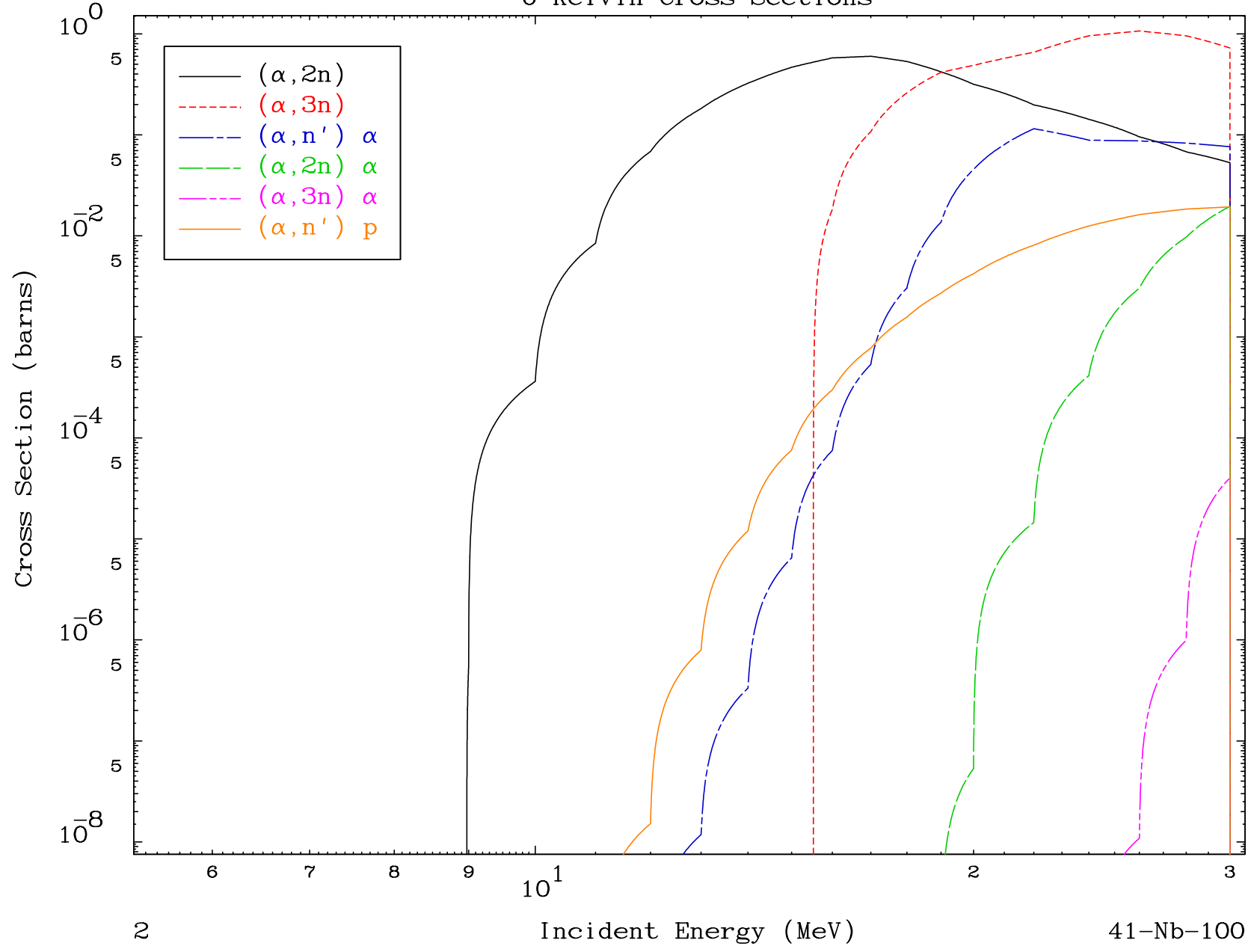


MAT 4147

α Neutron Production
0 Kelvin Cross Sections

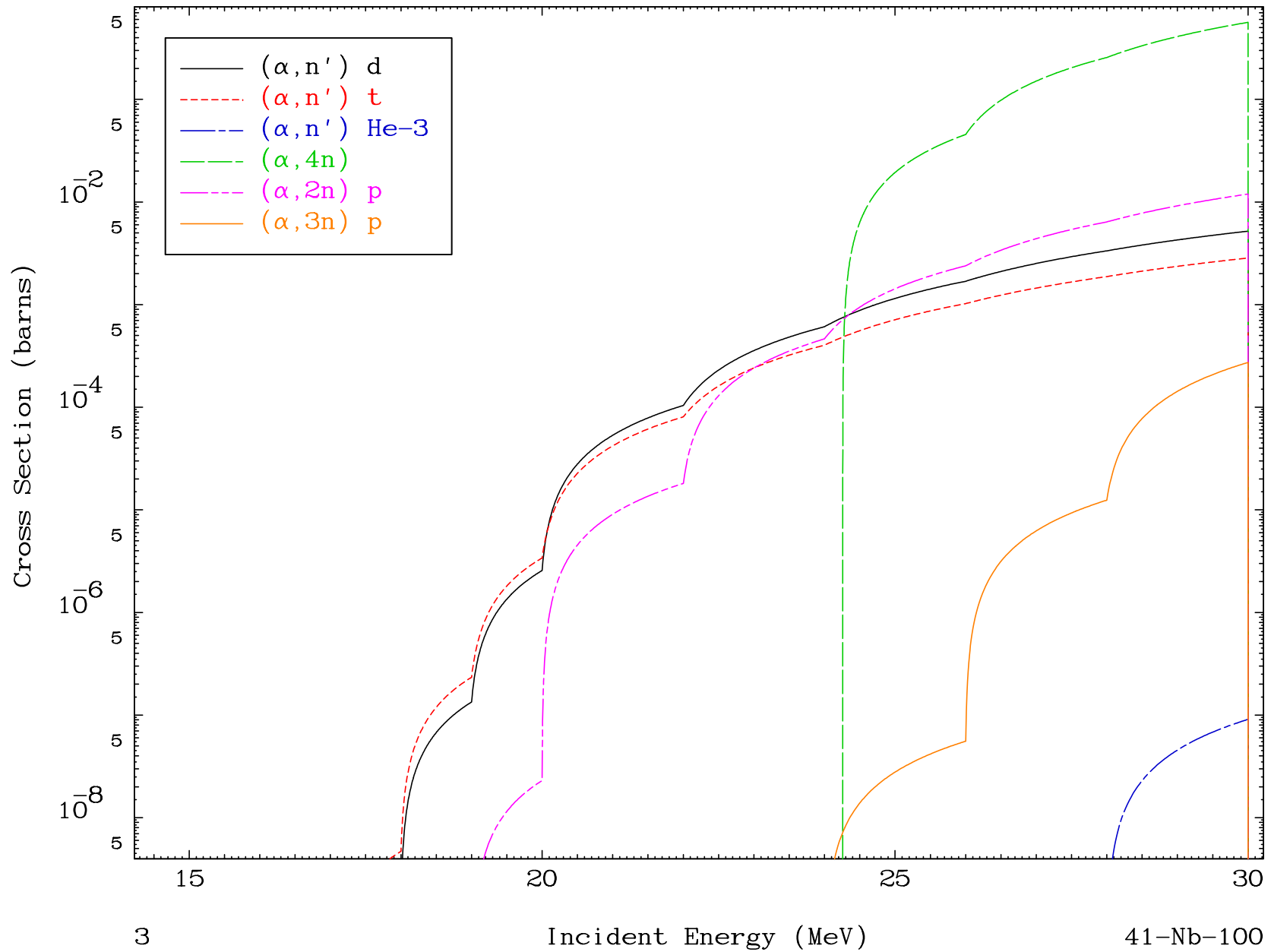
41-Nb-100

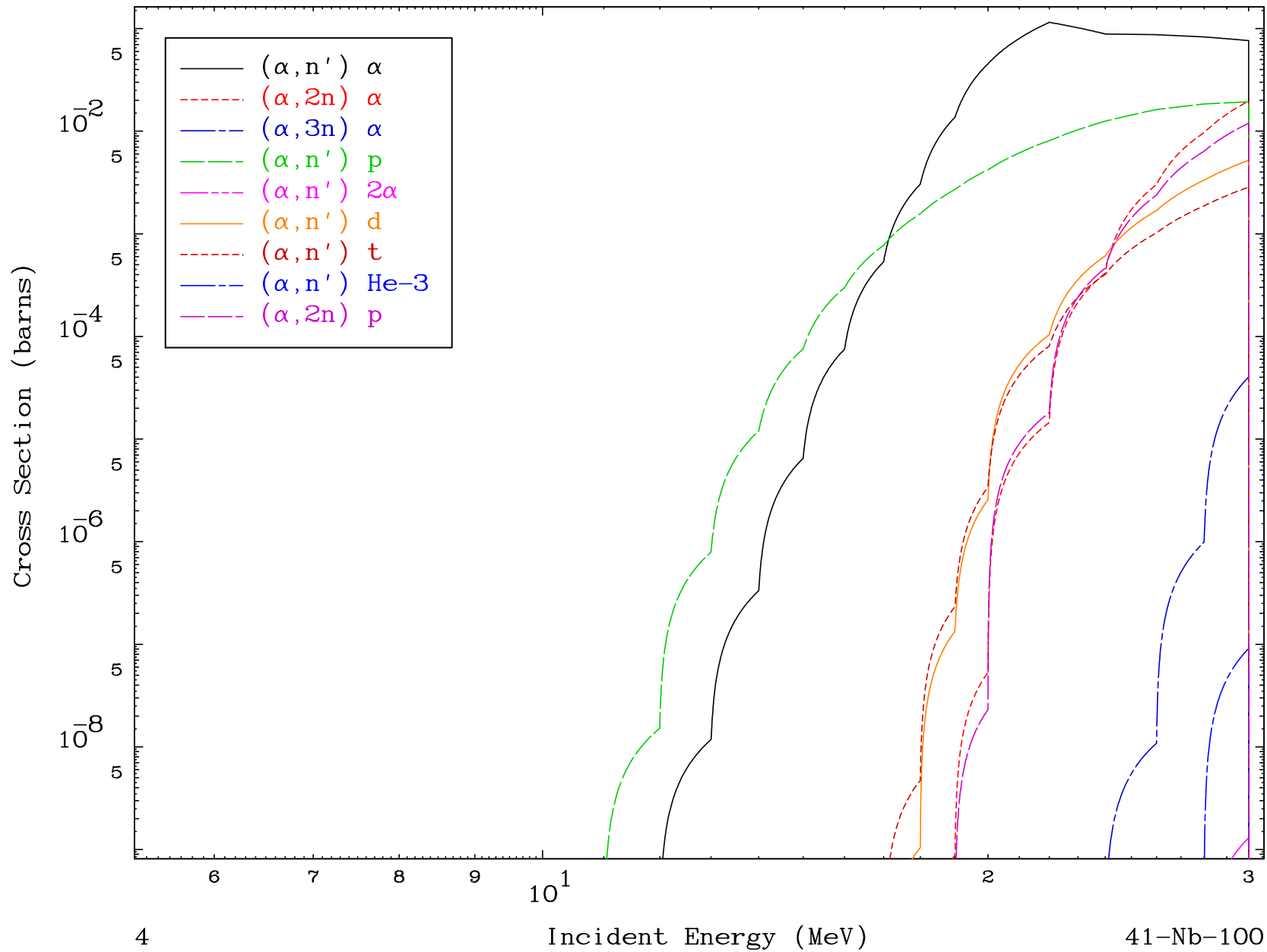


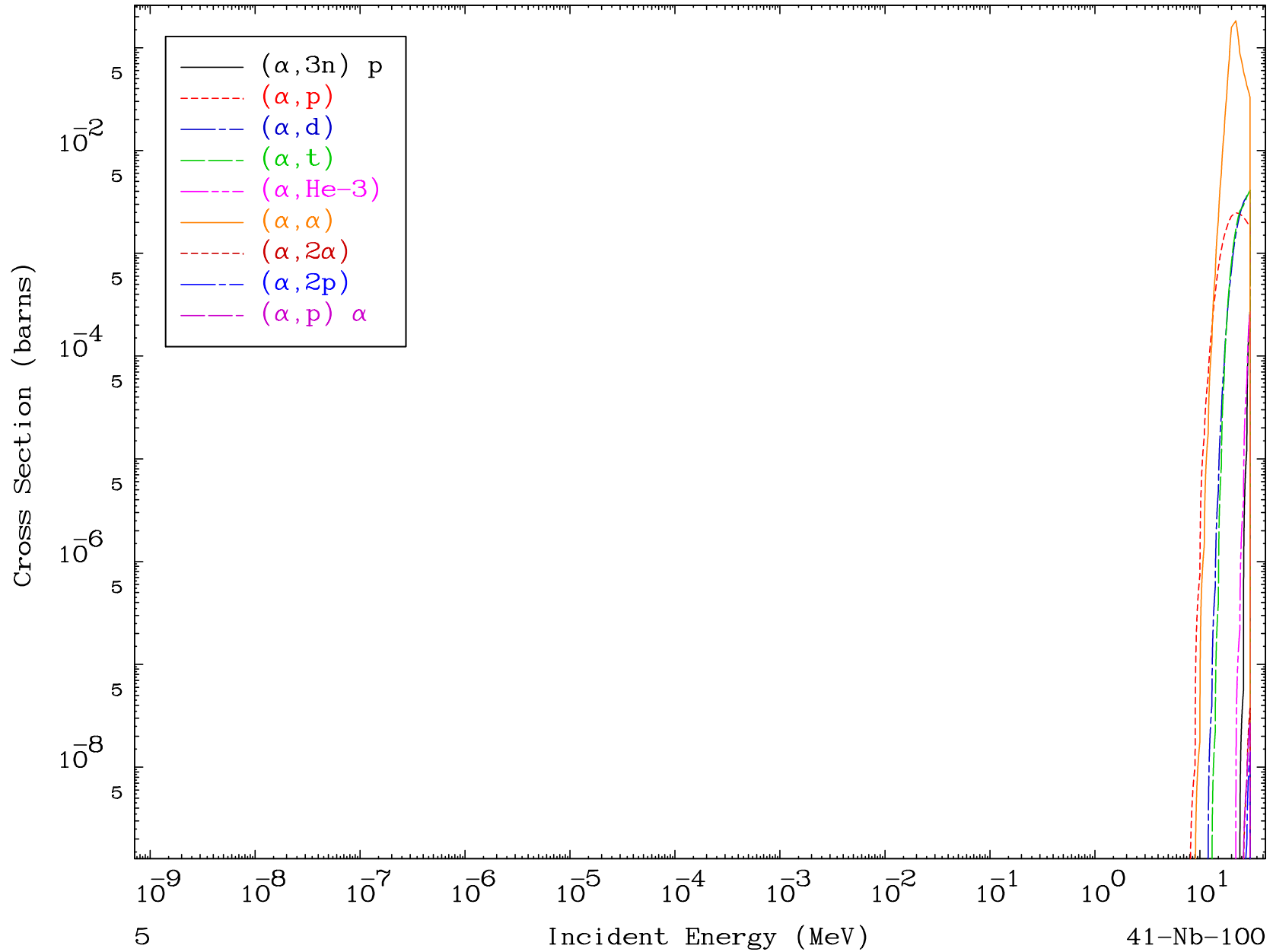
2

Incident Energy (MeV)

41-Nb-100



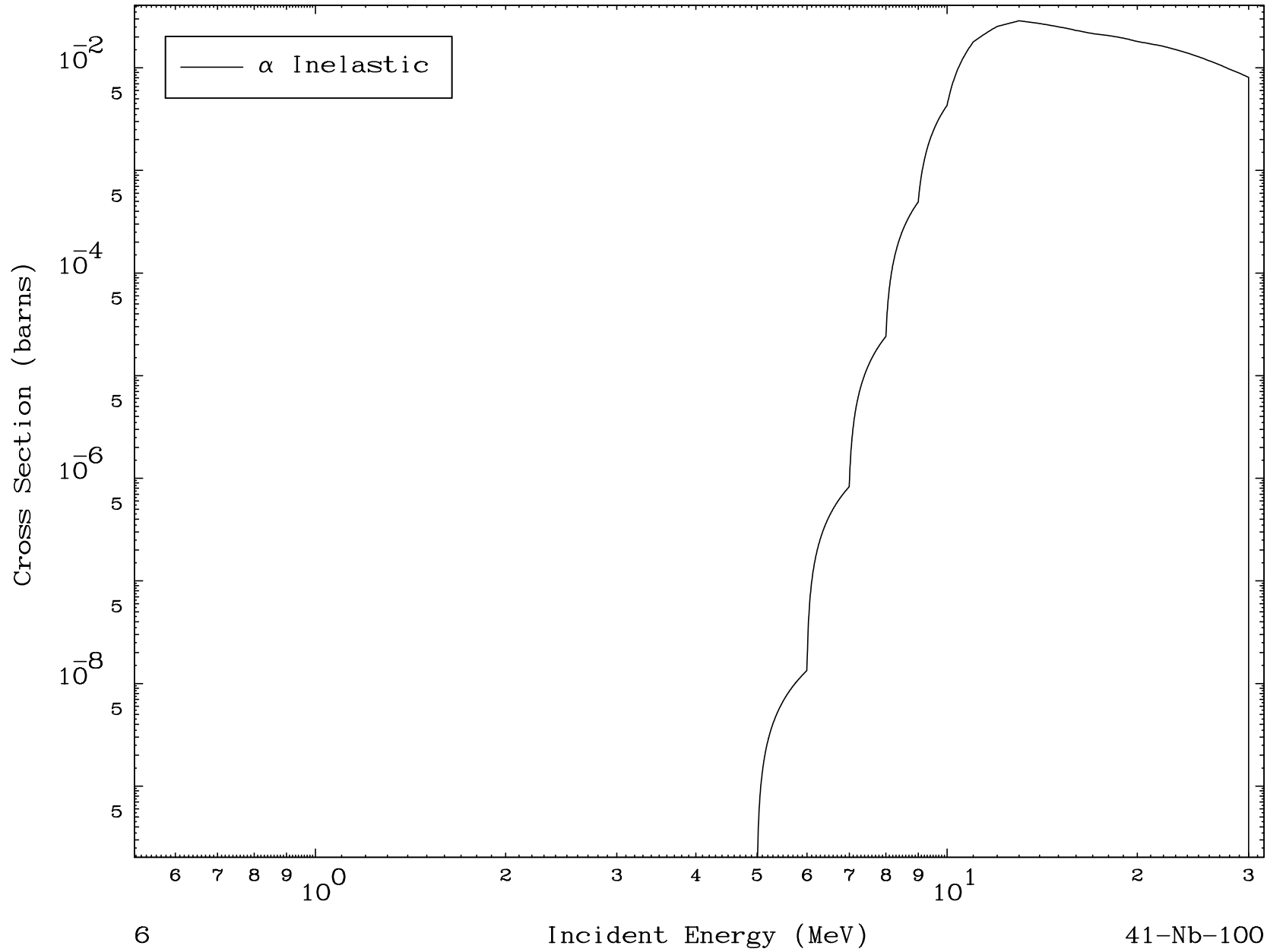




MAT 4147

(α, n') Level
0 Kelvin Cross Sections

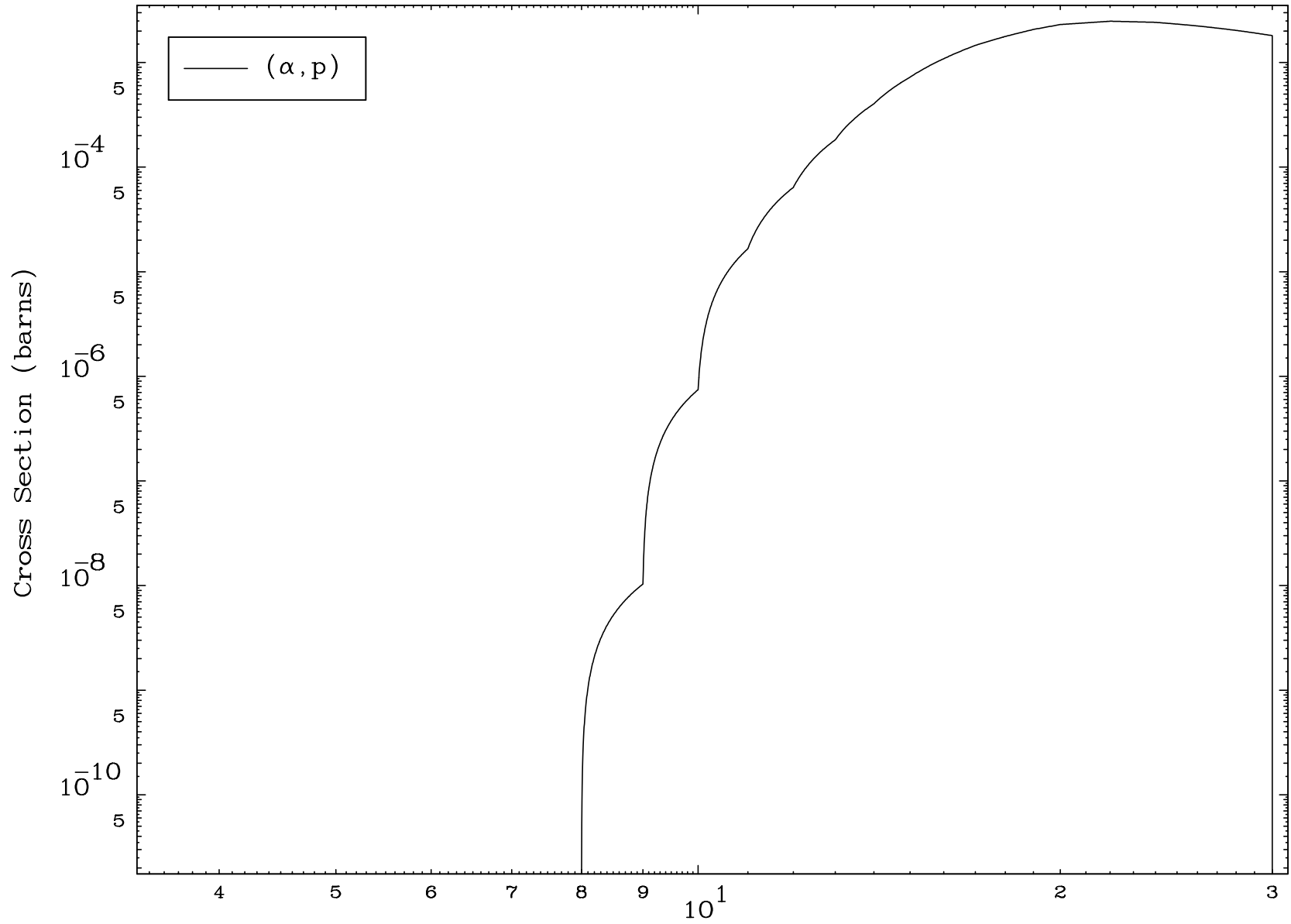
41-Nb-100



6

Incident Energy (MeV)

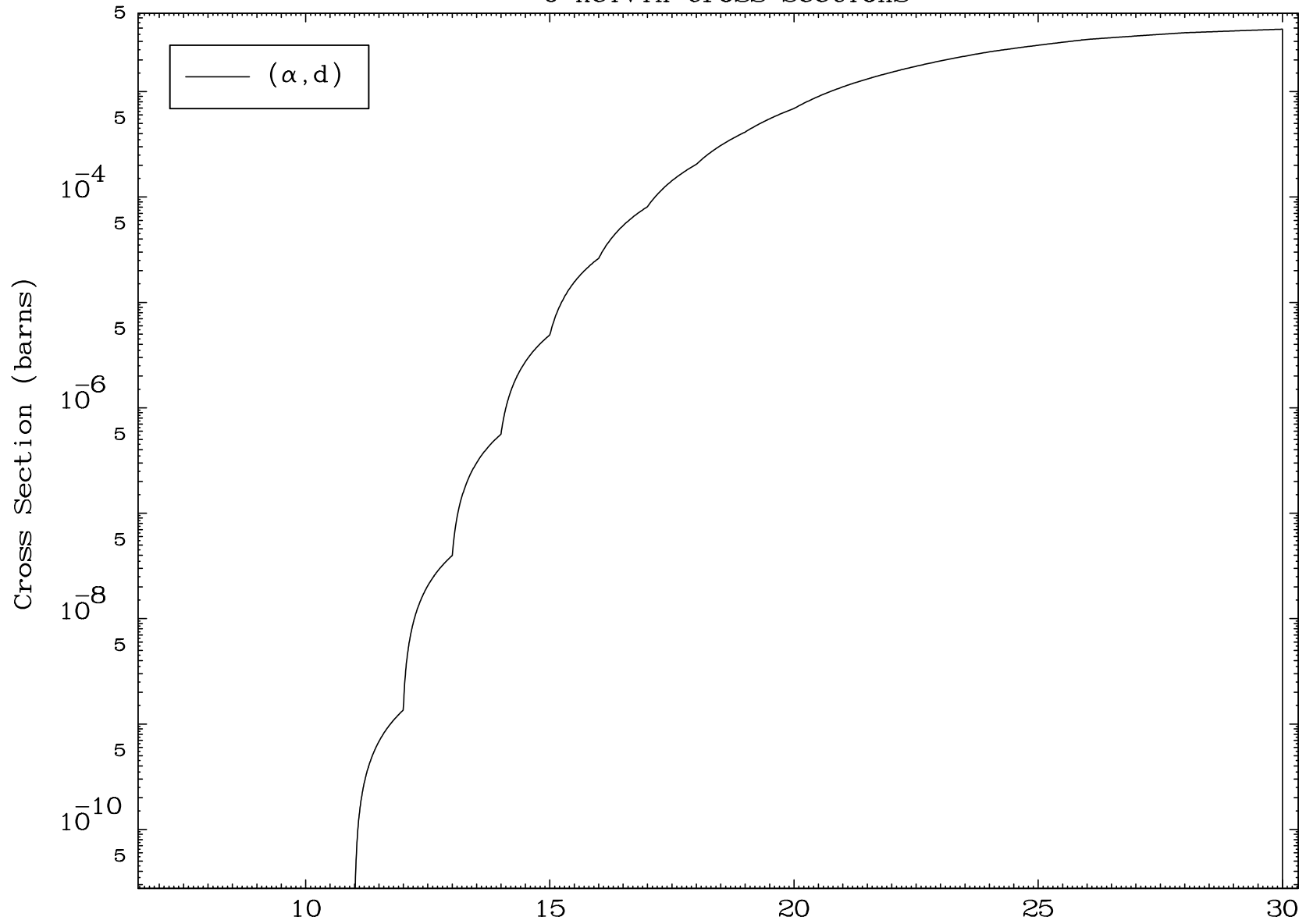
41-Nb-100



MAT 4147

(α ,d) Levels
0 Kelvin Cross Sections

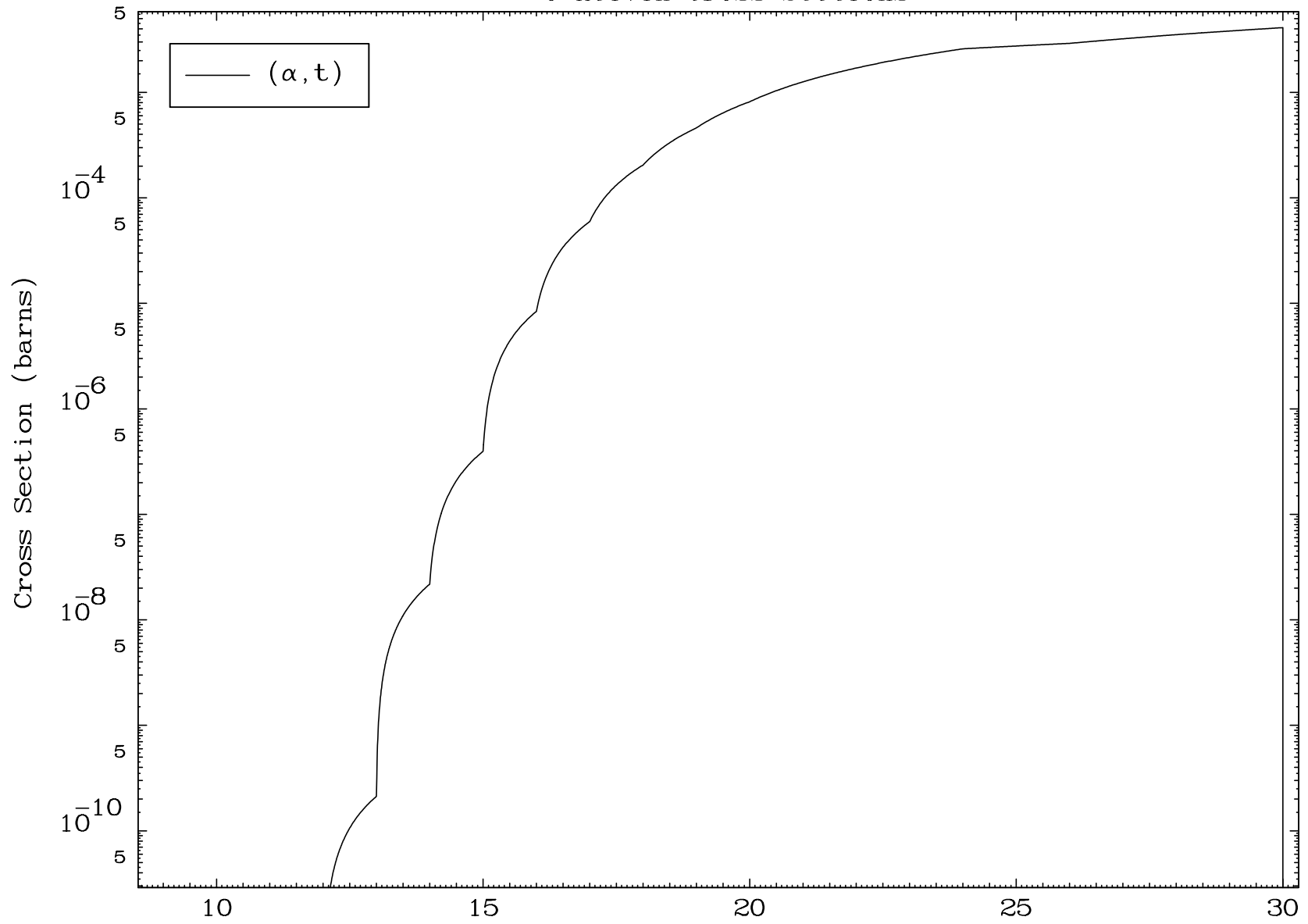
41-Nb-100



8

Incident Energy (MeV)

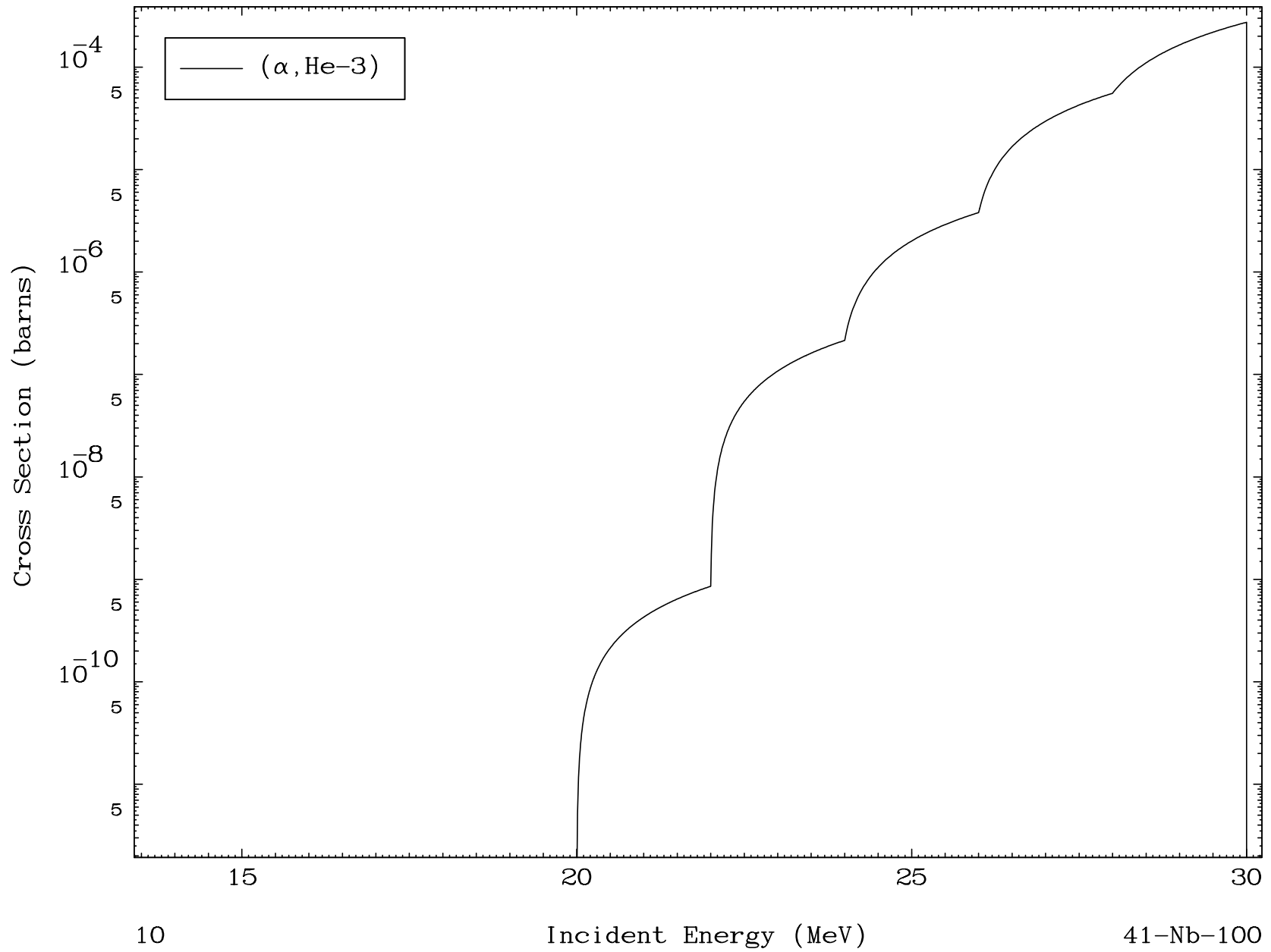
41-Nb-100

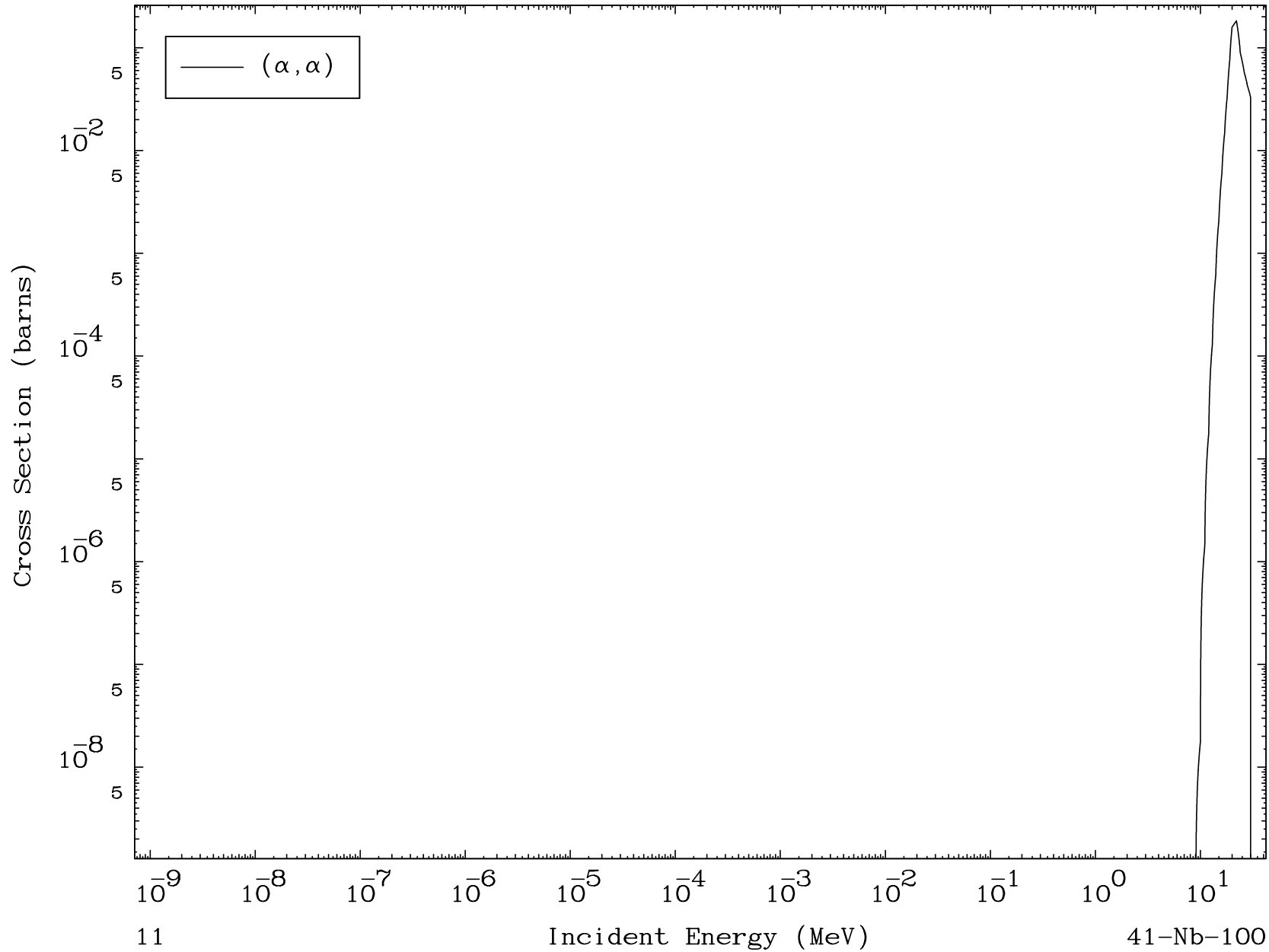


MAT 4147

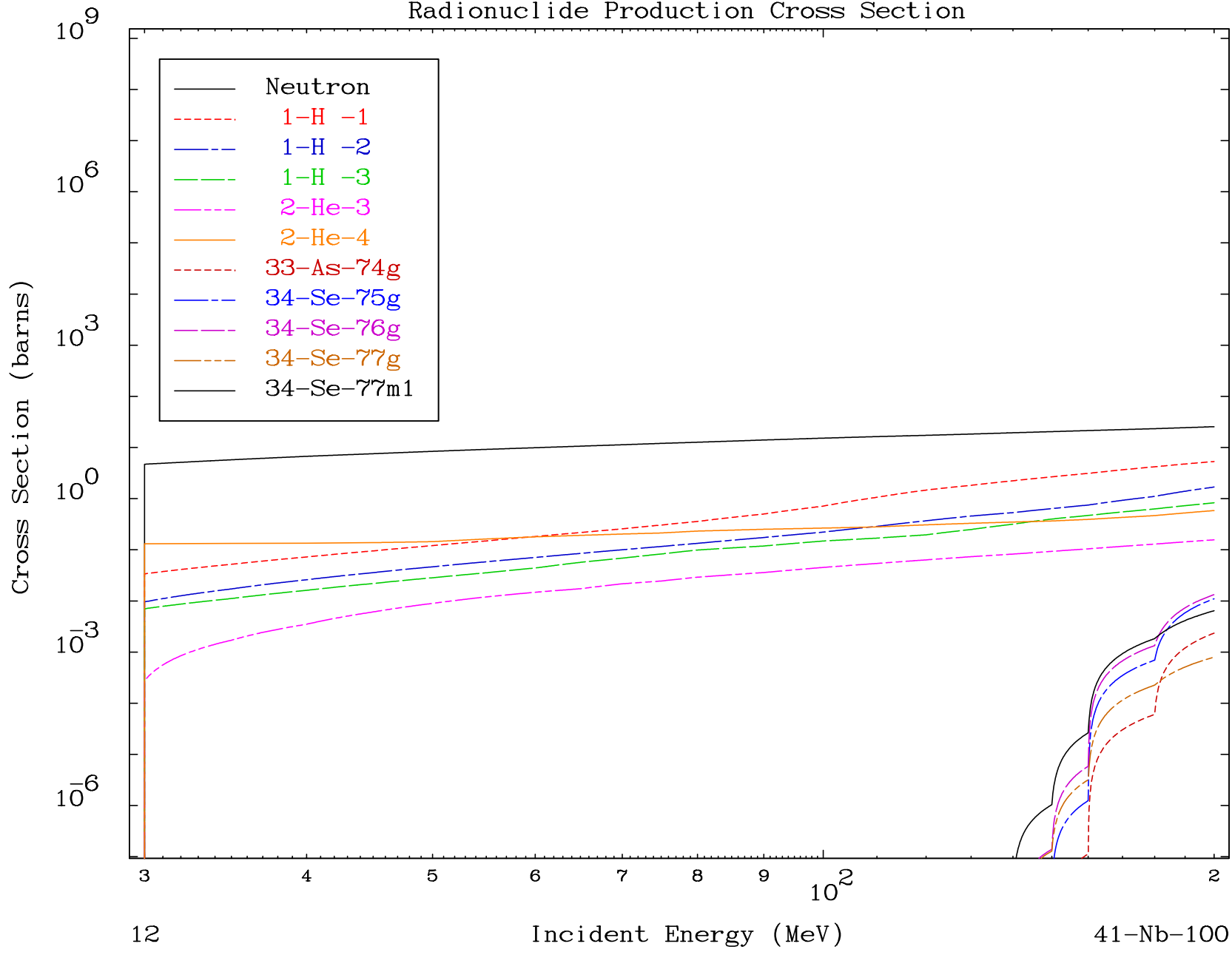
($\alpha, \text{He-3}$) Levels
0 Kelvin Cross Sections

41-Nb-100

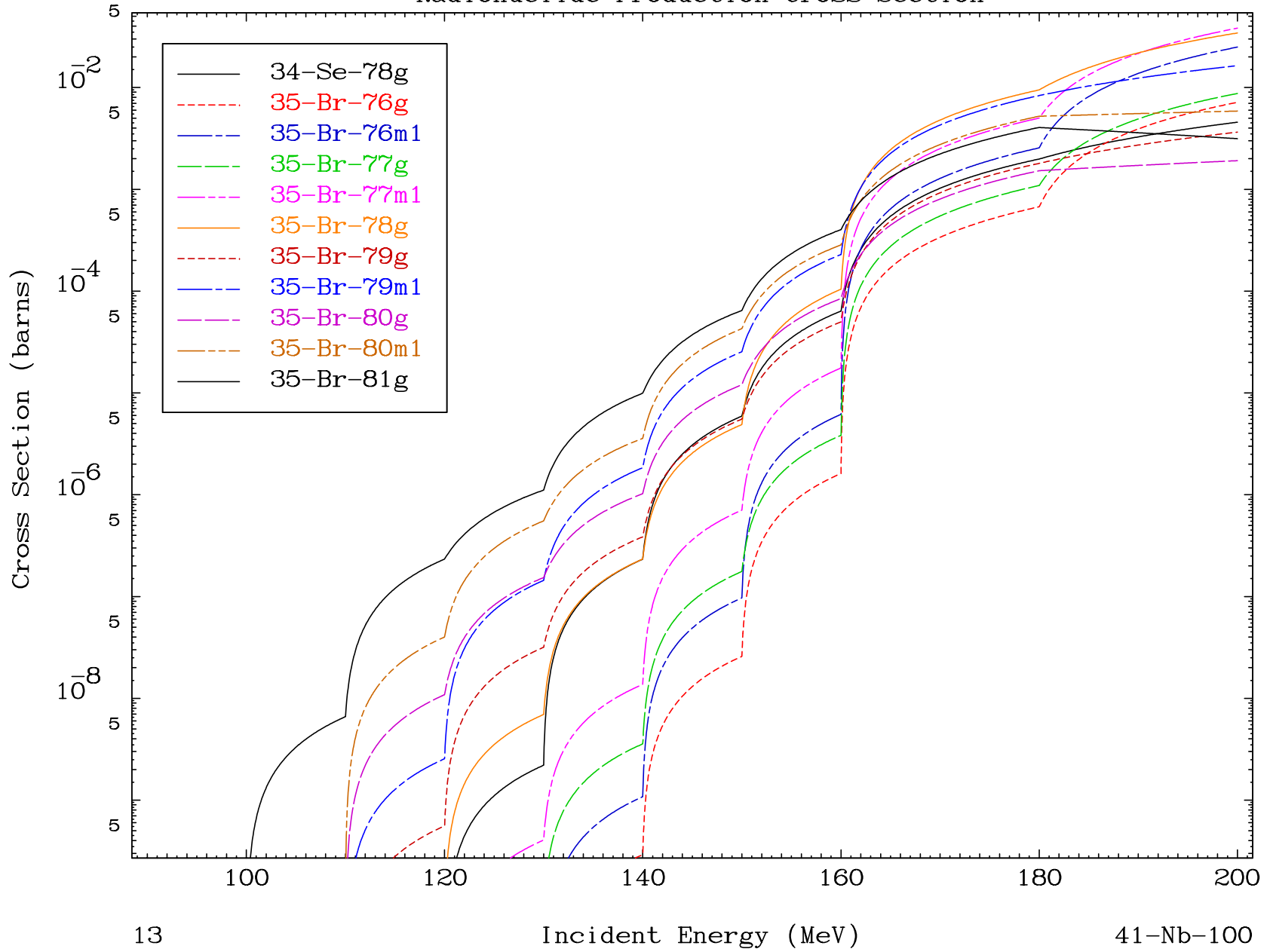




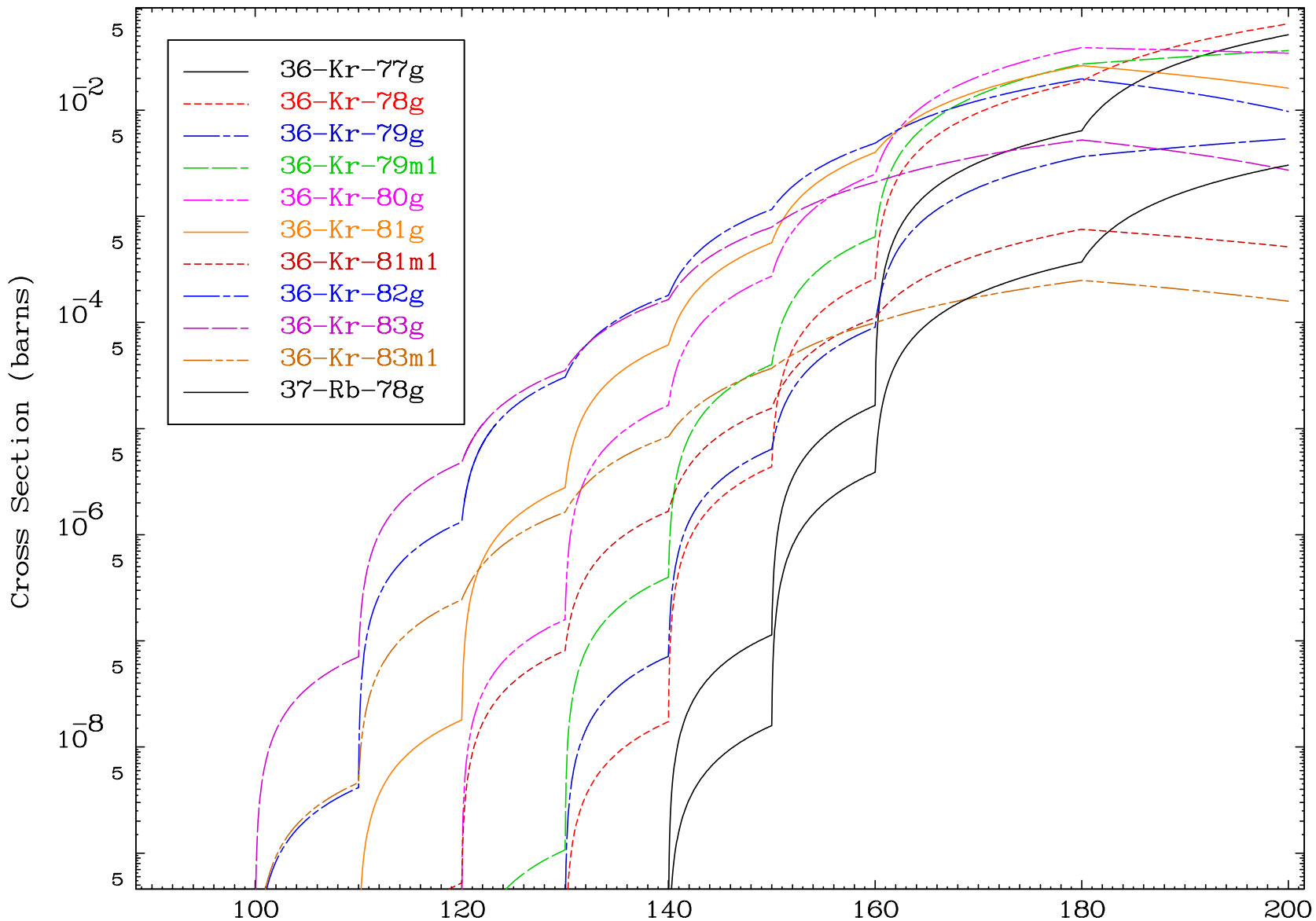
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

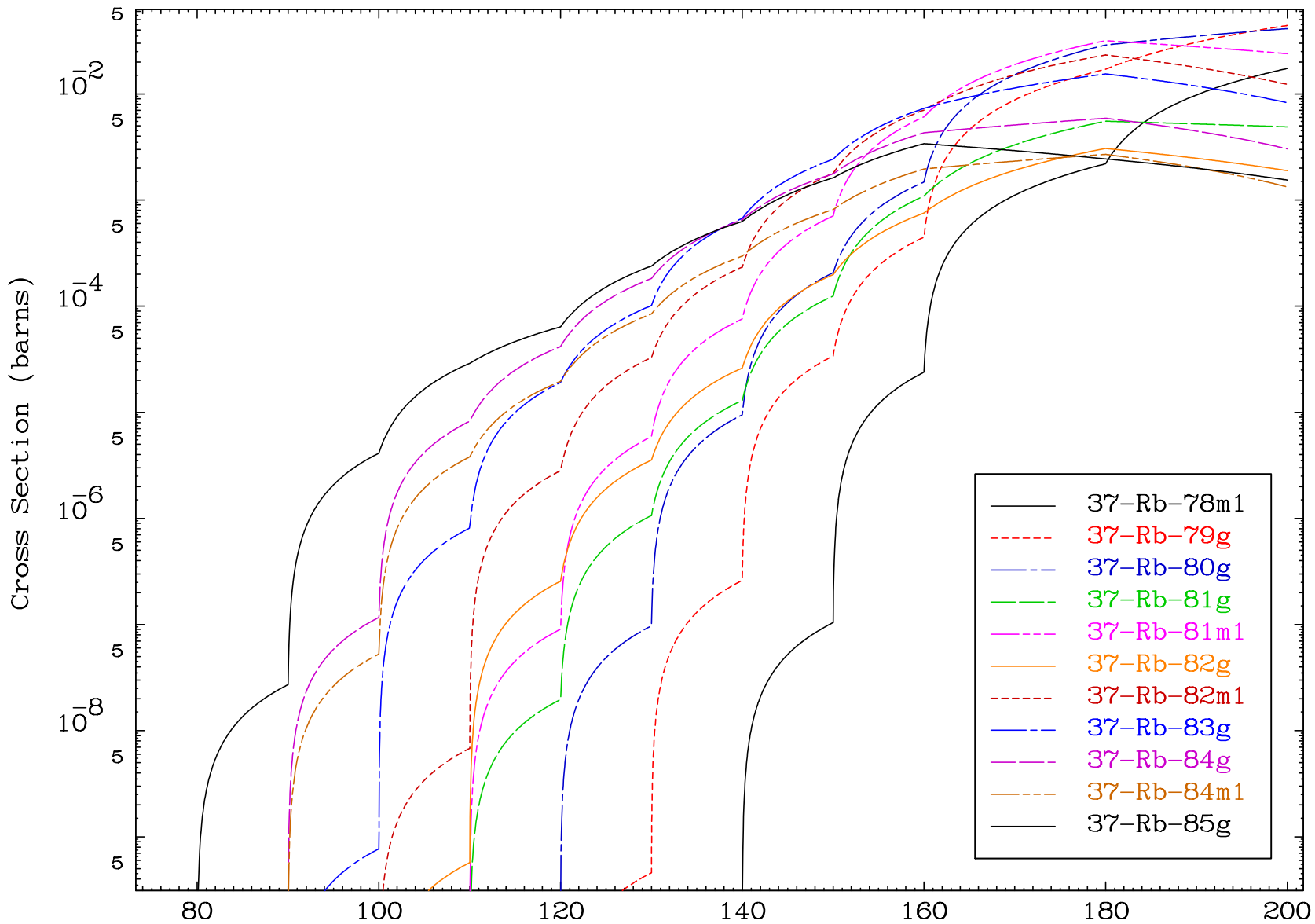


MAT 4147

(α , remainder)

41-Nb-100

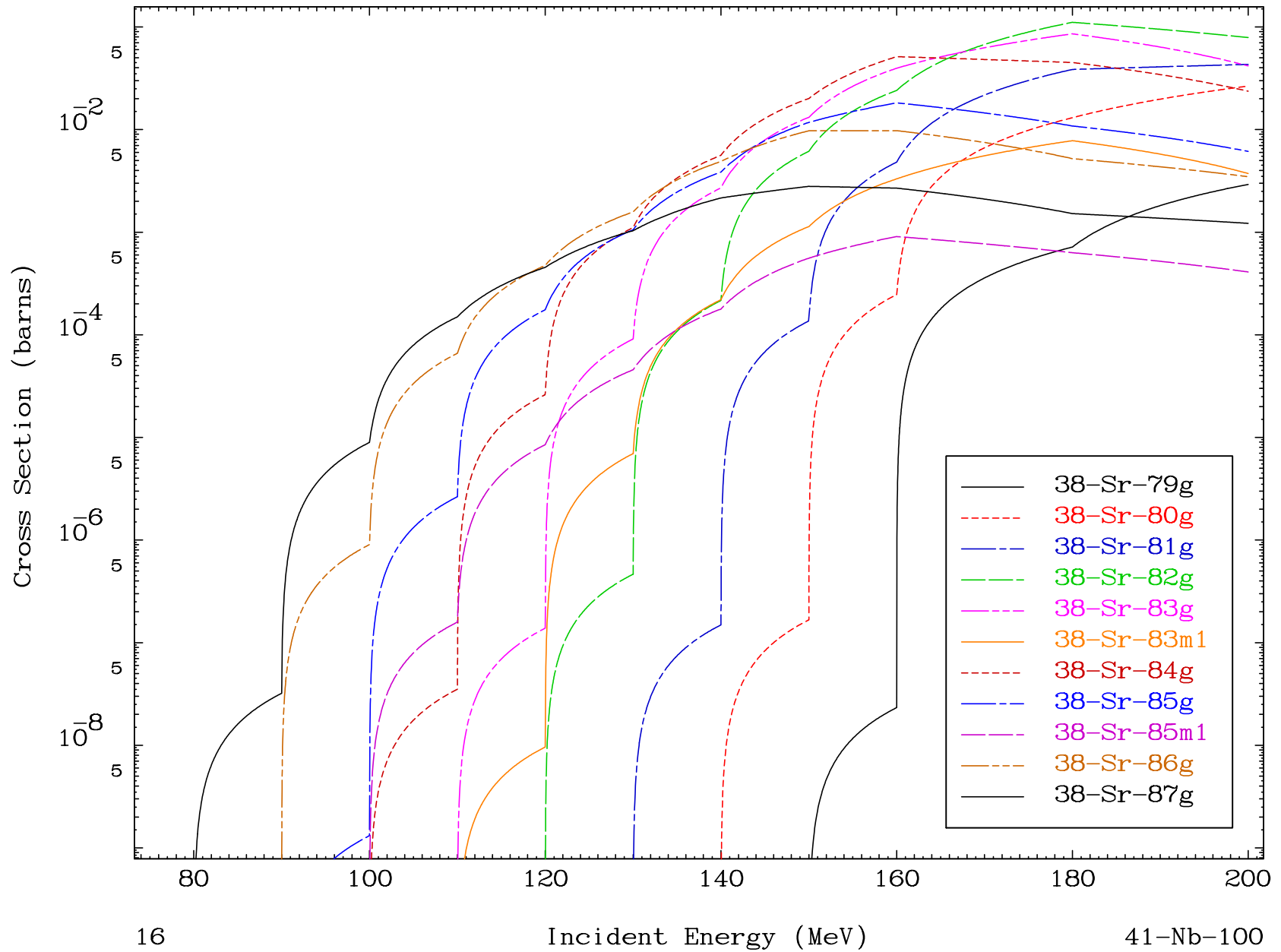
Radionuclide Production Cross Section

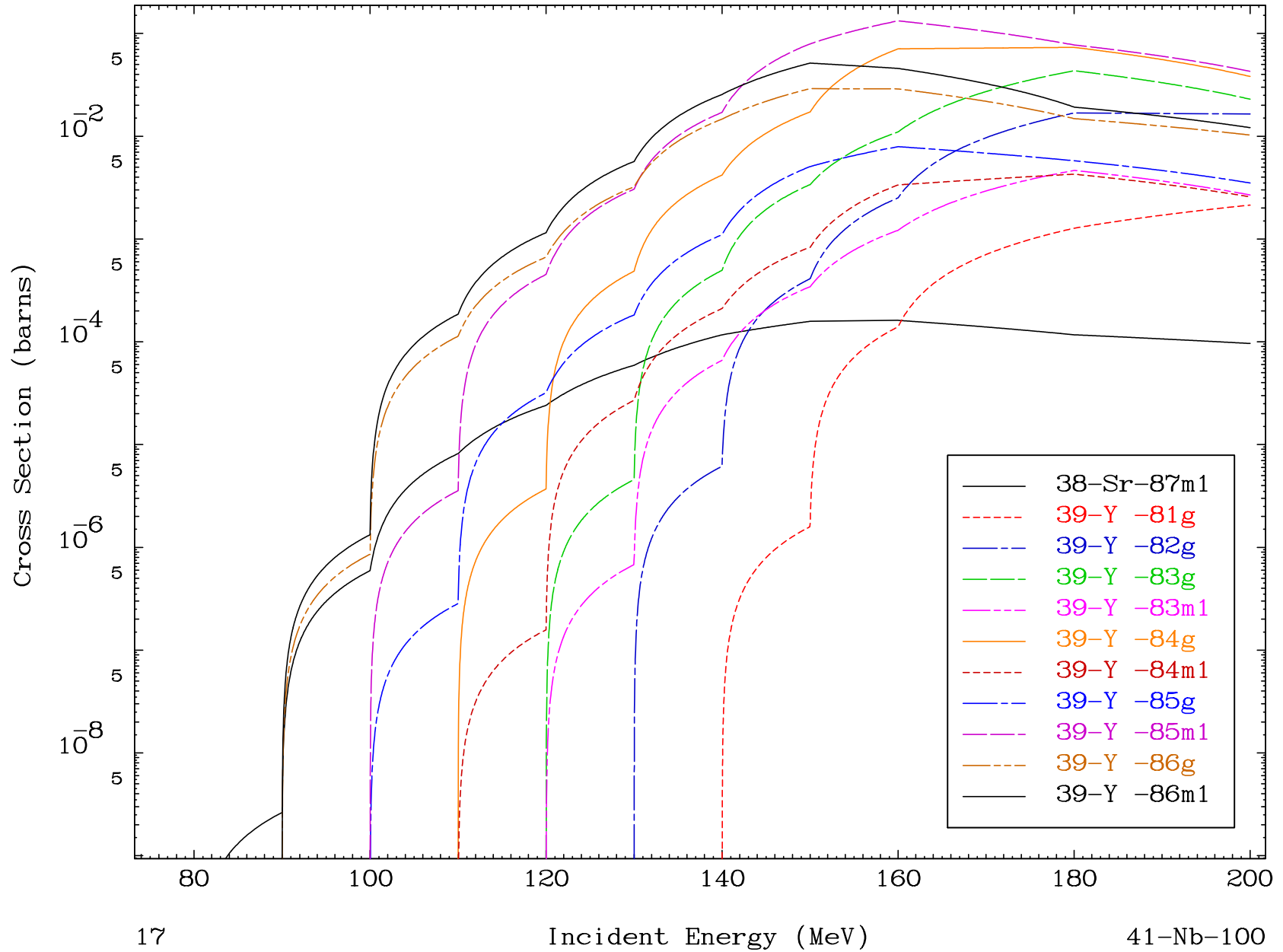


15

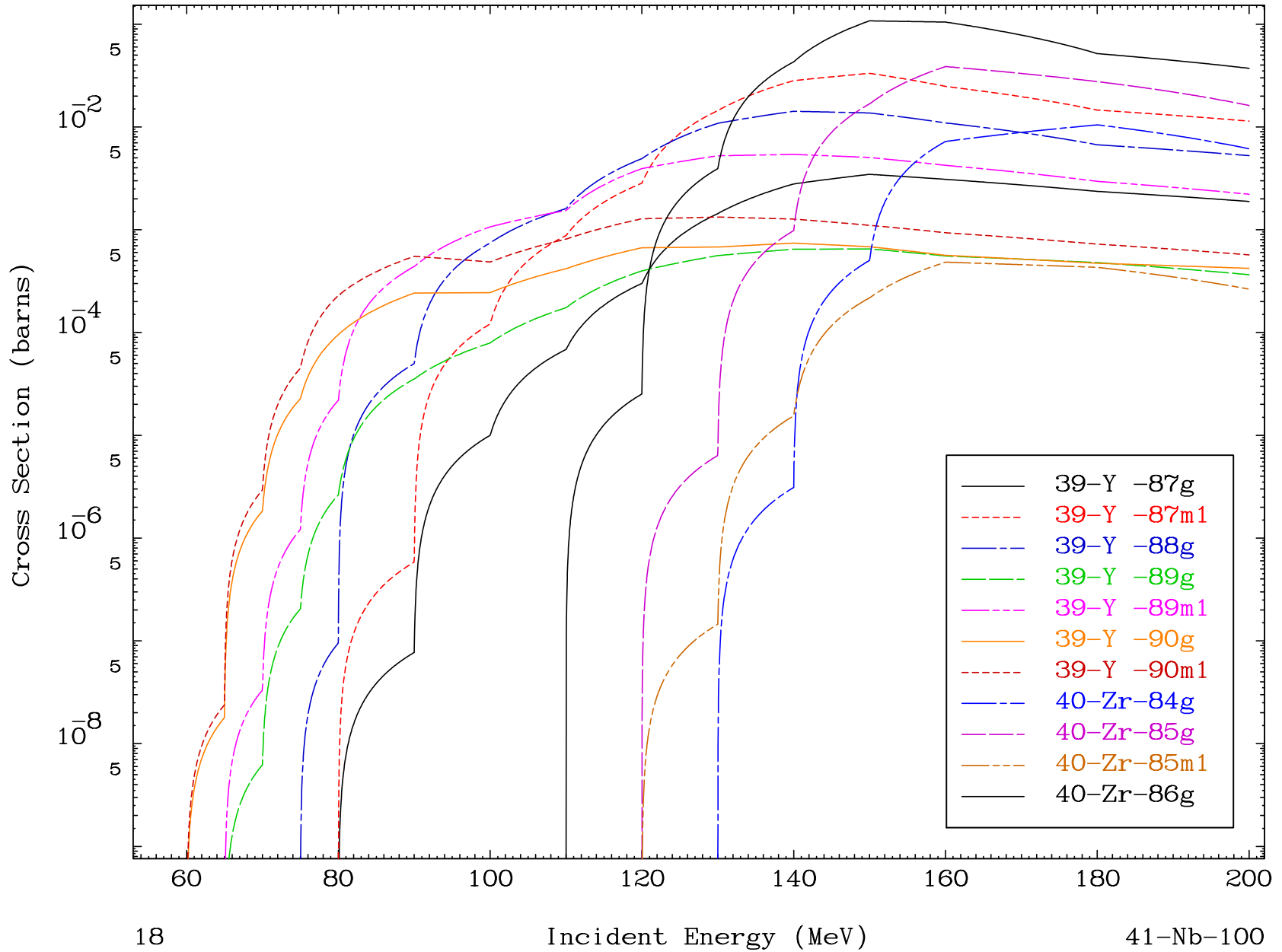
Incident Energy (MeV)

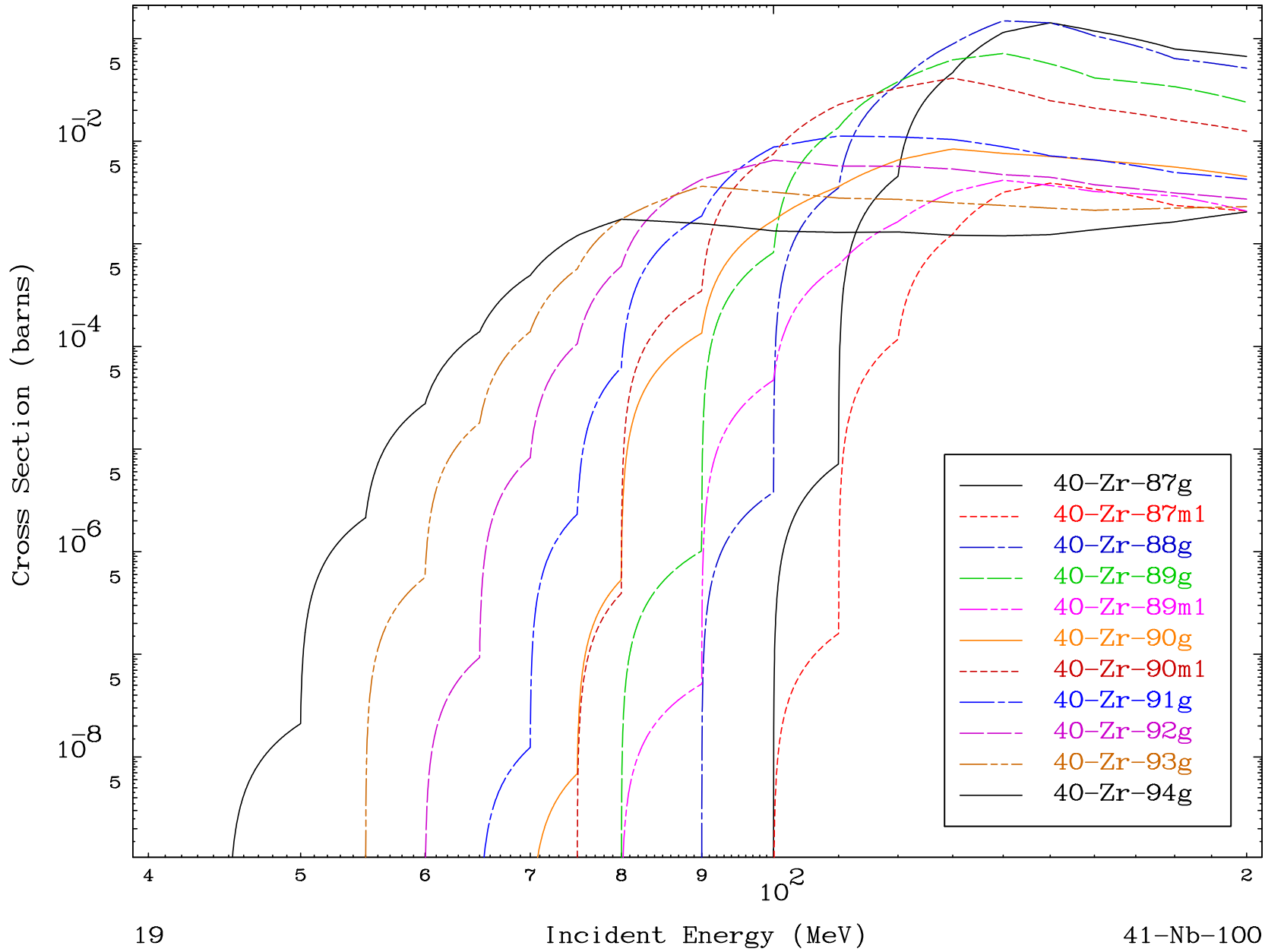
41-Nb-100

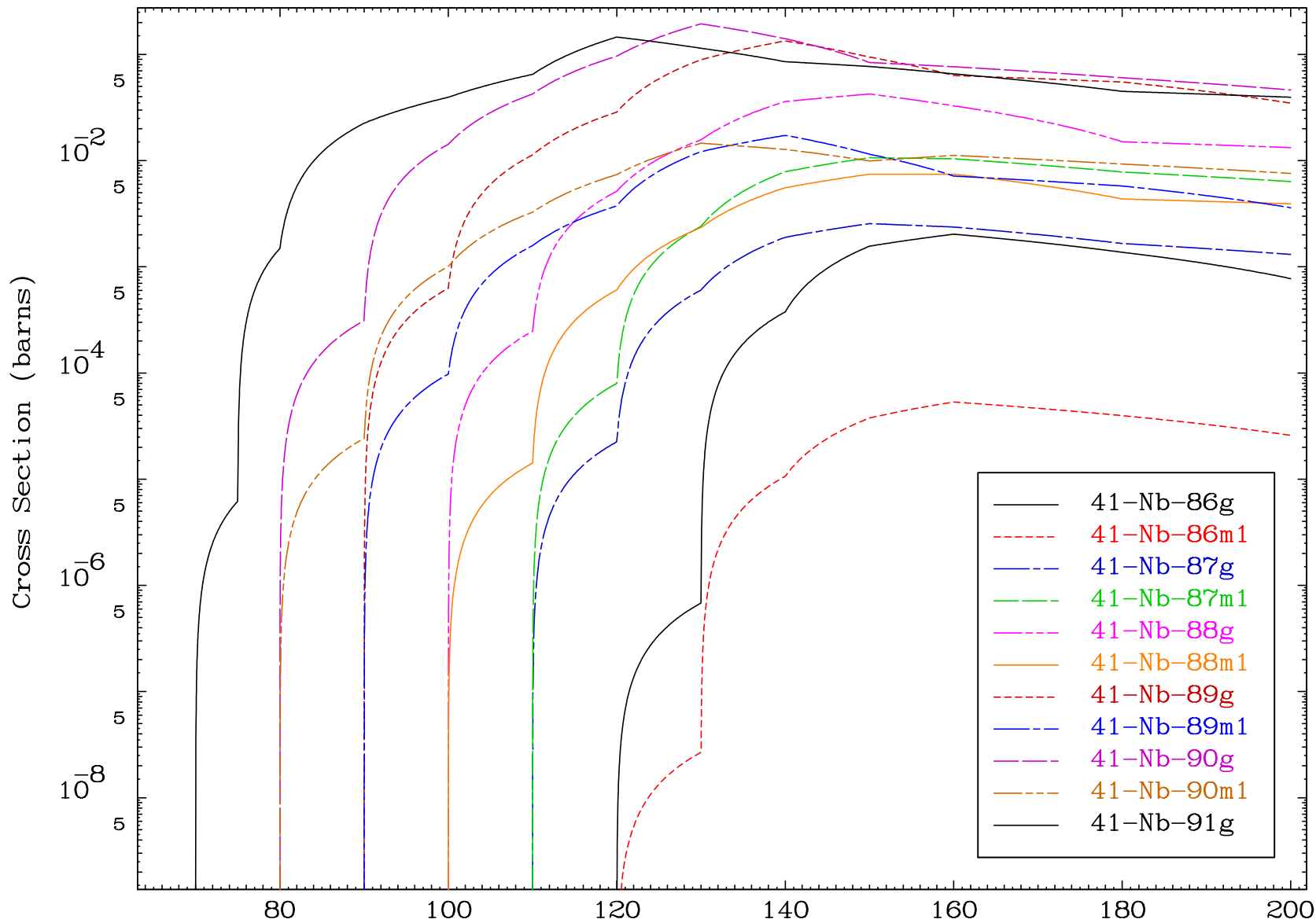


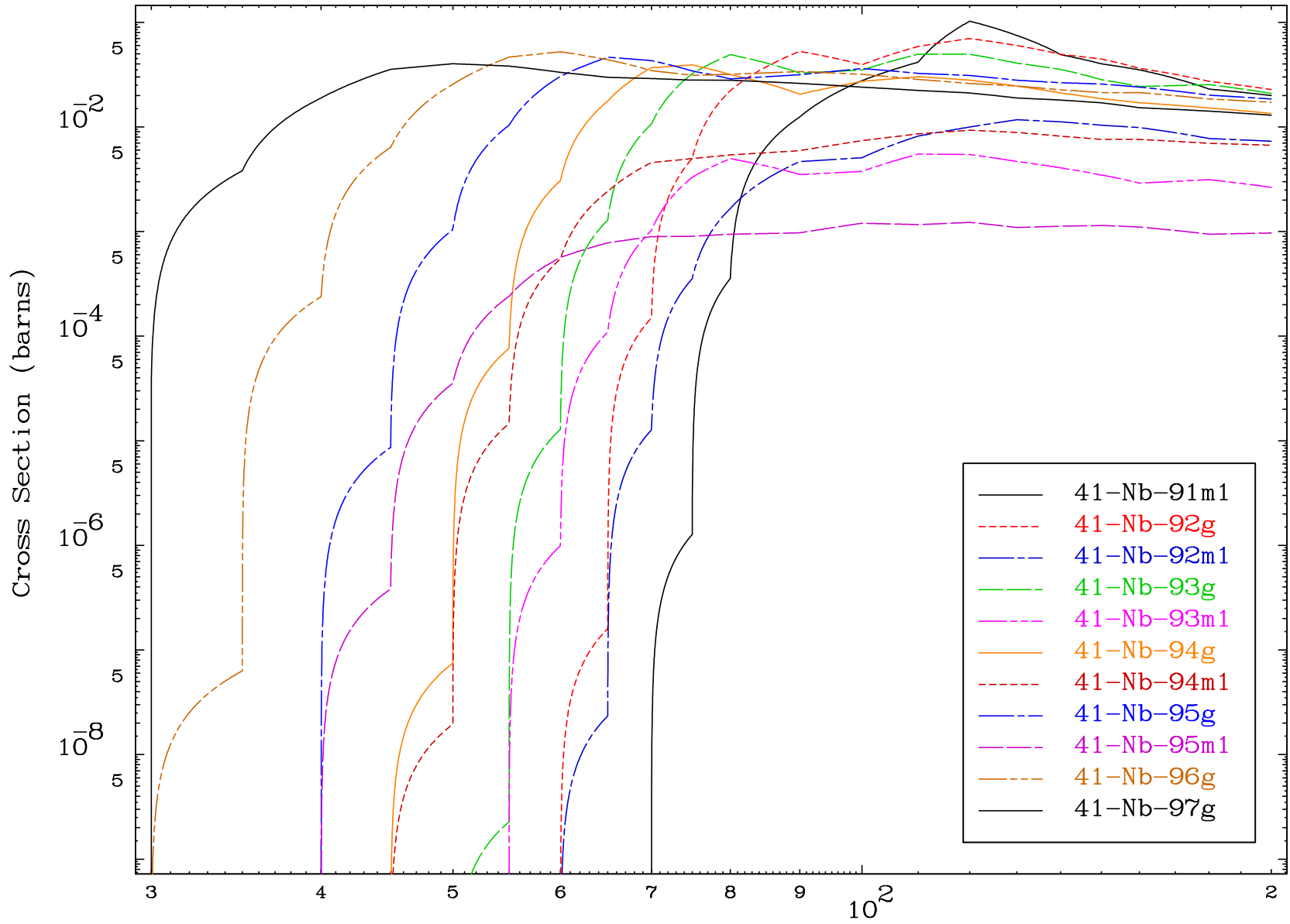


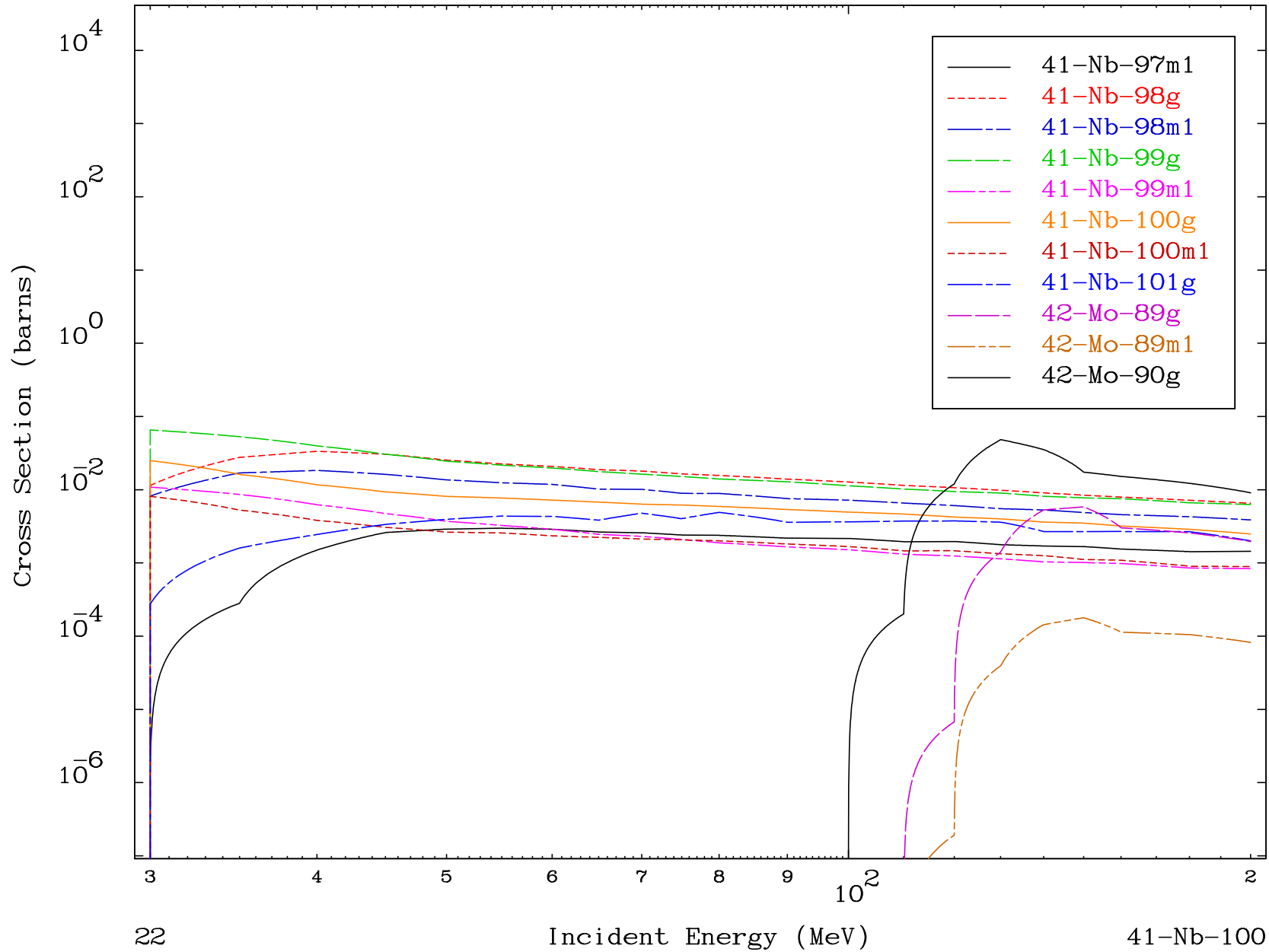
Radionuclide Production Cross Section

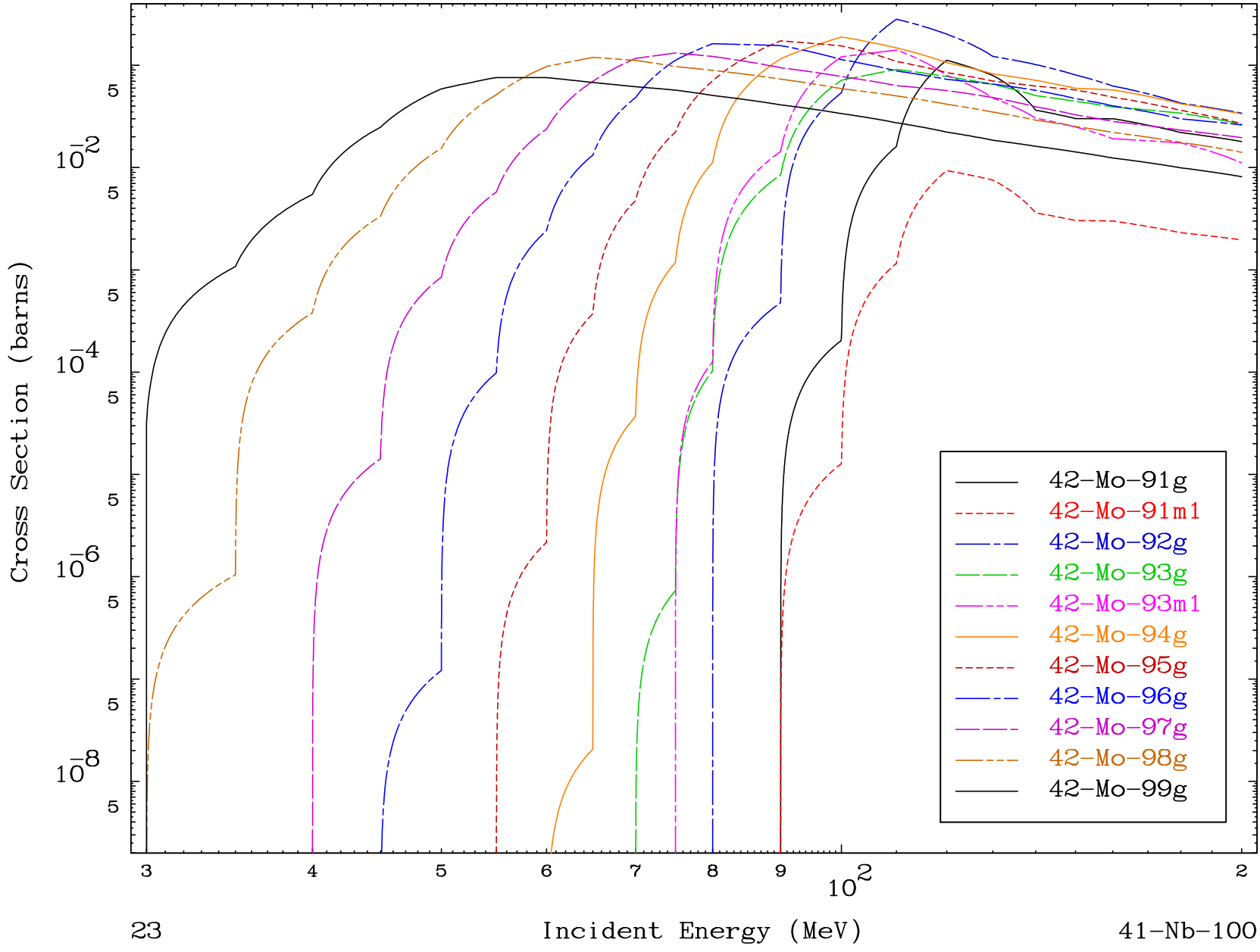


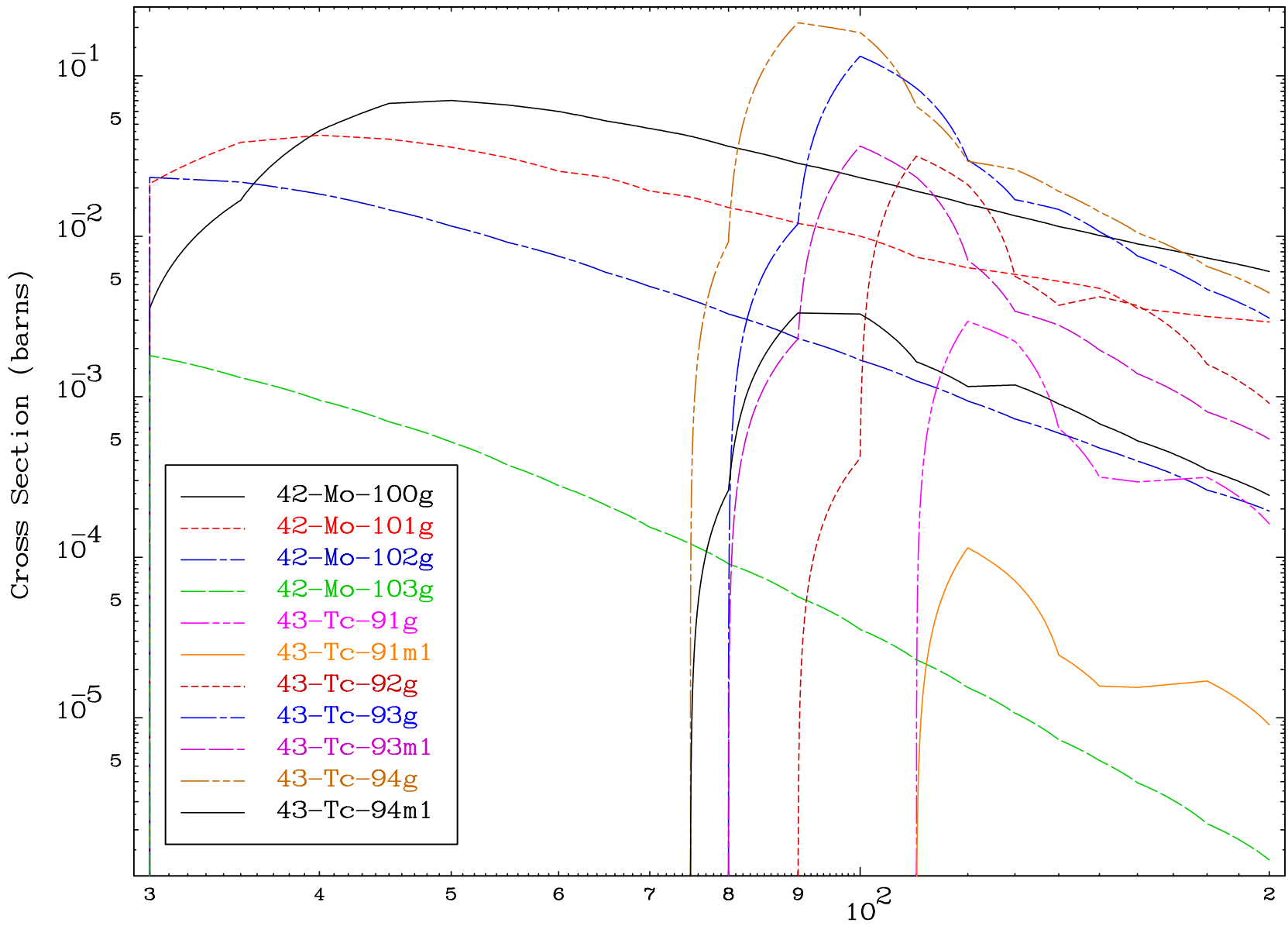


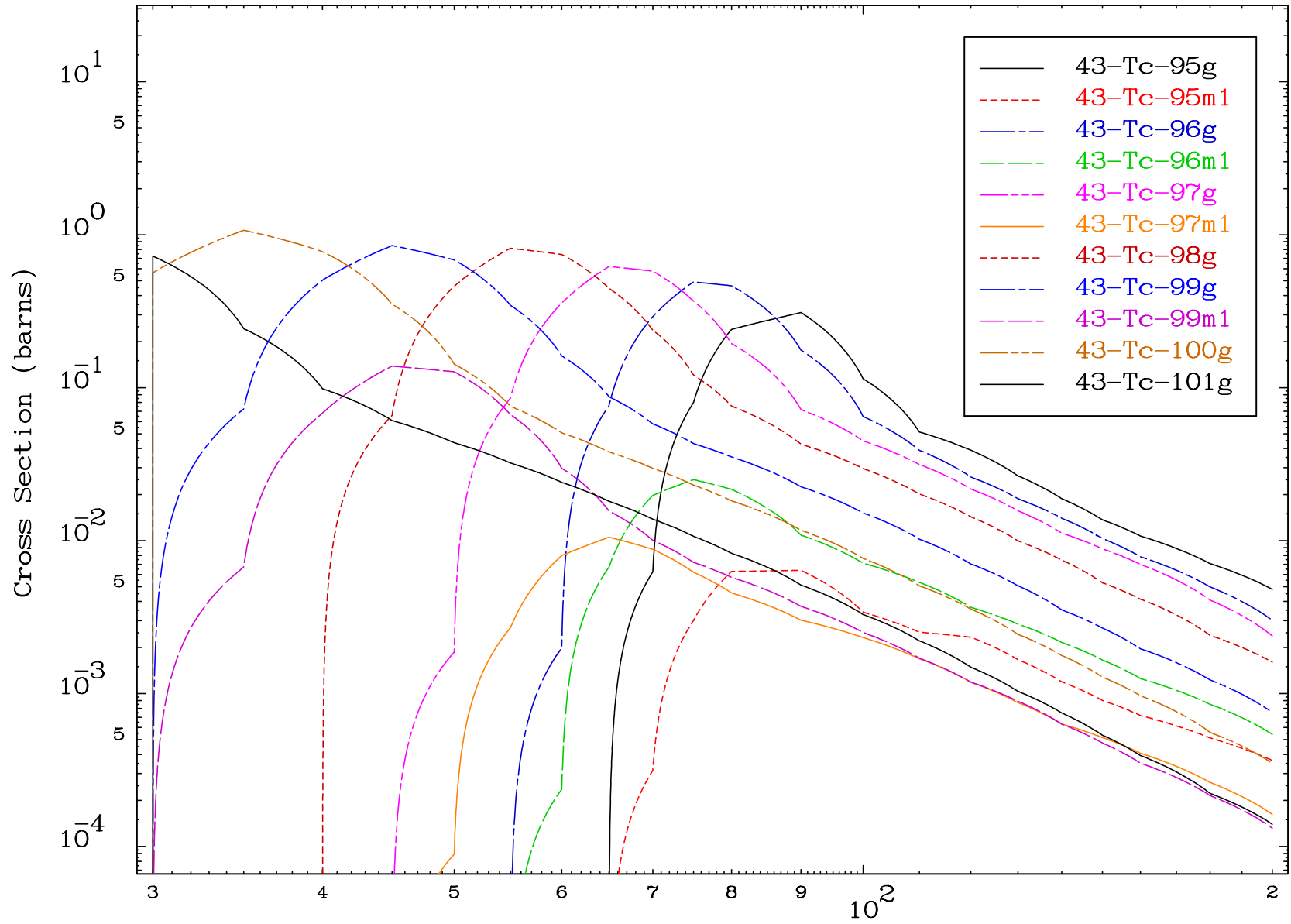


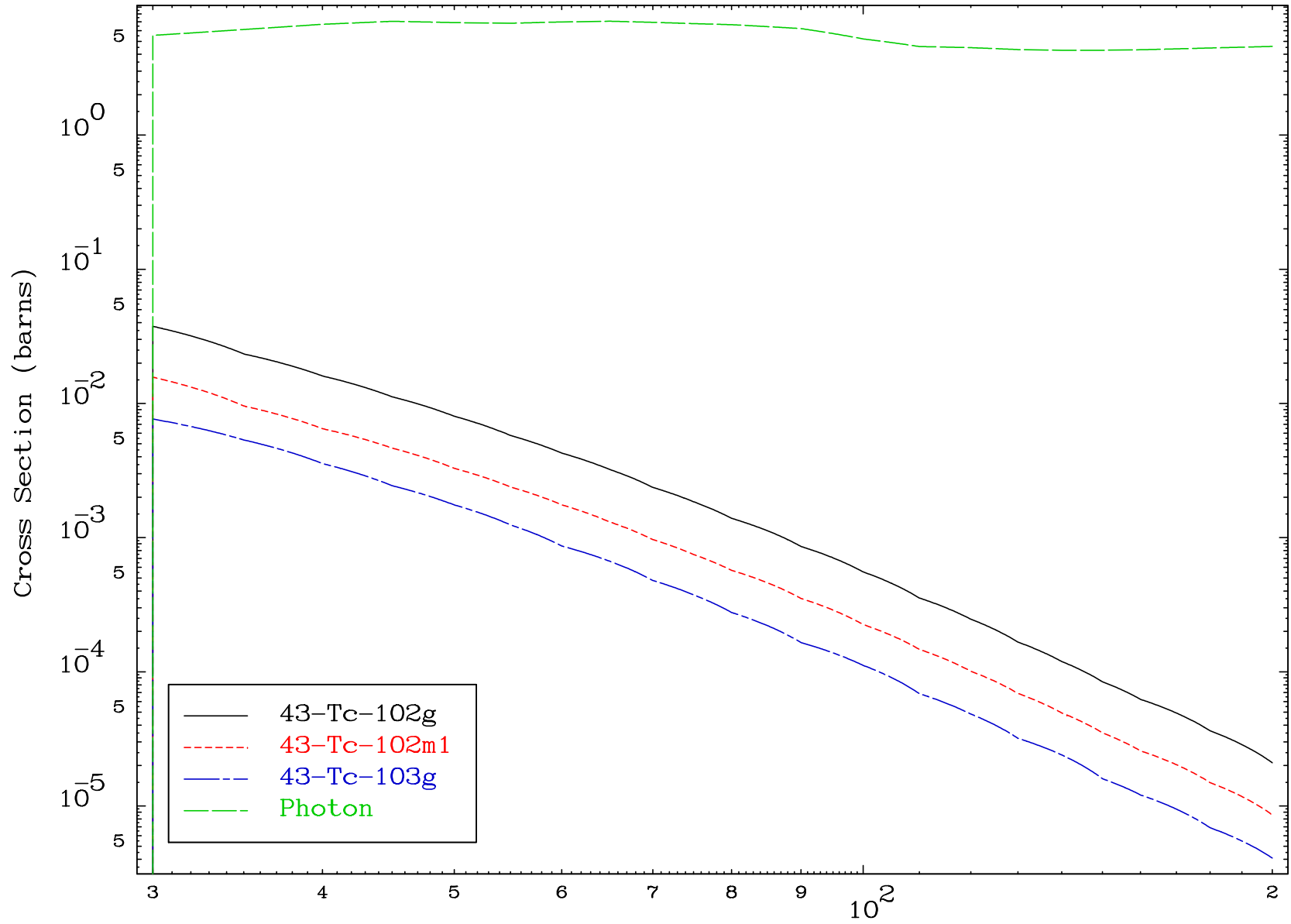










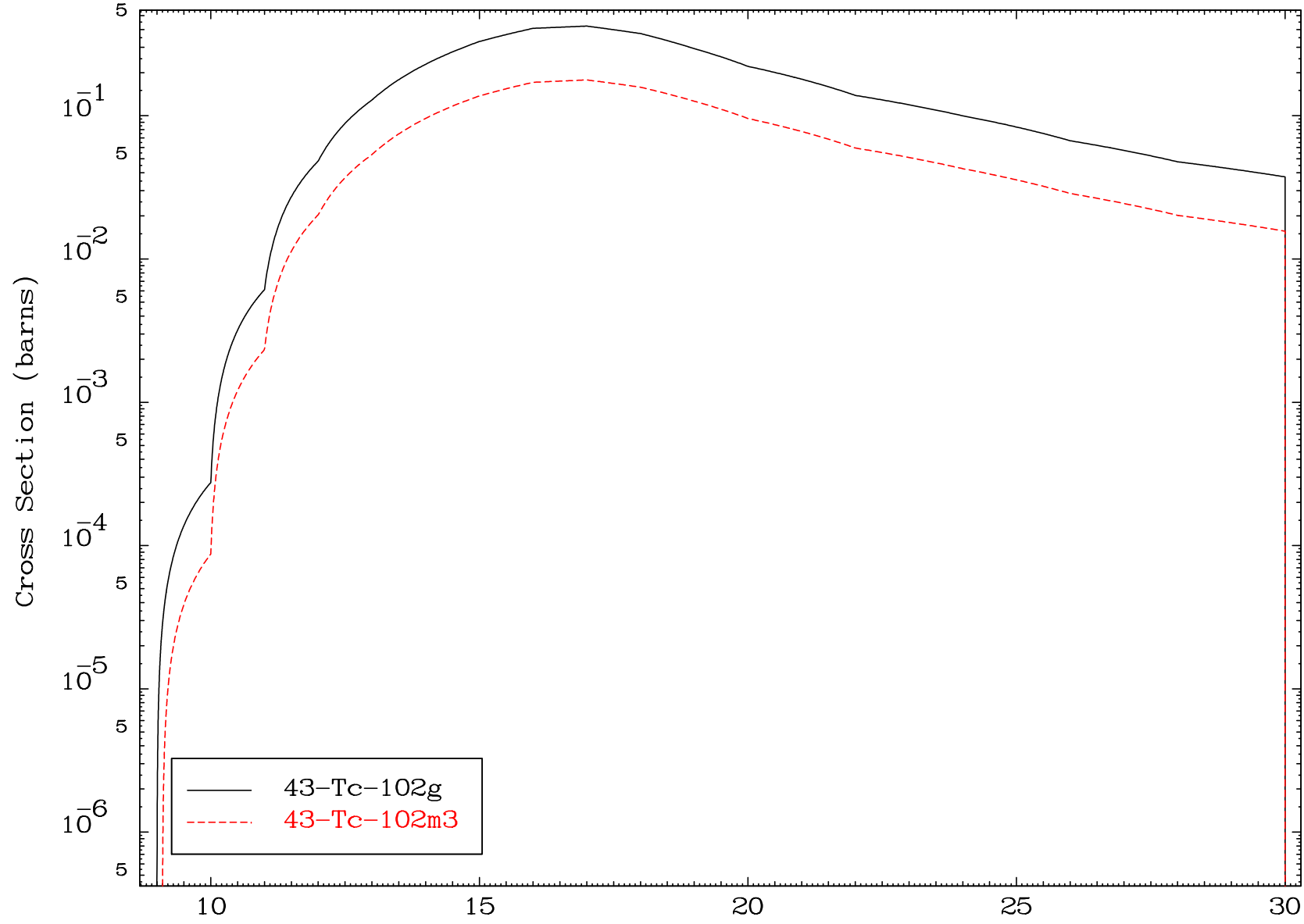


MAT 4147

($\alpha, 2n$)

41-Nb-100

Radionuclide Production Cross Section

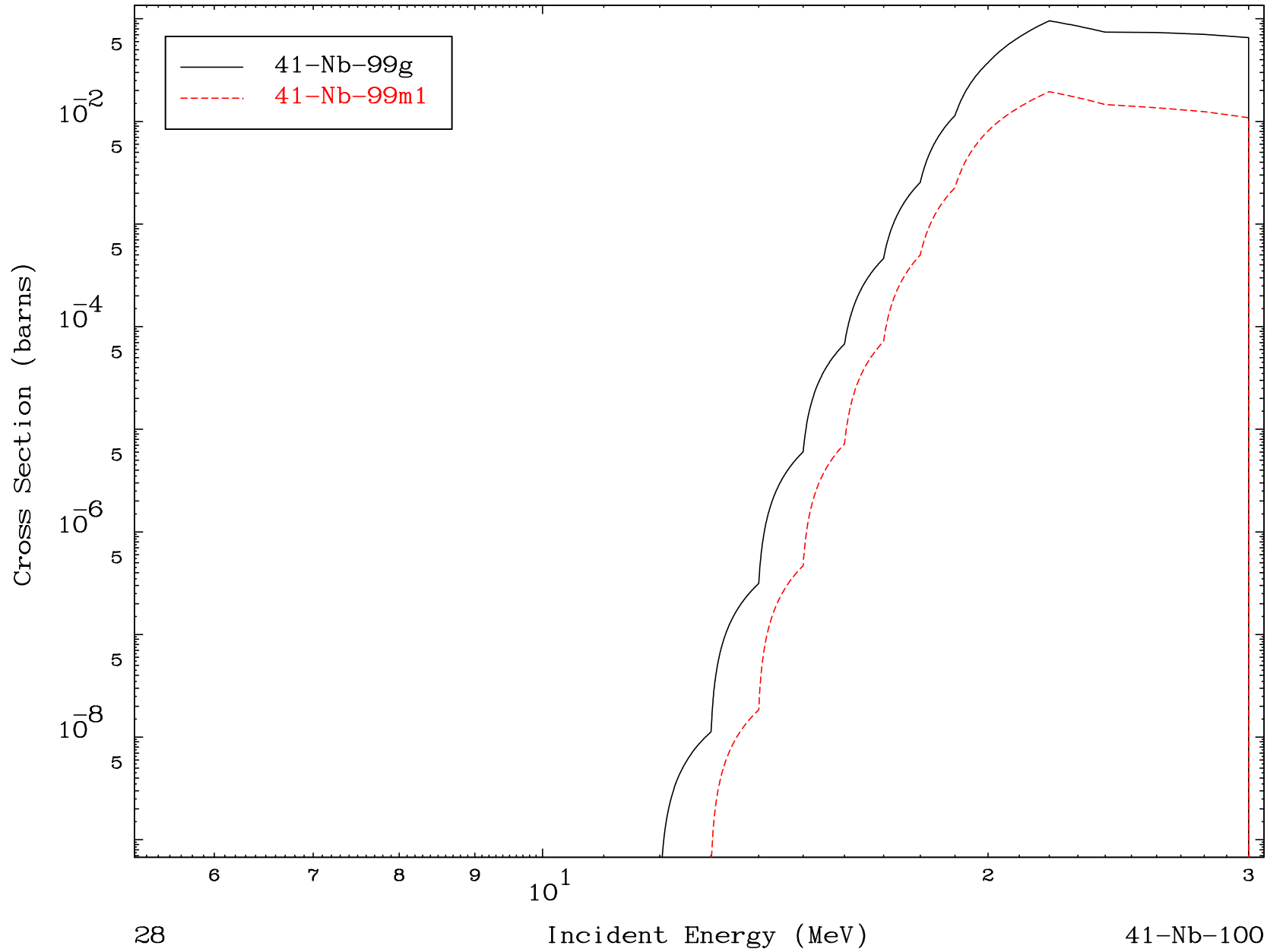


27

Incident Energy (MeV)

41-Nb-100

Radionuclide Production Cross Section

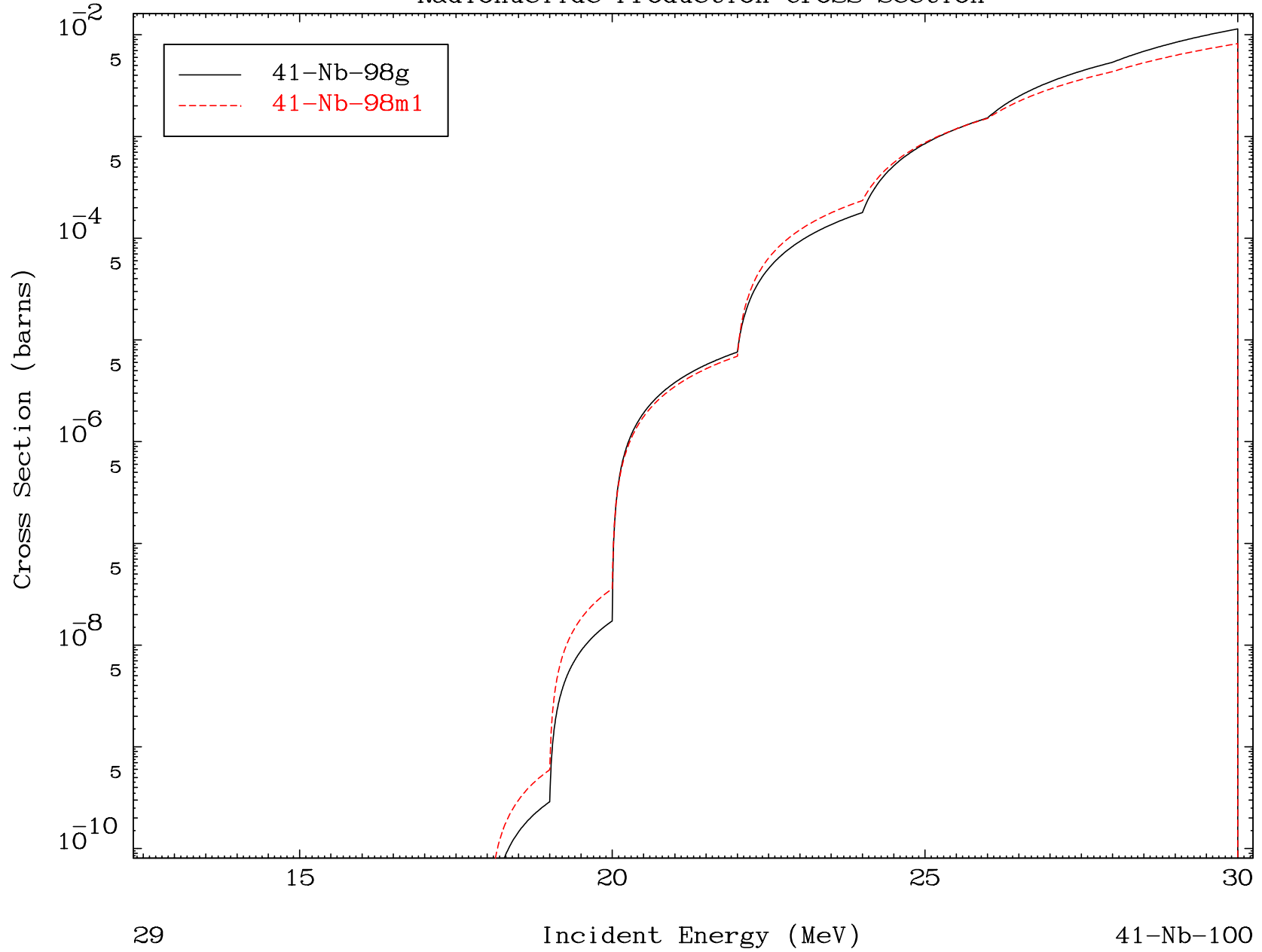


MAT 4147

($\alpha, 2n$) α

41-Nb-100

Radionuclide Production Cross Section

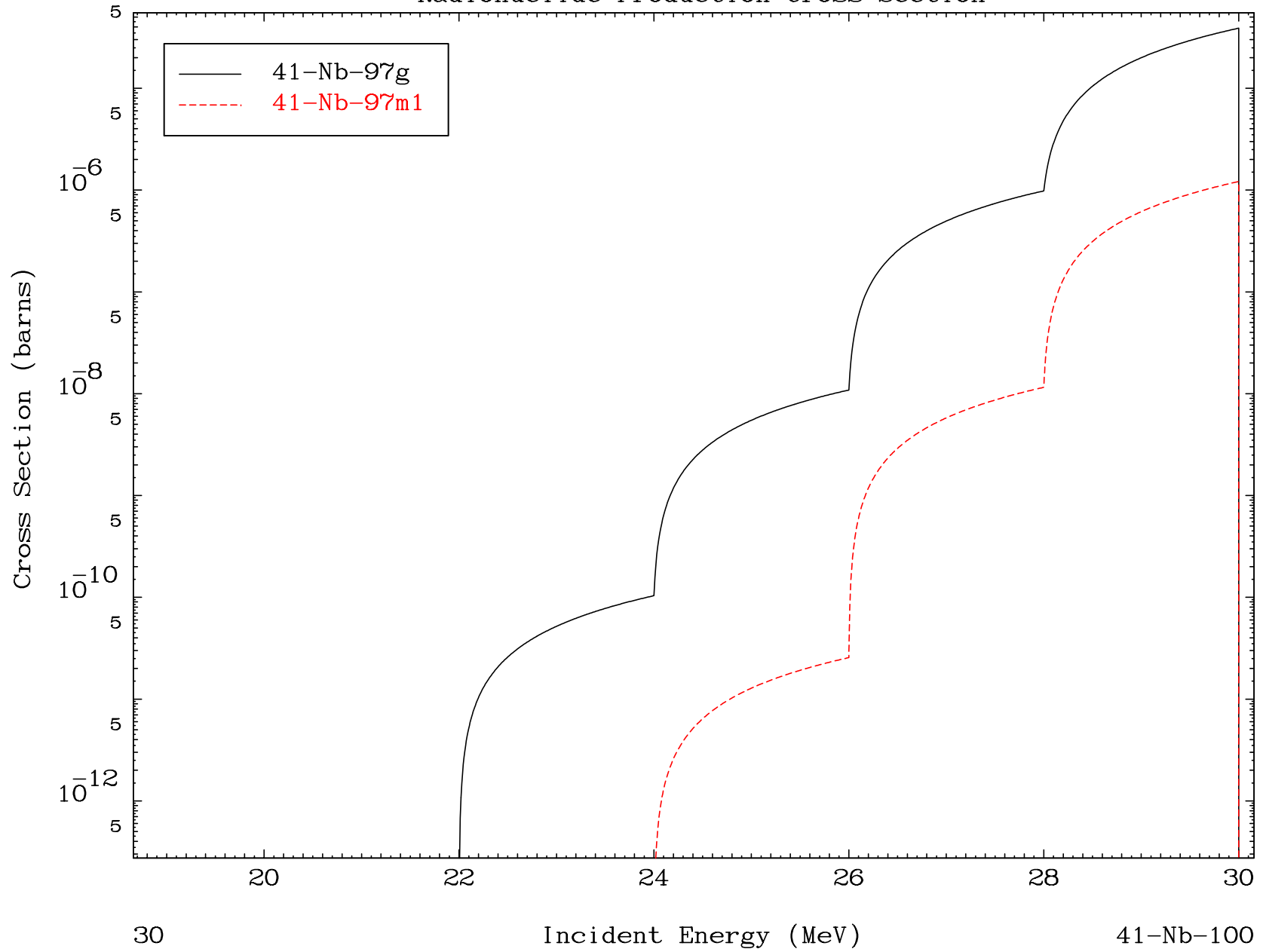


MAT 4147

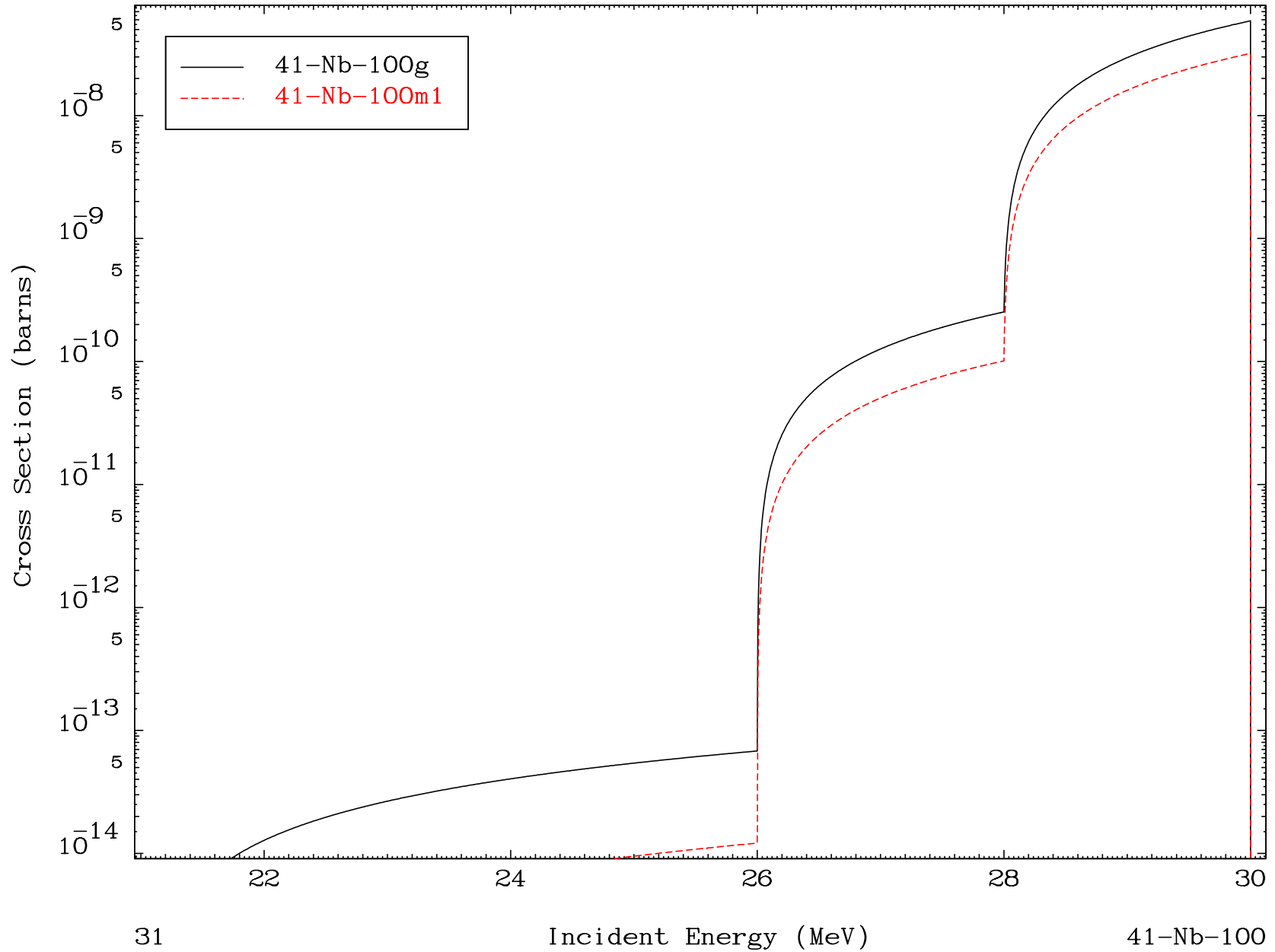
$(\alpha, 3n) \alpha$

41-Nb-100

Radionuclide Production Cross Section



Radionuclide Production Cross Section

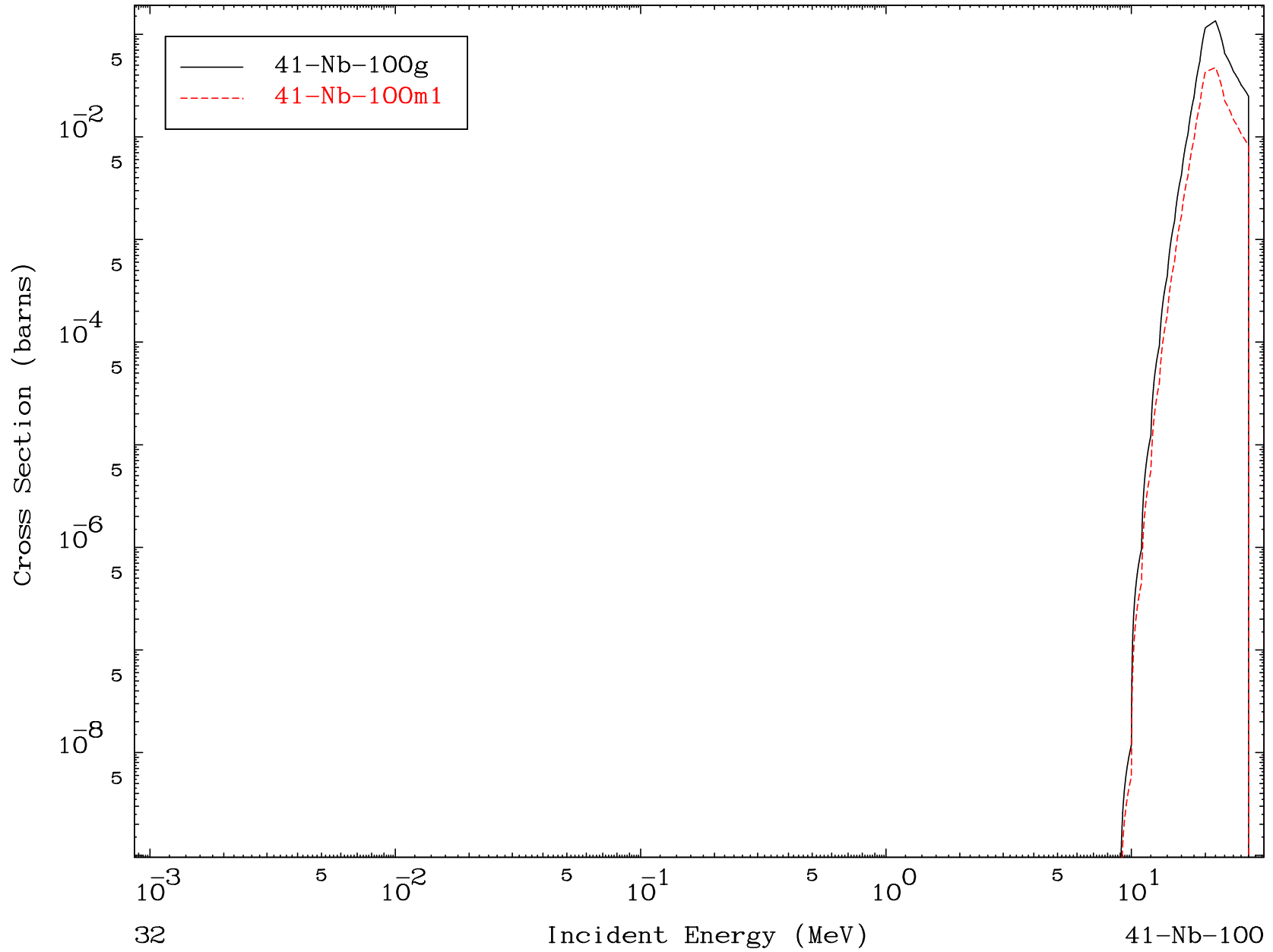


MAT 4147

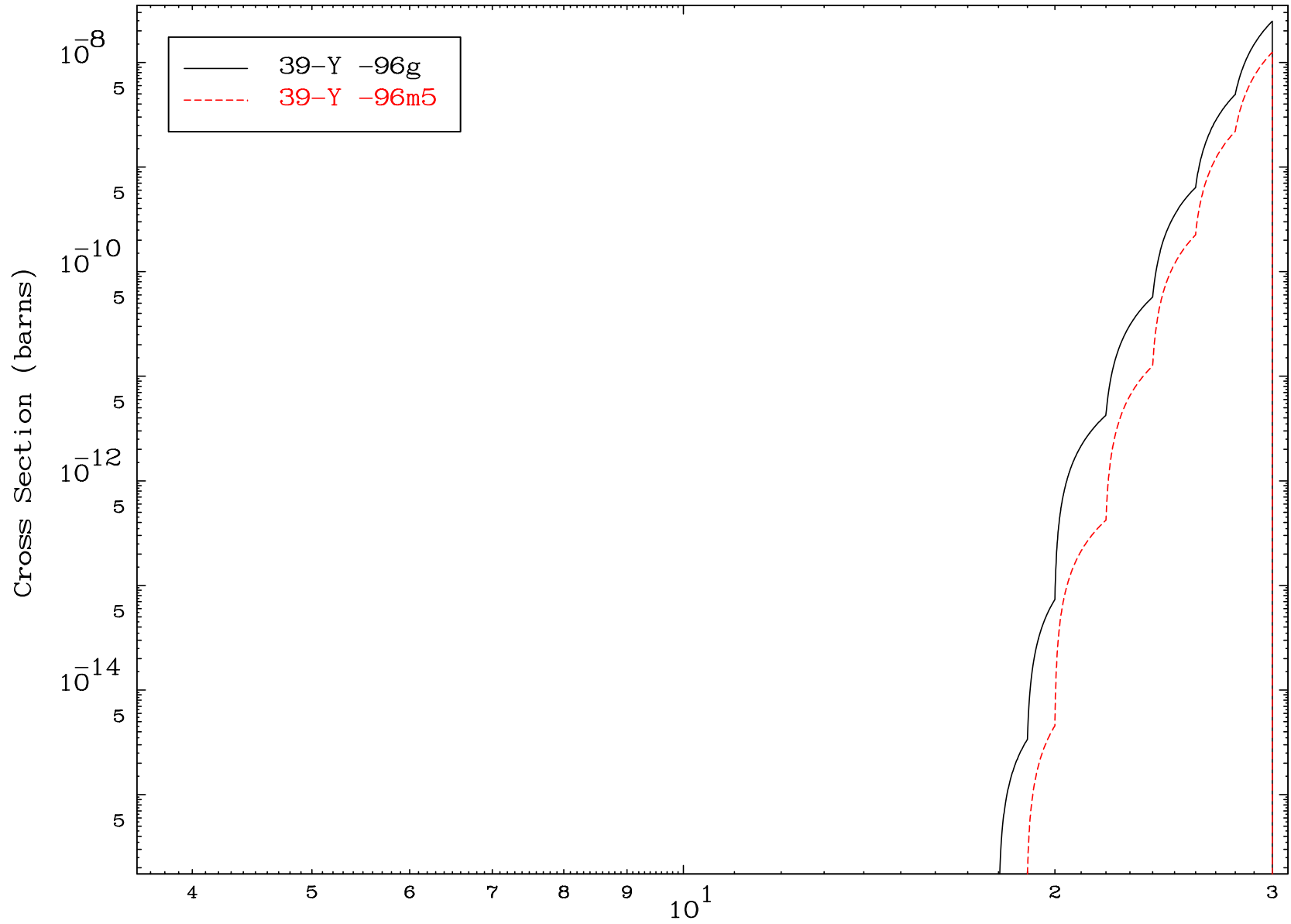
(α, α)

41-Nb-100

Radionuclide Production Cross Section



Radionuclide Production Cross Section



MAT 4147

($\alpha, 2p$)

41-Nb-100

Radionuclide Production Cross Section

