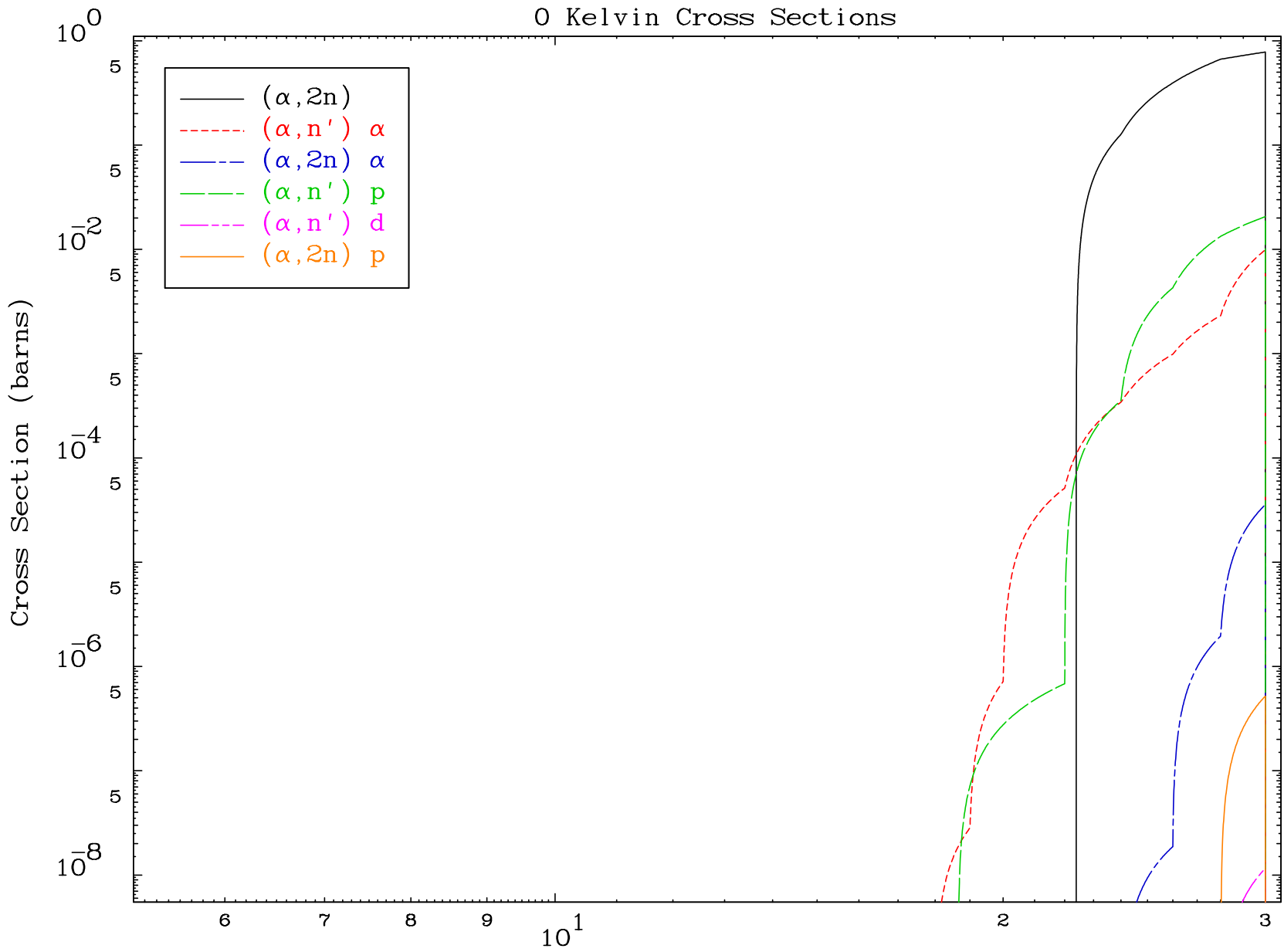


MAT 7613

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

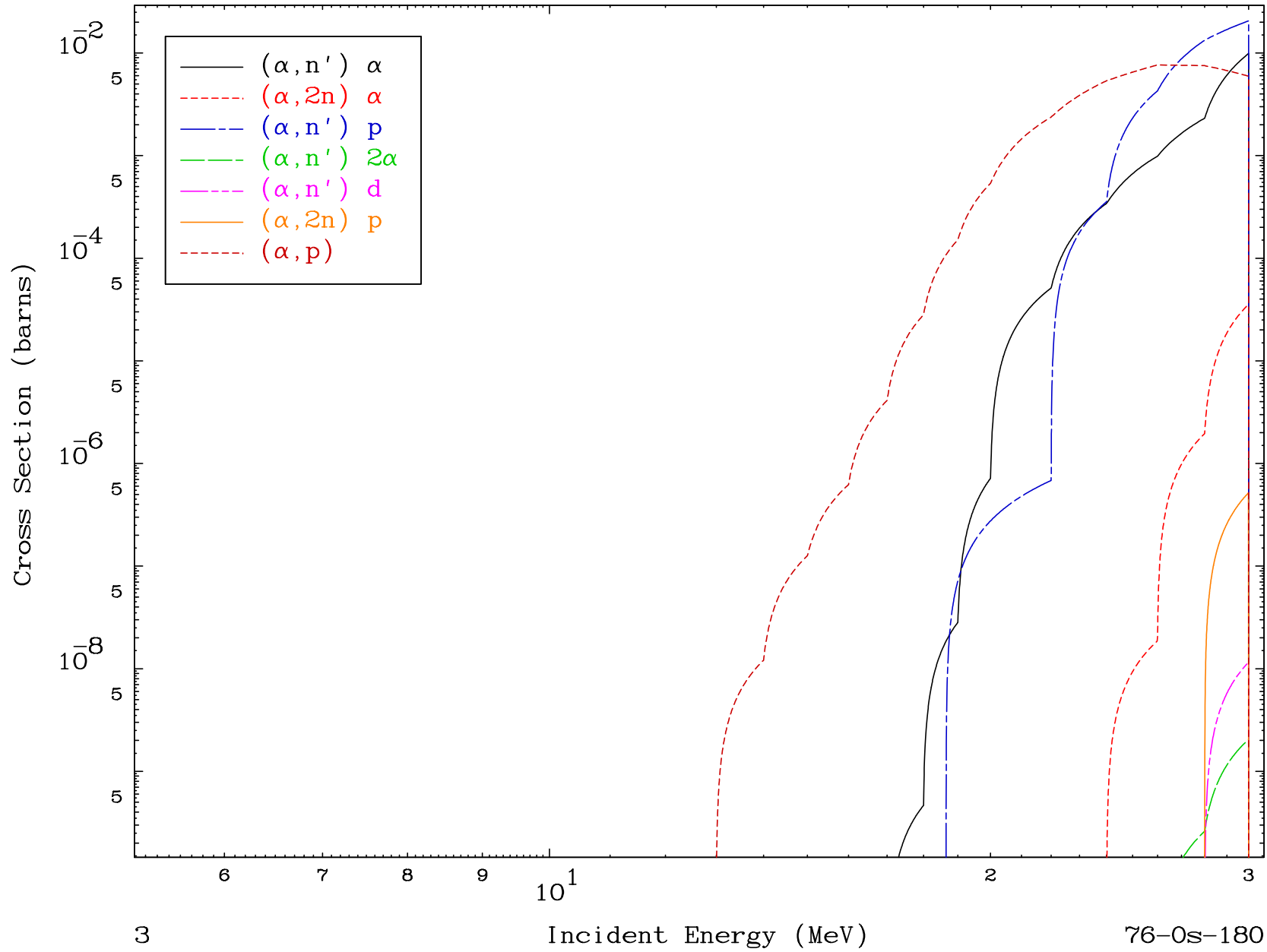
76-0s-180

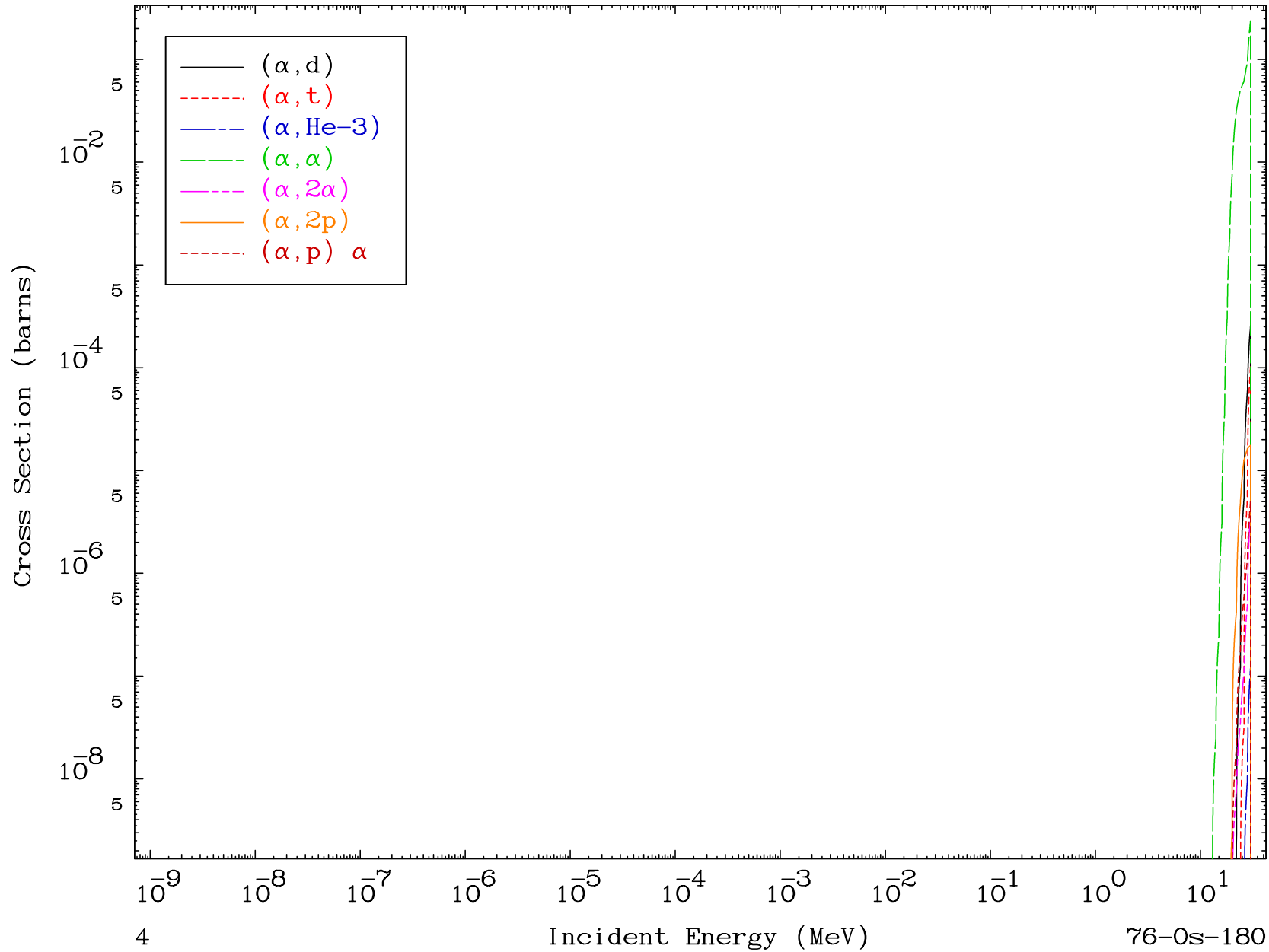


2

Incident Energy (MeV)

76-0s-180

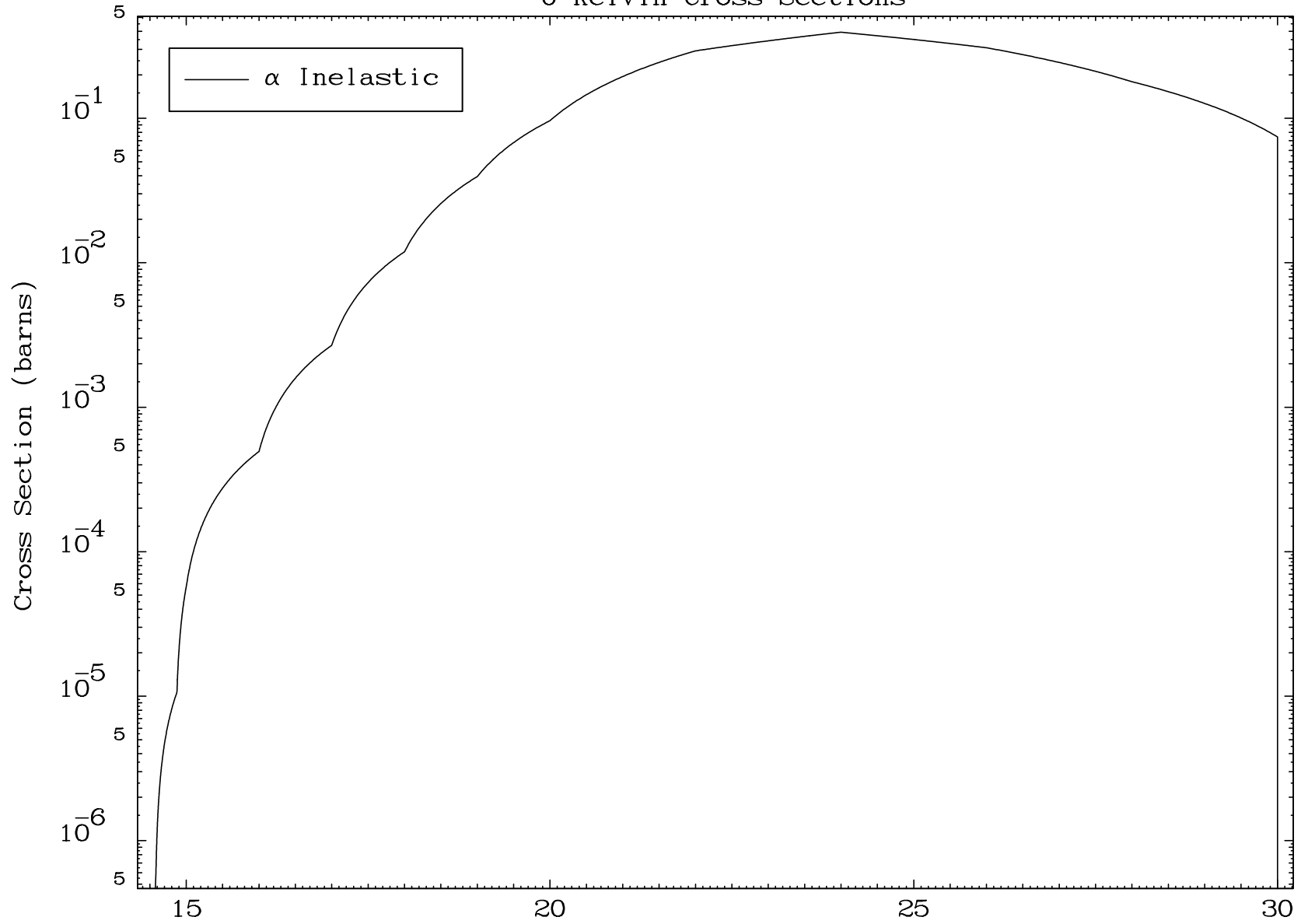




MAT 7613

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

76-0s-180



5

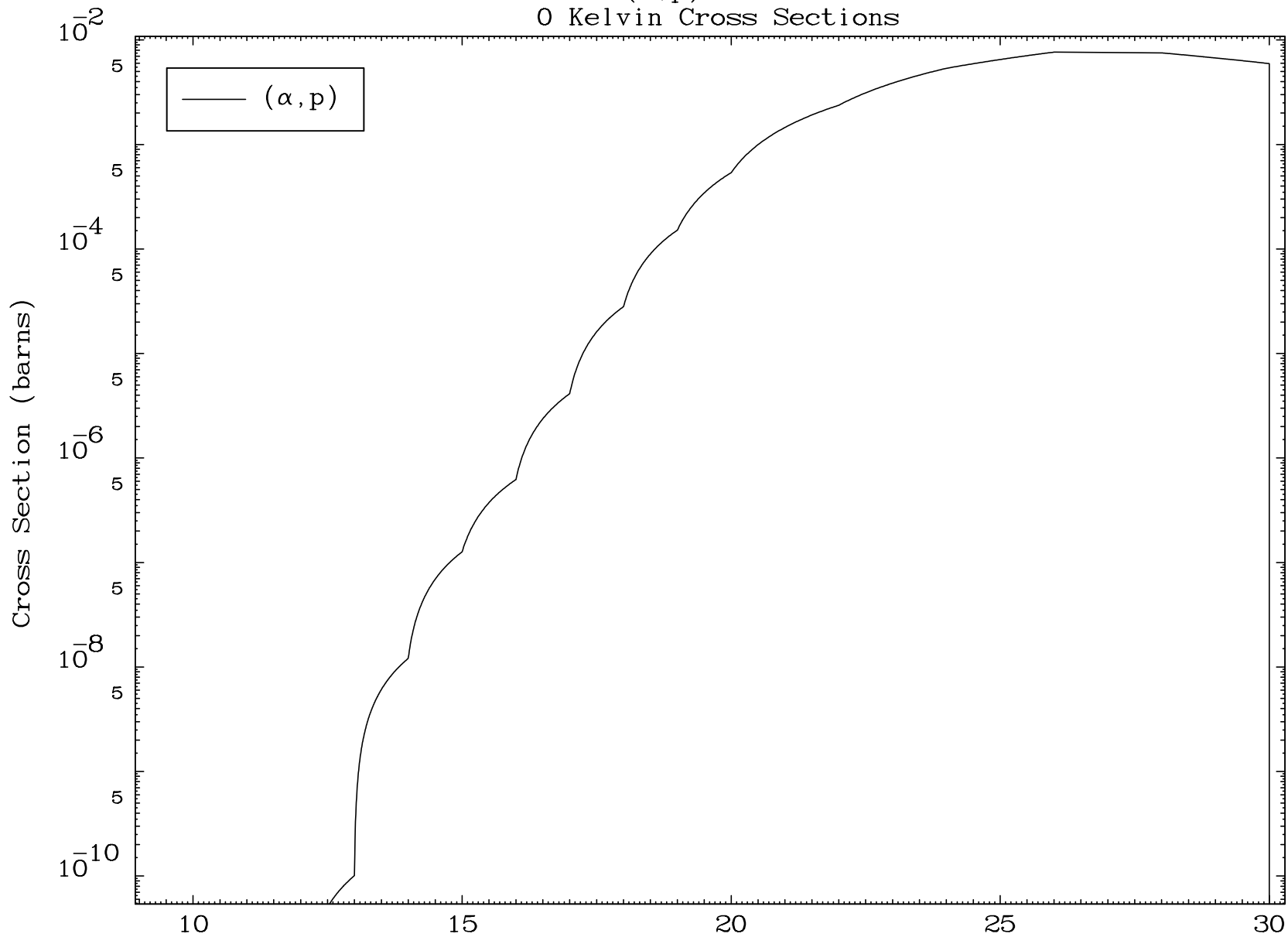
Incident Energy (MeV)

76-0s-180

MAT 7613

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections

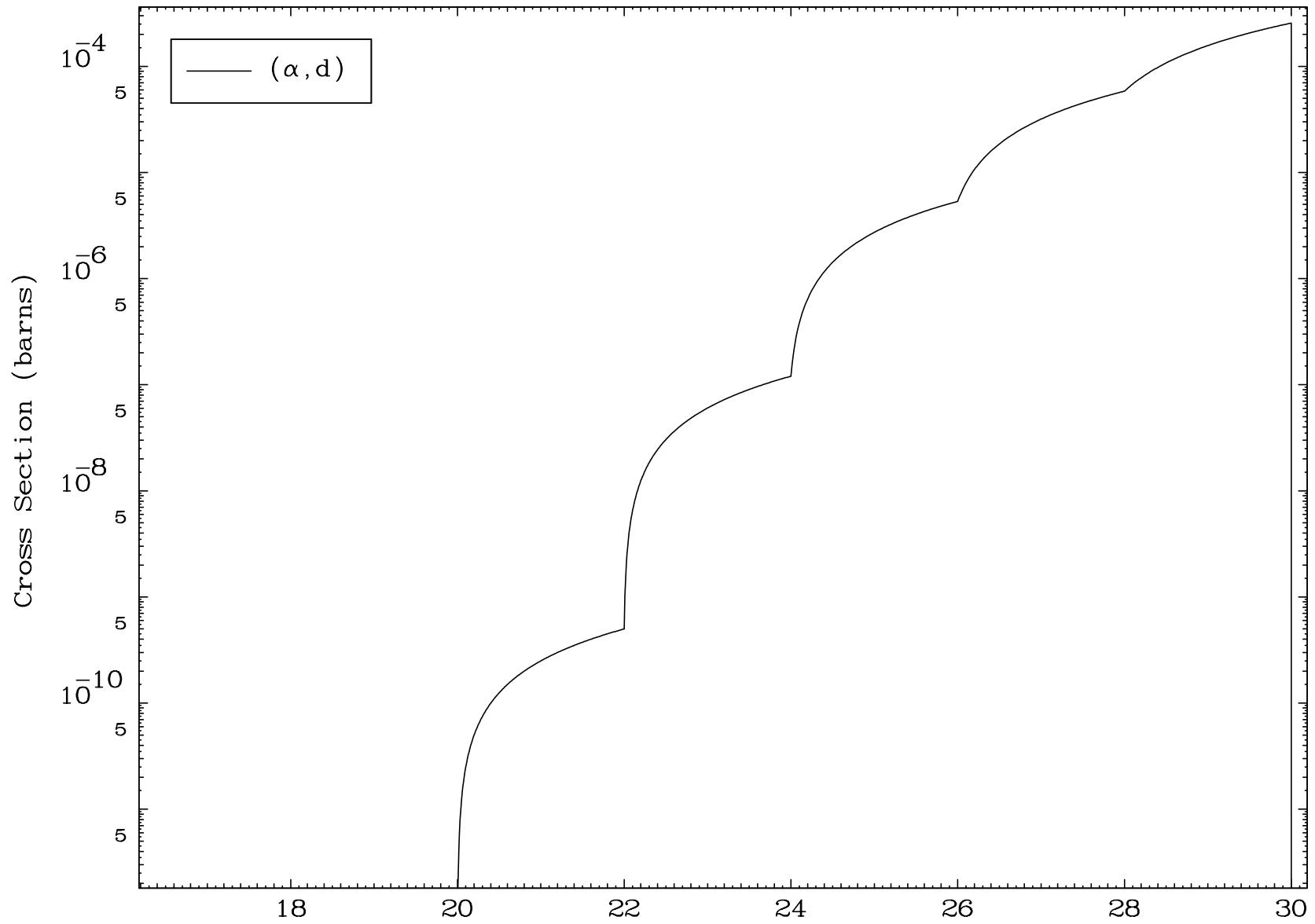
76-0s-180



6

Incident Energy (MeV)

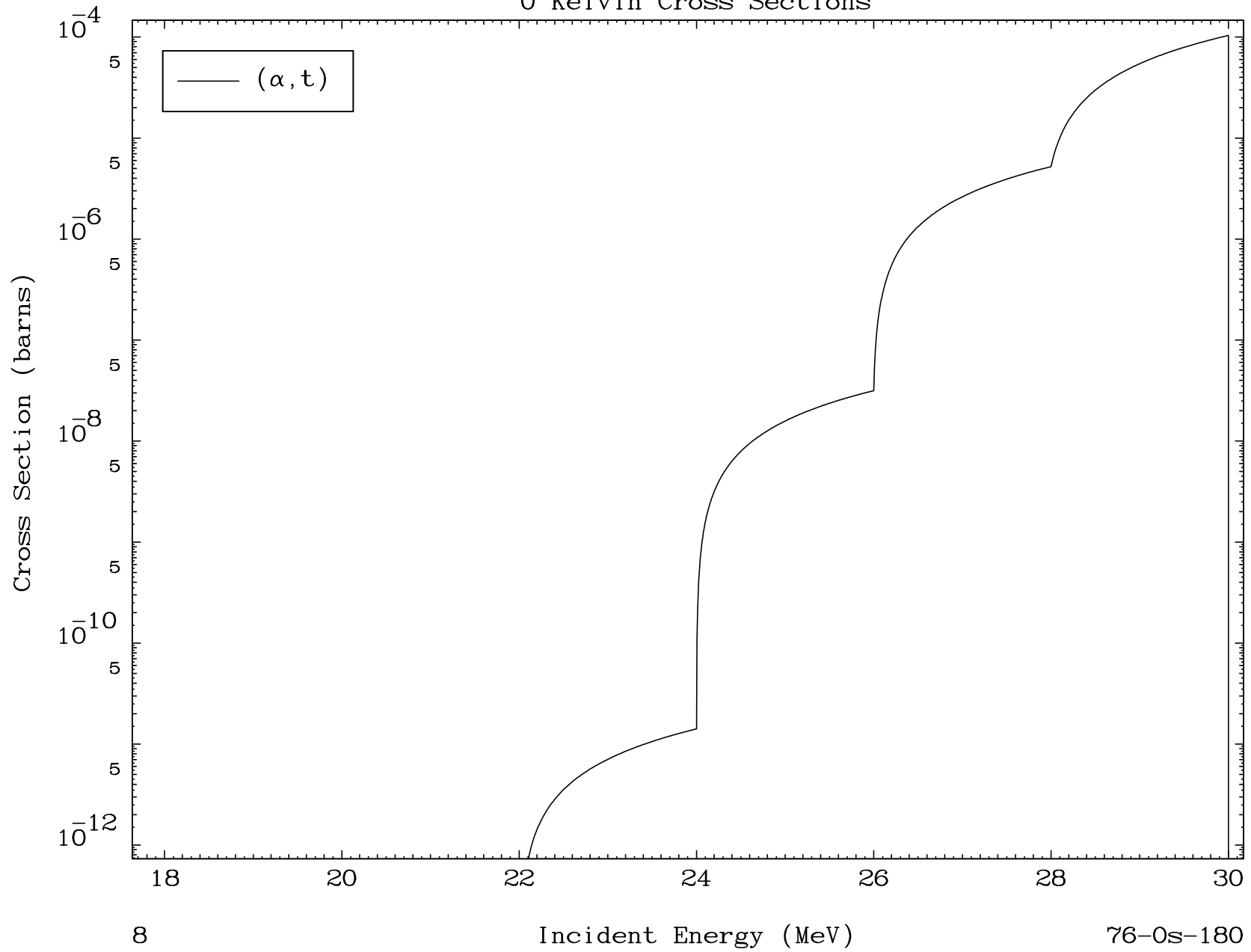
76-0s-180



MAT 7613

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

76-0s-180

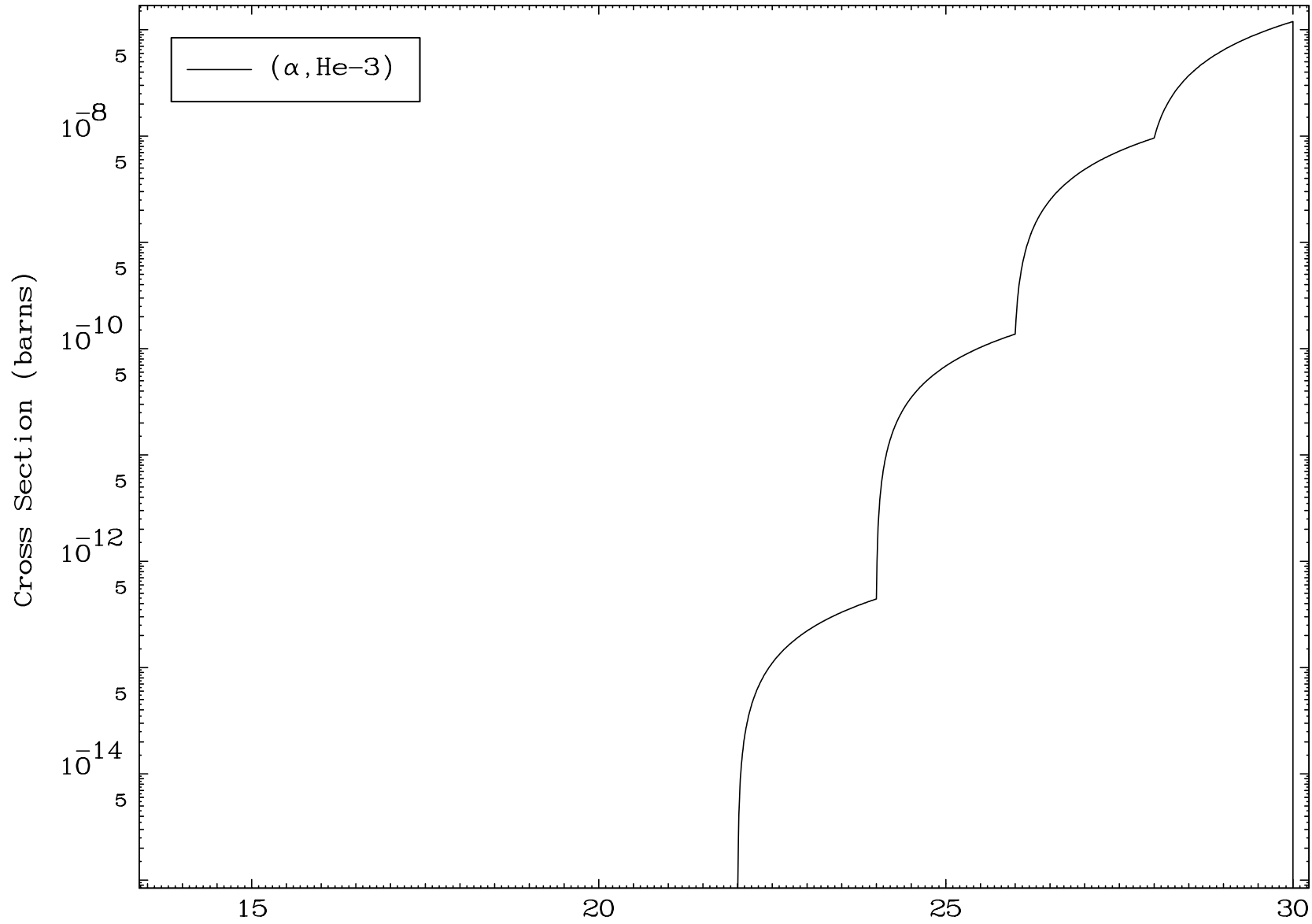


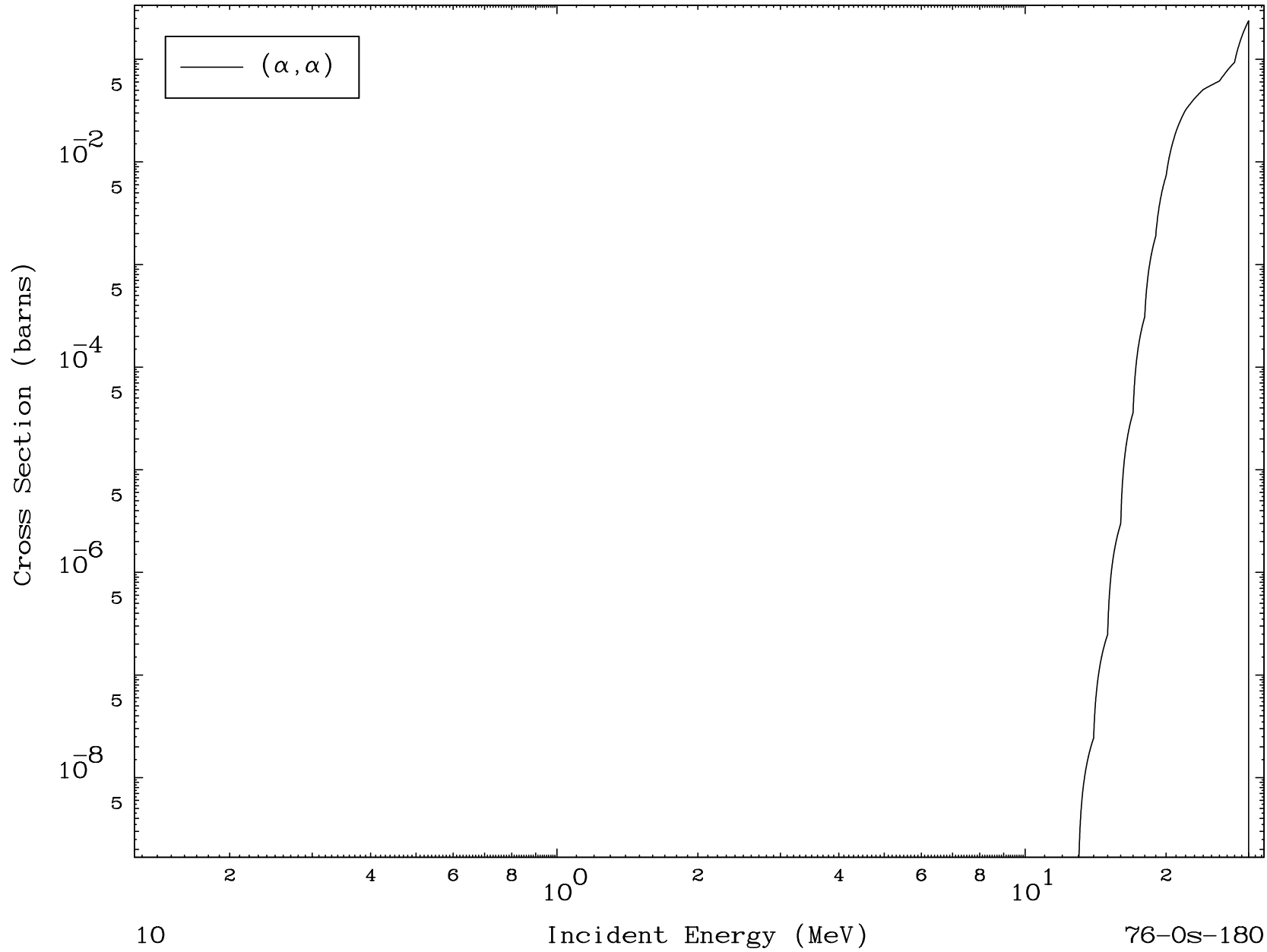
8

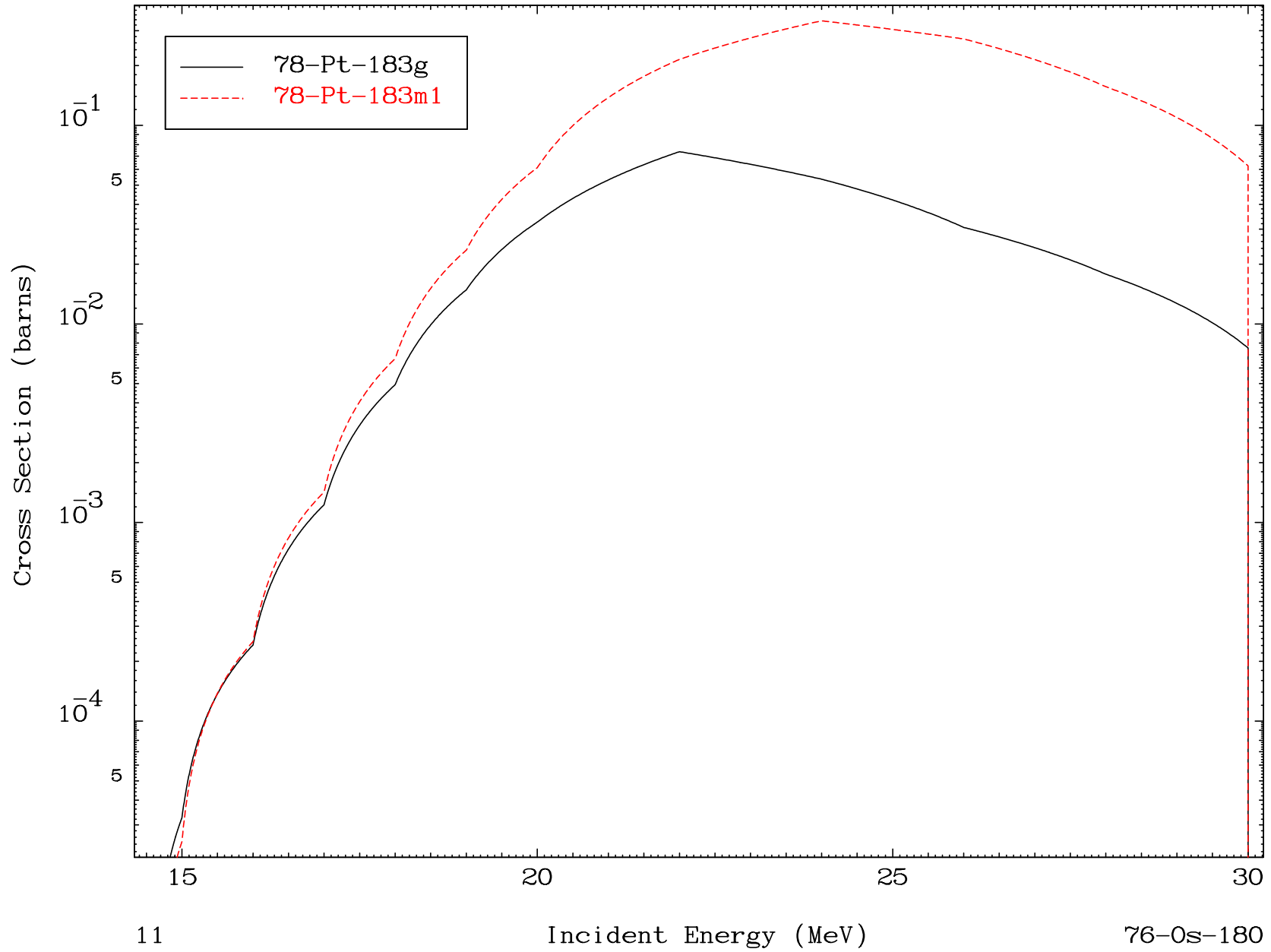
Incident Energy (MeV)

76-0s-180

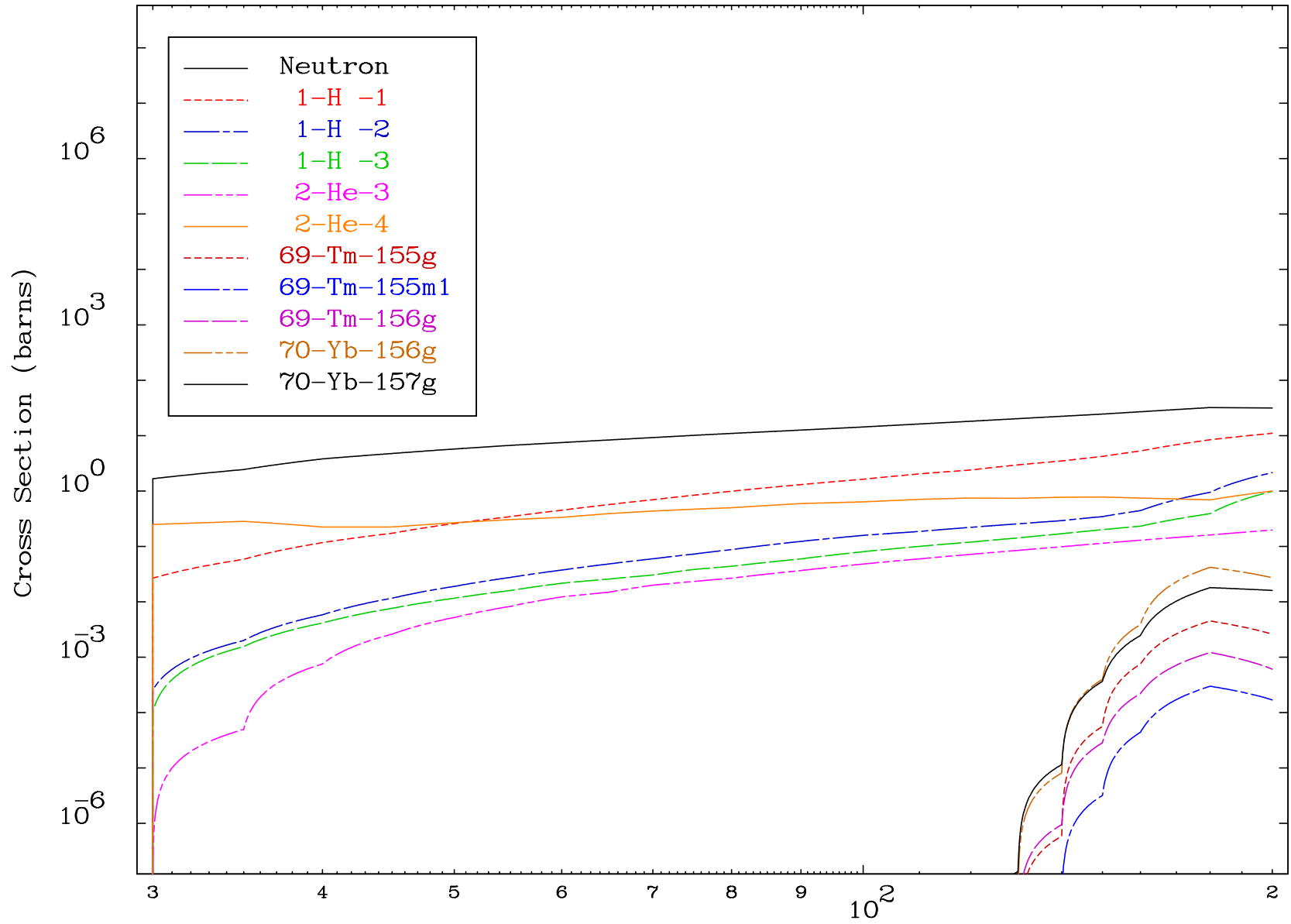




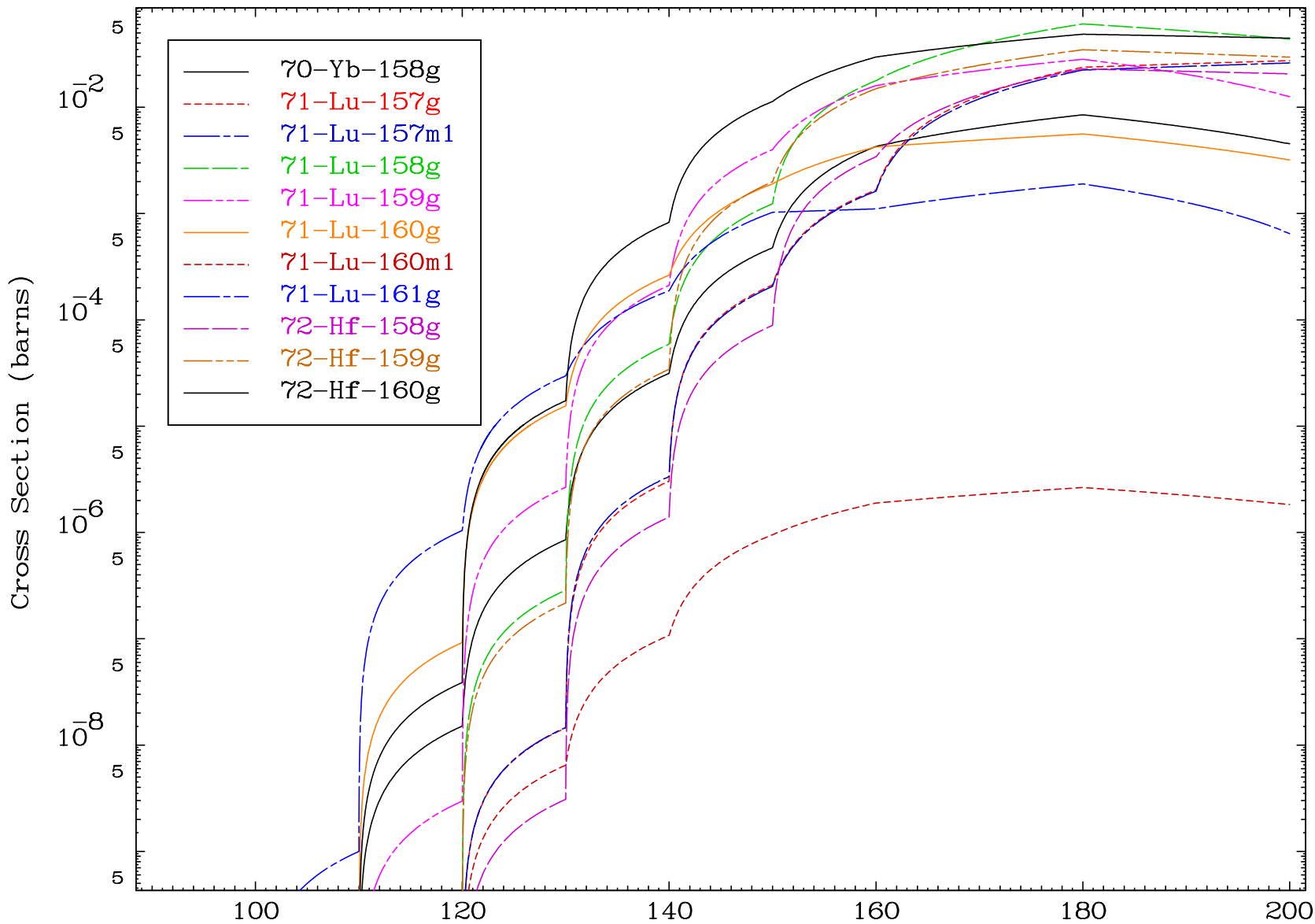


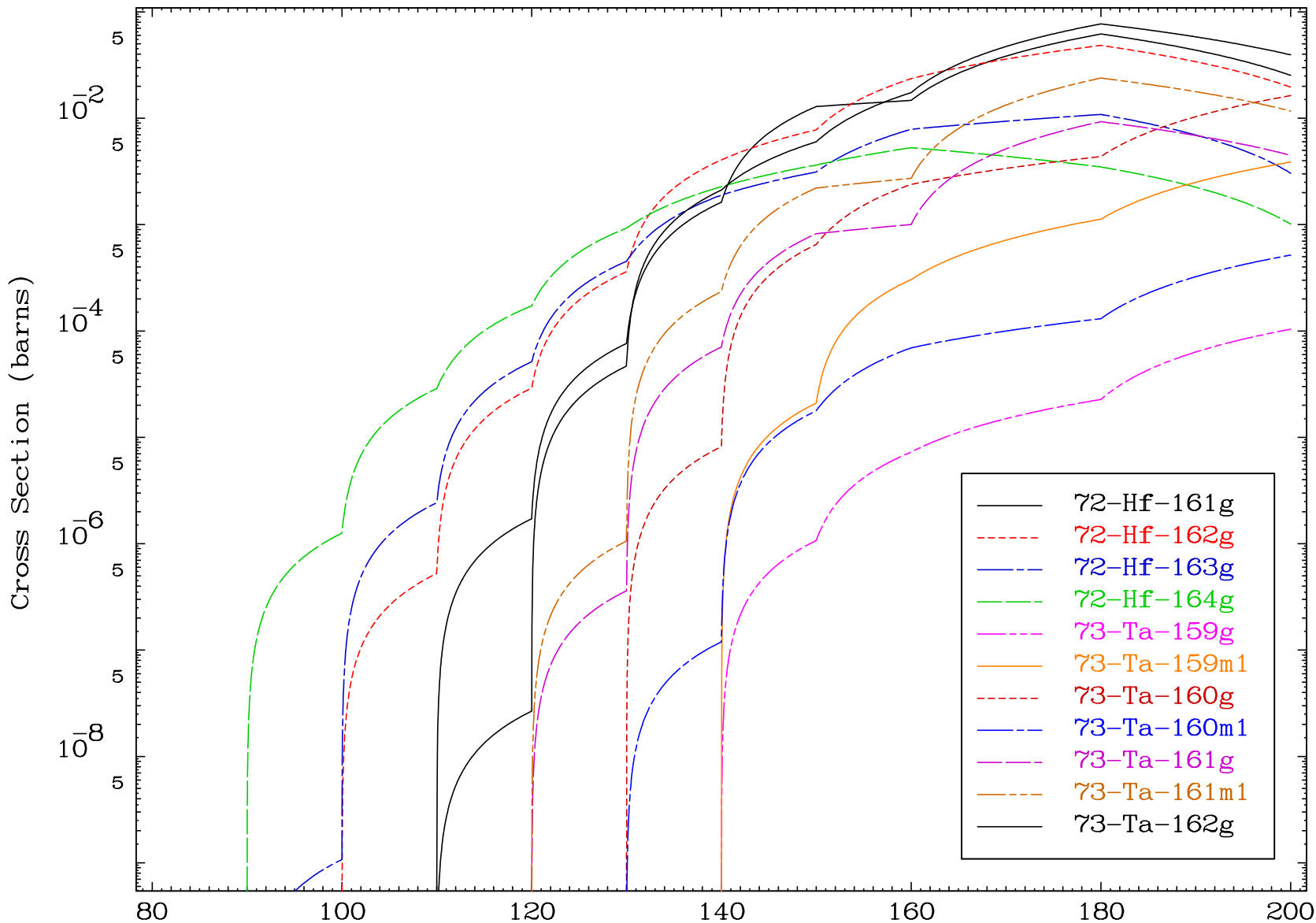


Radionuclide Production Cross Section

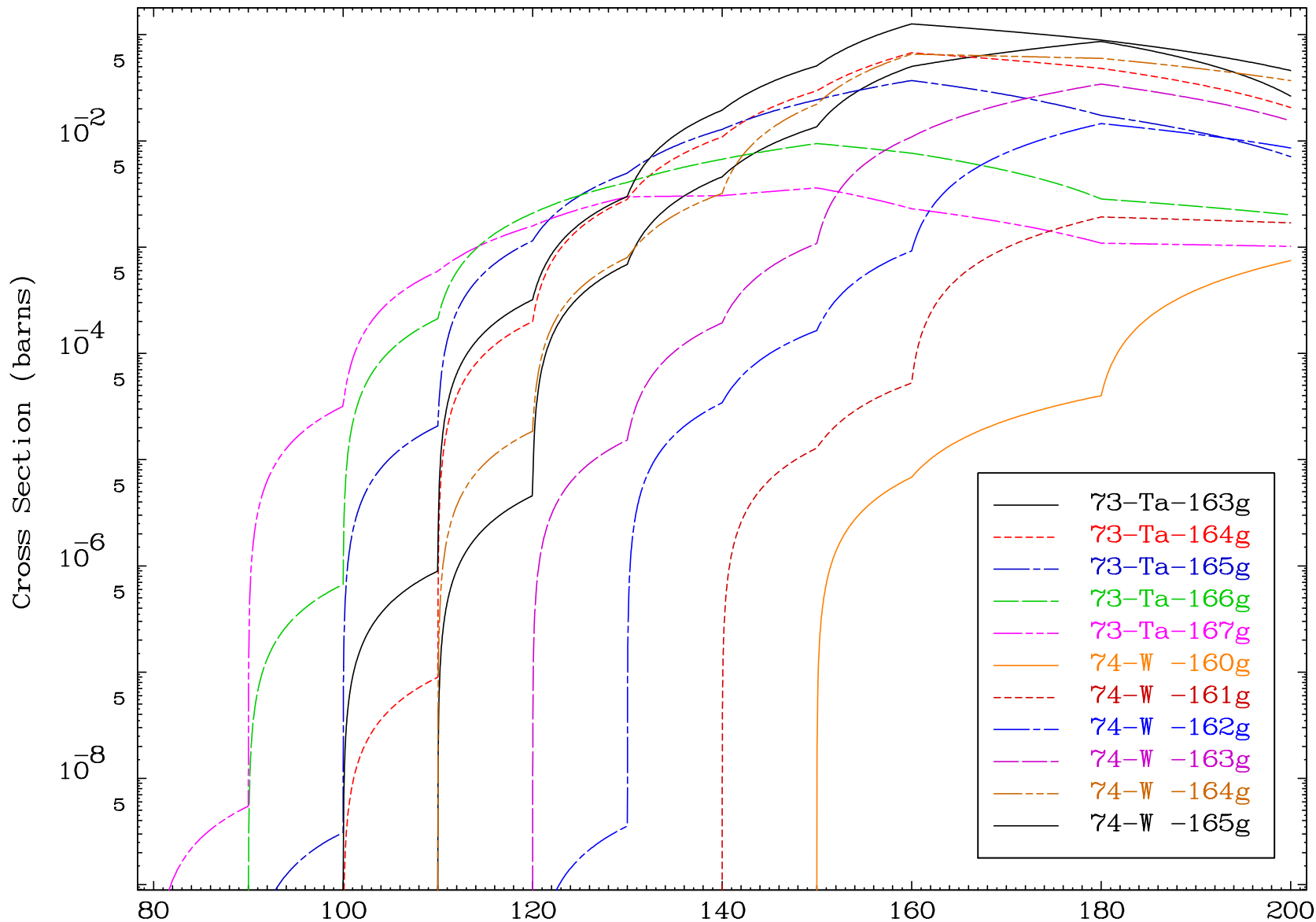


Radionuclide Production Cross Section

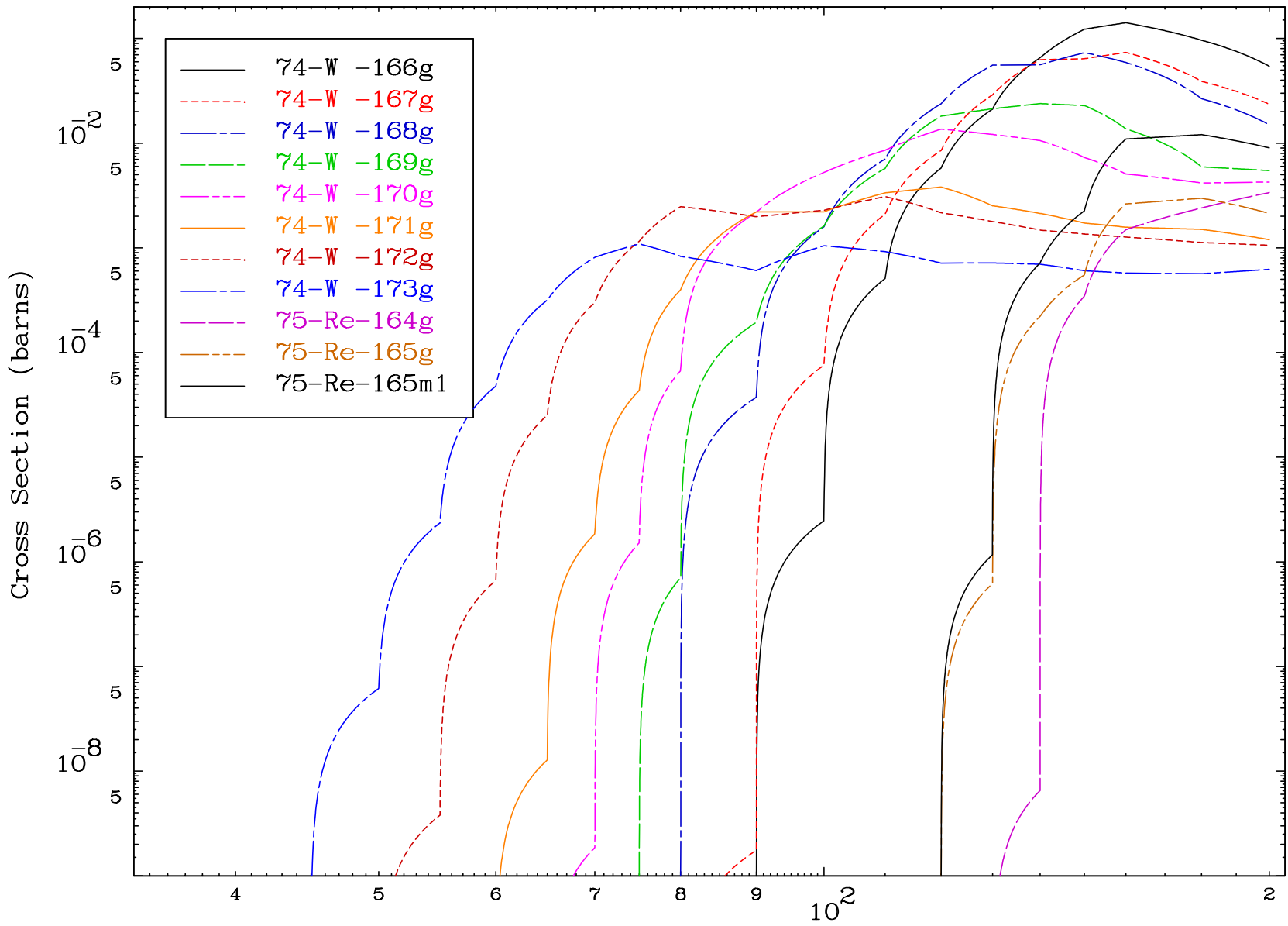




Radionuclide Production Cross Section

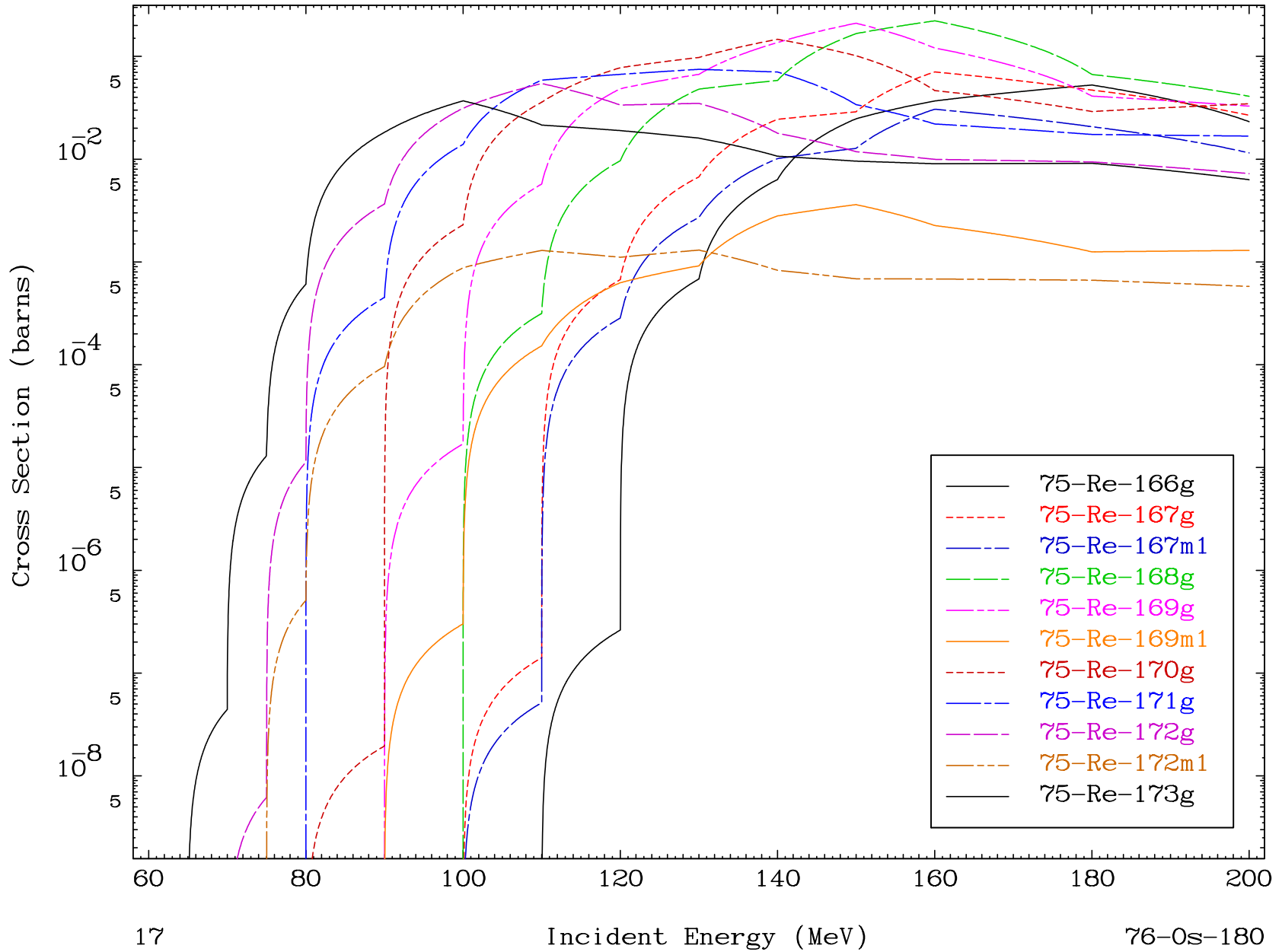


Radionuclide Production Cross Section

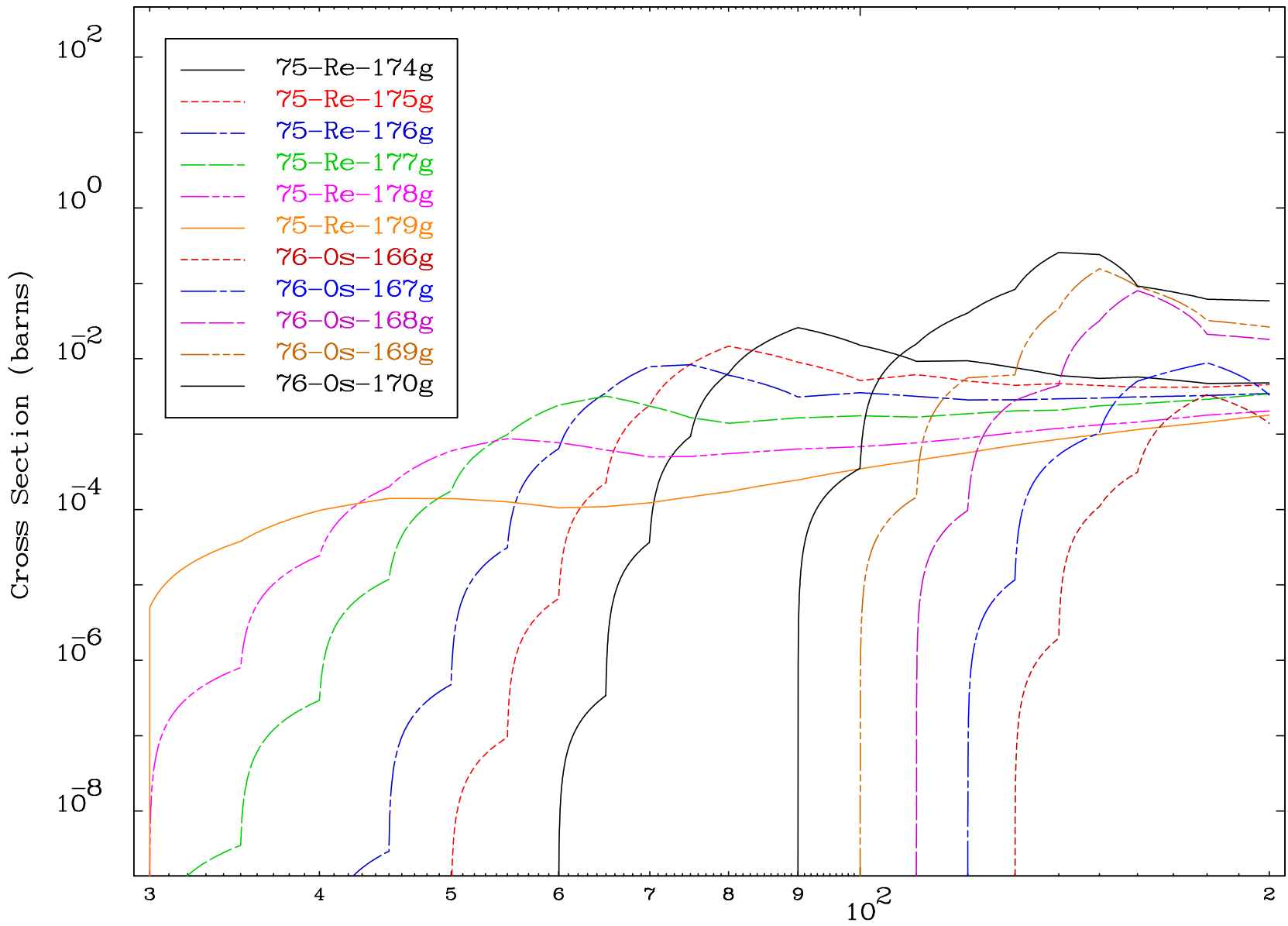




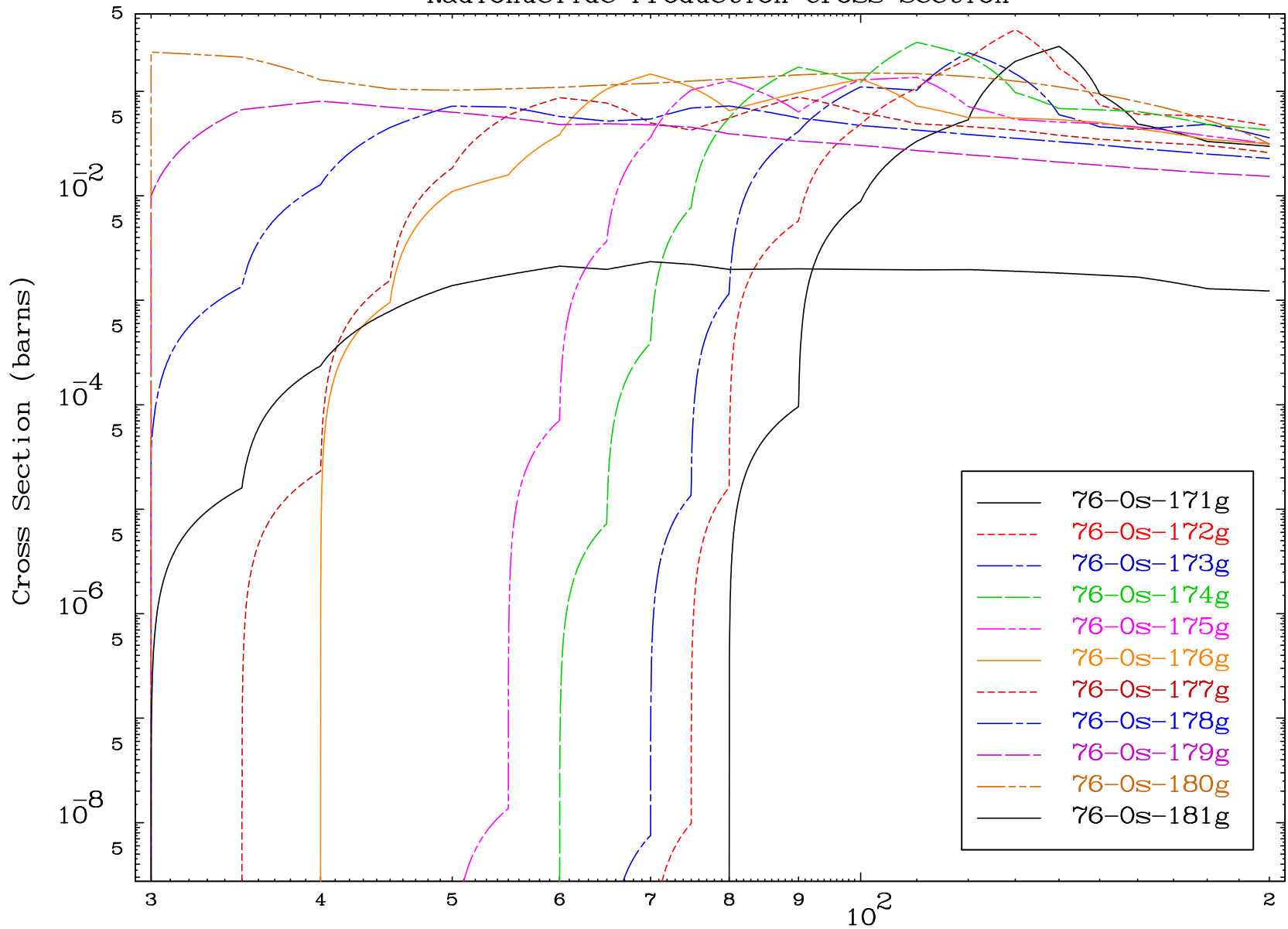
Radionuclide Production Cross Section



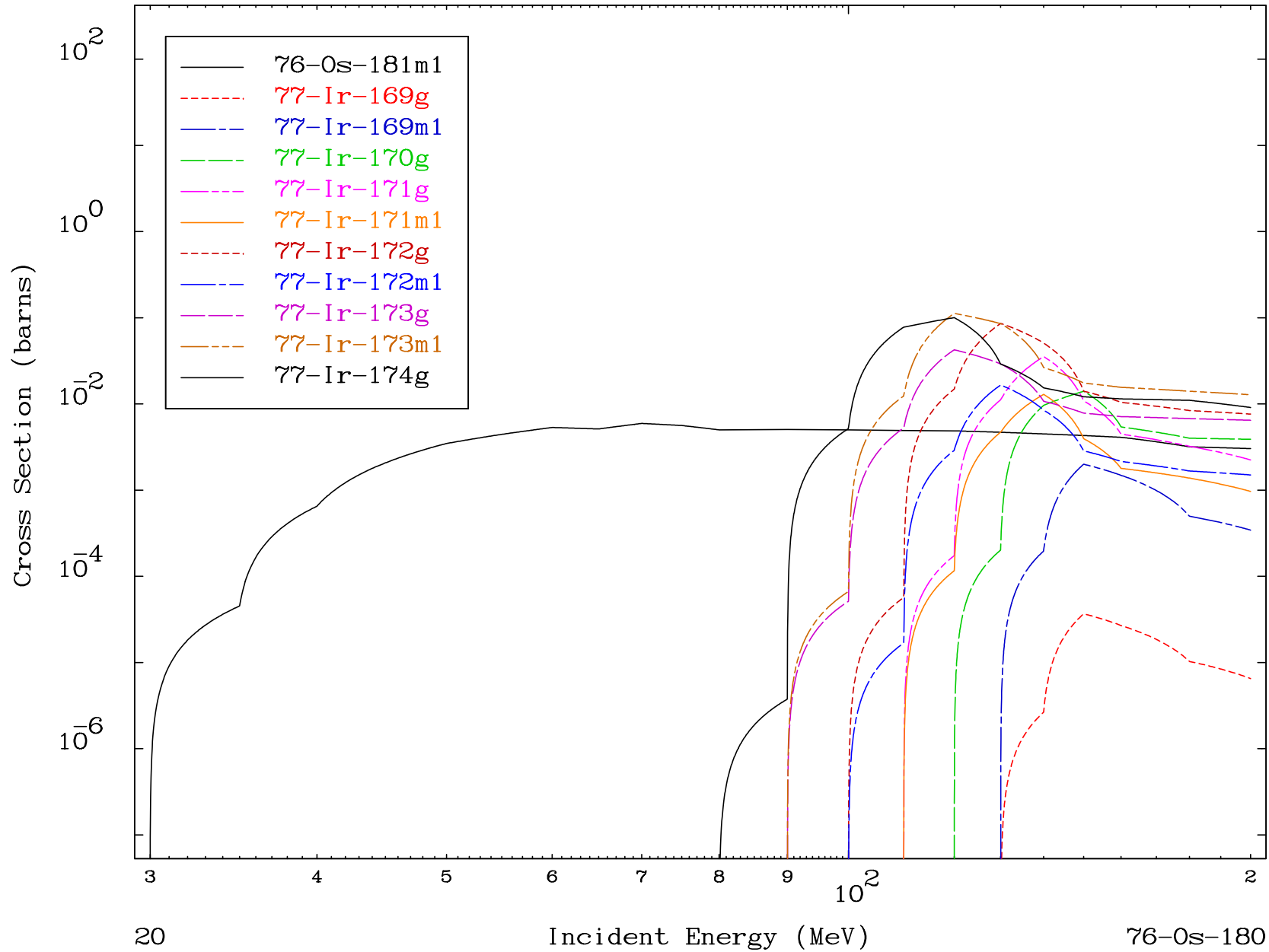
Radionuclide Production Cross Section



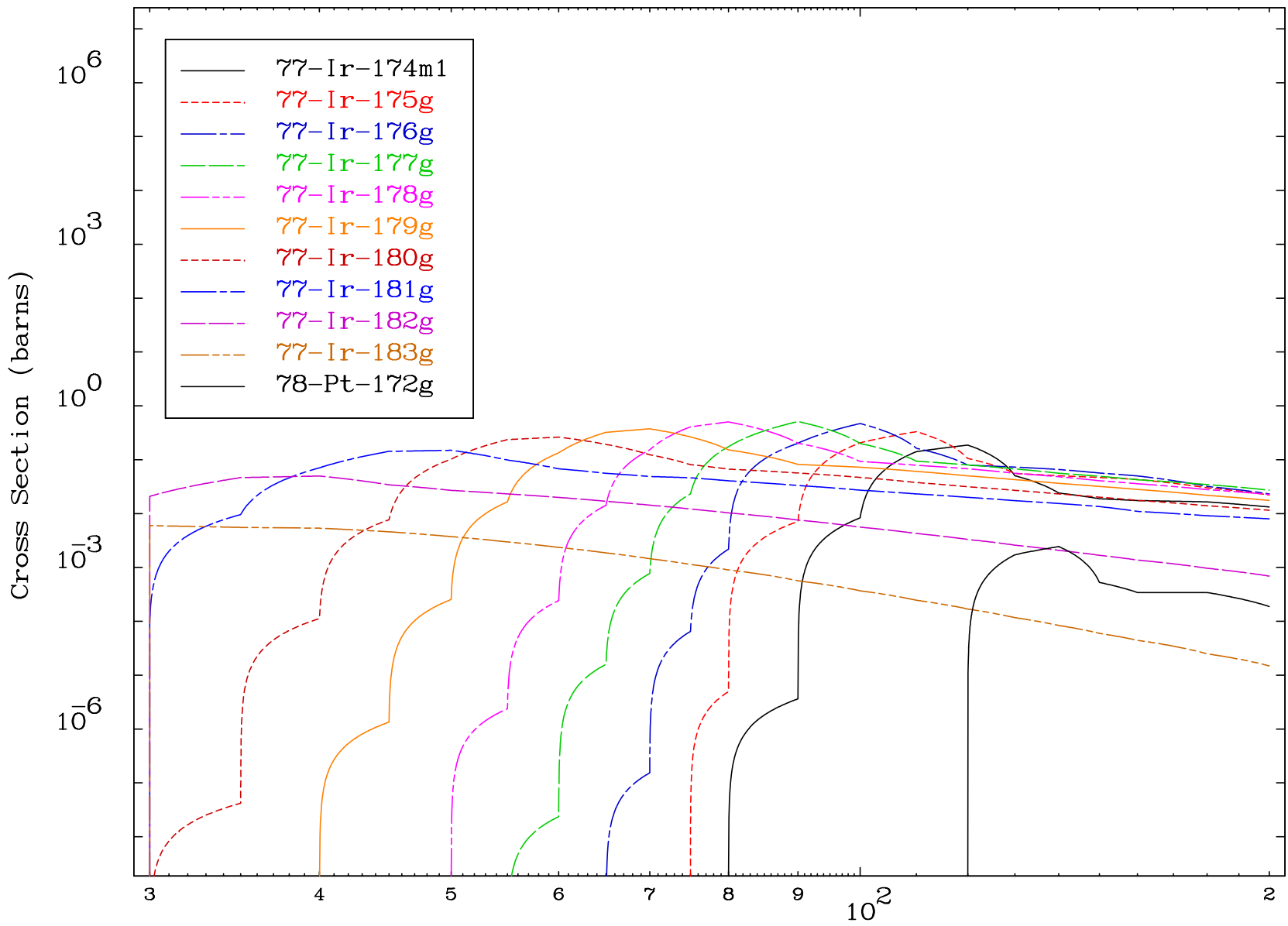
Radionuclide Production Cross Section

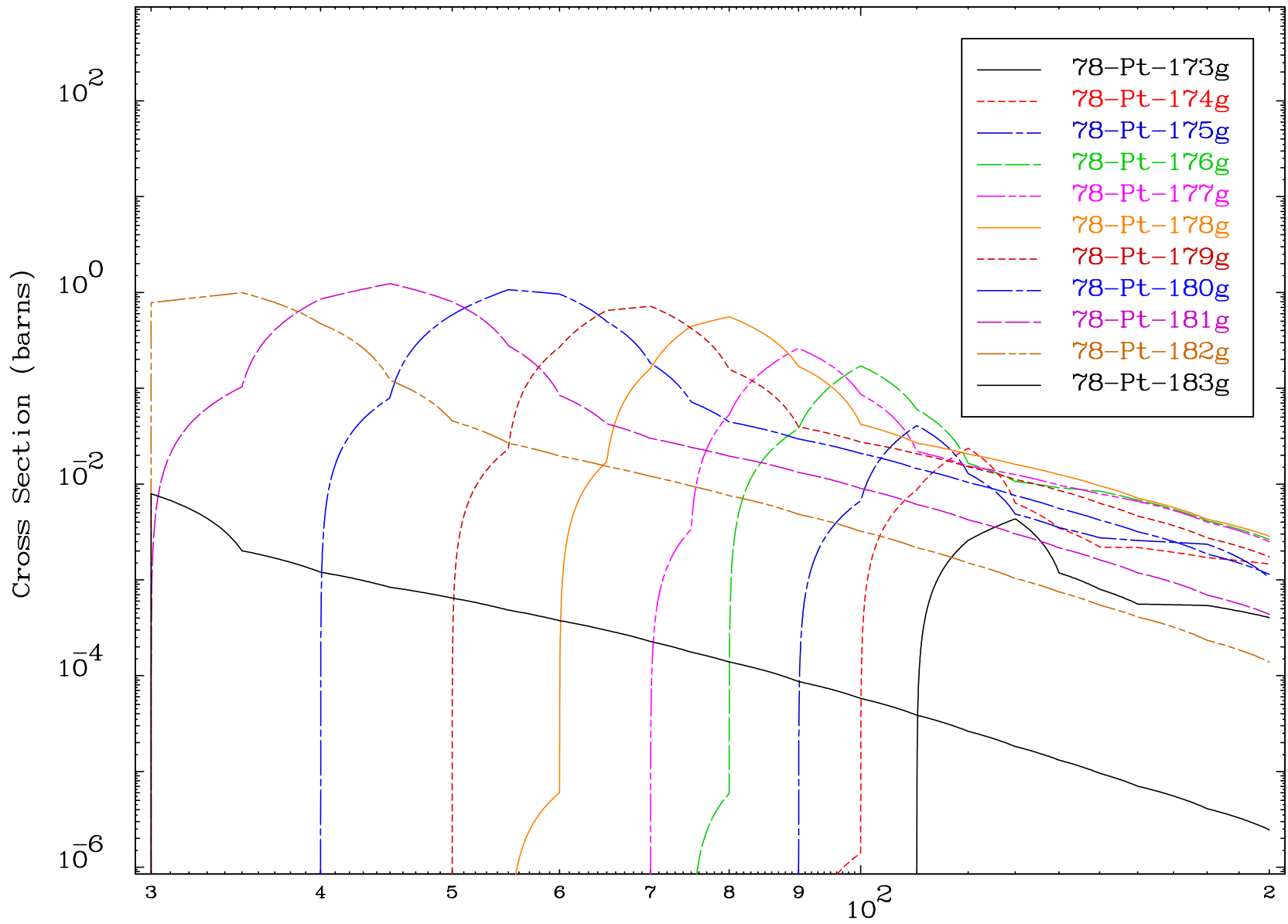


Radionuclide Production Cross Section



Radionuclide Production Cross Section



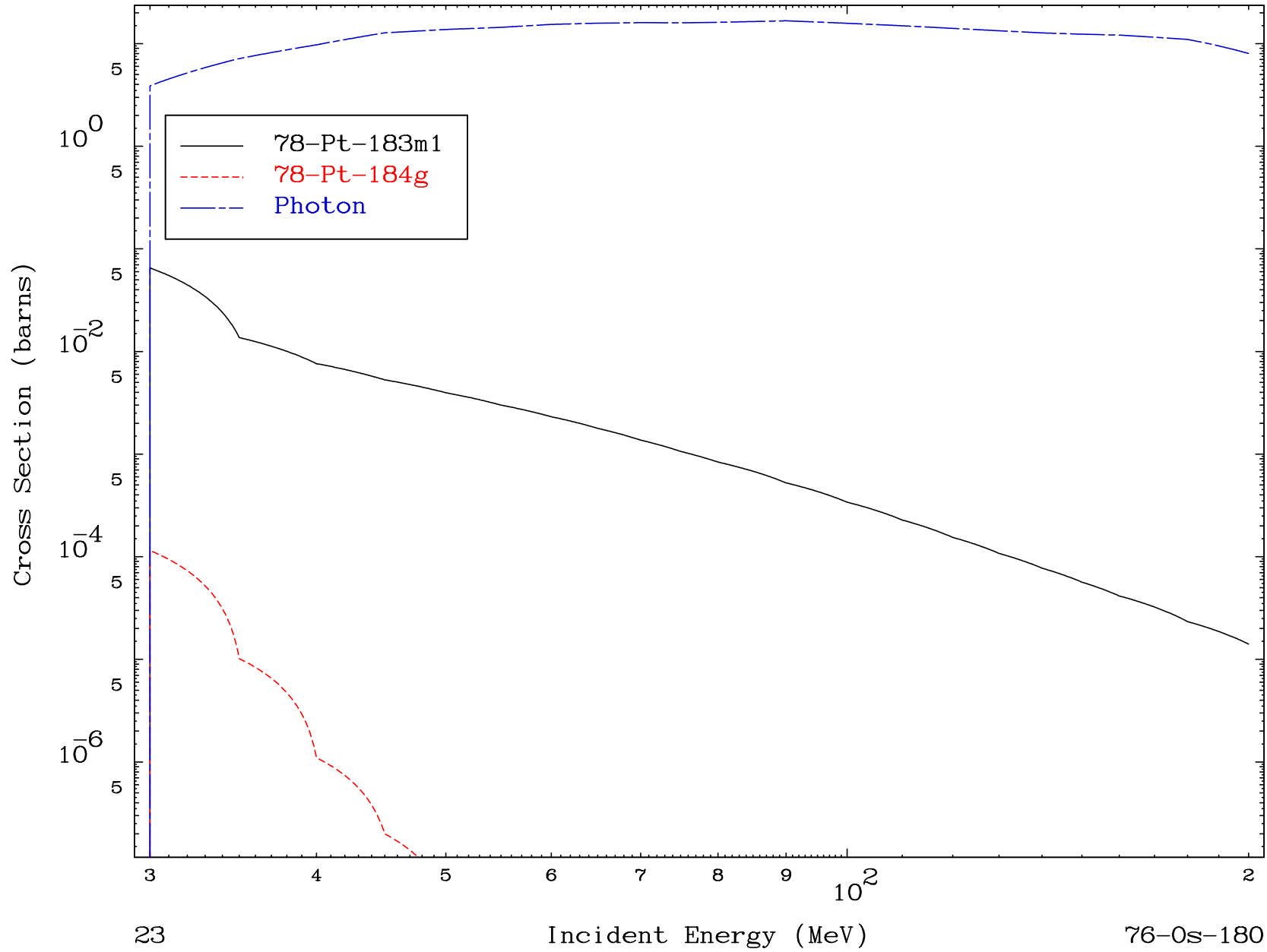


MAT 7613

( $\alpha$ , remainder)

76-0s-180

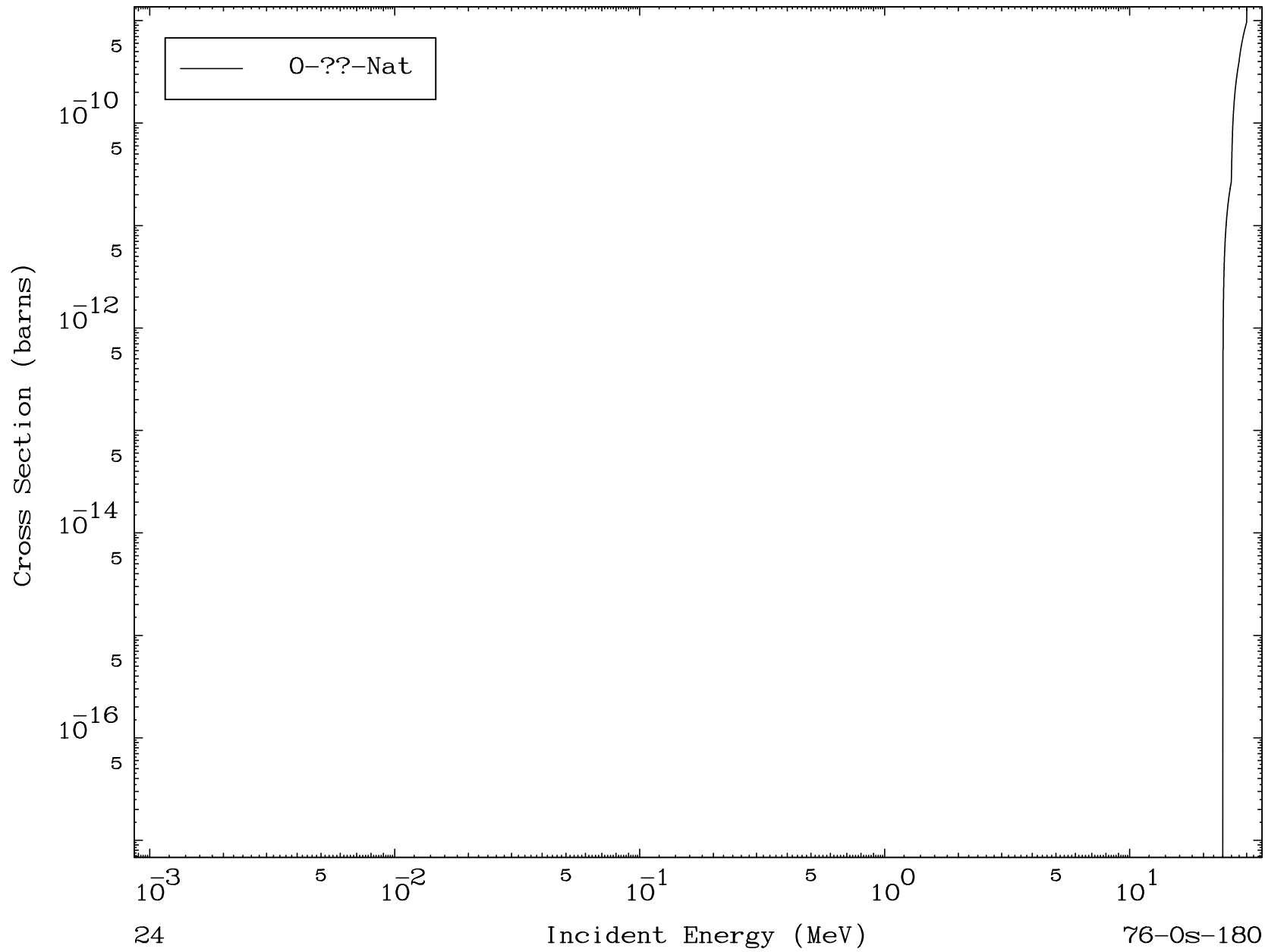
### Radionuclide Production Cross Section



23

Incident Energy (MeV)

76-0s-180



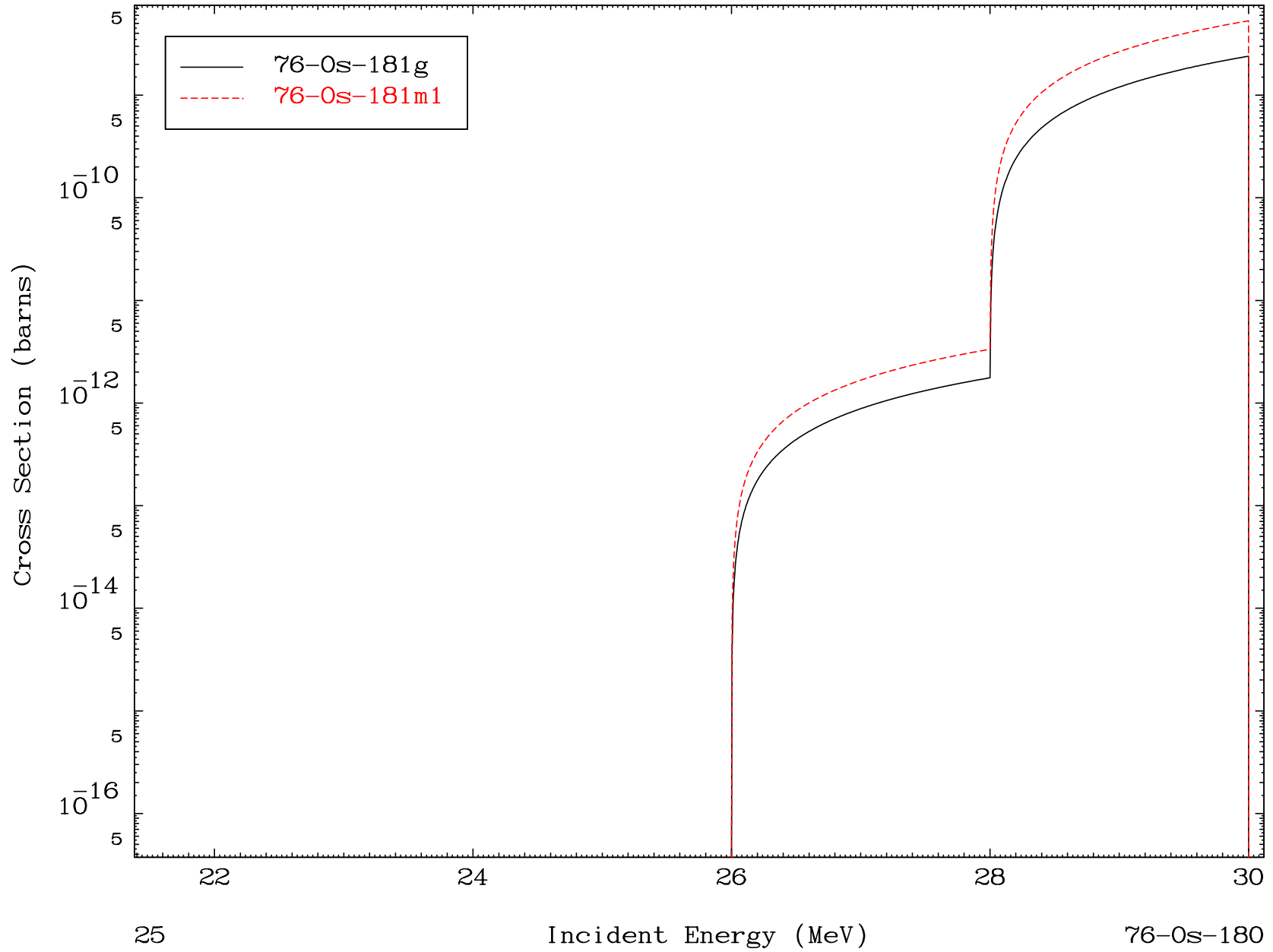


MAT 7613

( $\alpha, 2n$ ) p

76-0s-180

Radionuclide Production Cross Section



Radionuclide Production Cross Section

