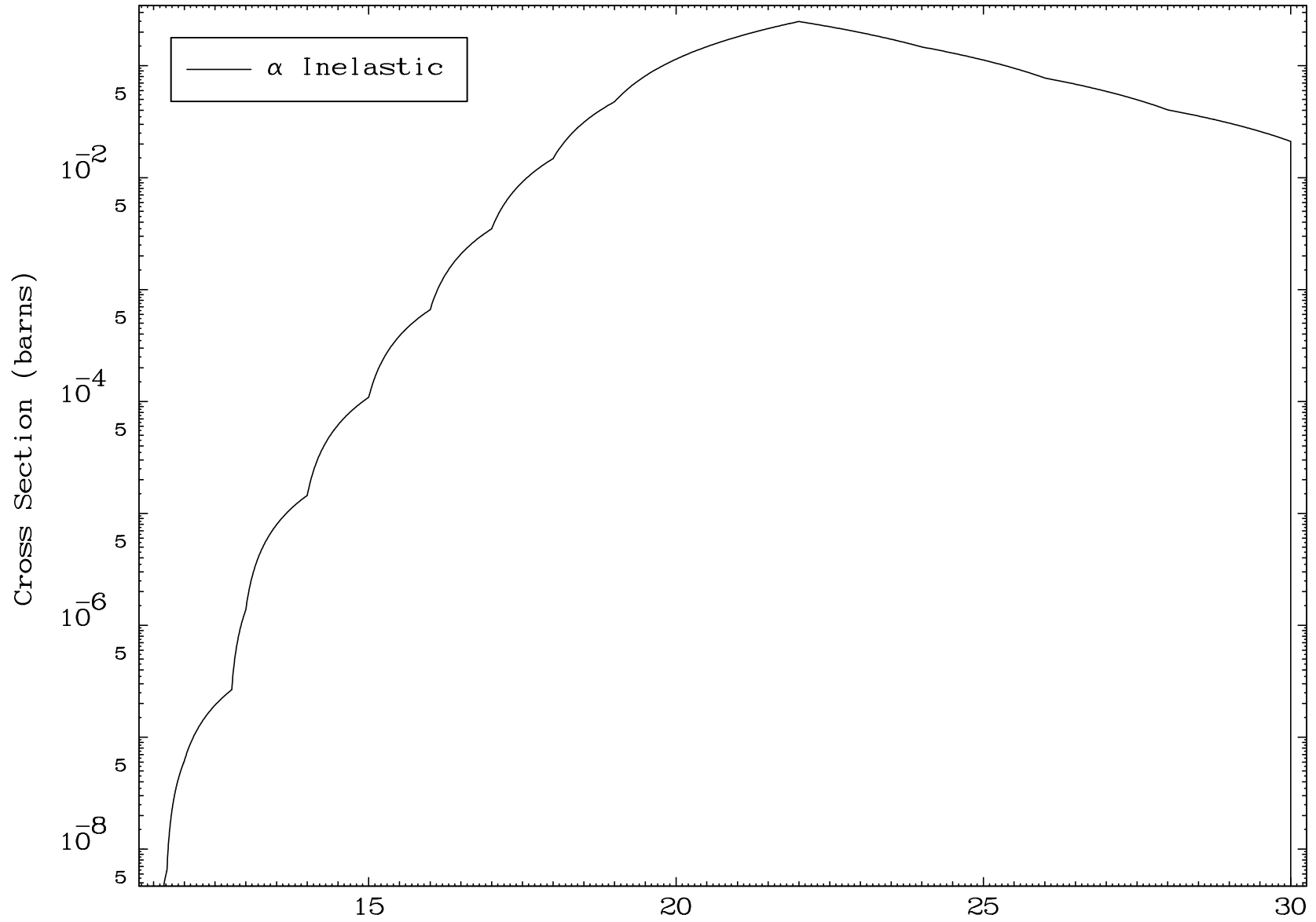


MAT 7623

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

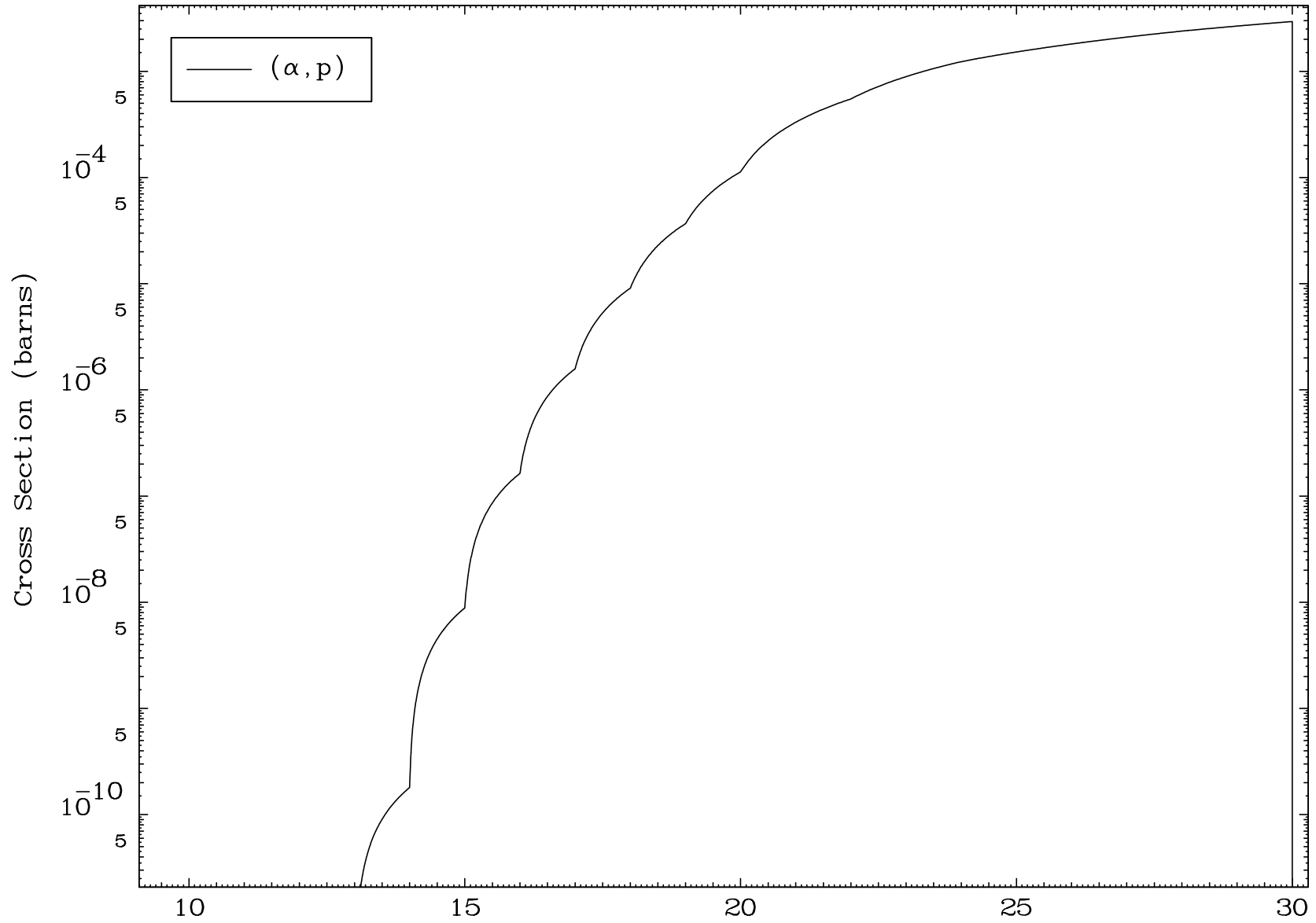
76-0s-183

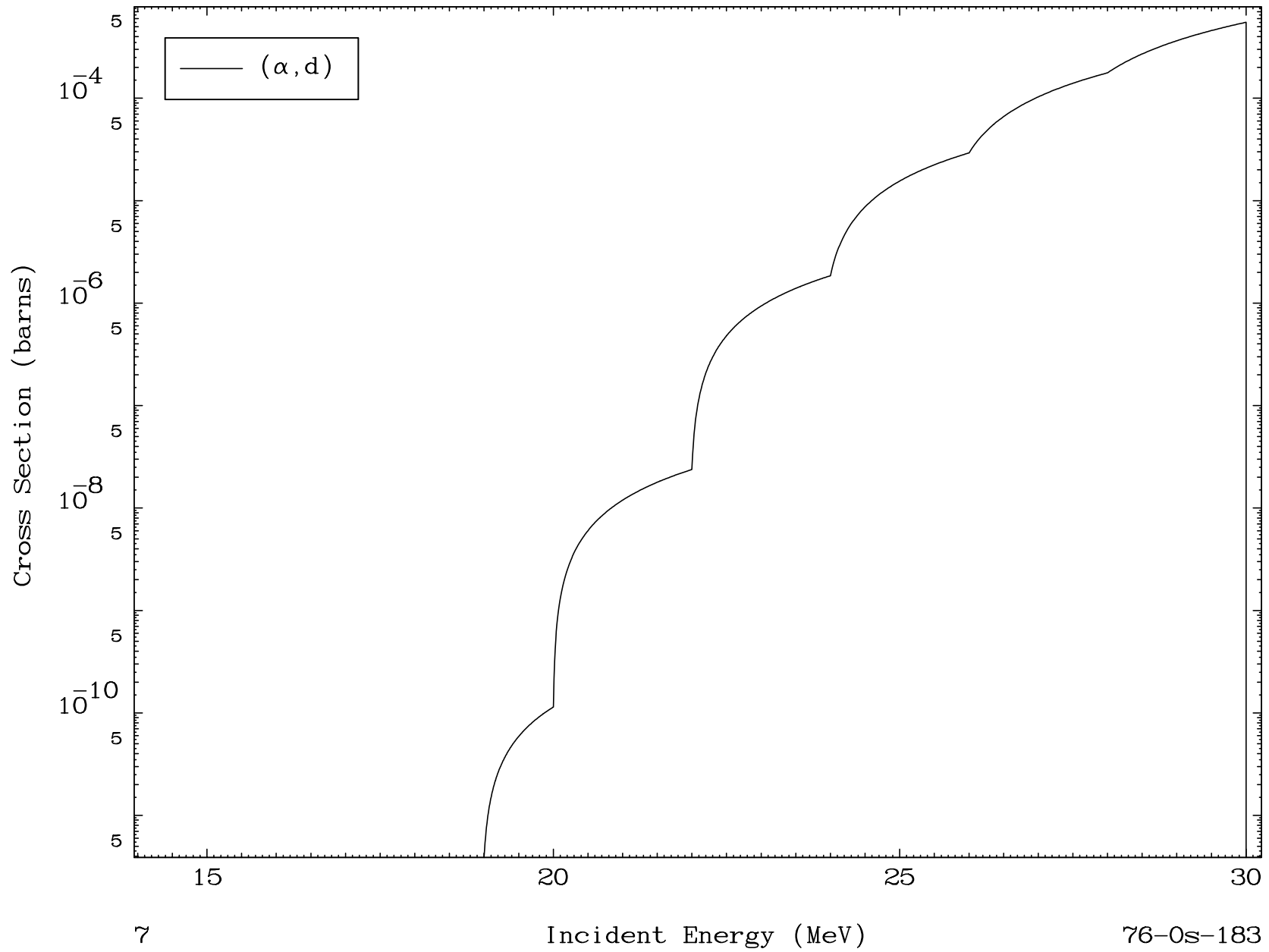


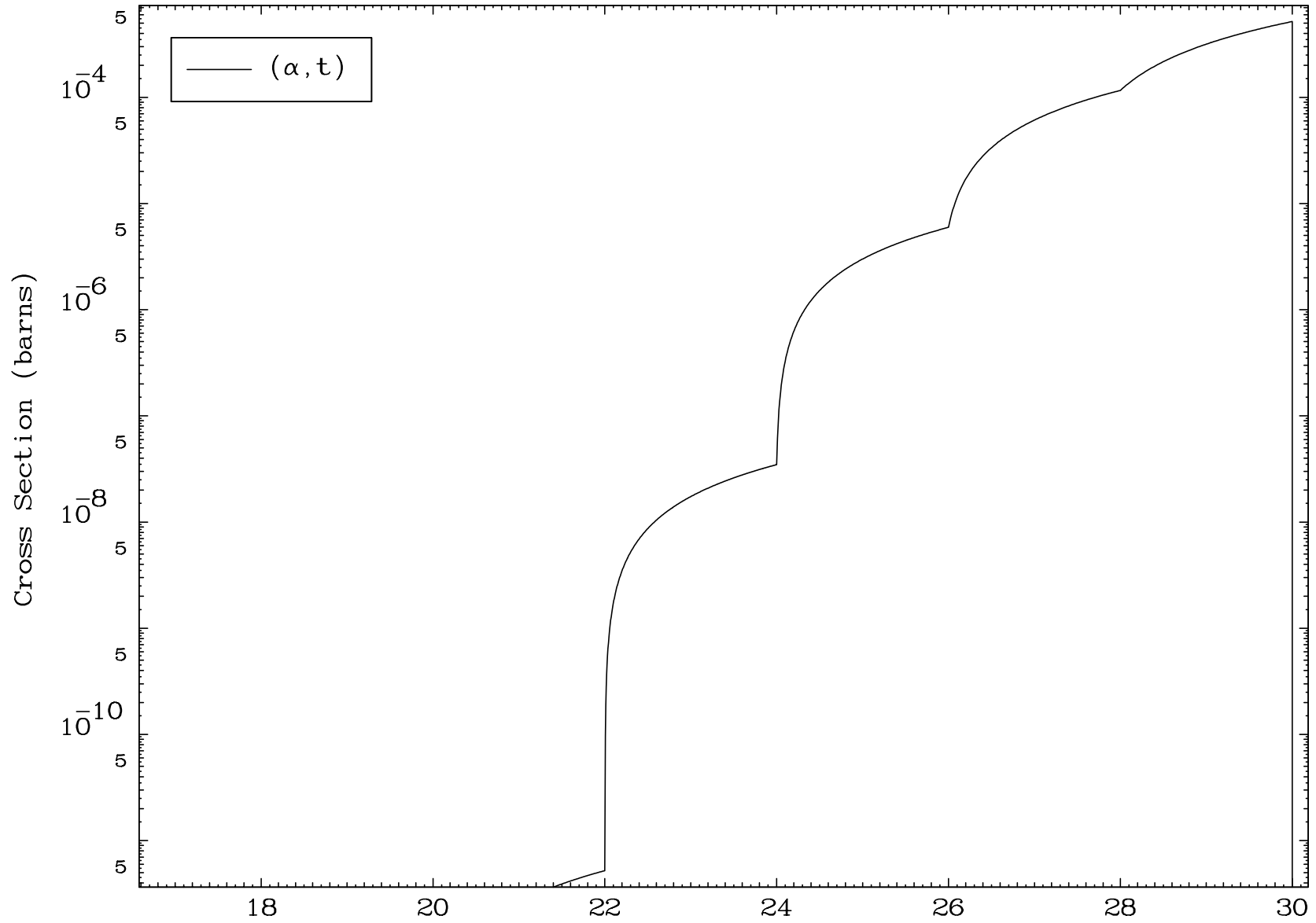
5

Incident Energy (MeV)

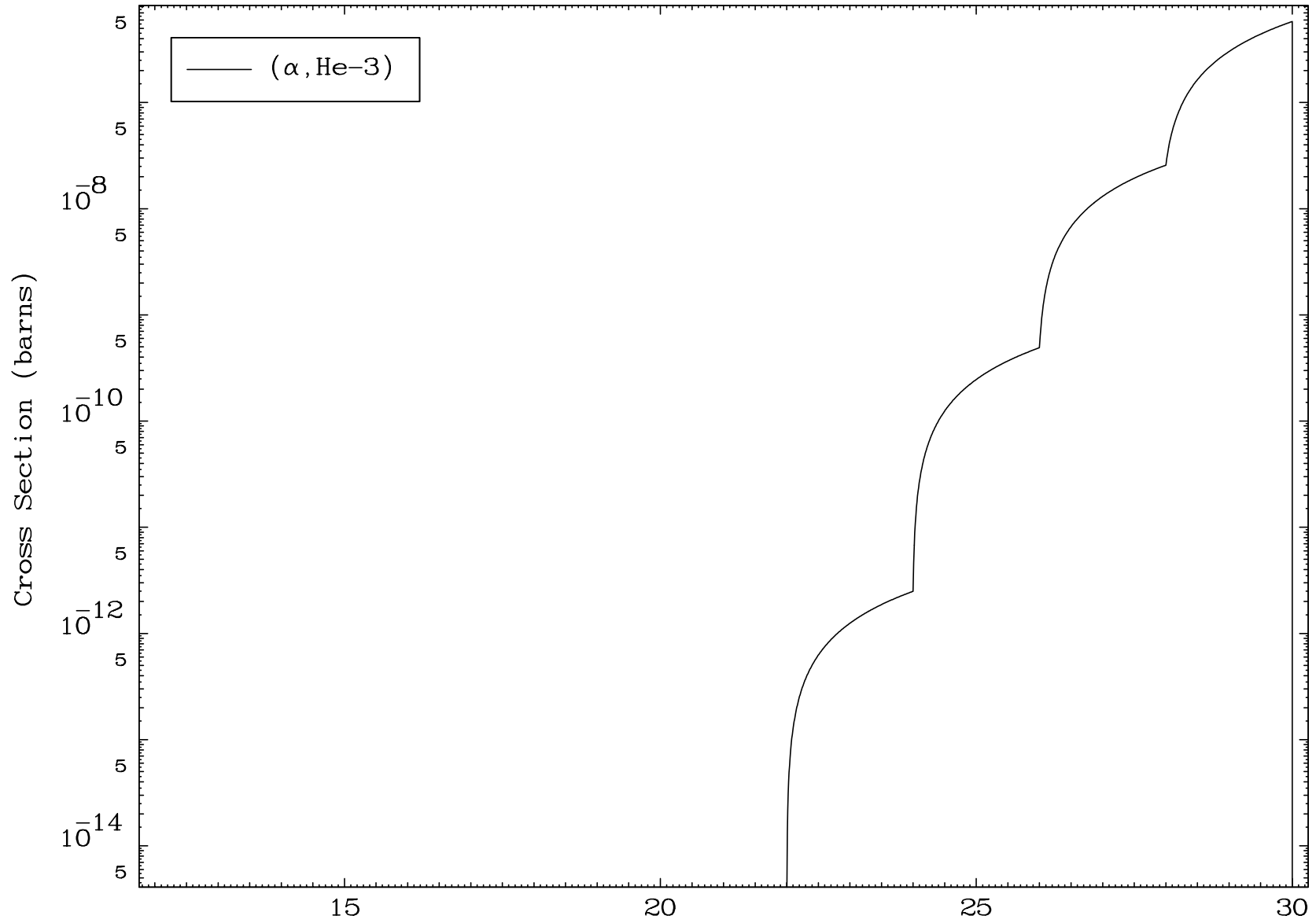
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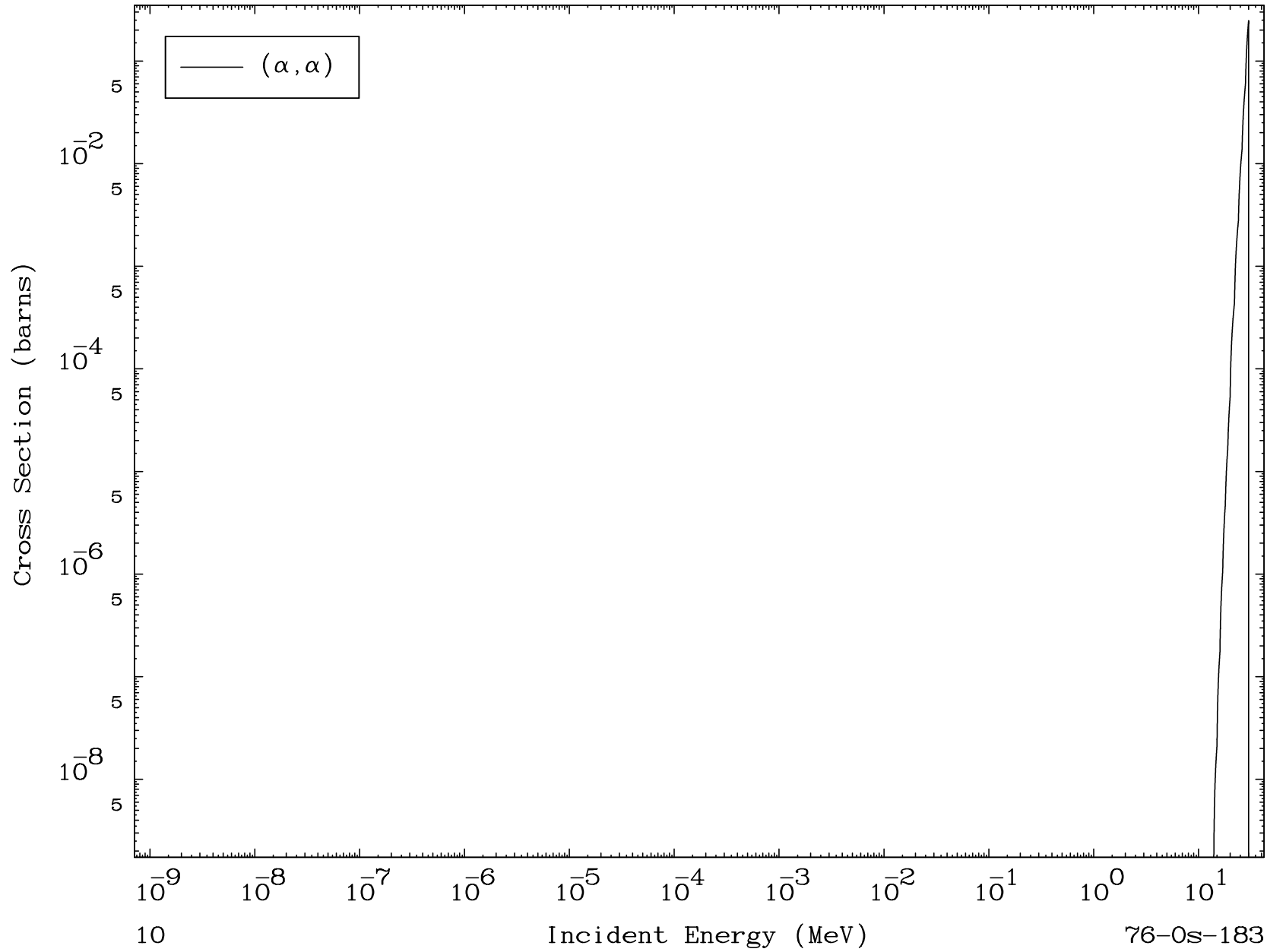




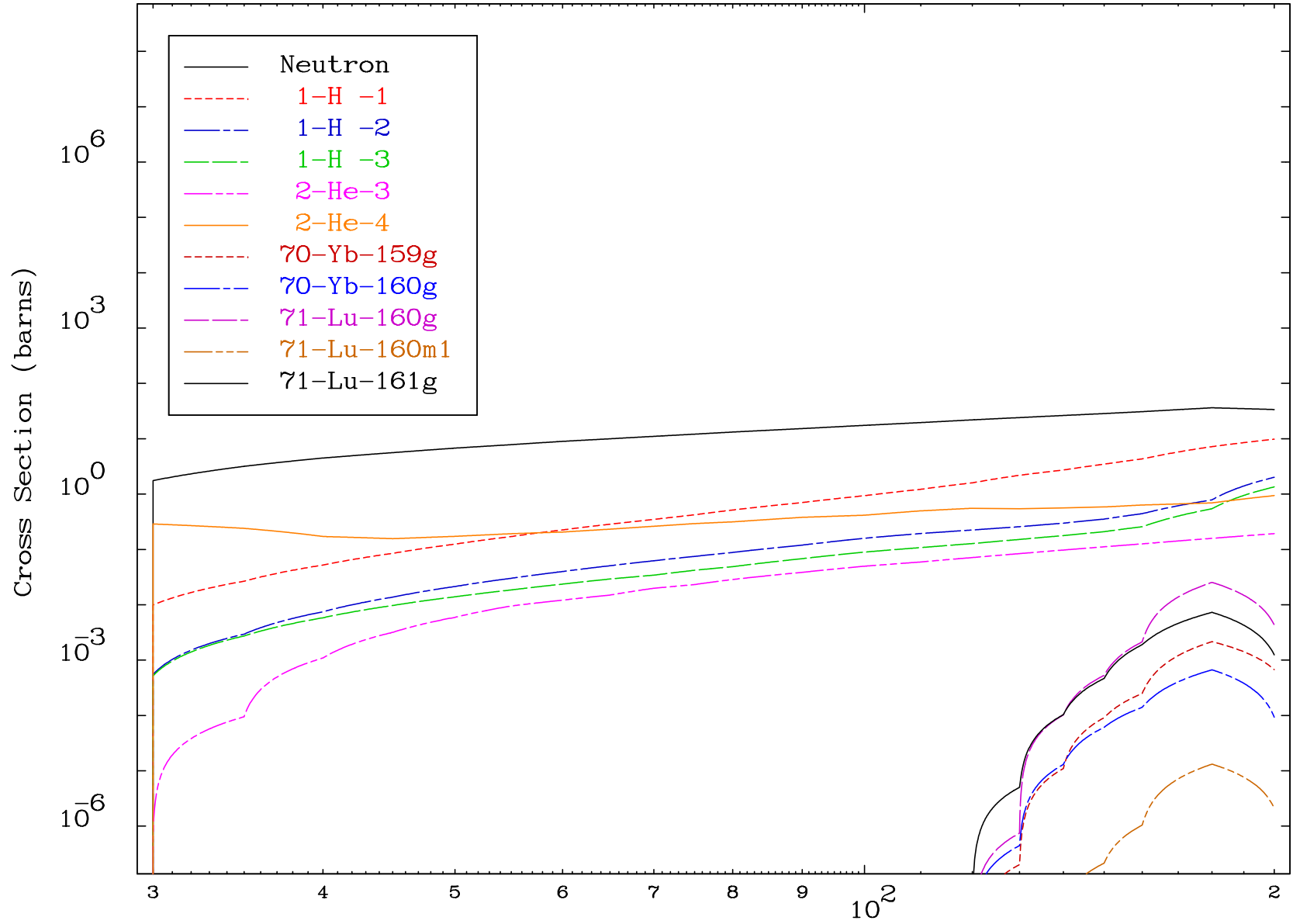




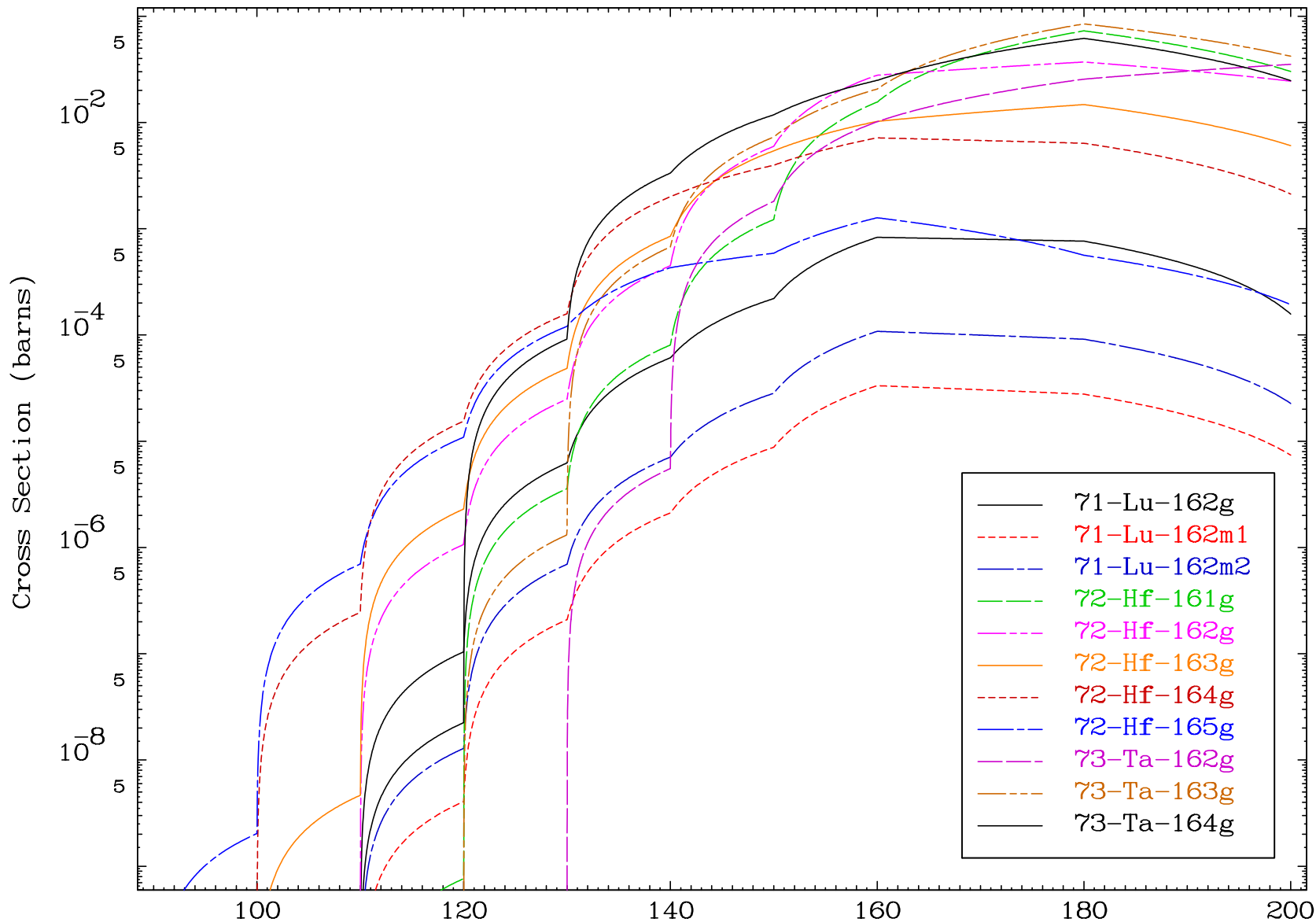




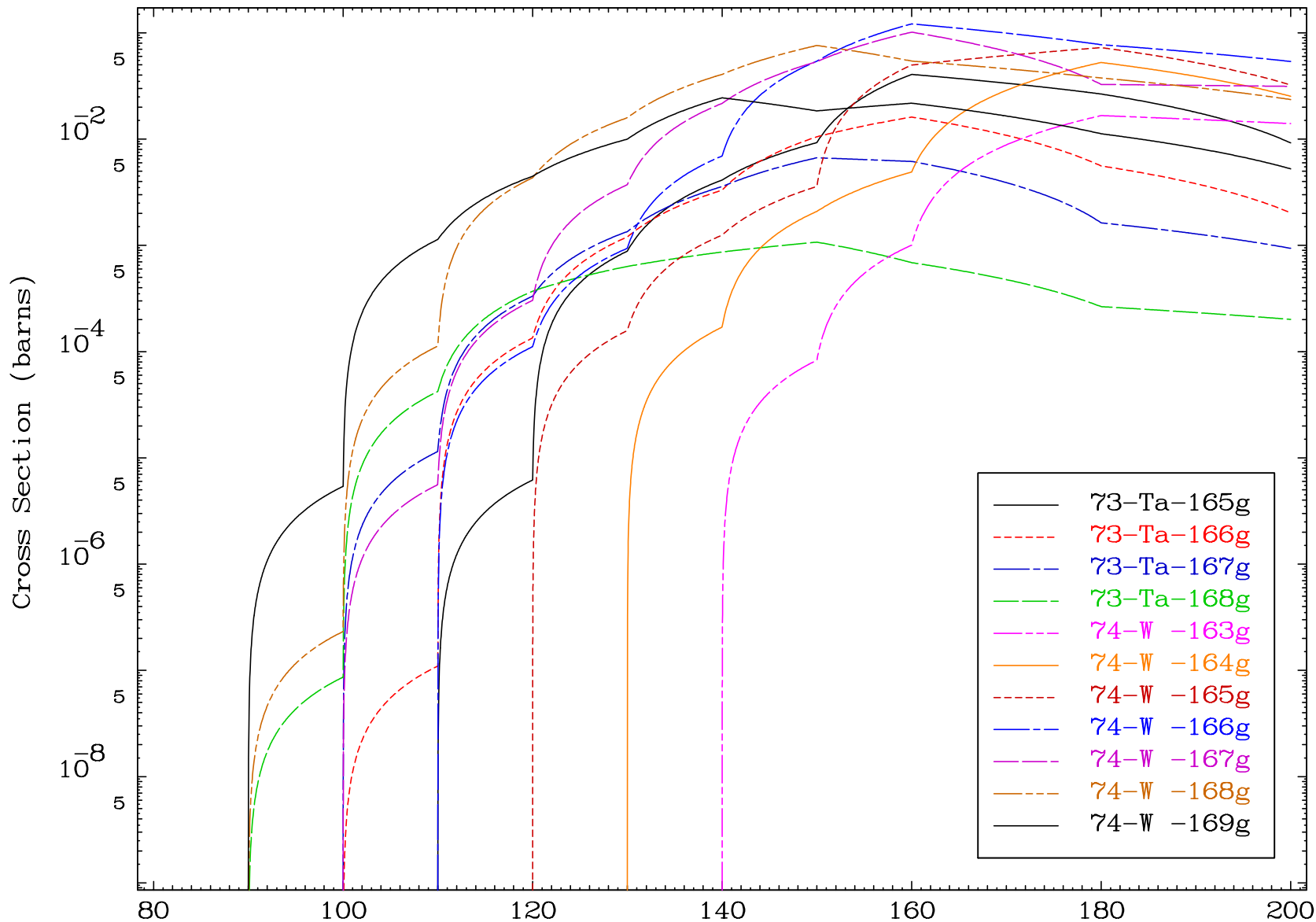
Radionuclide Production Cross Section



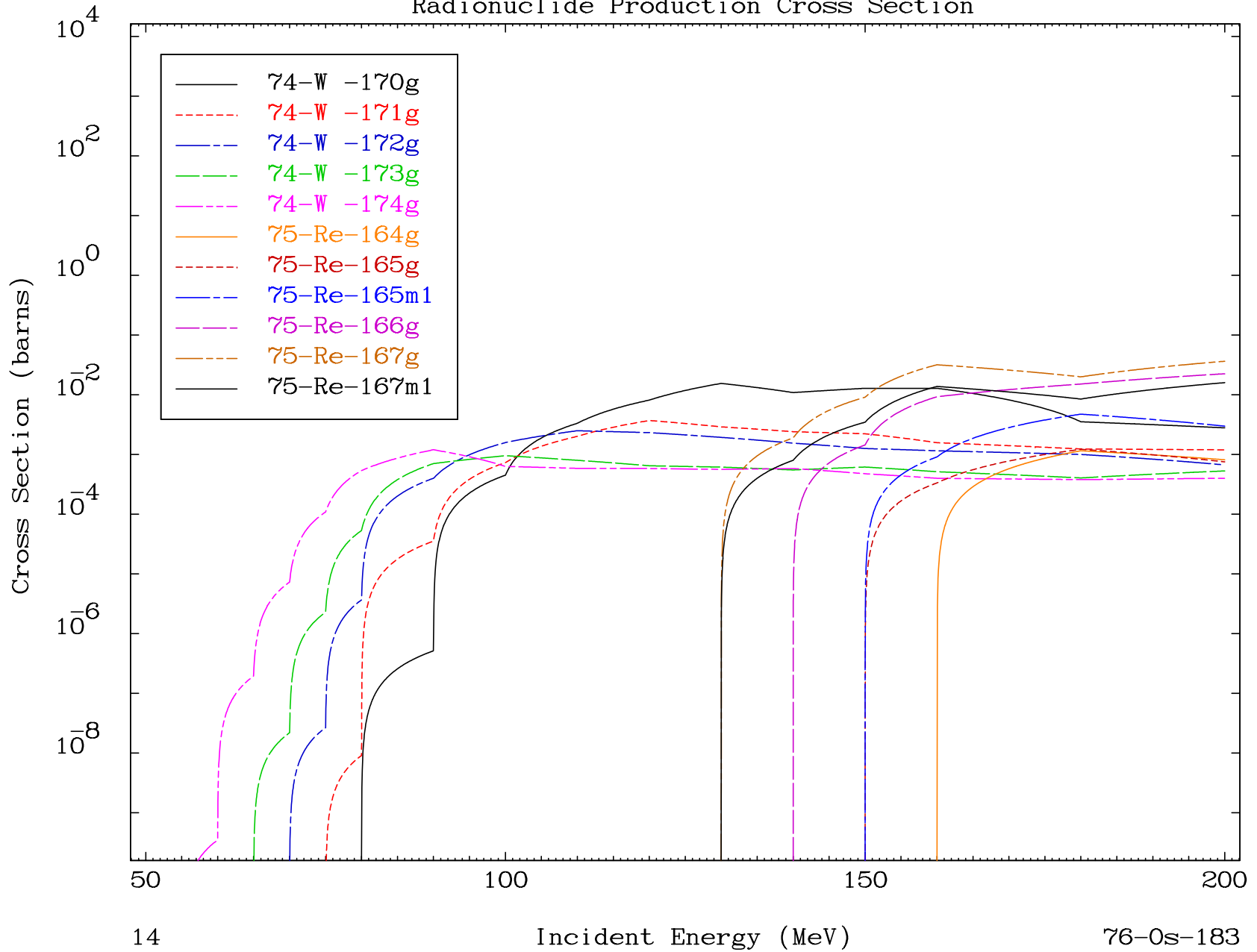
Radionuclide Production Cross Section



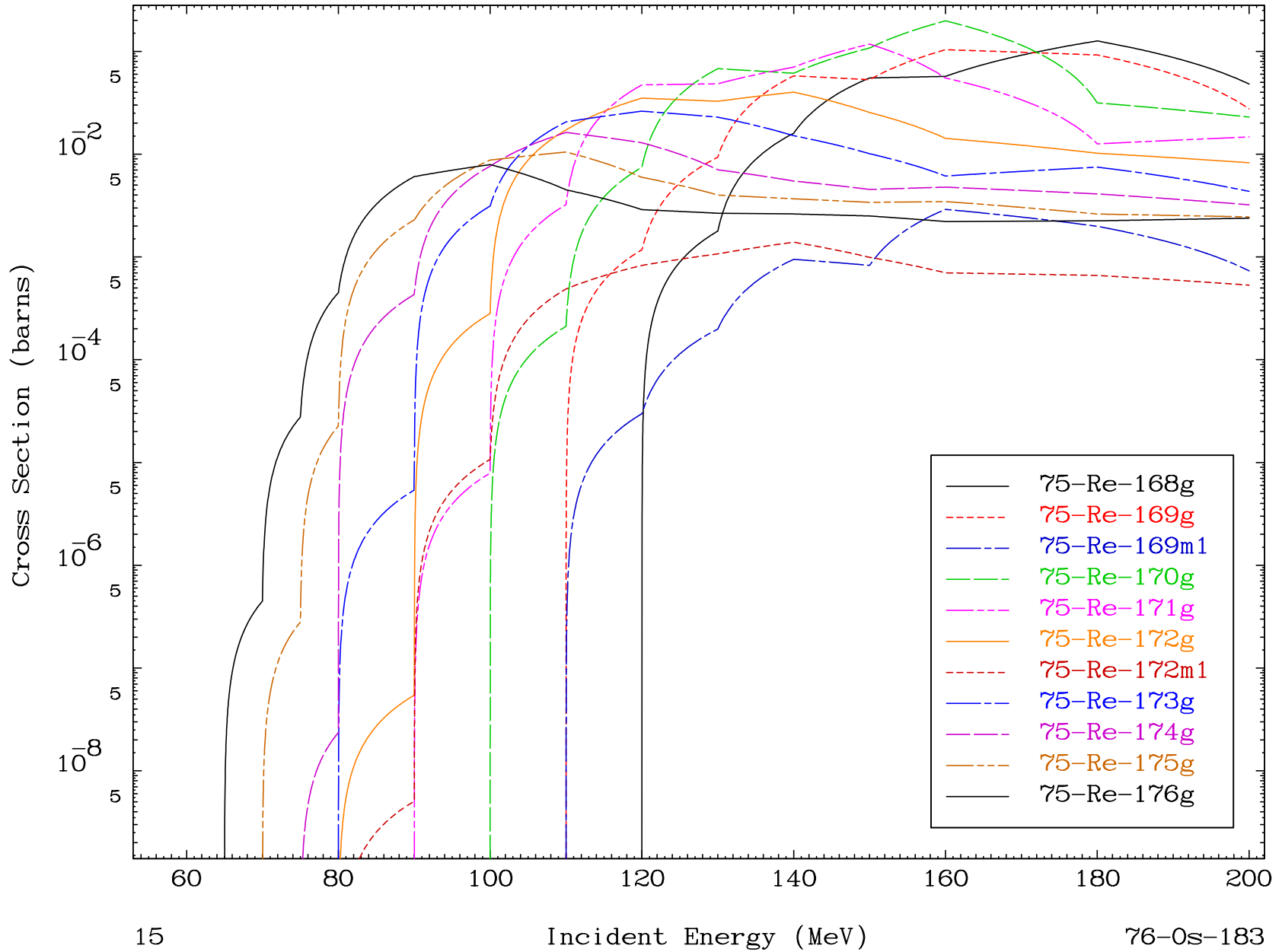
Radionuclide Production Cross Section



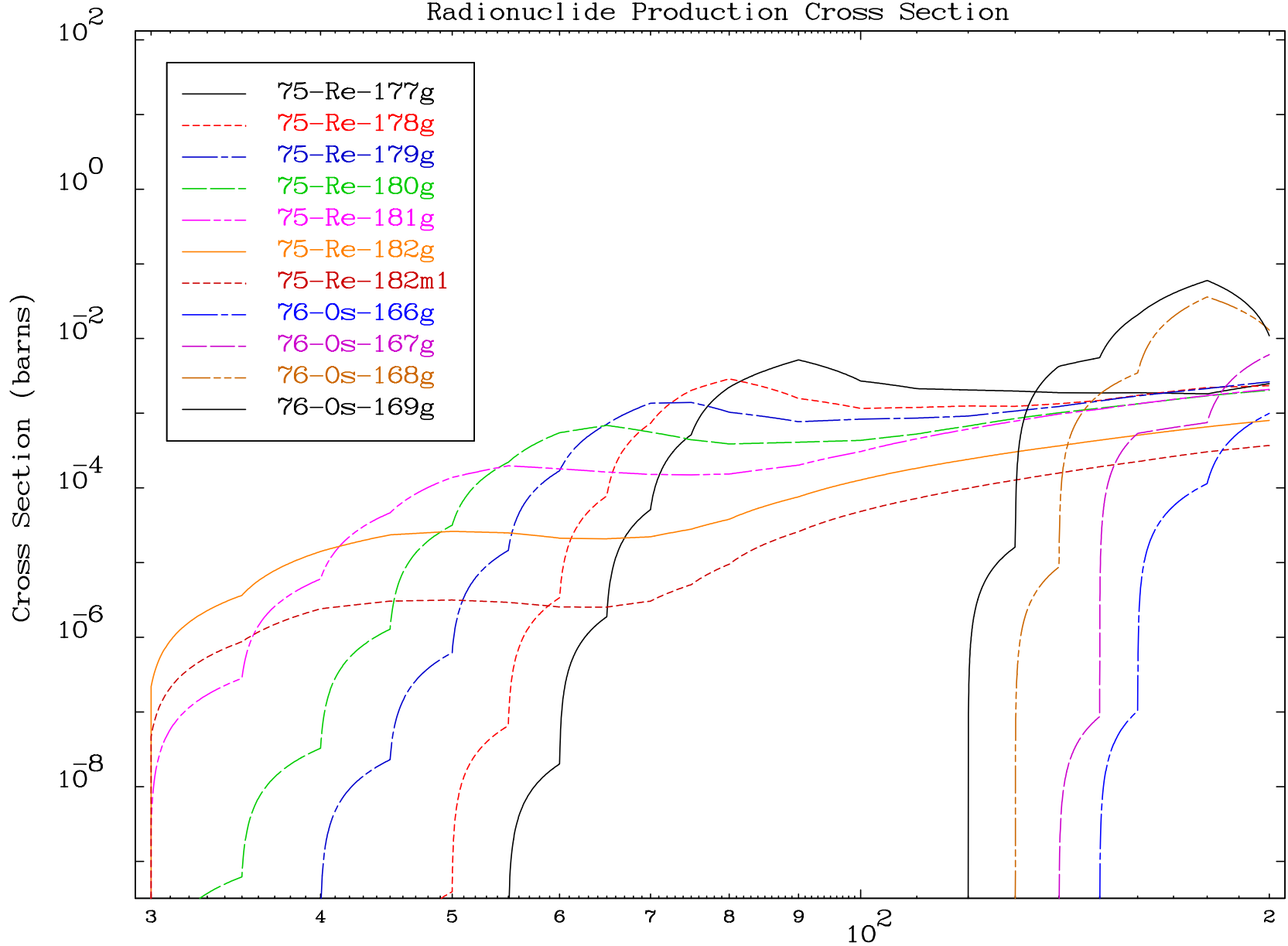
Radionuclide Production Cross Section



Radionuclide Production Cross Section

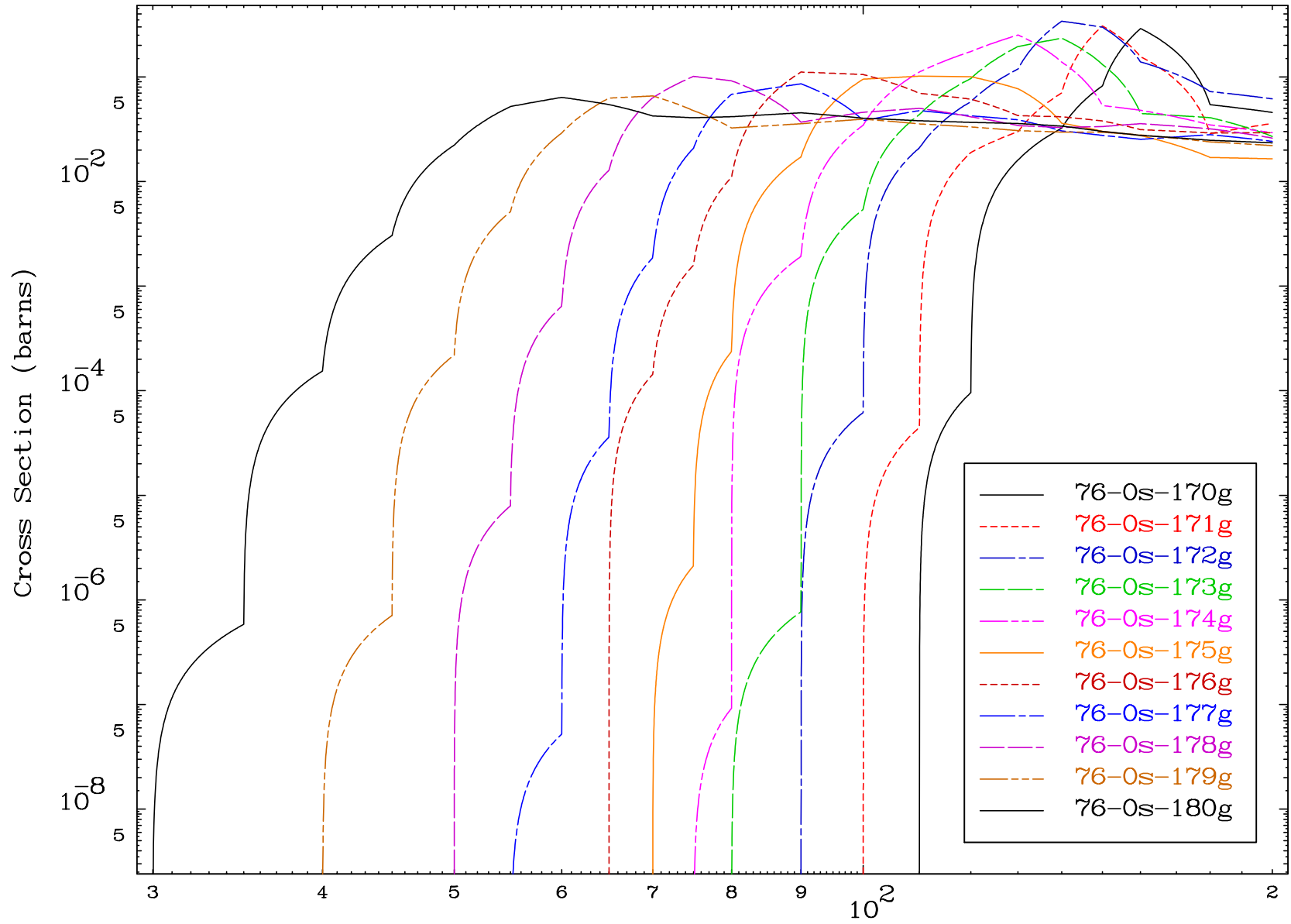


Radionuclide Production Cross Section

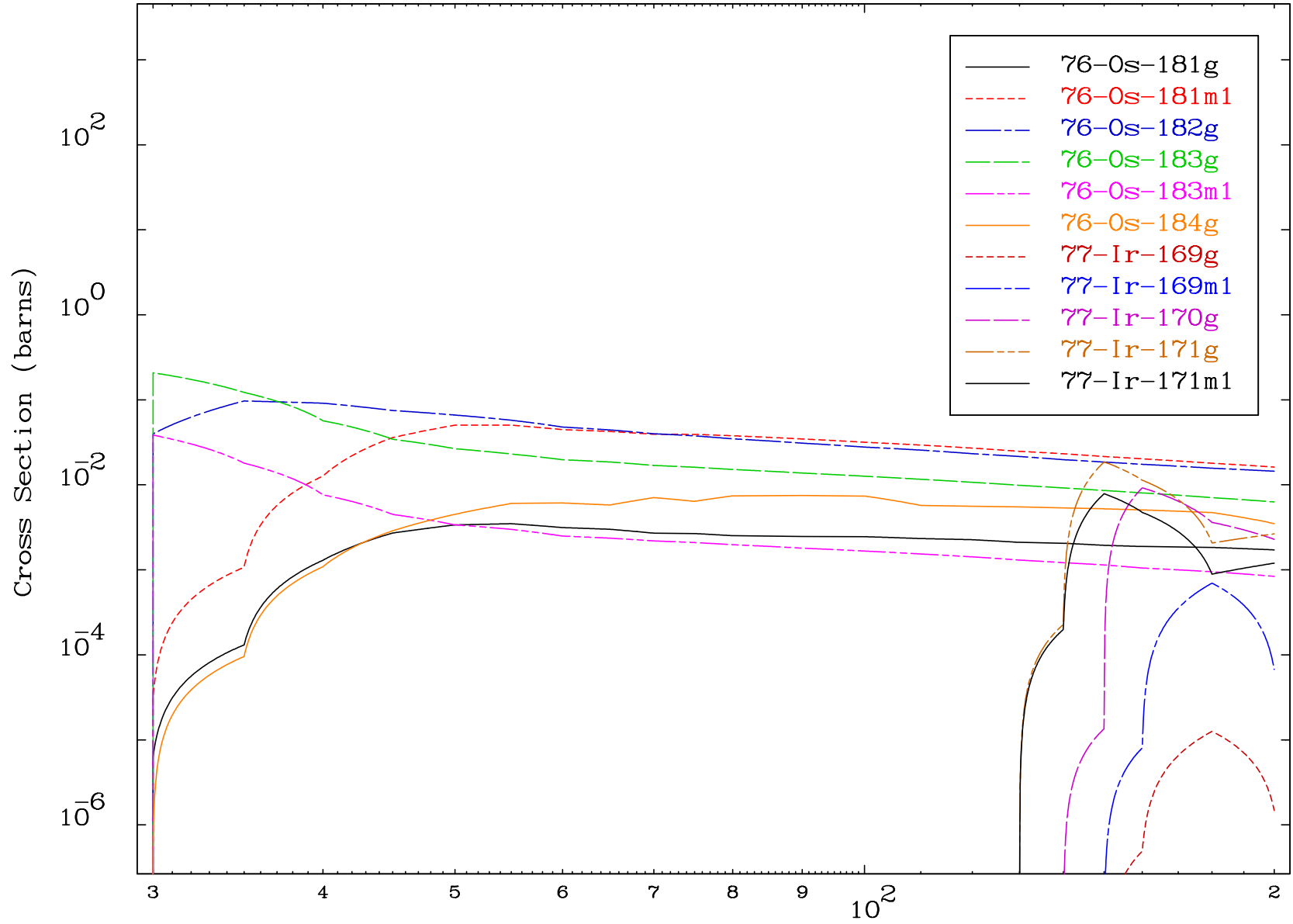




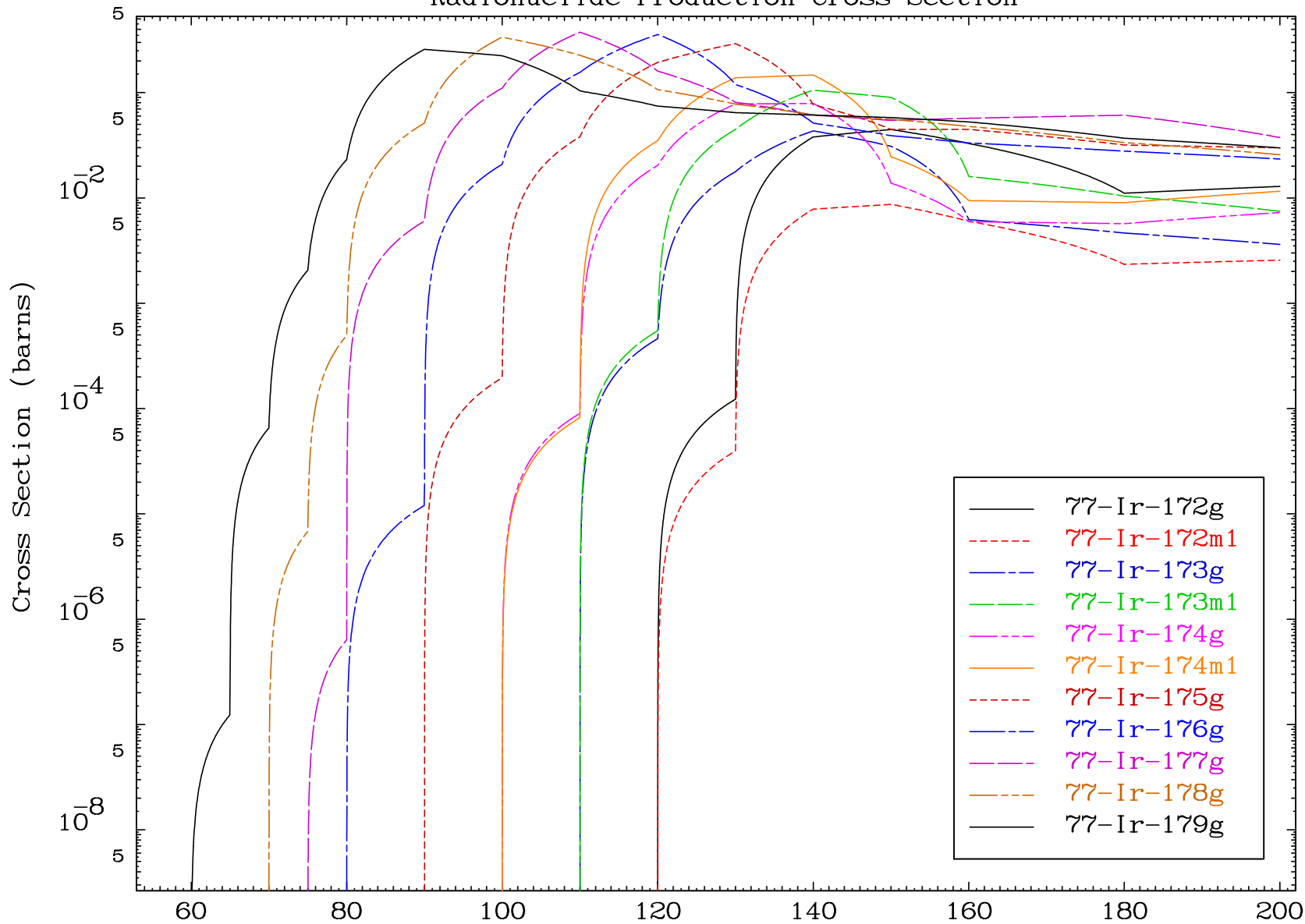
Radionuclide Production Cross Section



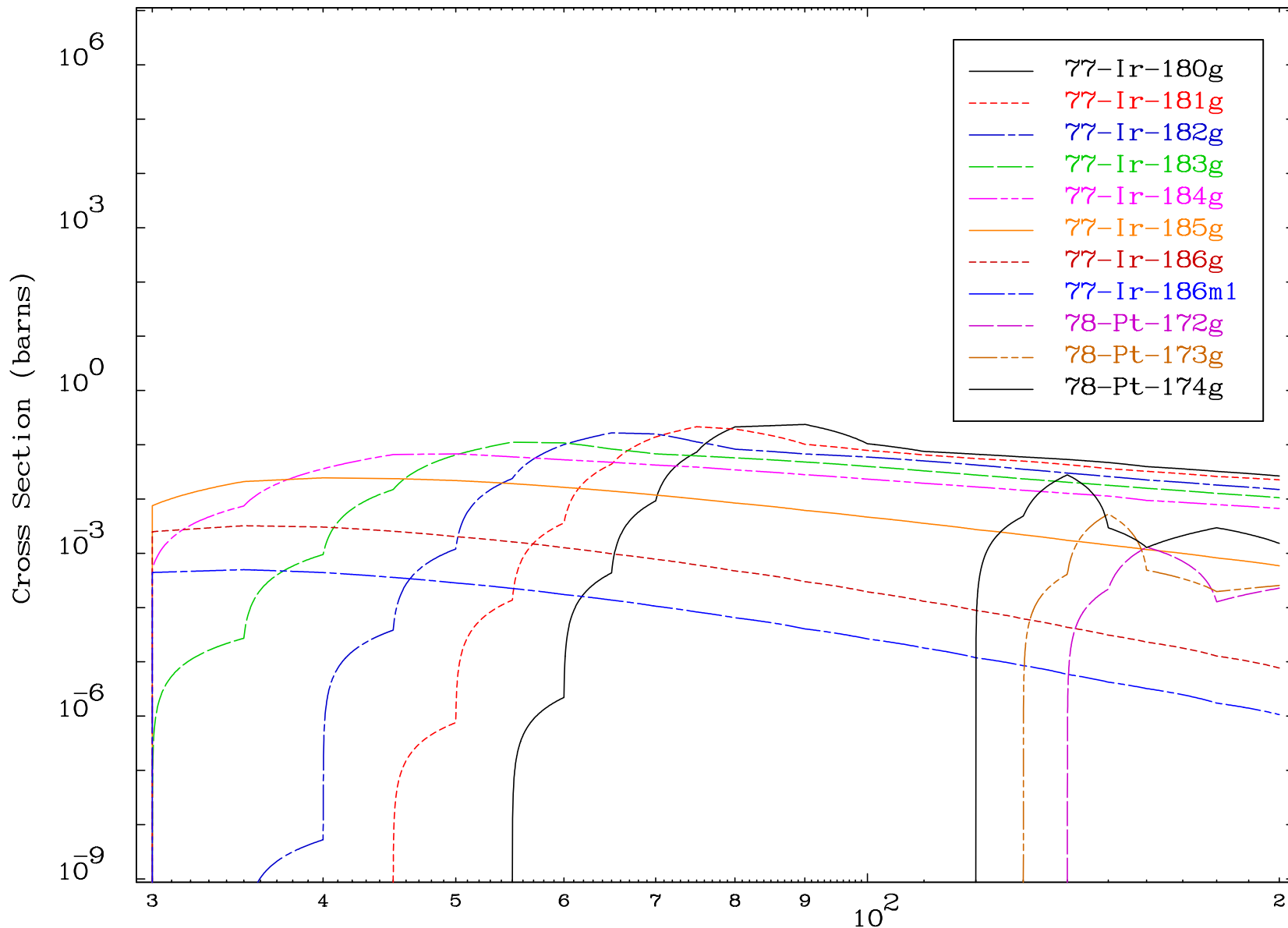
Radionuclide Production Cross Section



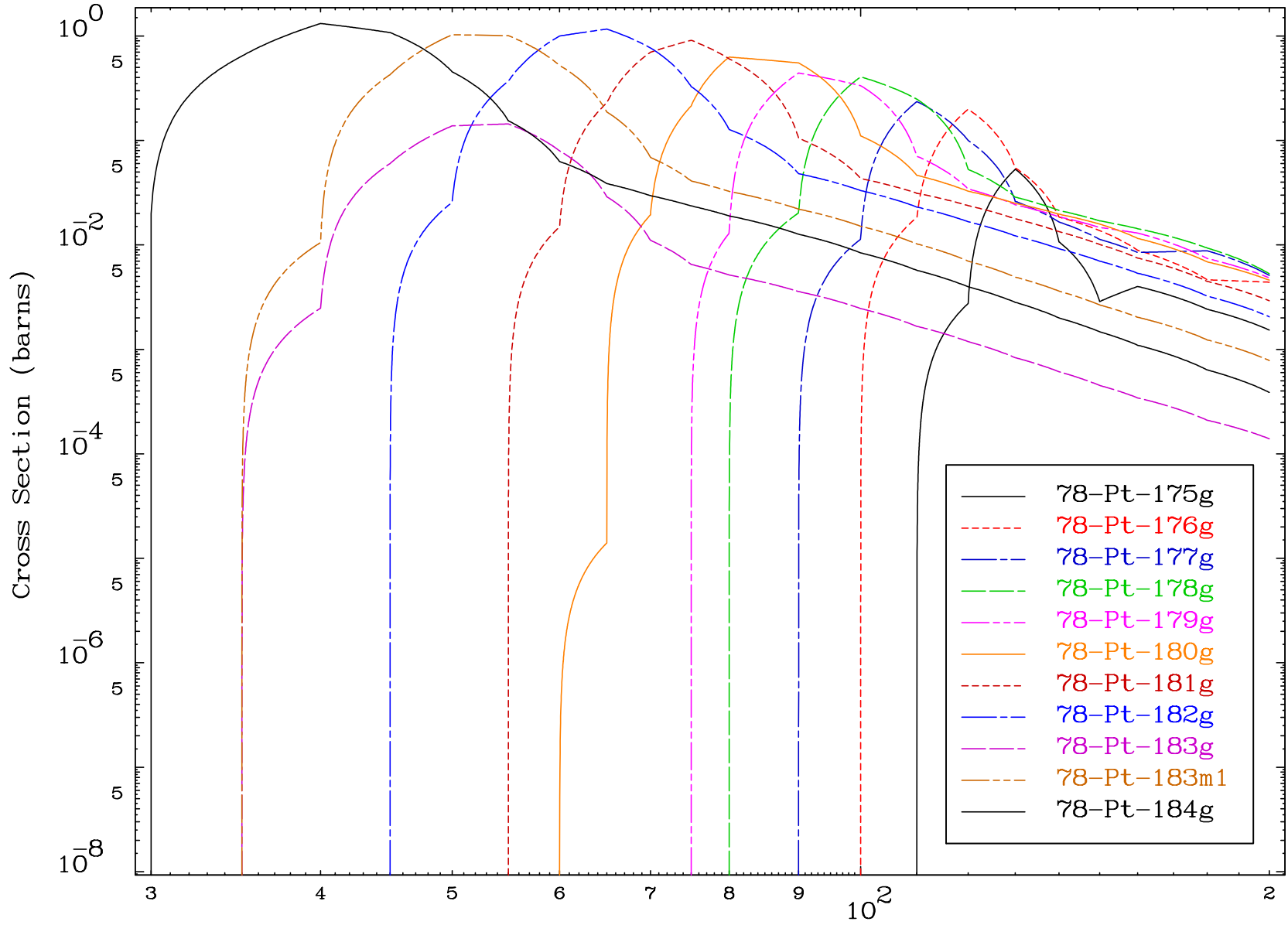
Radionuclide Production Cross Section



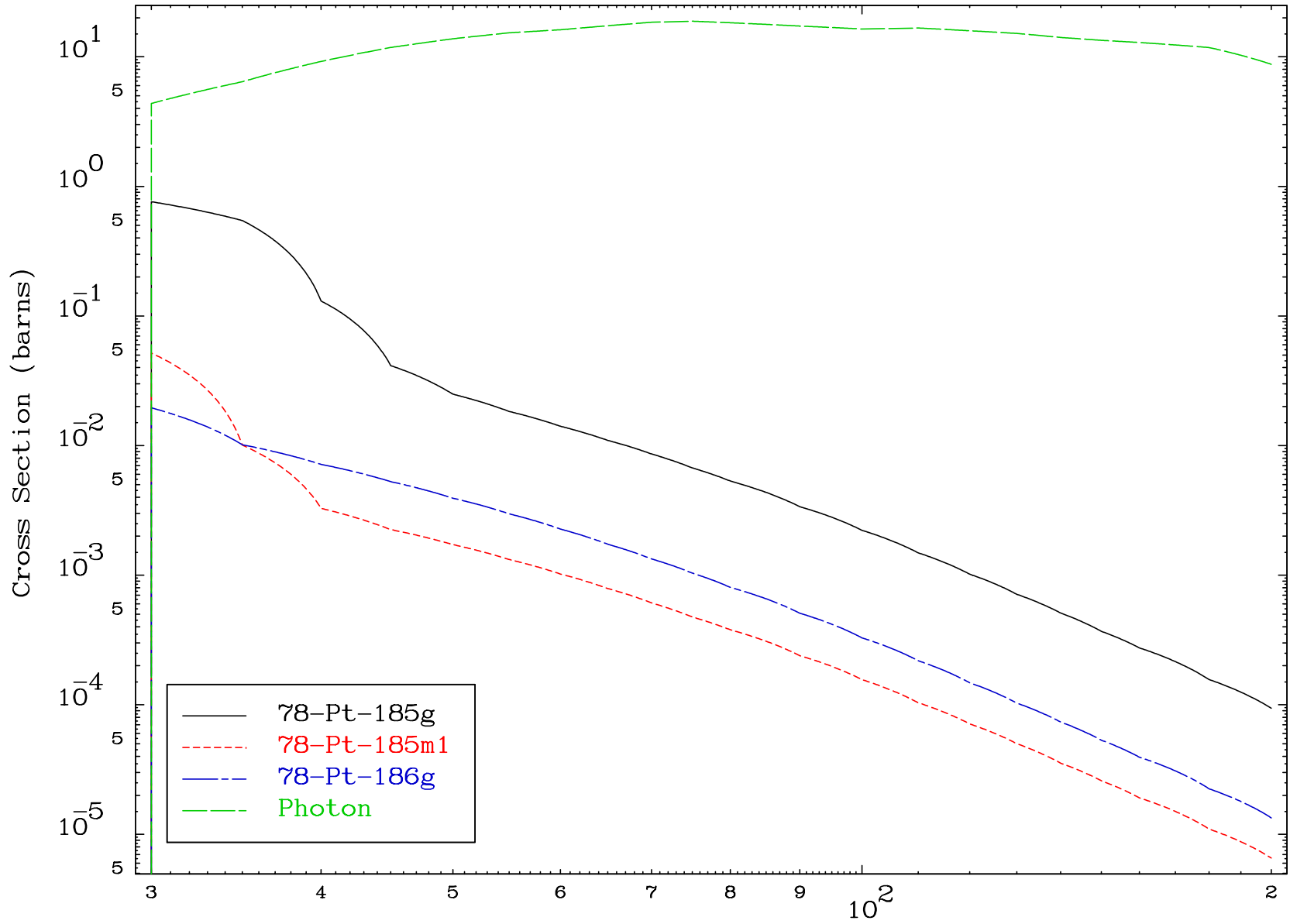
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

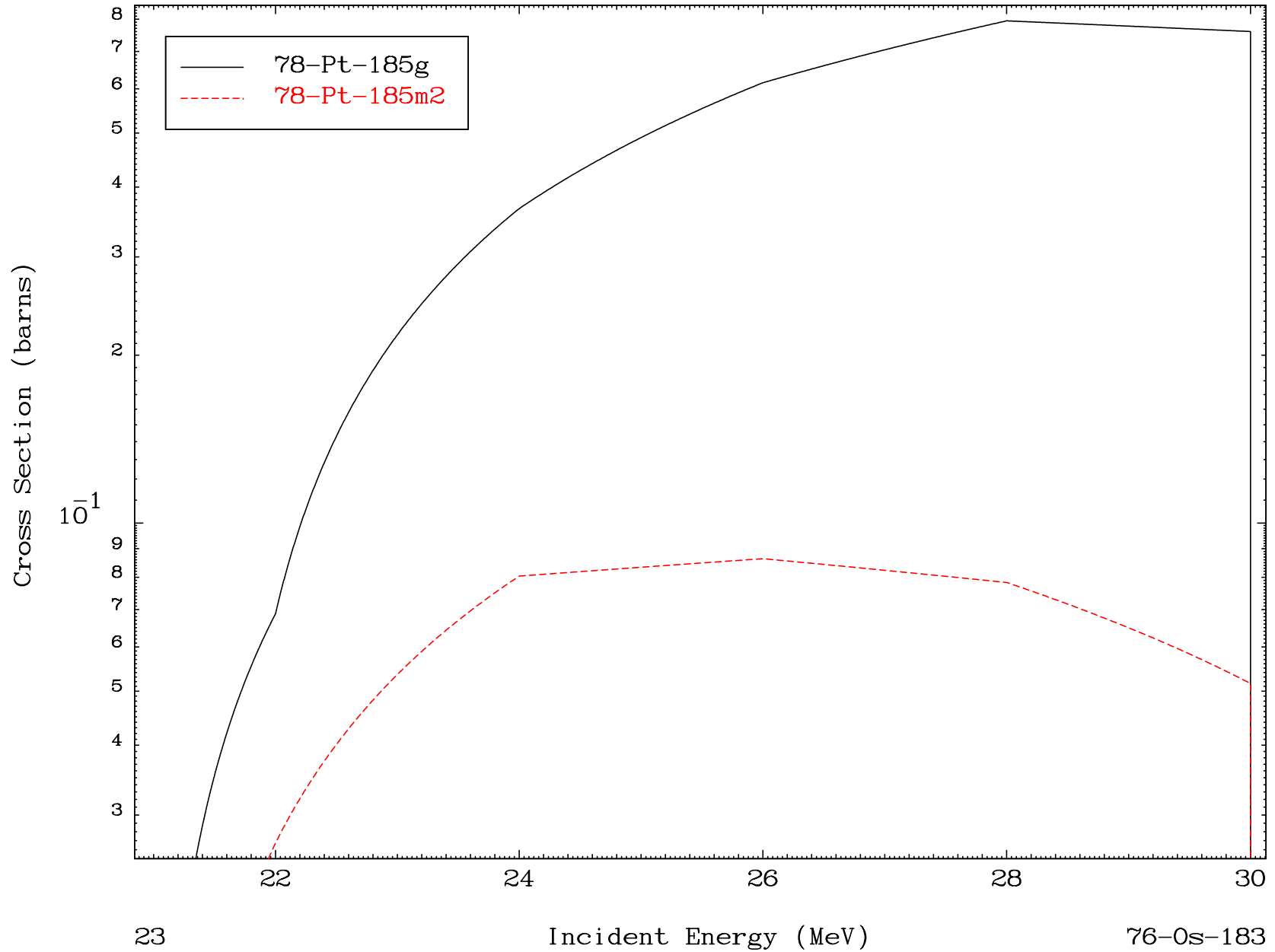


MAT 7623

( $\alpha, 2n$ )

76-0s-183

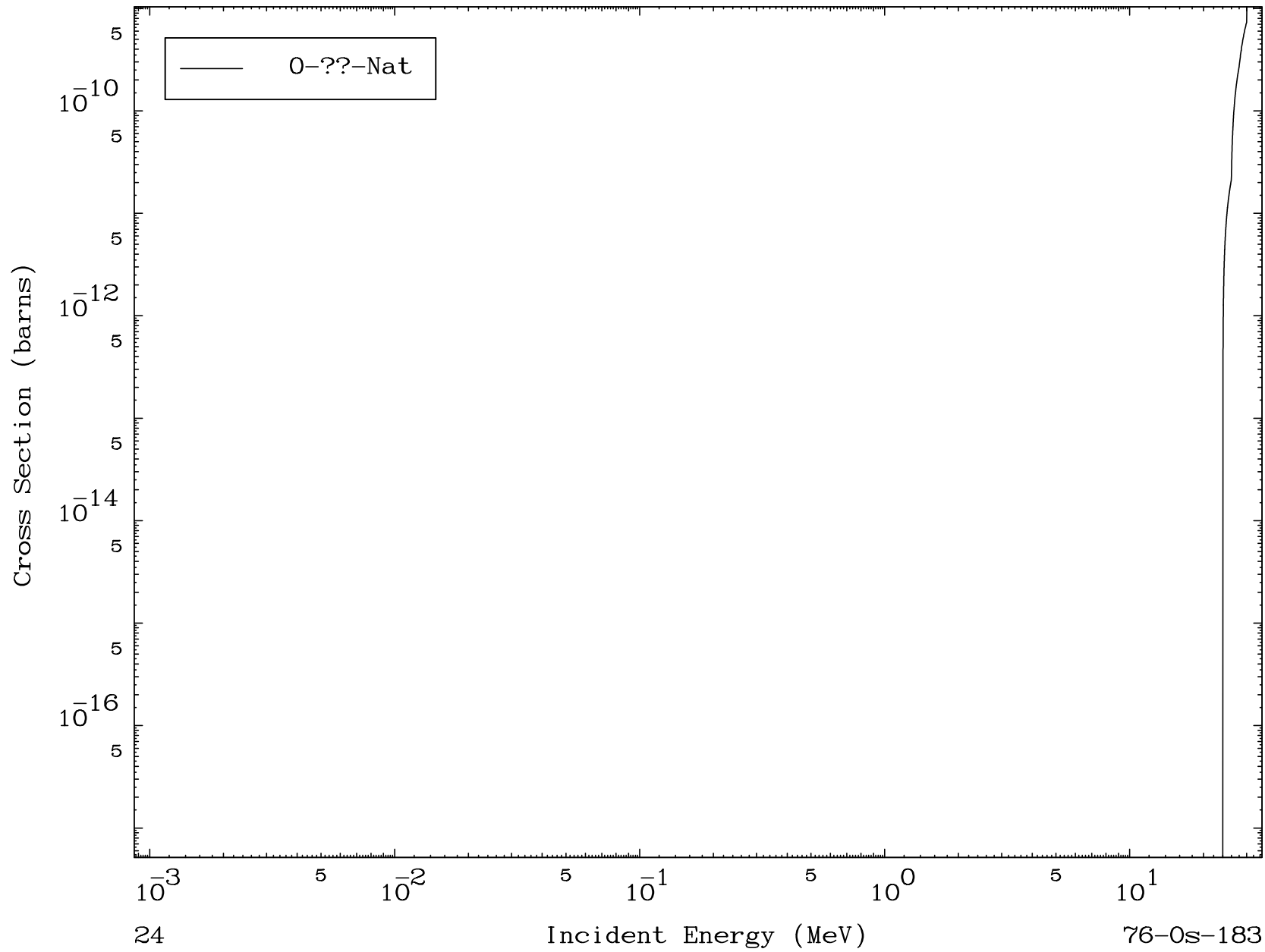
Radionuclide Production Cross Section



23

Incident Energy (MeV)

76-0s-183



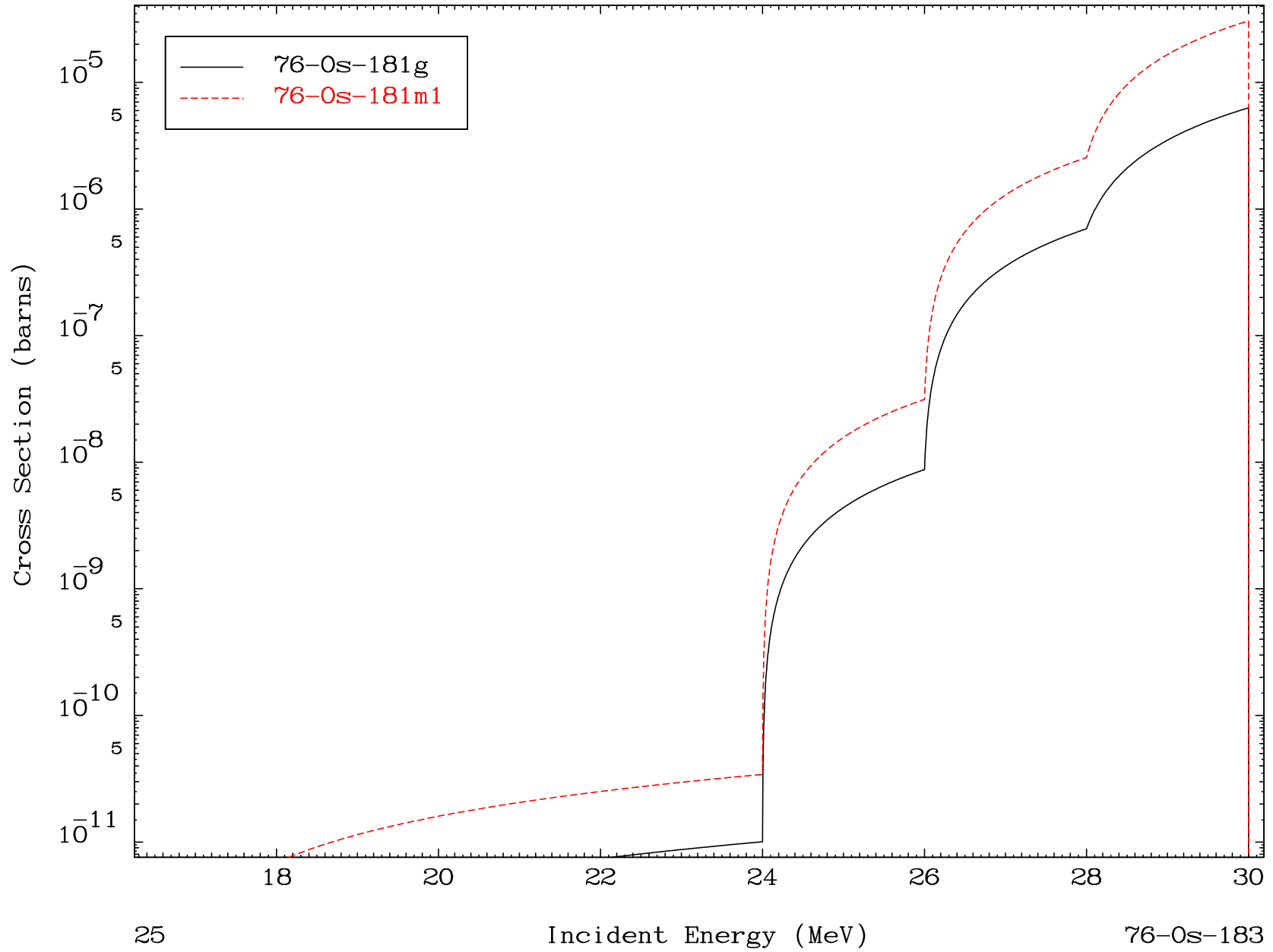


MAT 7623

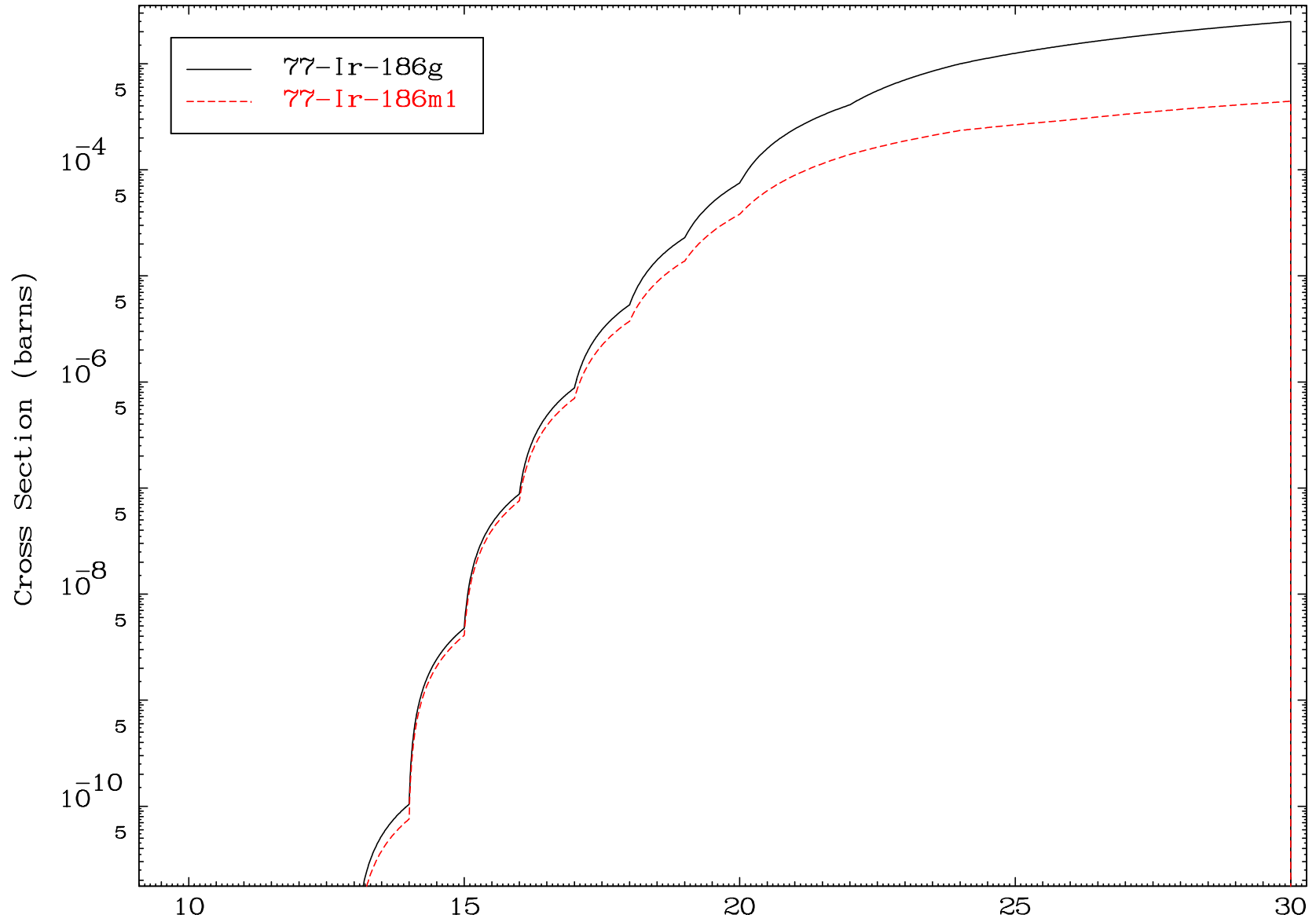
( $\alpha, 2n$ )  $\alpha$

76-0s-183

Radionuclide Production Cross Section



Radionuclide Production Cross Section

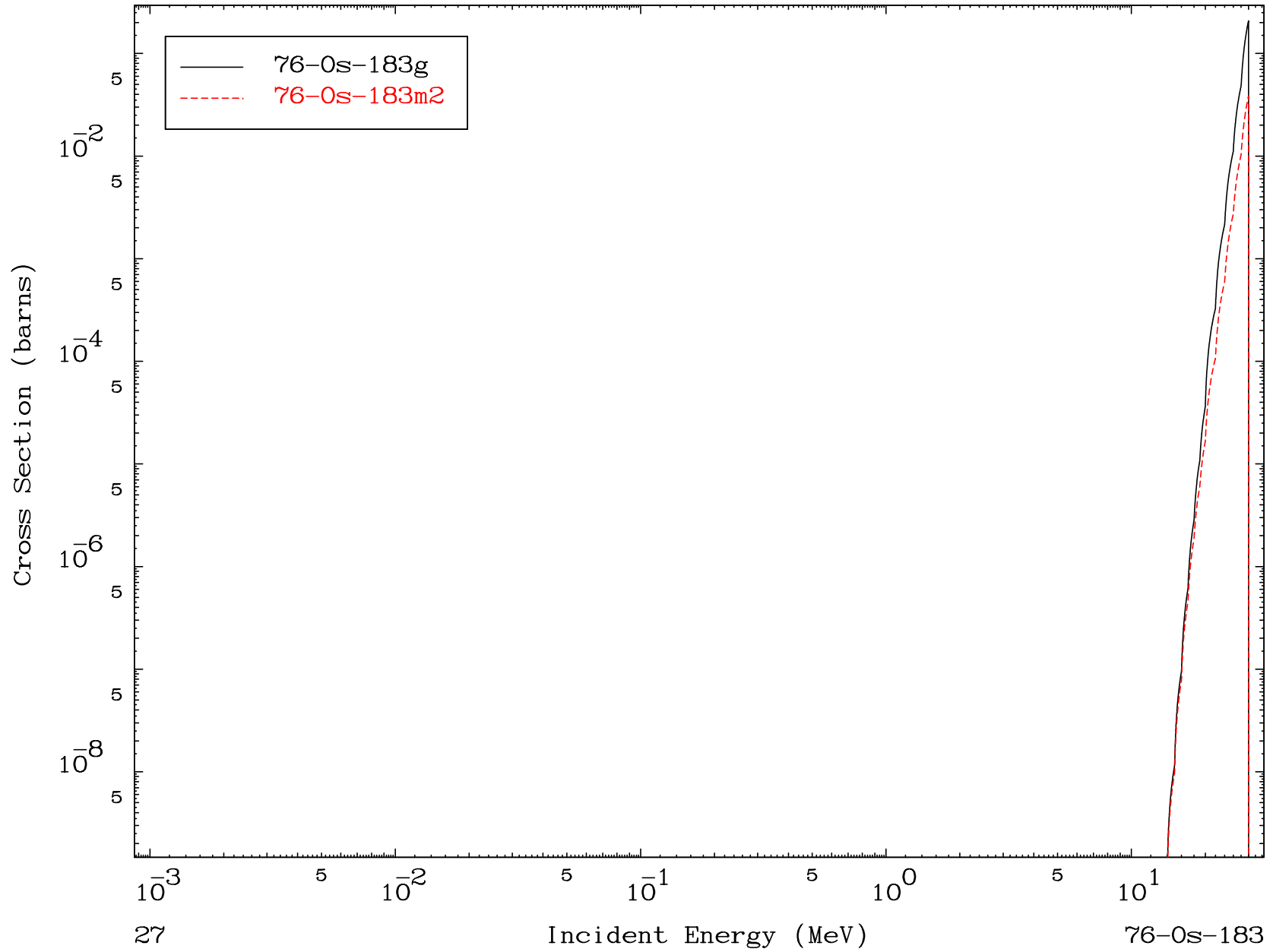


MAT 7623

( $\alpha, \alpha$ )

76-0s-183

Radionuclide Production Cross Section



27

Incident Energy (MeV)

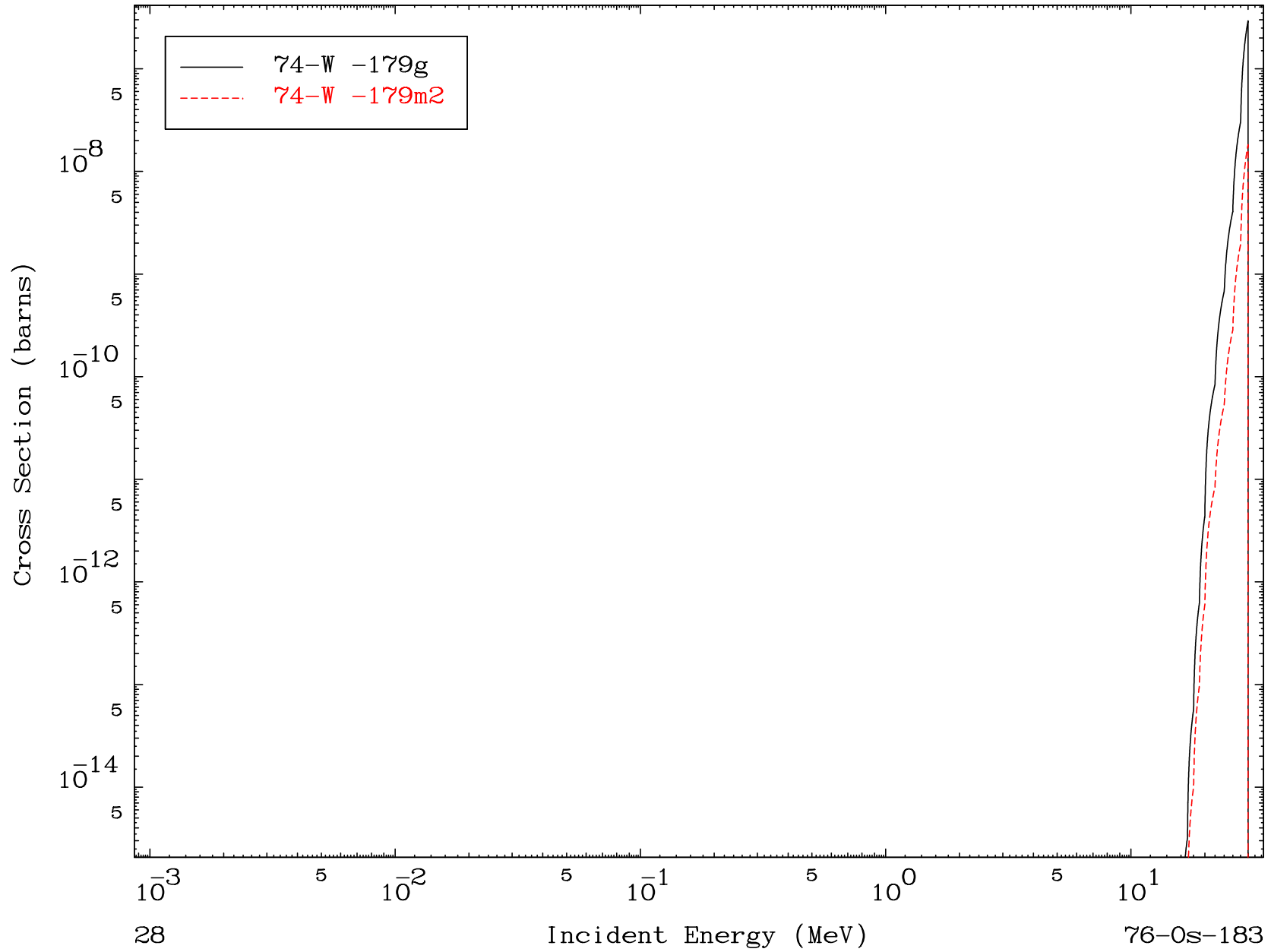
76-0s-183

MAT 7623

( $\alpha, 2\alpha$ )

76-0s-183

Radionuclide Production Cross Section



Radionuclide Production Cross Section

