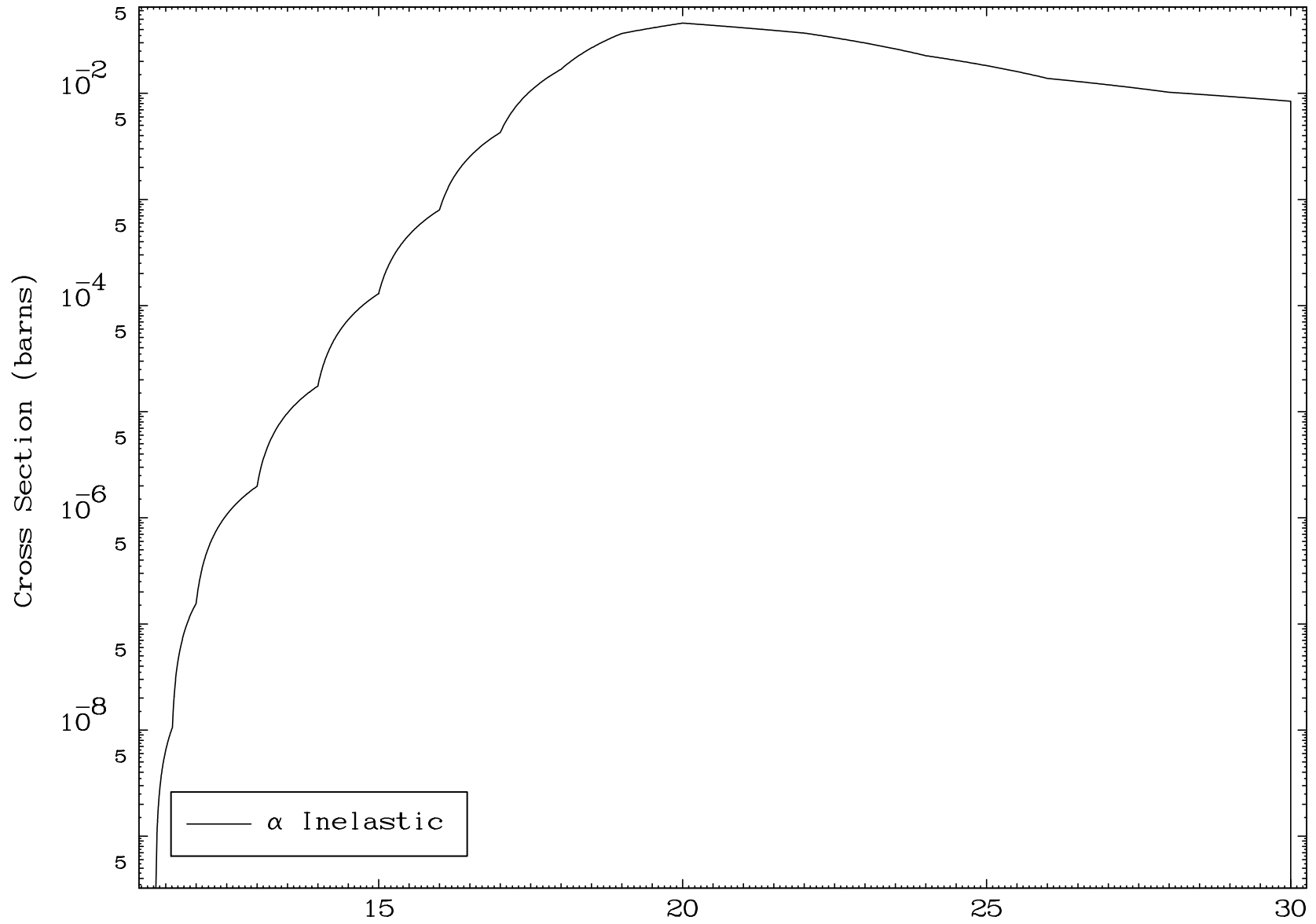


MAT 7637

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

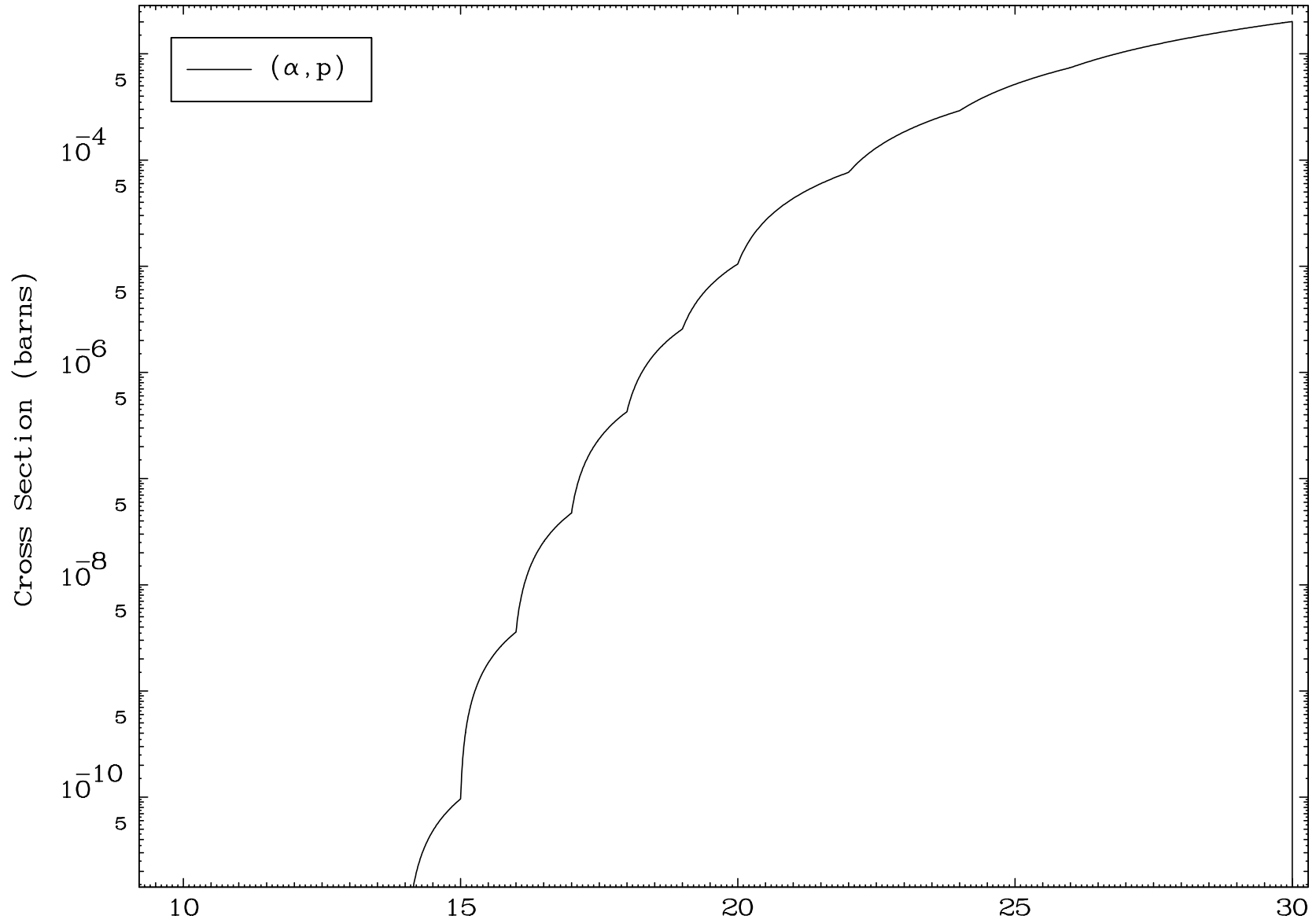
76-0s-188

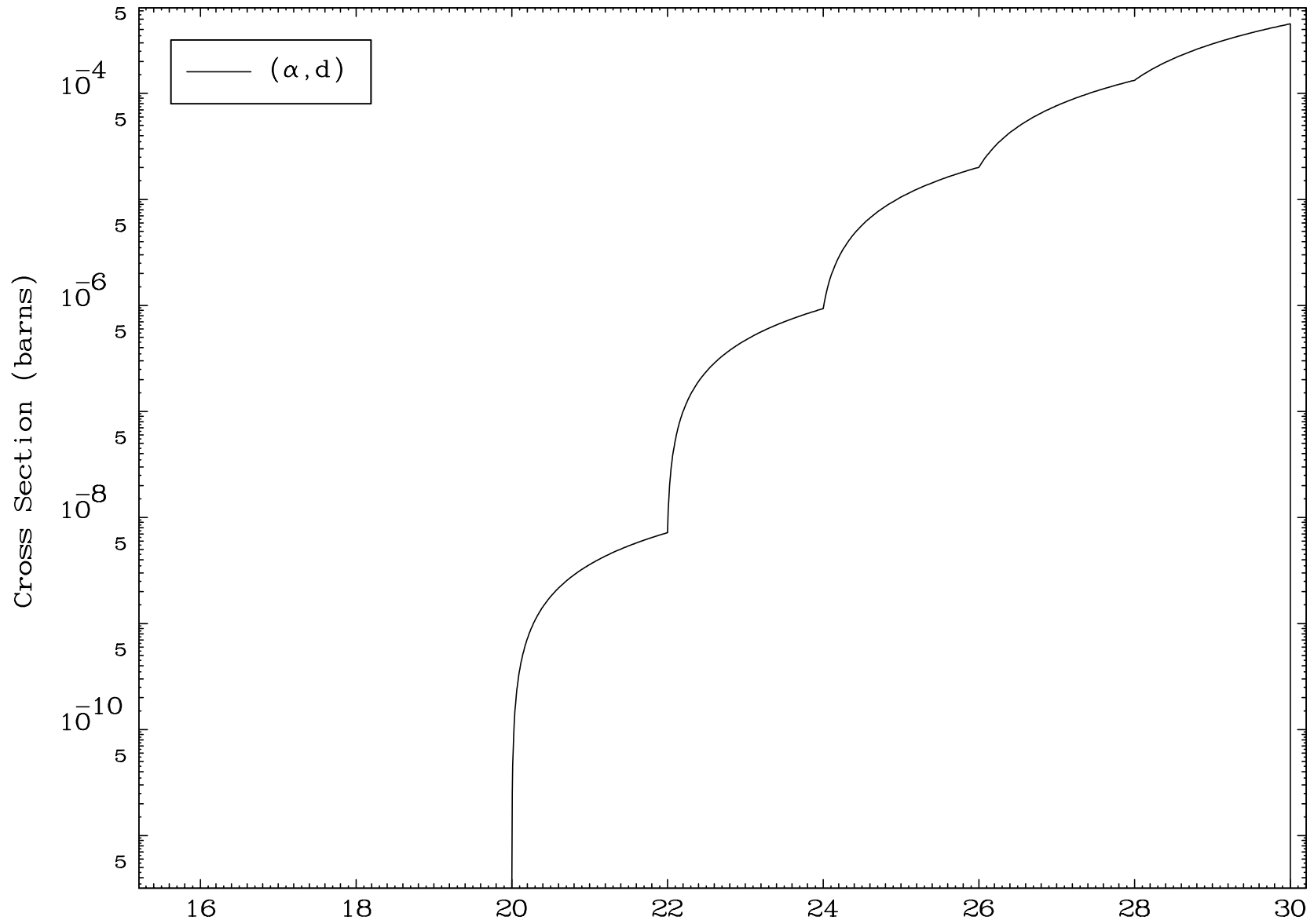


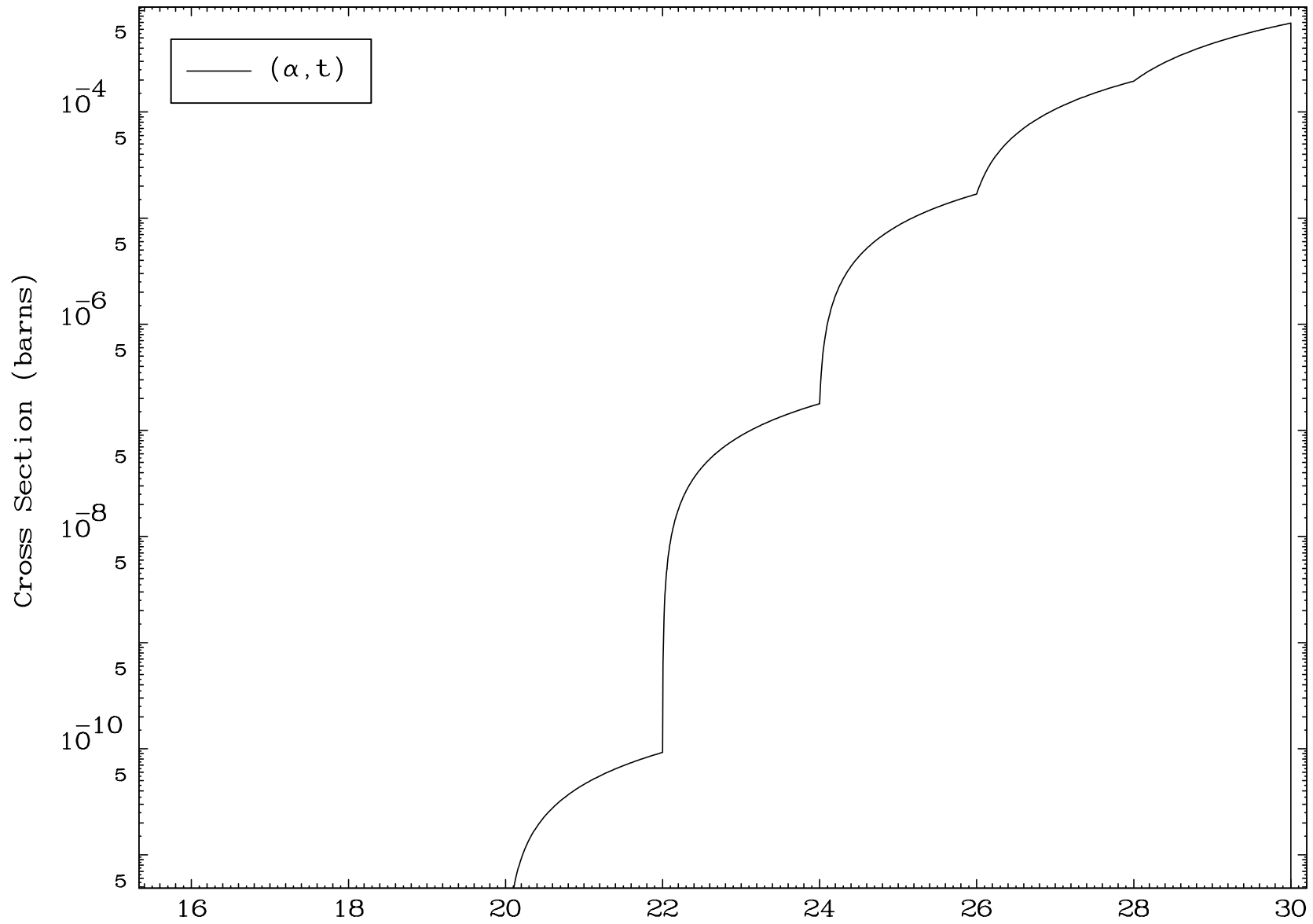
5

Incident Energy (MeV)

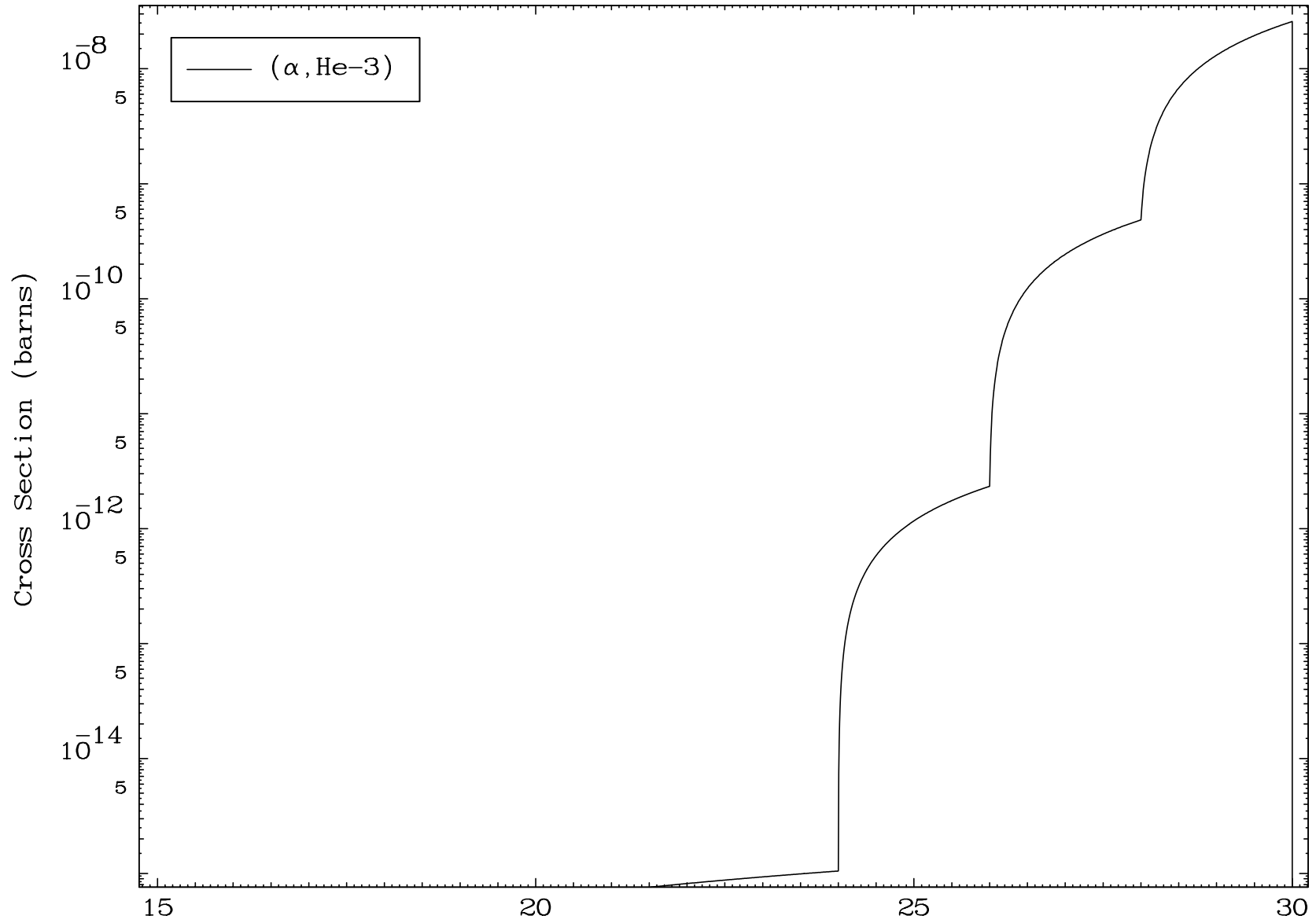
76-0s-188

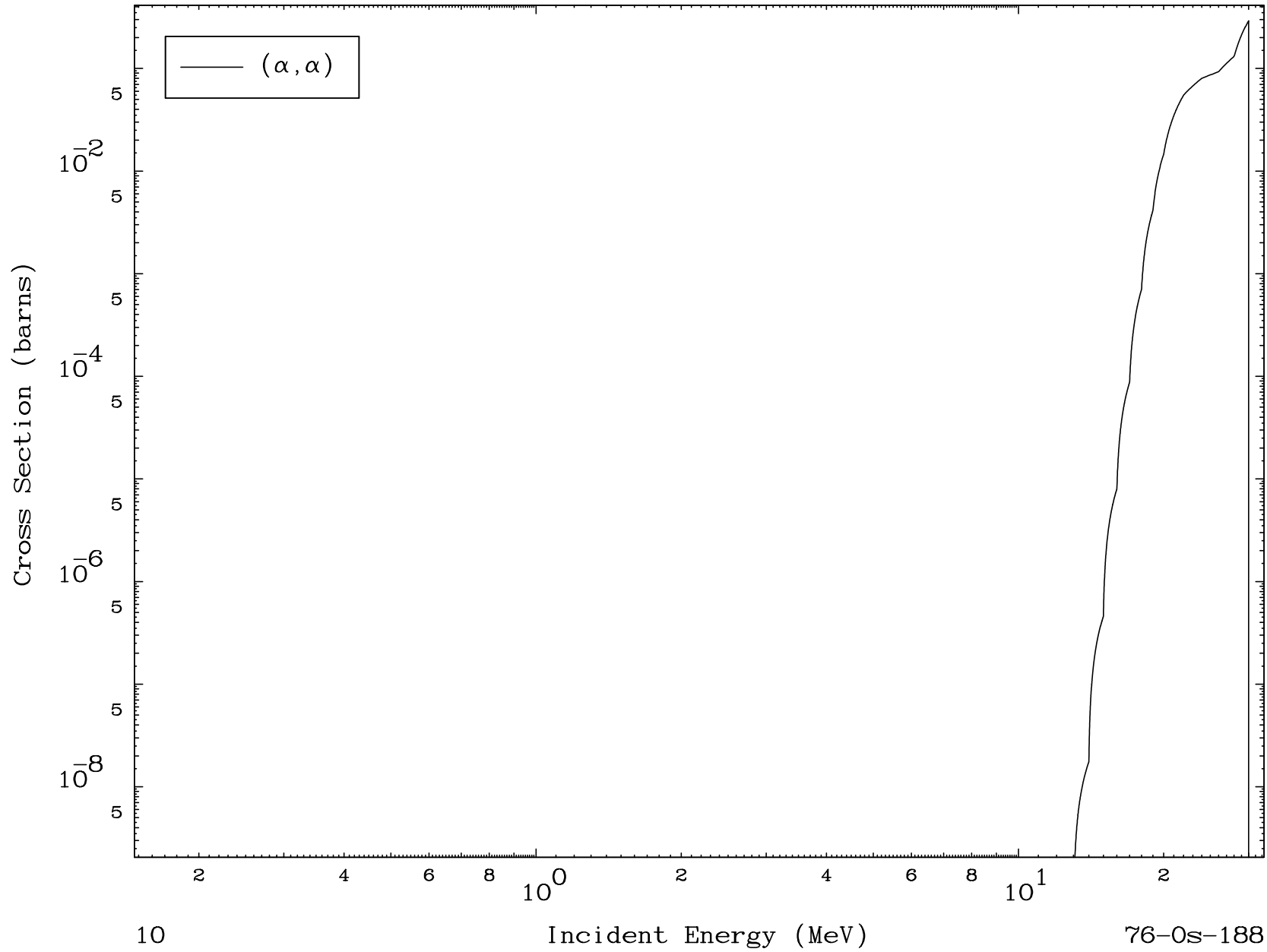




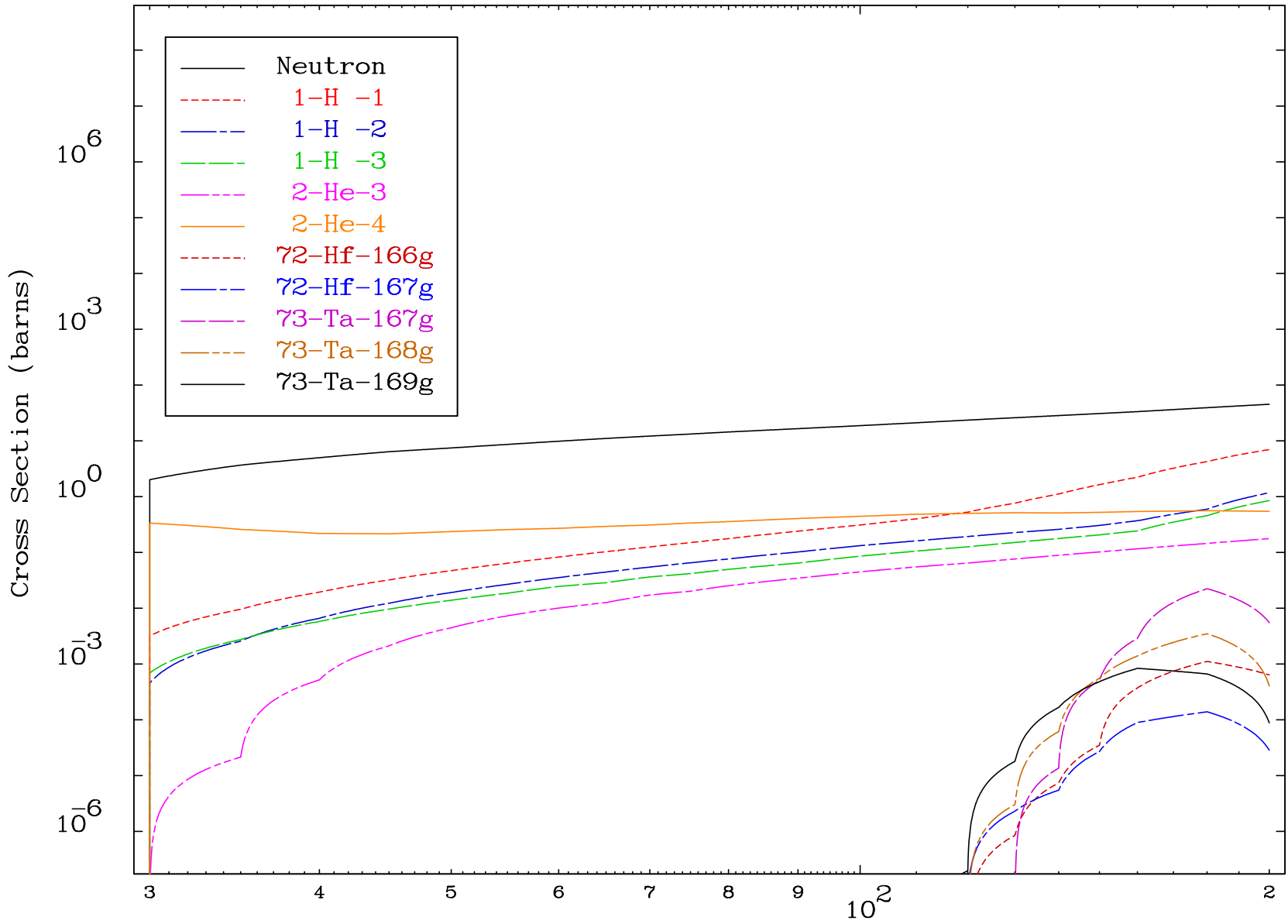




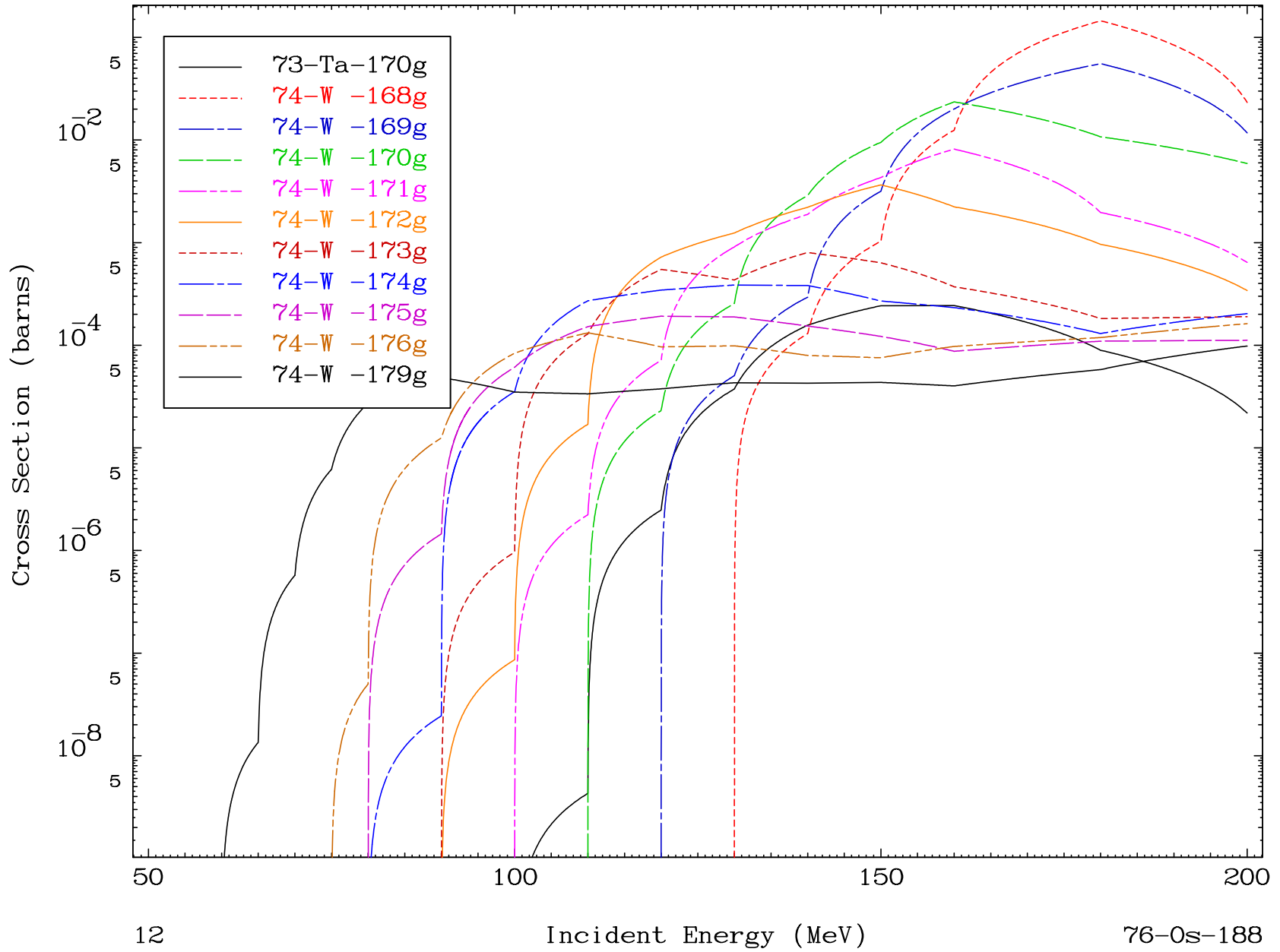




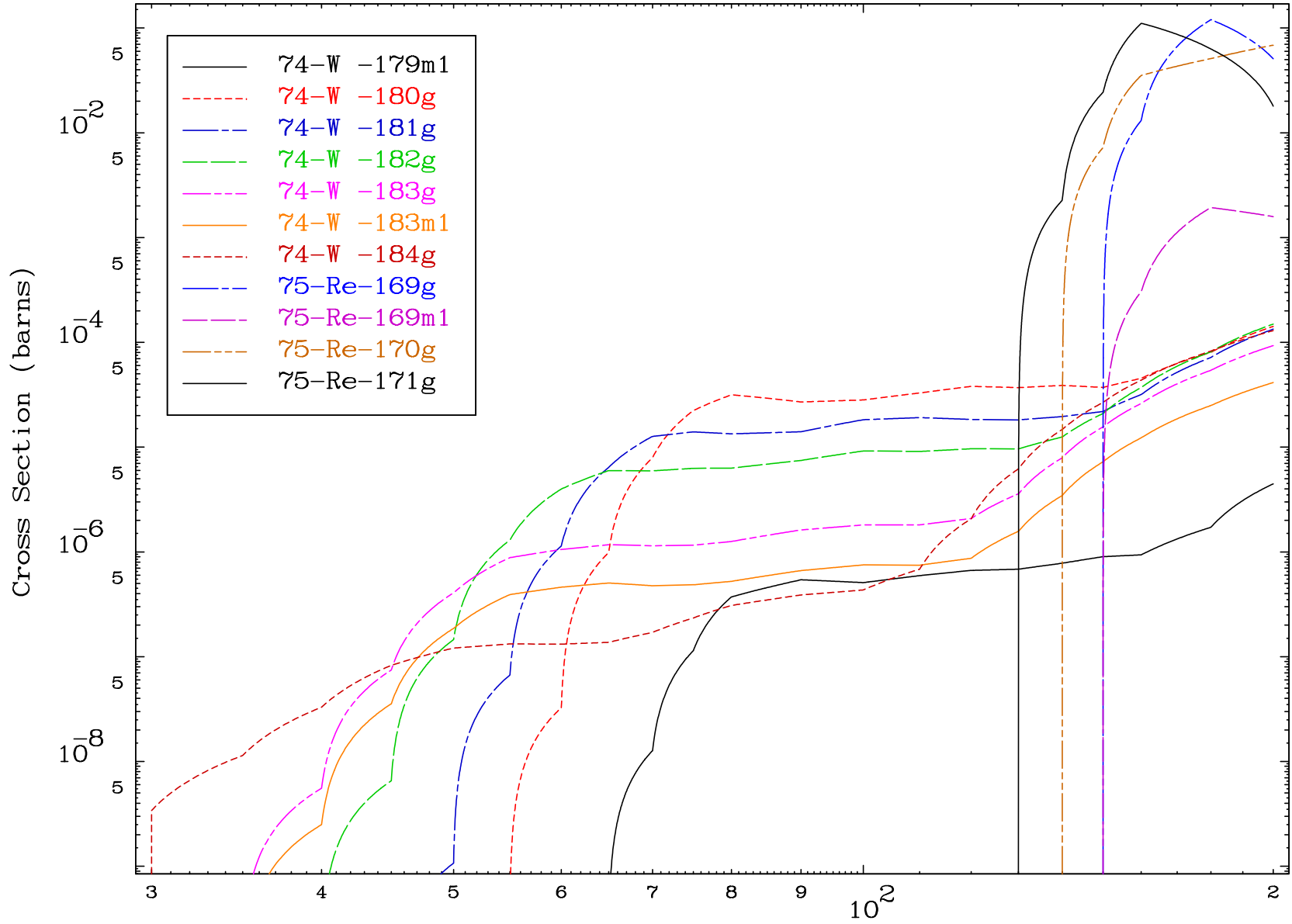
Radionuclide Production Cross Section



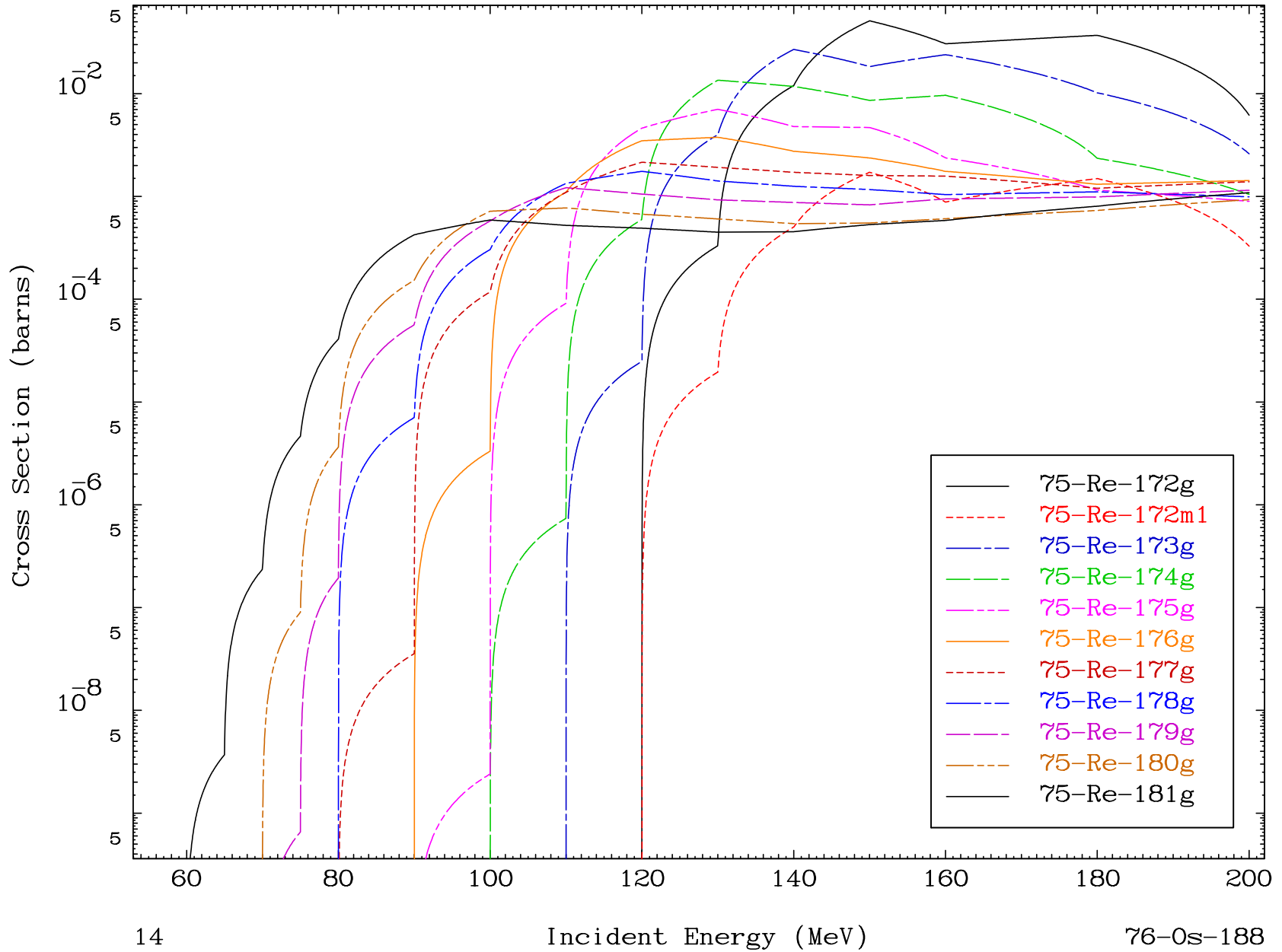
Radionuclide Production Cross Section



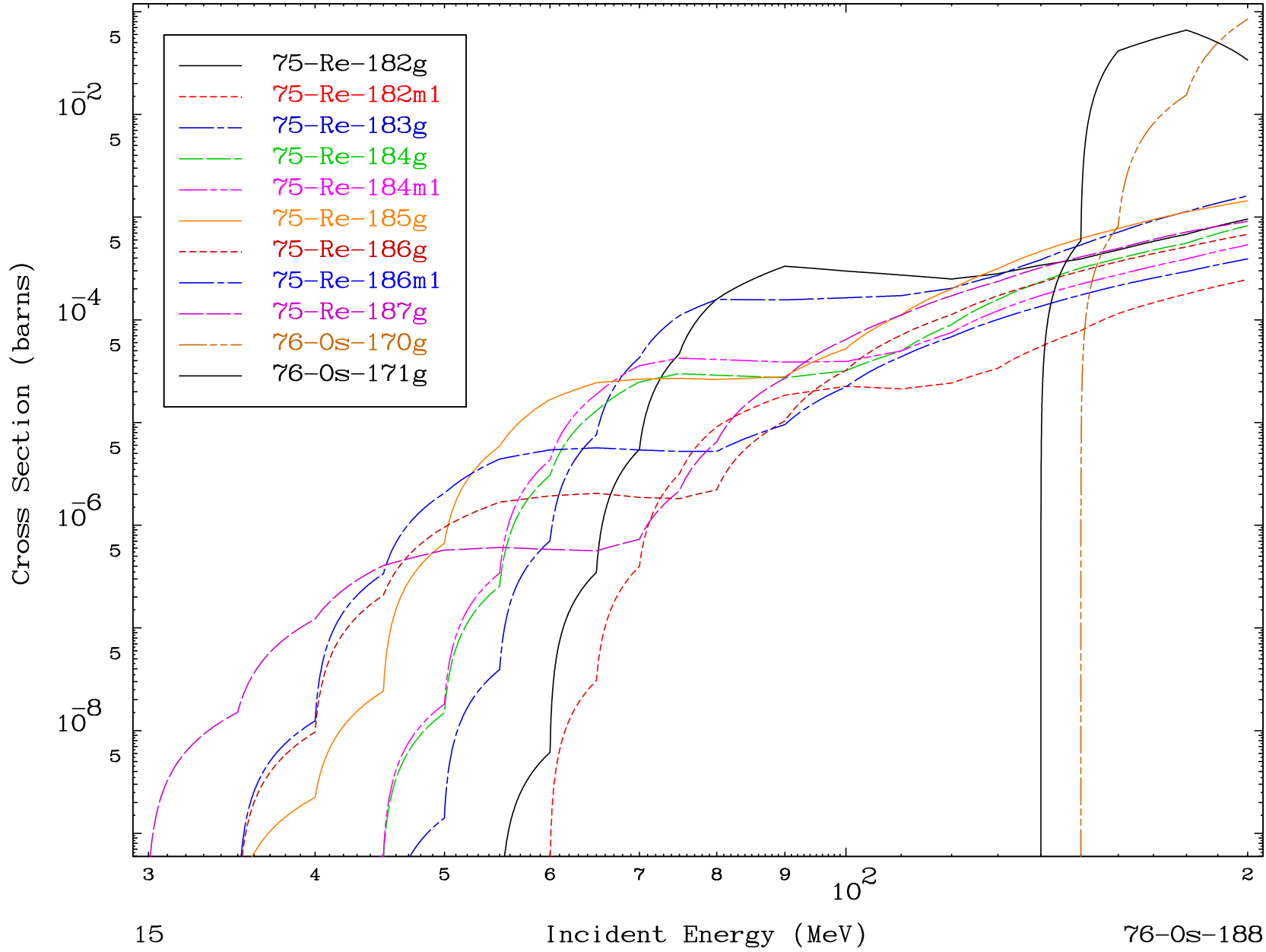
Radionuclide Production Cross Section



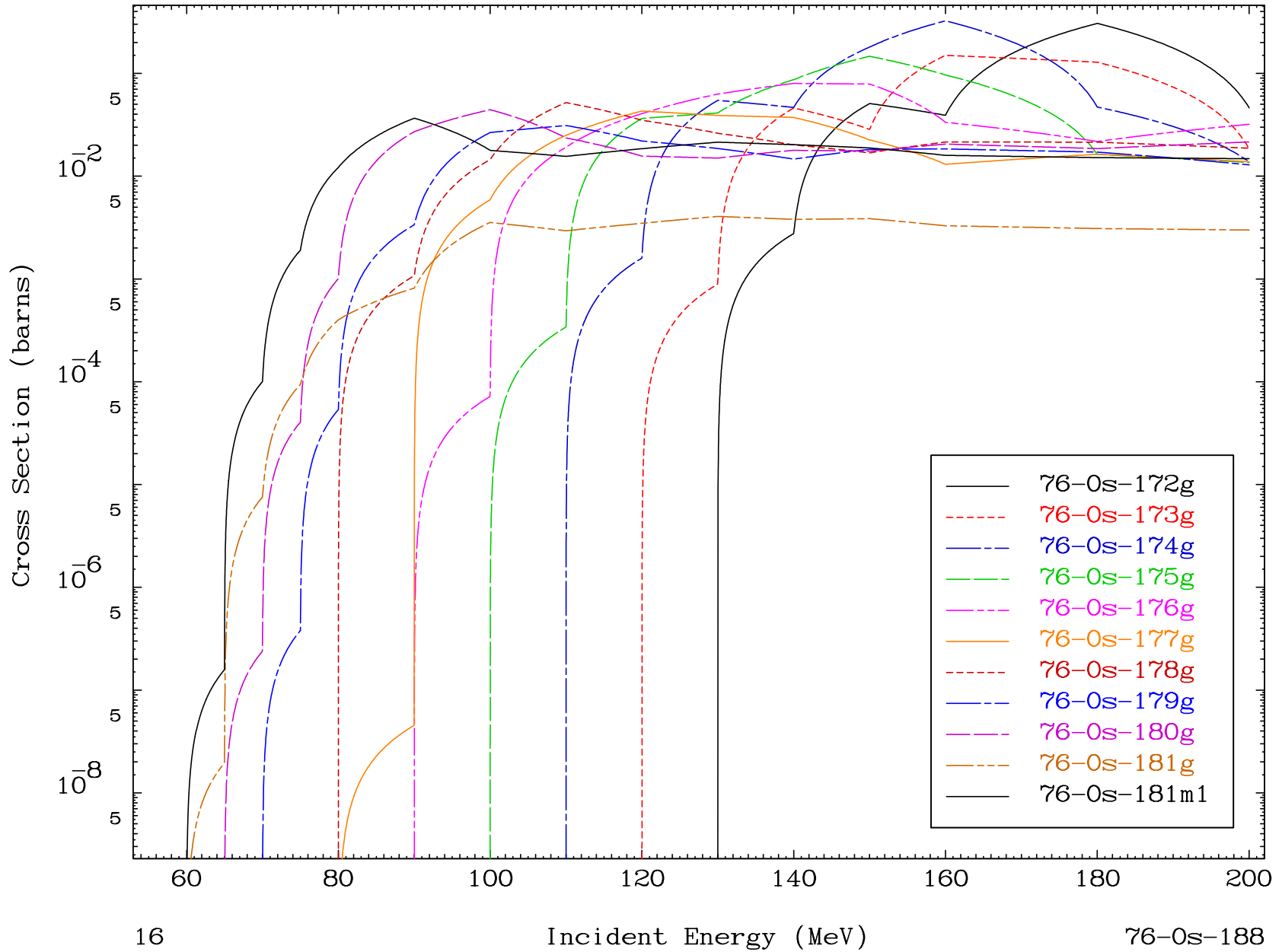
Radionuclide Production Cross Section



Radionuclide Production Cross Section

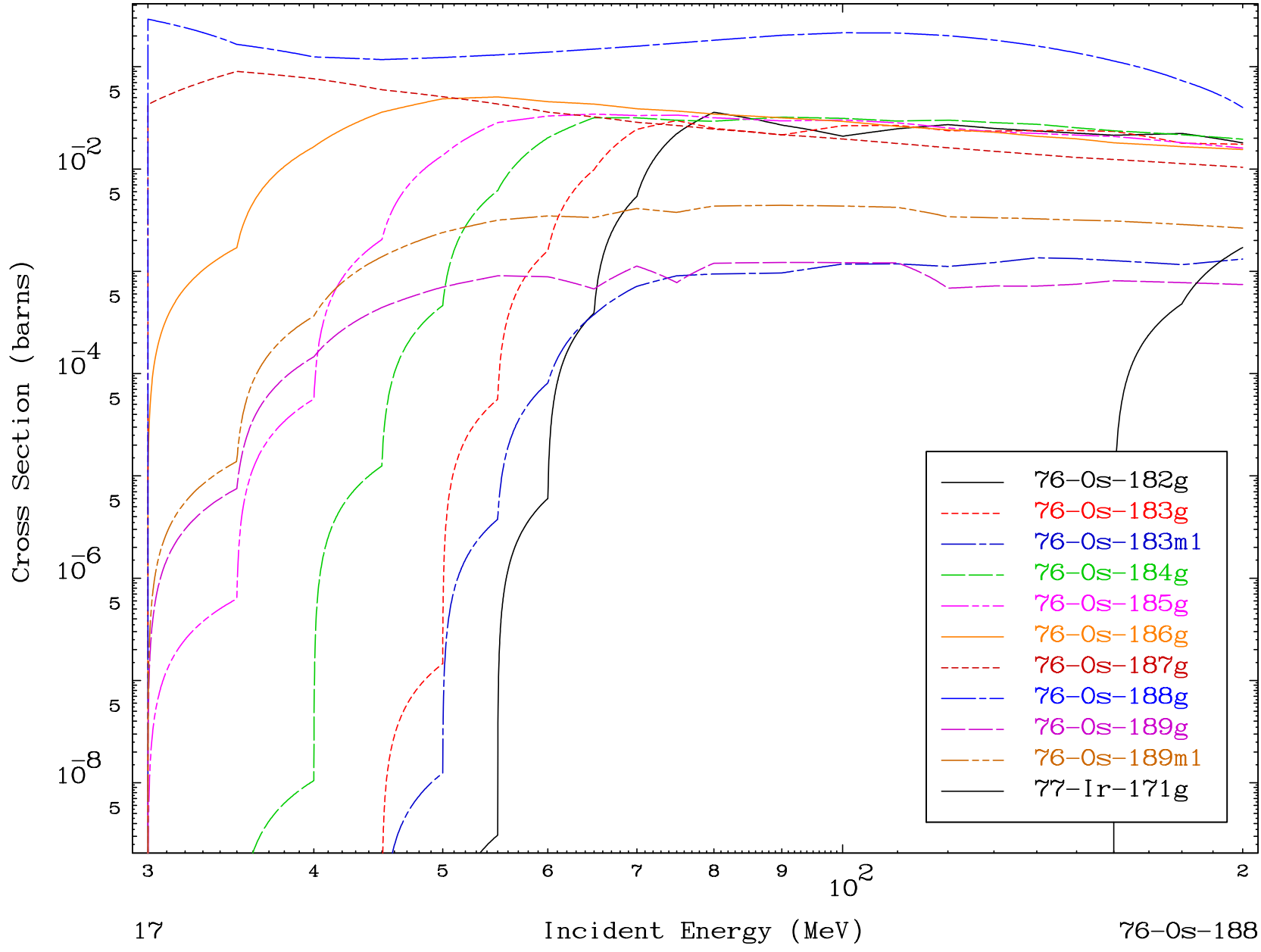


Radionuclide Production Cross Section

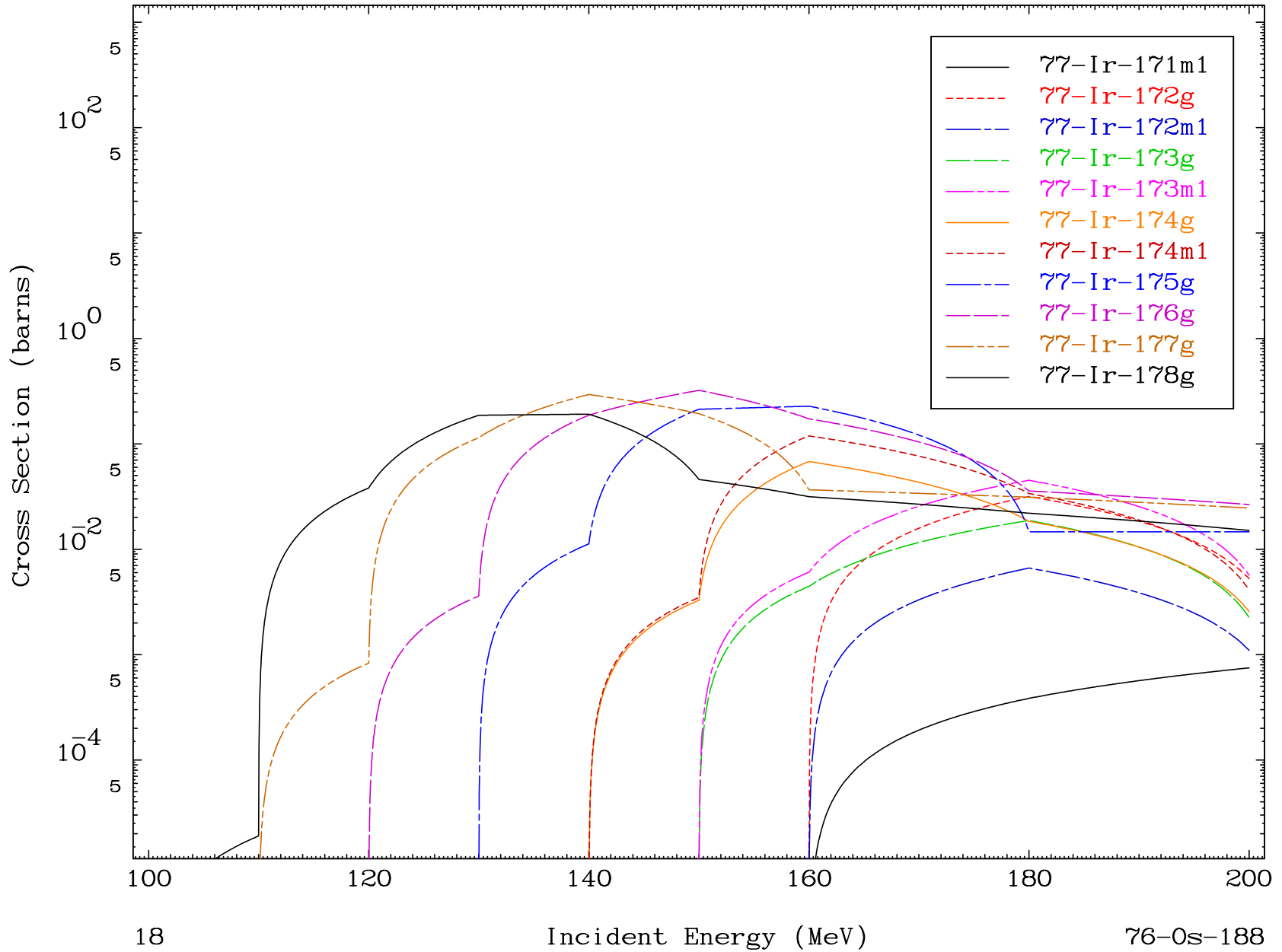




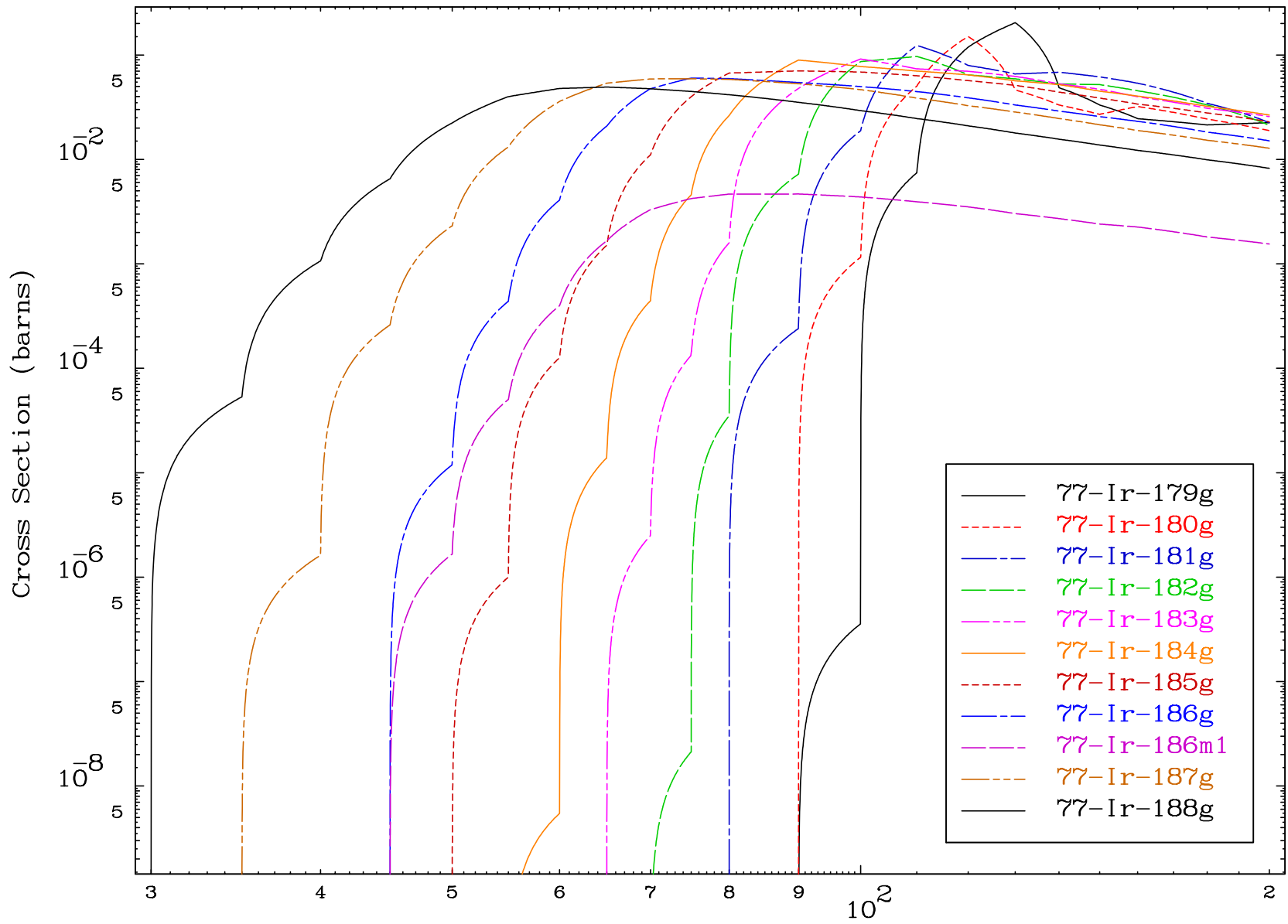
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

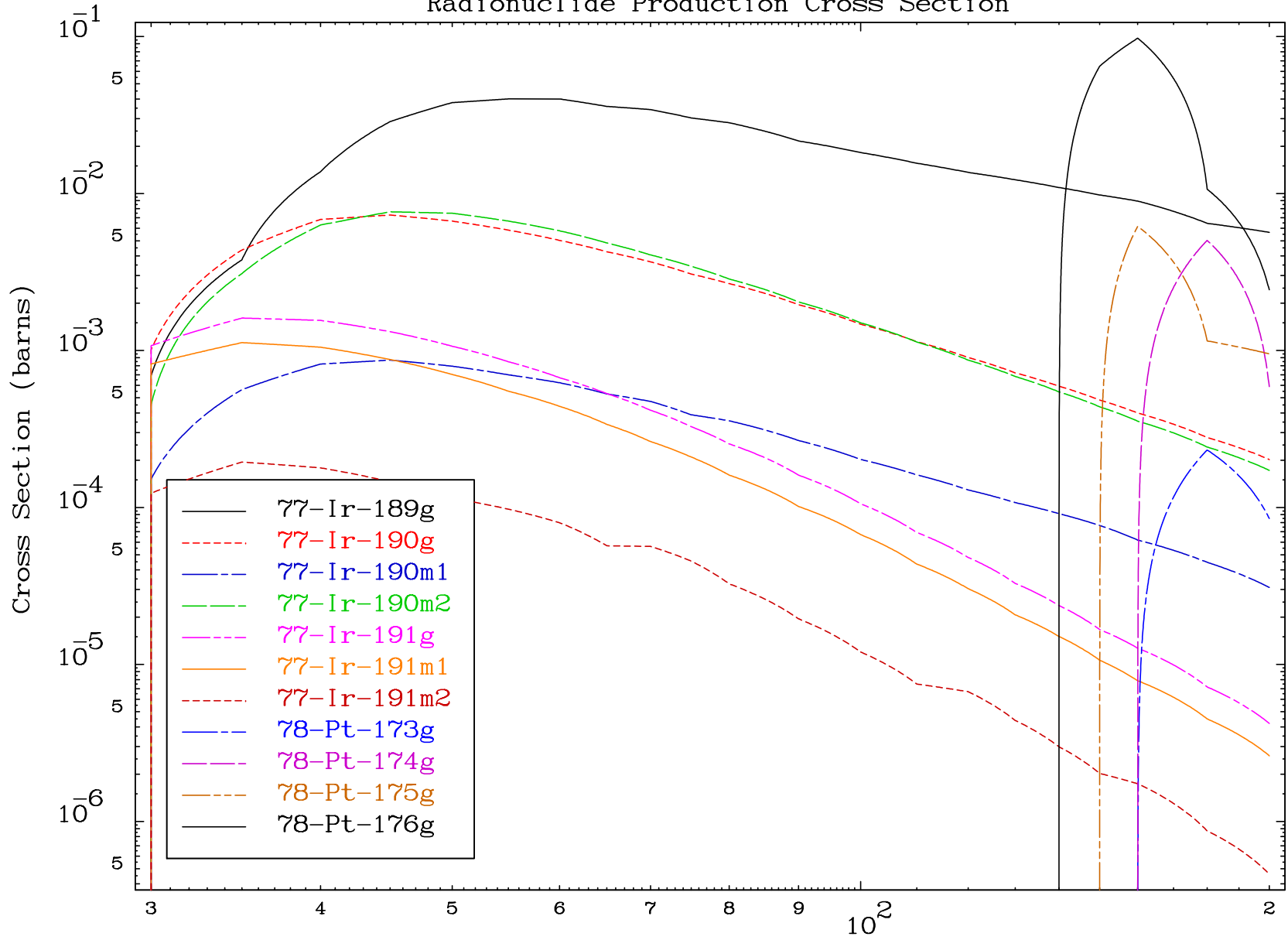


MAT 7637

( $\alpha$ , remainder)

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### Radionuclide Production Cross Section

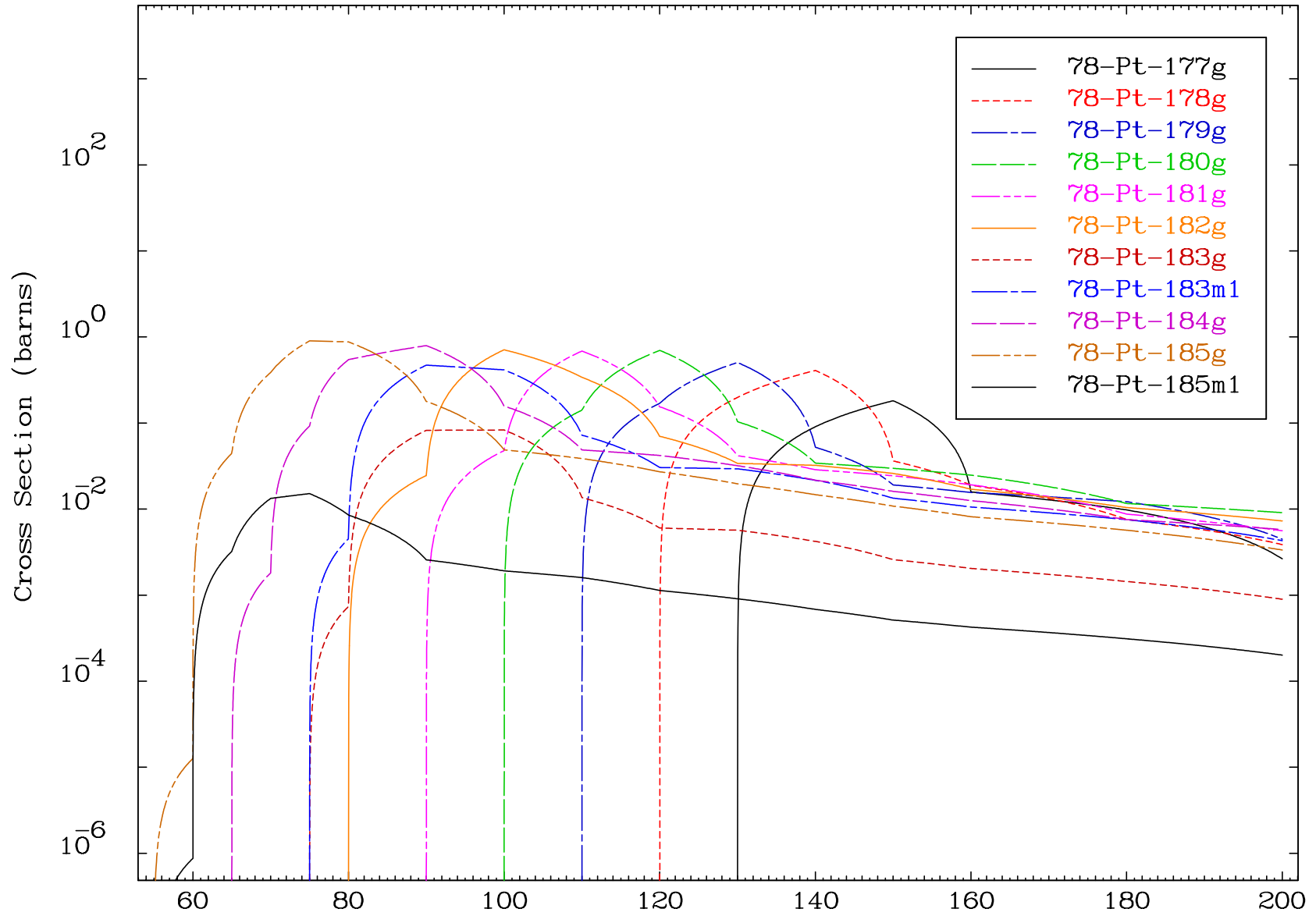


20

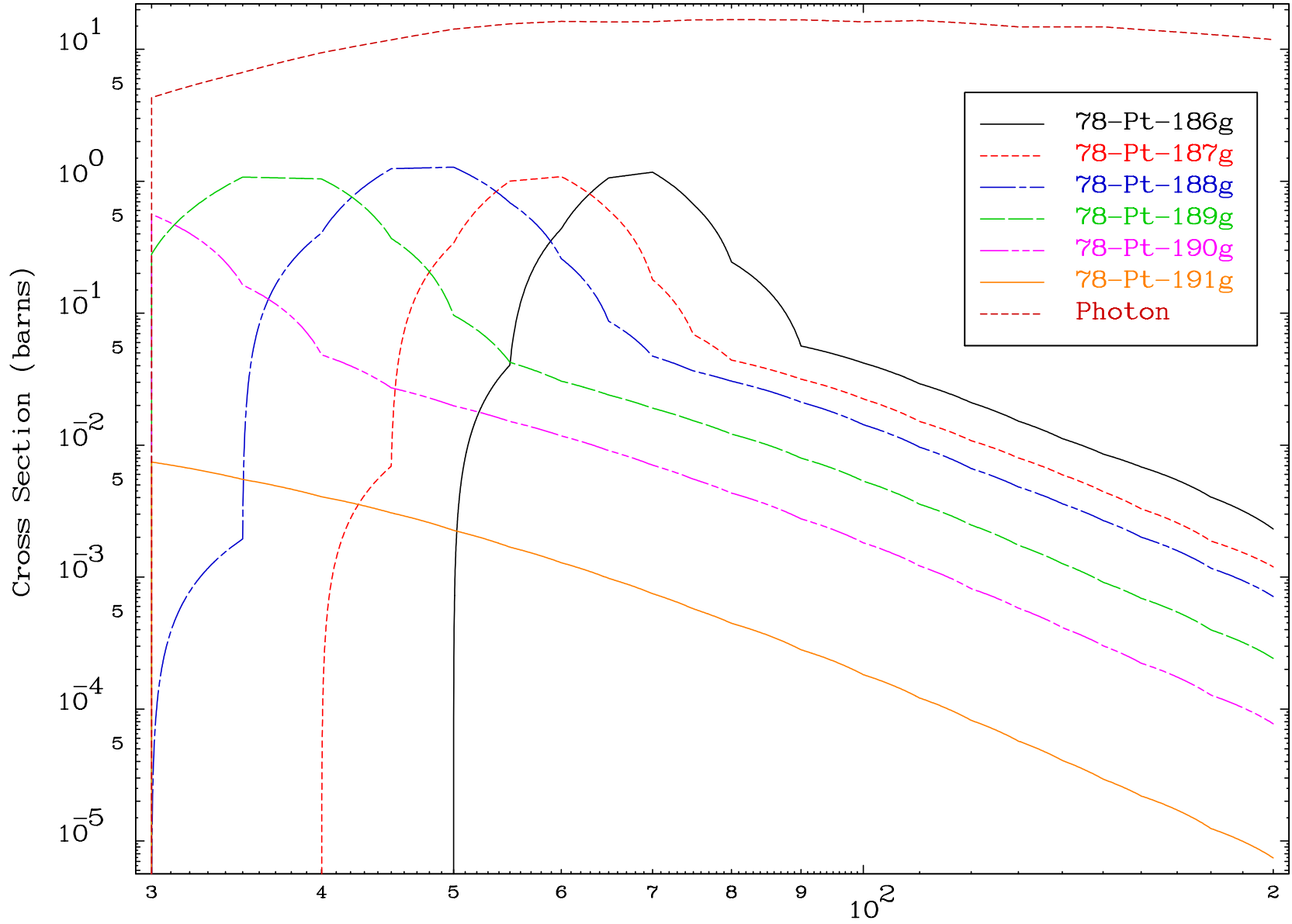
Incident Energy (MeV)

76-0s-188

Radionuclide Production Cross Section



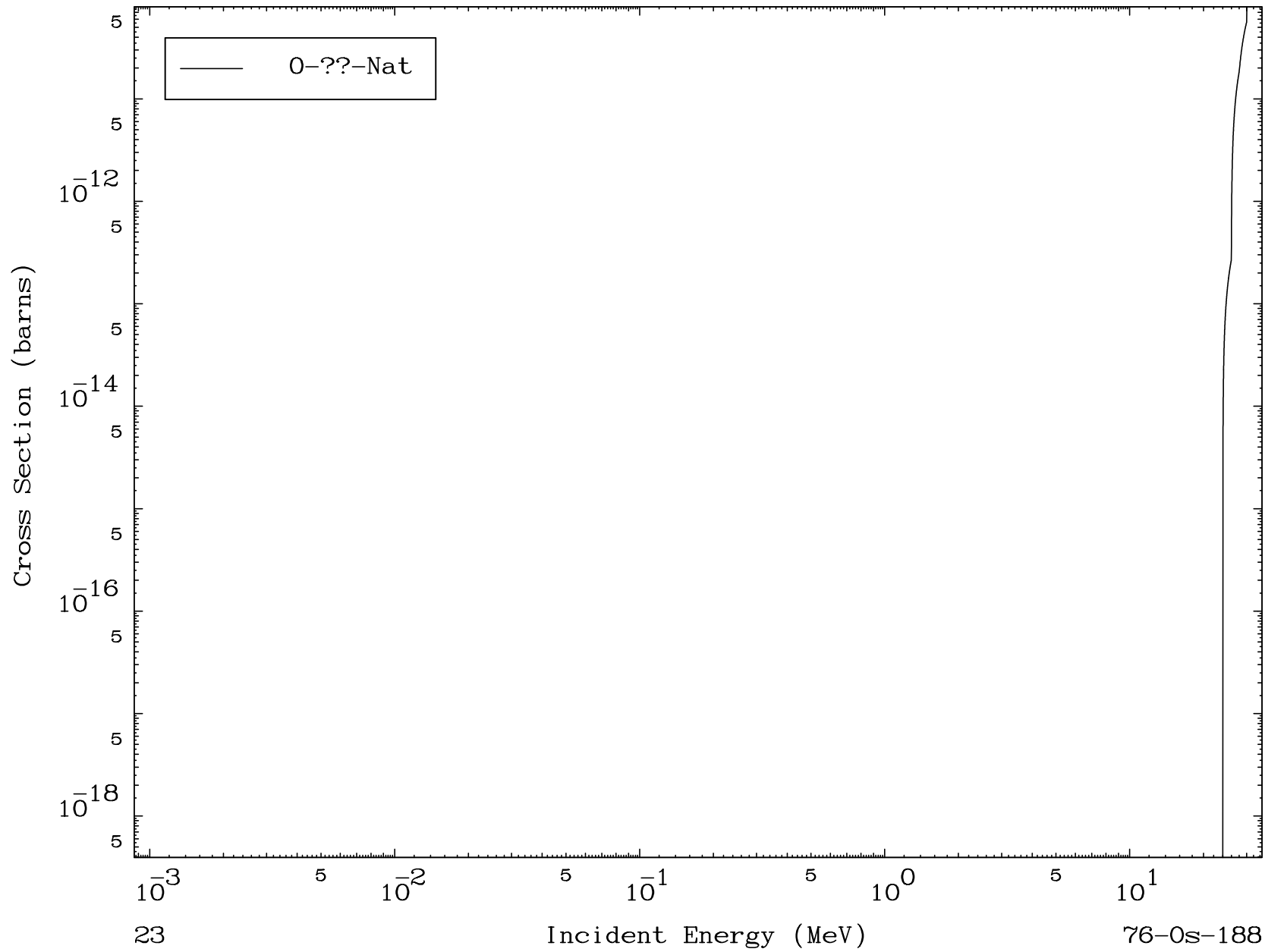
Radionuclide Production Cross Section



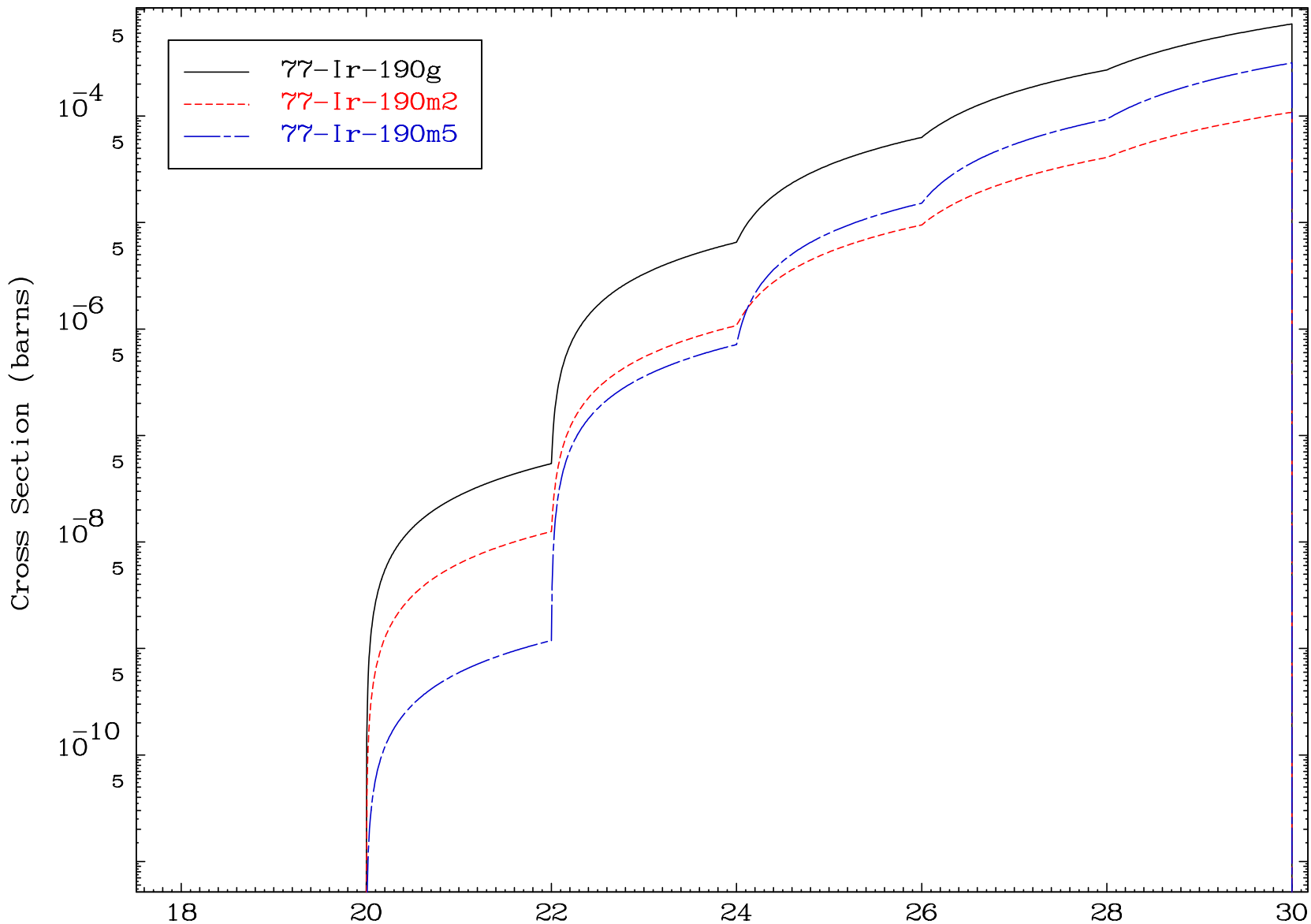
MAT 7637

$\alpha$  Fission  
Radionuclide Production Cross Section

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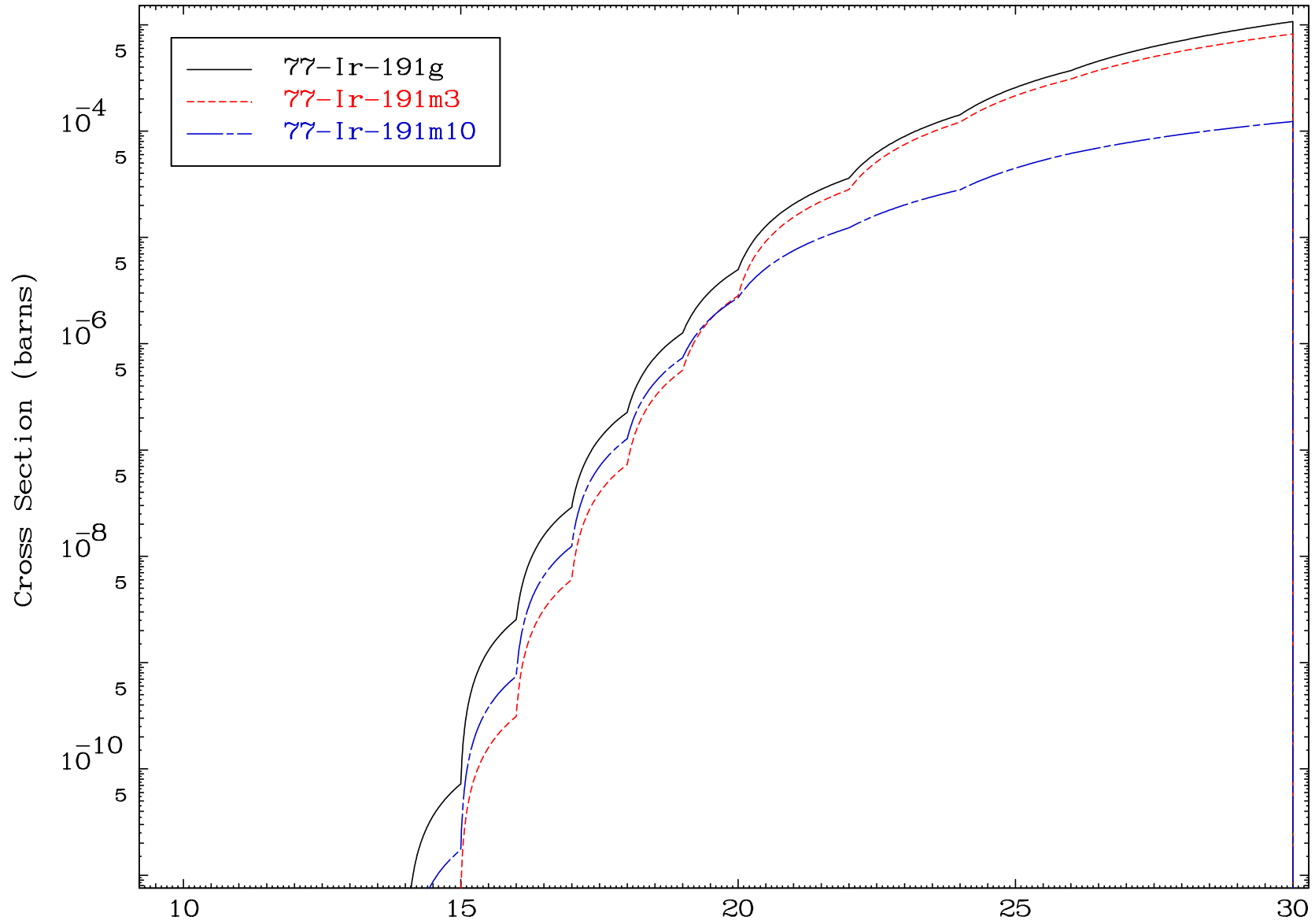


Radionuclide Production Cross Section

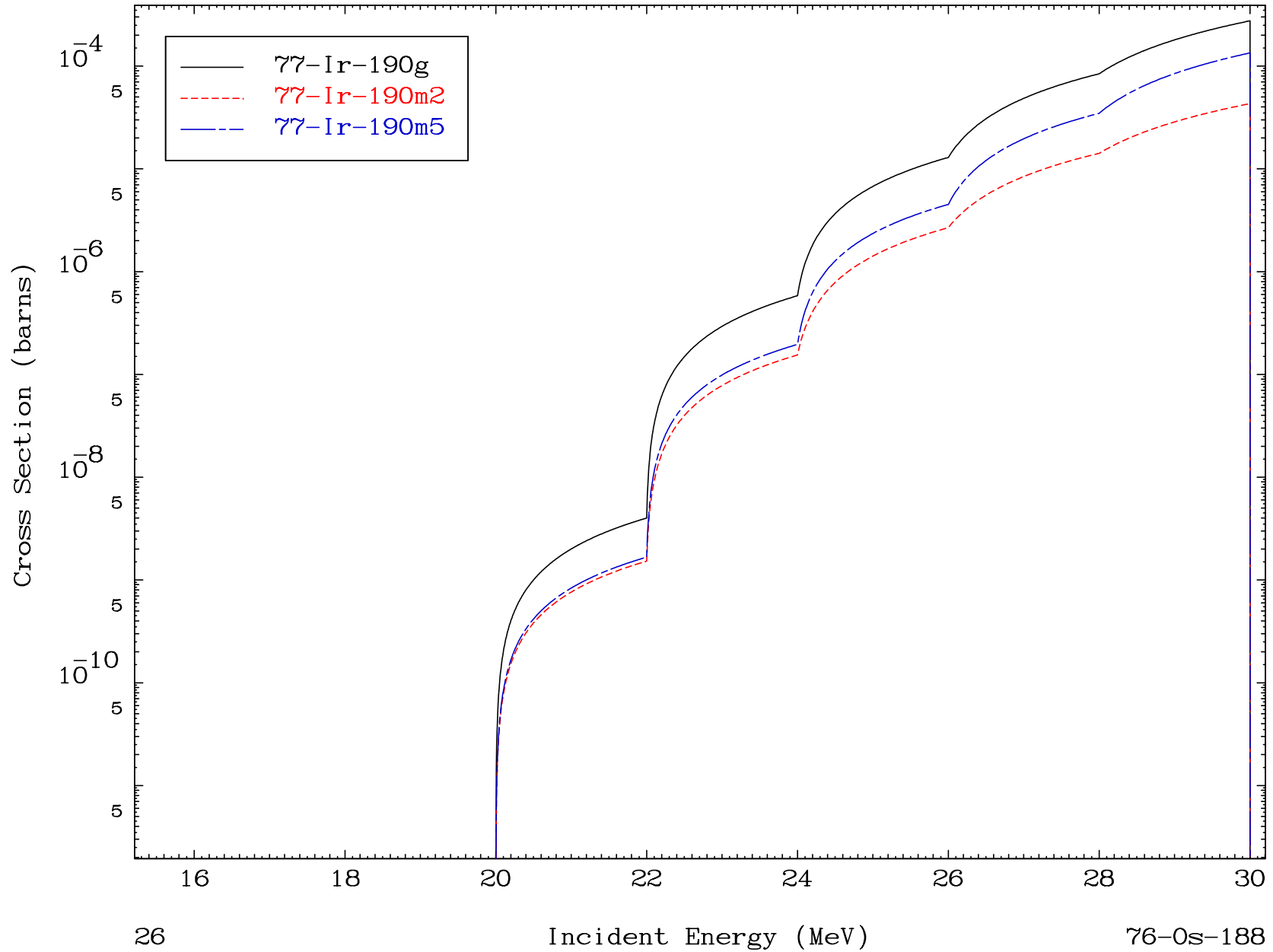




Radionuclide Production Cross Section



Radionuclide Production Cross Section



MAT 7637

( $\alpha, \text{He-3}$ )

76-0s-188

Radionuclide Production Cross Section

