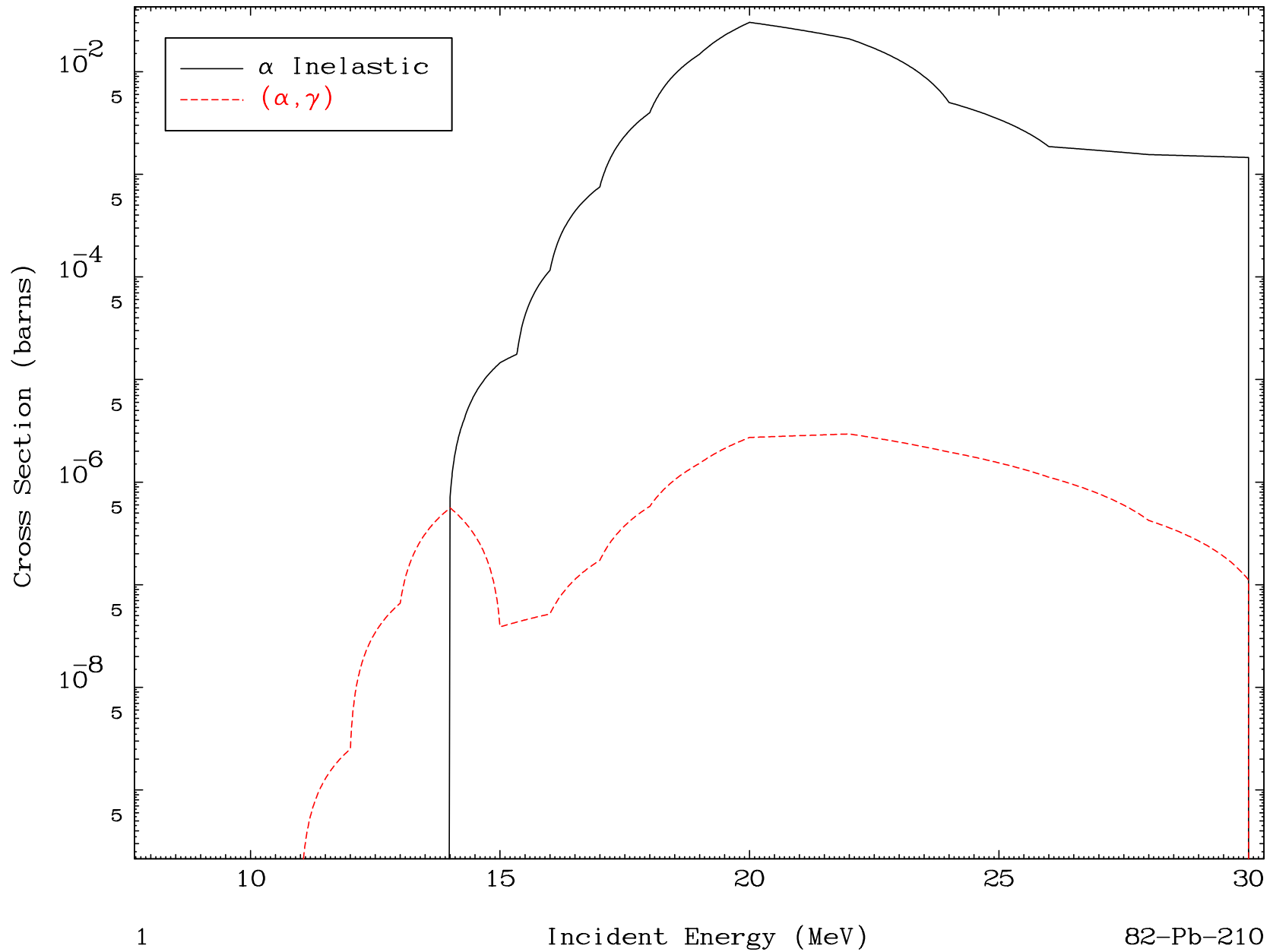


MAT 8243

$\alpha$  Major  
0 Kelvin Cross Sections

82-Pb-210



1

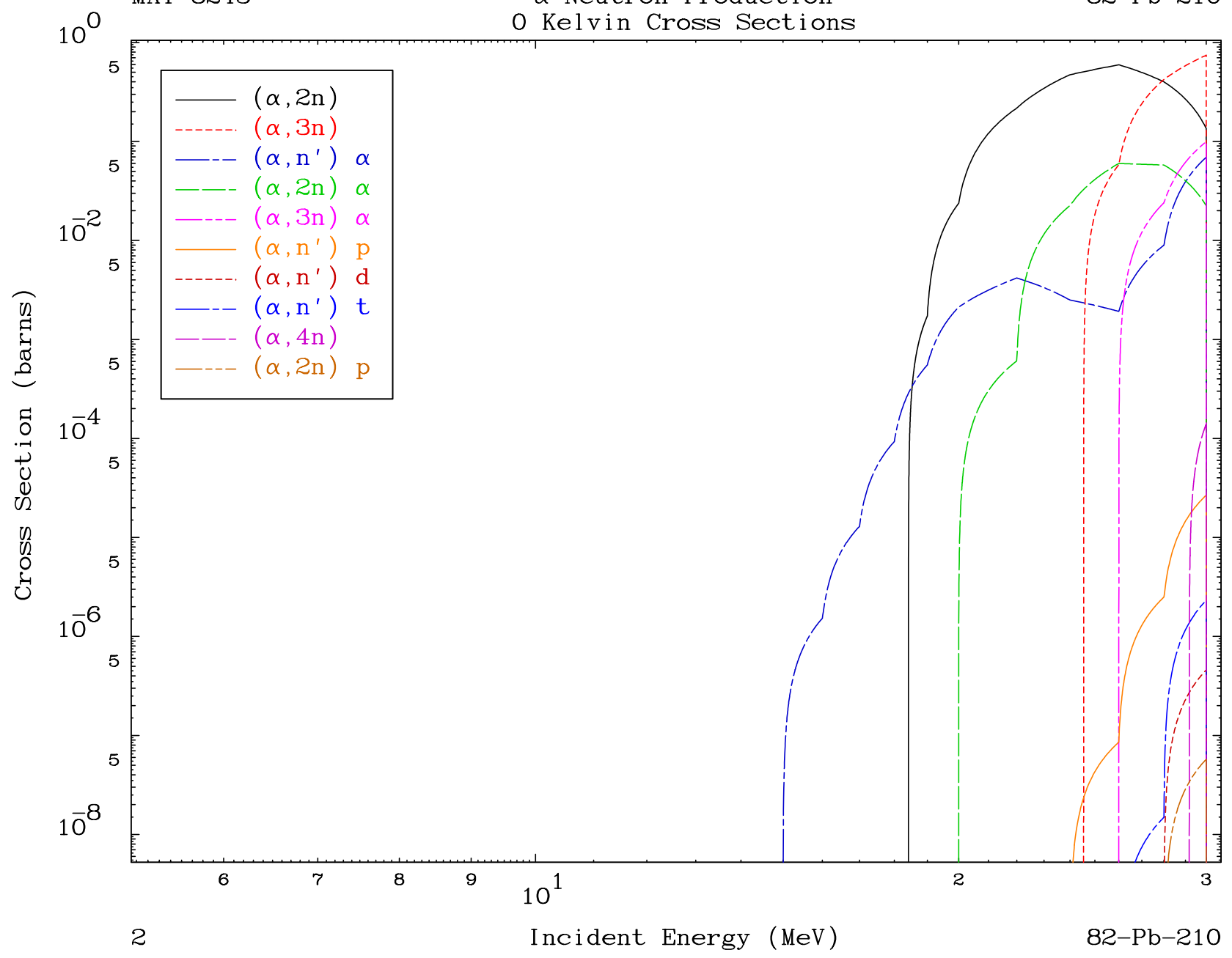
Incident Energy (MeV)

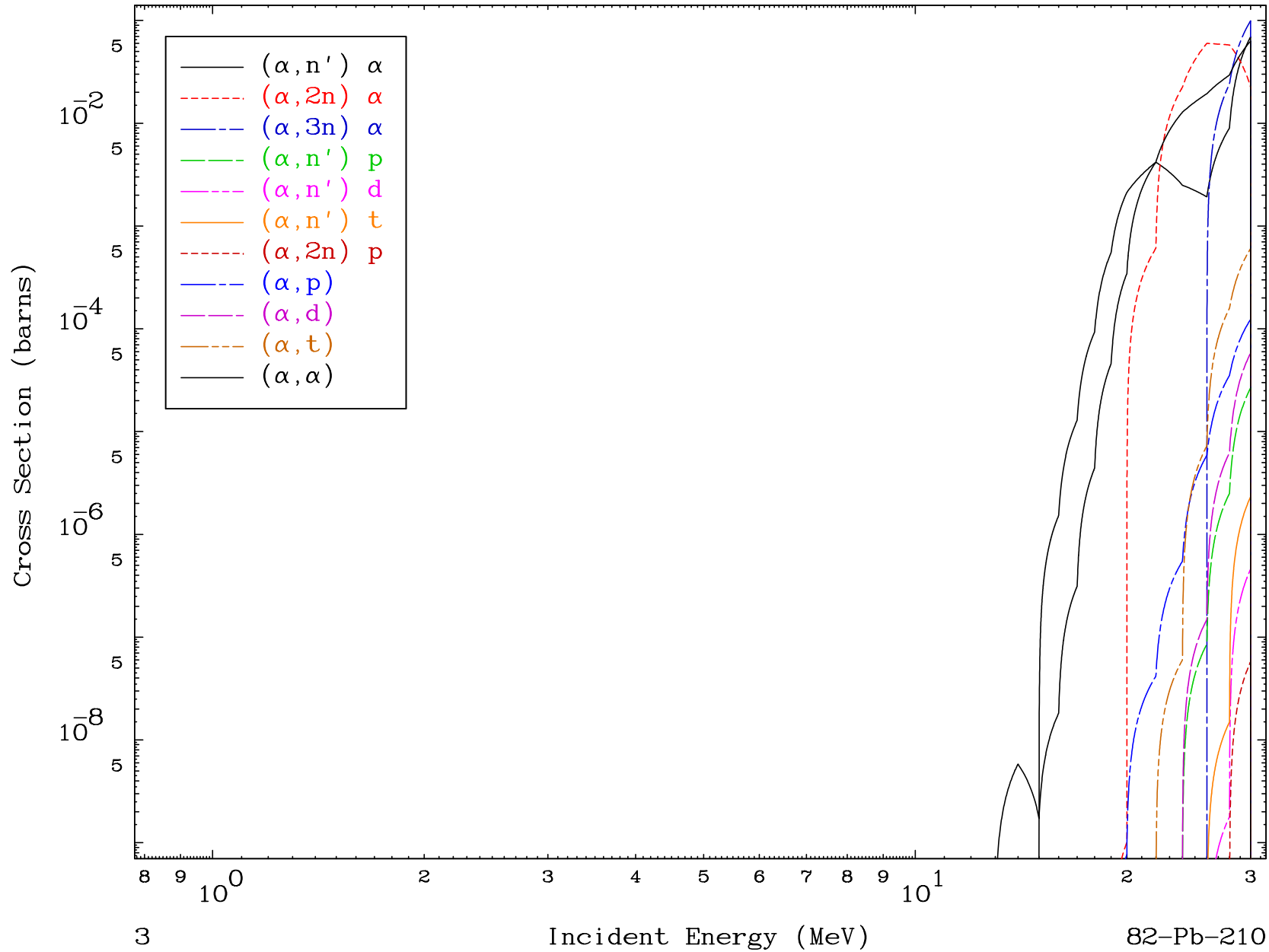
82-Pb-210

MAT 8243

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

82-Pb-210

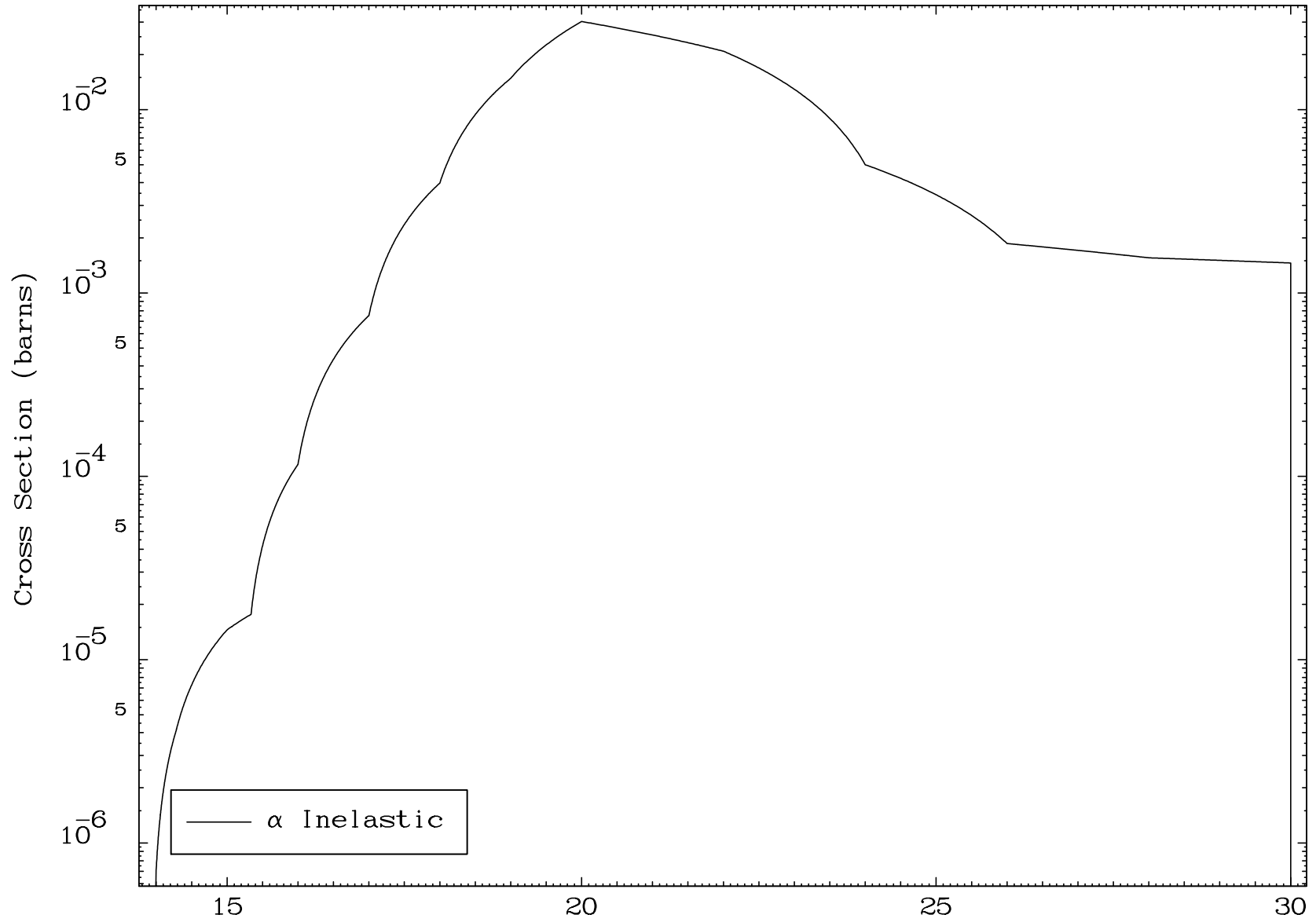




MAT 8243

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

82-Pb-210



4

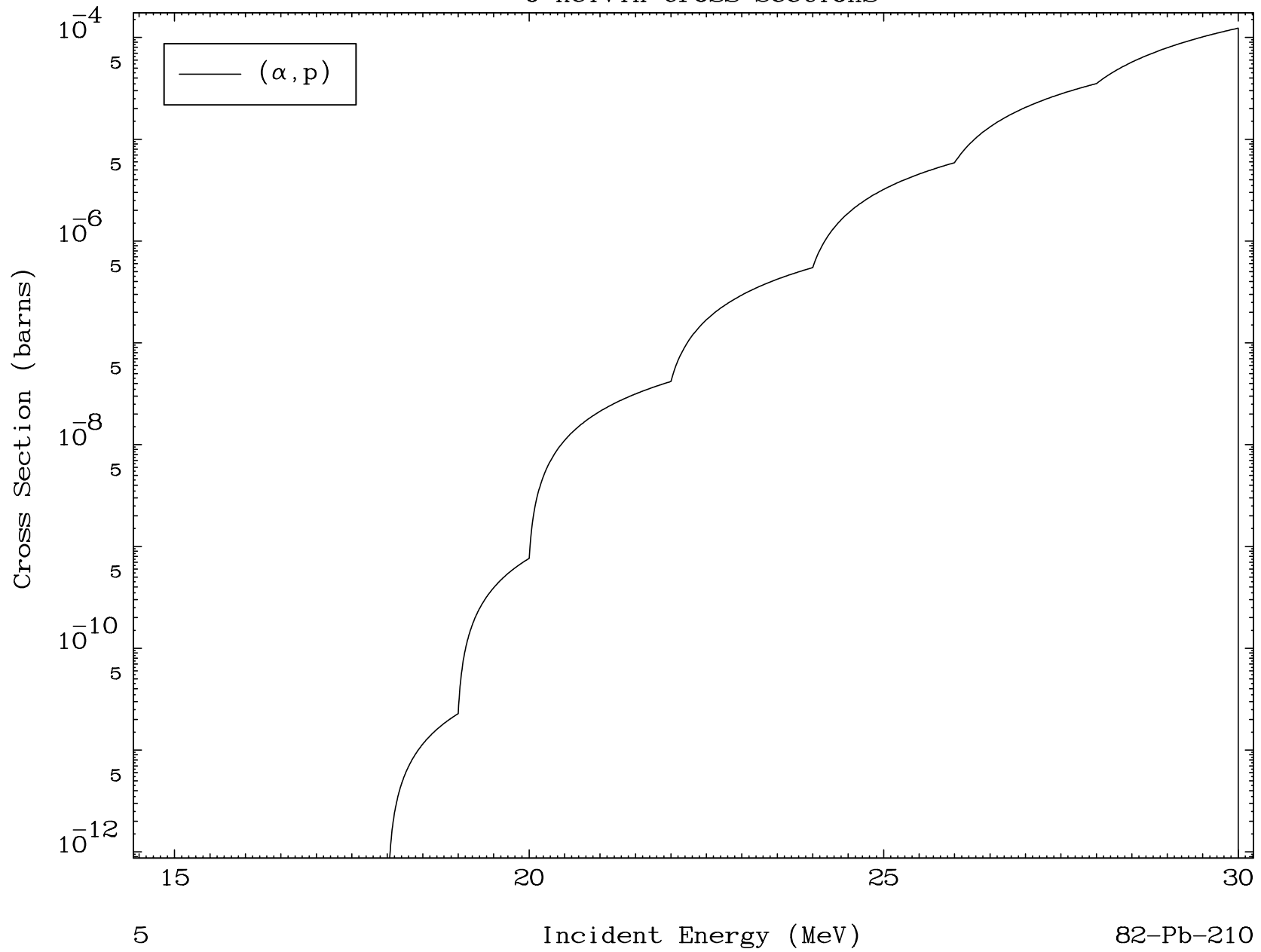
Incident Energy (MeV)

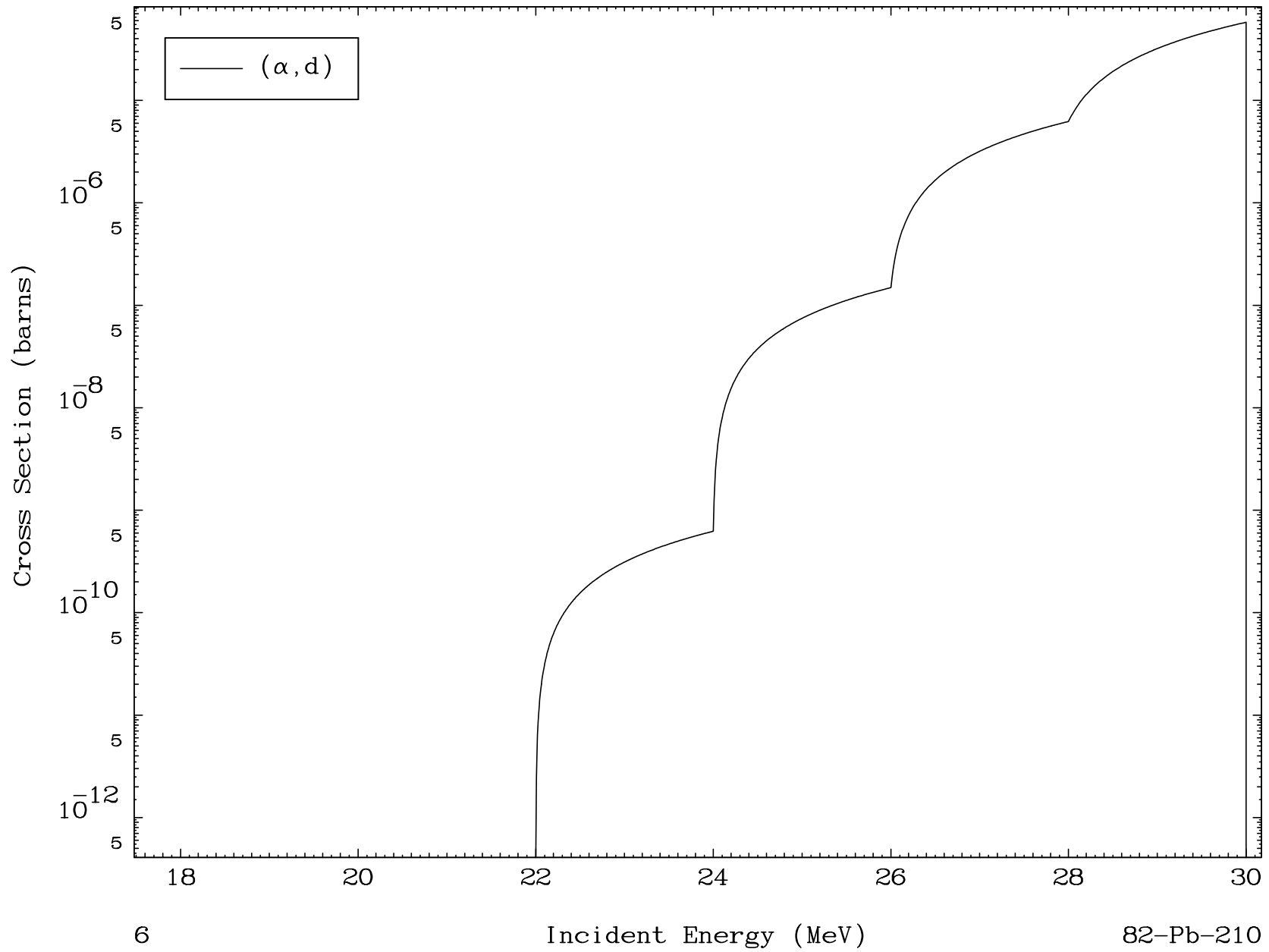
82-Pb-210

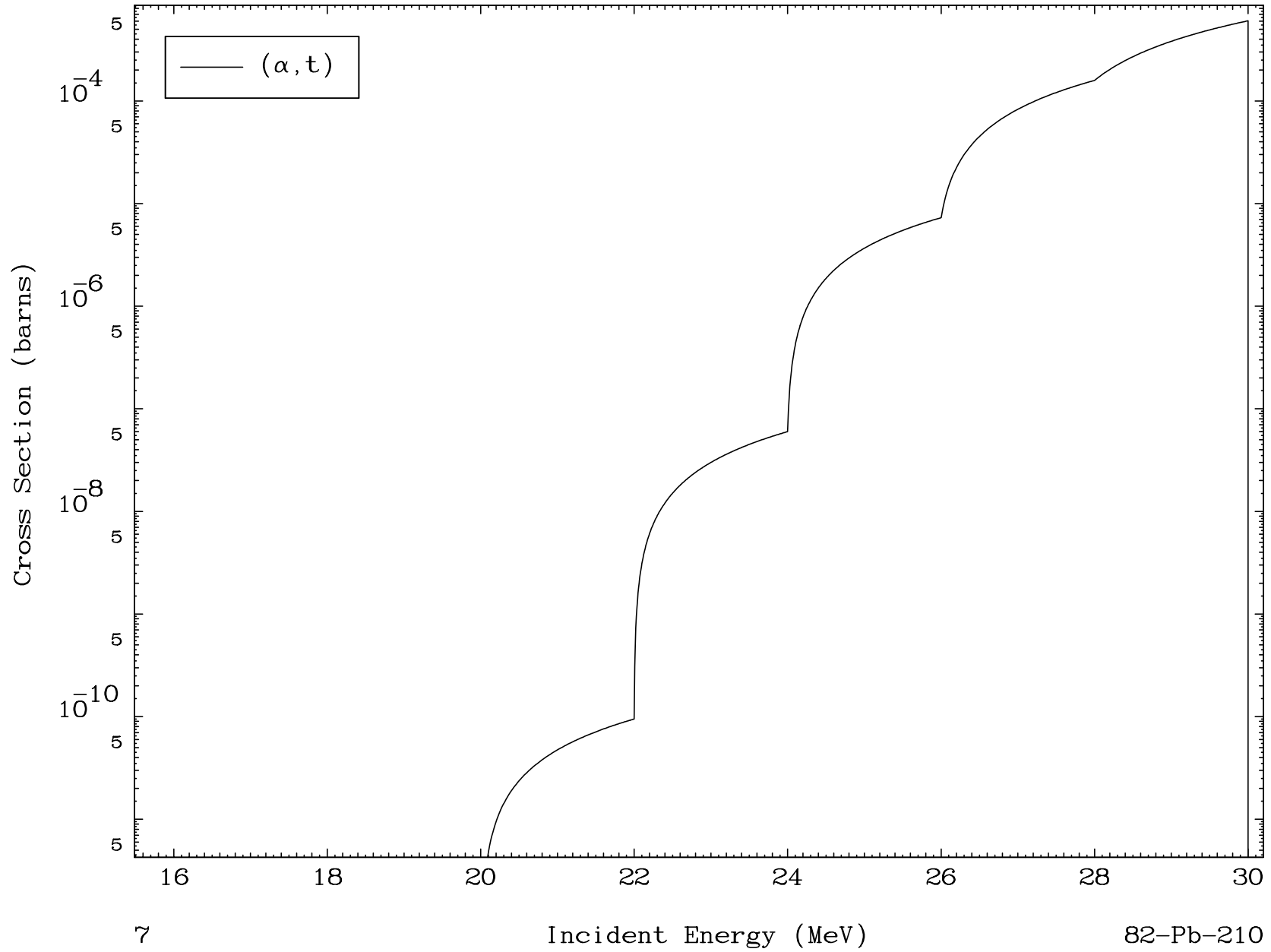
MAT 8243

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections

82-Pb-210



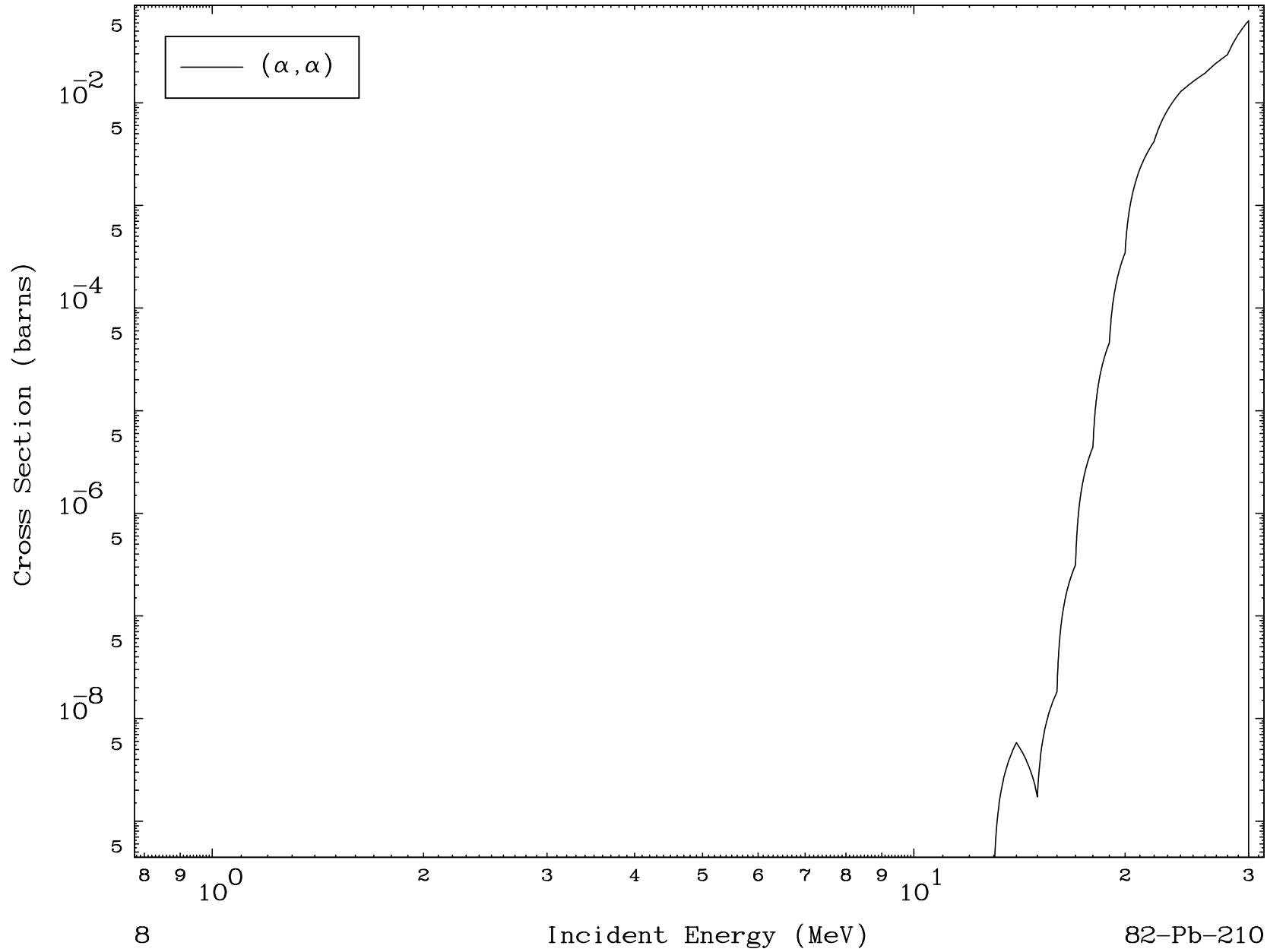




MAT 8243

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

82-Pb-210



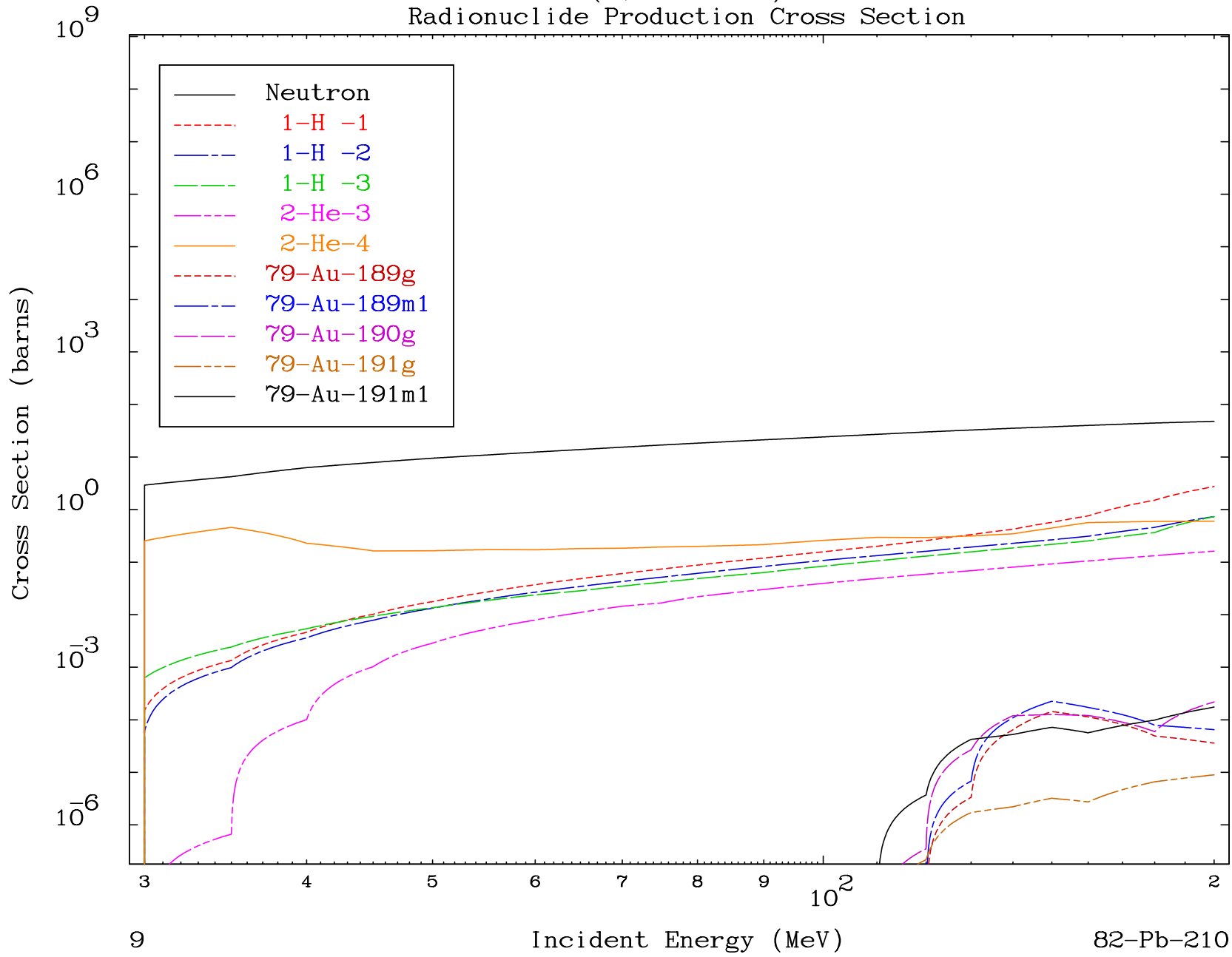


MAT 8243

( $\alpha$ , remainder)

82-Pb-210

Radionuclide Production Cross Section



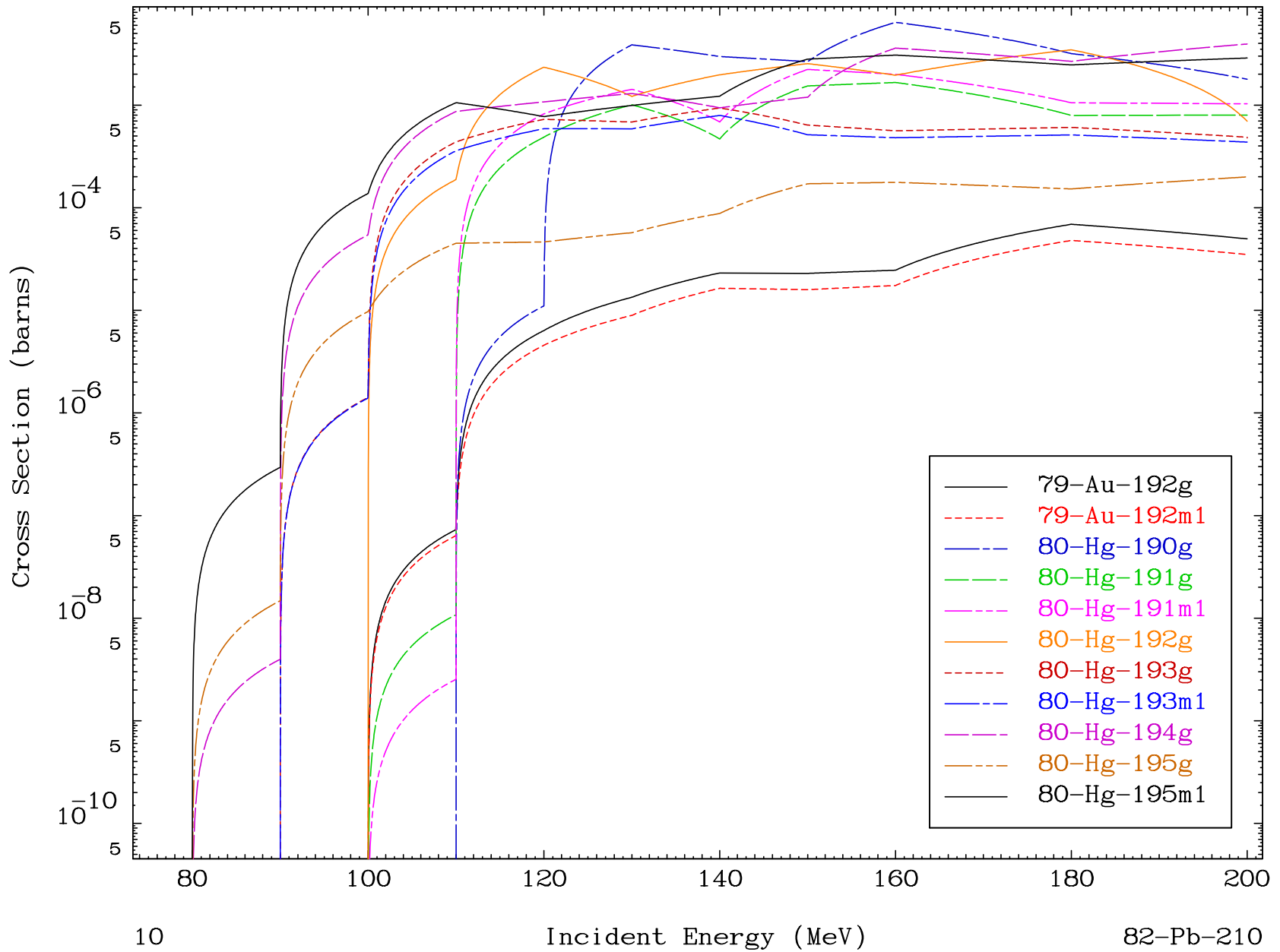
9

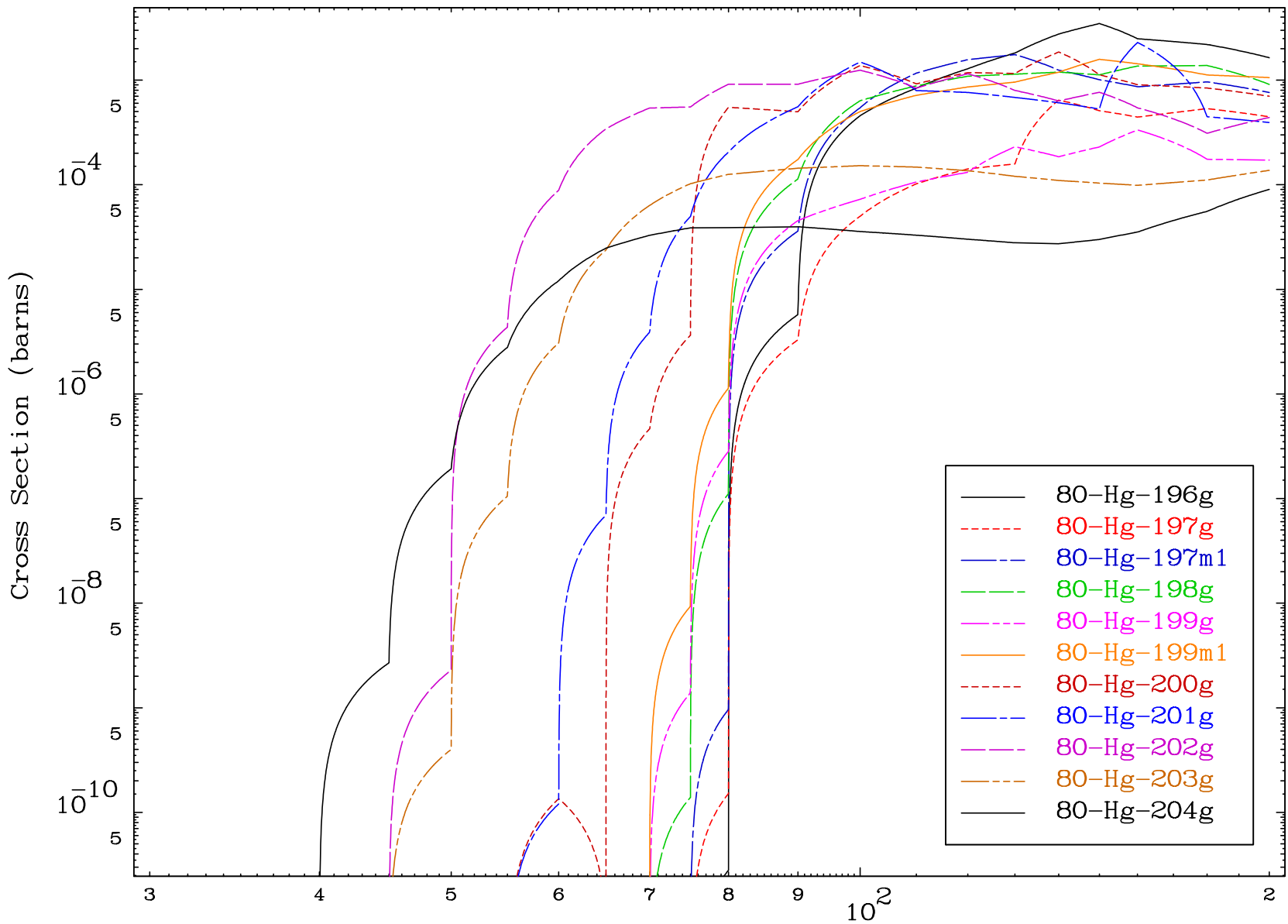
82-Pb-210

MAT 8243

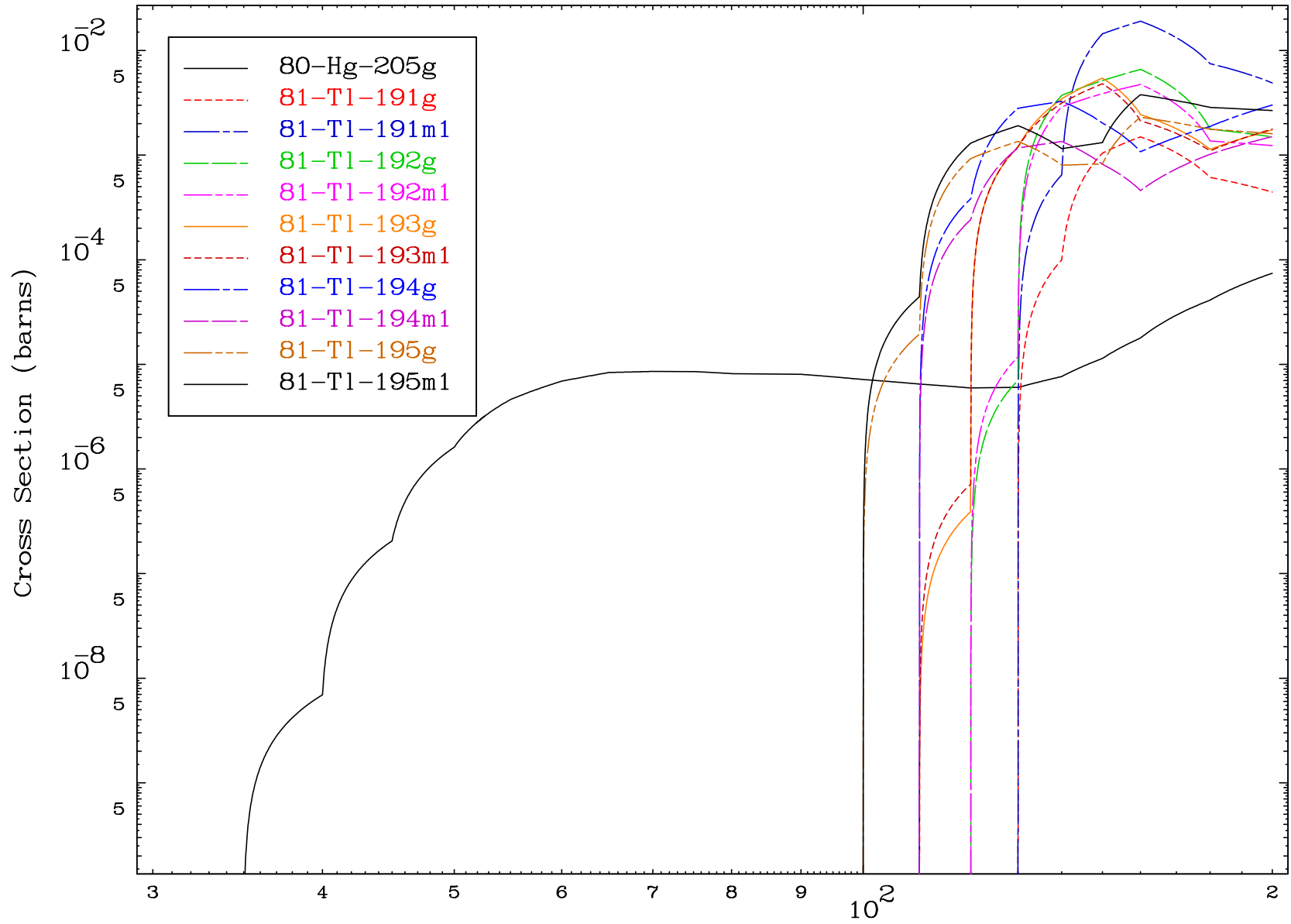
( $\alpha$ , remainder)  
Radionuclide Production Cross Section

82-Pb-210





Radionuclide Production Cross Section

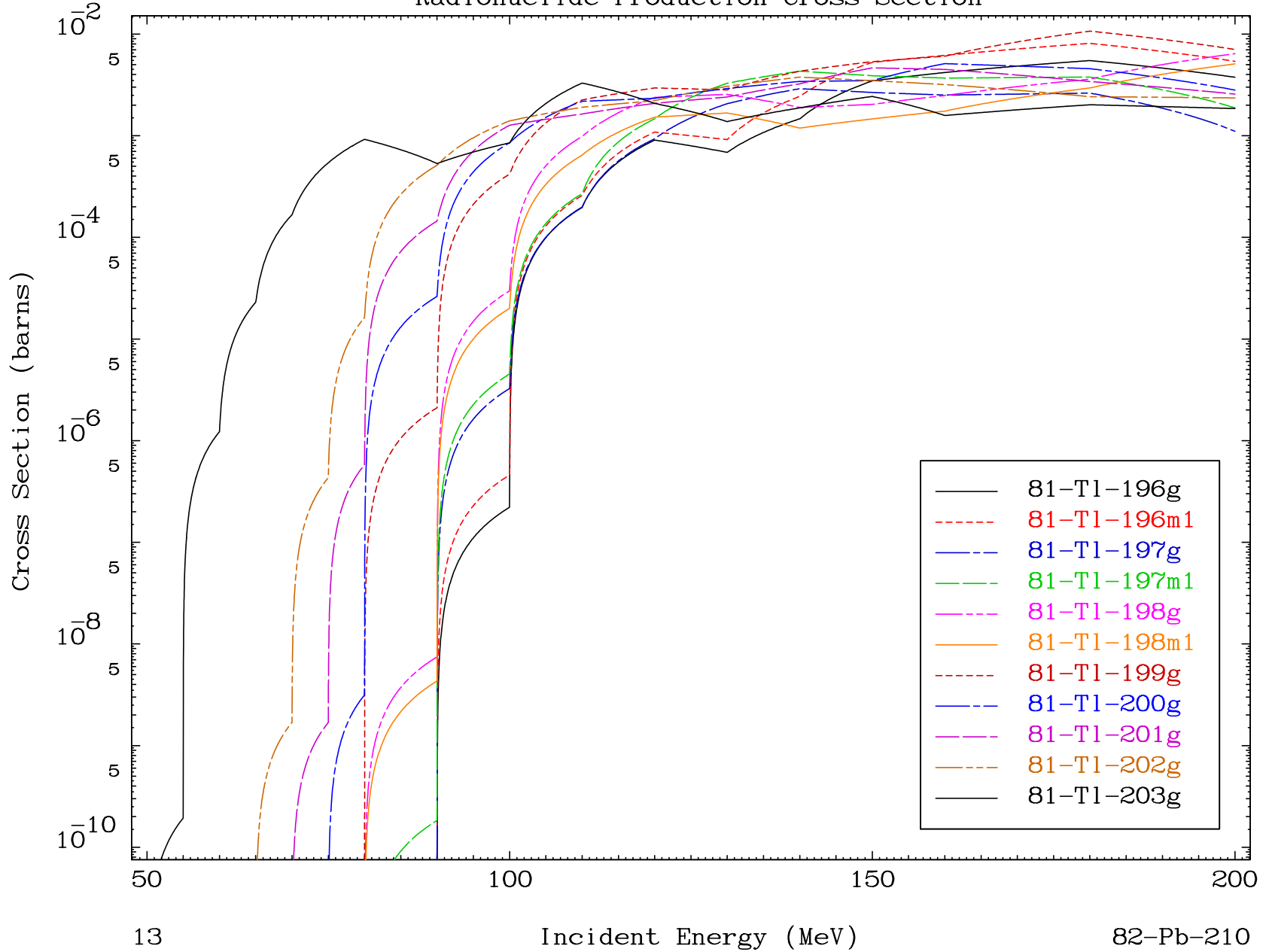


MAT 8243

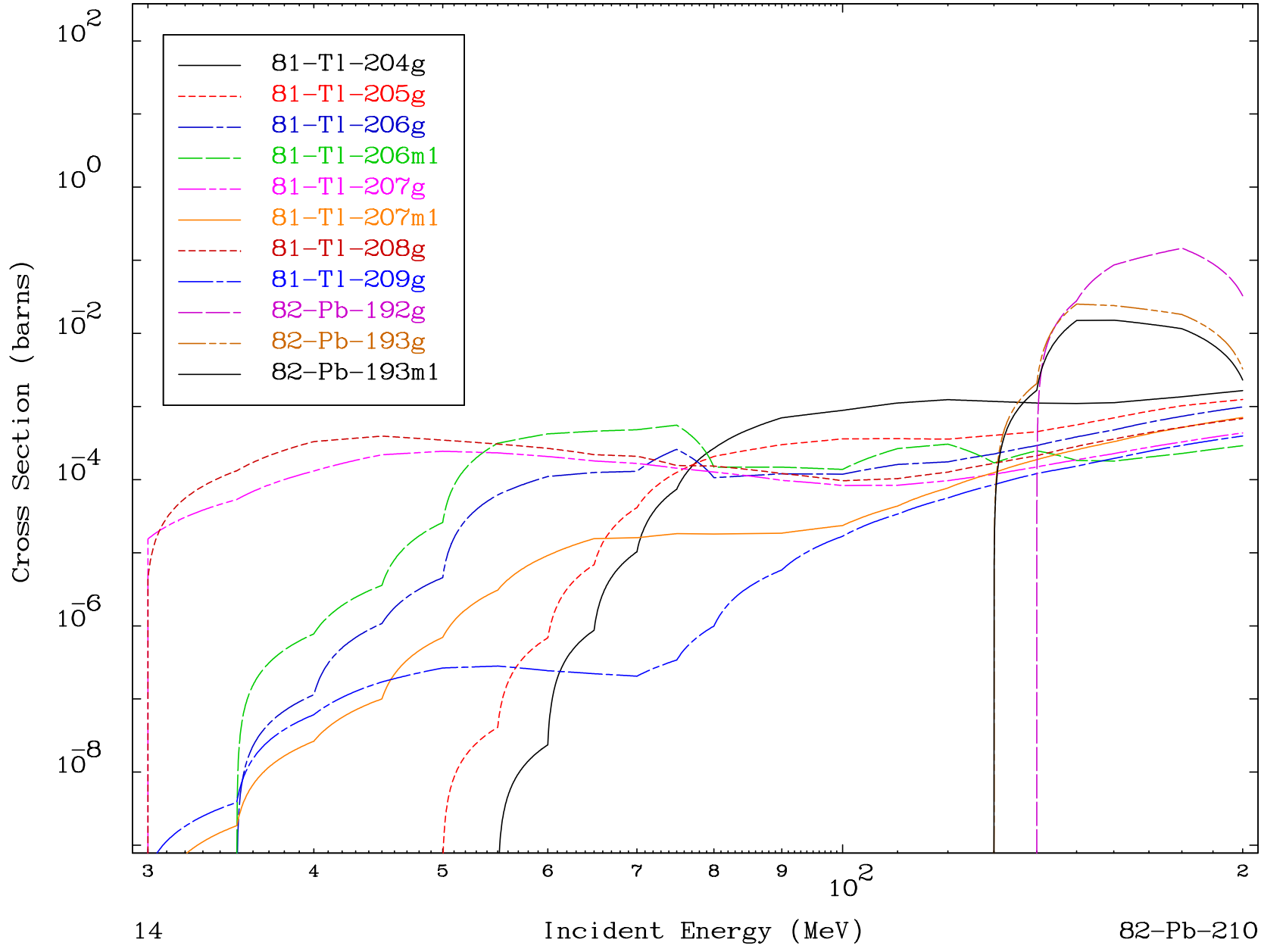
( $\alpha$ , remainder)

82-Pb-210

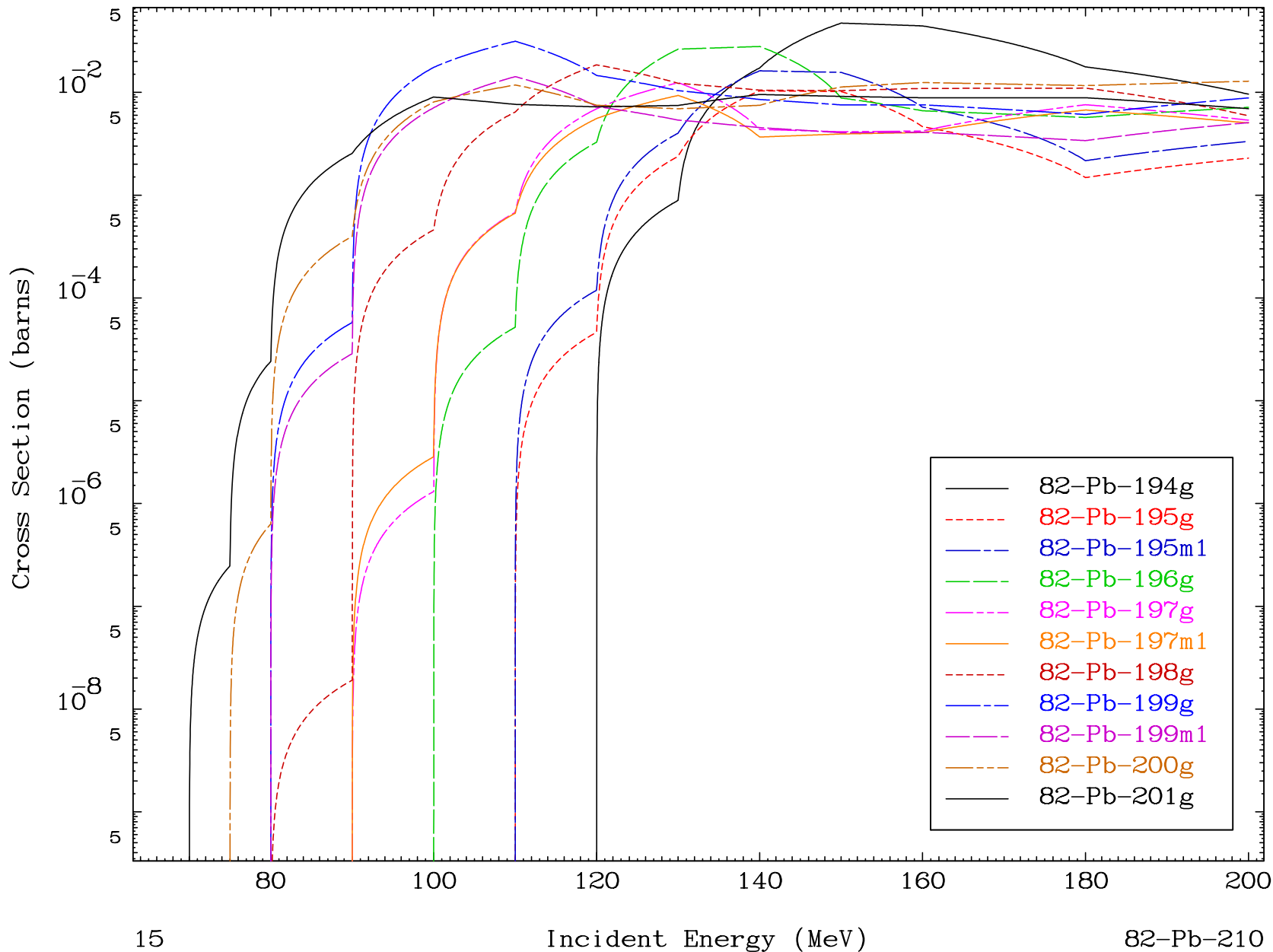
### Radionuclide Production Cross Section

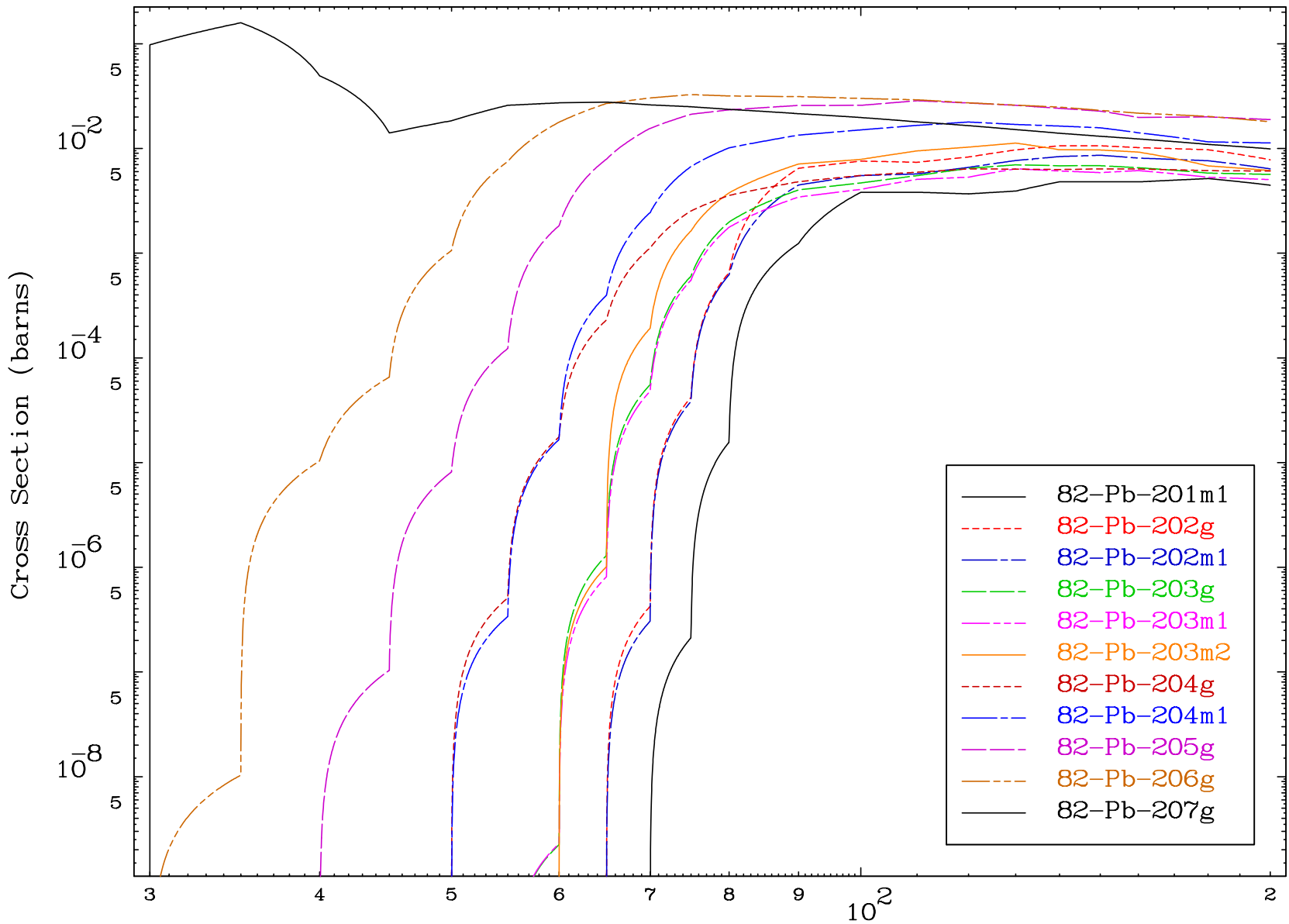


Radionuclide Production Cross Section



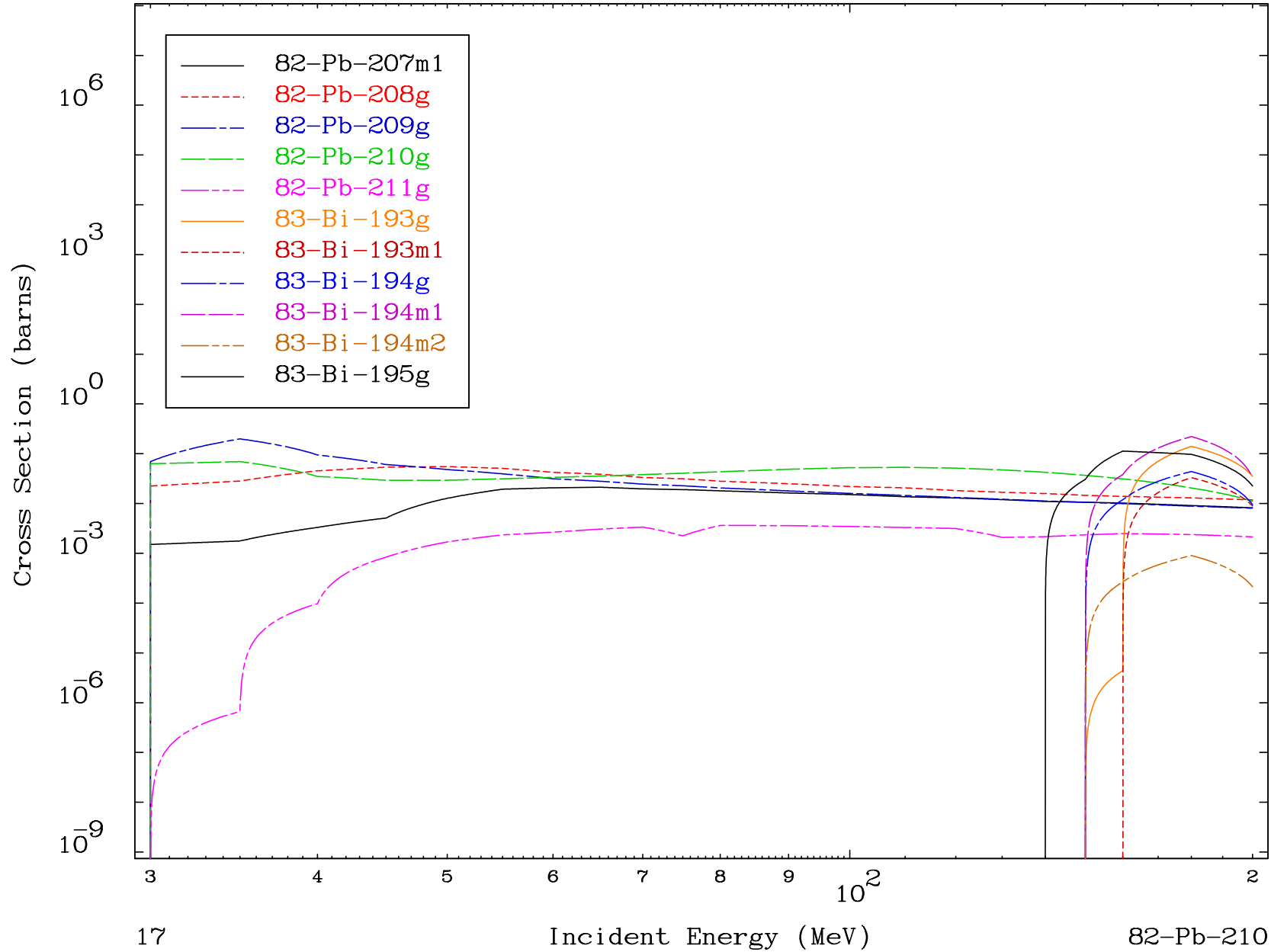
Radionuclide Production Cross Section







Radionuclide Production Cross Section

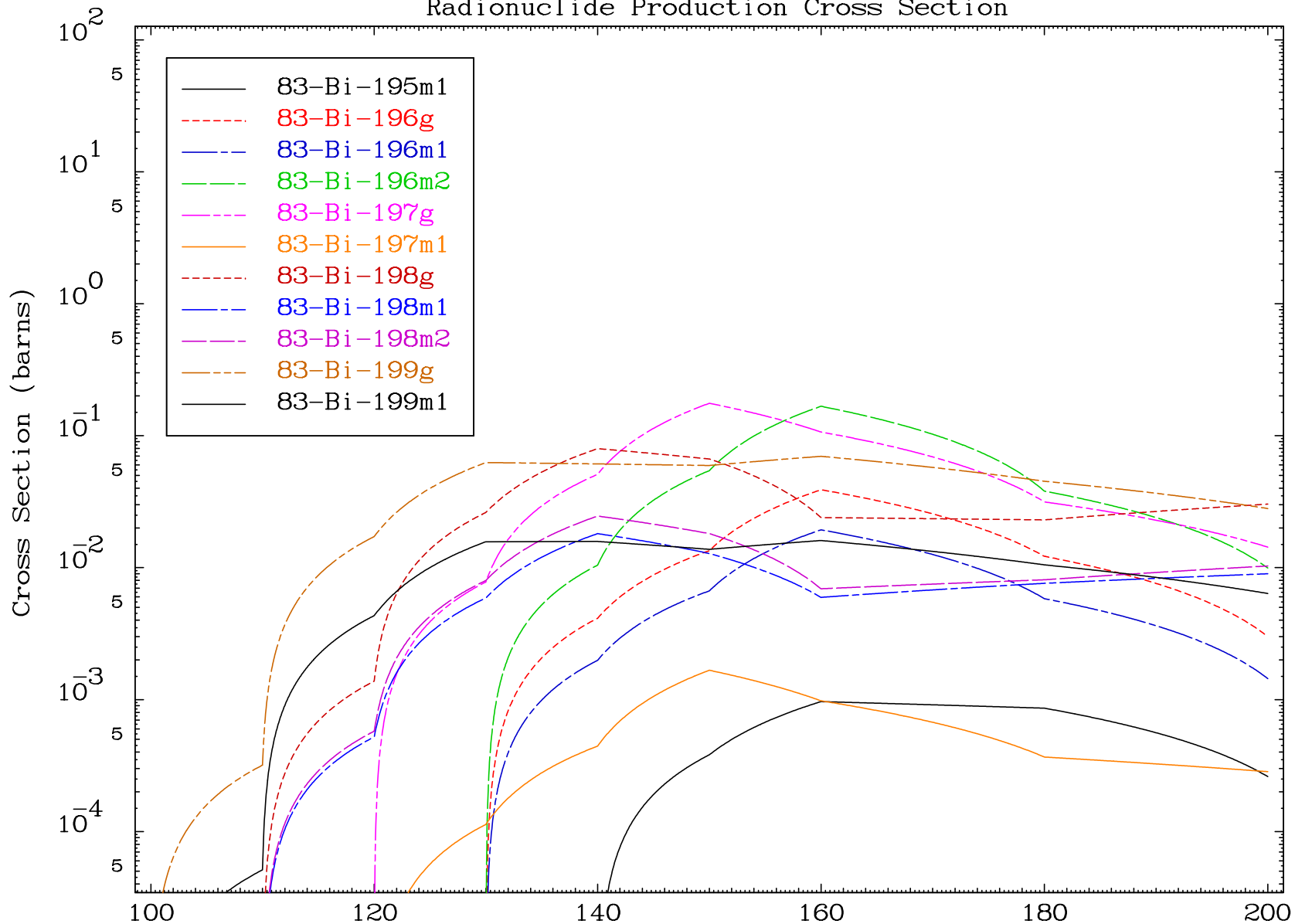


MAT 8243

( $\alpha$ , remainder)

82-Pb-210

### Radionuclide Production Cross Section



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Incident Energy (MeV)

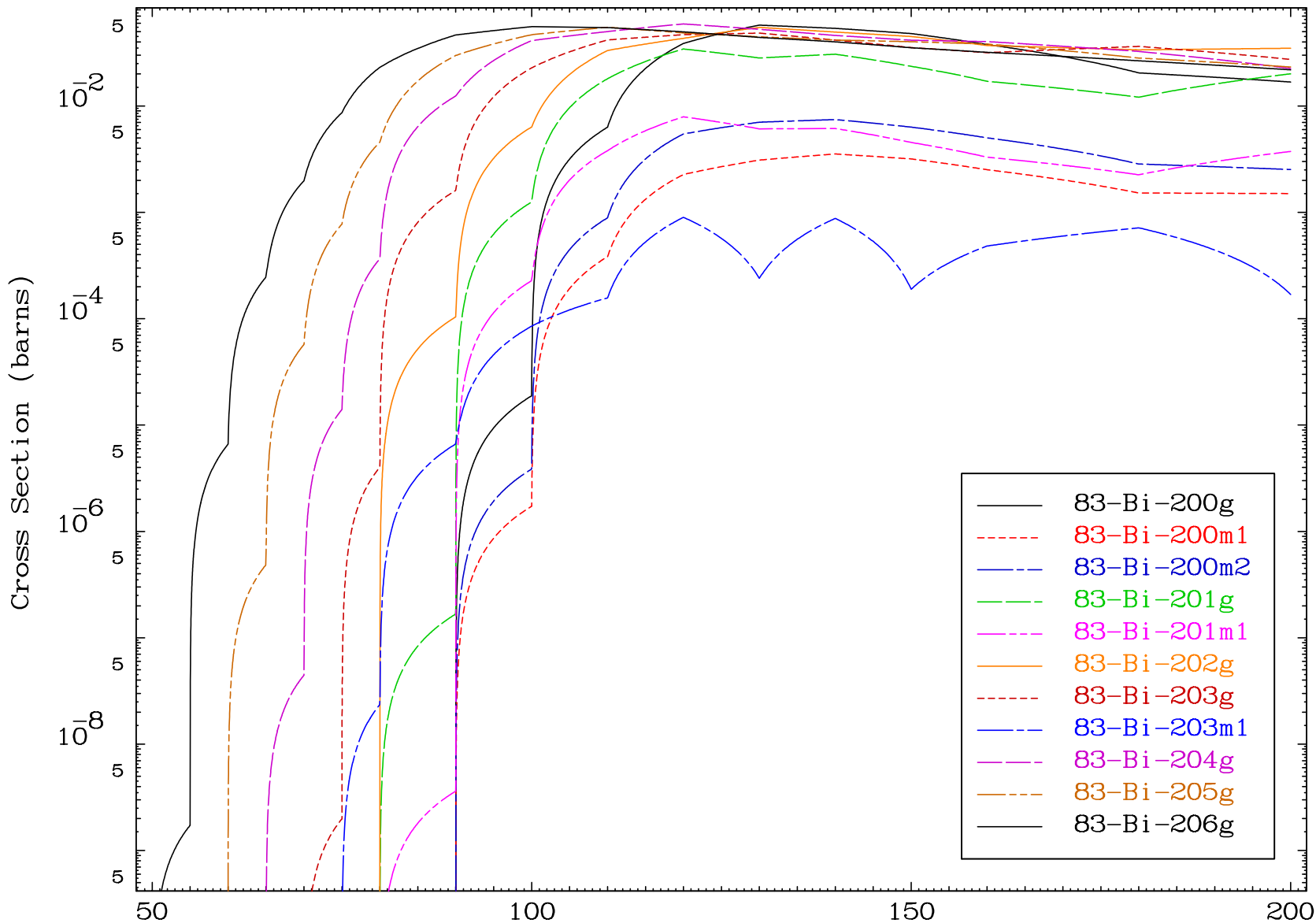
82-Pb-210

MAT 8243

( $\alpha$ , remainder)

82-Pb-210

### Radionuclide Production Cross Section

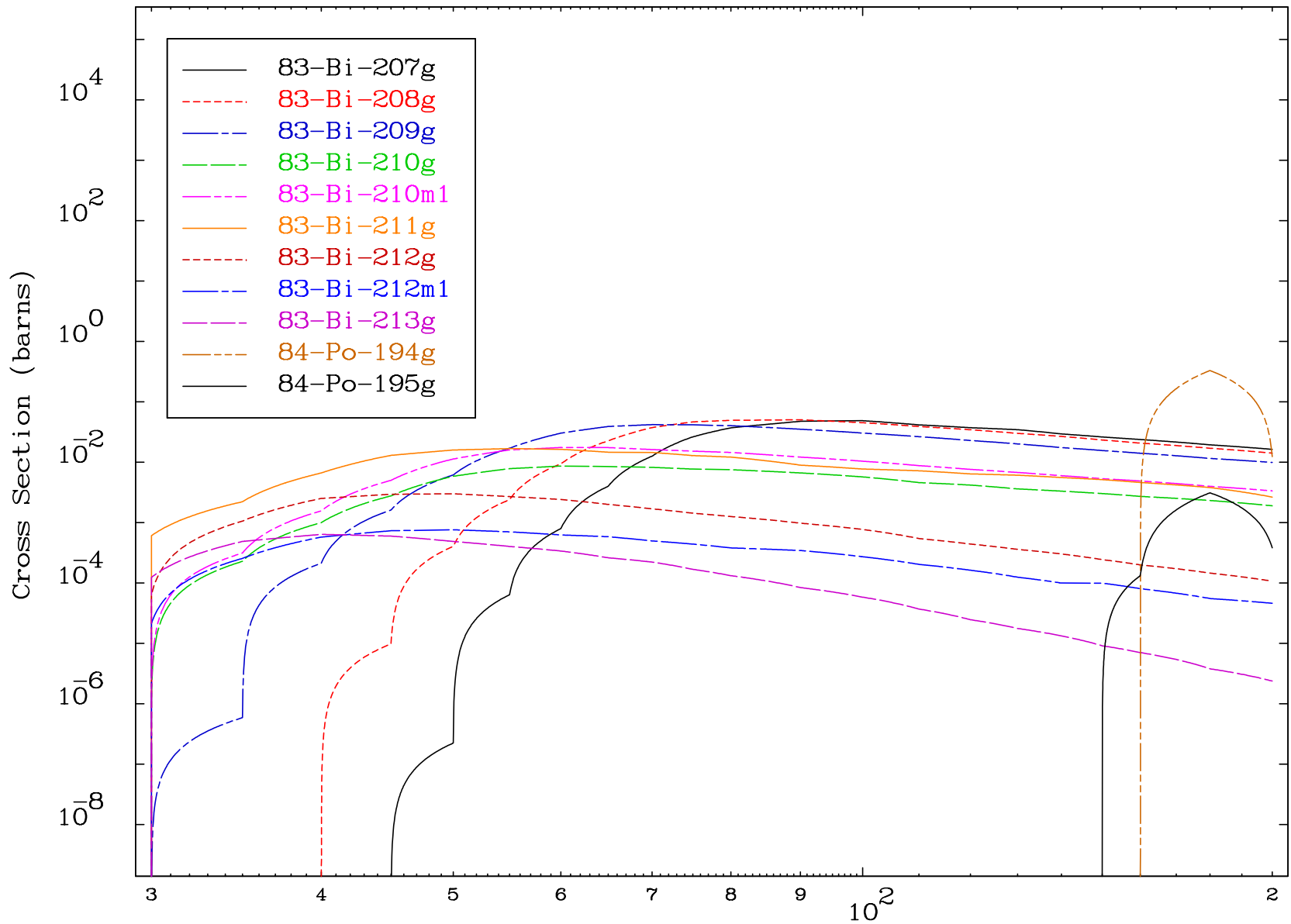


19

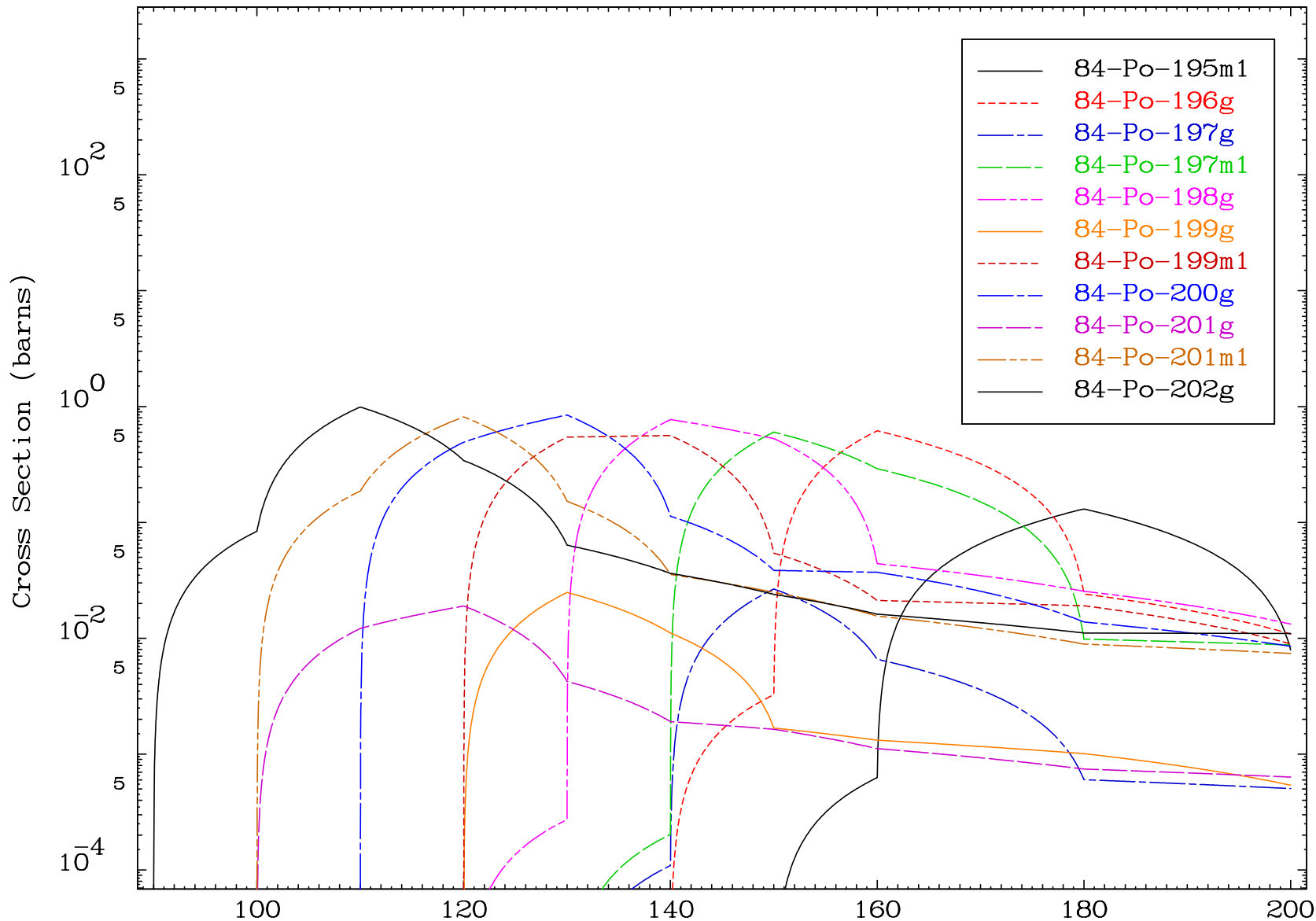
Incident Energy (MeV)

82-Pb-210

Radionuclide Production Cross Section



Radionuclide Production Cross Section

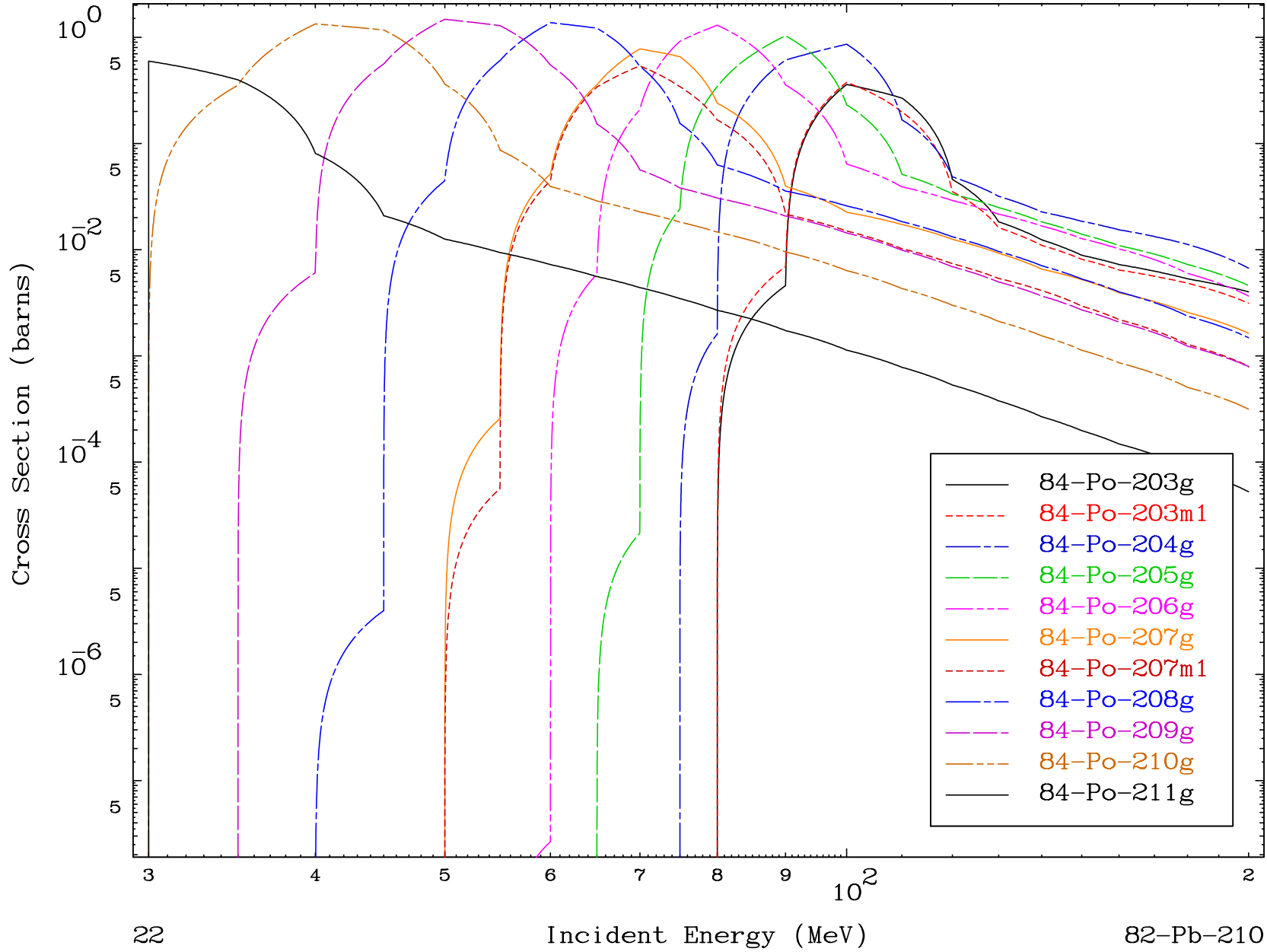


MAT 8243

( $\alpha$ , remainder)

82-Pb-210

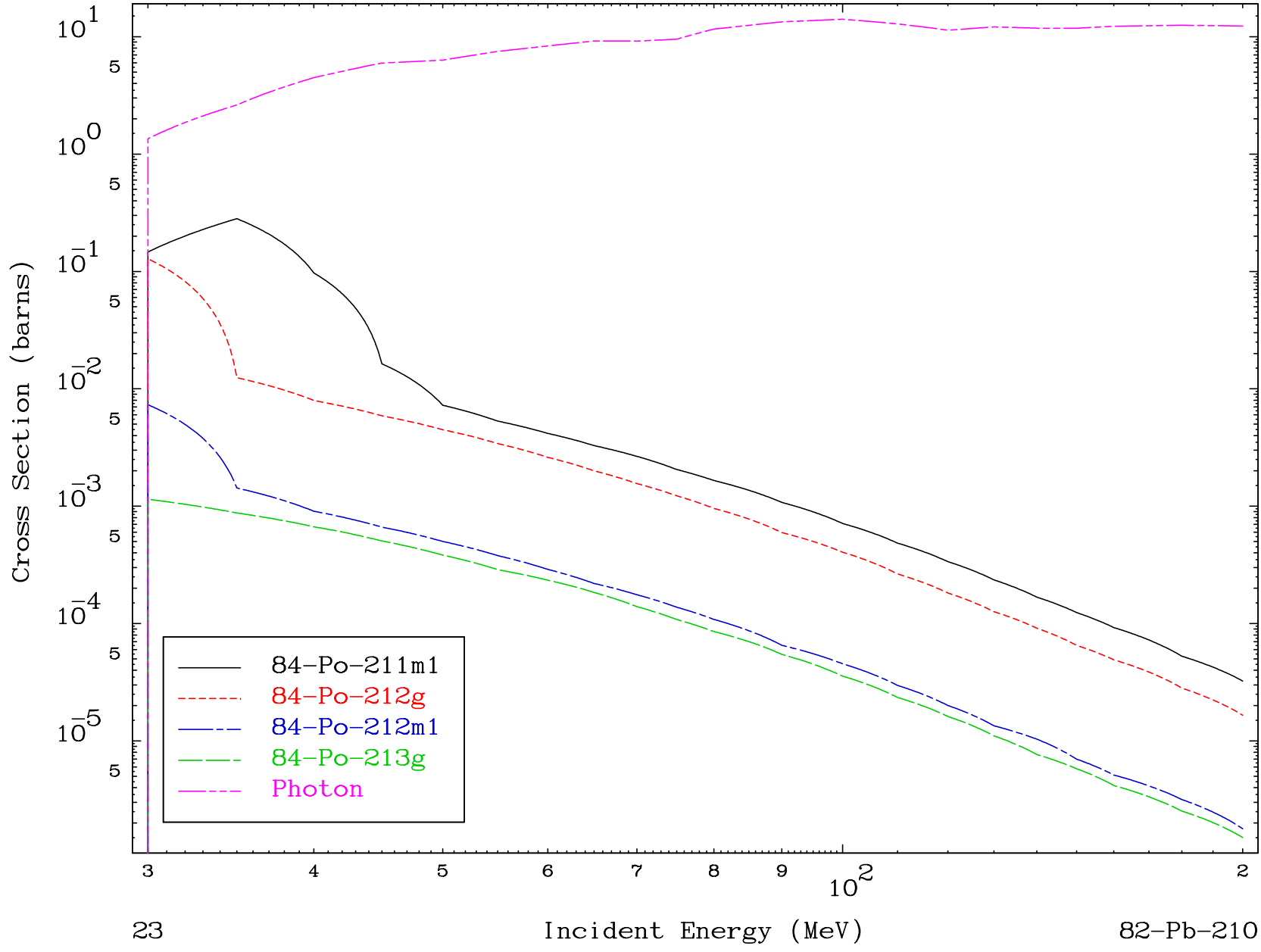
### Radionuclide Production Cross Section



MAT 8243

( $\alpha$ , remainder)  
Radionuclide Production Cross Section

82-Pb-210



23

Incident Energy (MeV)

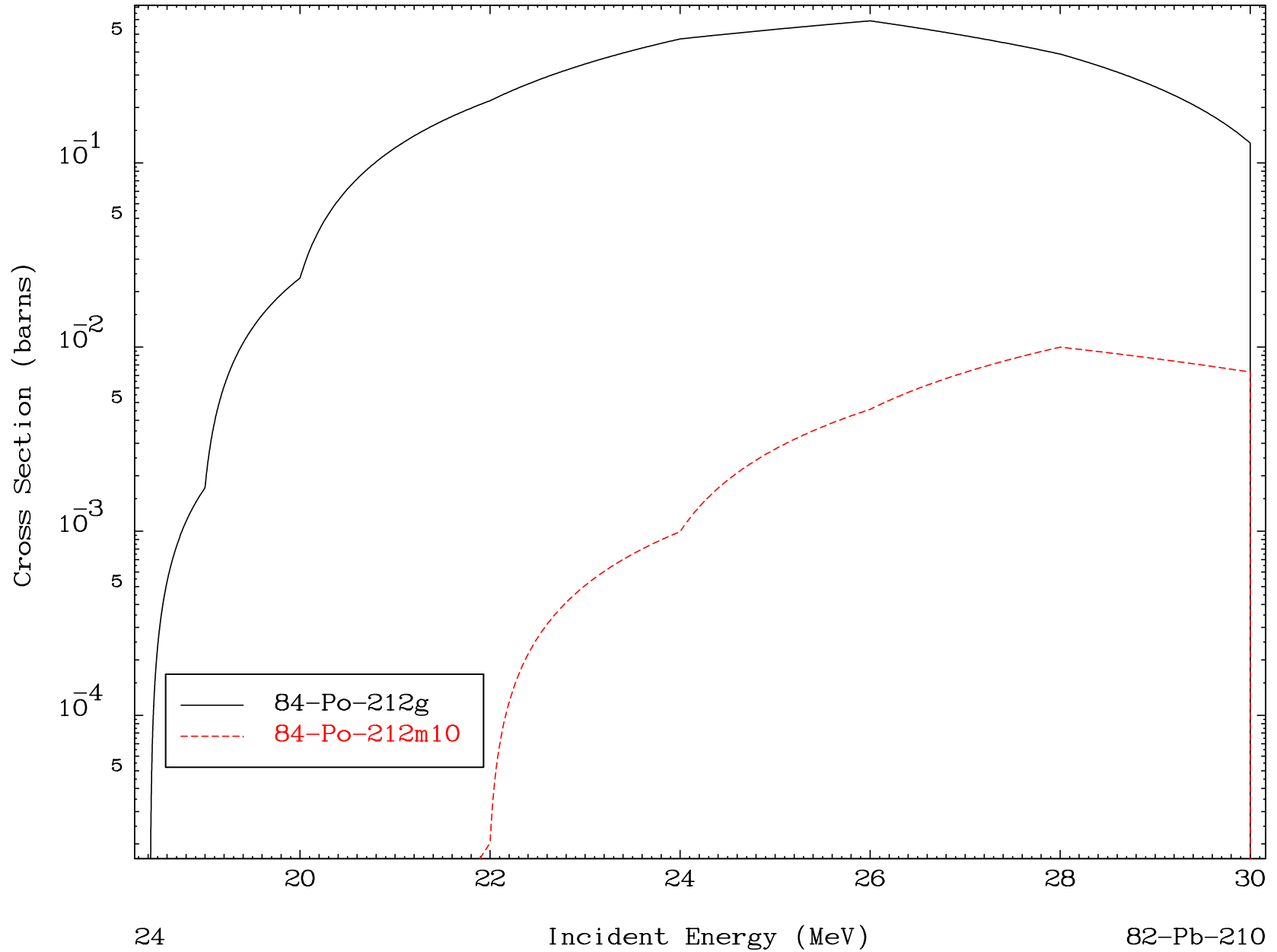
82-Pb-210

MAT 8243

( $\alpha, 2n$ )

82-Pb-210

Radionuclide Production Cross Section



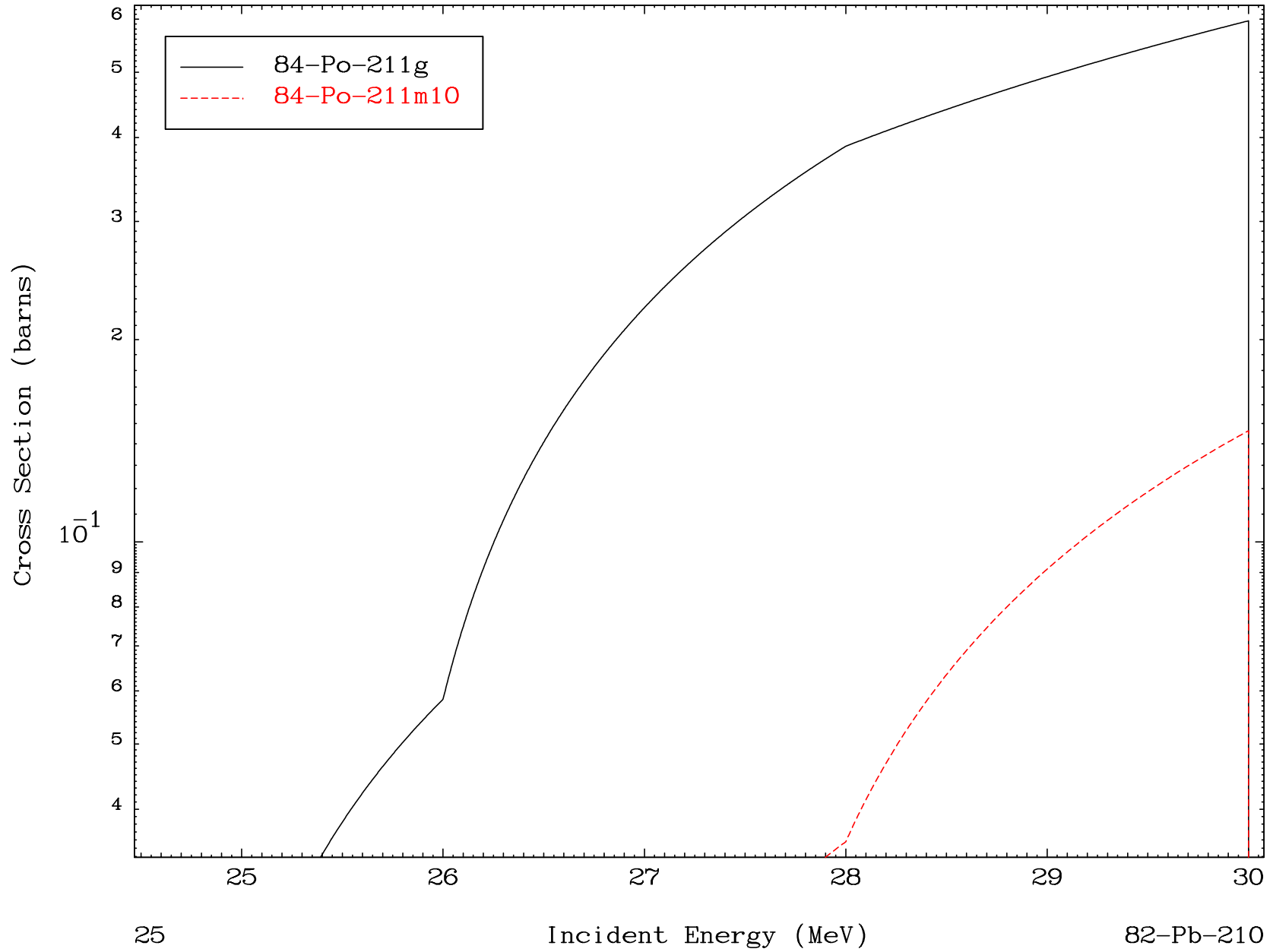


MAT 8243

( $\alpha, 3n$ )

82-Pb-210

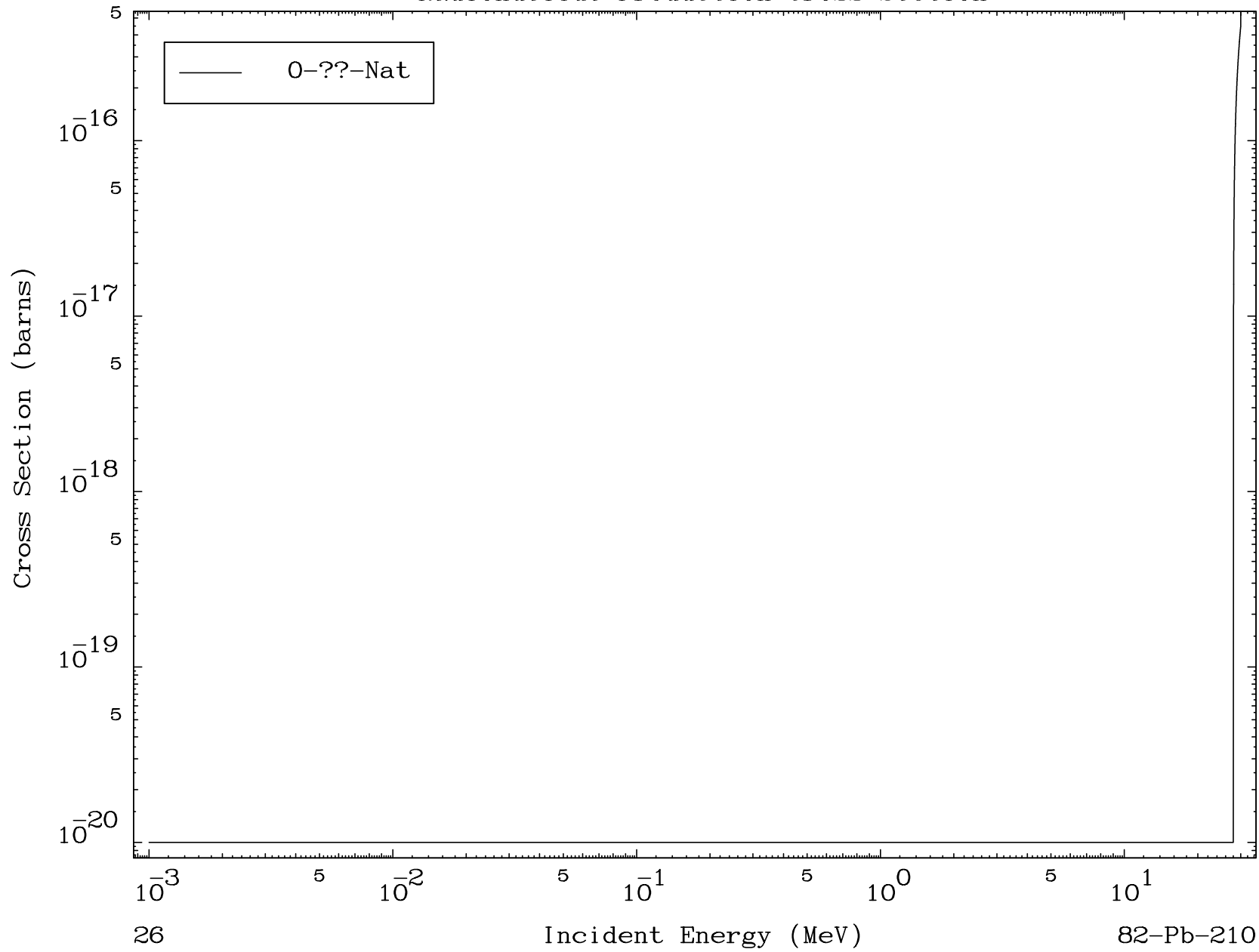
Radionuclide Production Cross Section



MAT 8243

$\alpha$  Fission  
Radionuclide Production Cross Section

82-Pb-210

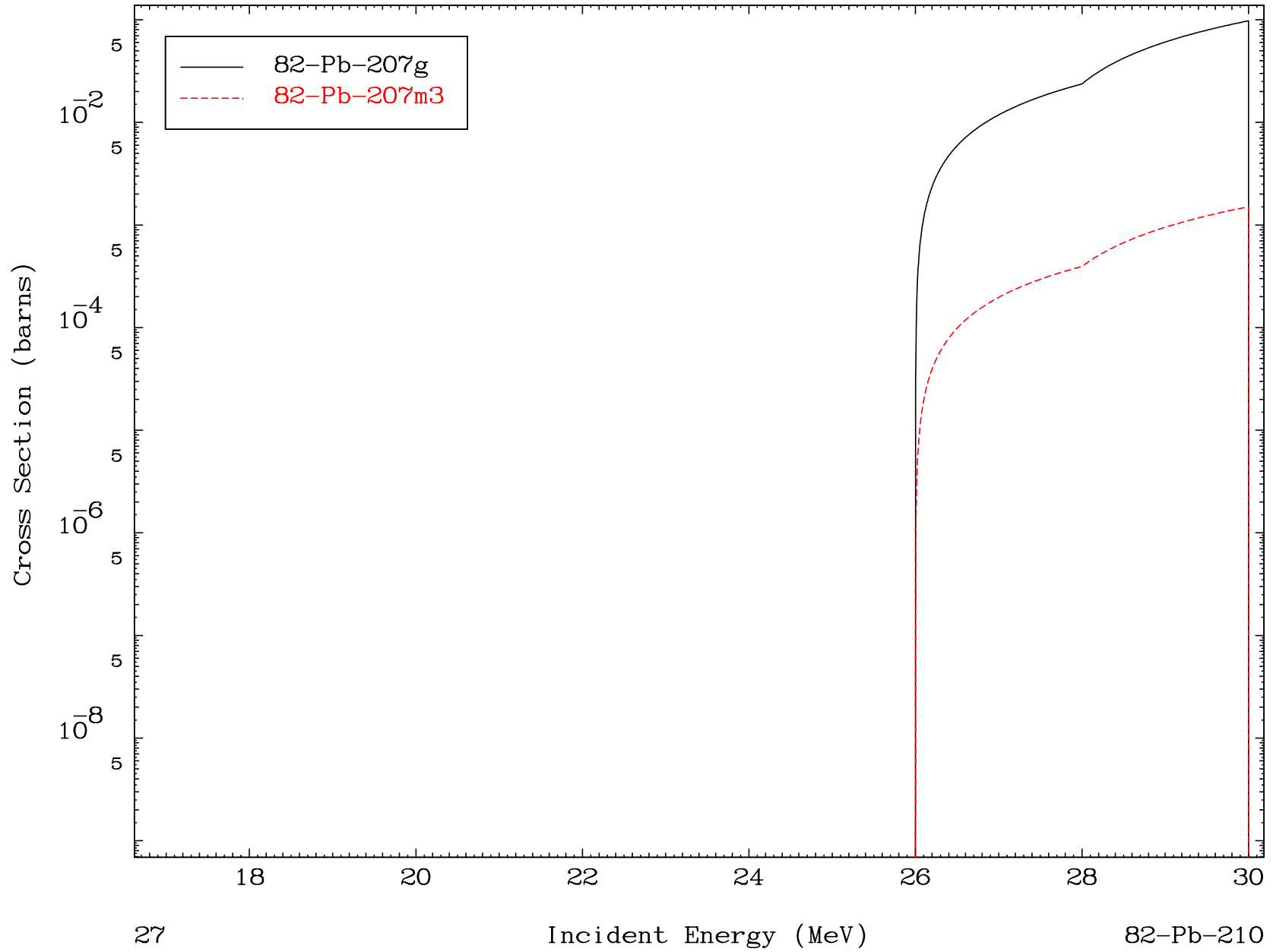


MAT 8243

$(\alpha, 3n) \alpha$

82-Pb-210

Radionuclide Production Cross Section

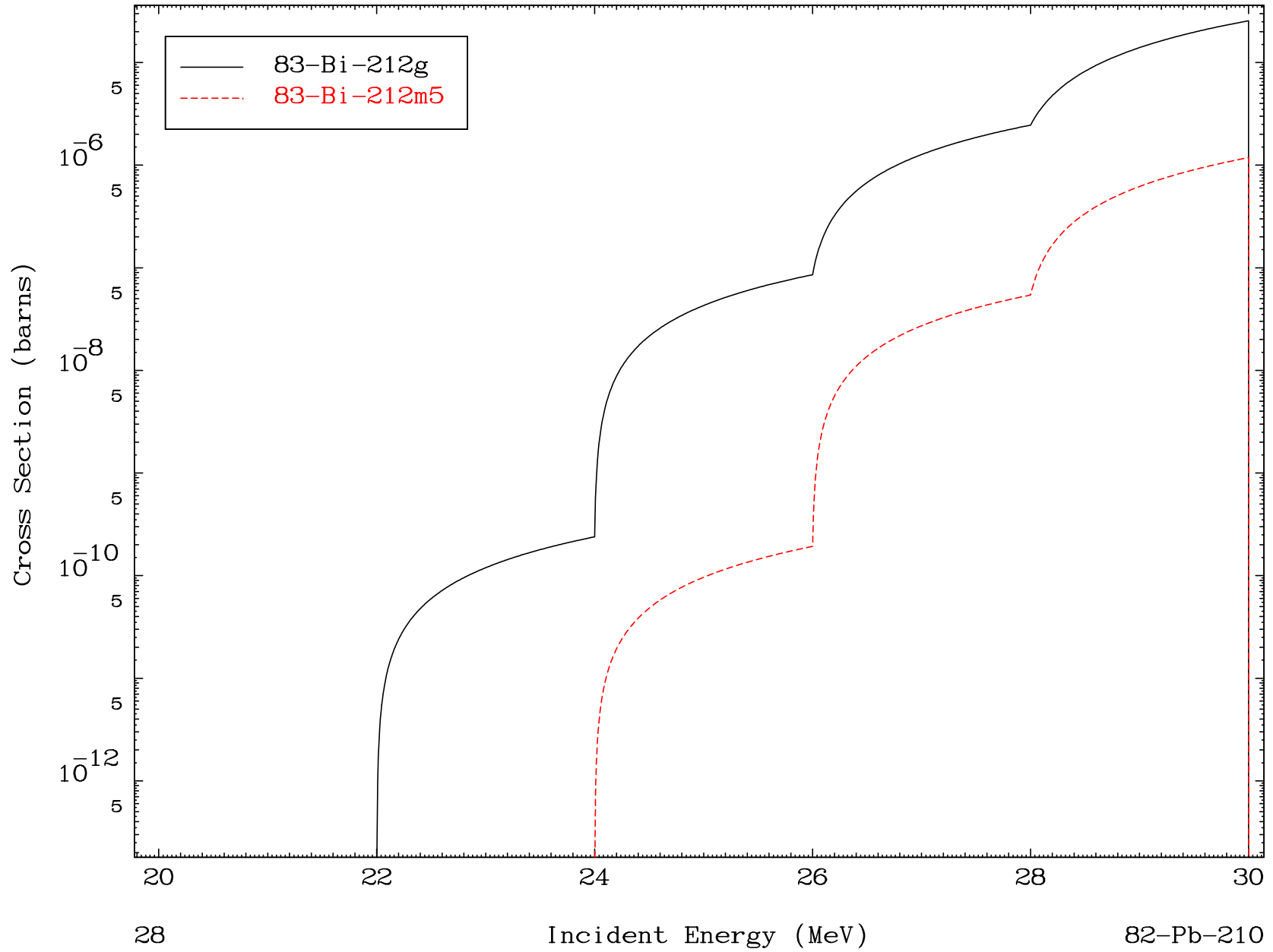


MAT 8243

( $\alpha, n'$ ) p

82-Pb-210

Radionuclide Production Cross Section

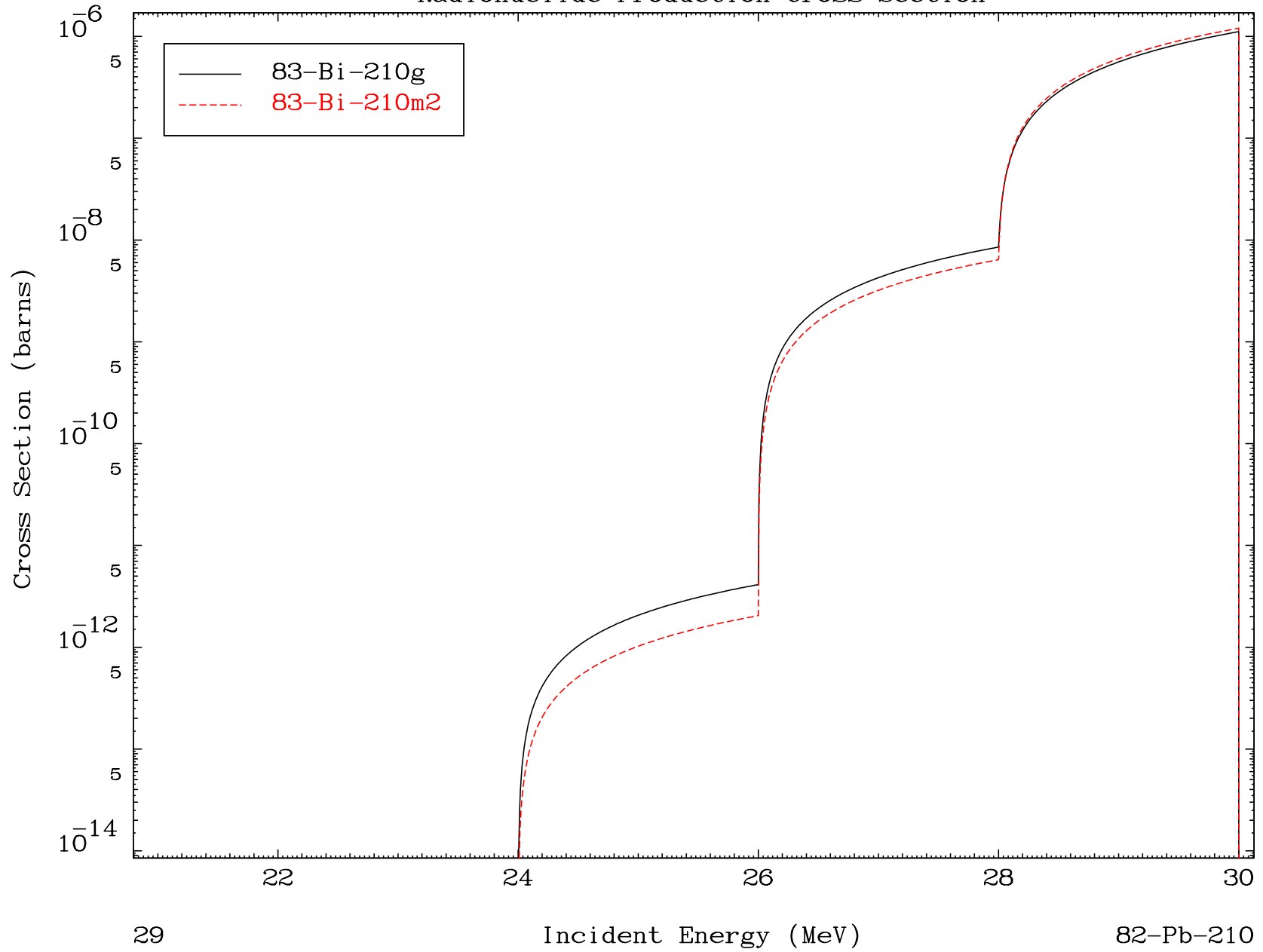


MAT 8243

( $\alpha, n'$ ) t

82-Pb-210

Radionuclide Production Cross Section

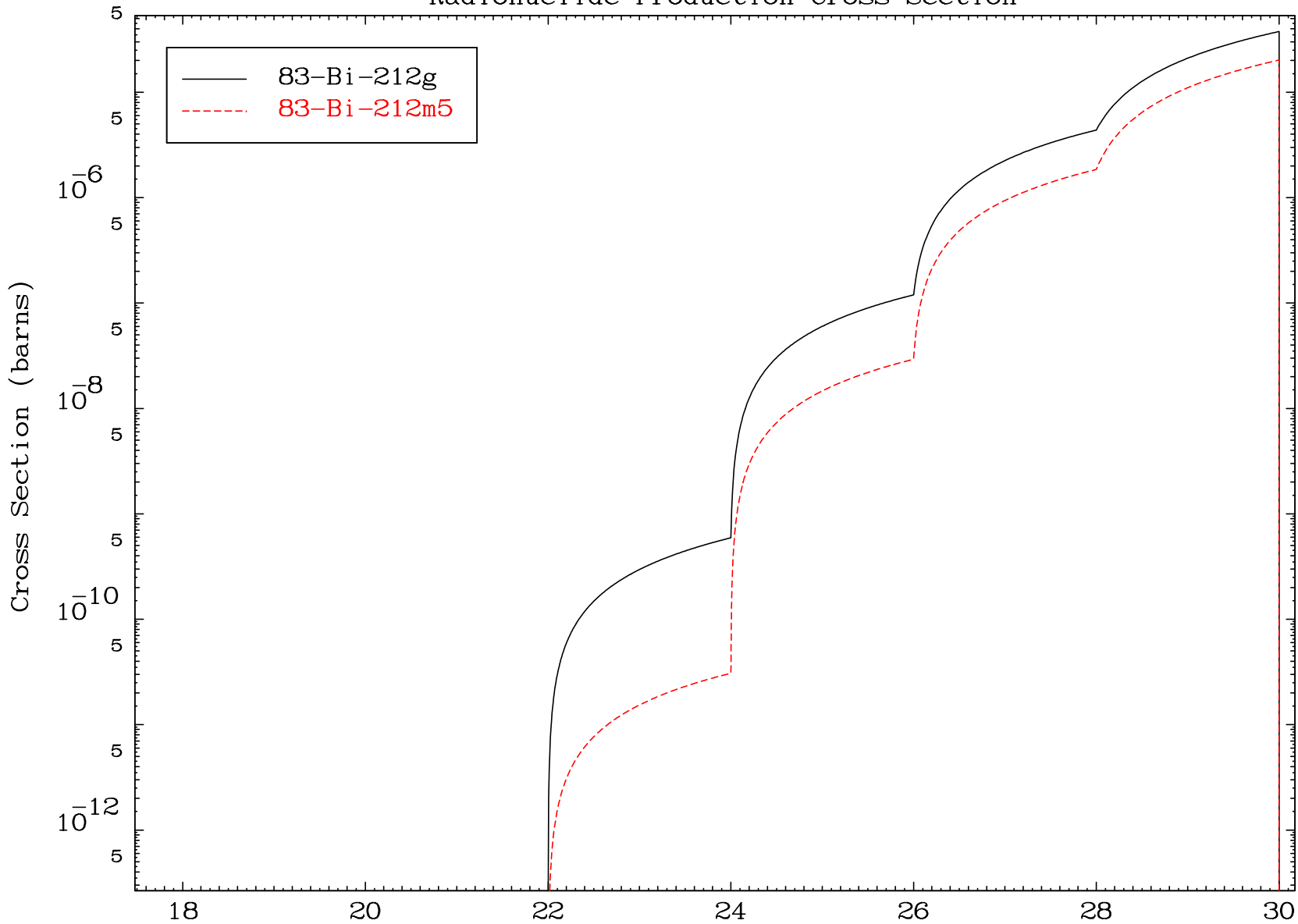


MAT 8243

( $\alpha, d$ )

82-Pb-210

Radionuclide Production Cross Section



30

Incident Energy (MeV)

82-Pb-210