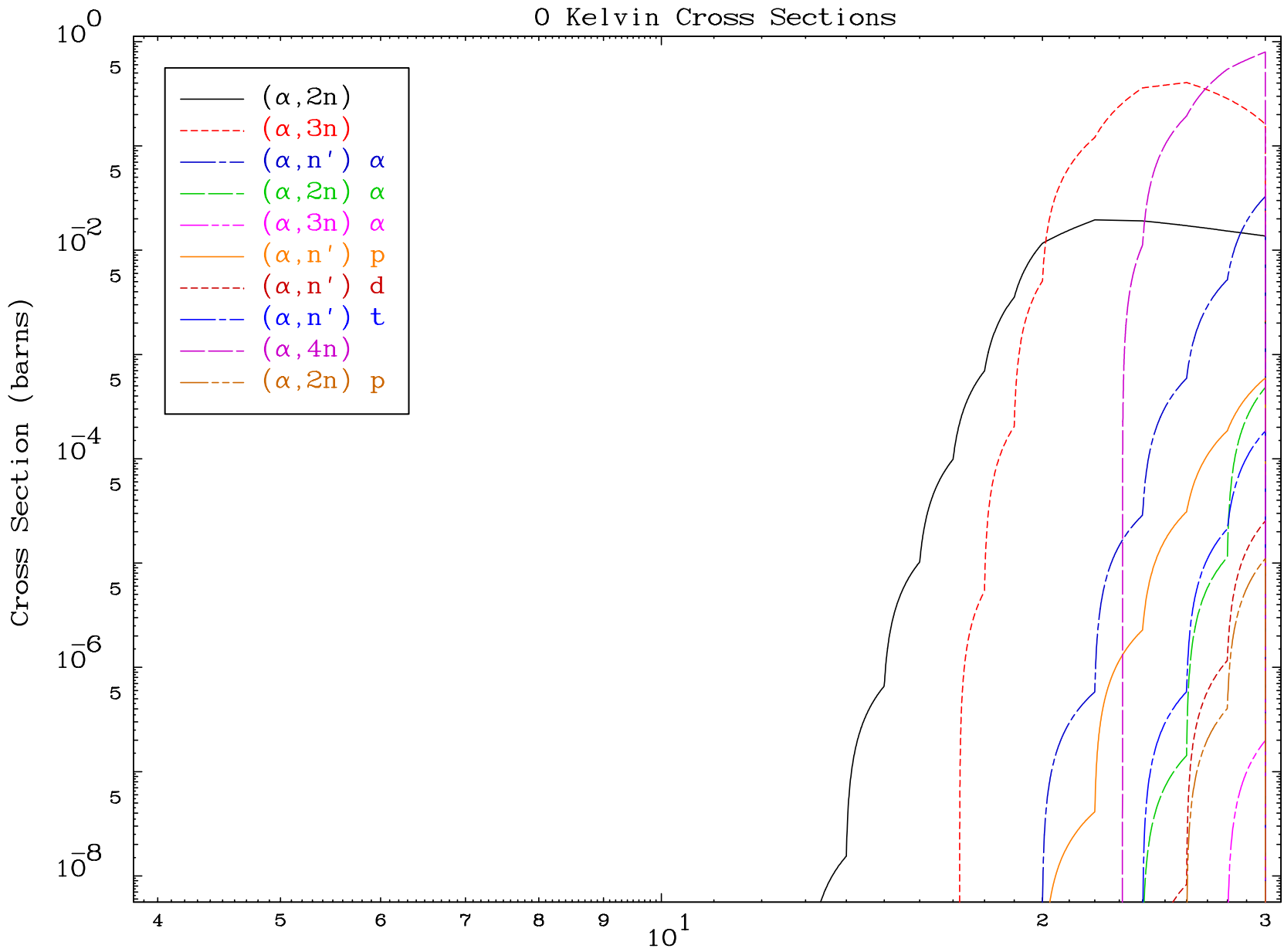


MAT 8861

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

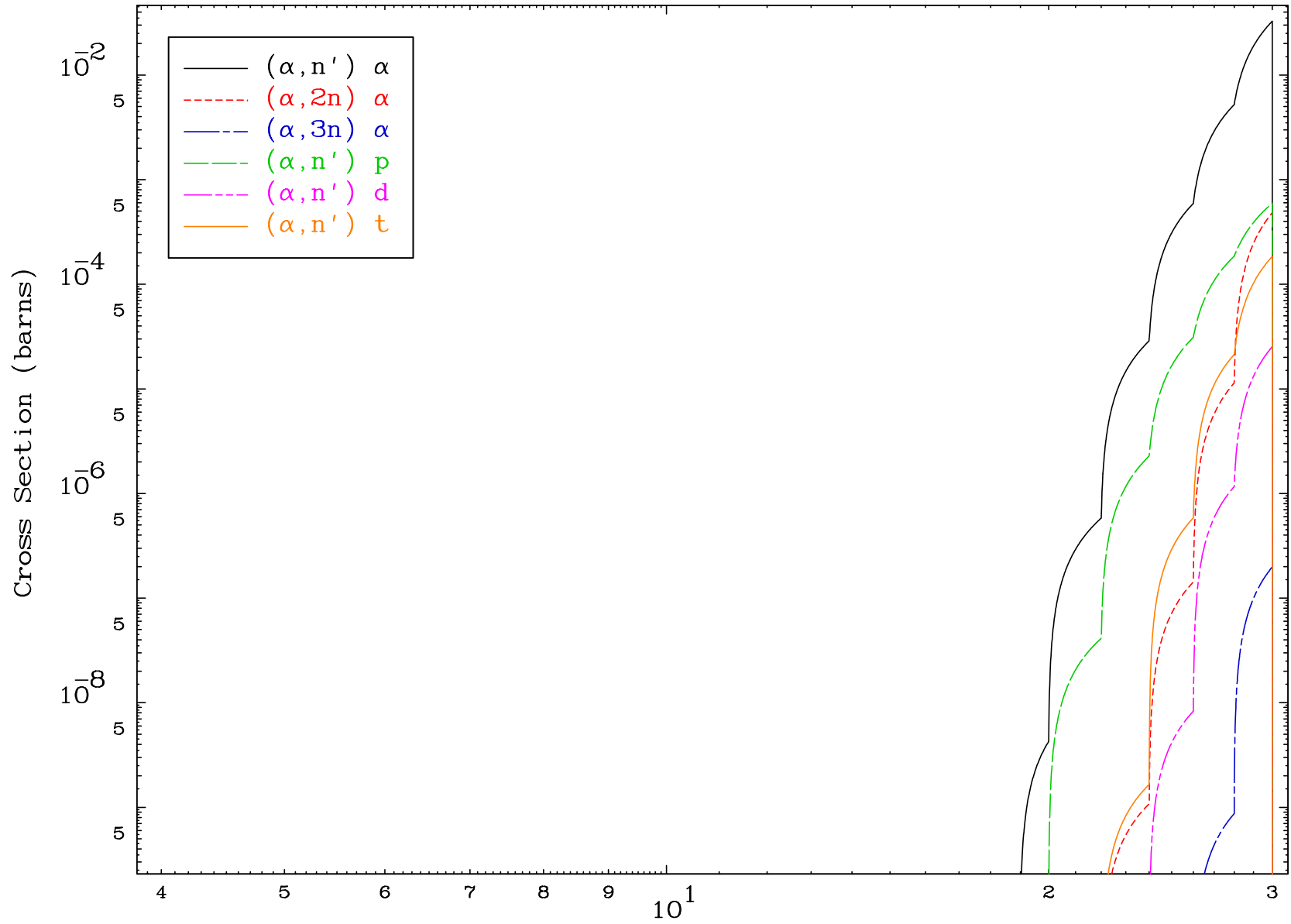
88-Ra-235

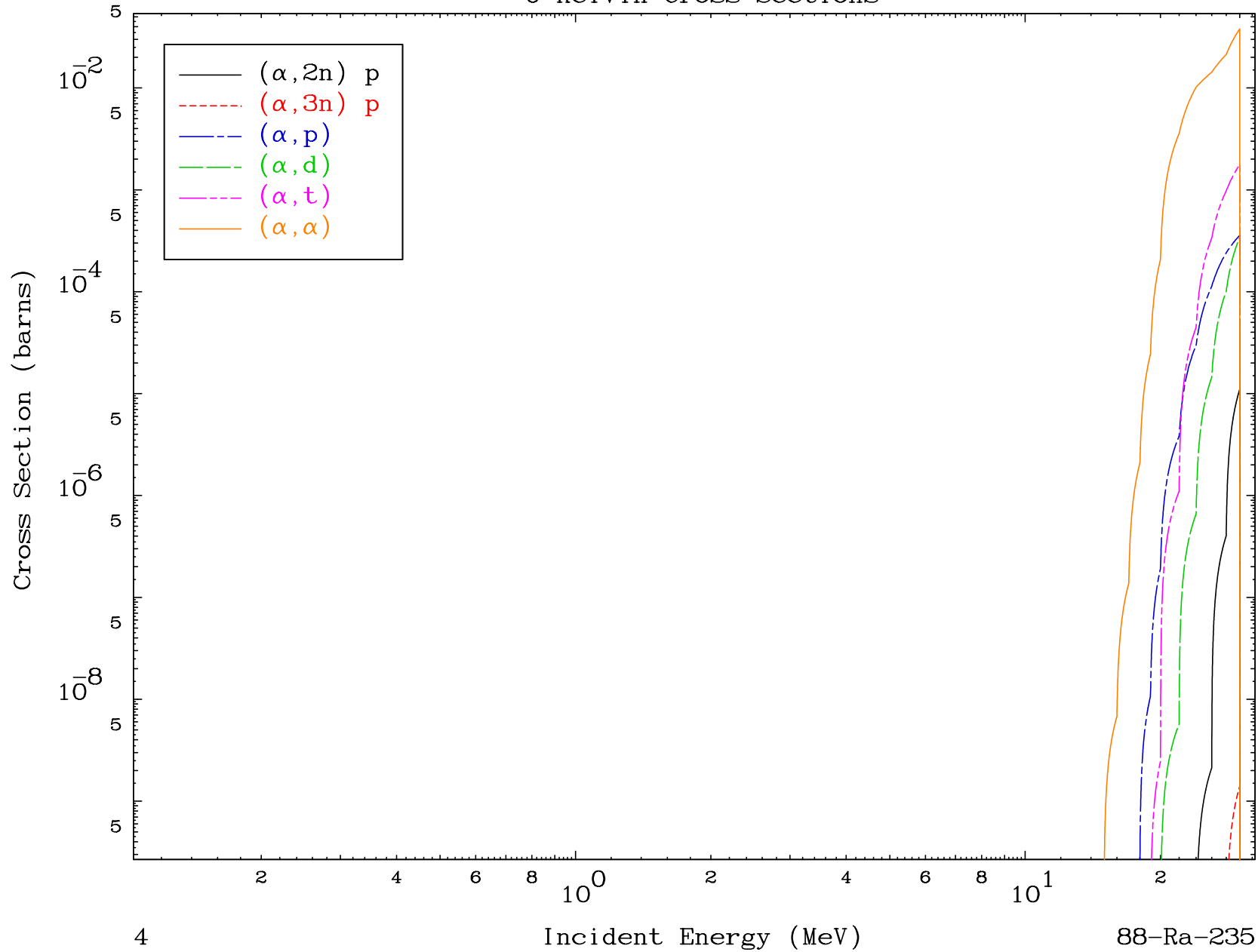


2

Incident Energy (MeV)

88-Ra-235

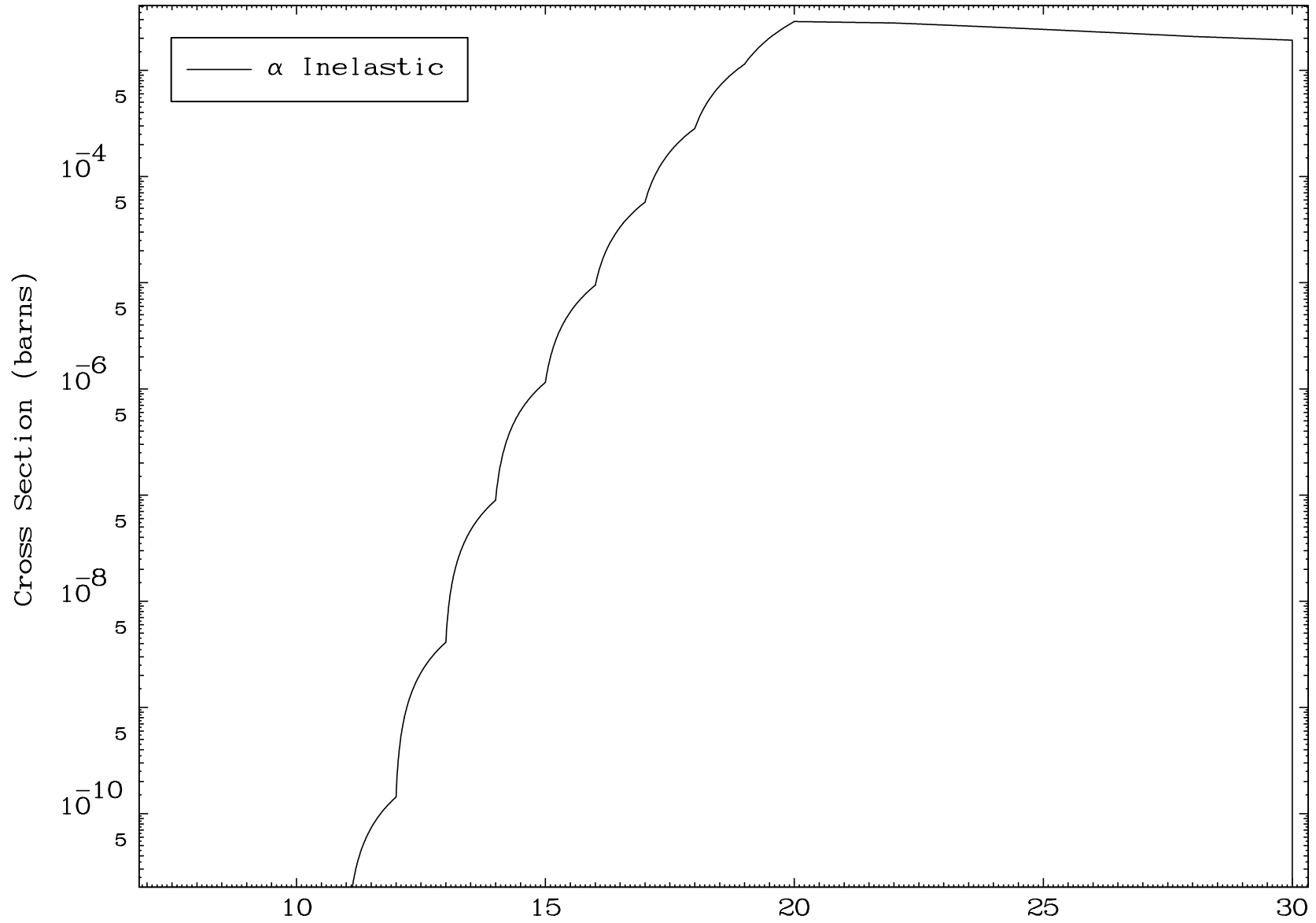




MAT 8861

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

88-Ra-235



5

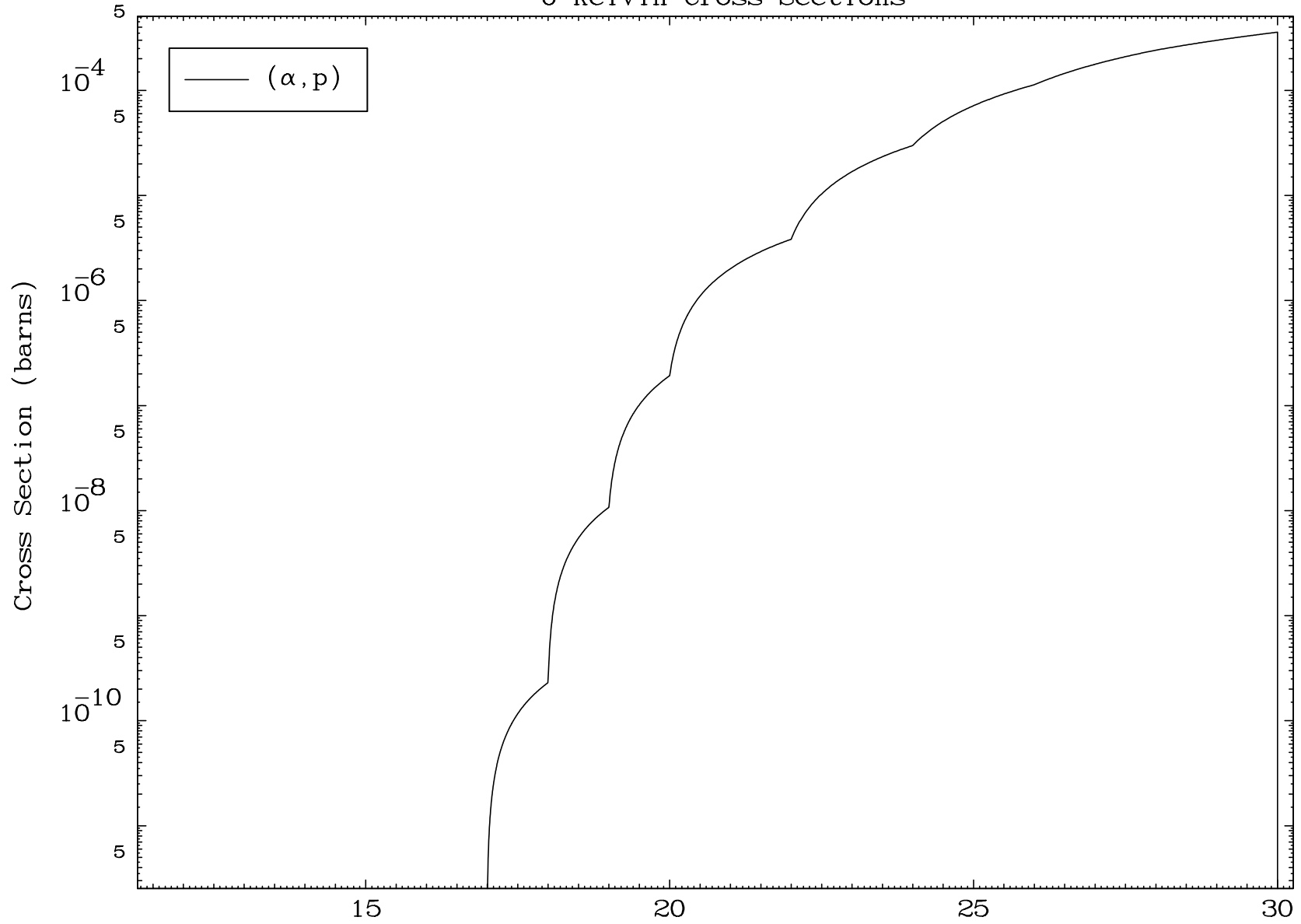
Incident Energy (MeV)

88-Ra-235

MAT 8861

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections

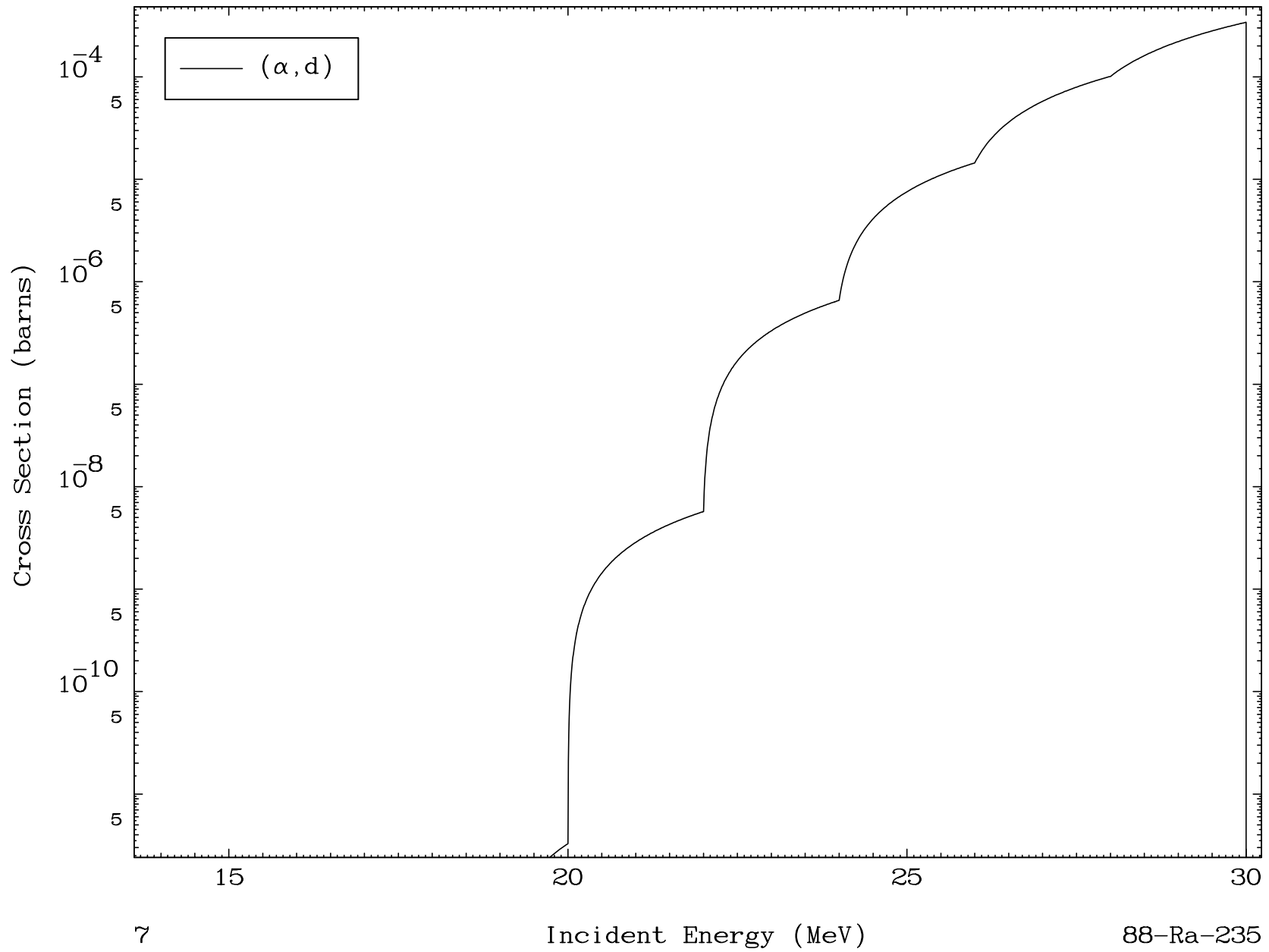
88-Ra-235

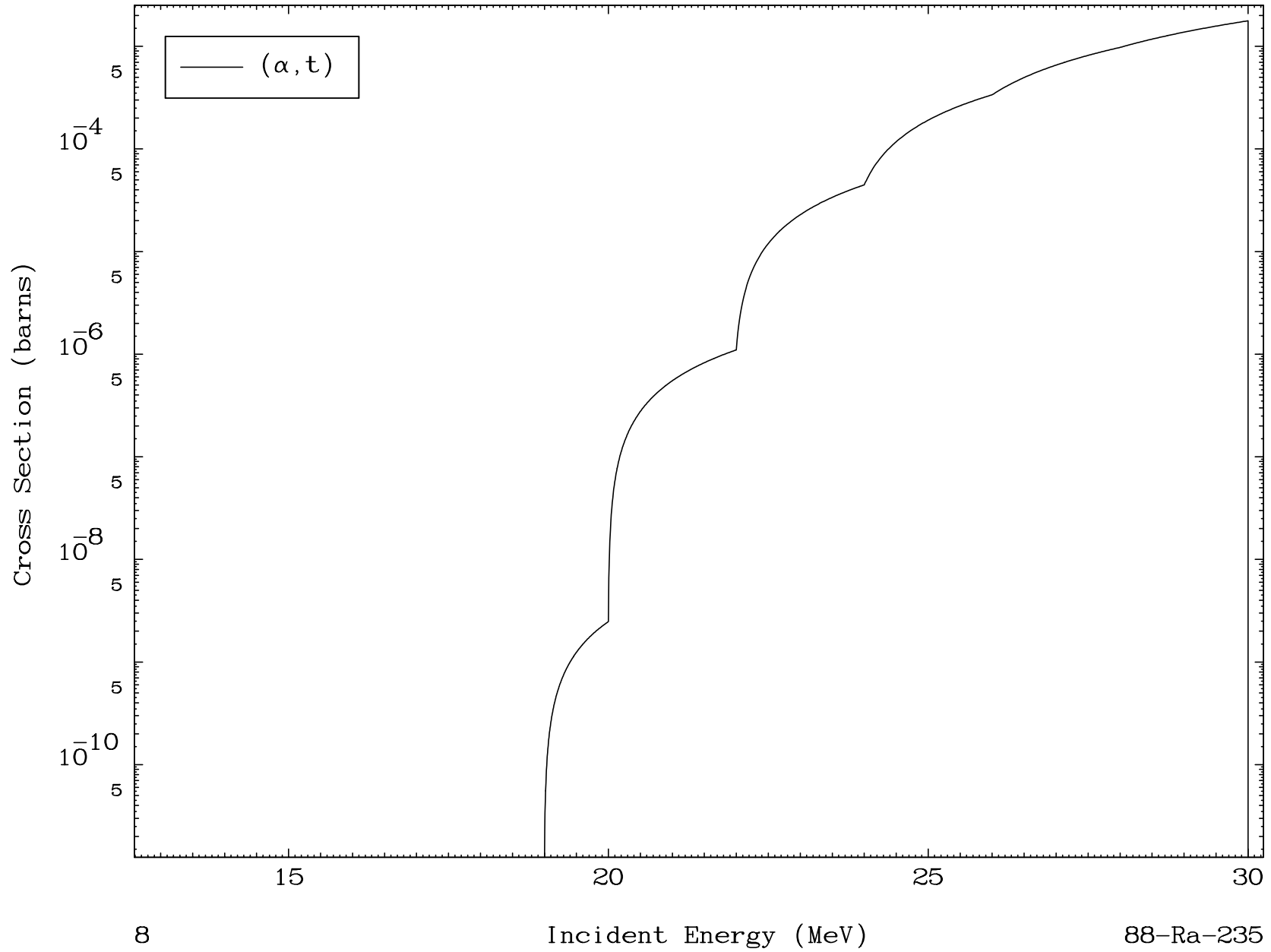


6

Incident Energy (MeV)

88-Ra-235



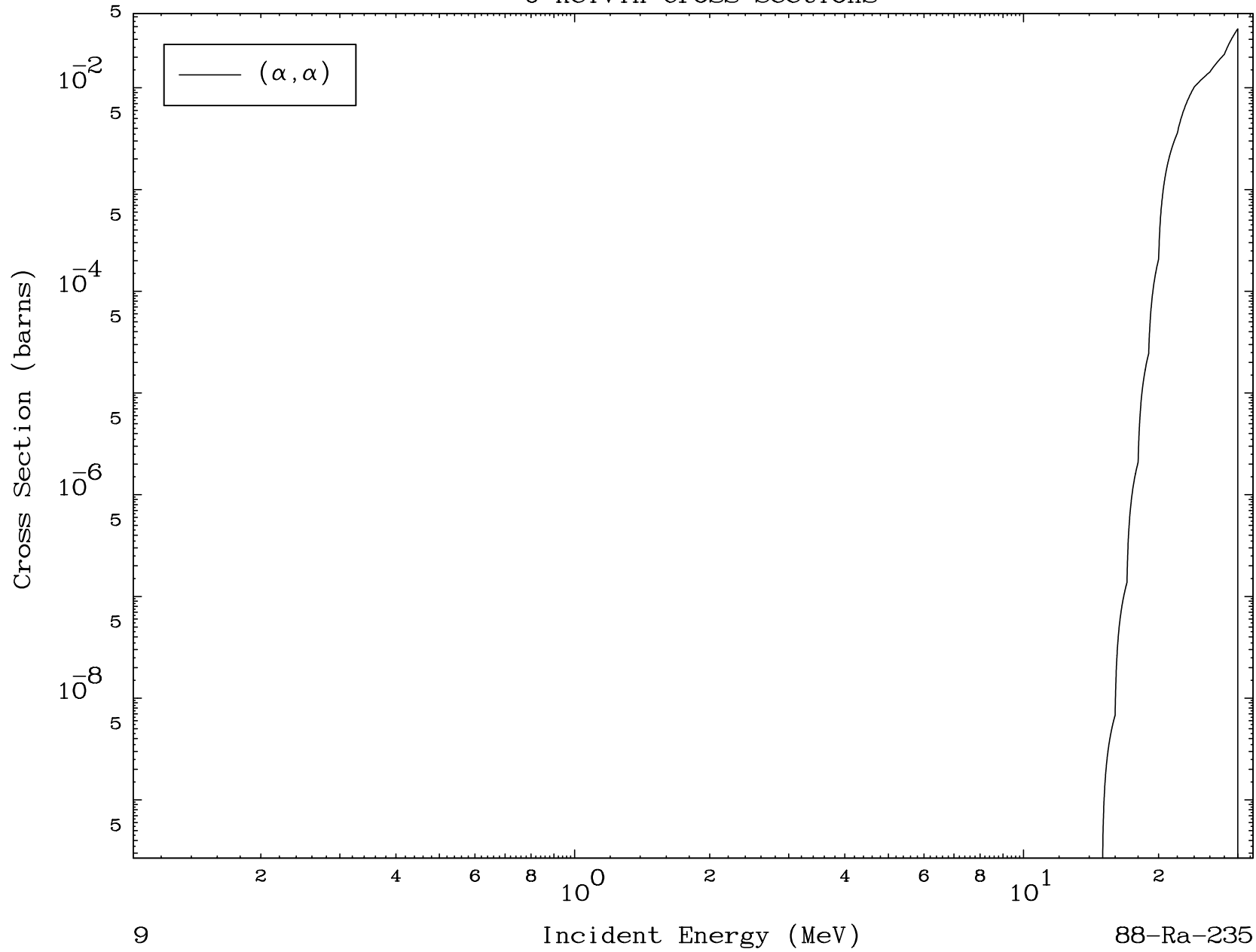




MAT 8861

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

88-Ra-235



9

Incident Energy (MeV)

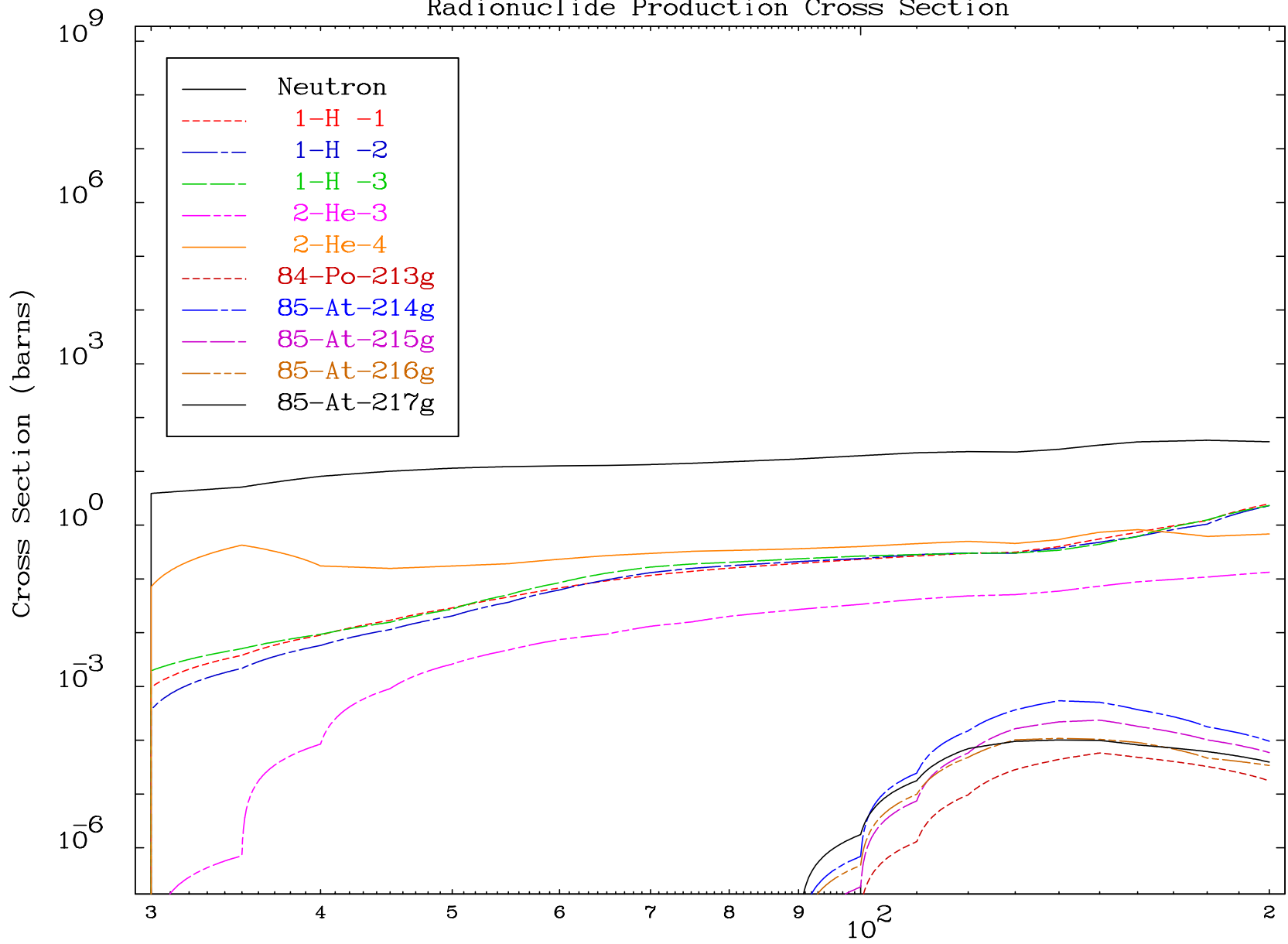
88-Ra-235

MAT 8861

( $\alpha$ , remainder)

88-Ra-235

Radionuclide Production Cross Section

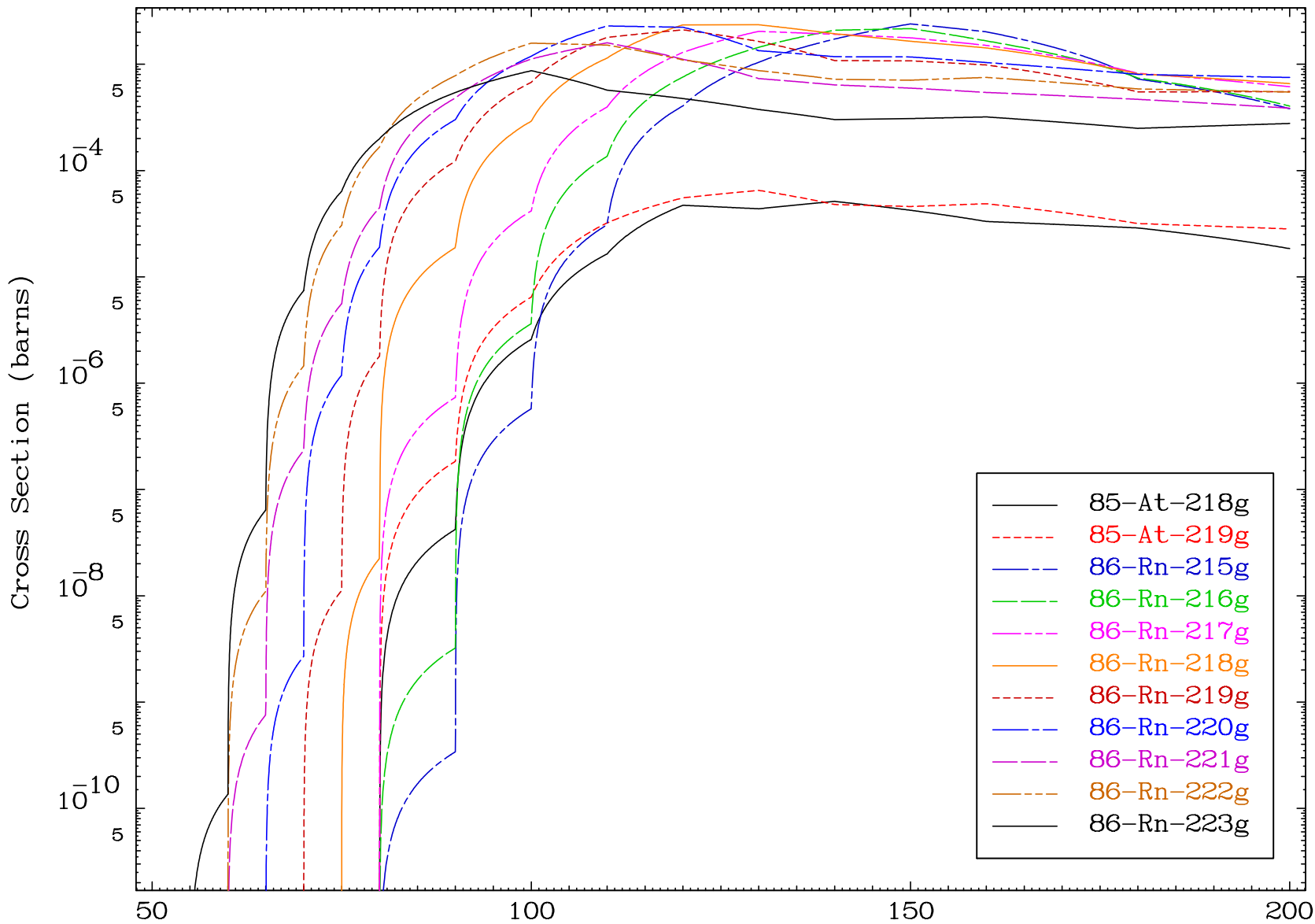


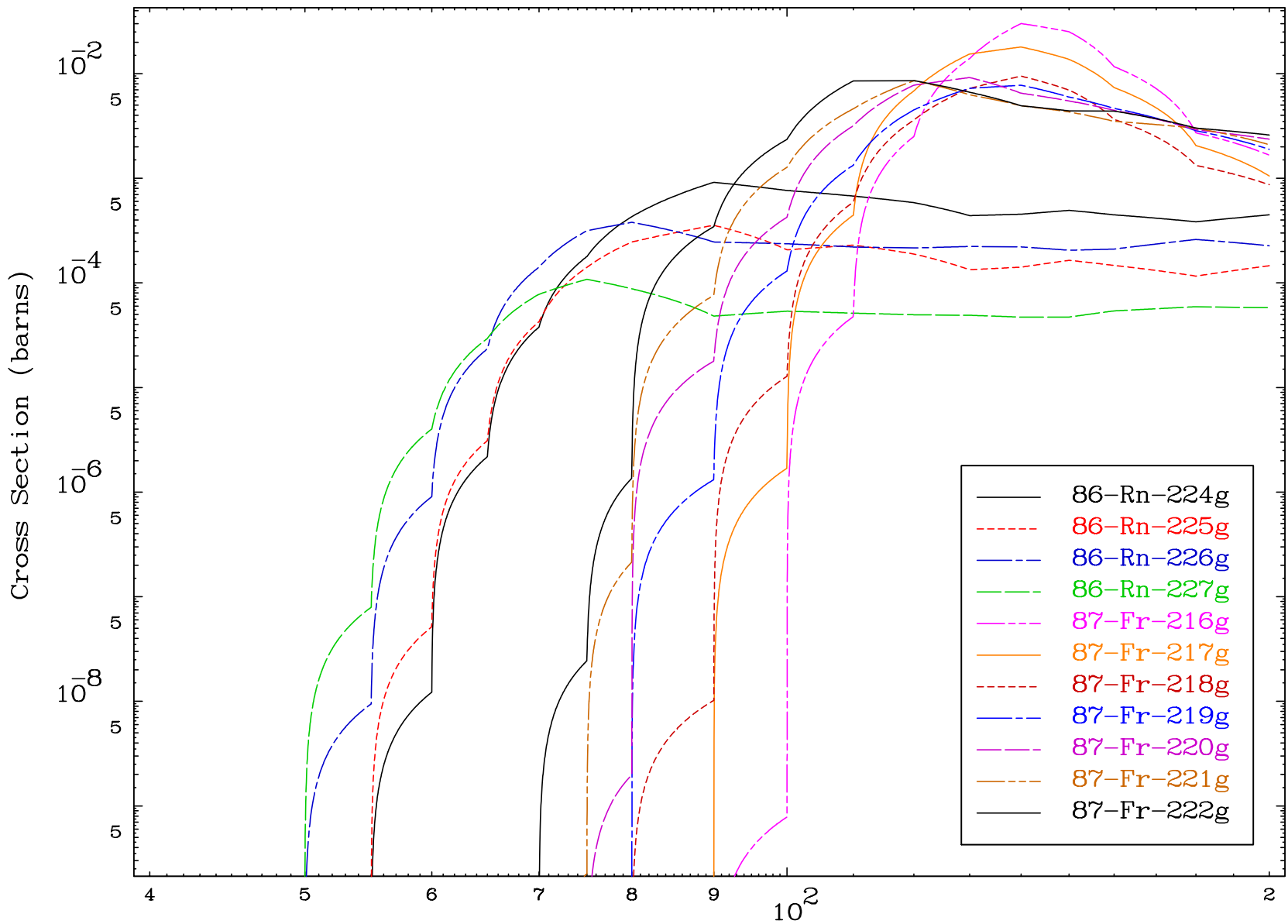
10

Incident Energy (MeV)

88-Ra-235

Radionuclide Production Cross Section



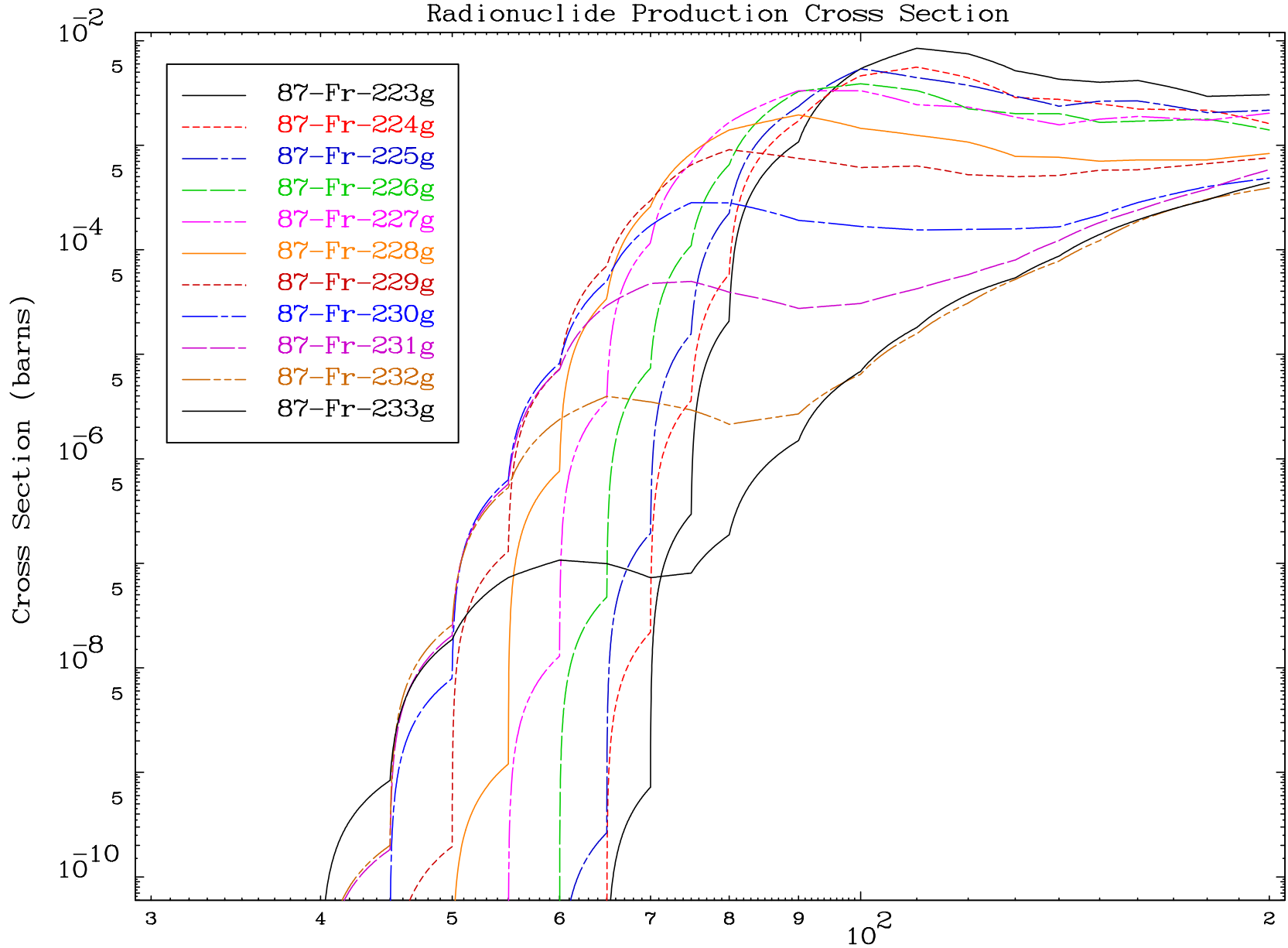


MAT 8861

( $\alpha$ , remainder)

88-Ra-235

### Radionuclide Production Cross Section

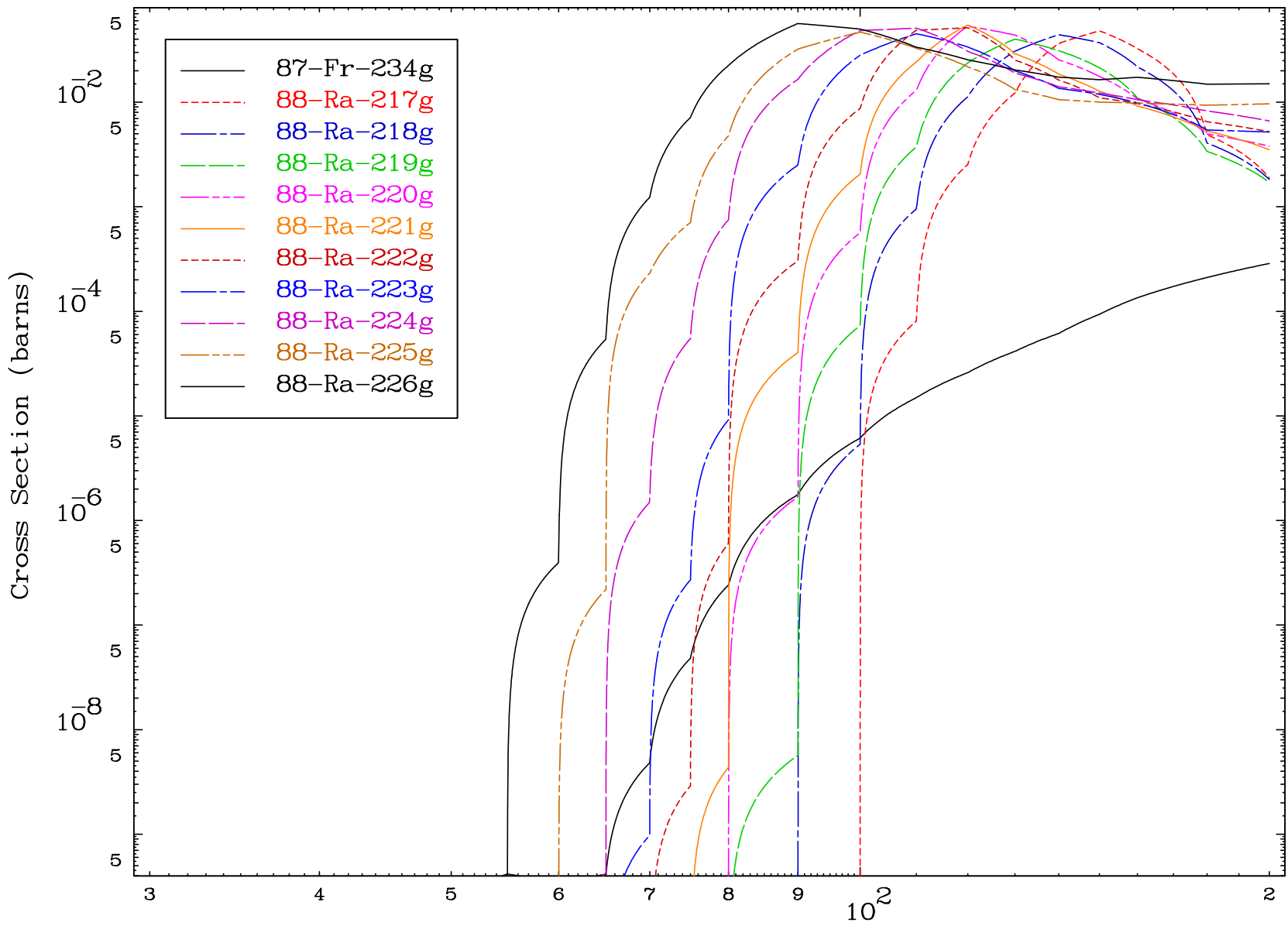


13

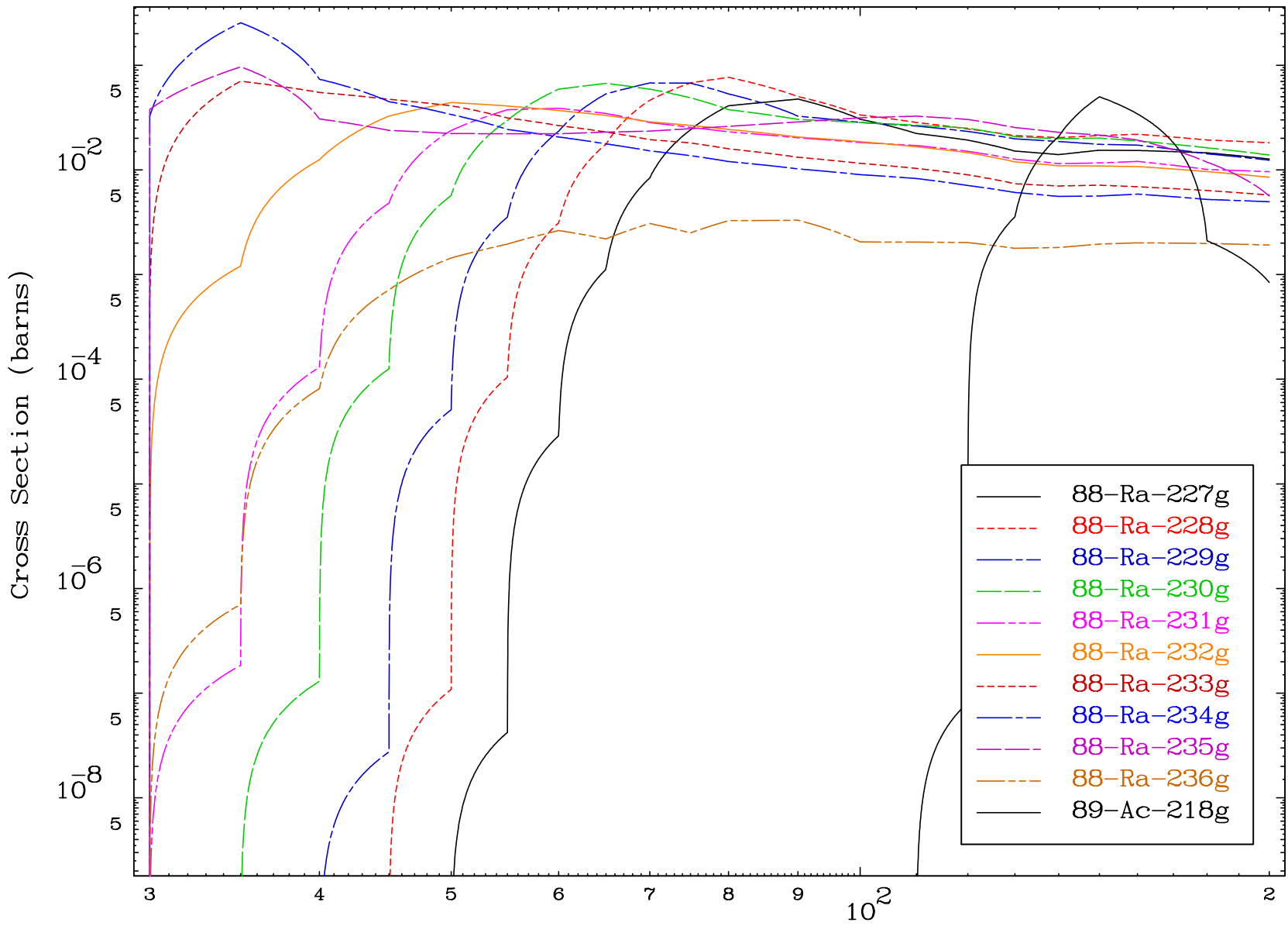
Incident Energy (MeV)

88-Ra-235

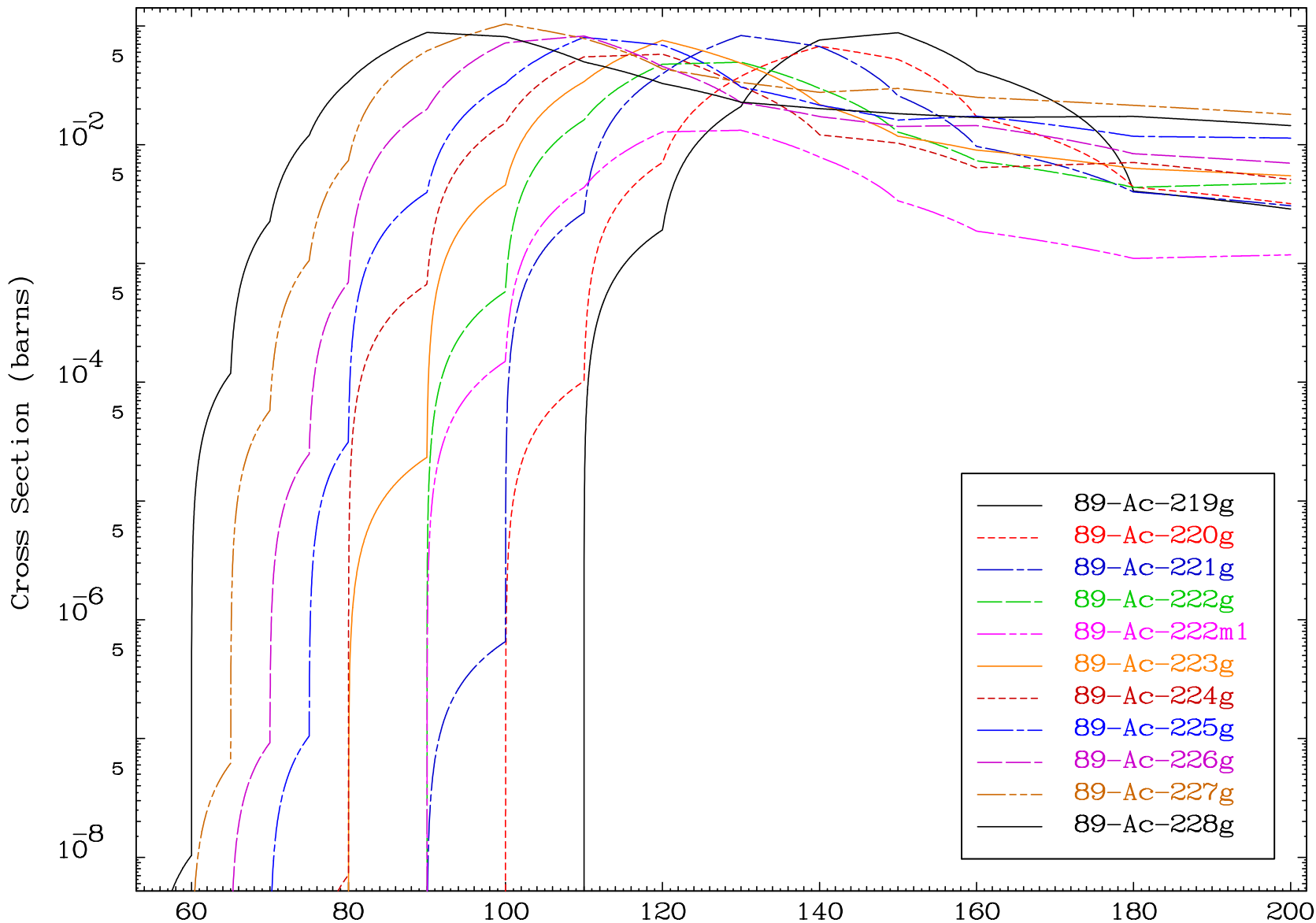
Radionuclide Production Cross Section



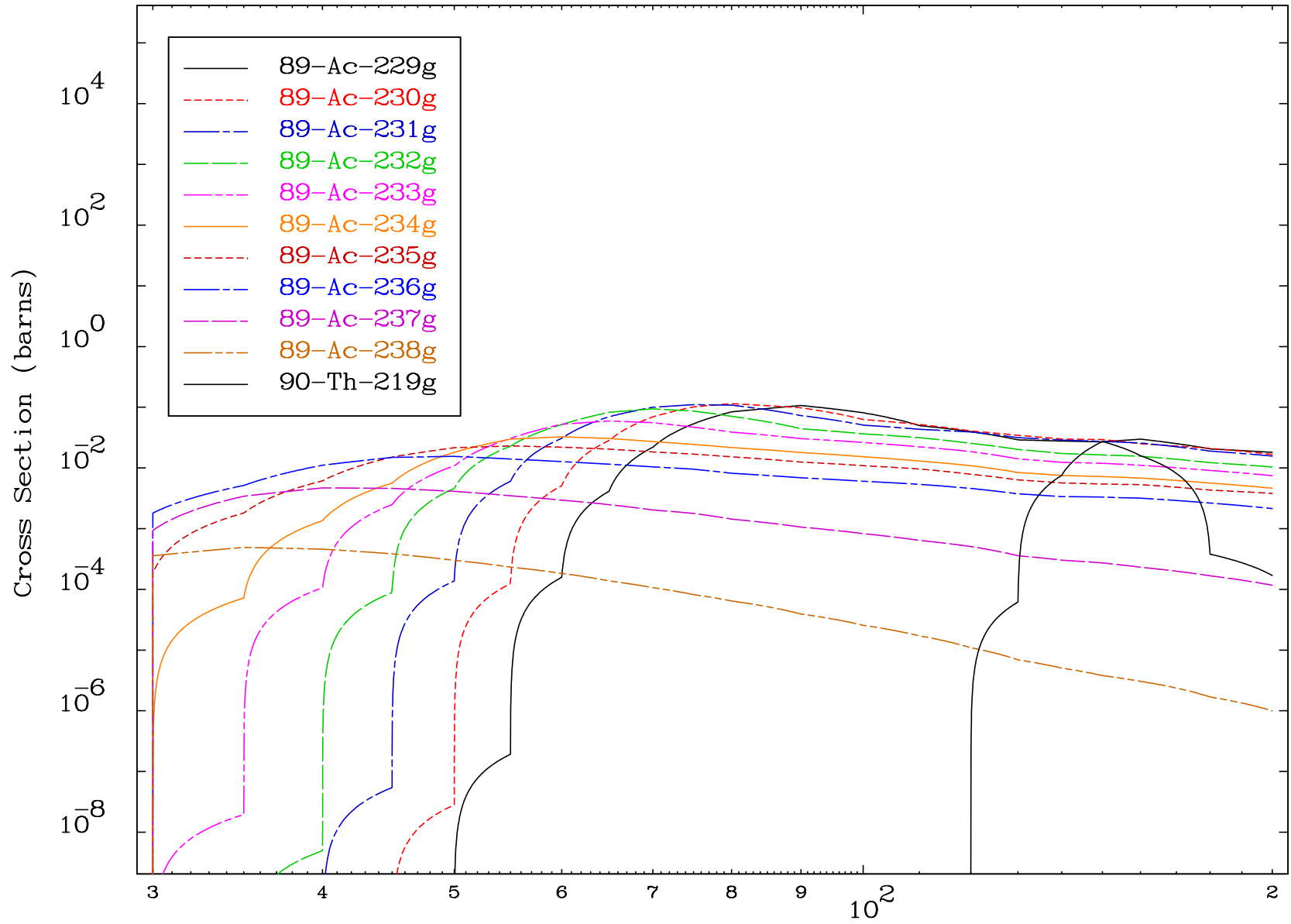
Radionuclide Production Cross Section



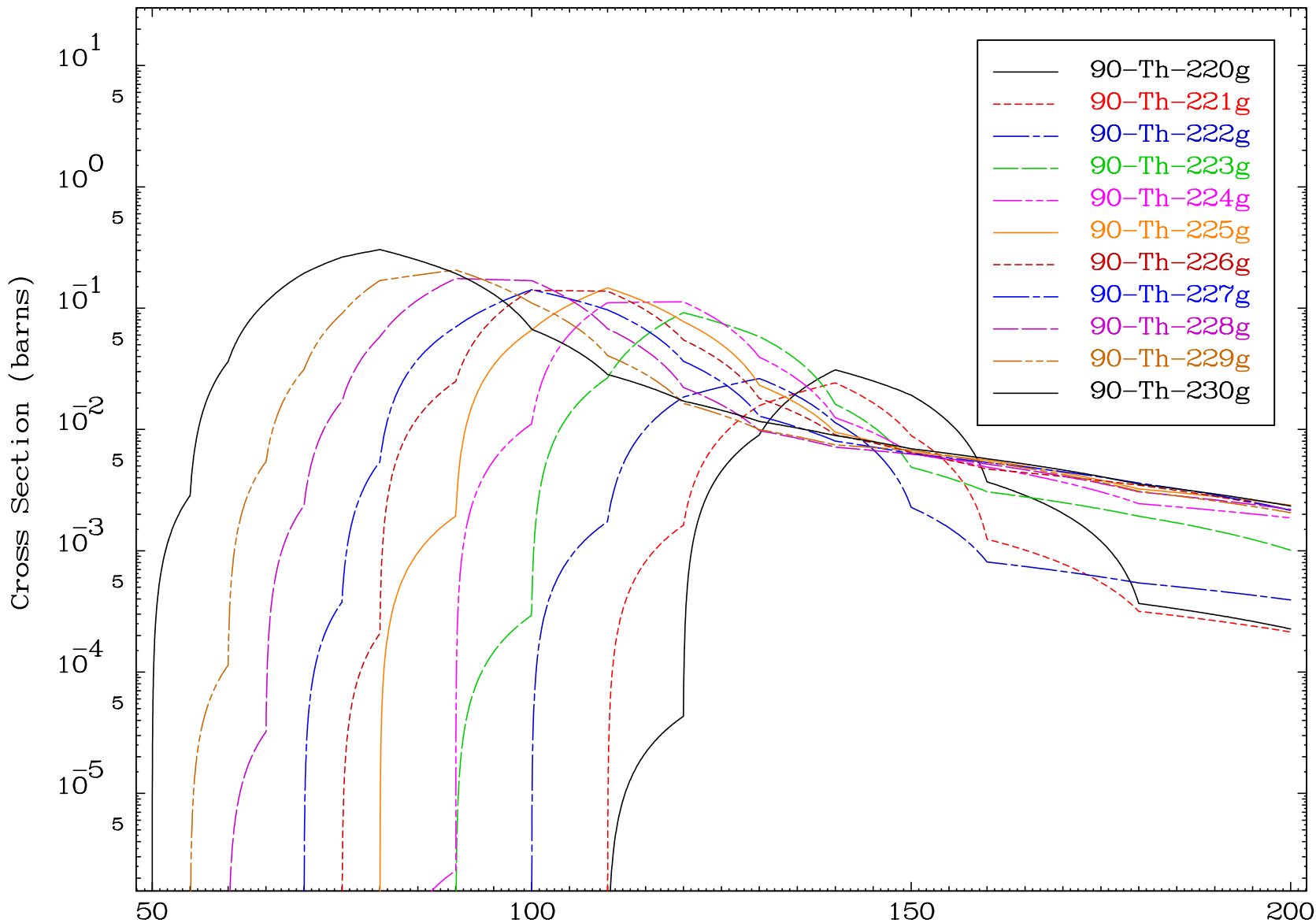
Radionuclide Production Cross Section







Radionuclide Production Cross Section

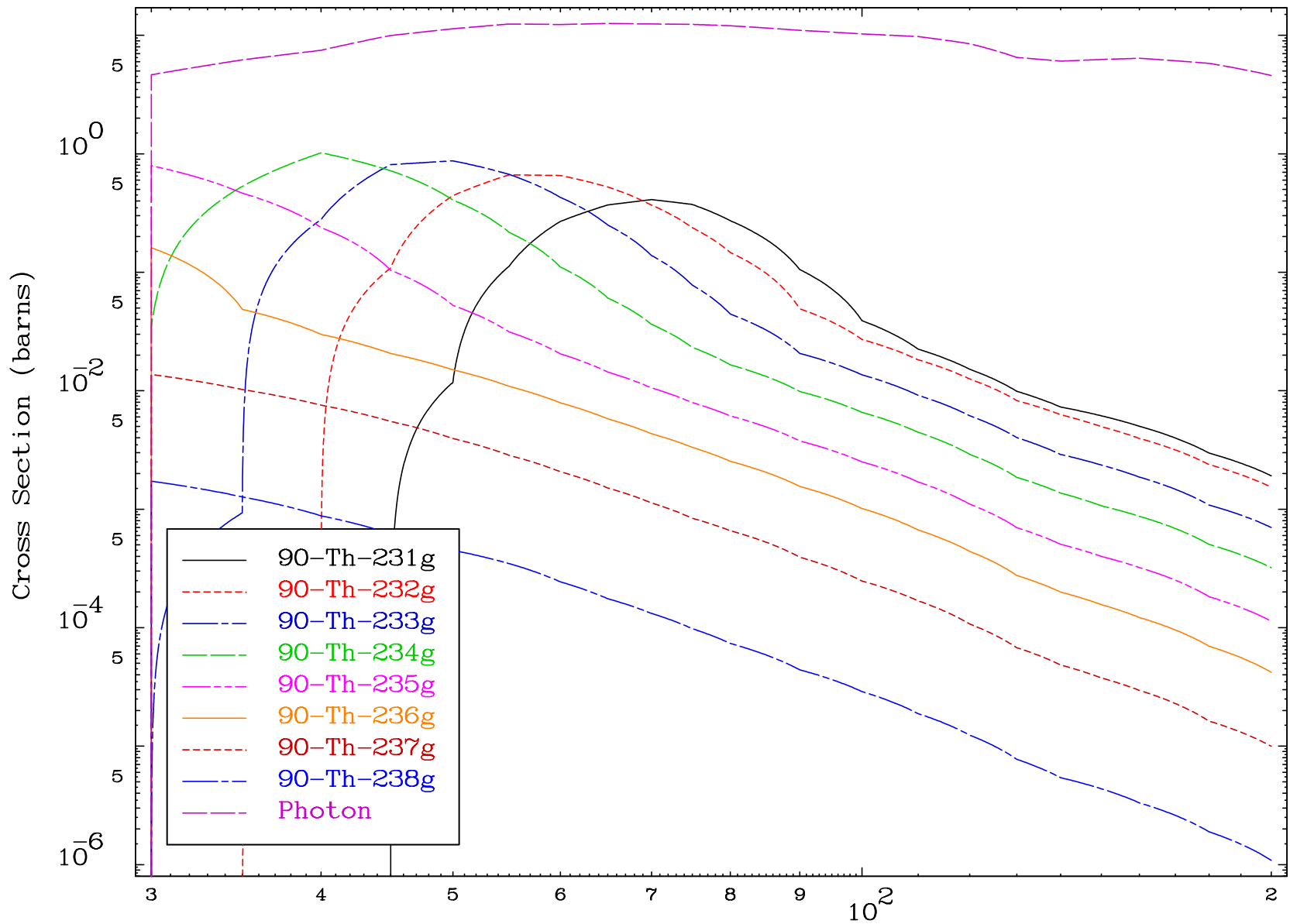


MAT 8861

( $\alpha$ , remainder)

88-Ra-235

### Radionuclide Production Cross Section



19

Incident Energy (MeV)

88-Ra-235

MAT 8861

$\alpha$  Fission  
Radionuclide Production Cross Section

88-Ra-235

