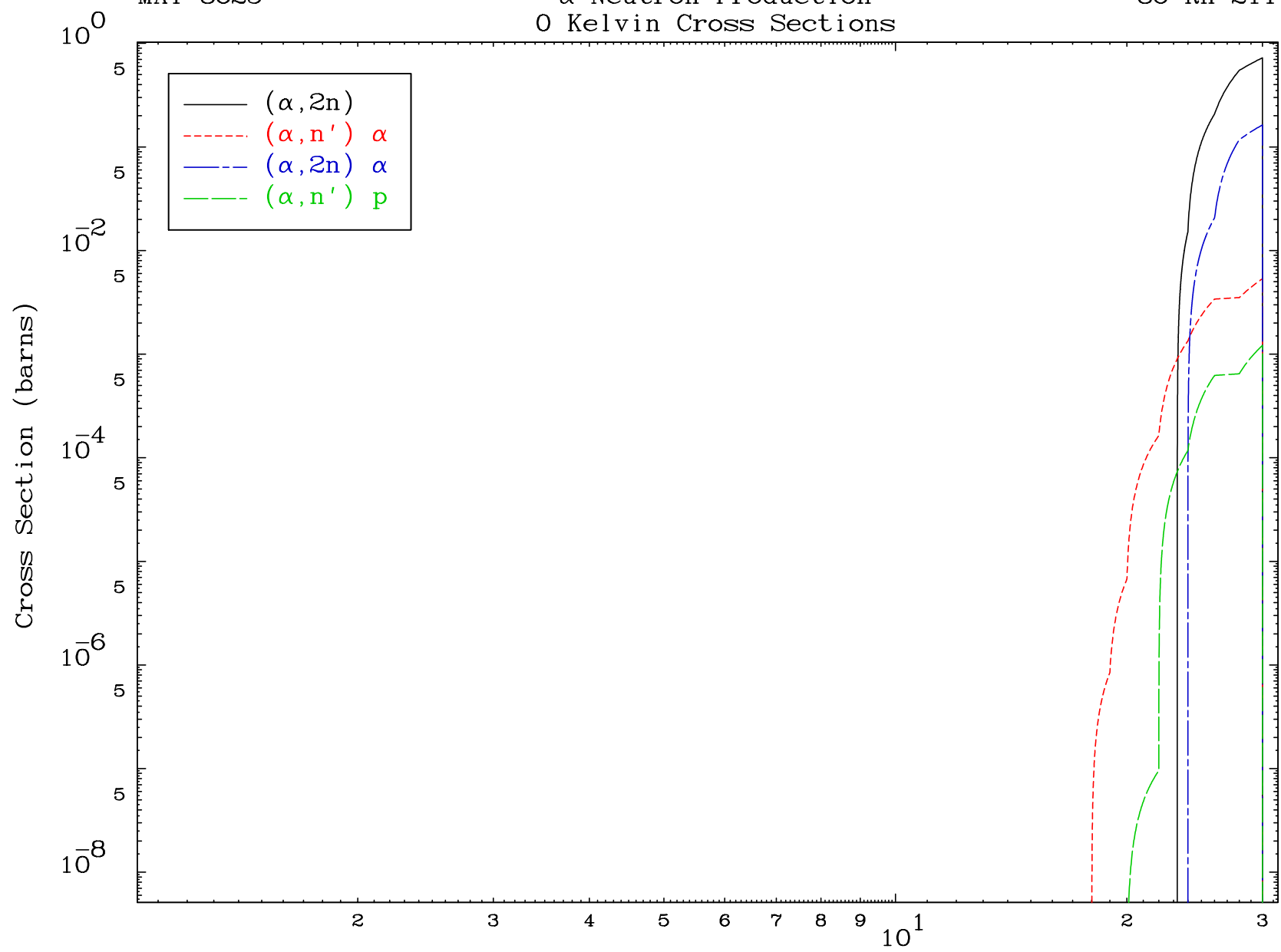


MAT 8625

α Neutron Production
0 Kelvin Cross Sections

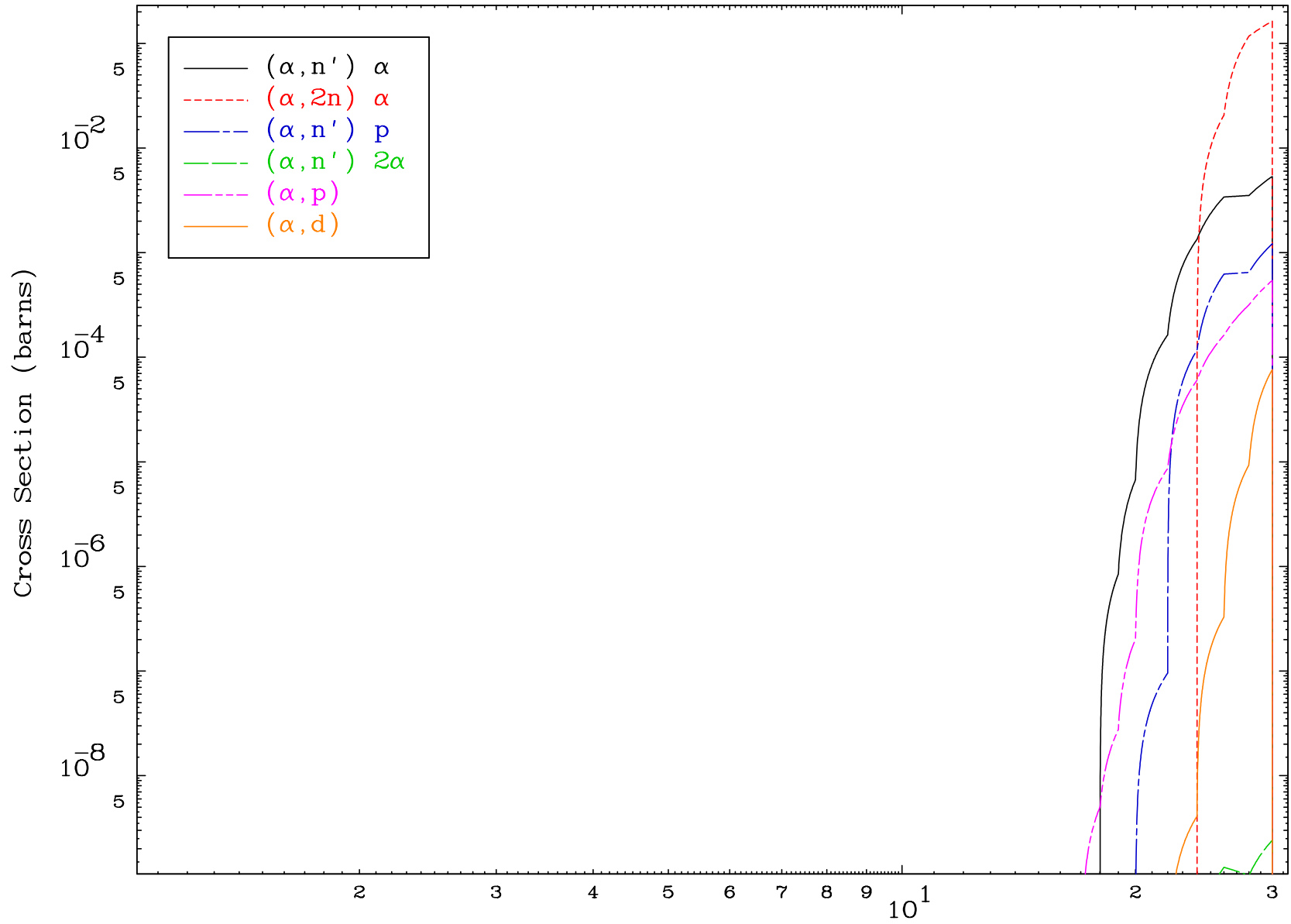
86-Rn-211

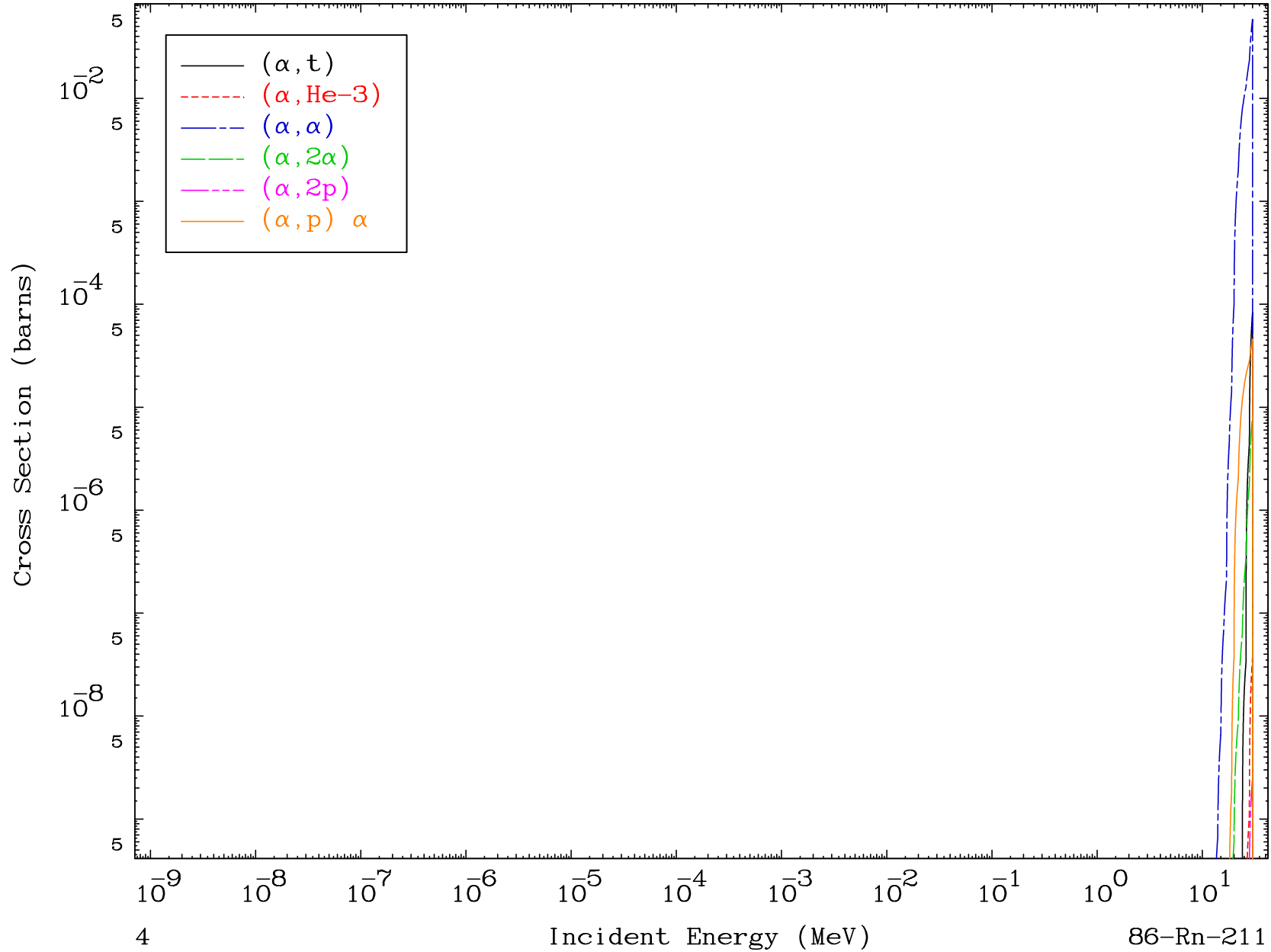


2

Incident Energy (MeV)

86-Rn-211

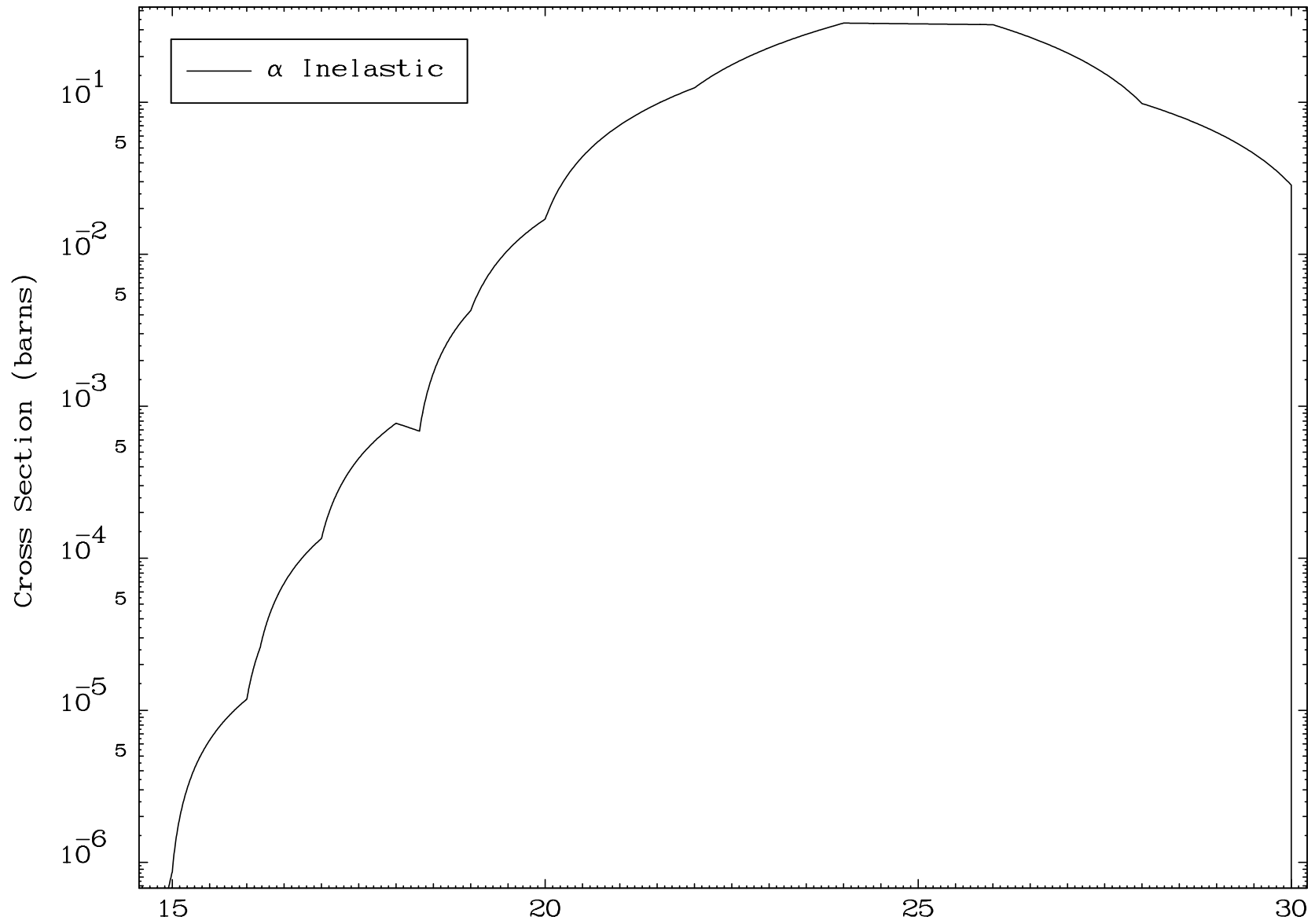




MAT 8625

(α, n') Level
0 Kelvin Cross Sections

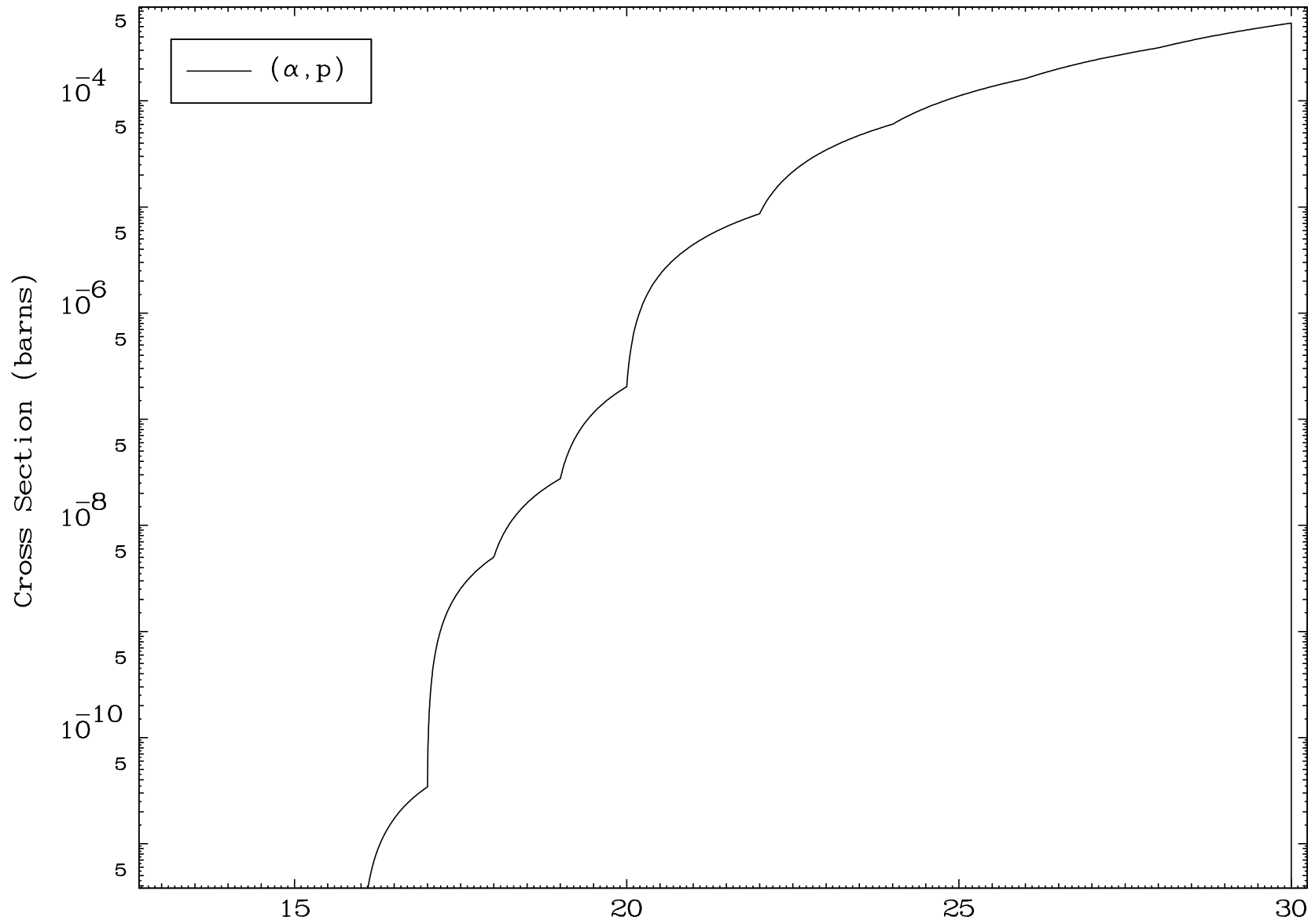
86-Rn-211



5

Incident Energy (MeV)

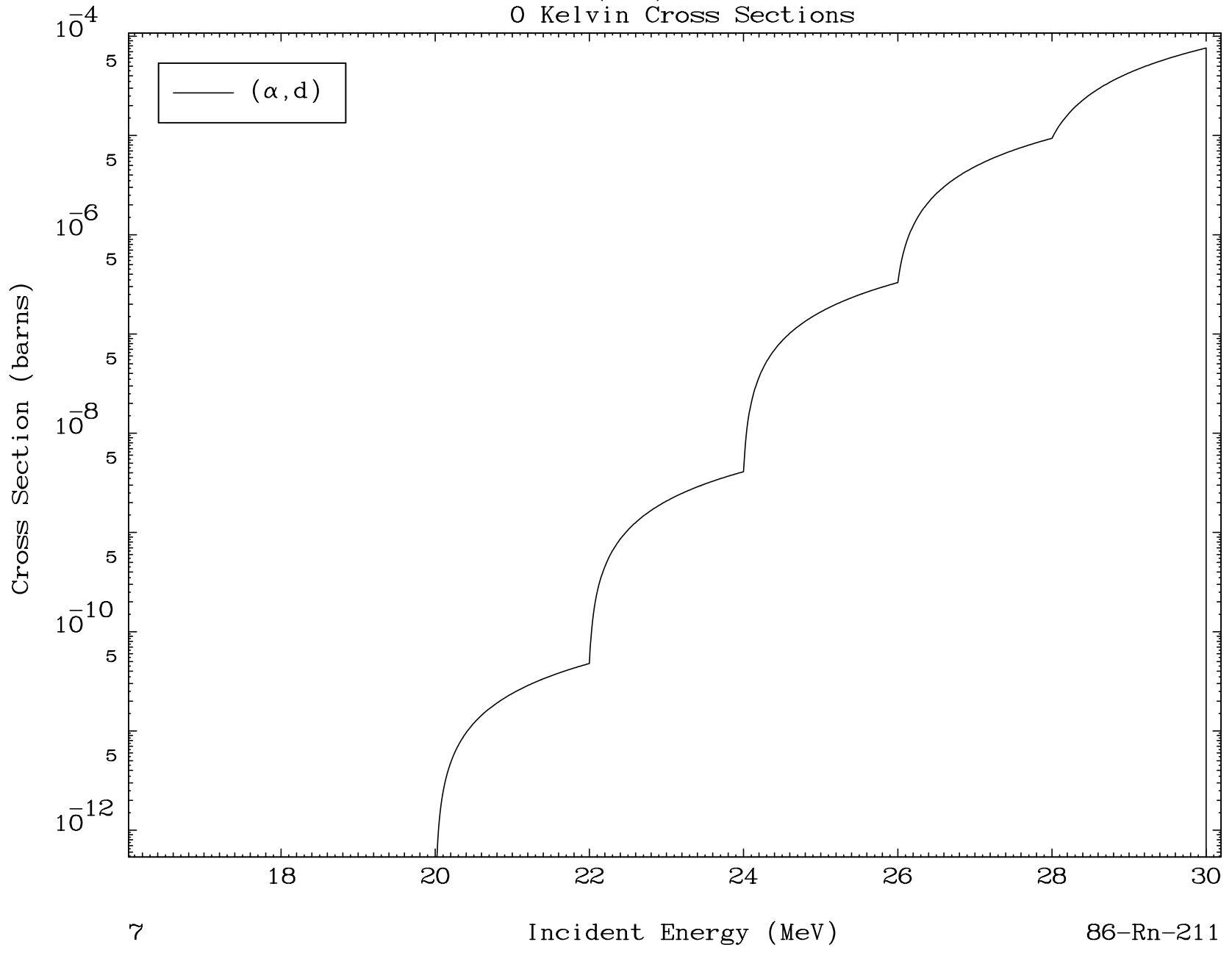
86-Rn-211



MAT 8625

(α ,d) Levels
0 Kelvin Cross Sections

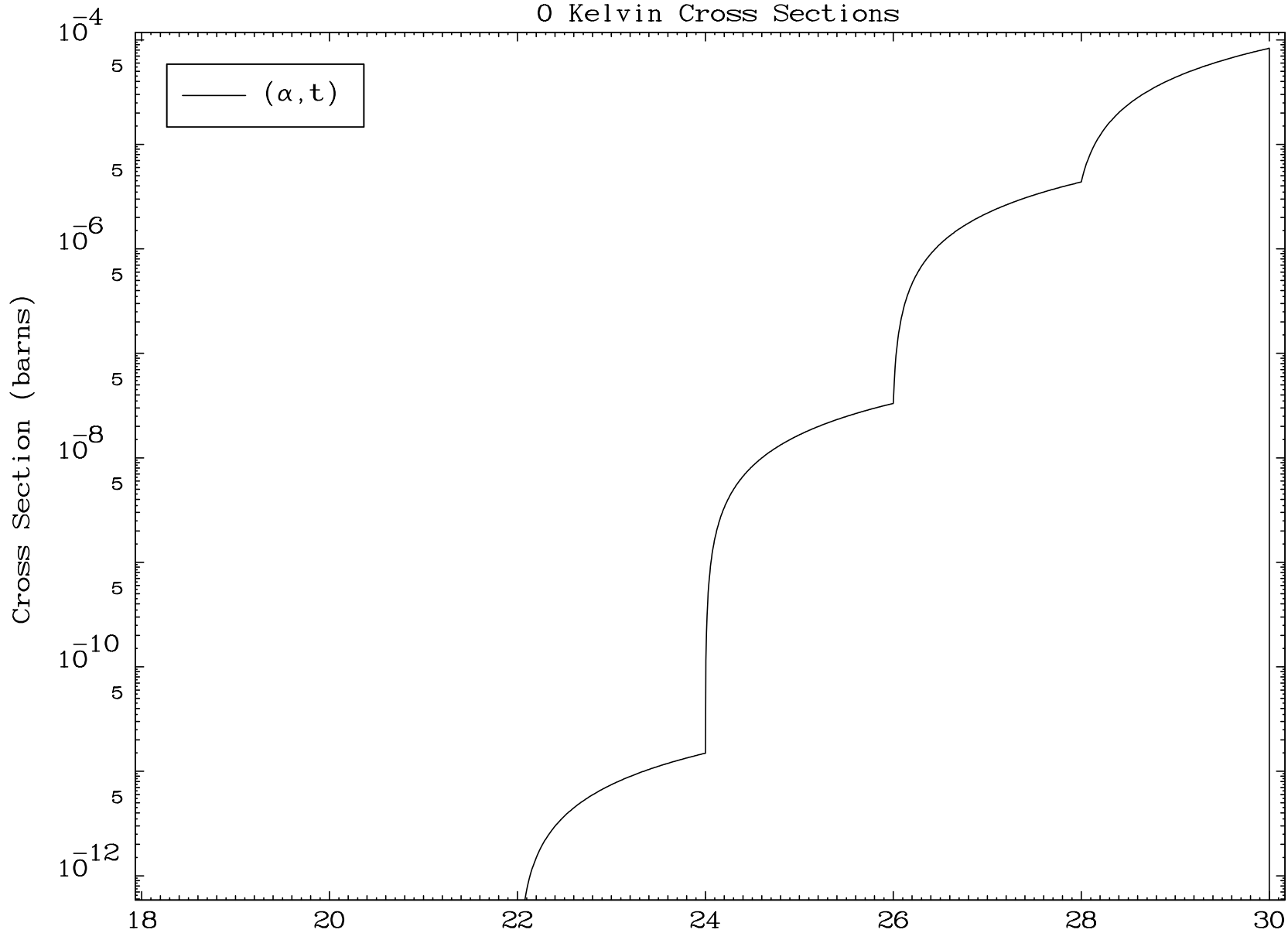
86-Rn-211



MAT 8625

(α, t) Levels
0 Kelvin Cross Sections

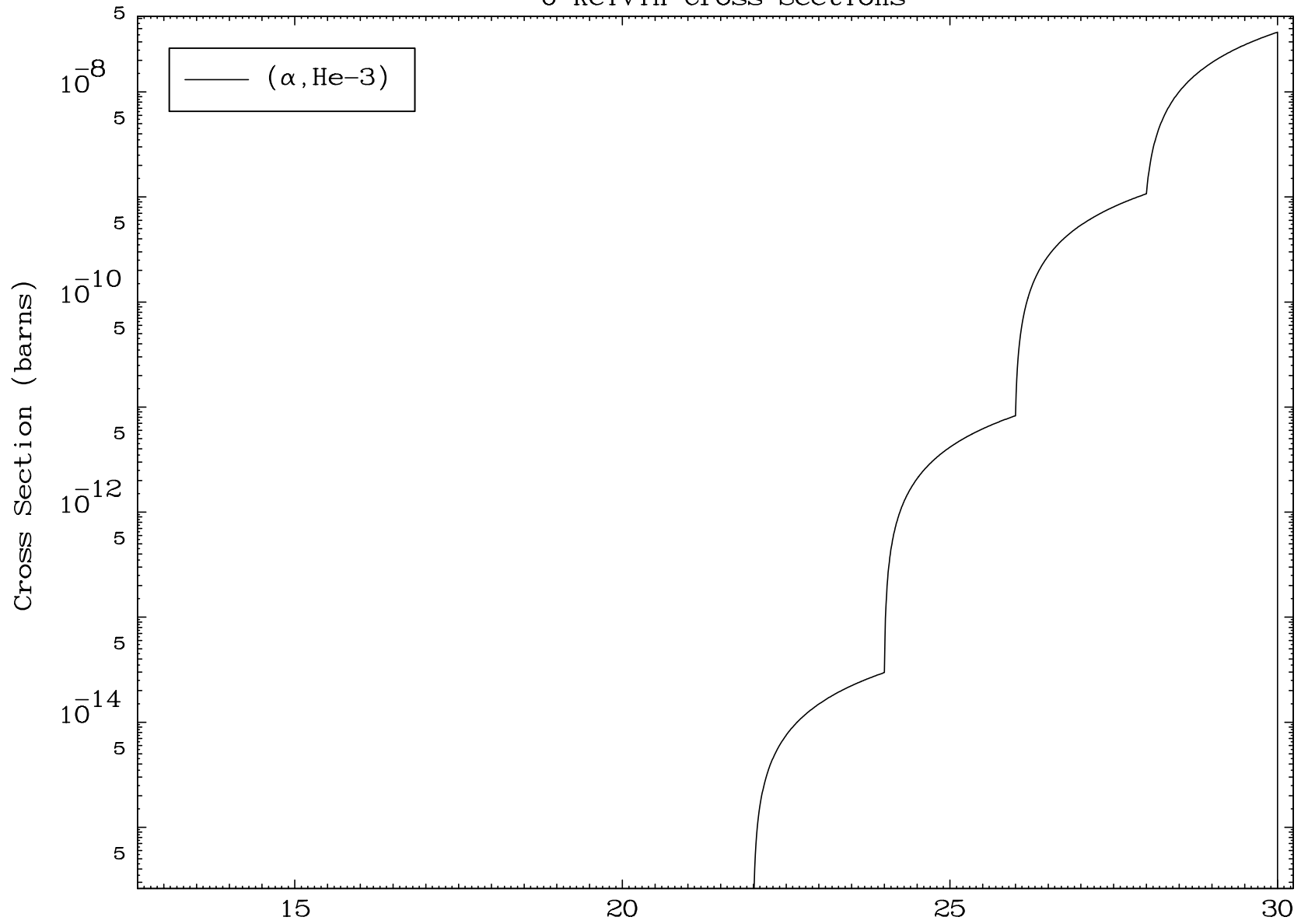
86-Rn-211



8

Incident Energy (MeV)

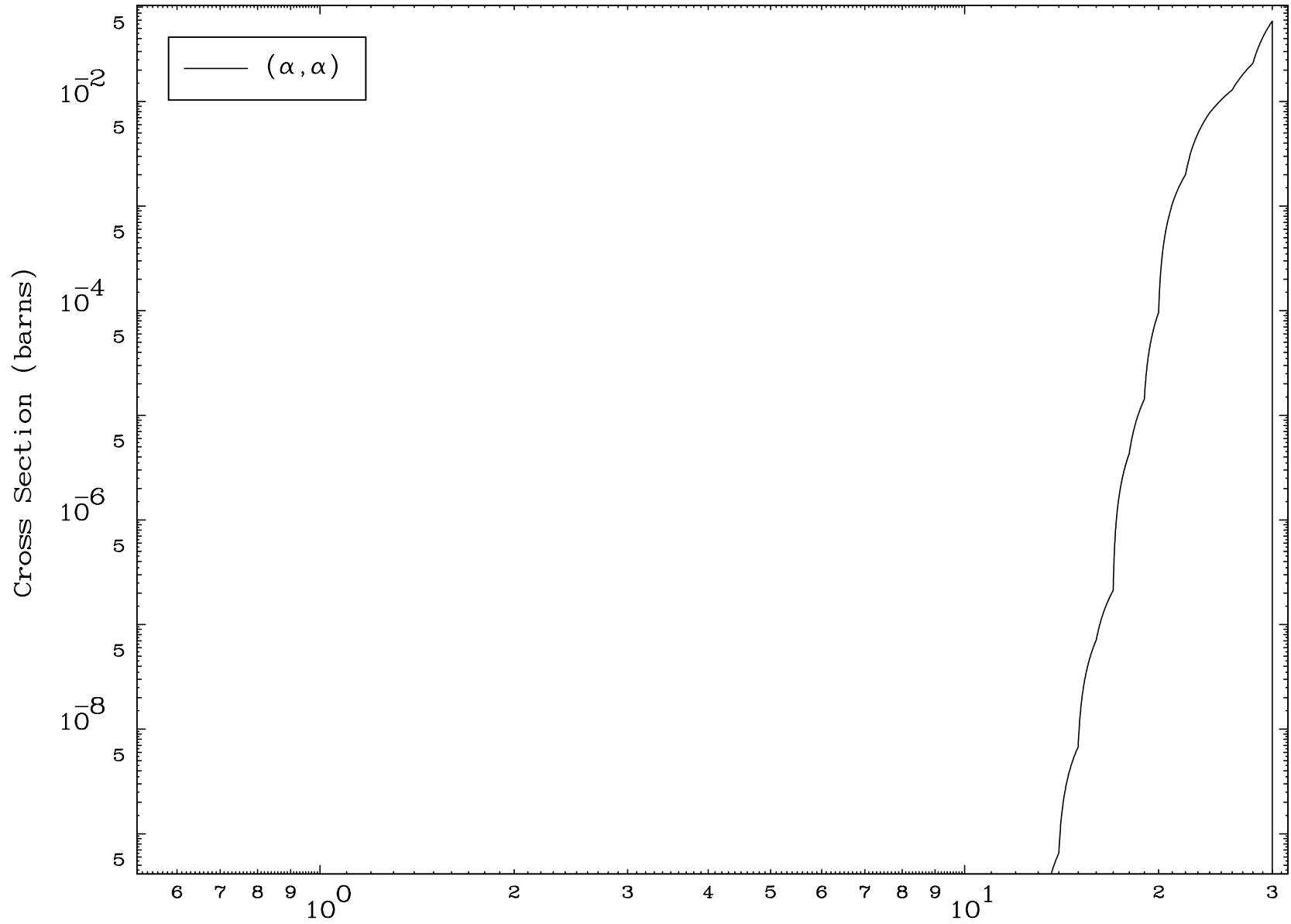
86-Rn-211



MAT 8625

(α, α) Levels
0 Kelvin Cross Sections

86-Rn-211

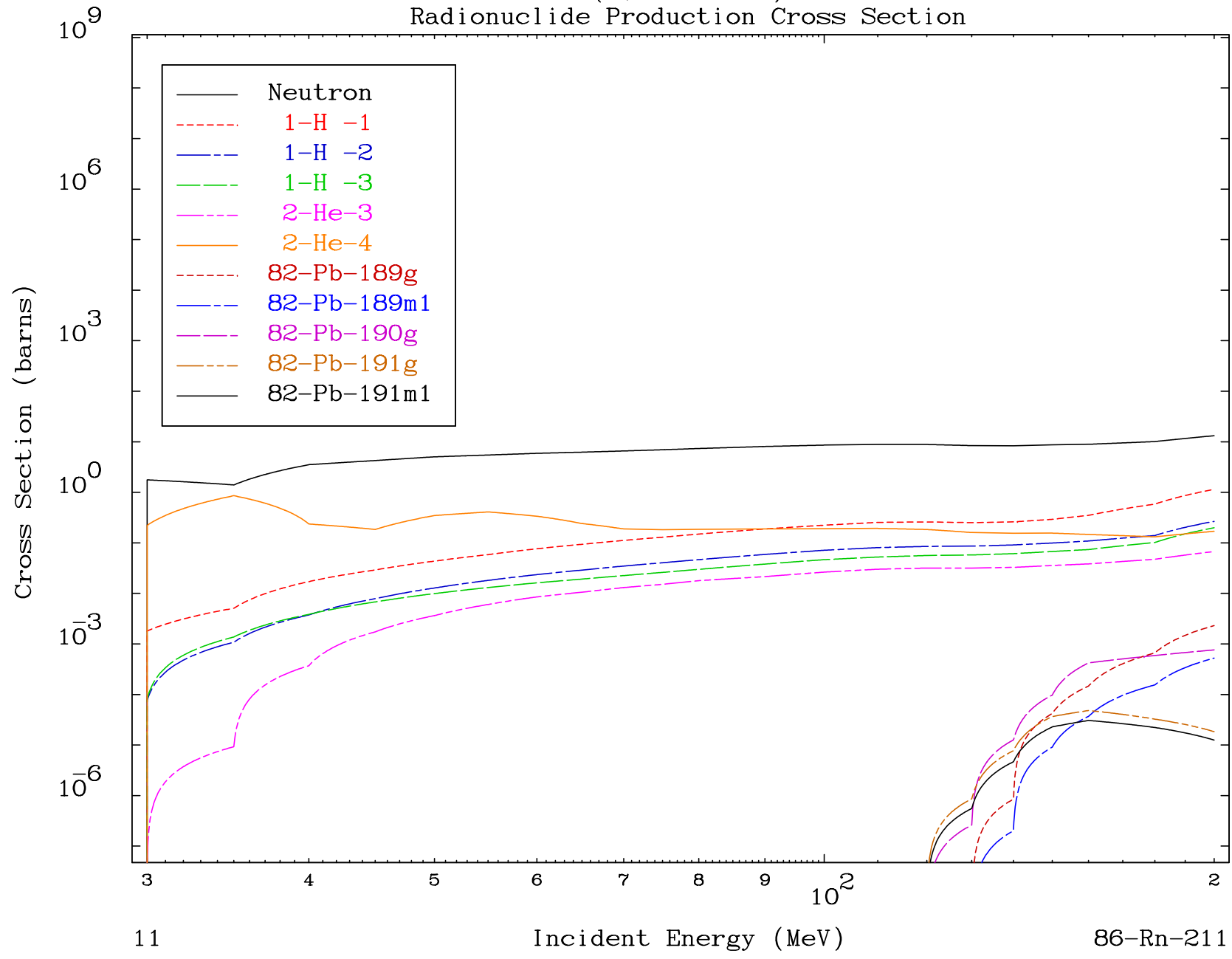


10

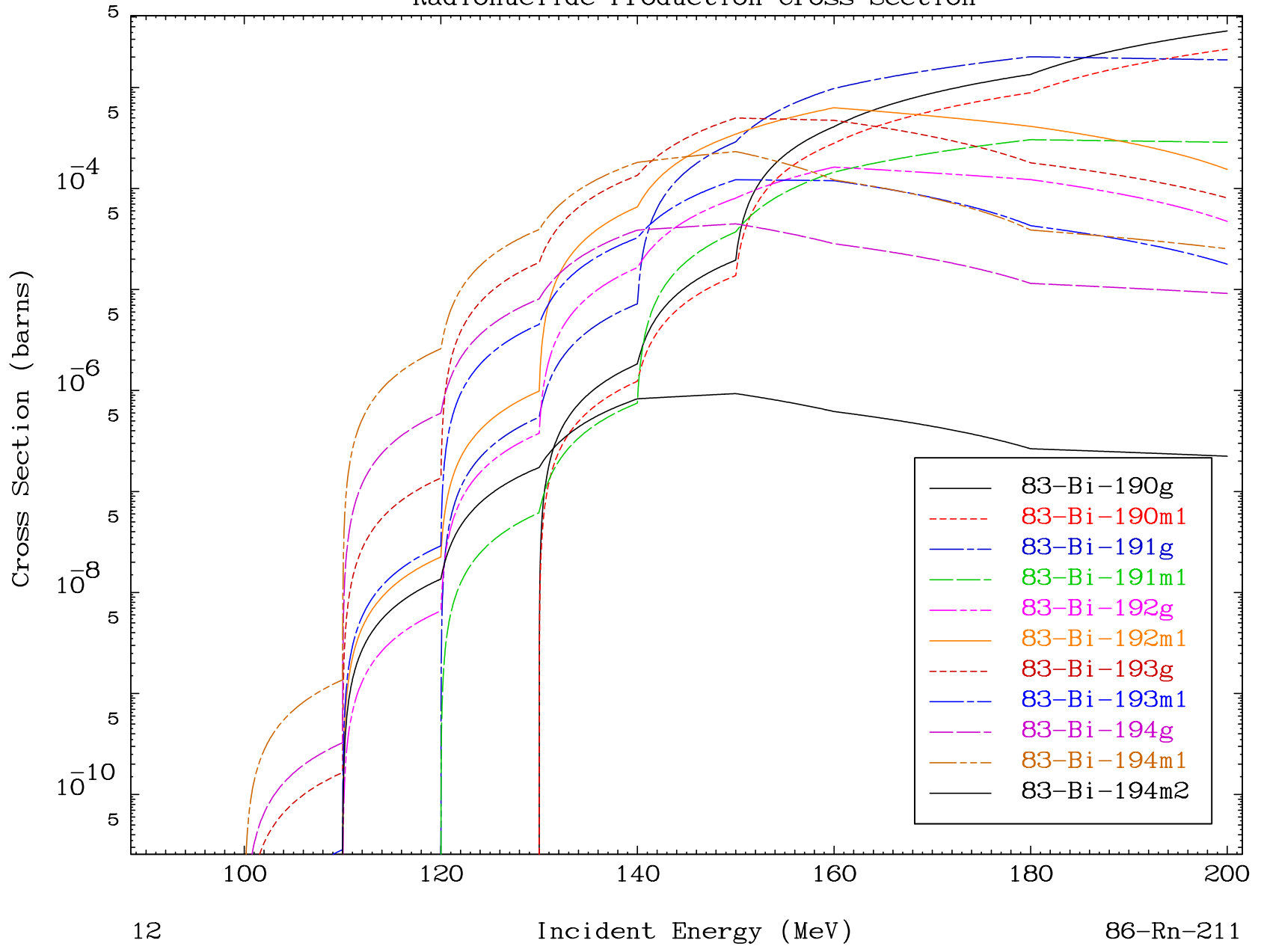
Incident Energy (MeV)

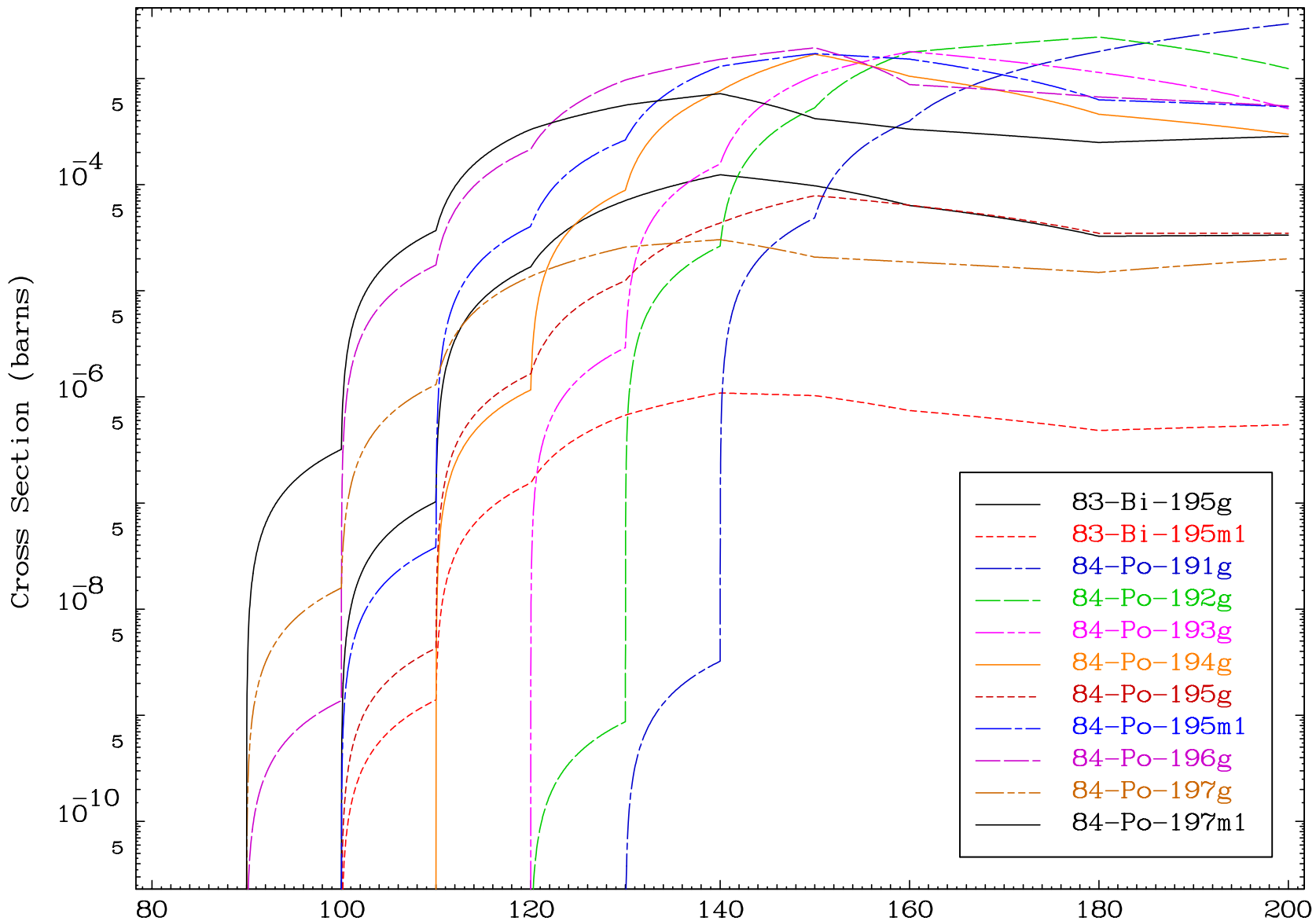
86-Rn-211

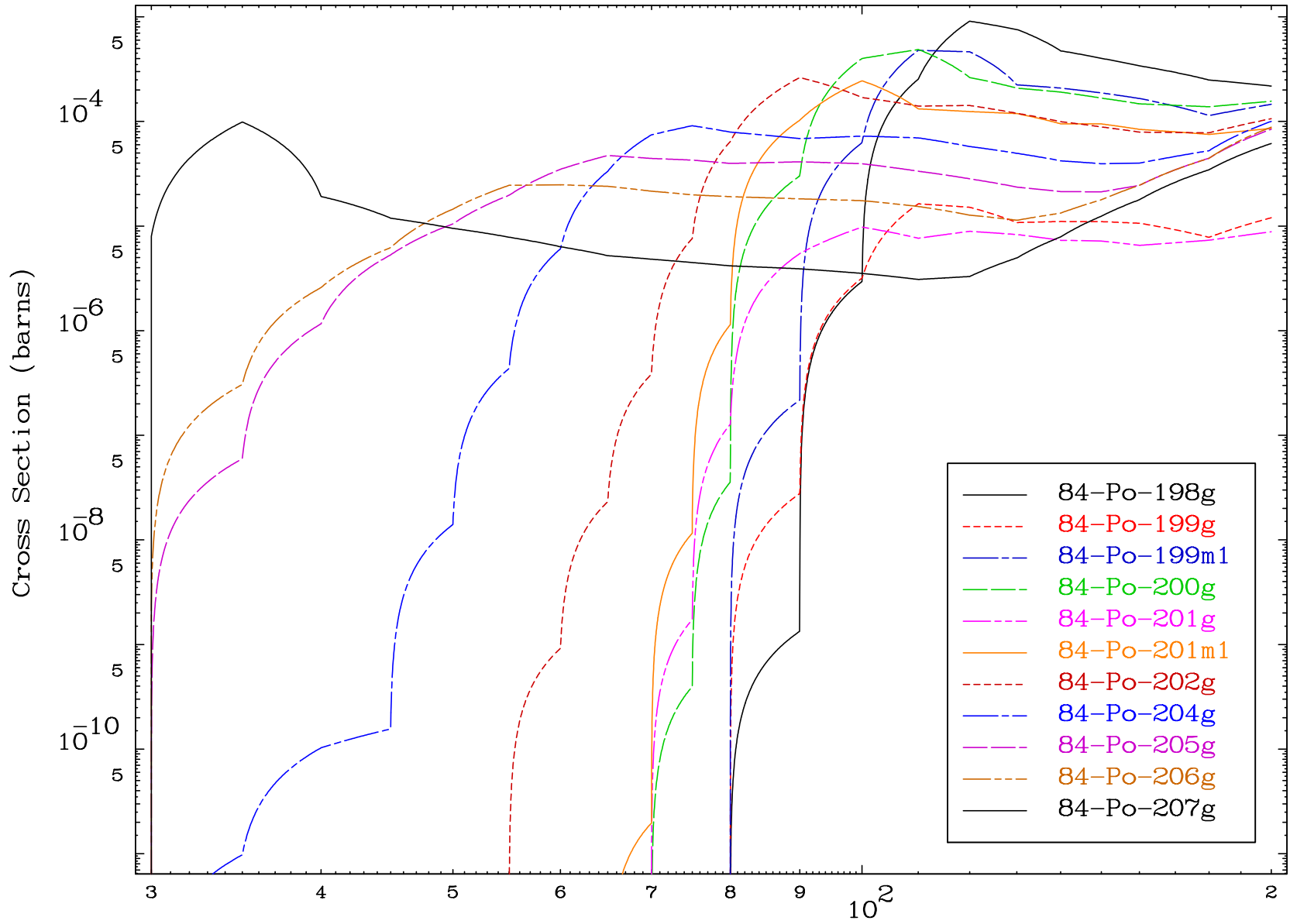
Radionuclide Production Cross Section

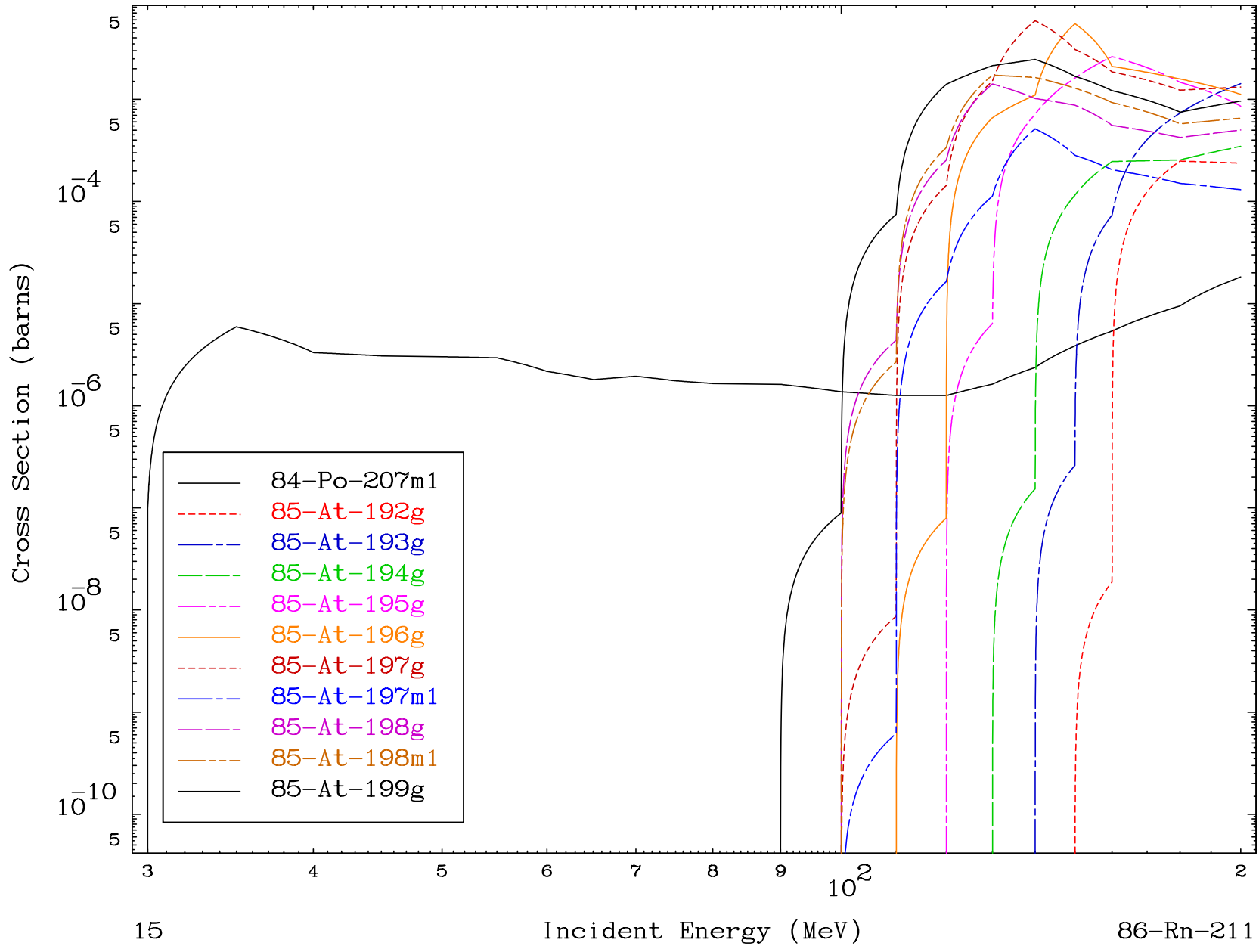


Radionuclide Production Cross Section

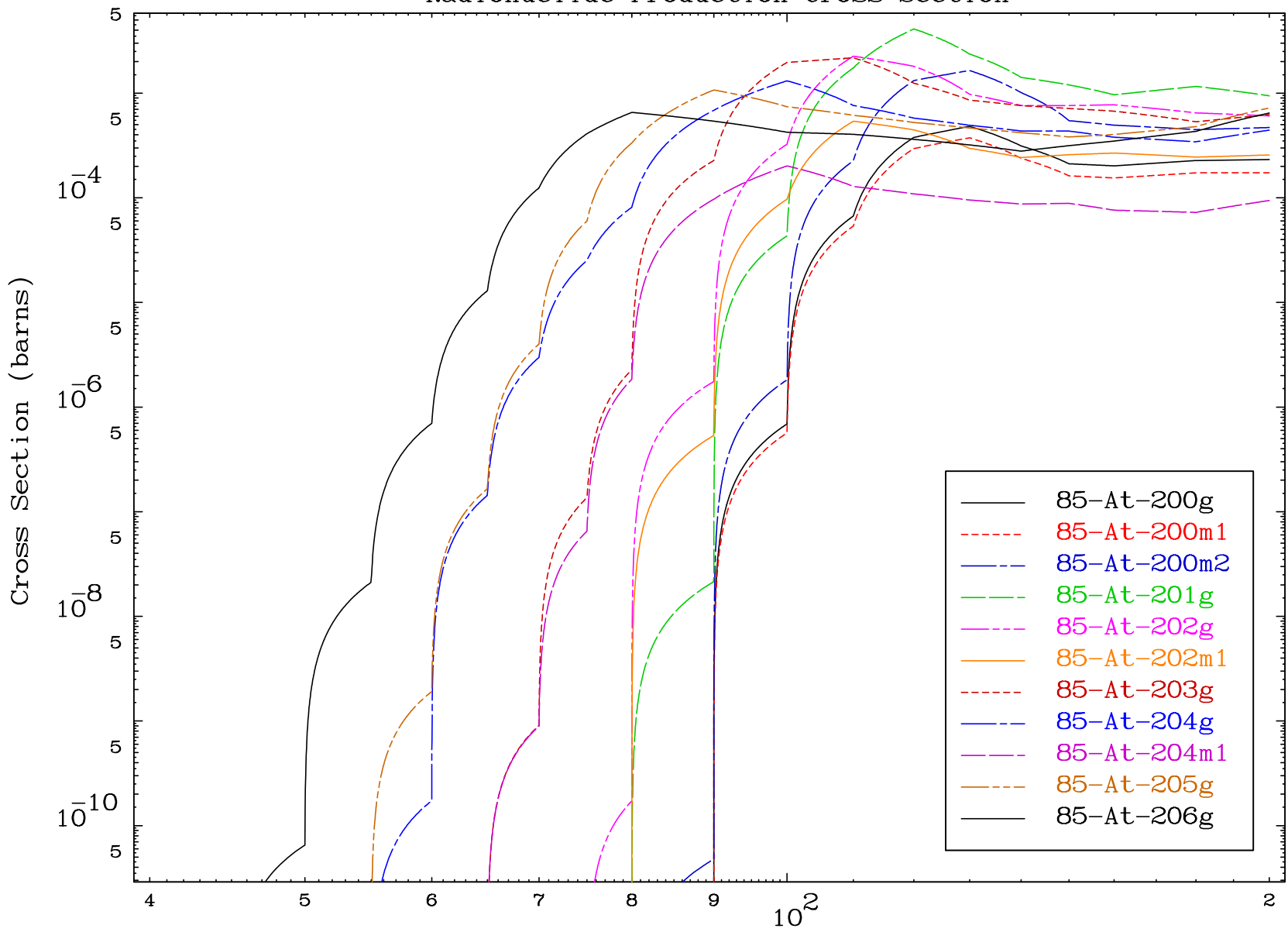


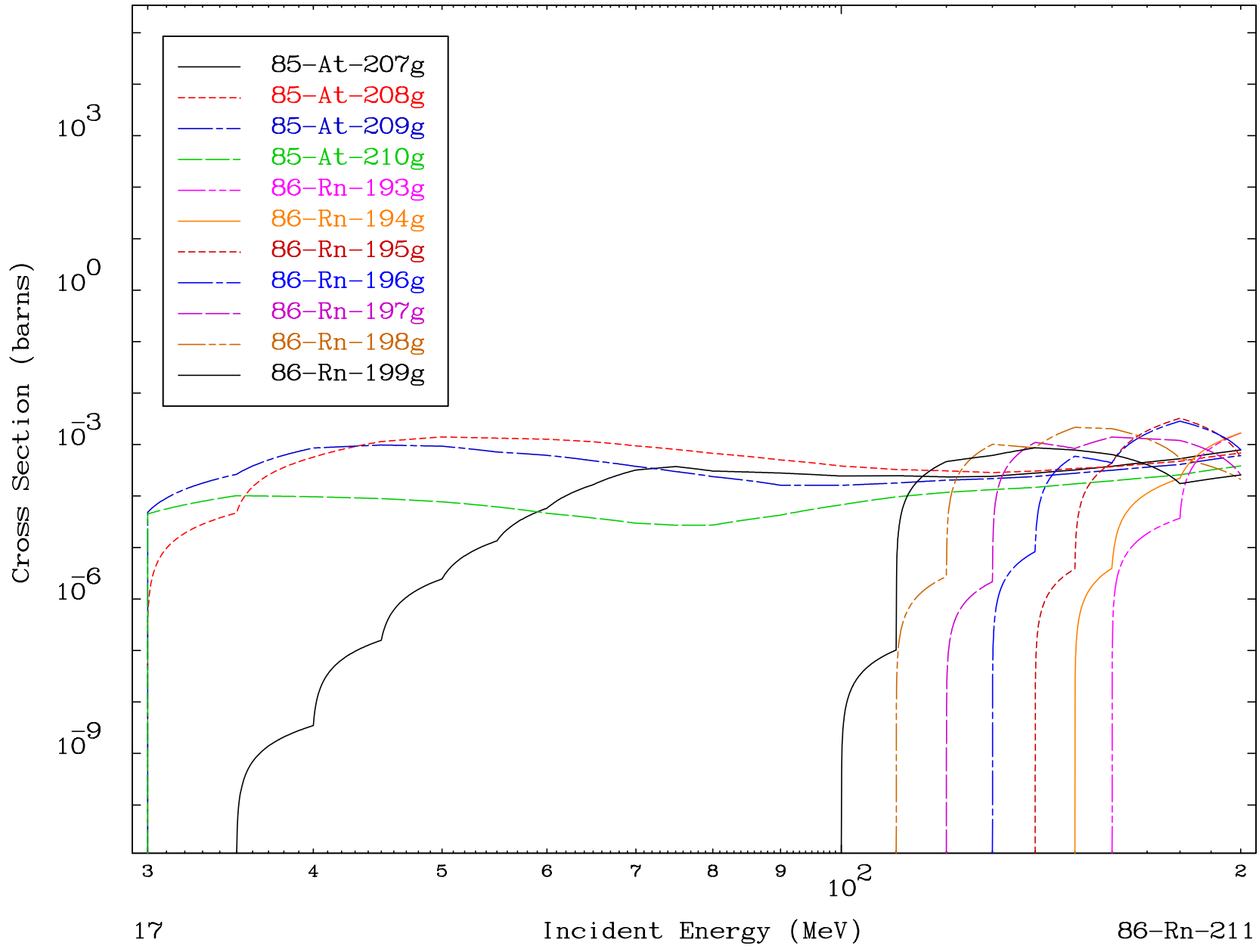


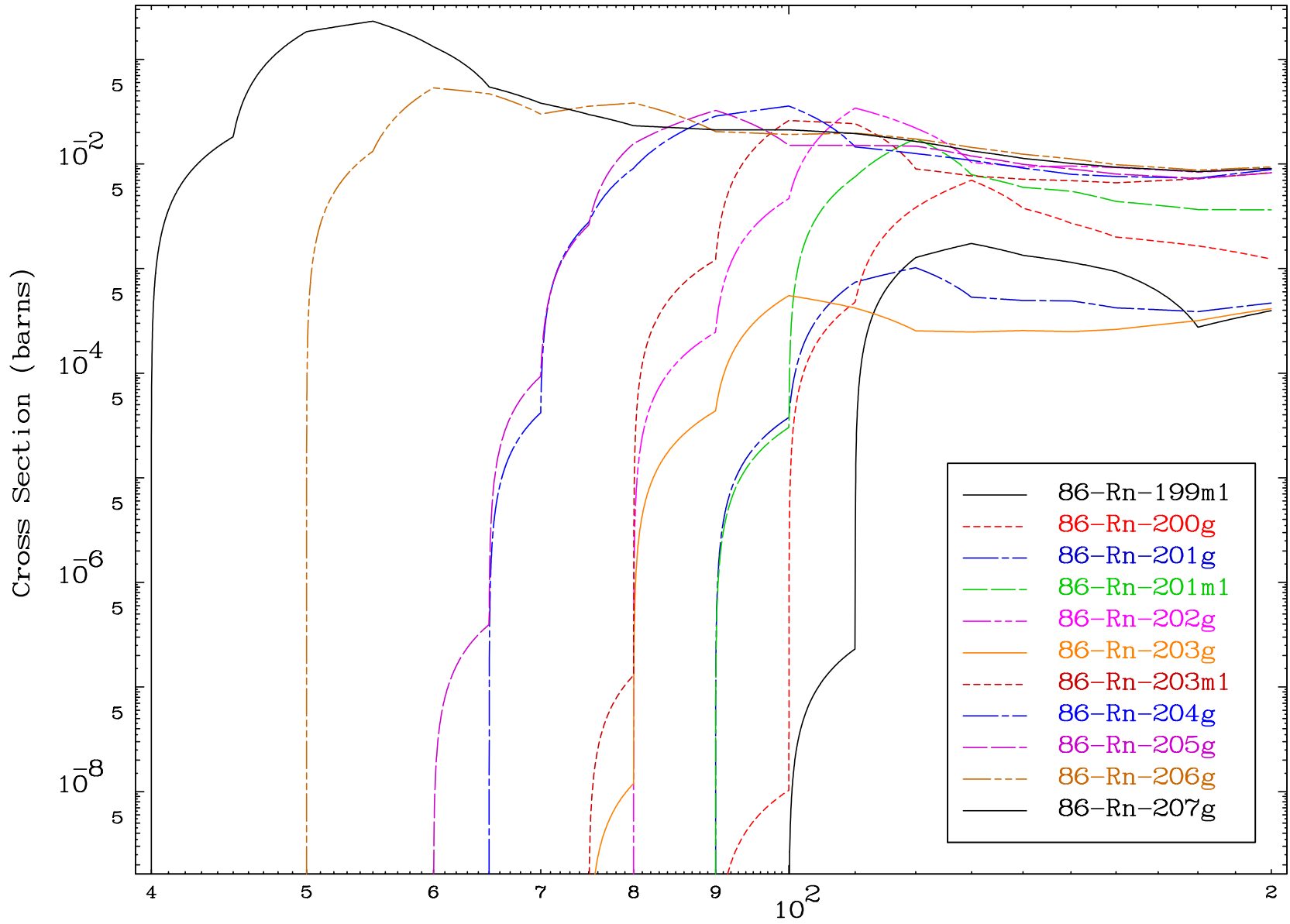


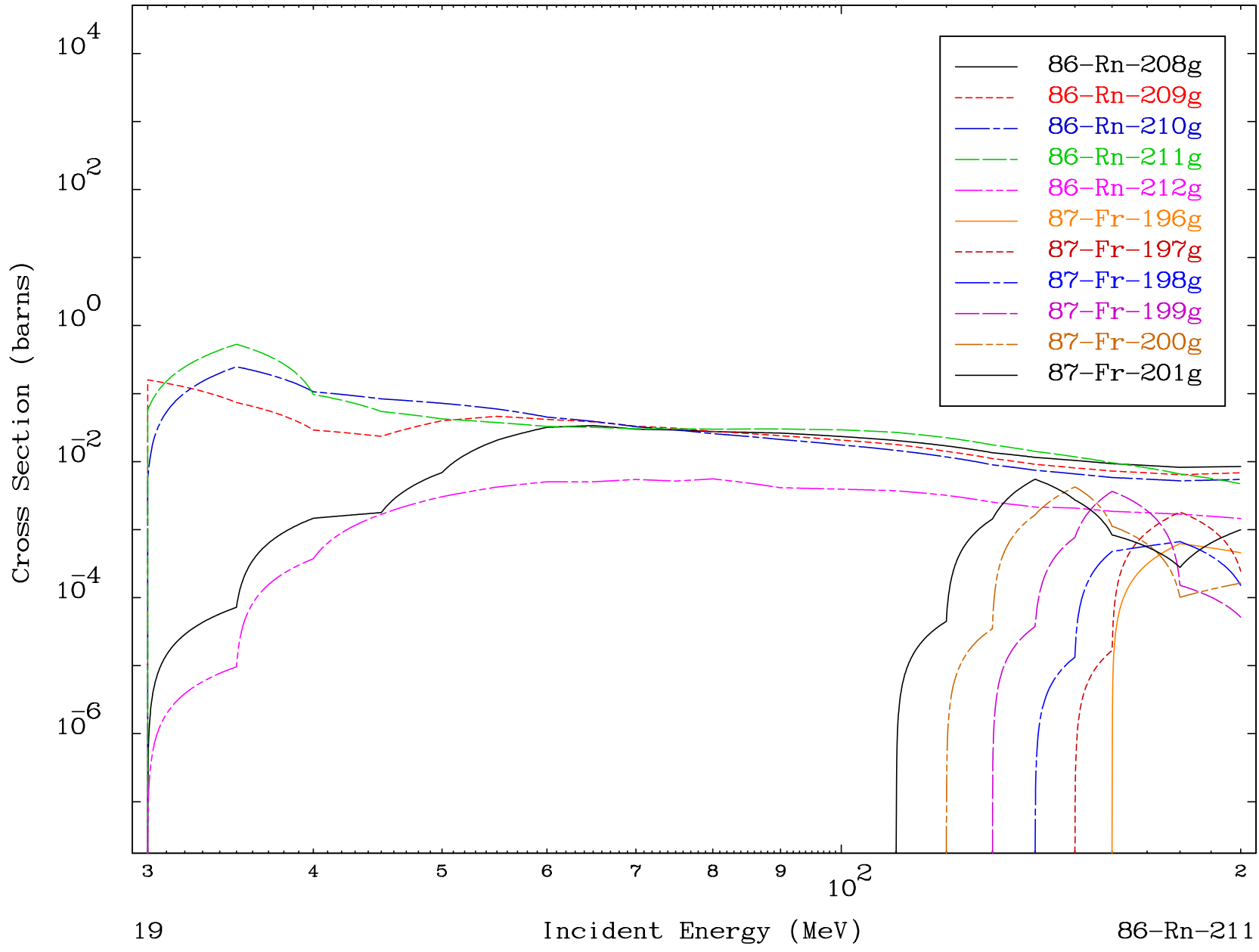


Radionuclide Production Cross Section







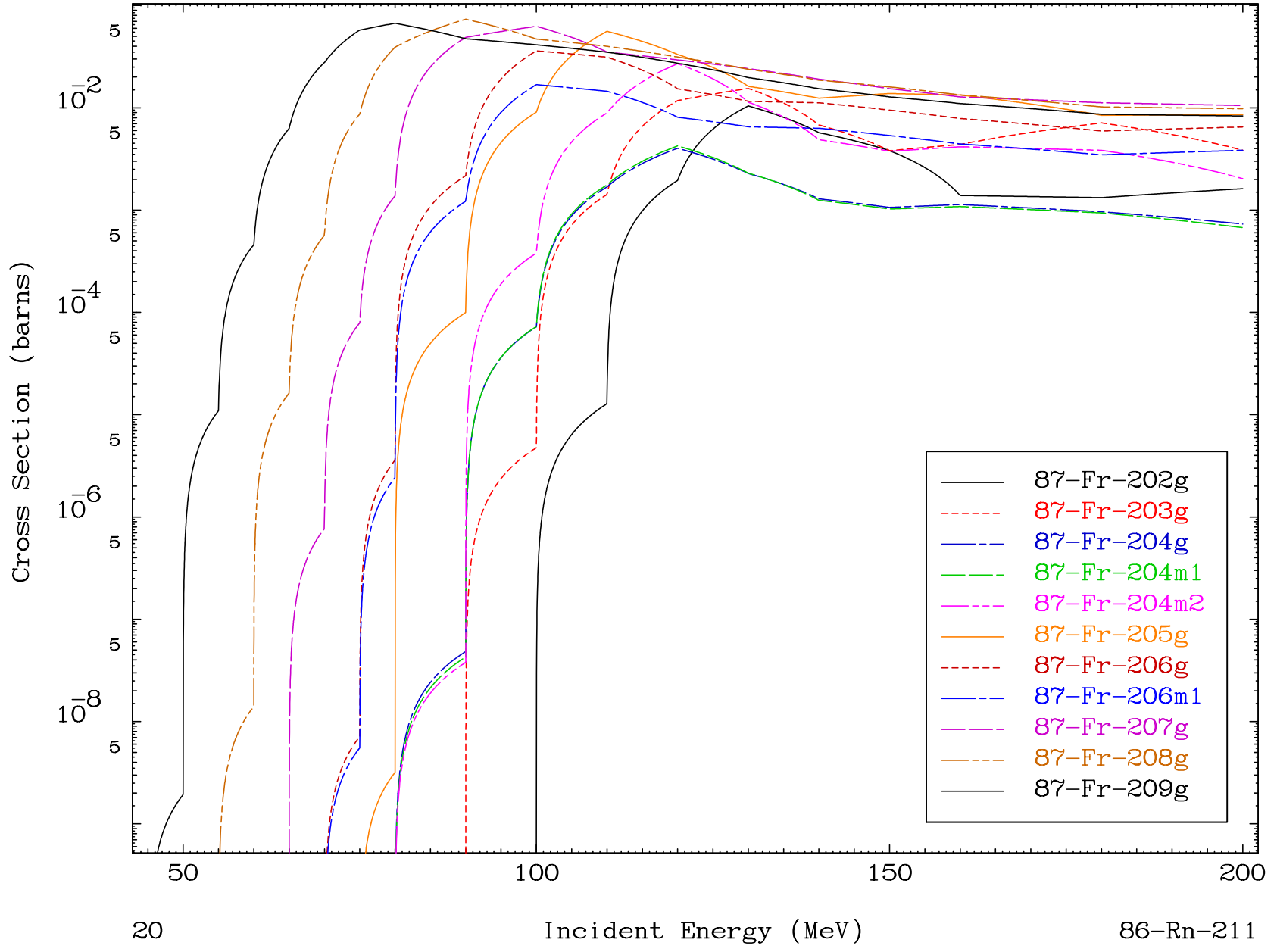


MAT 8625

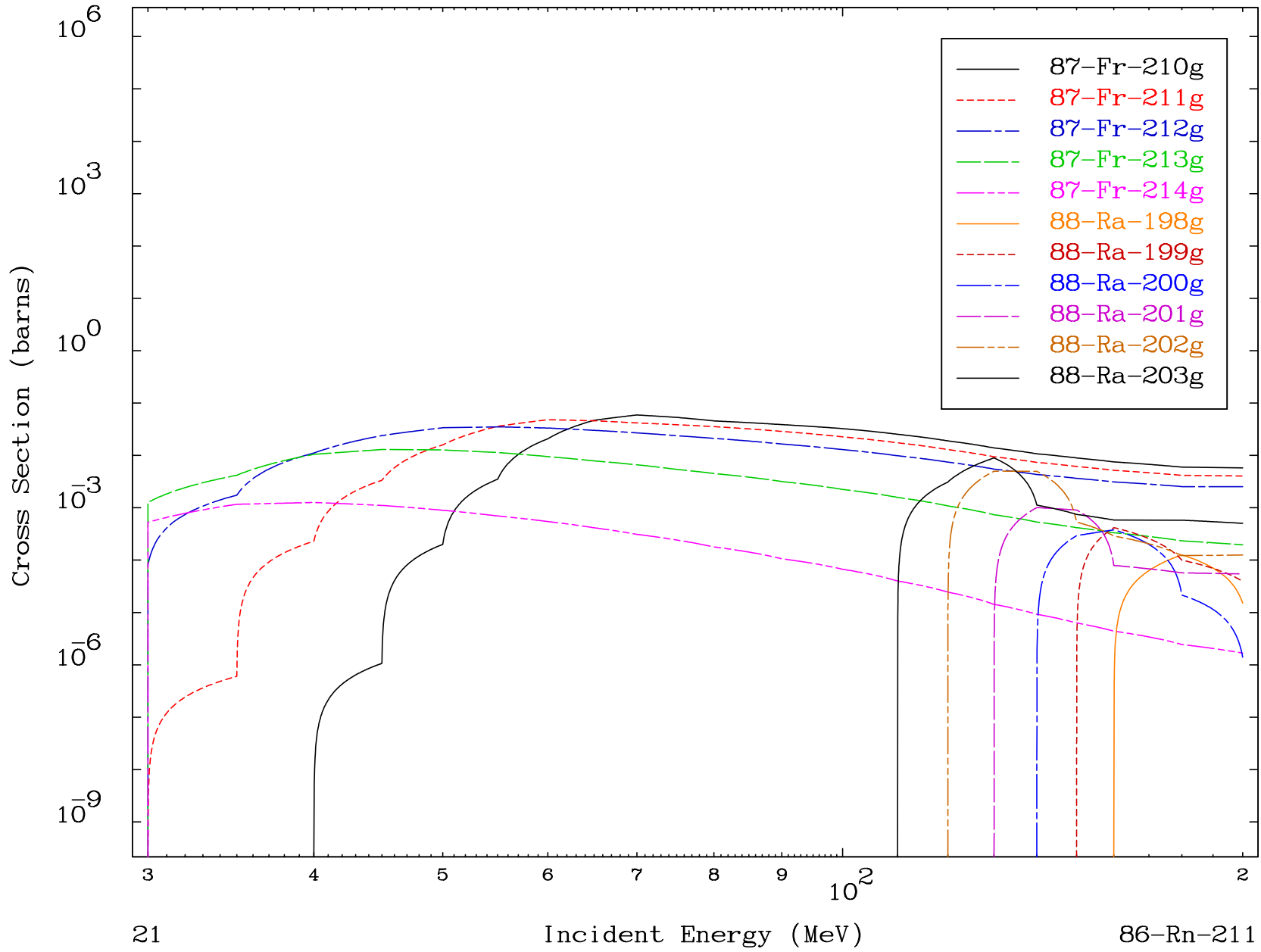
(α , remainder)

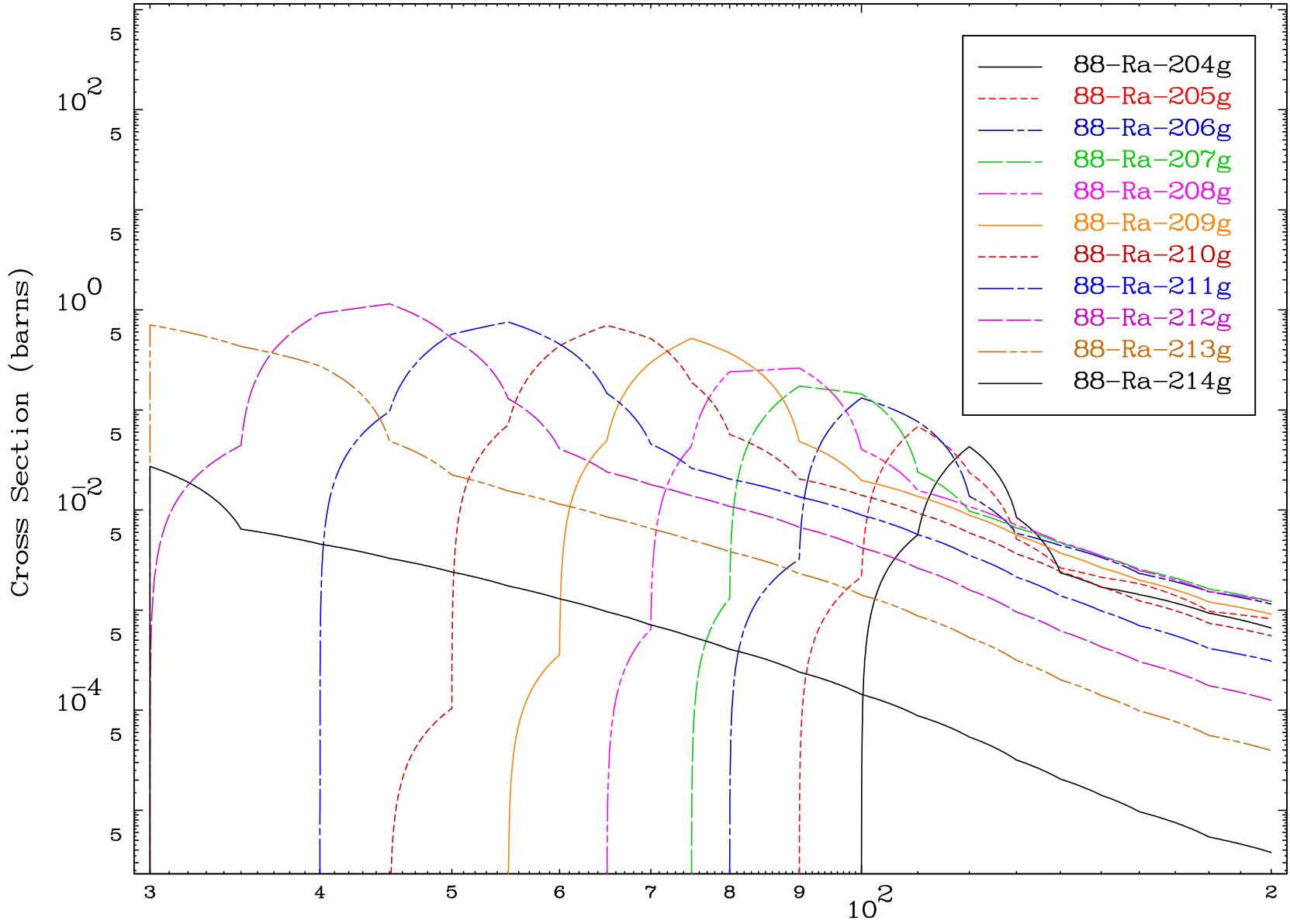
86-Rn-211

Radionuclide Production Cross Section



Radionuclide Production Cross Section



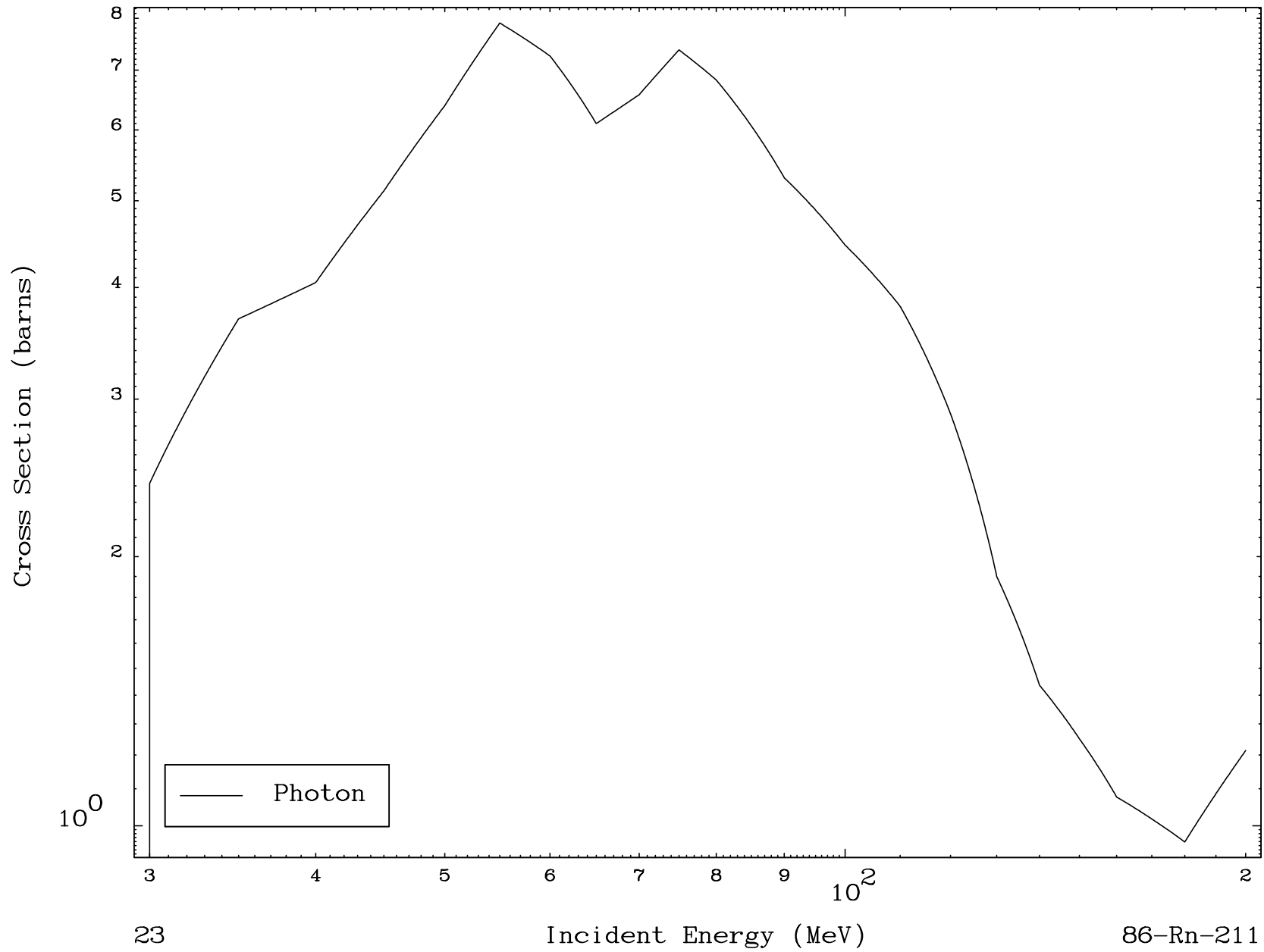


MAT 8625

(α , remainder)

86-Rn-211

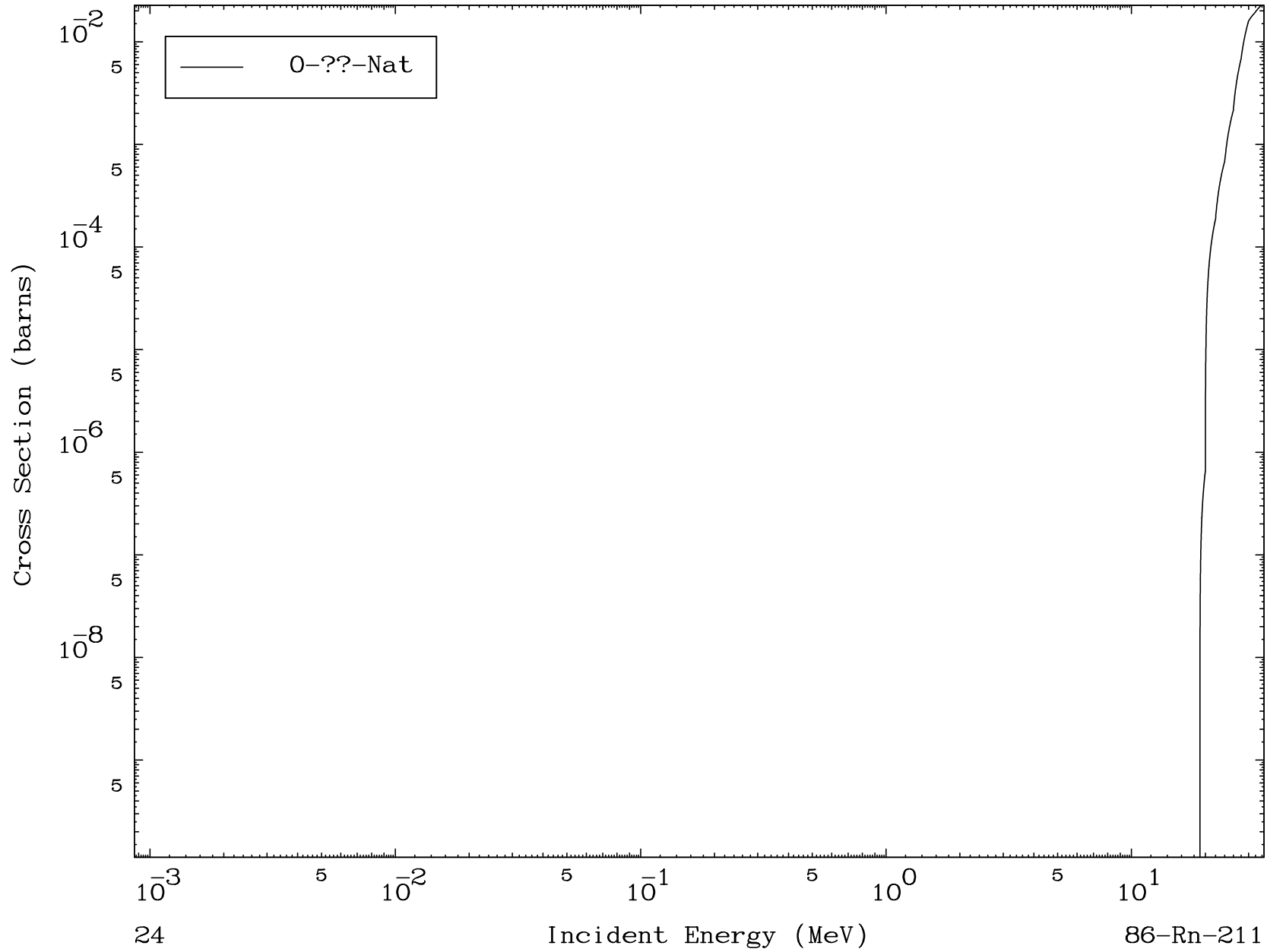
Radionuclide Production Cross Section



MAT 8625

α Fission
Radionuclide Production Cross Section

86-Rn-211



24

Incident Energy (MeV)

86-Rn-211

MAT 8625

($\alpha, 2\alpha$)

86-Rn-211

Radionuclide Production Cross Section

