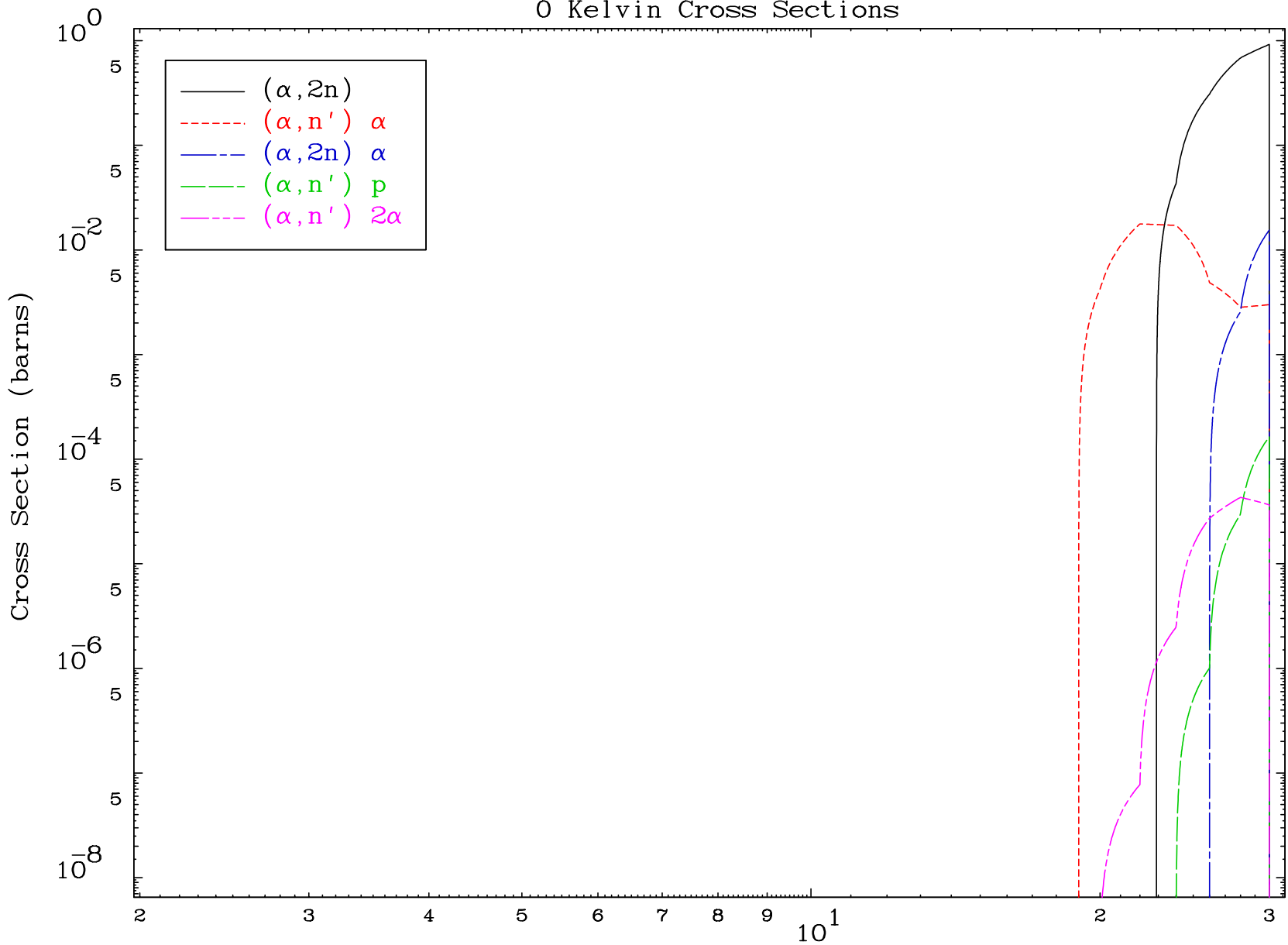


MAT 8628

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

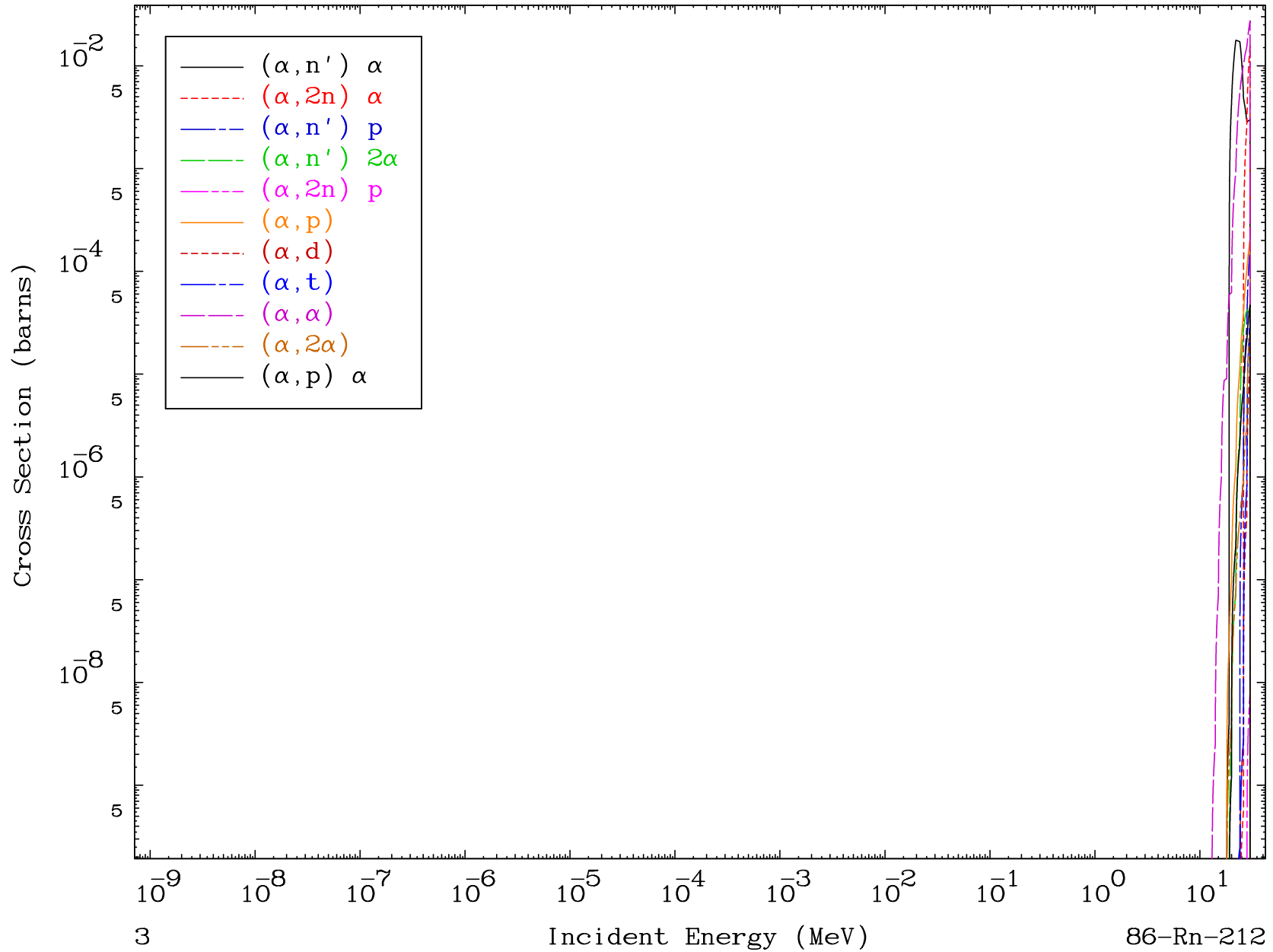
86-Rn-212



2

Incident Energy (MeV)

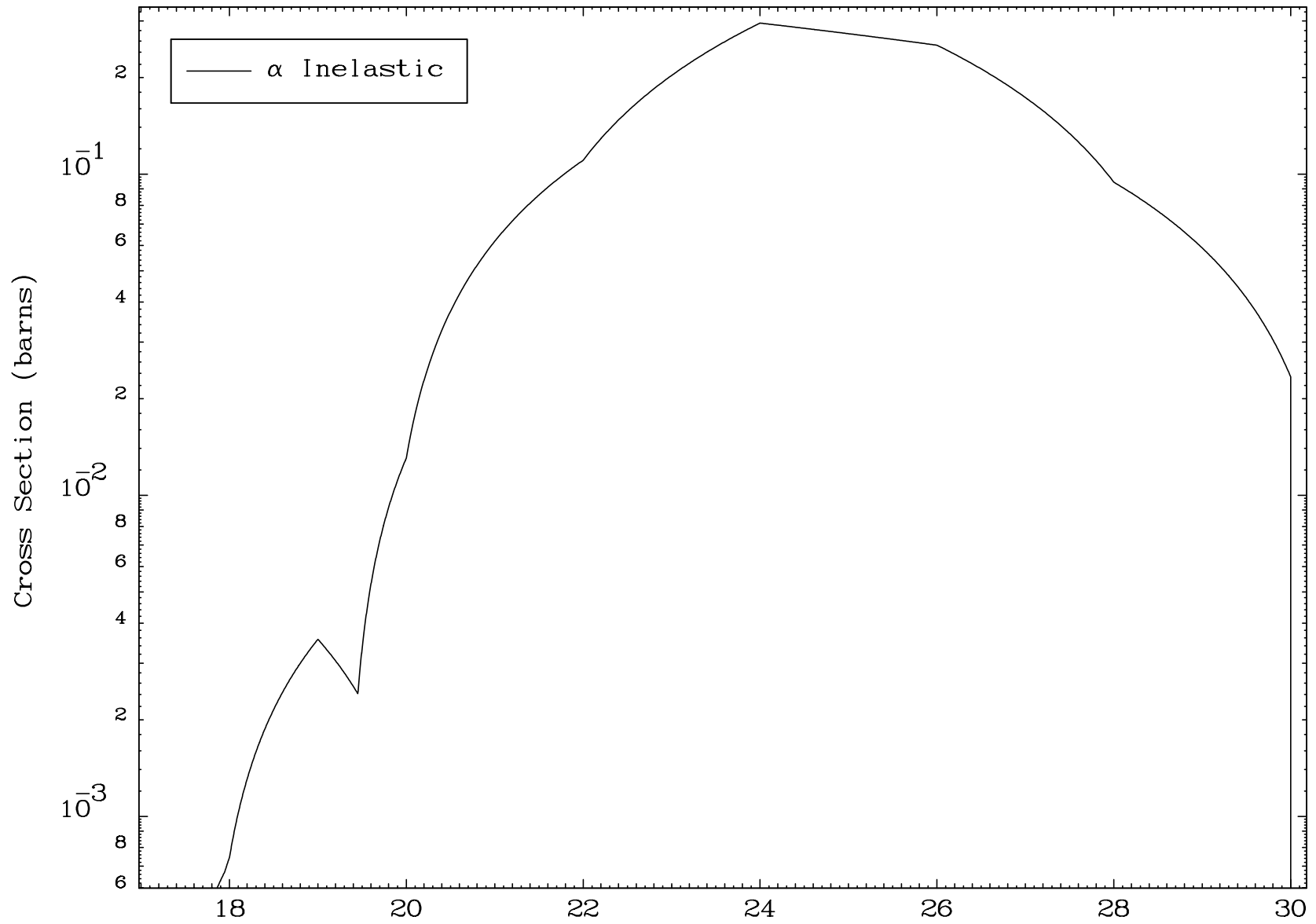
86-Rn-212



MAT 8628

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

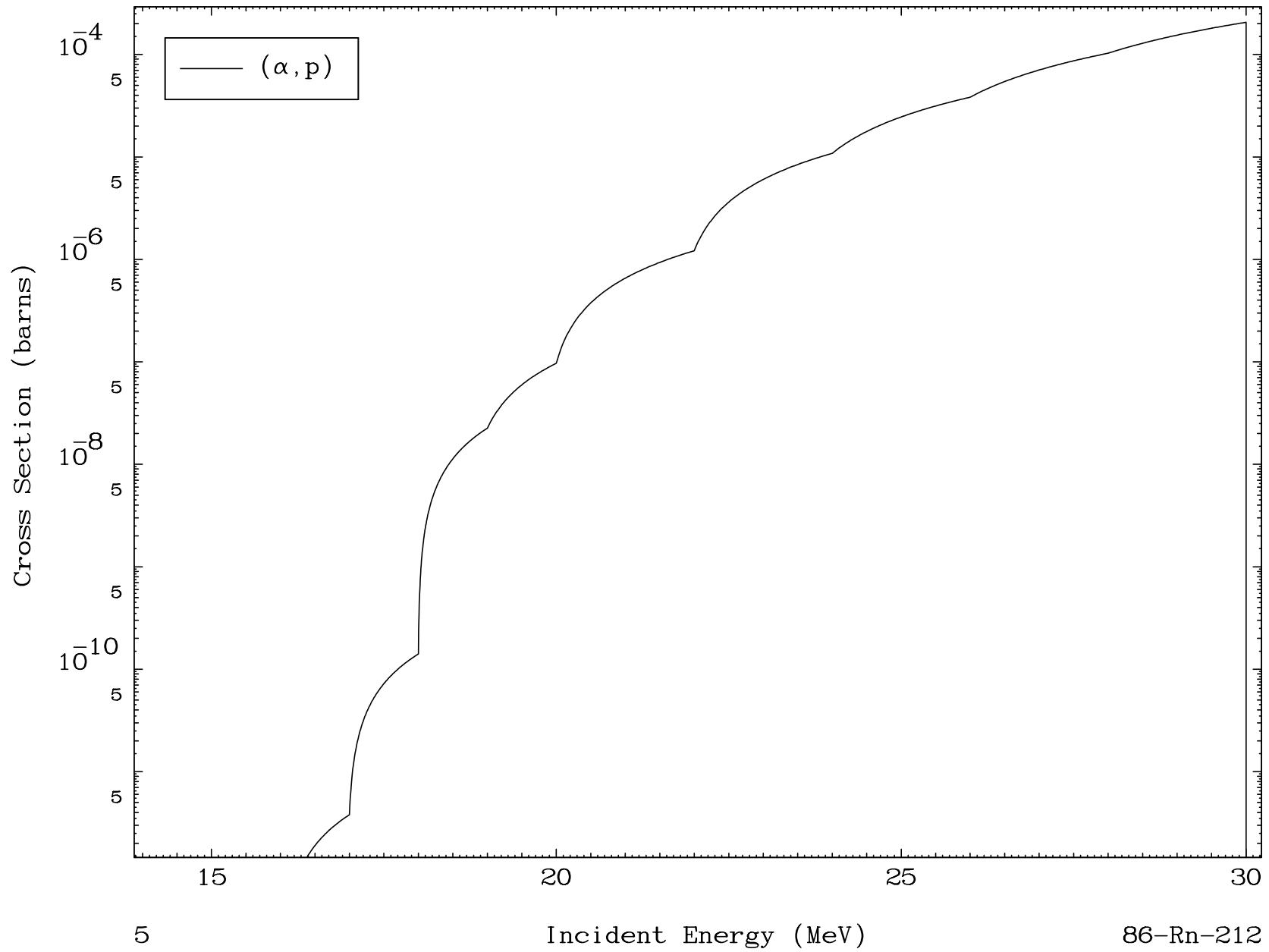
86-Rn-212

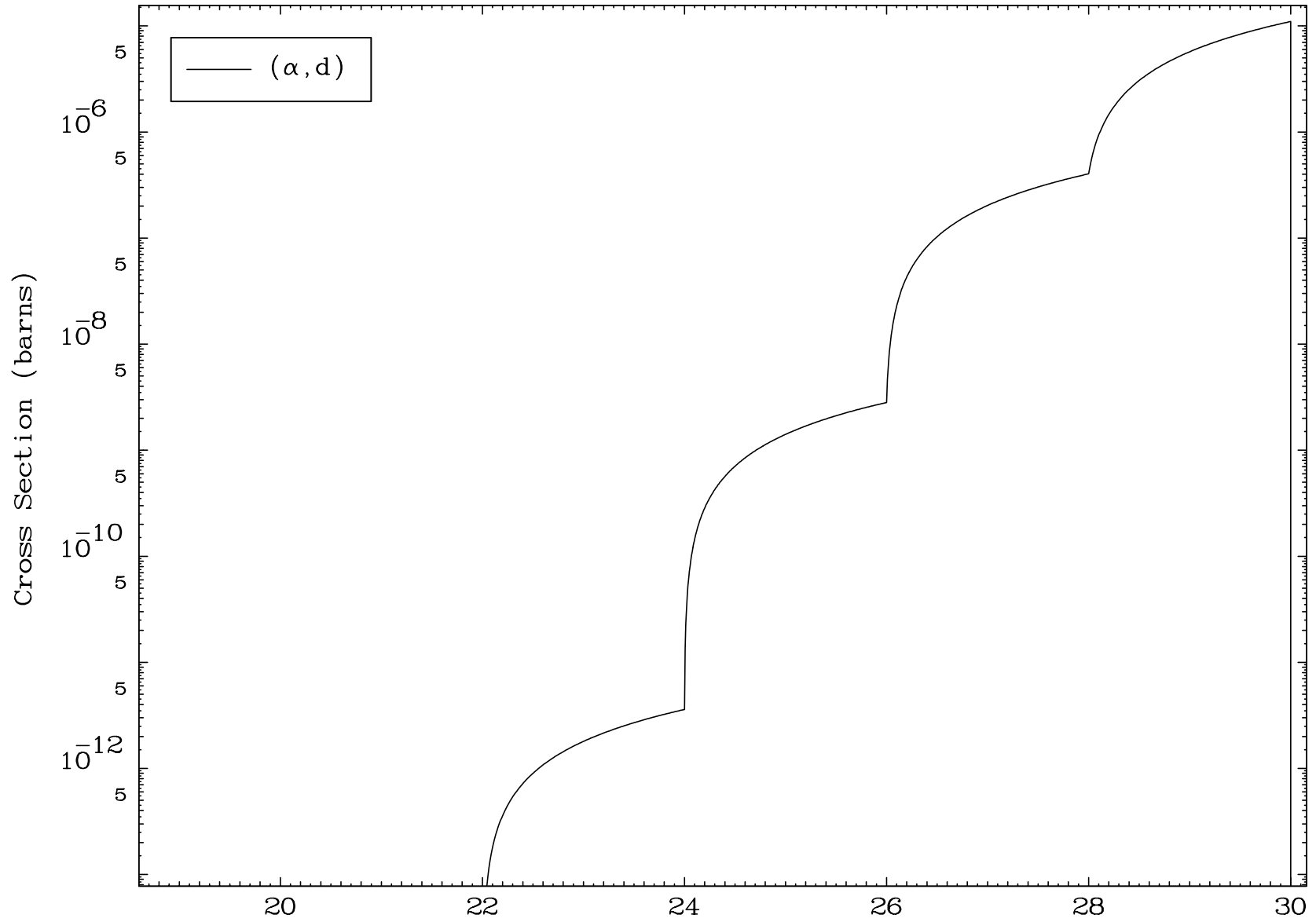


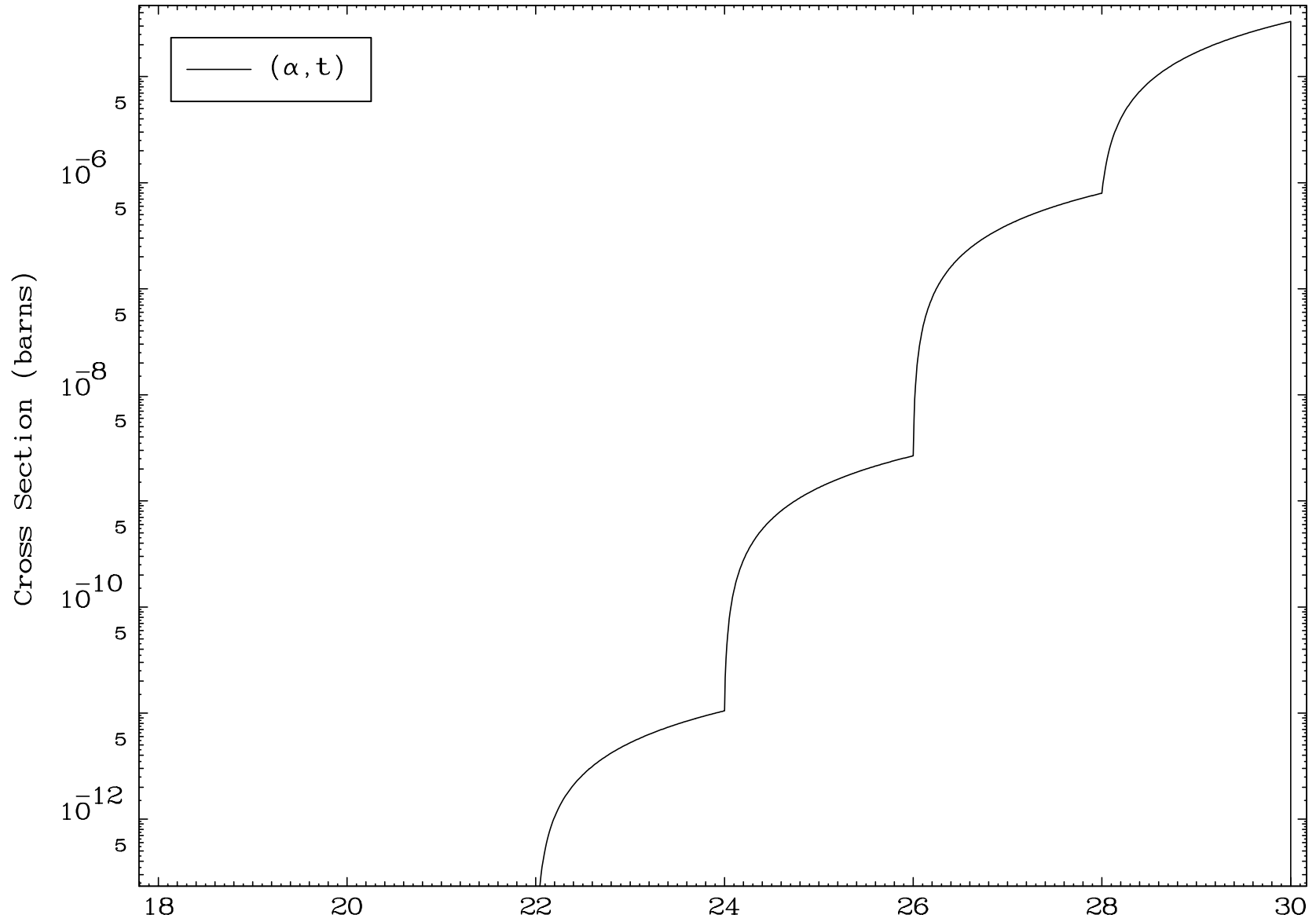
4

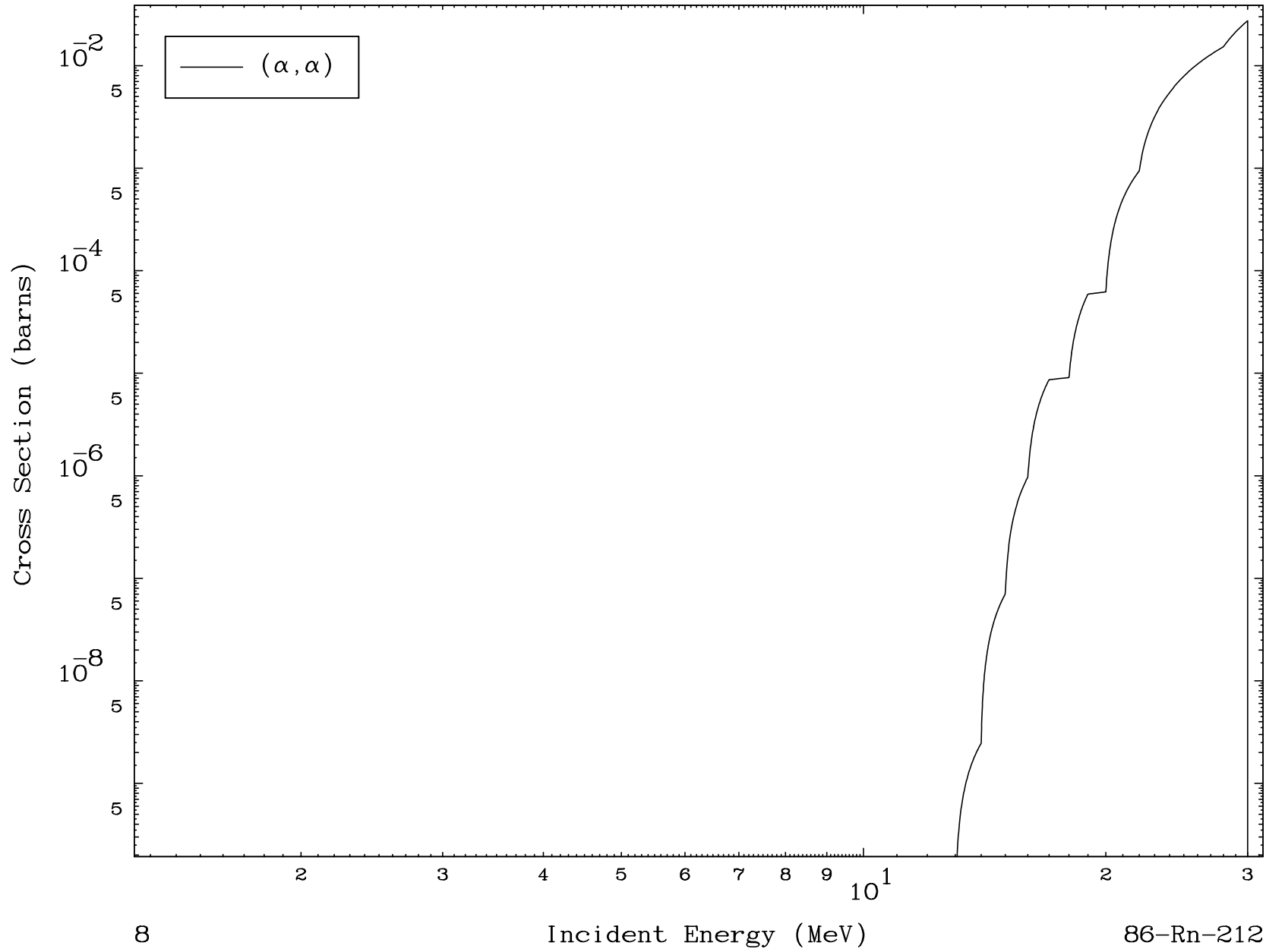
Incident Energy (MeV)

86-Rn-212



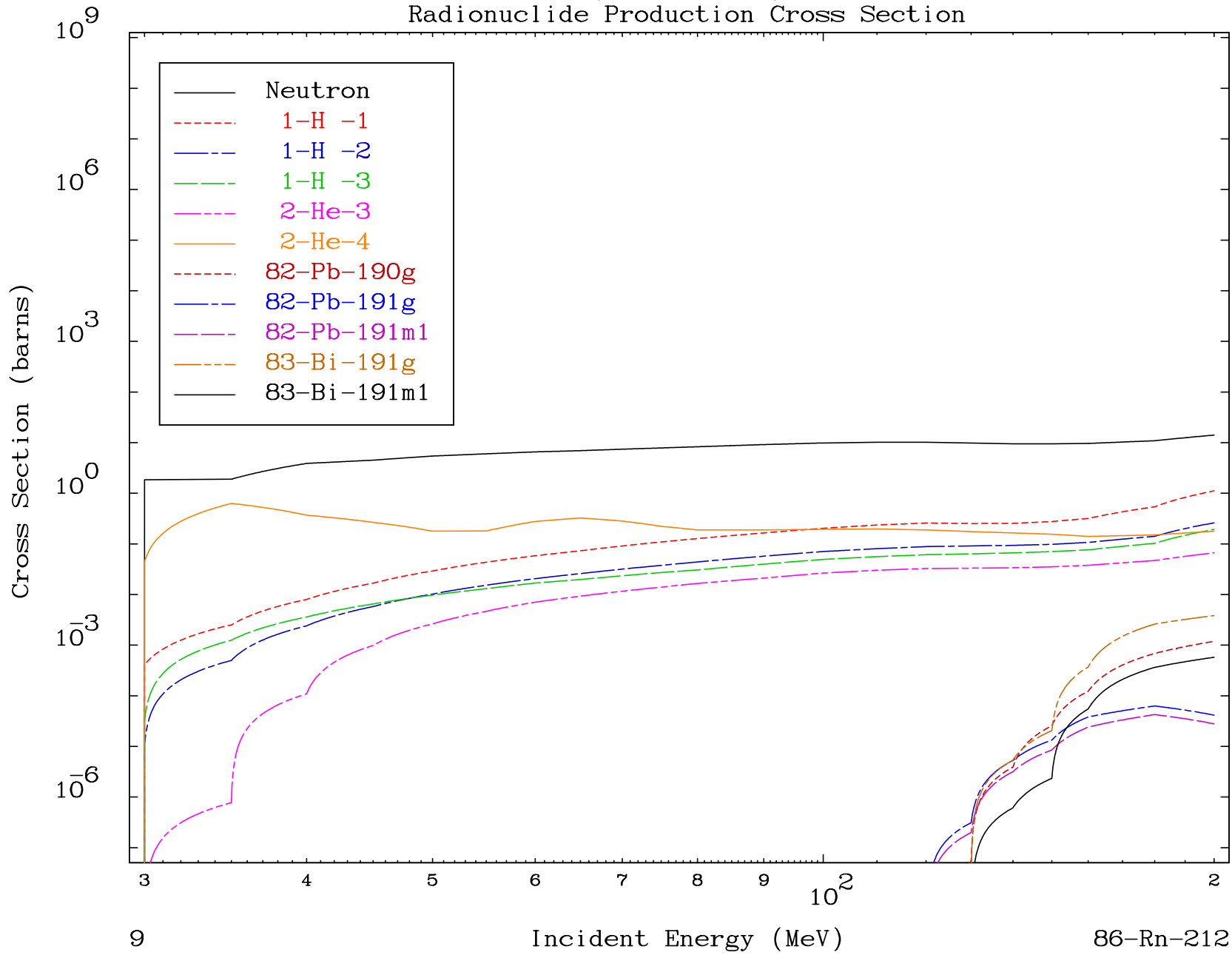








Radionuclide Production Cross Section

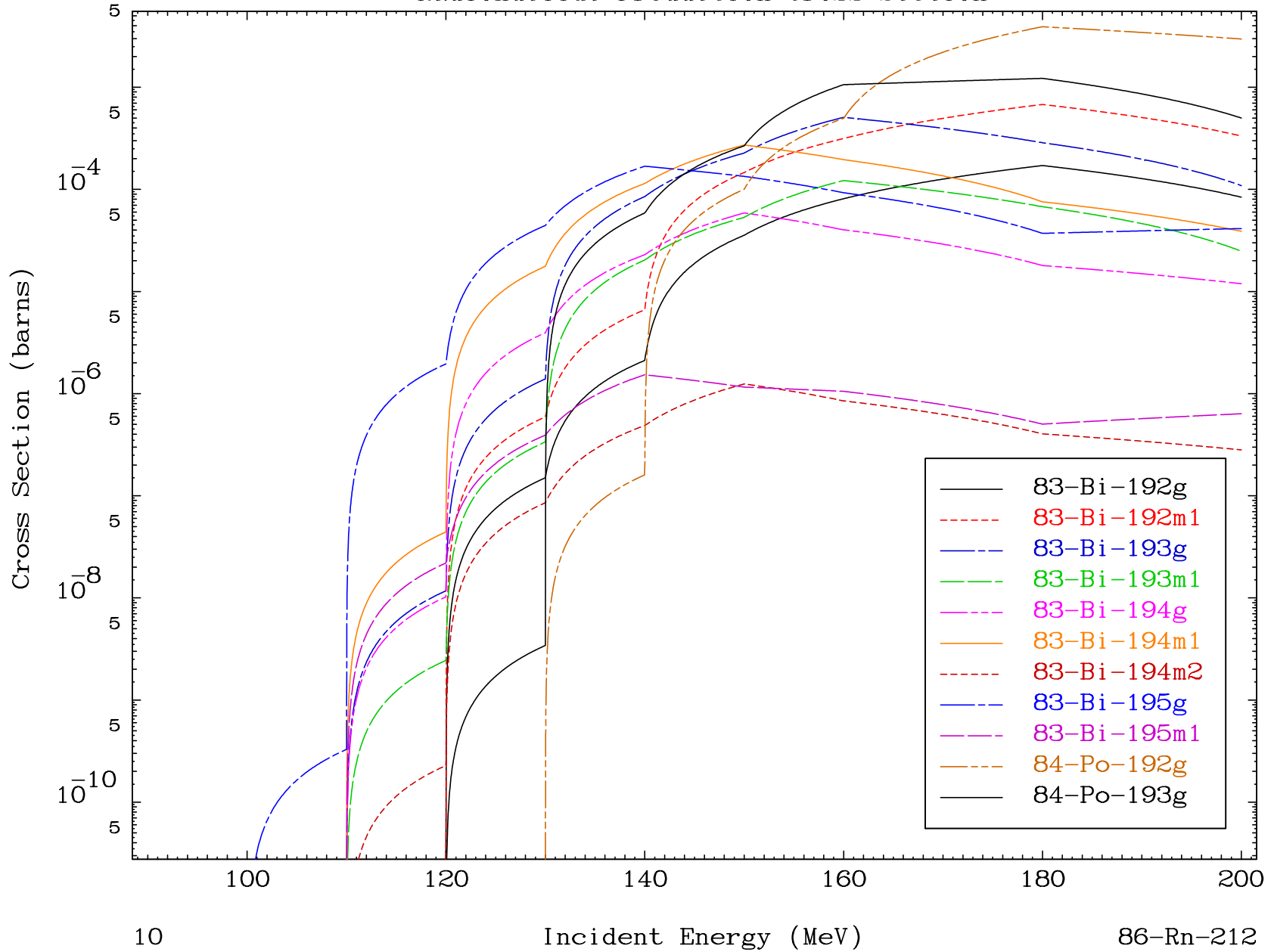


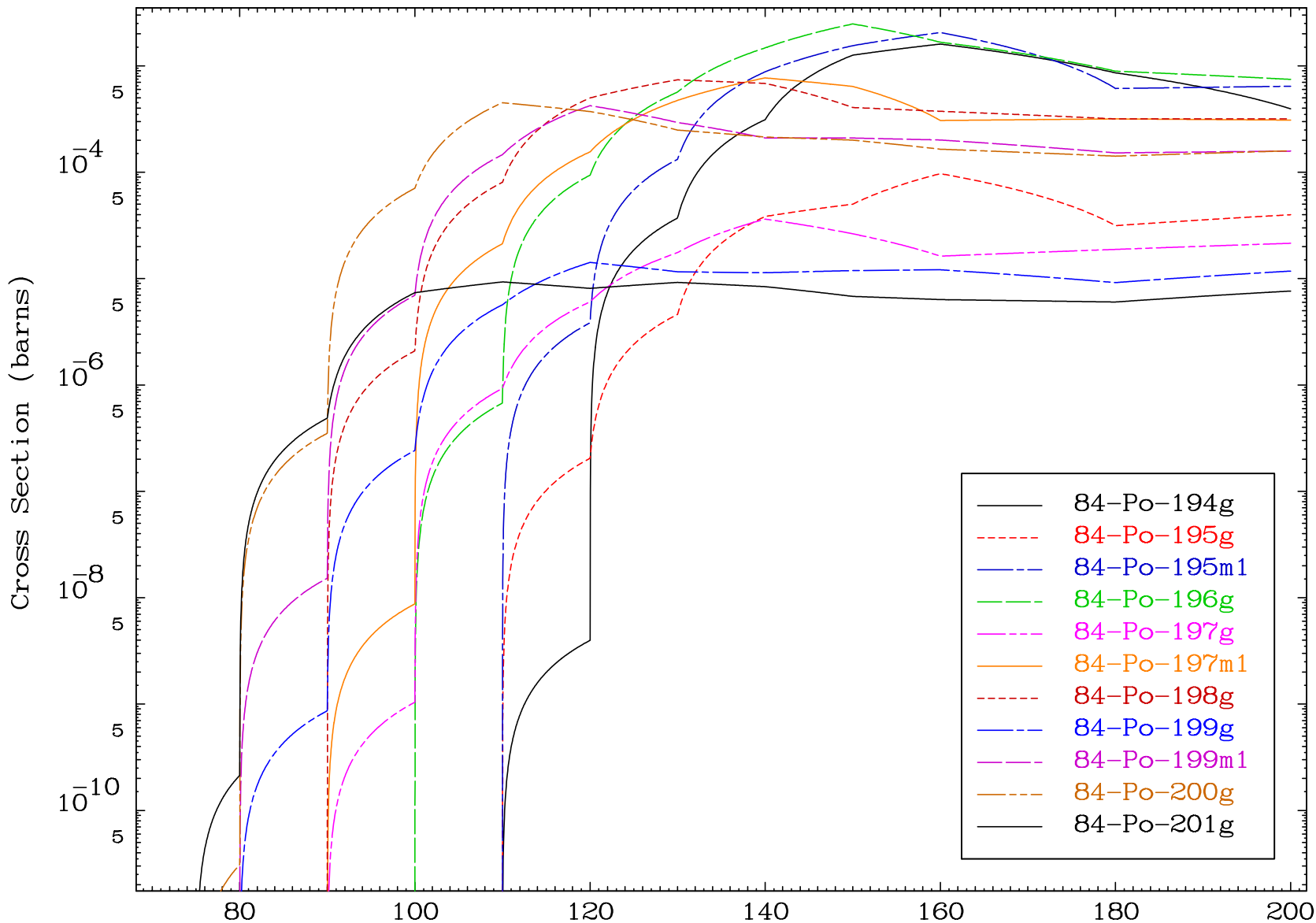
MAT 8628

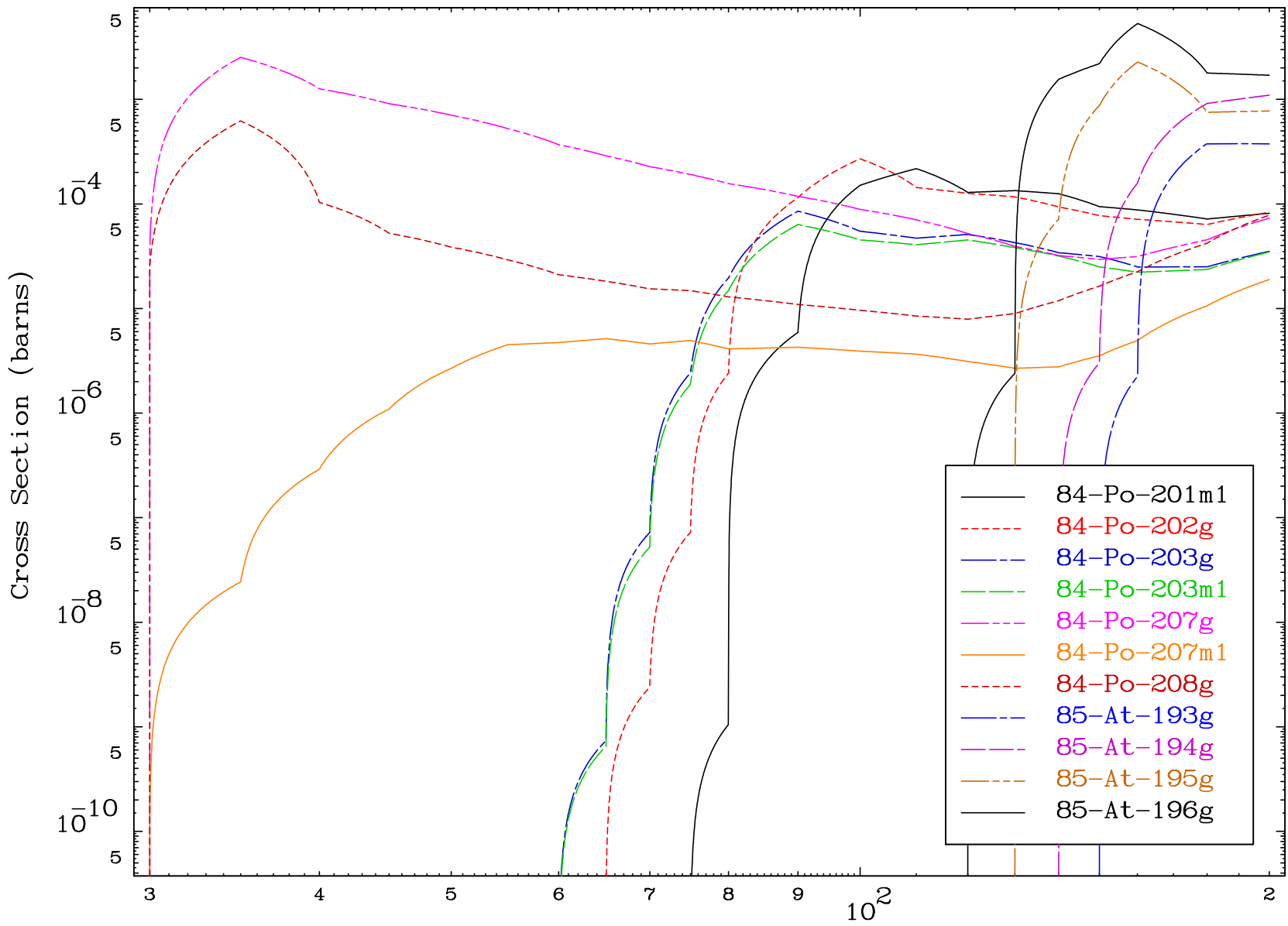
( $\alpha$ , remainder)

86-Rn-212

### Radionuclide Production Cross Section





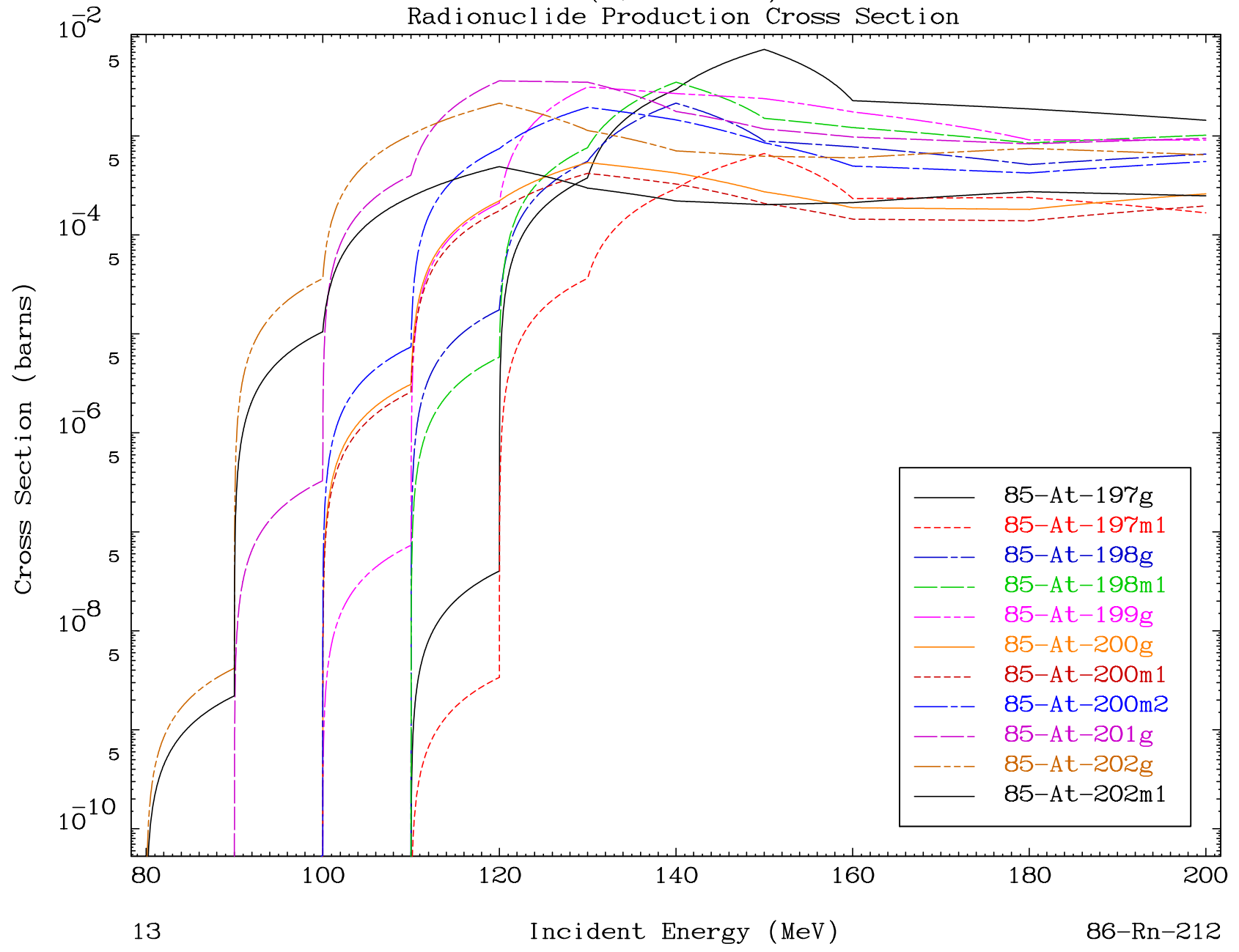


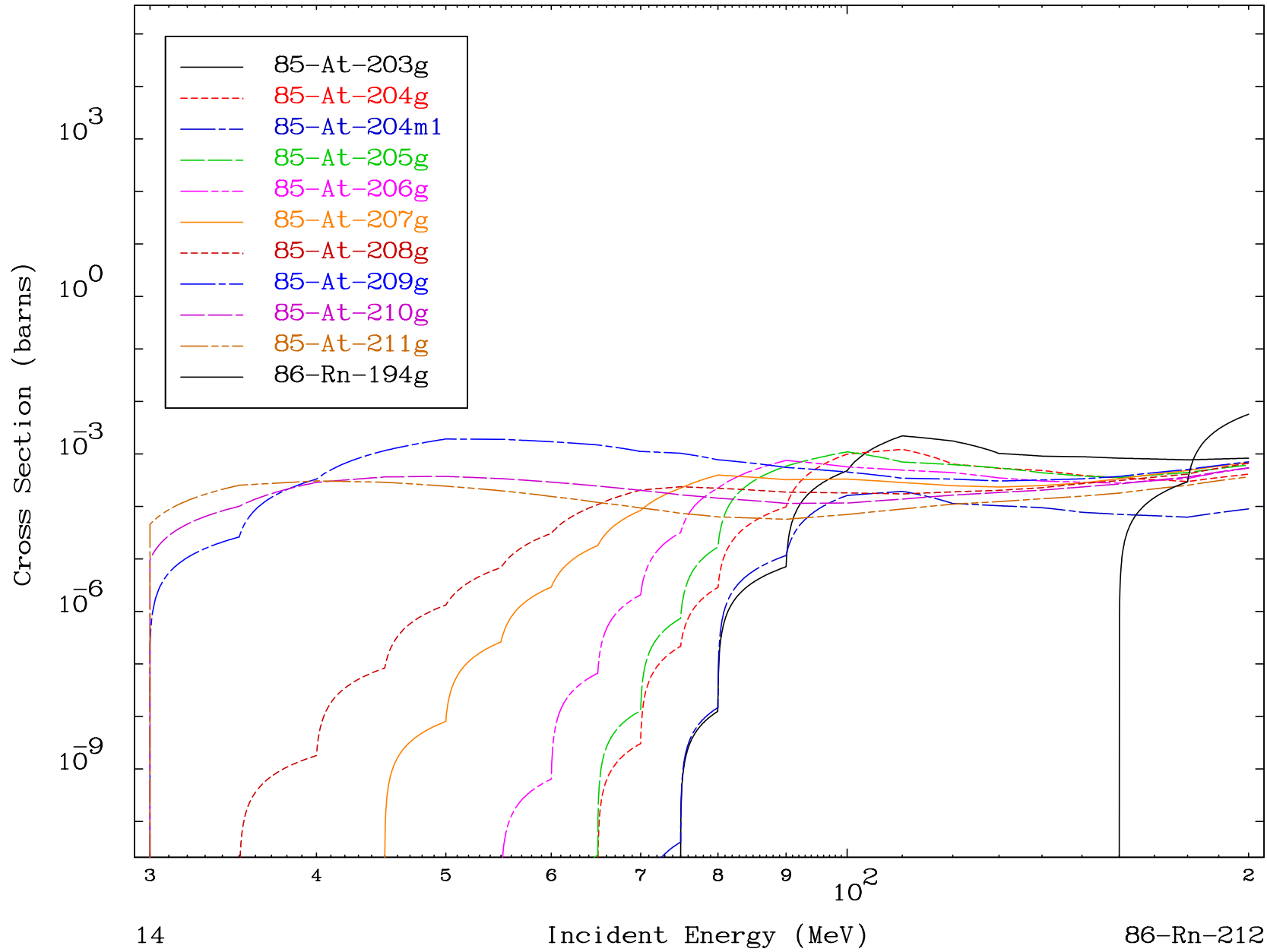
MAT 8628

( $\alpha$ , remainder)

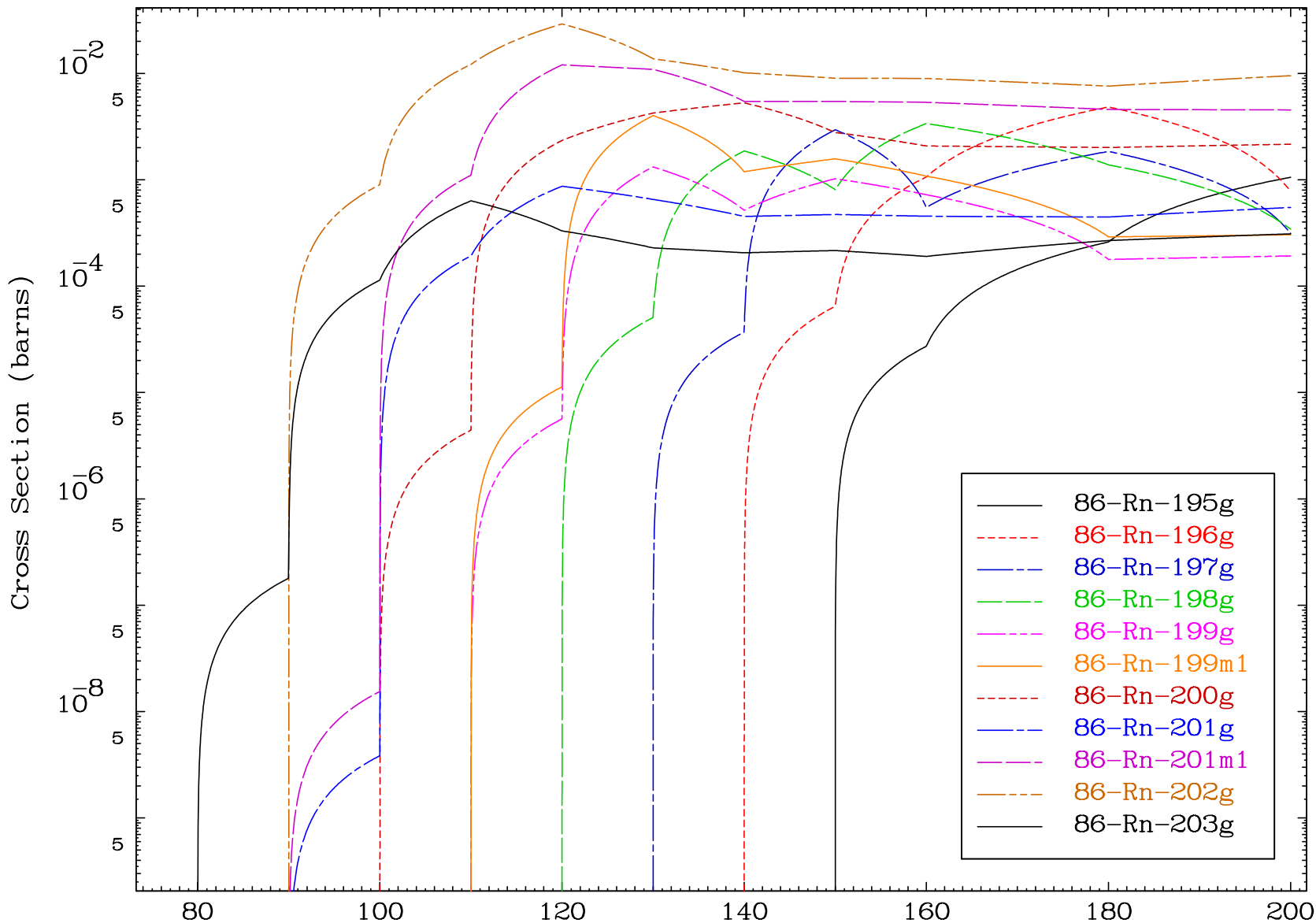
86-Rn-212

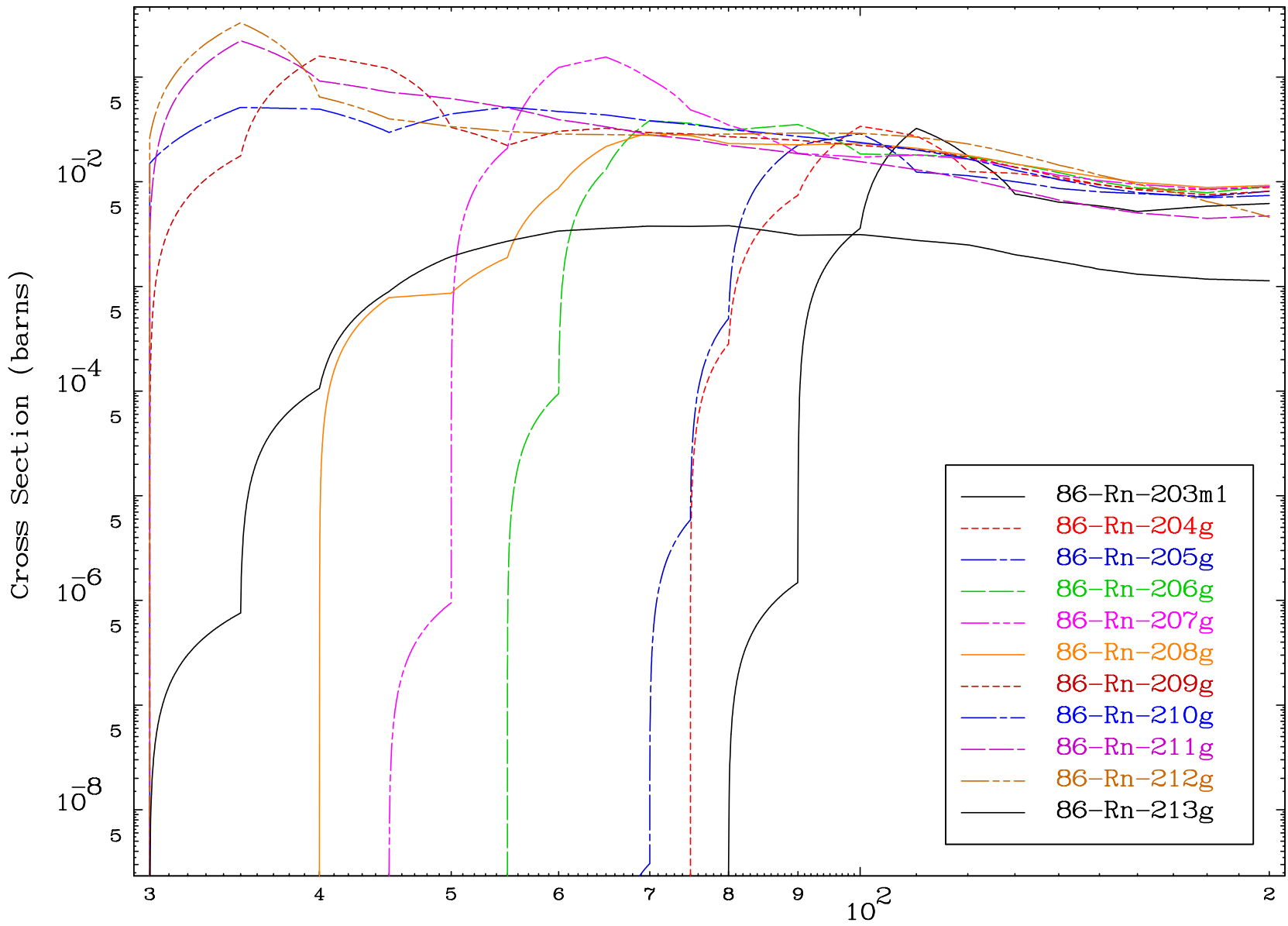
### Radionuclide Production Cross Section





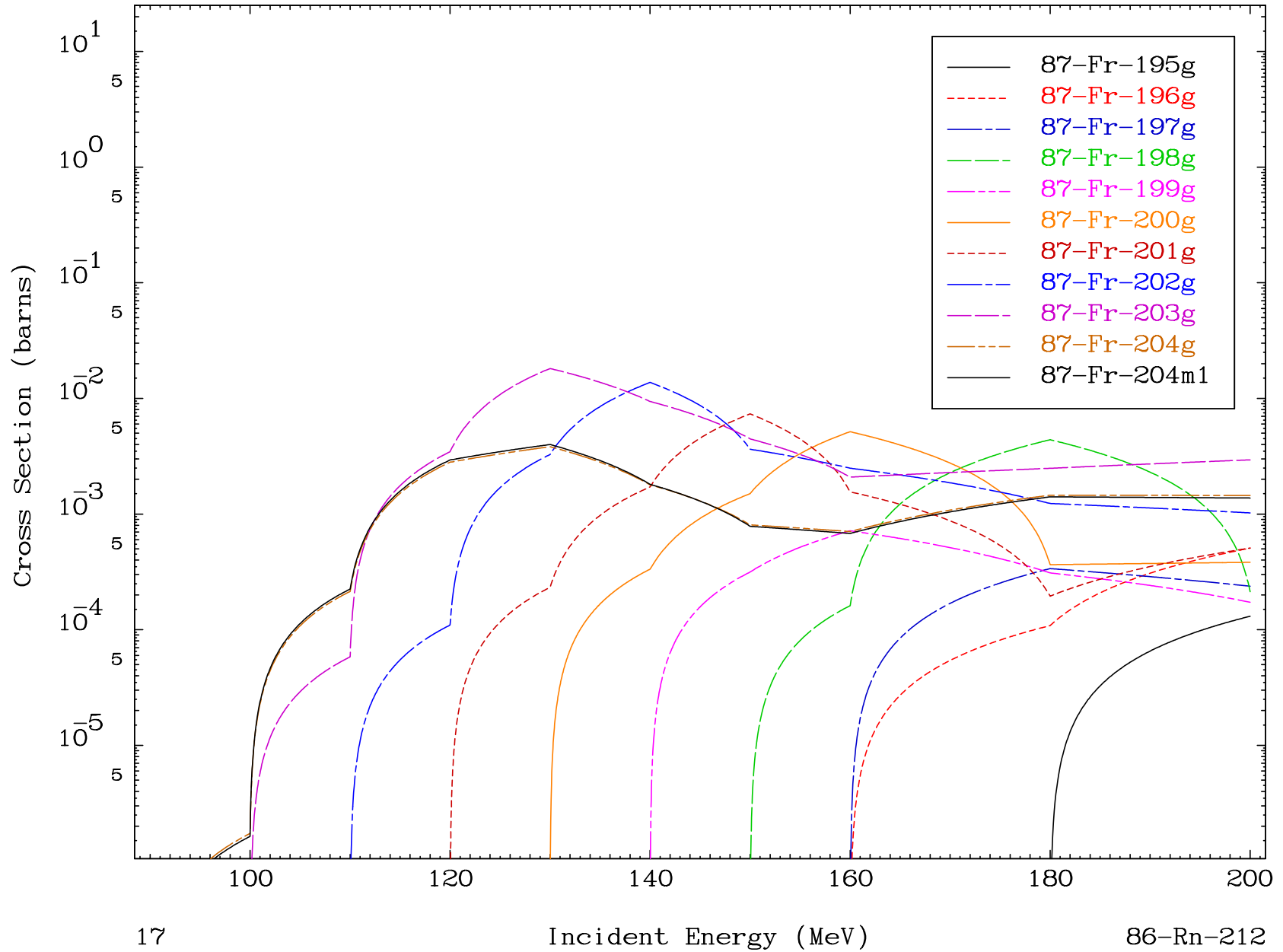
Radionuclide Production Cross Section



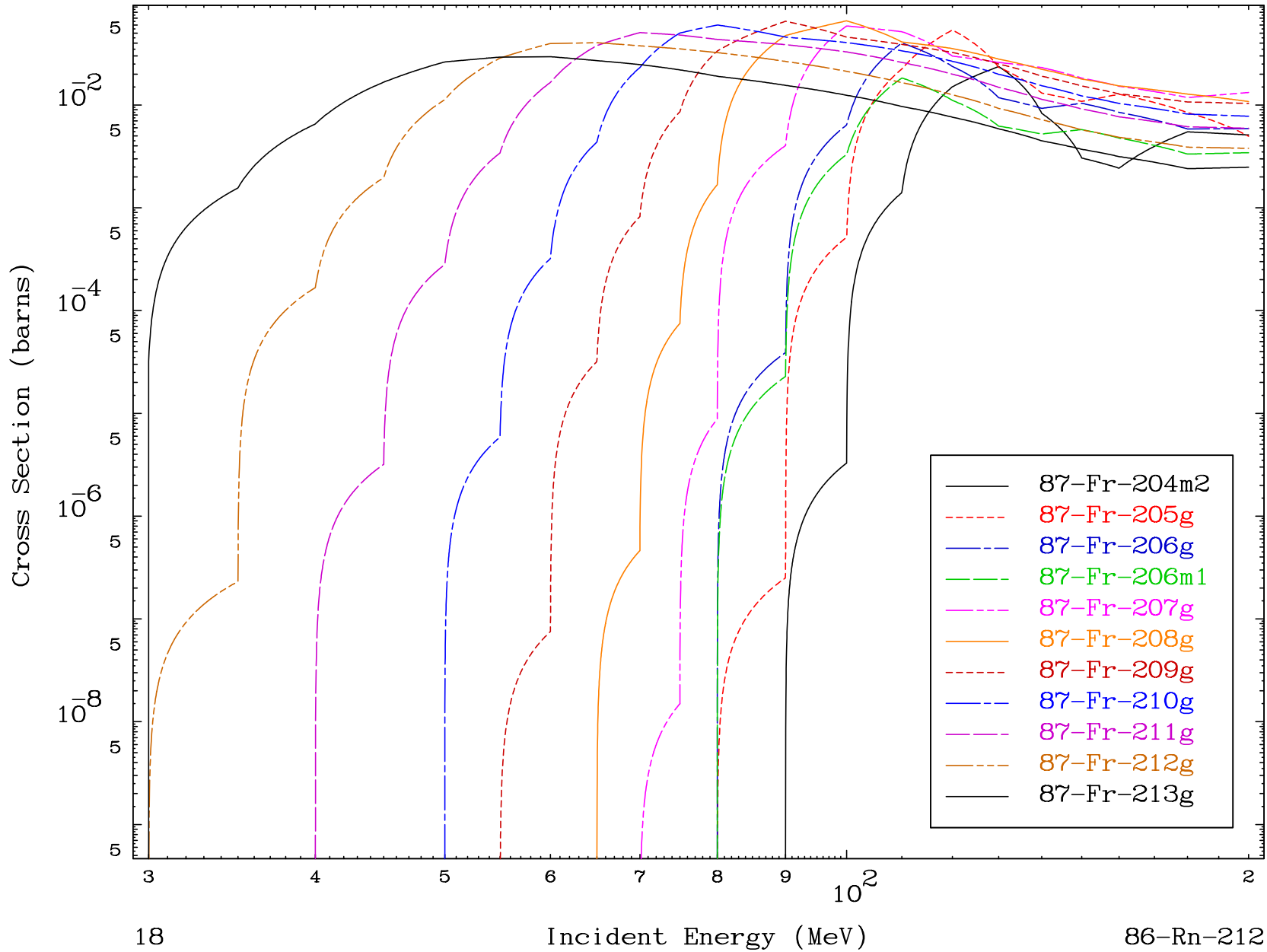




Radionuclide Production Cross Section



Radionuclide Production Cross Section

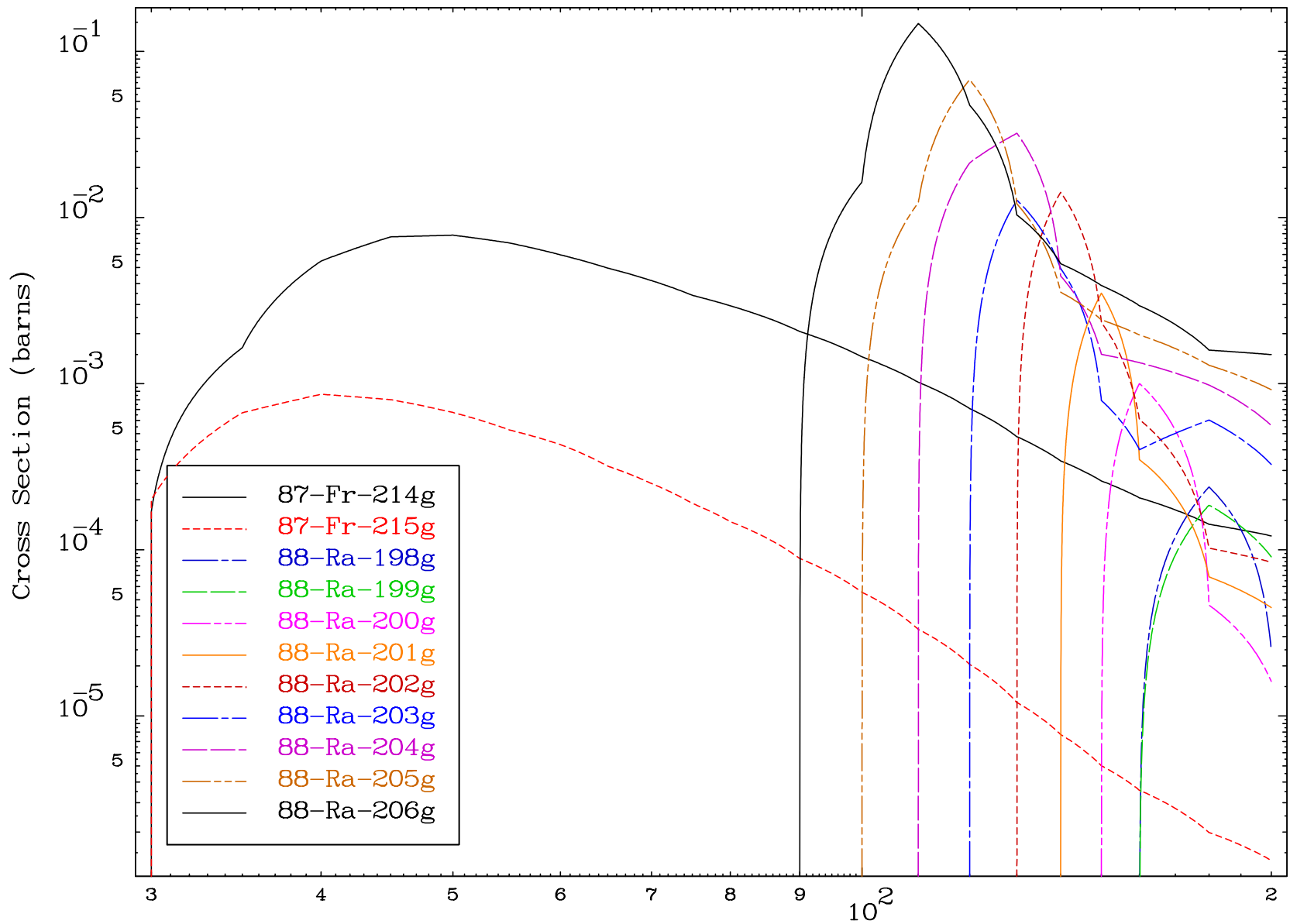


MAT 8628

( $\alpha$ , remainder)

86-Rn-212

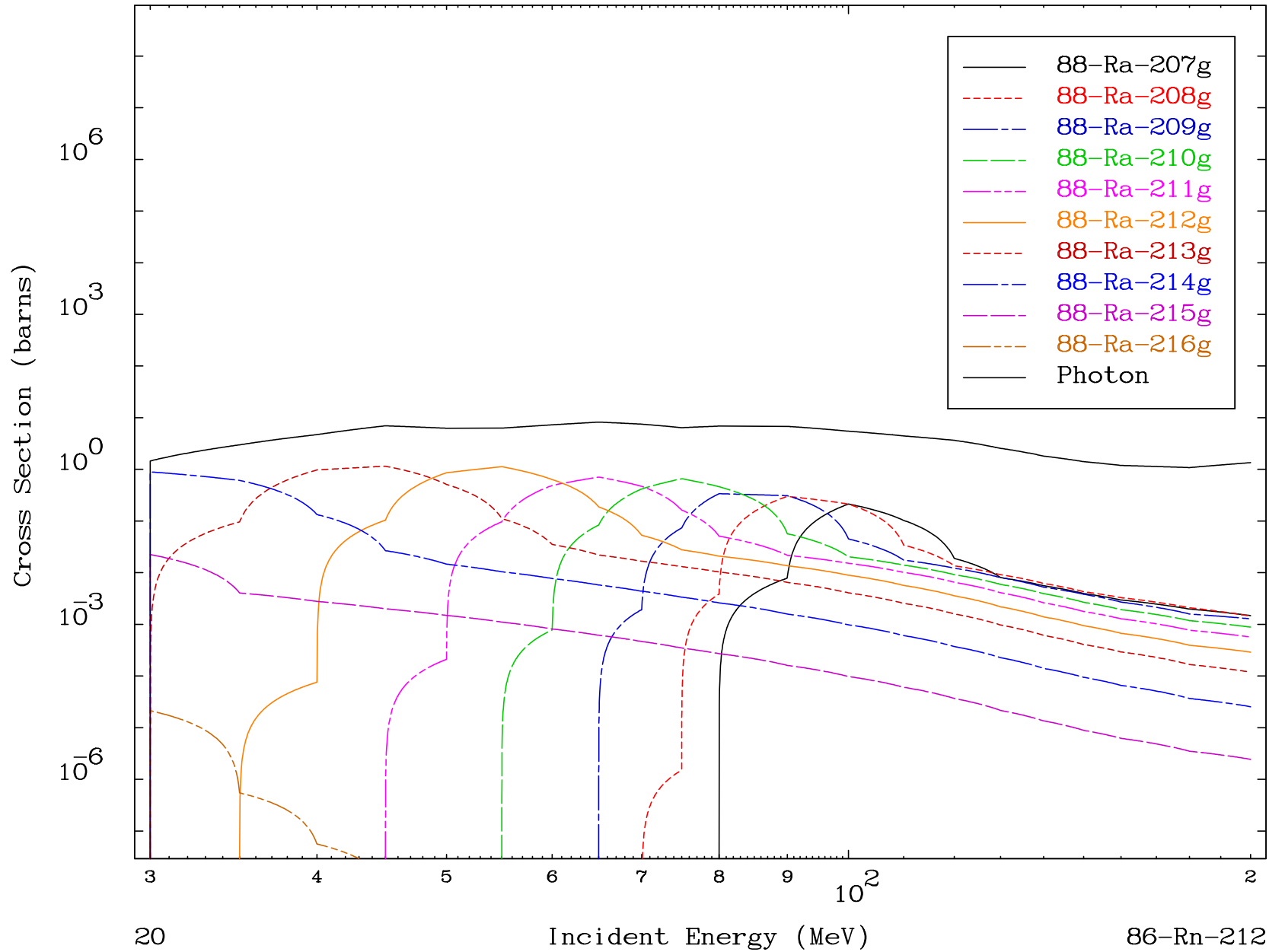
### Radionuclide Production Cross Section



19

Incident Energy (MeV)

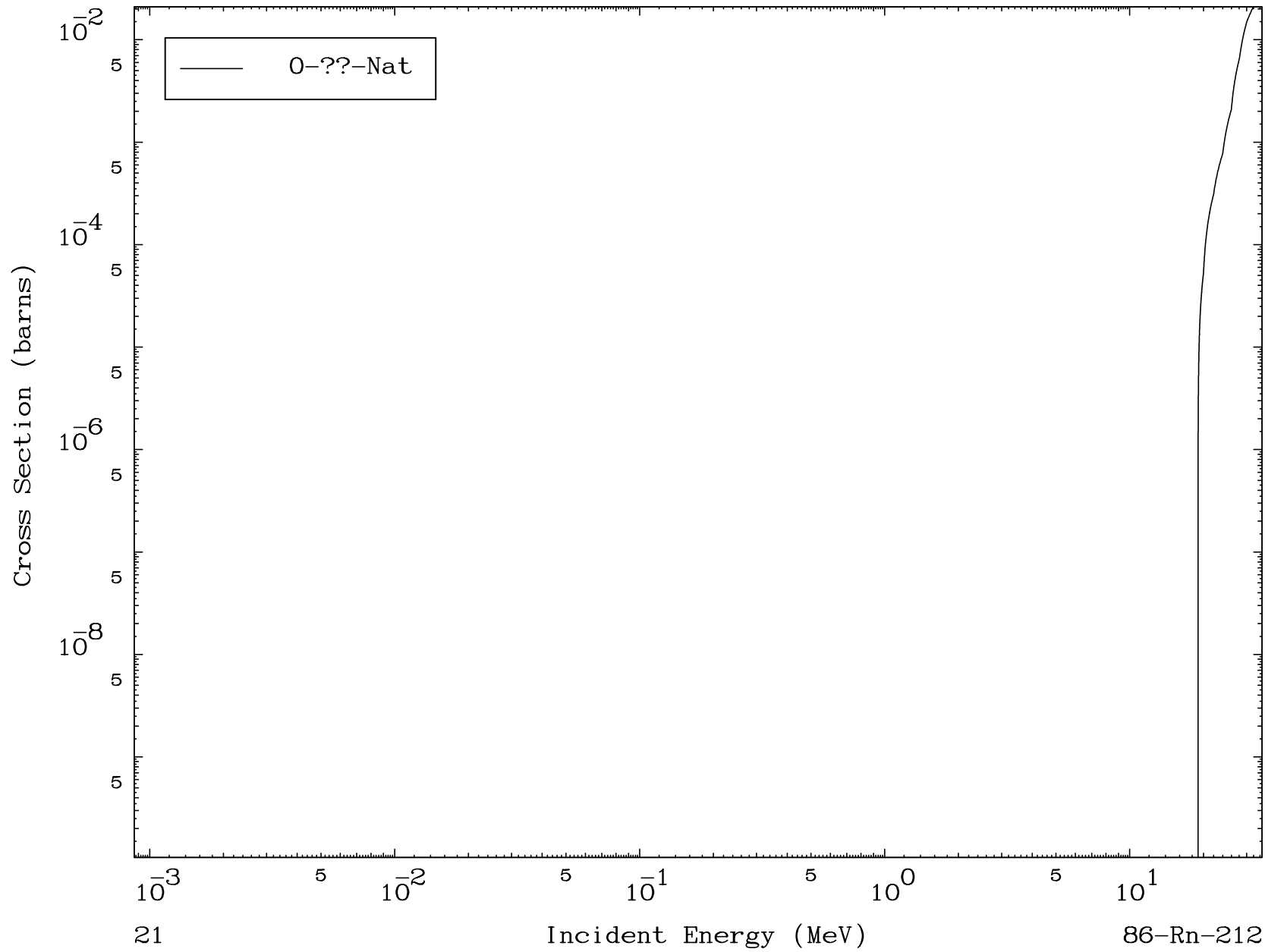
86-Rn-212



MAT 8628

$\alpha$  Fission  
Radionuclide Production Cross Section

86-Rn-212



MAT 8628

( $\alpha, n'$ )  $2\alpha$

86-Rn-212

Radionuclide Production Cross Section

