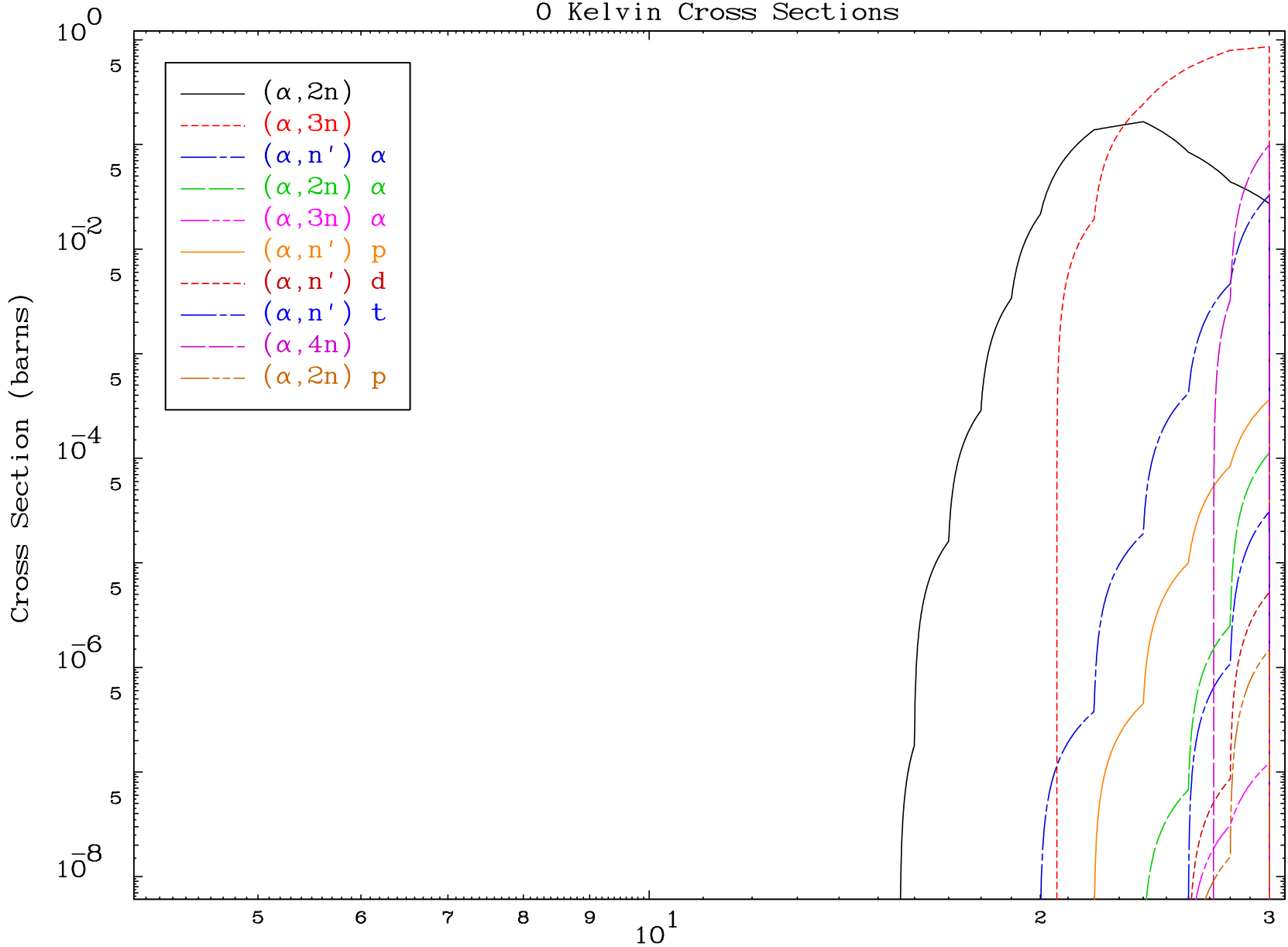


MAT 8661

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

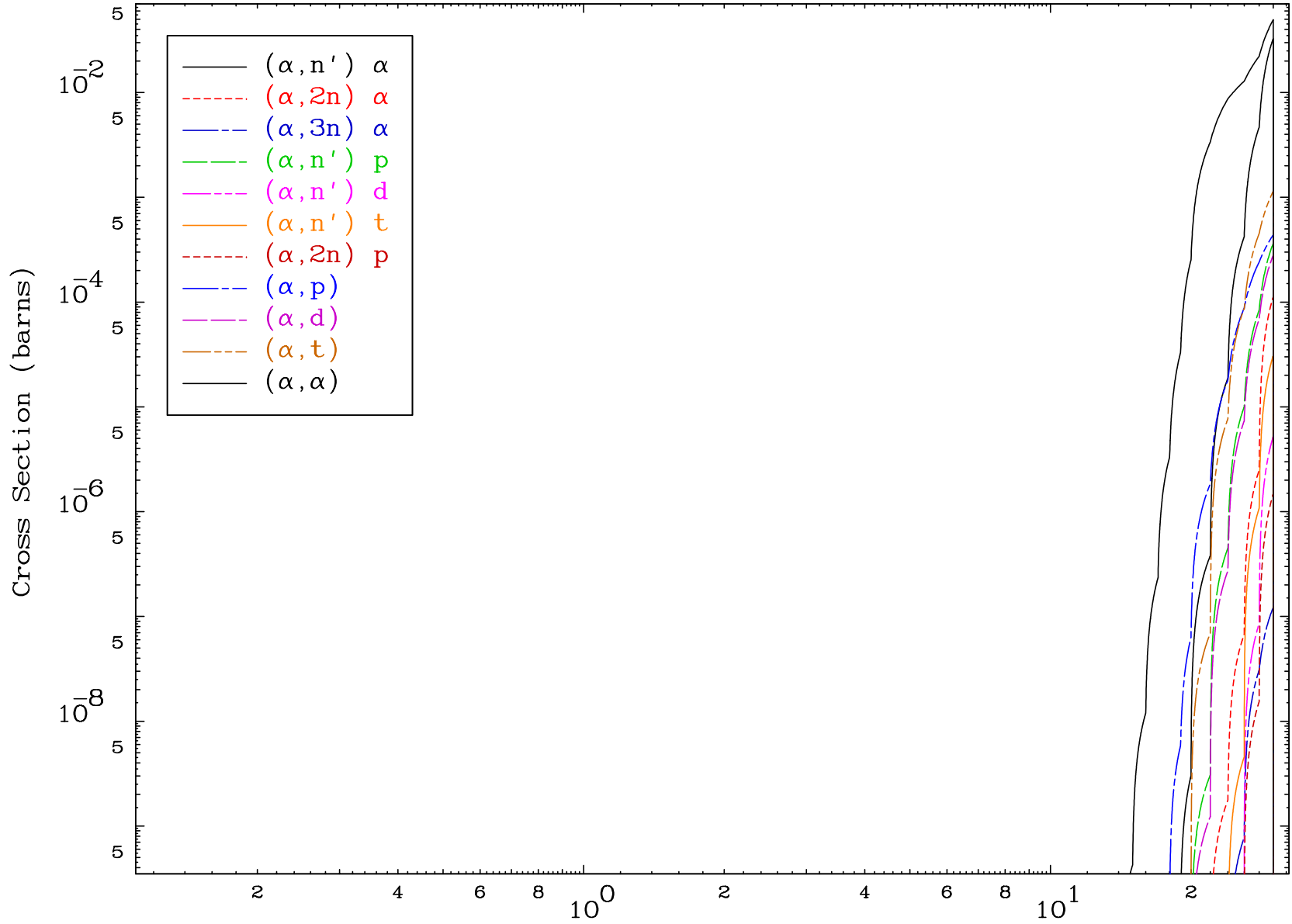
86-Rn-223

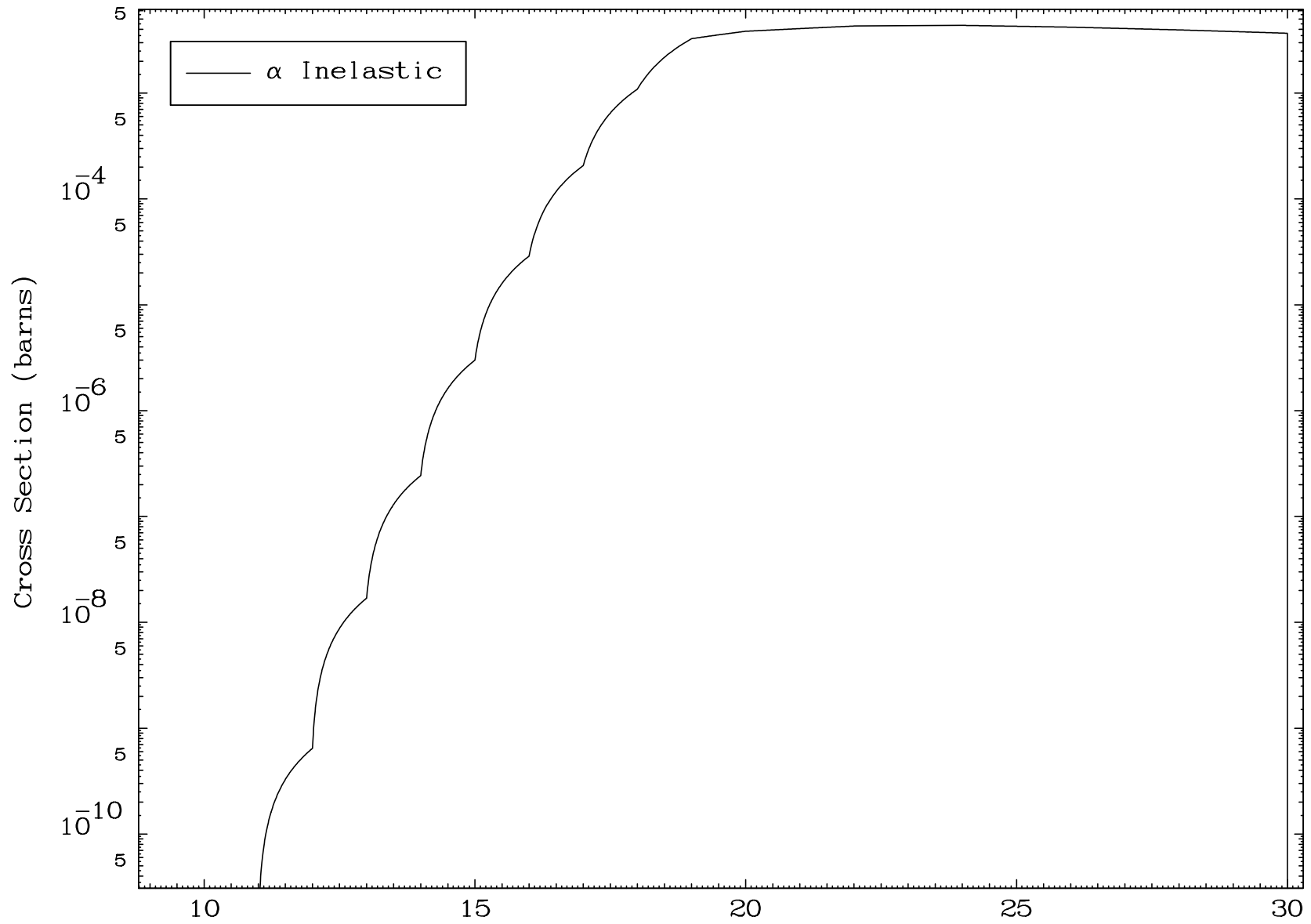


2

Incident Energy (MeV)

86-Rn-223

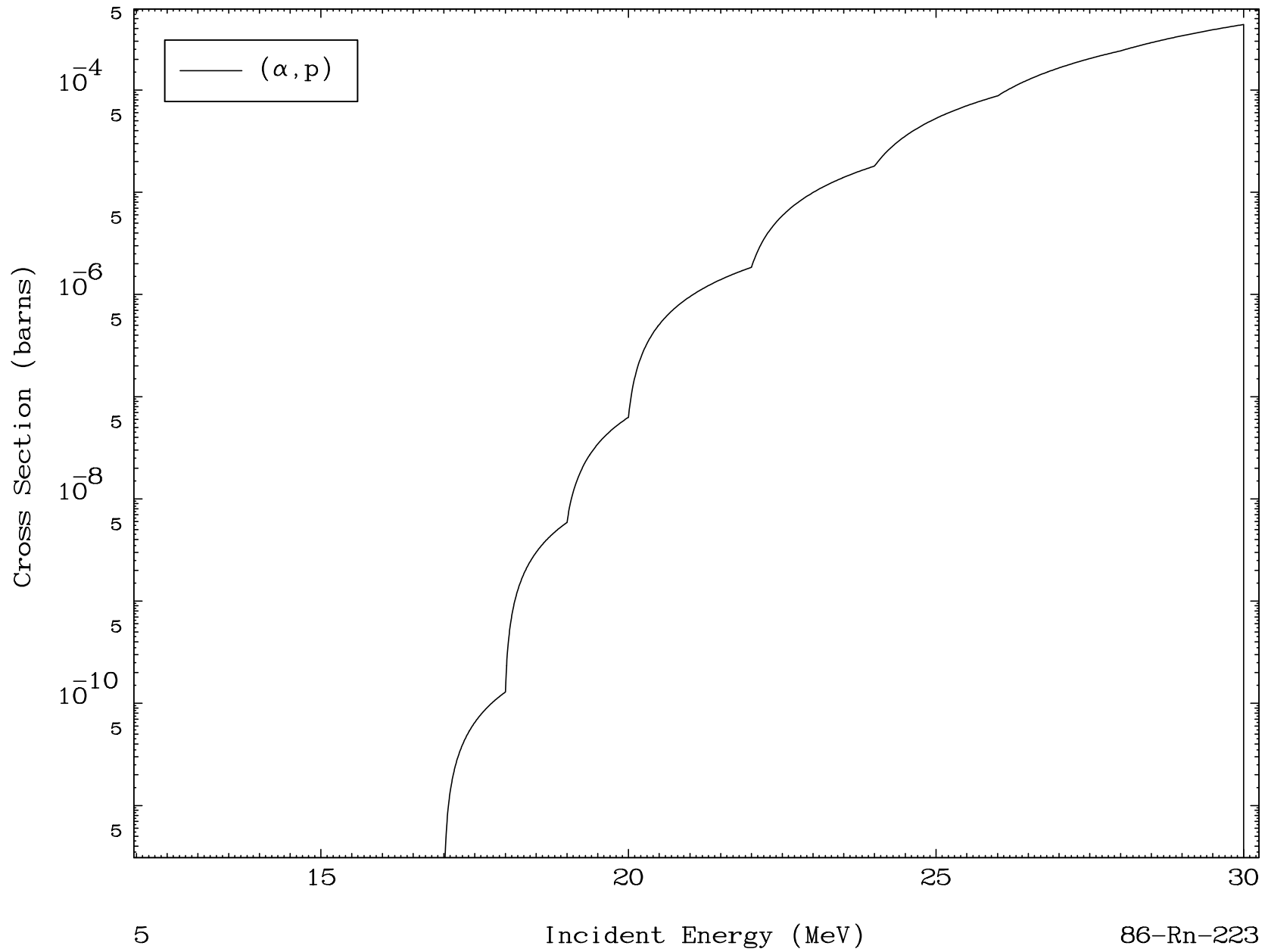


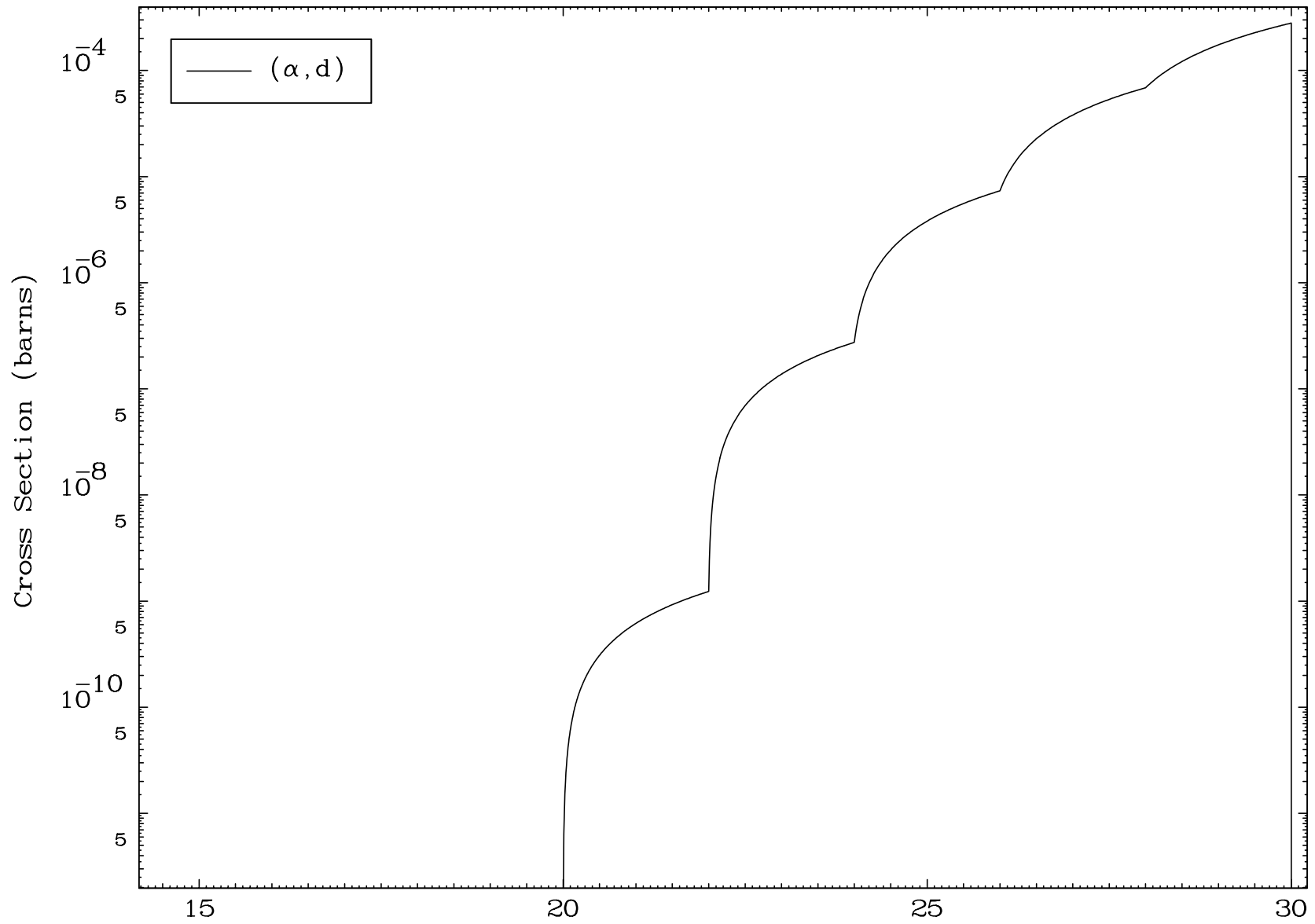


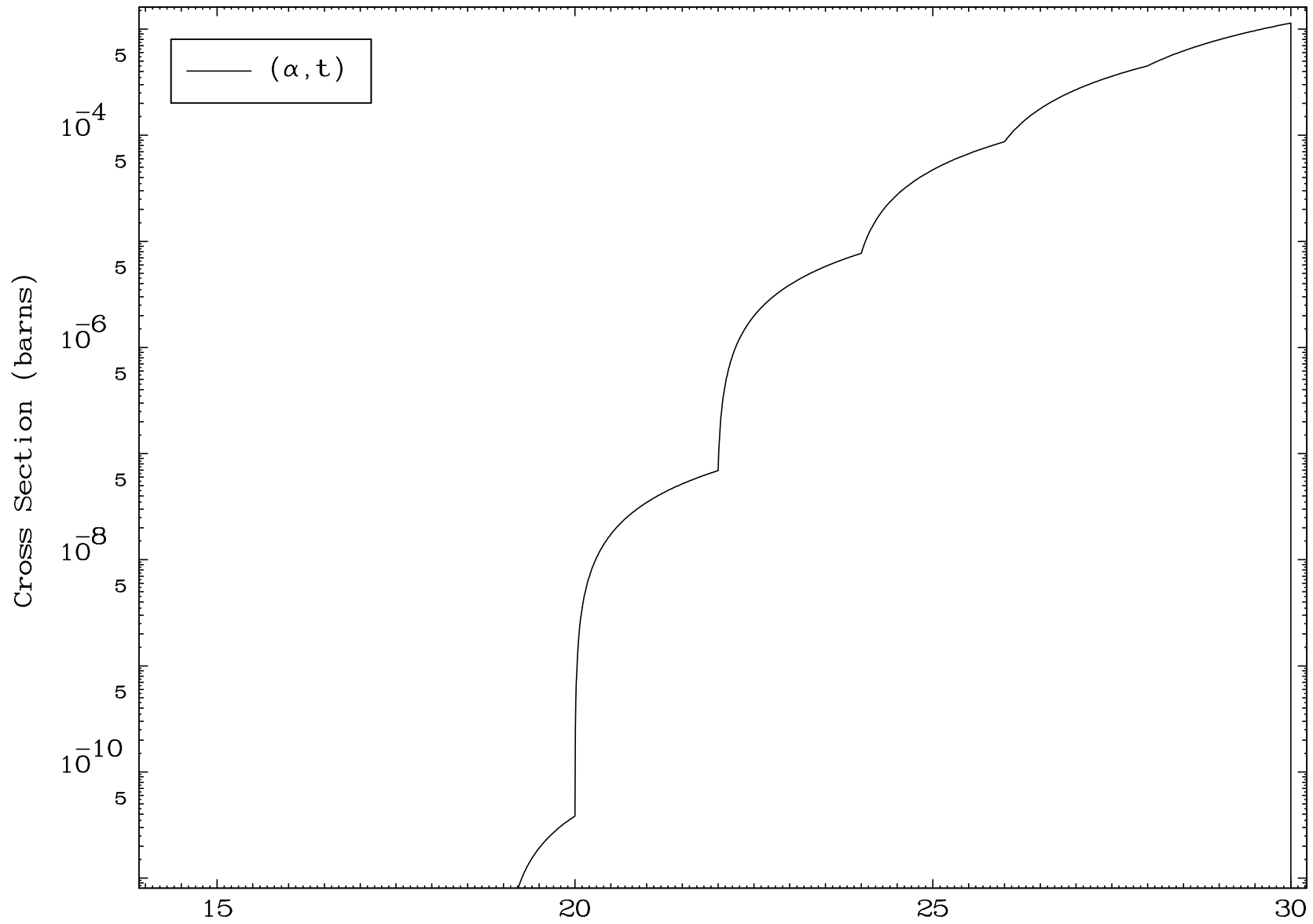
MAT 8661

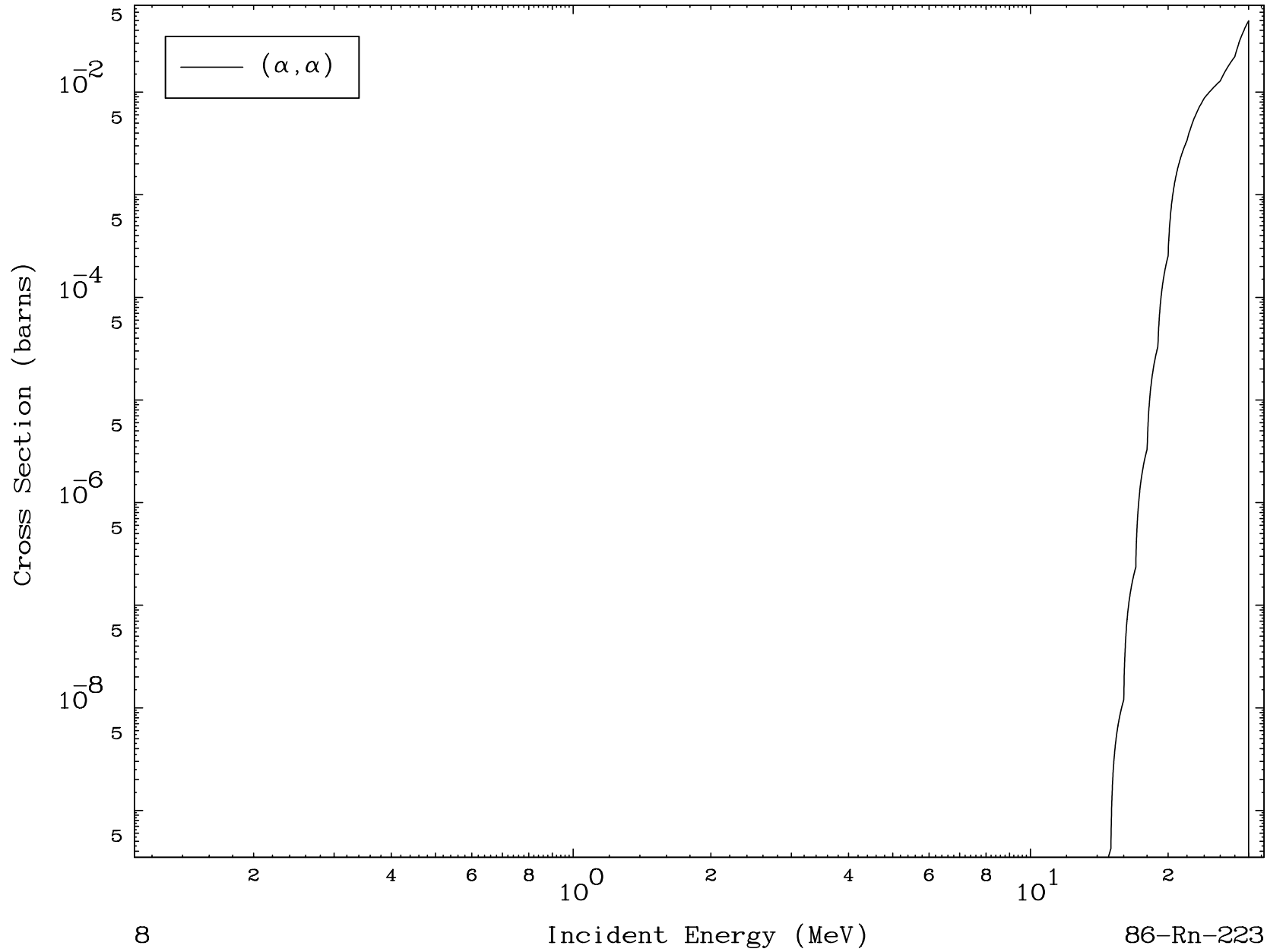
( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections

86-Rn-223



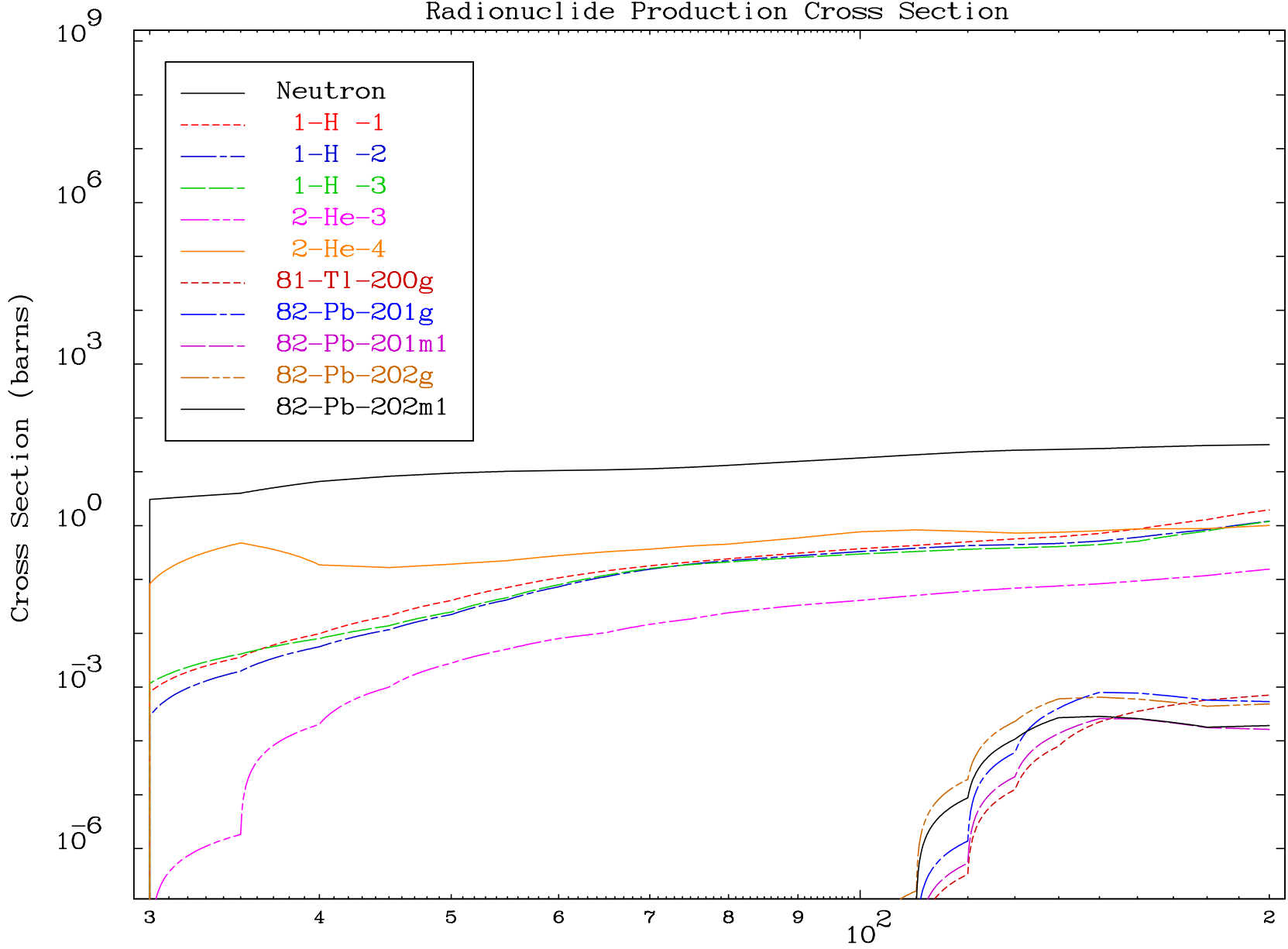




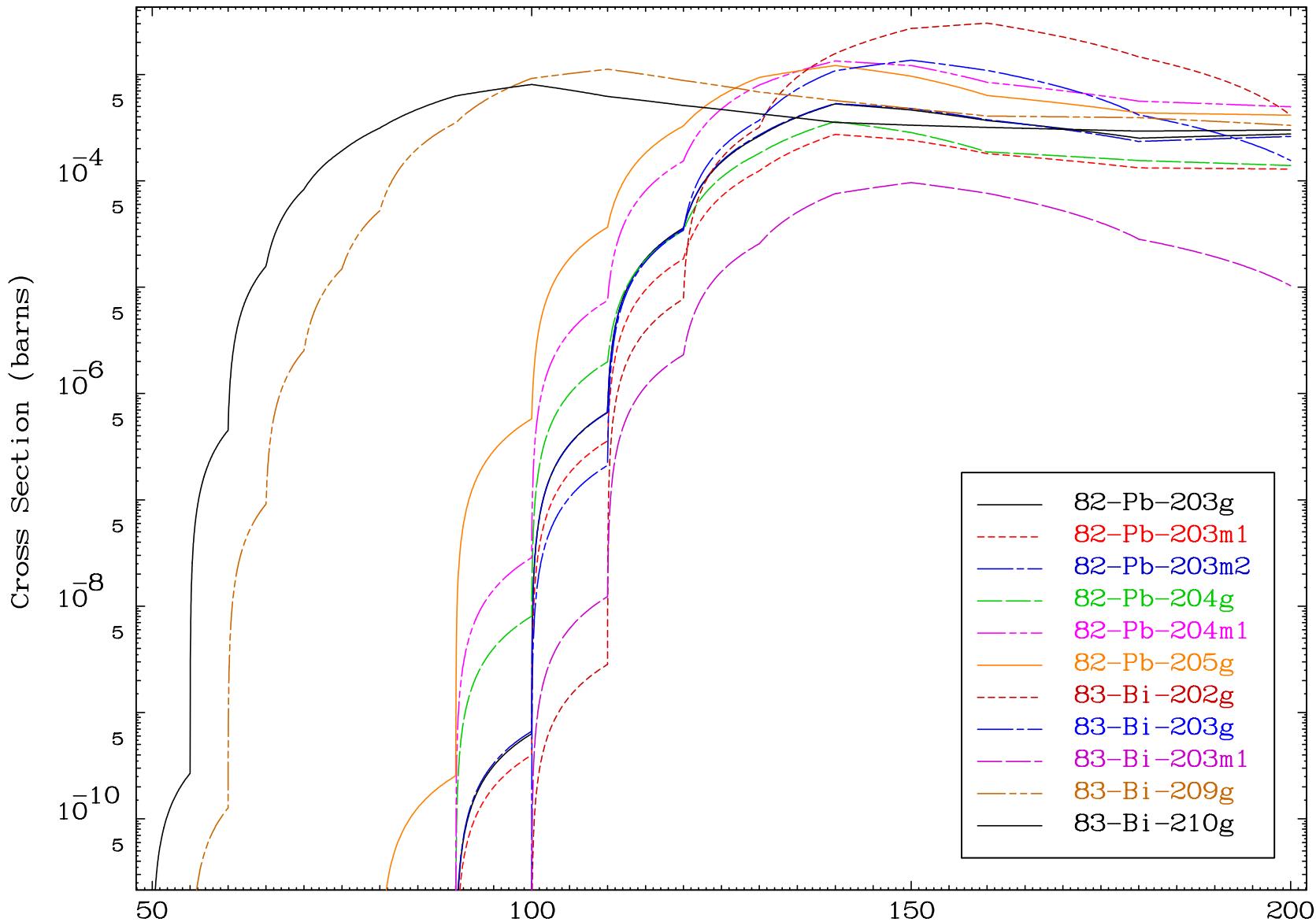




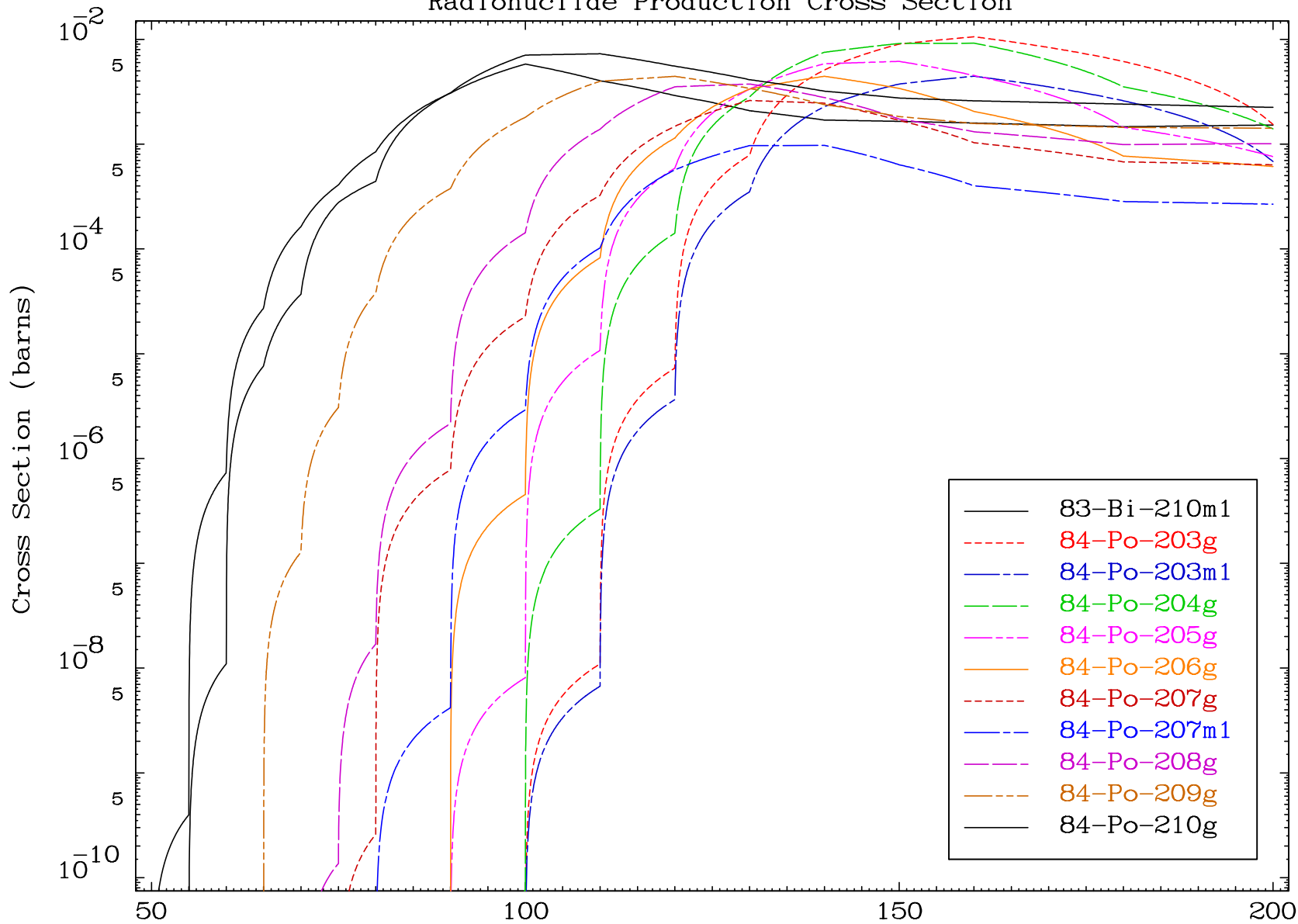
Radionuclide Production Cross Section



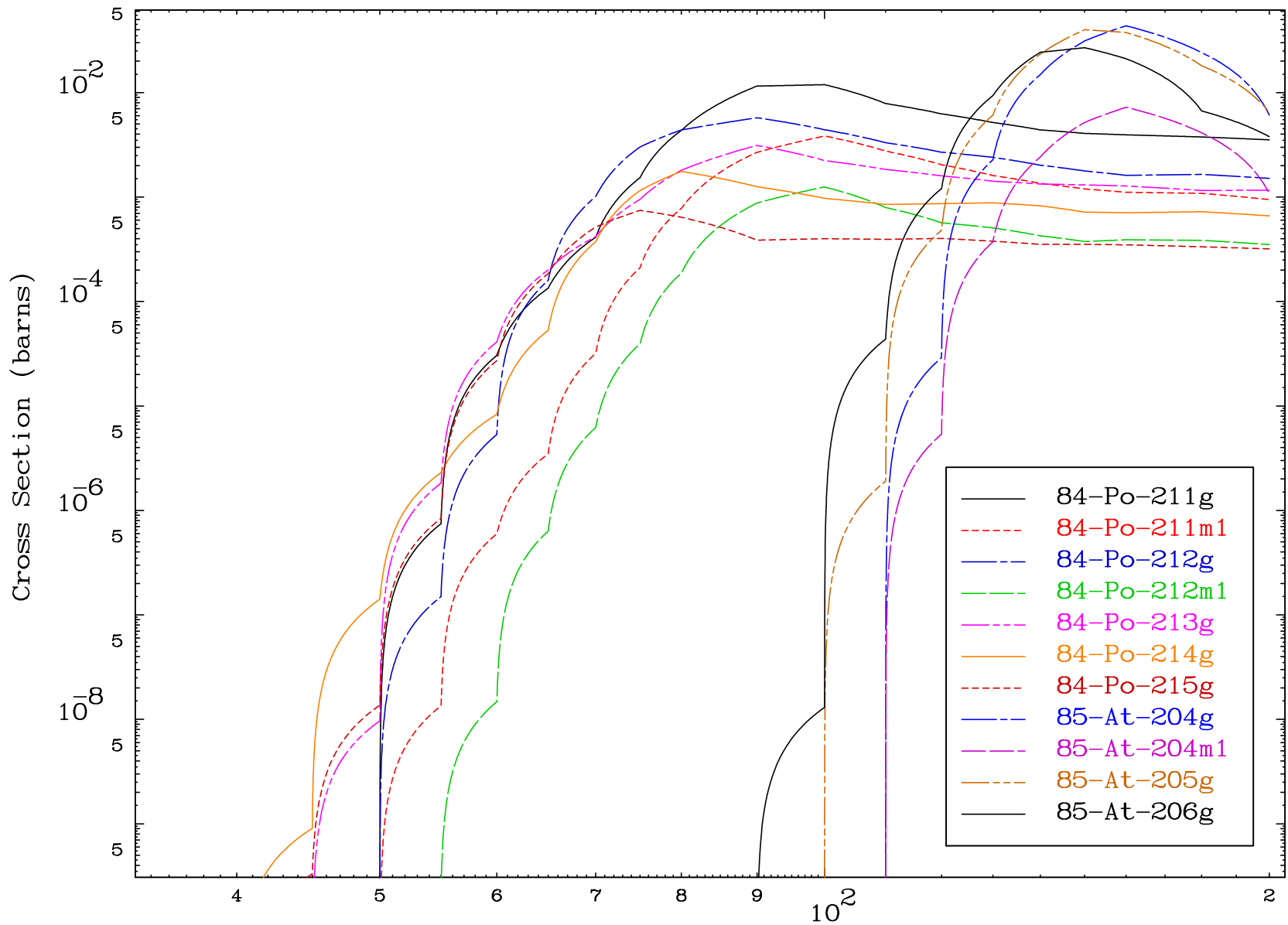
Radionuclide Production Cross Section

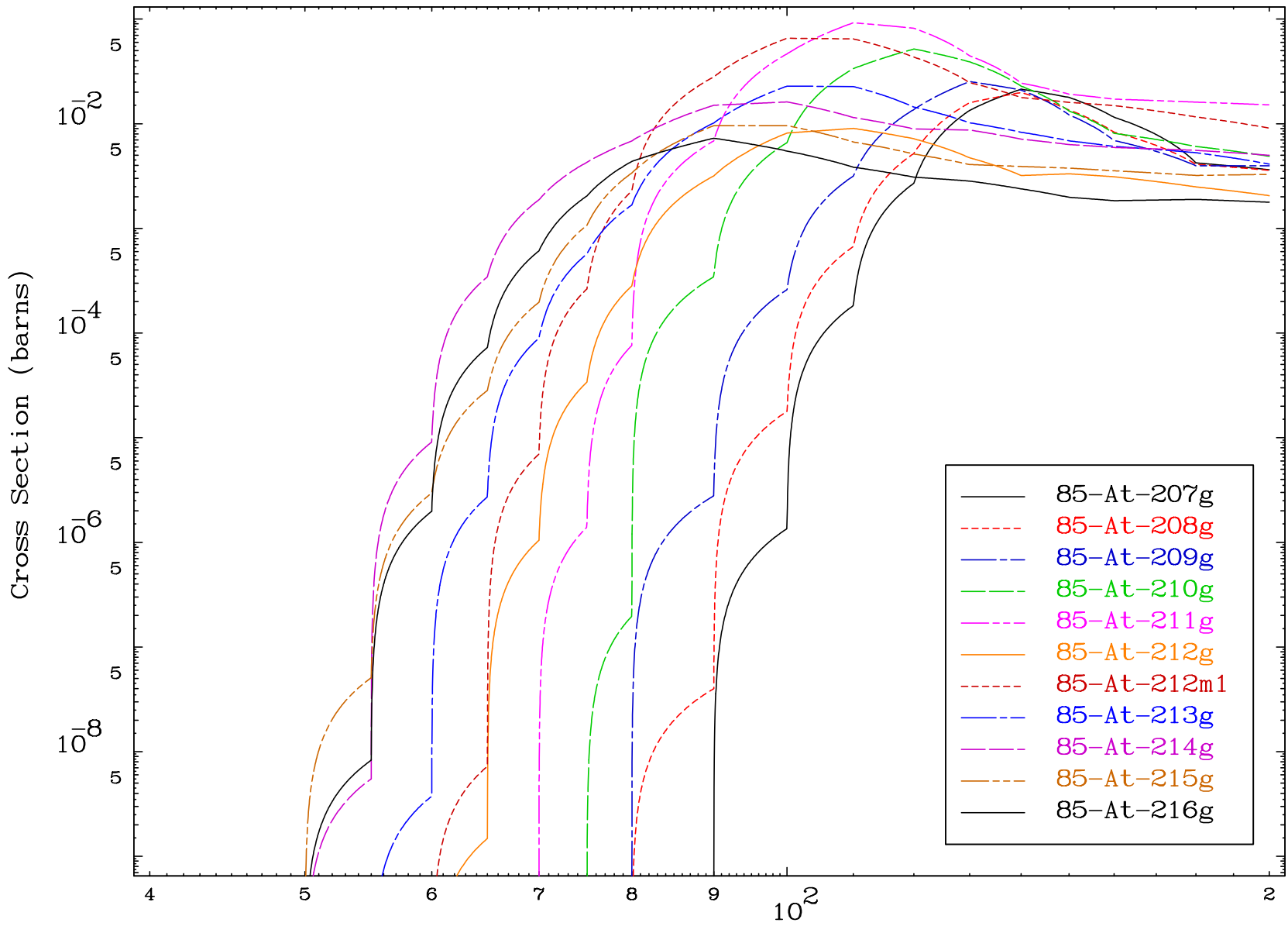


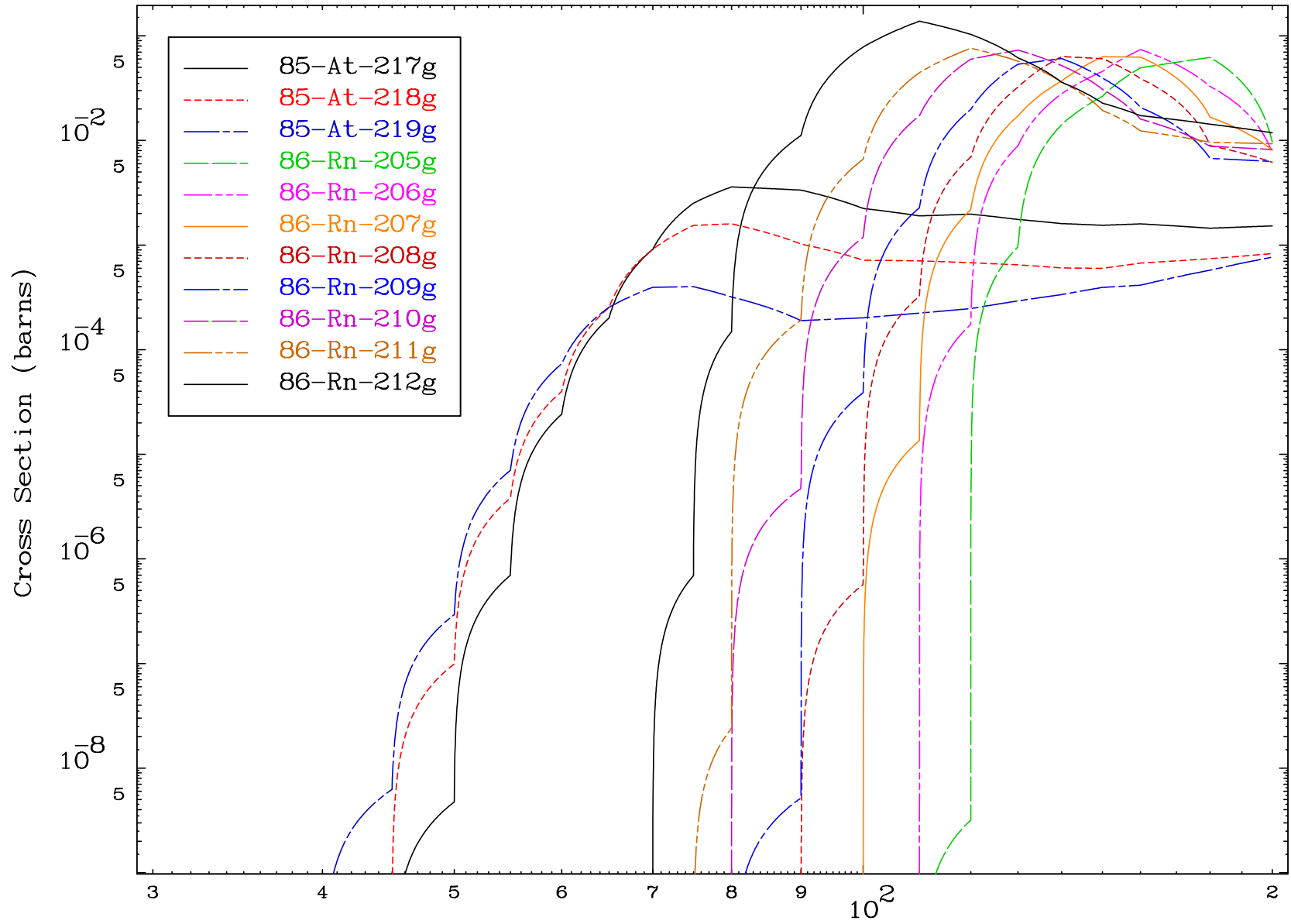
Radionuclide Production Cross Section

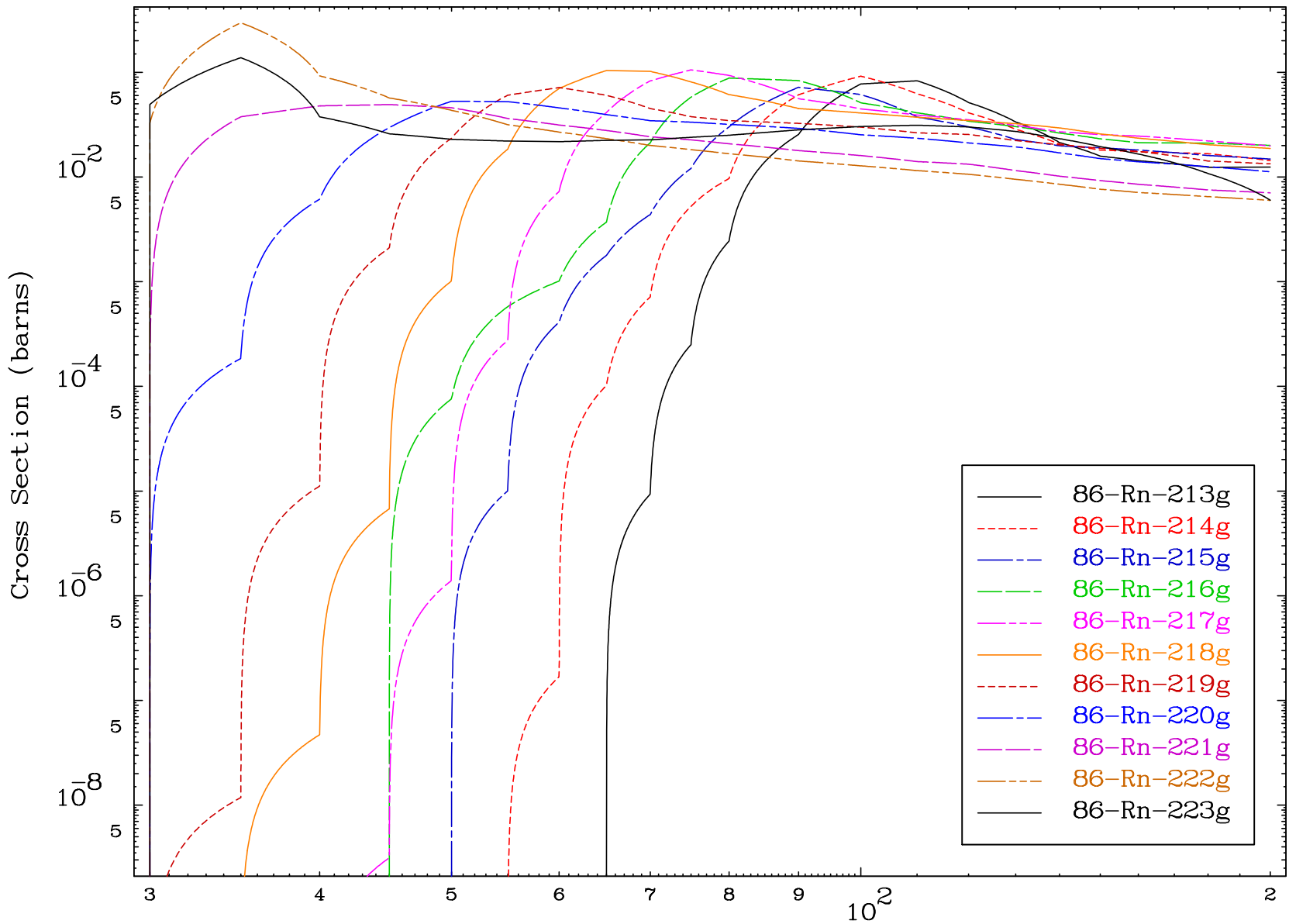


Radionuclide Production Cross Section

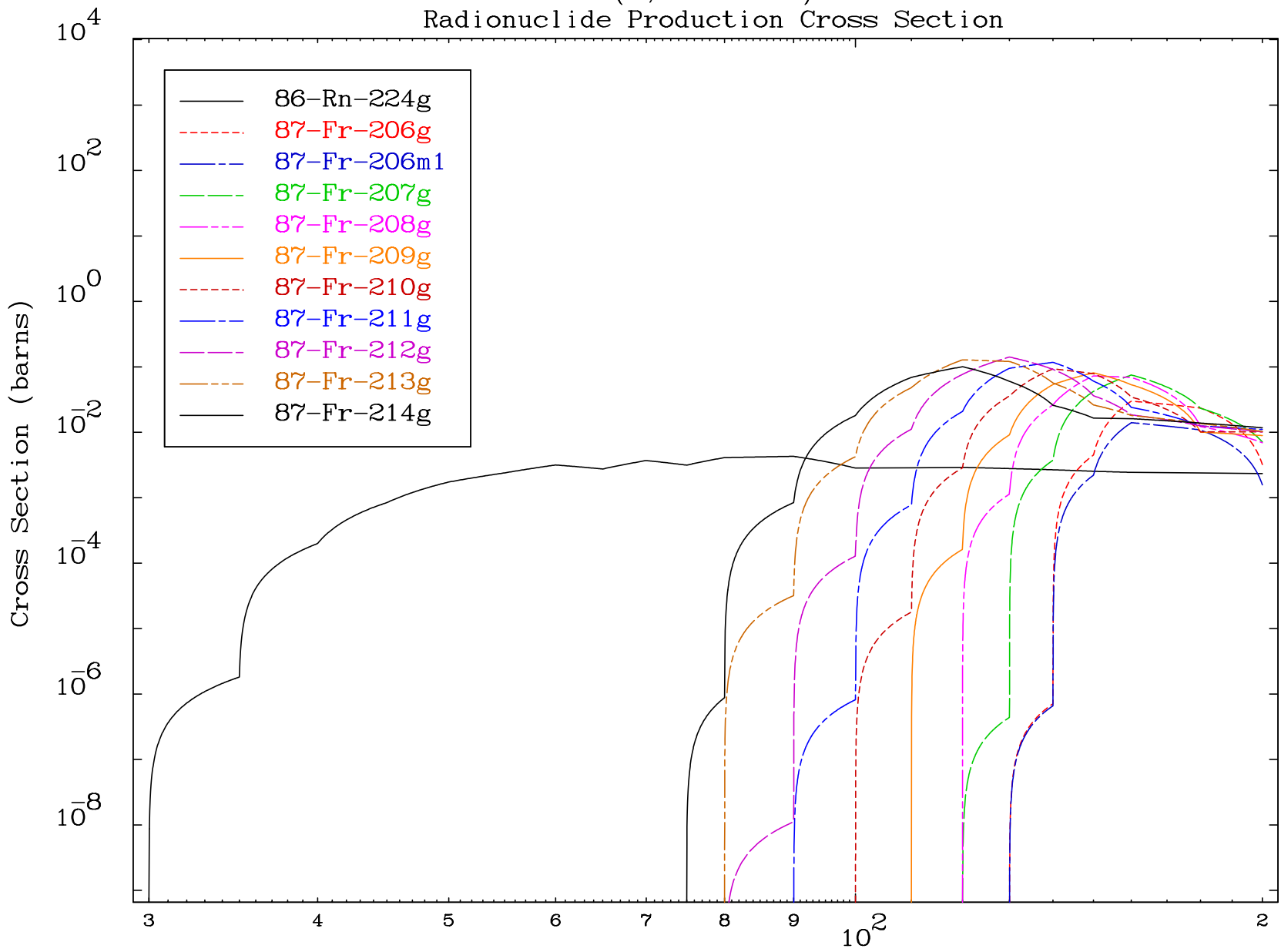




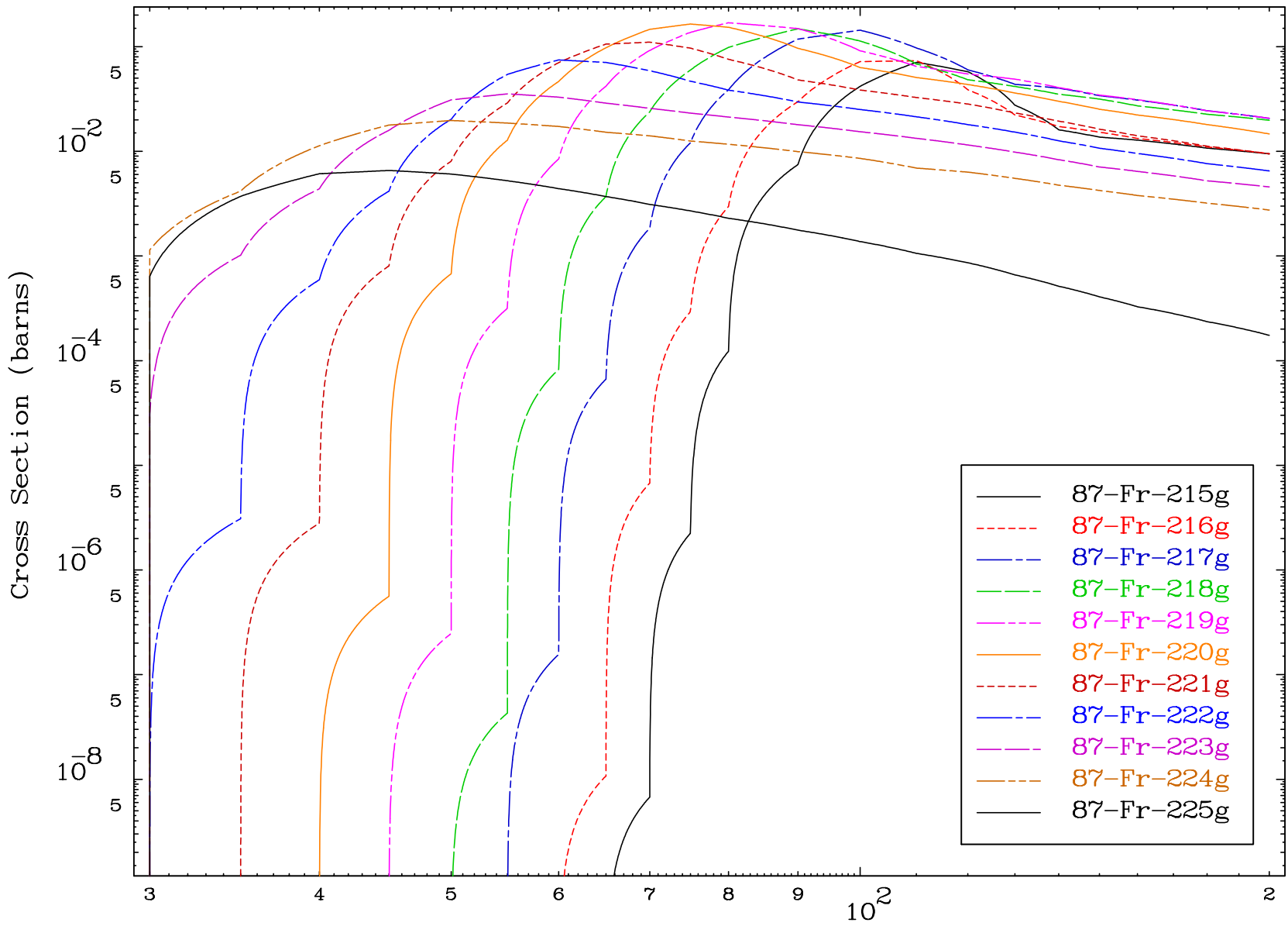




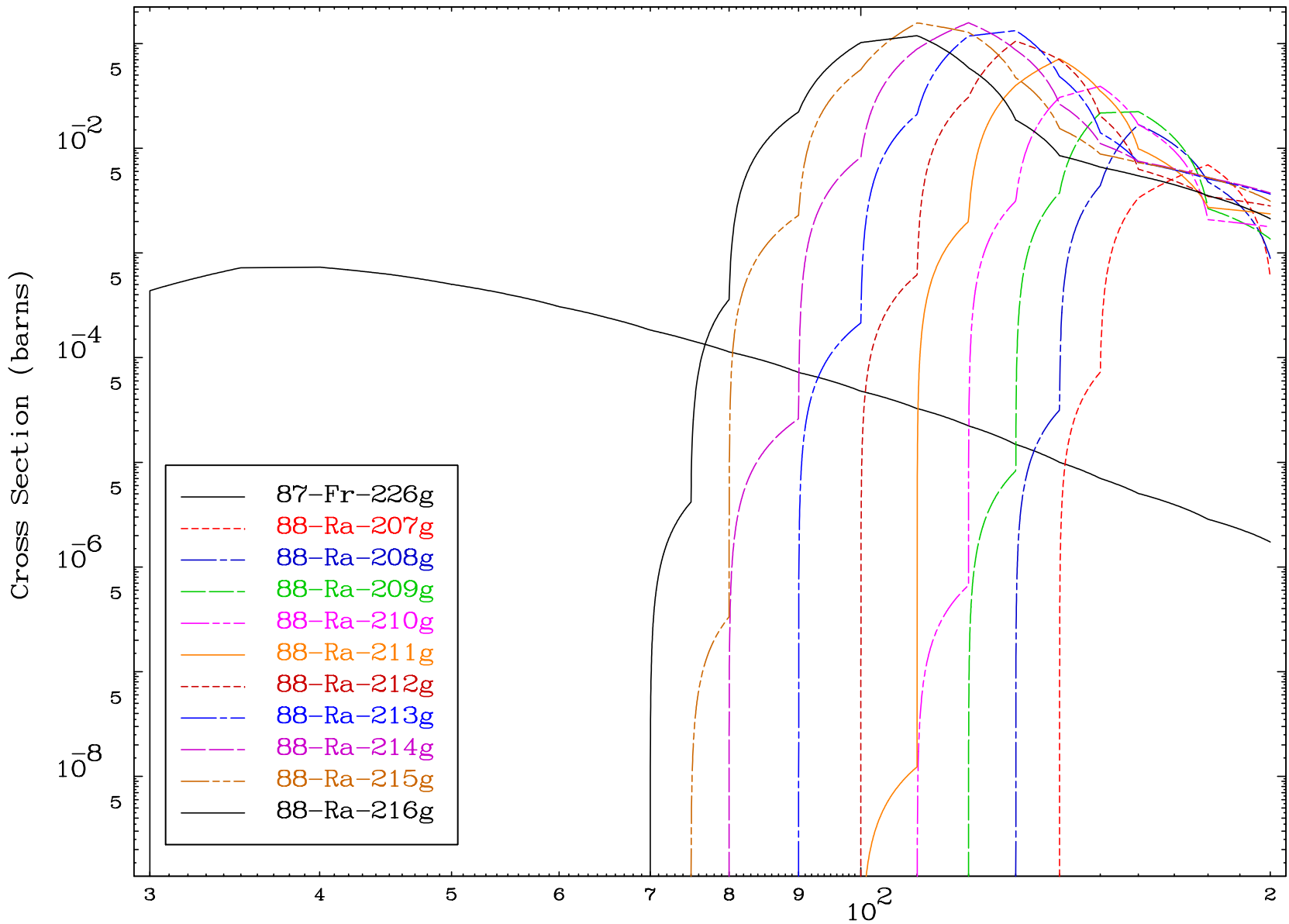
Radionuclide Production Cross Section







Radionuclide Production Cross Section



Radionuclide Production Cross Section

