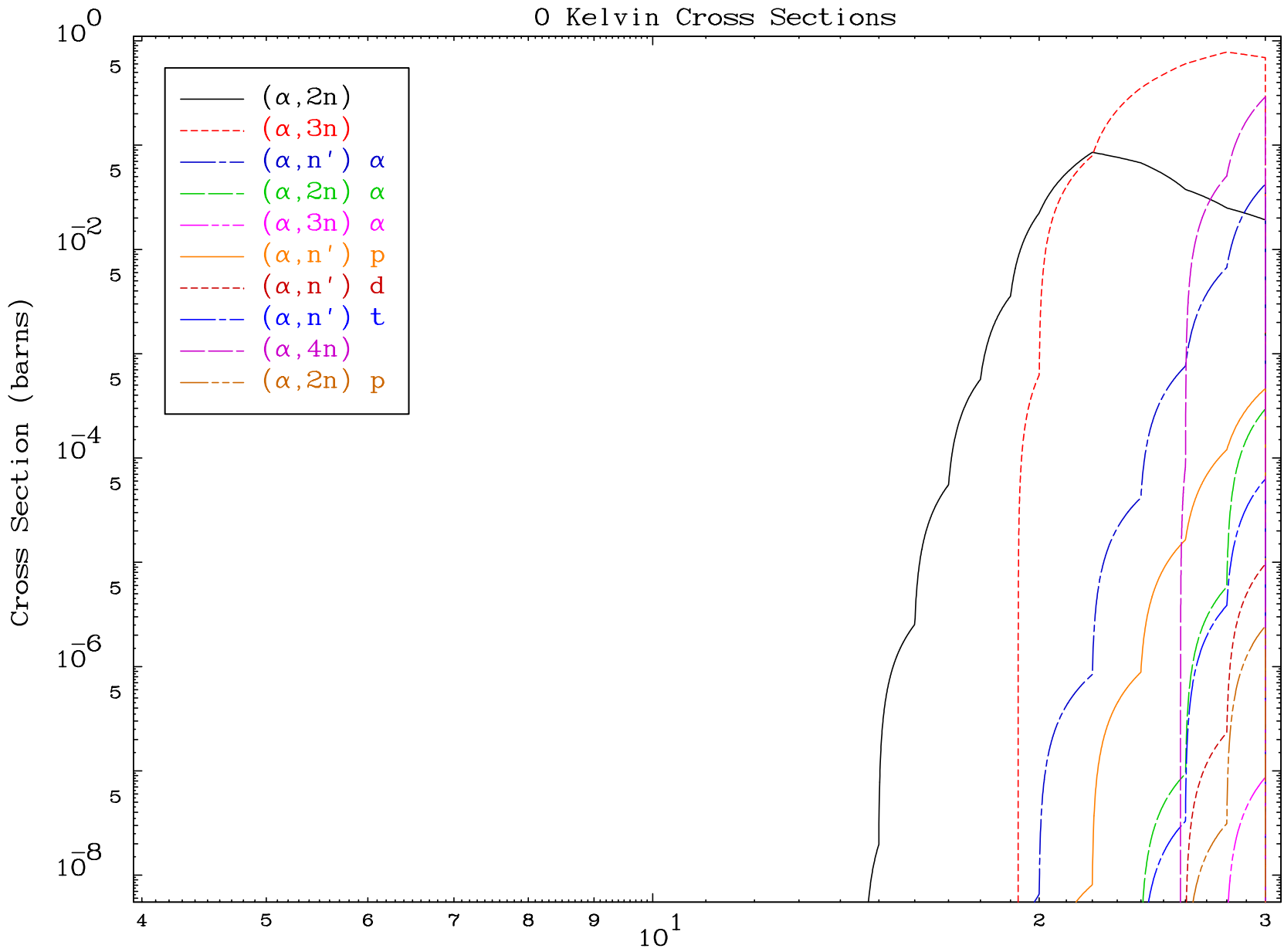


MAT 8667

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

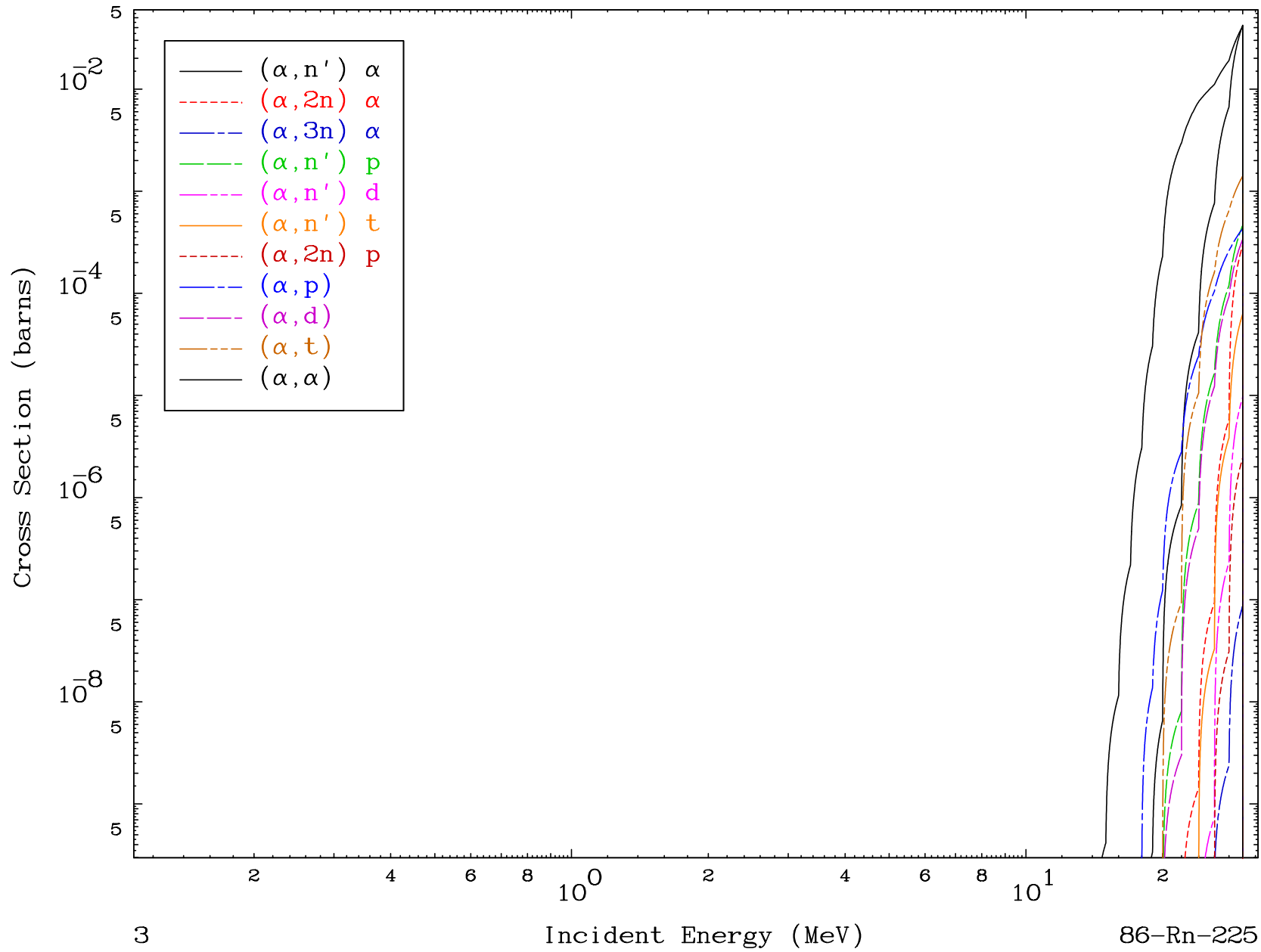
86-Rn-225

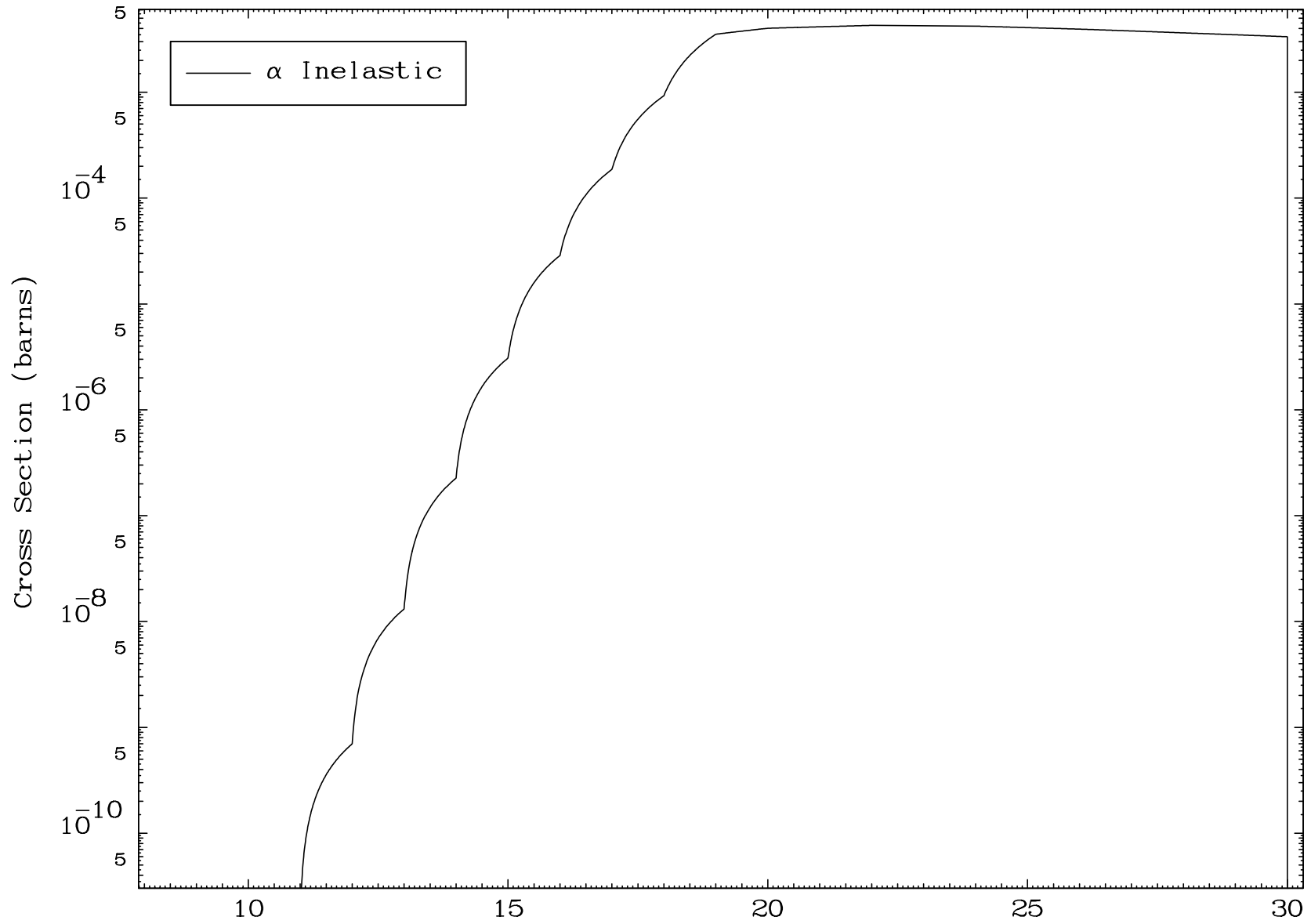


2

Incident Energy (MeV)

86-Rn-225

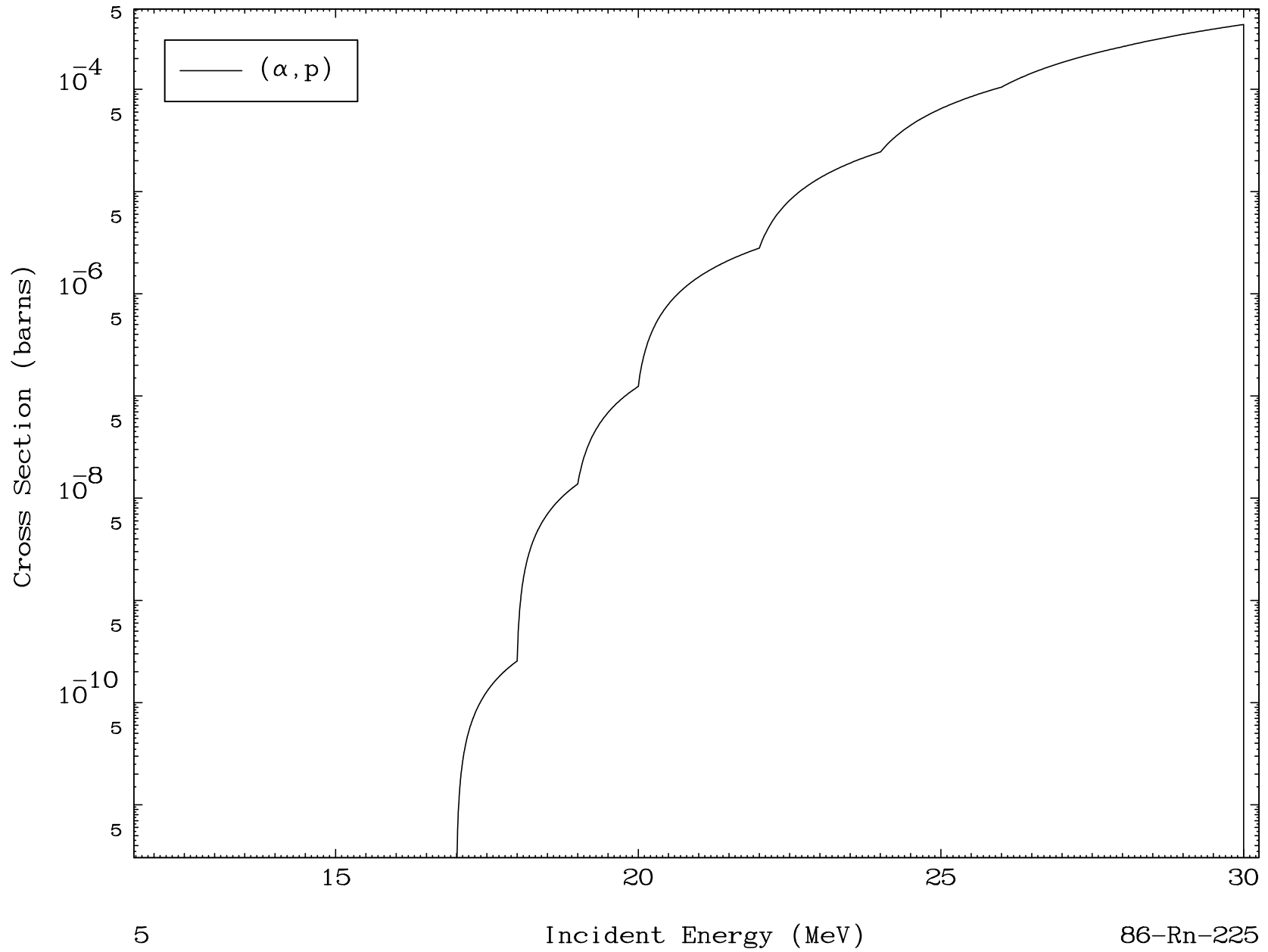


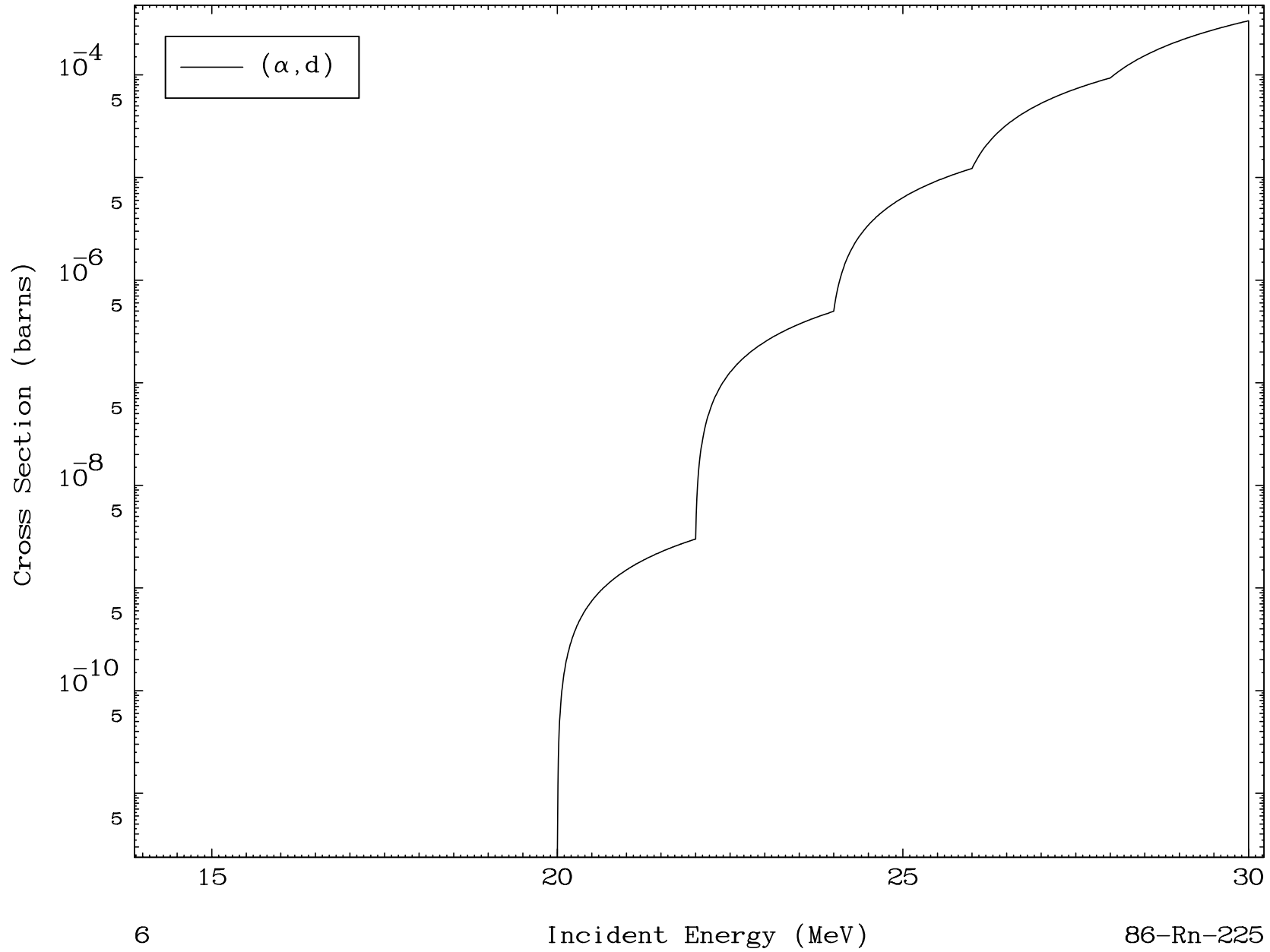


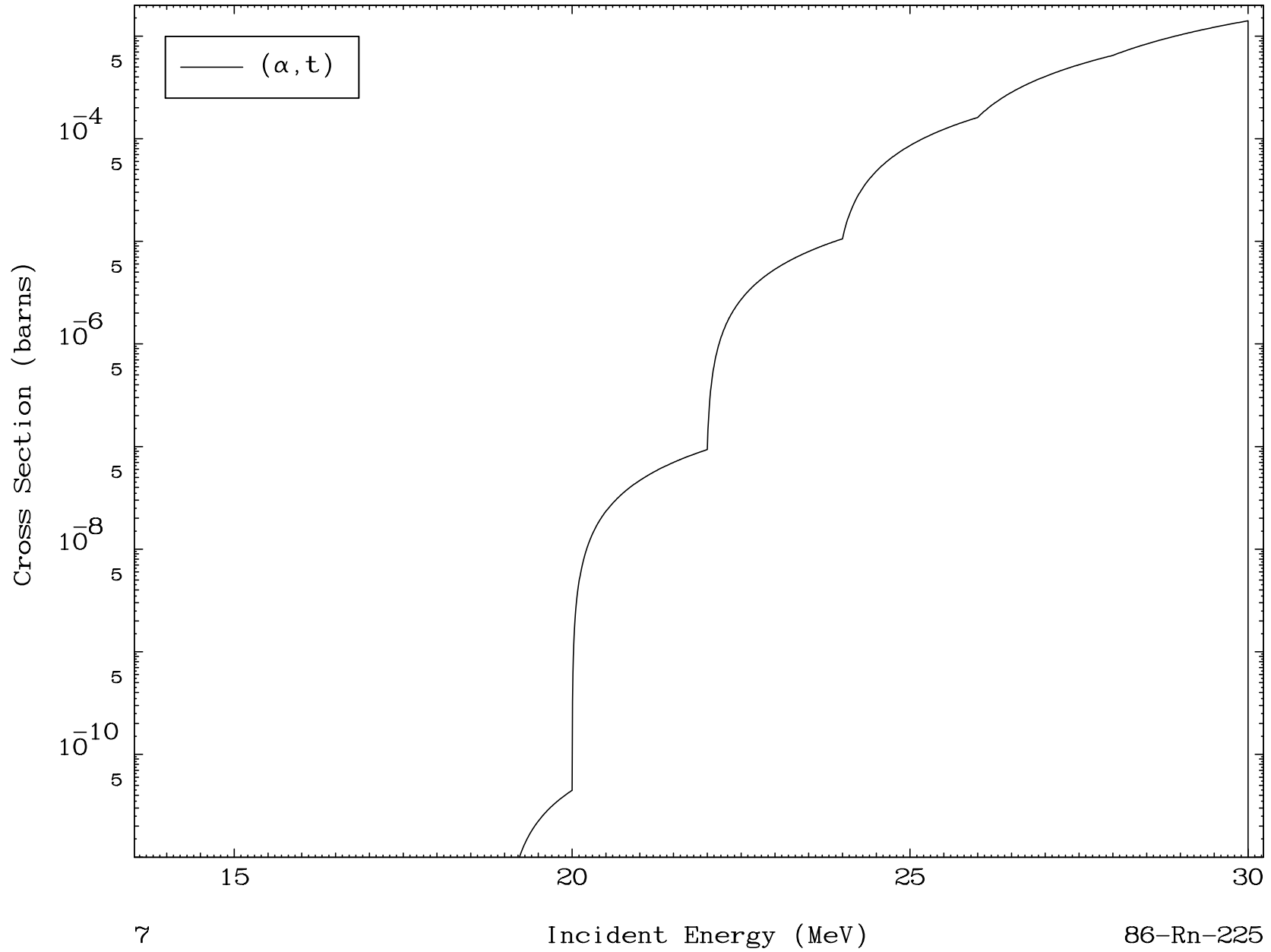
MAT 8667

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections

86-Rn-225



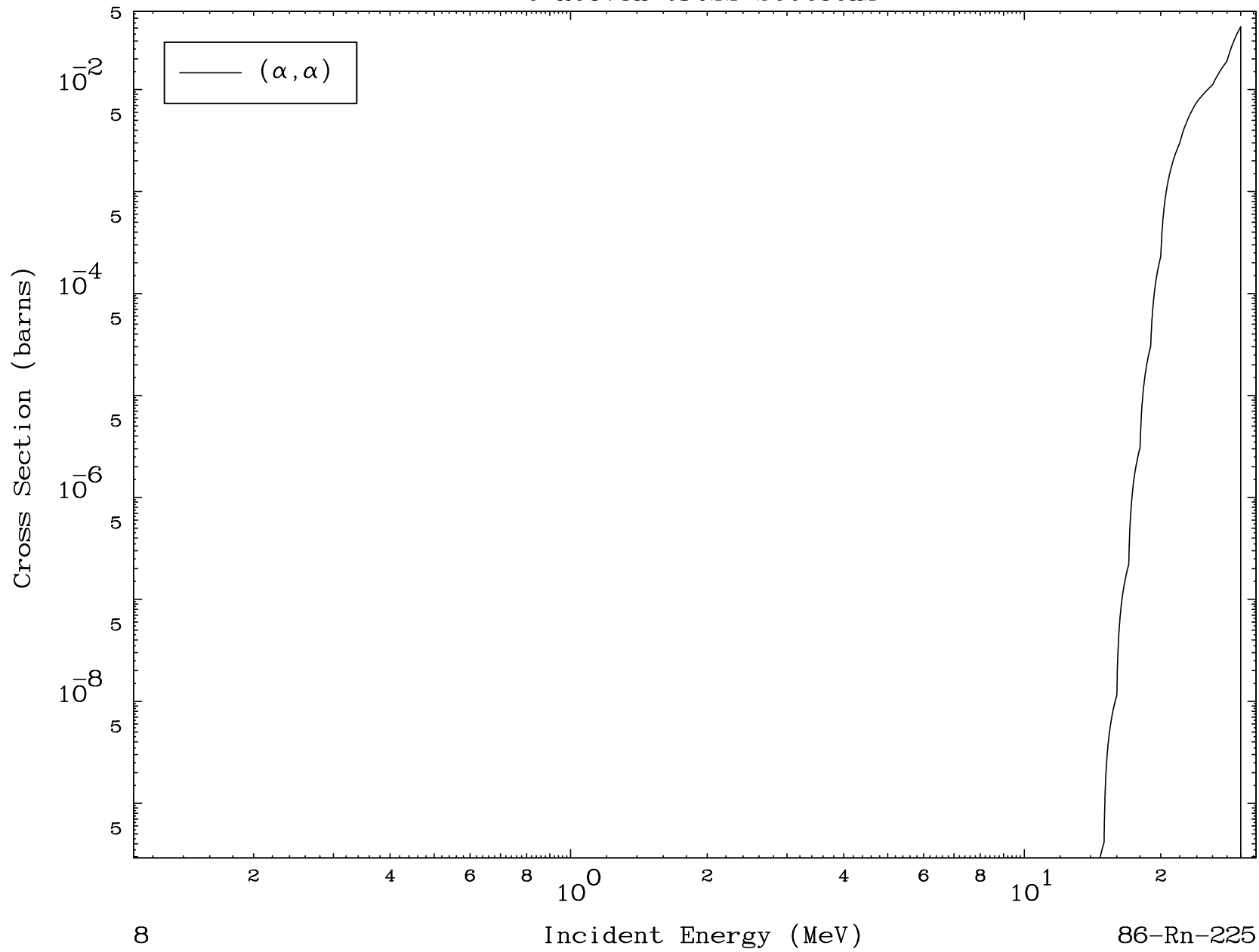




MAT 8667

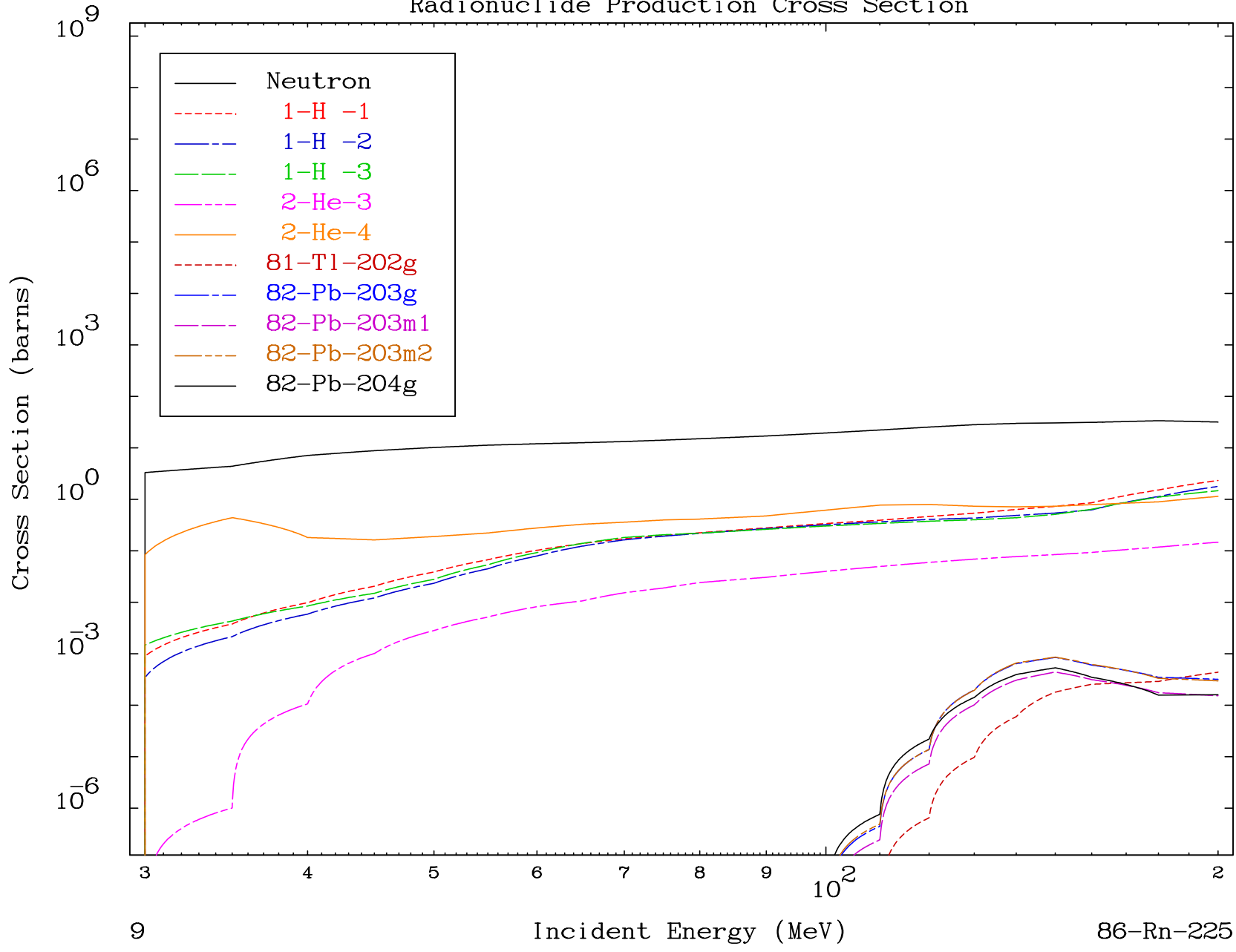
( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

86-Rn-225





Radionuclide Production Cross Section

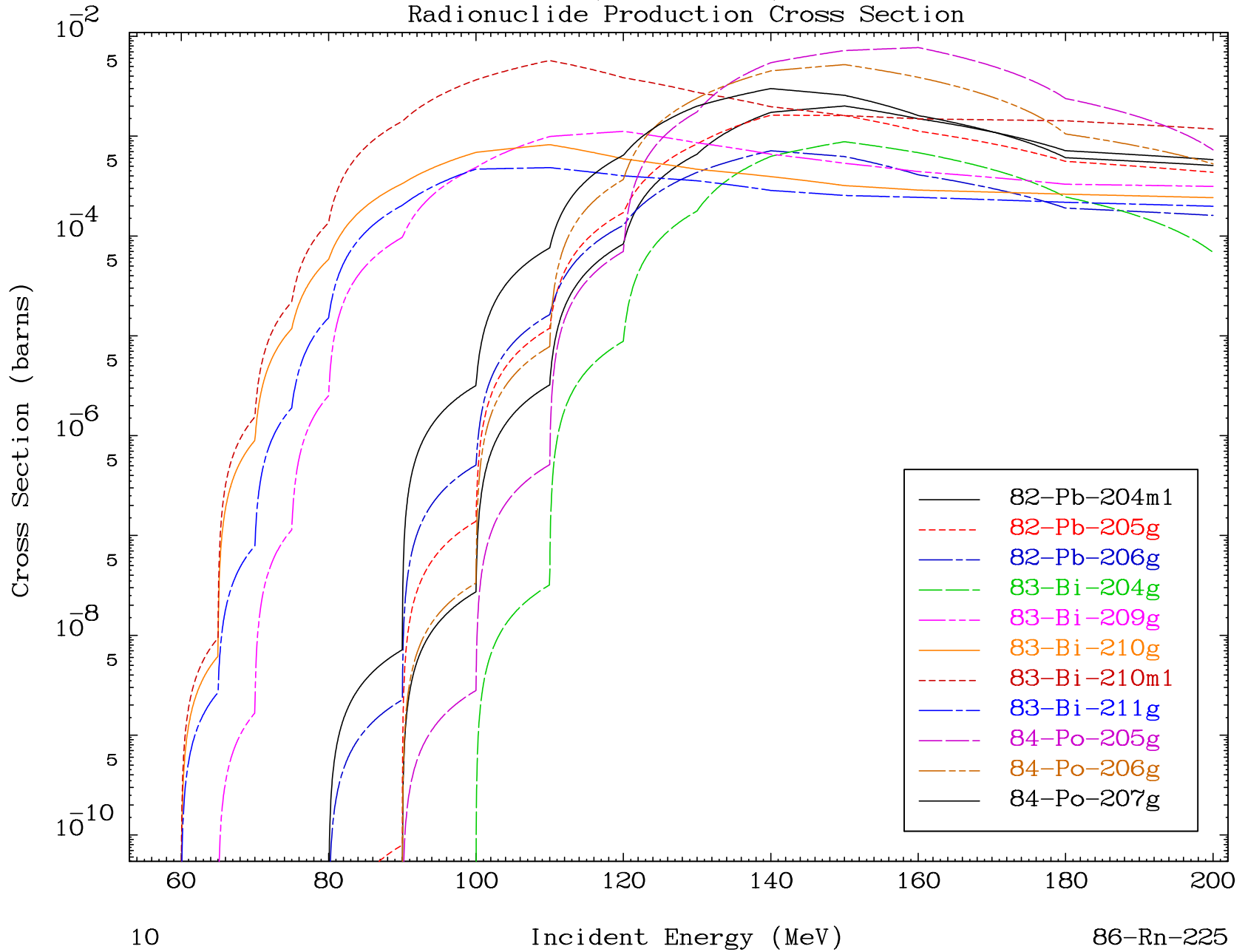


MAT 8667

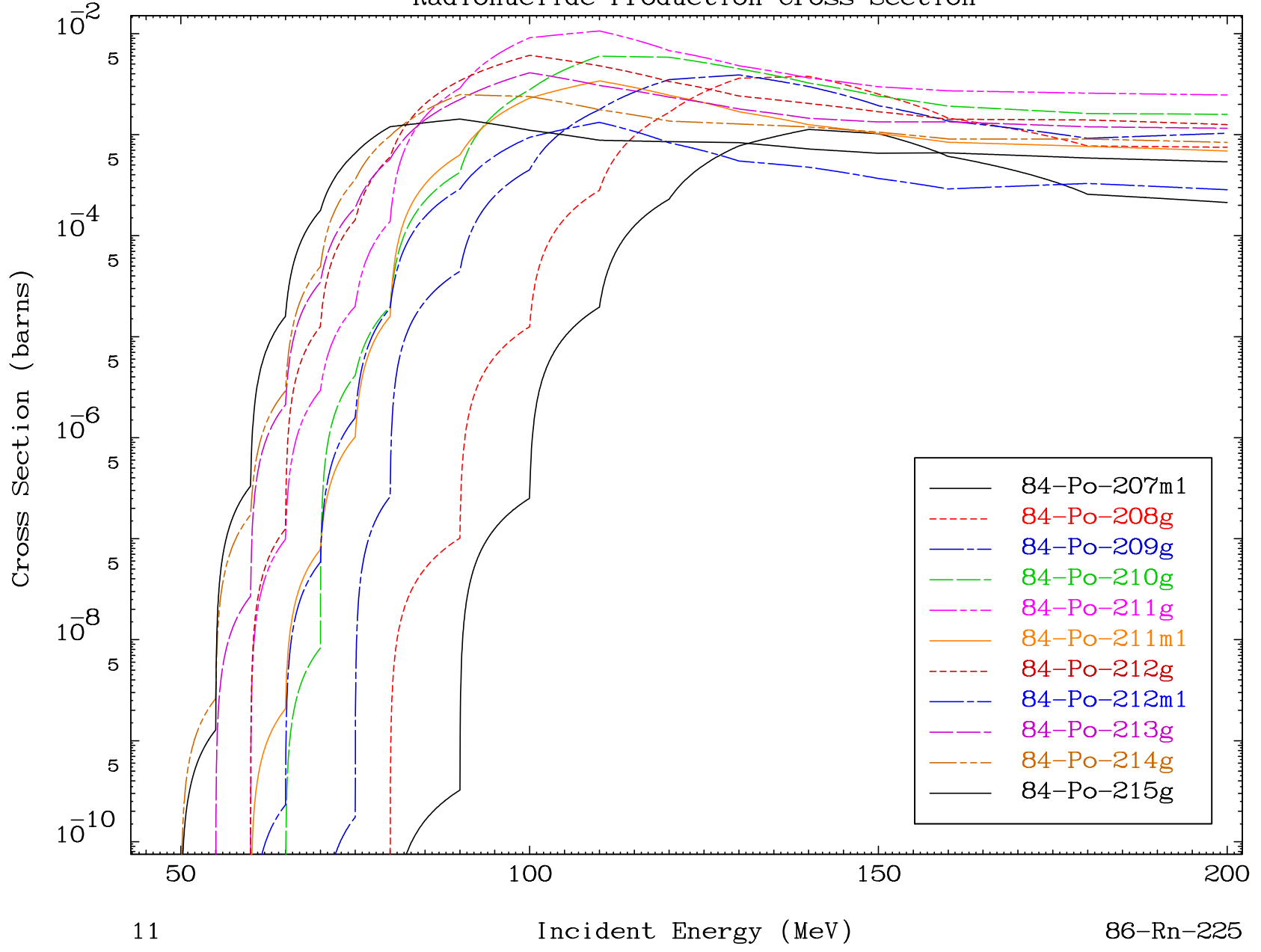
( $\alpha$ , remainder)

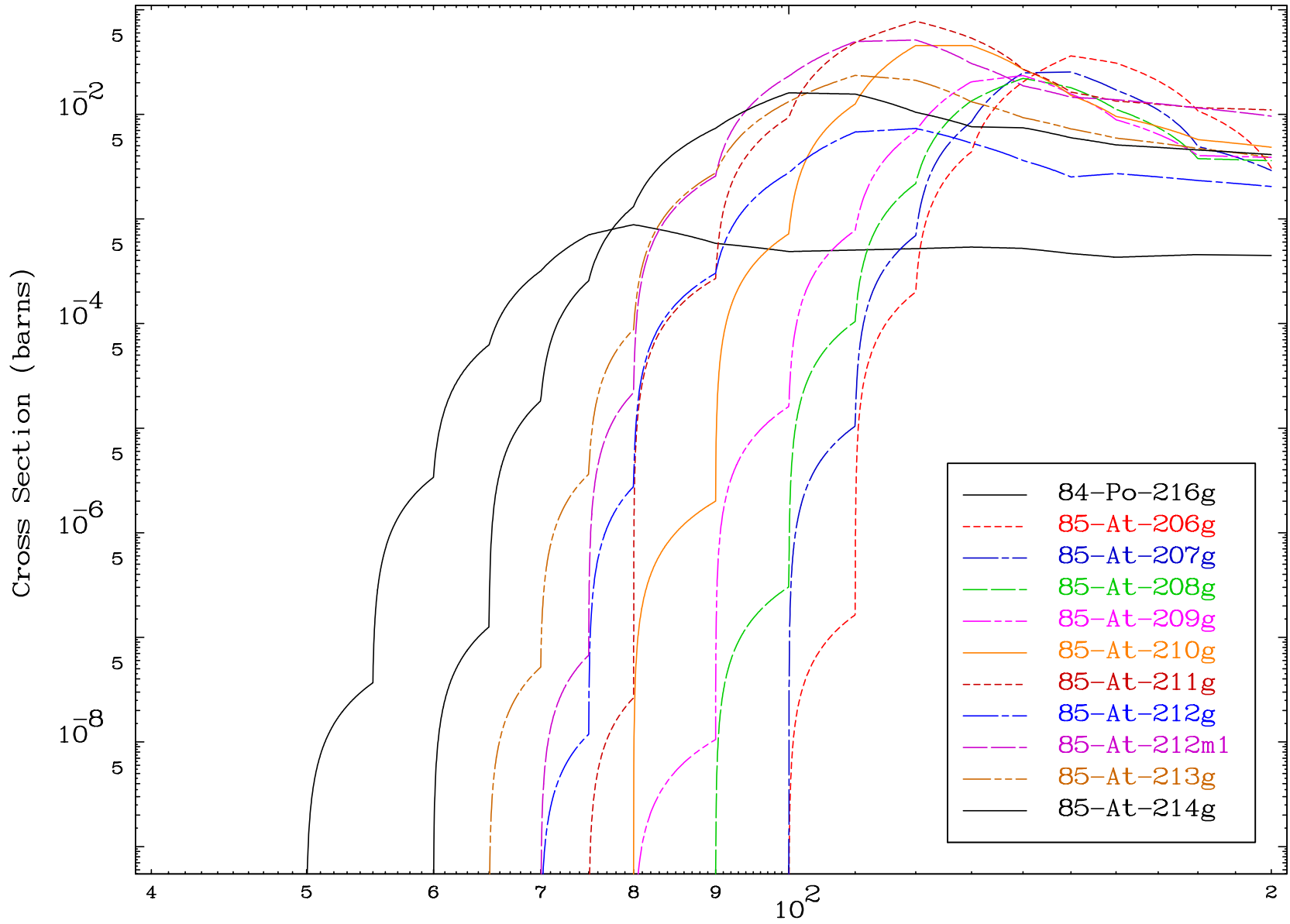
86-Rn-225

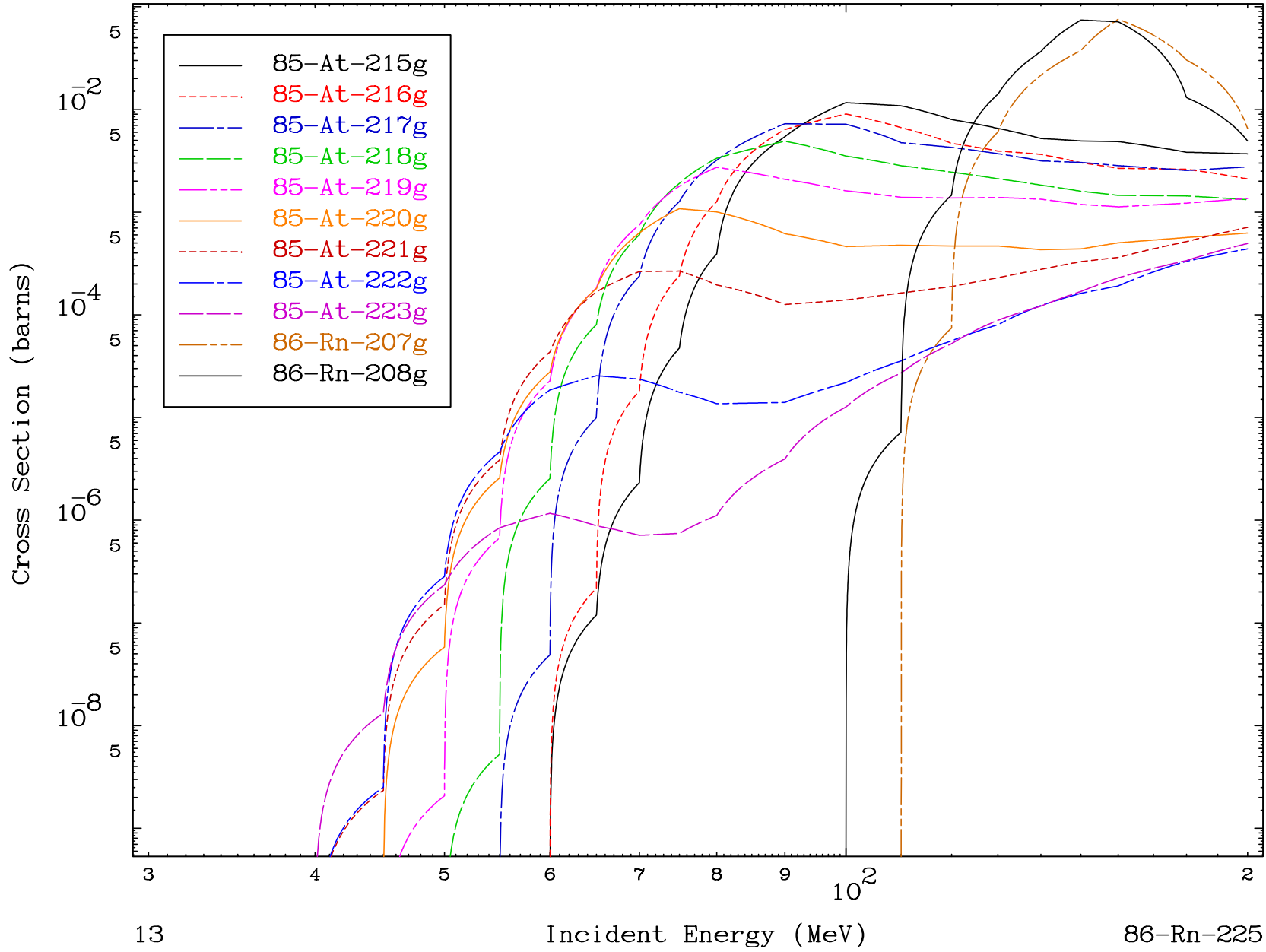
### Radionuclide Production Cross Section

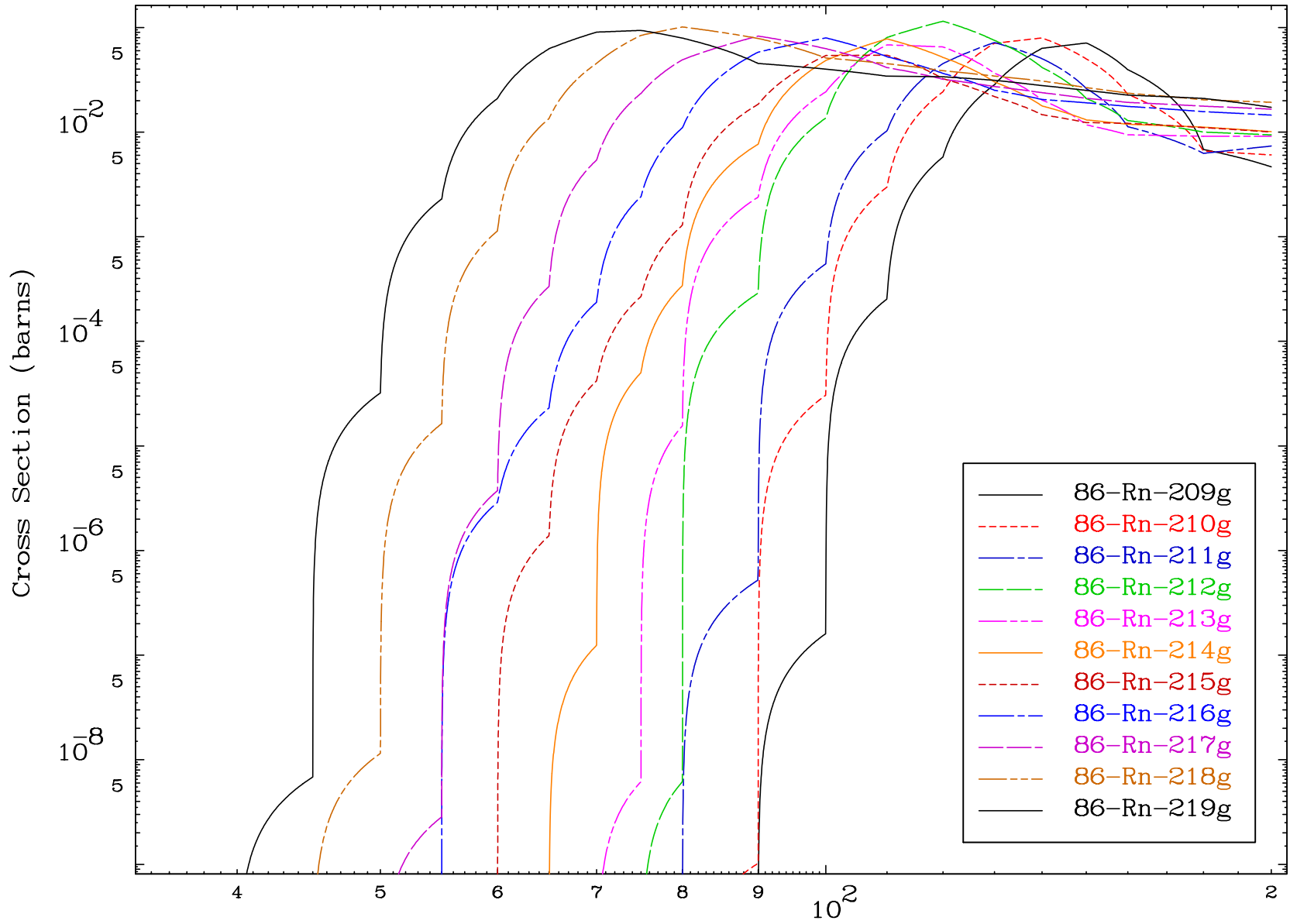


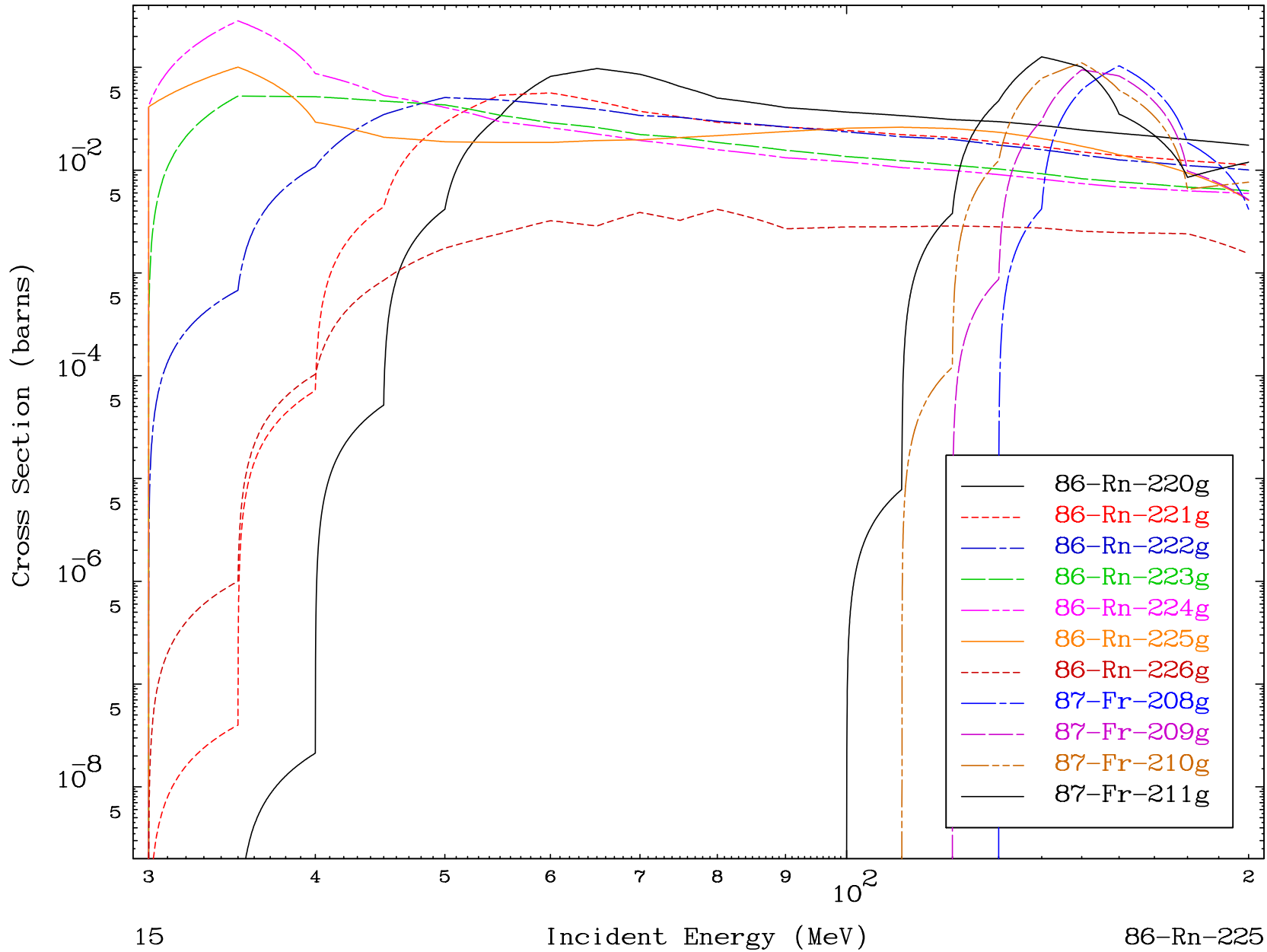
Radionuclide Production Cross Section

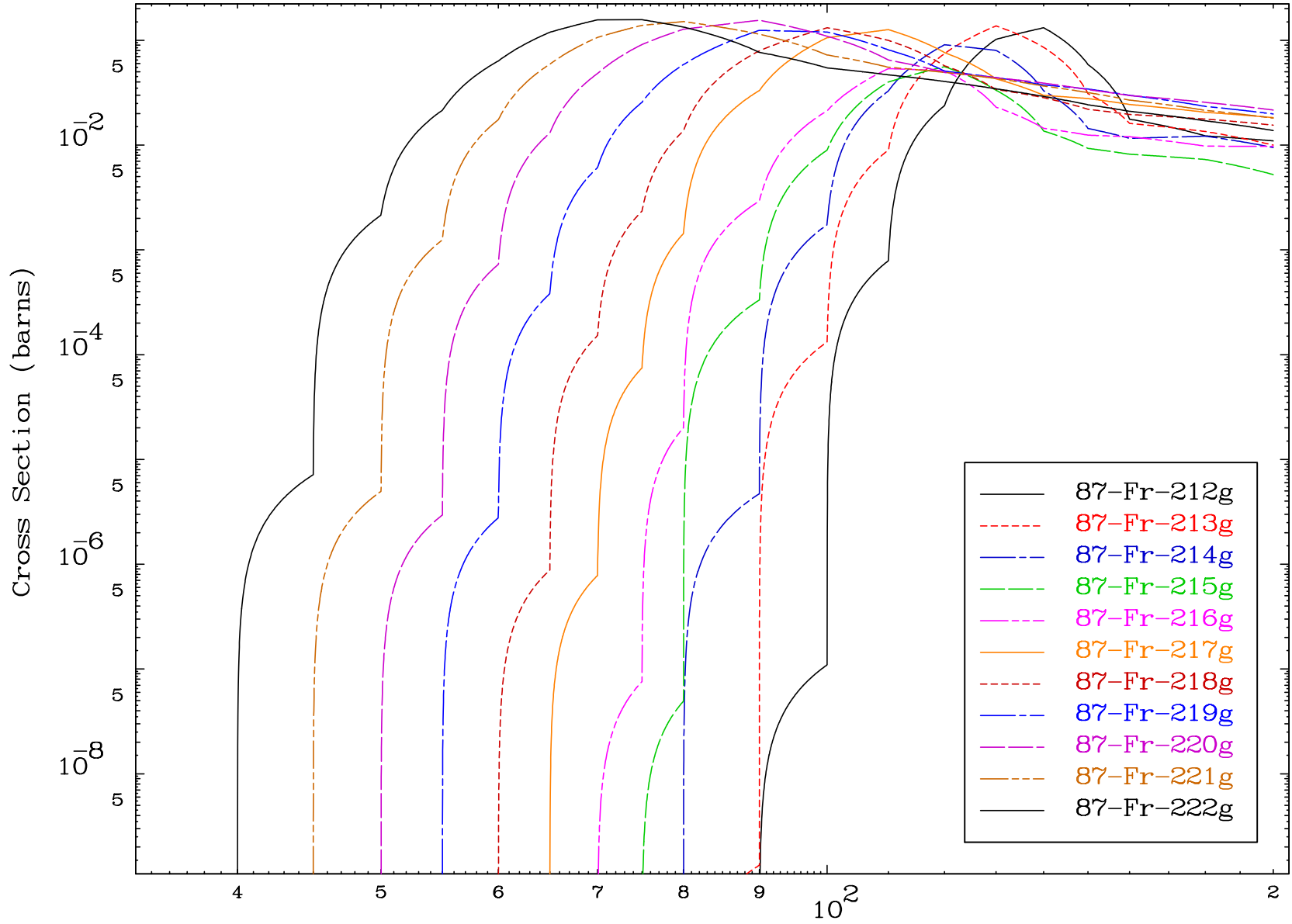






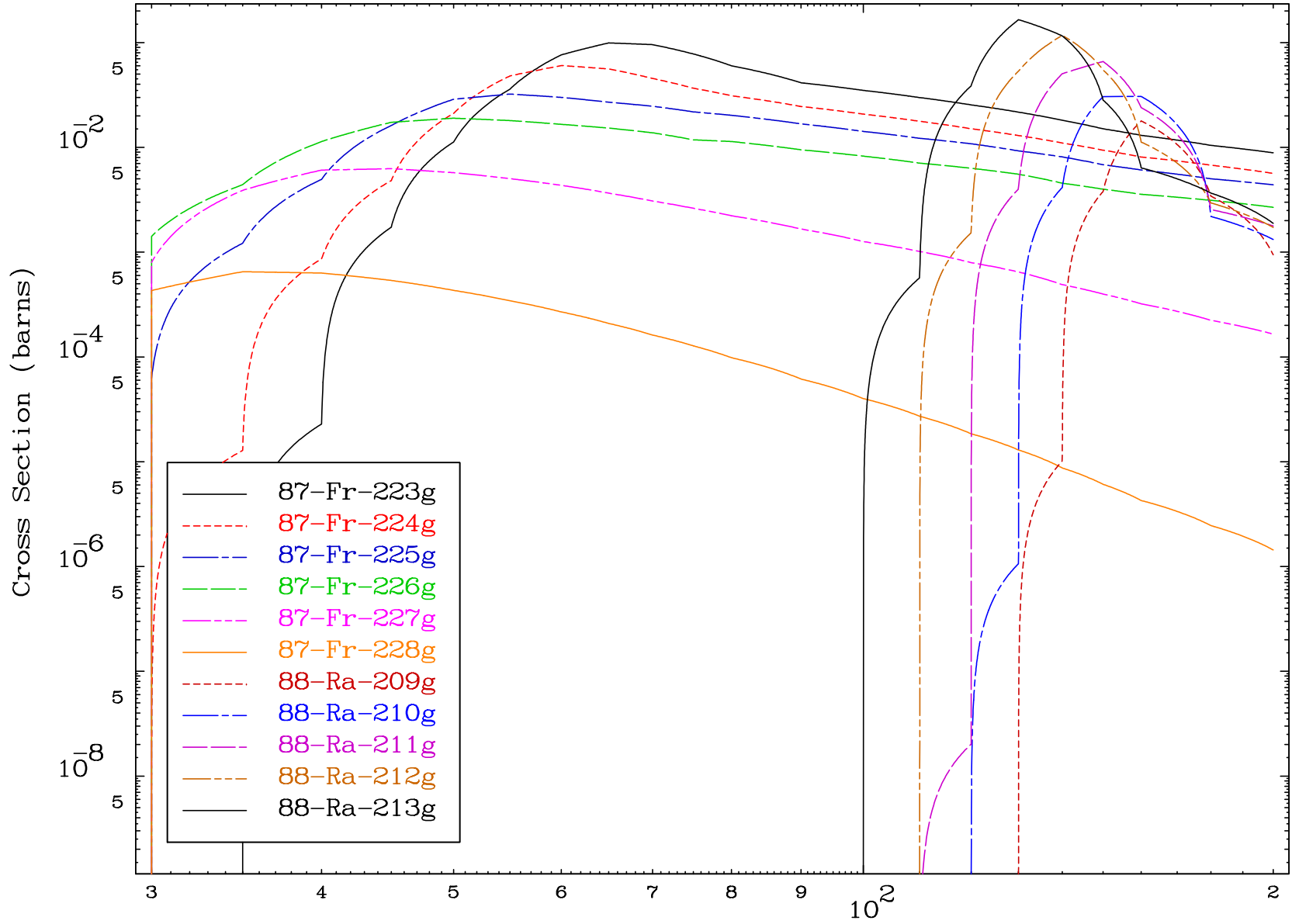








Radionuclide Production Cross Section

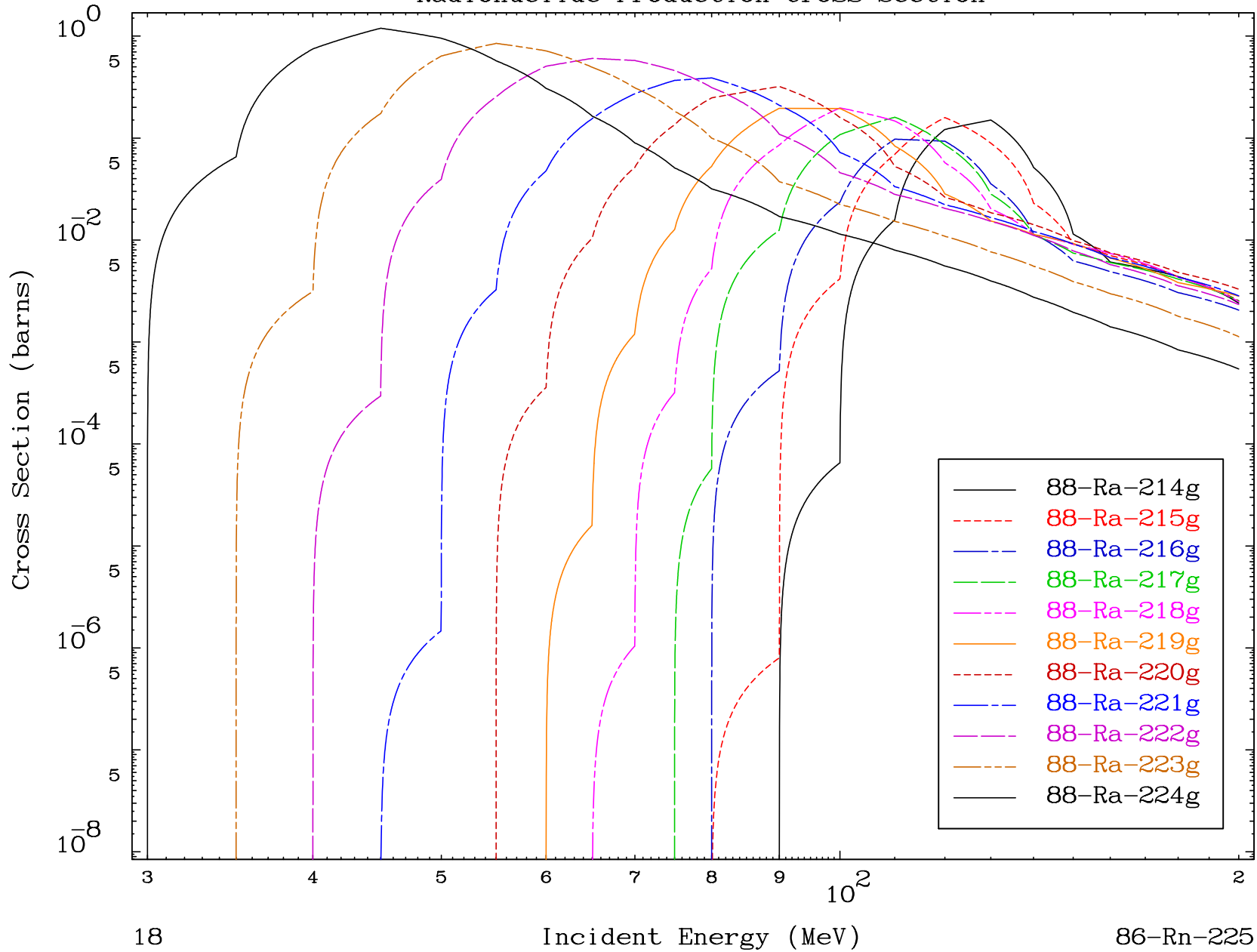


MAT 8667

( $\alpha$ , remainder)

86-Rn-225

### Radionuclide Production Cross Section

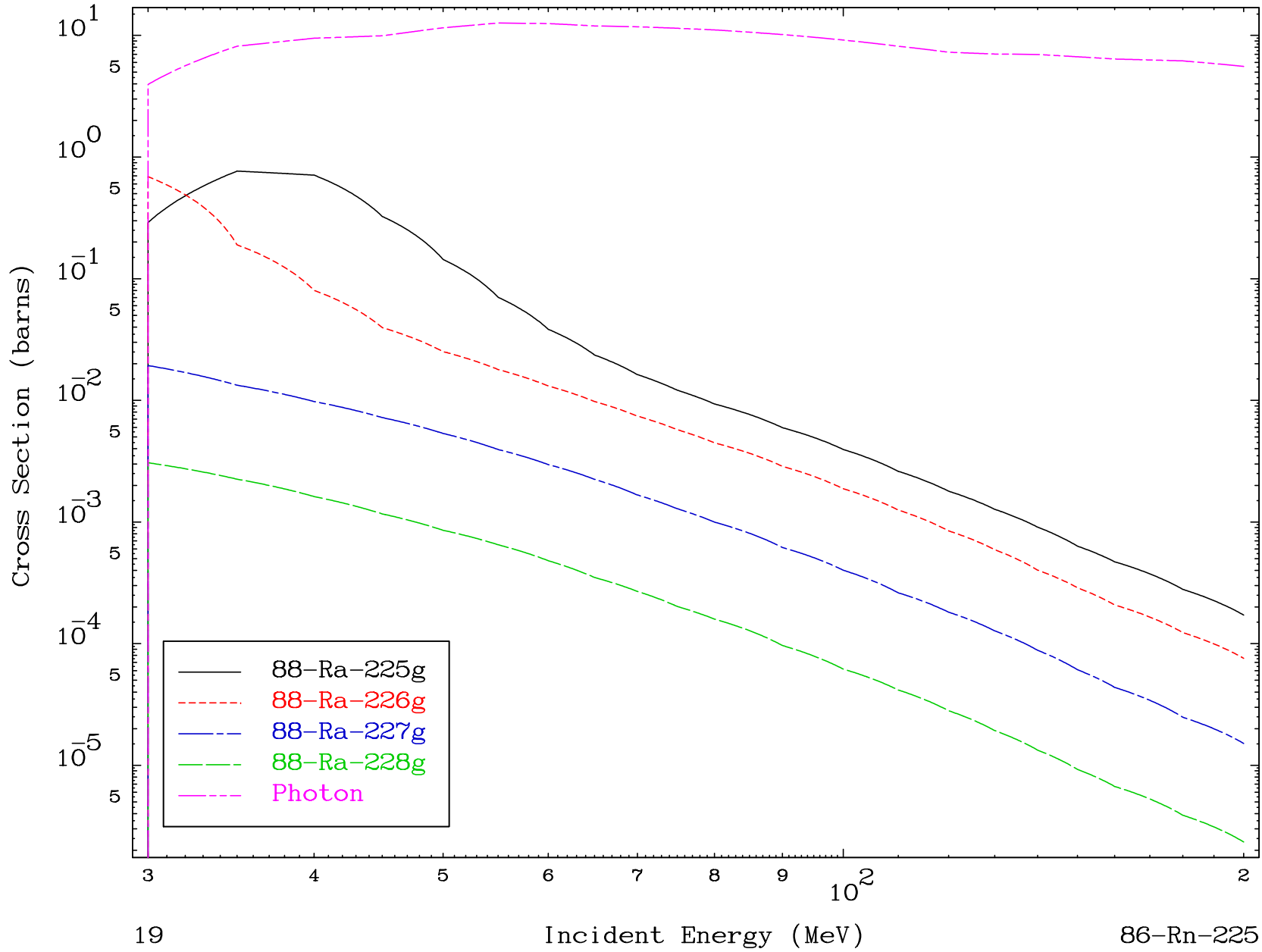


MAT 8667

( $\alpha$ , remainder)

86-Rn-225

### Radionuclide Production Cross Section



19

86-Rn-225

MAT 8667

$\alpha$  Fission  
Radionuclide Production Cross Section

86-Rn-225

