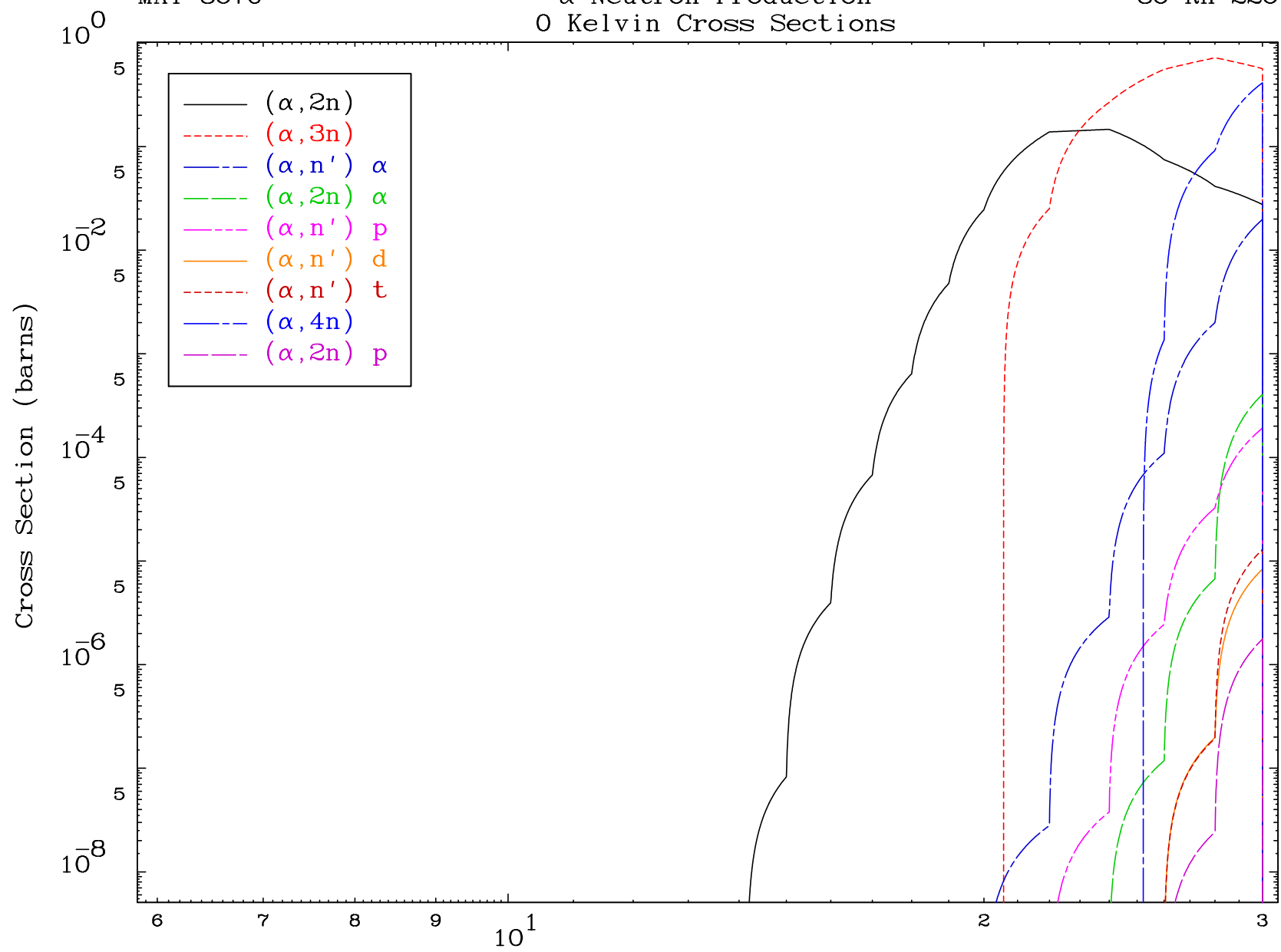


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$\alpha$  Neutron Production  
0 Kelvin Cross Sections

86-Rn-226

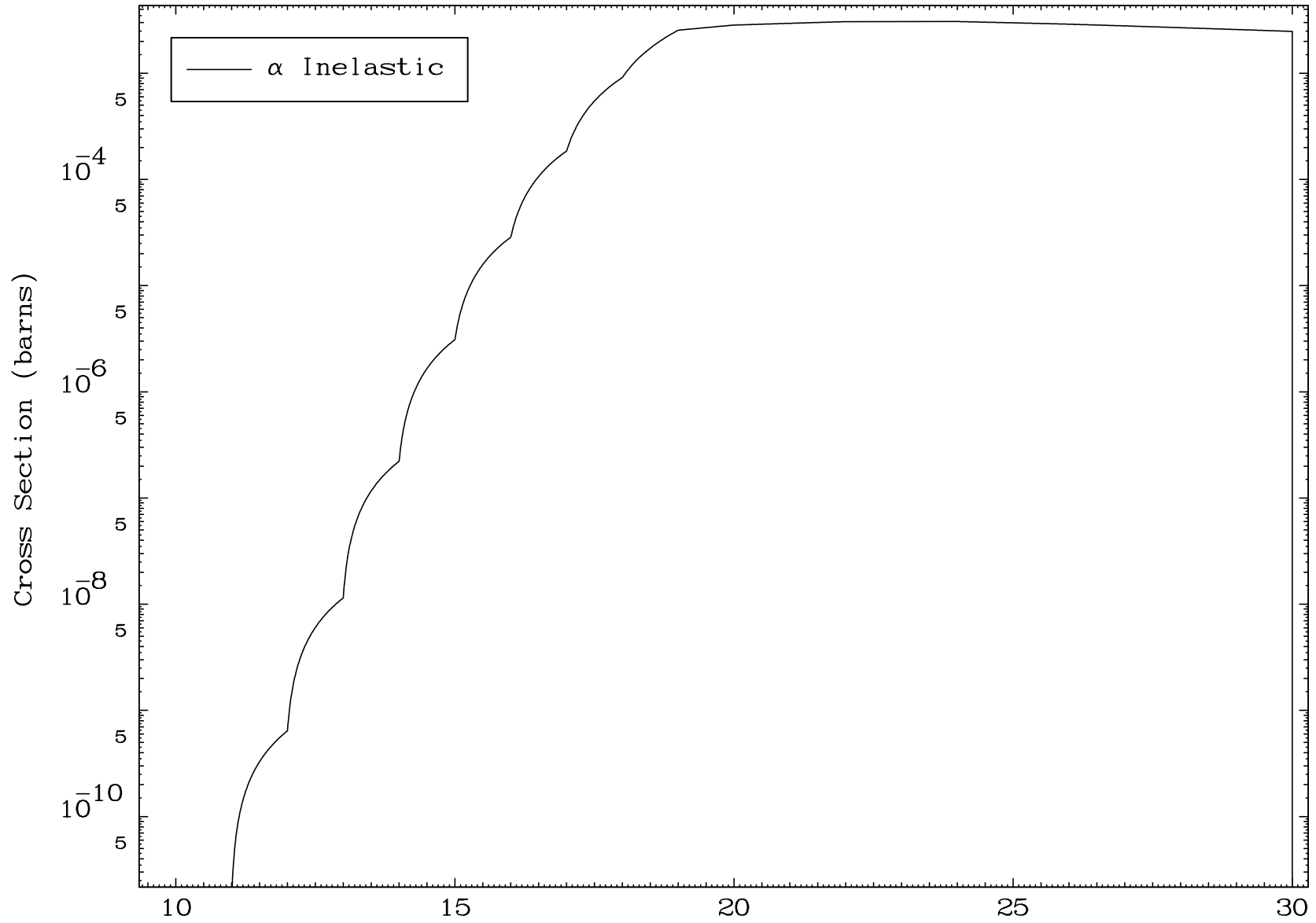


2

Incident Energy (MeV)

86-Rn-226

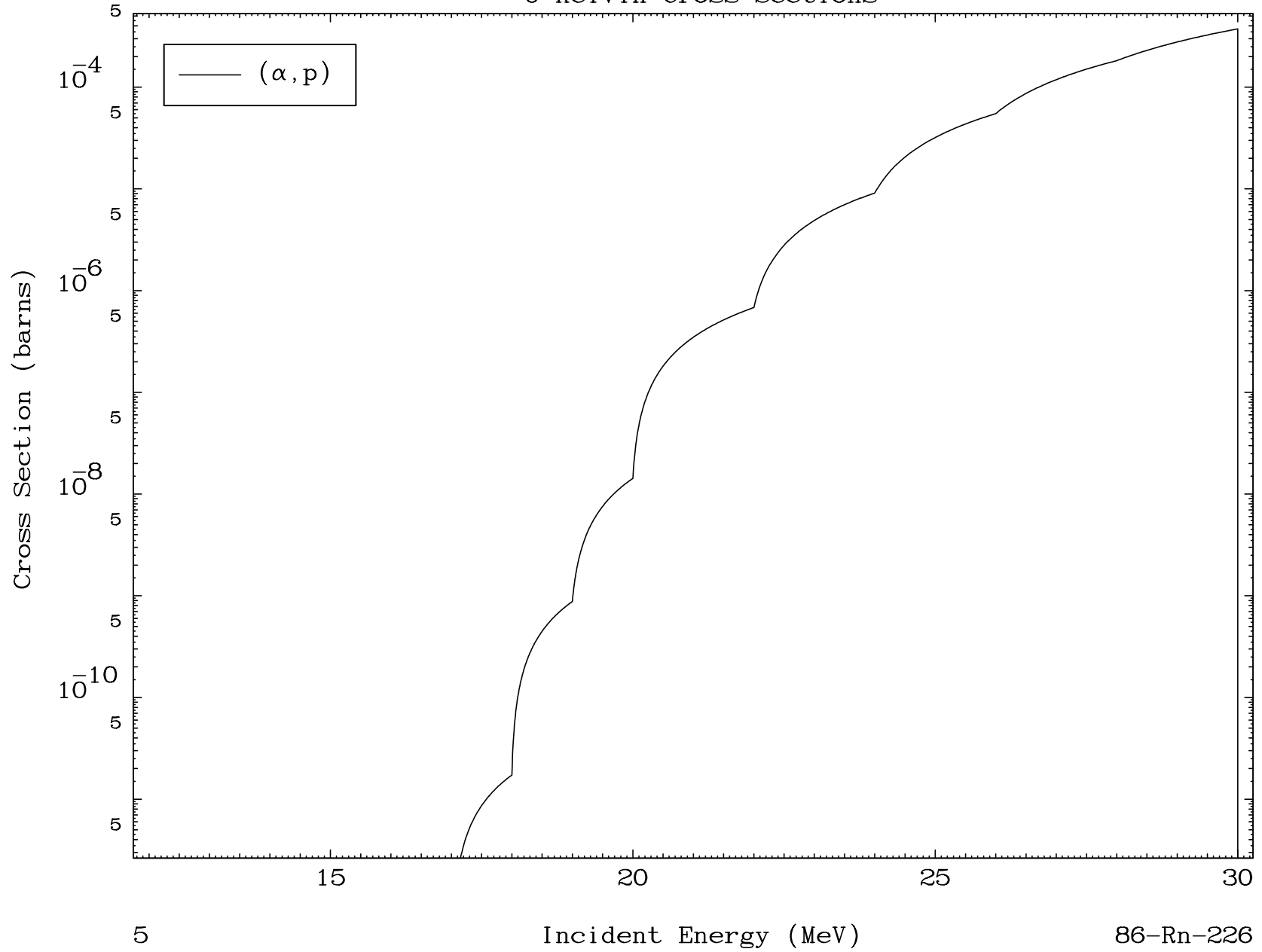


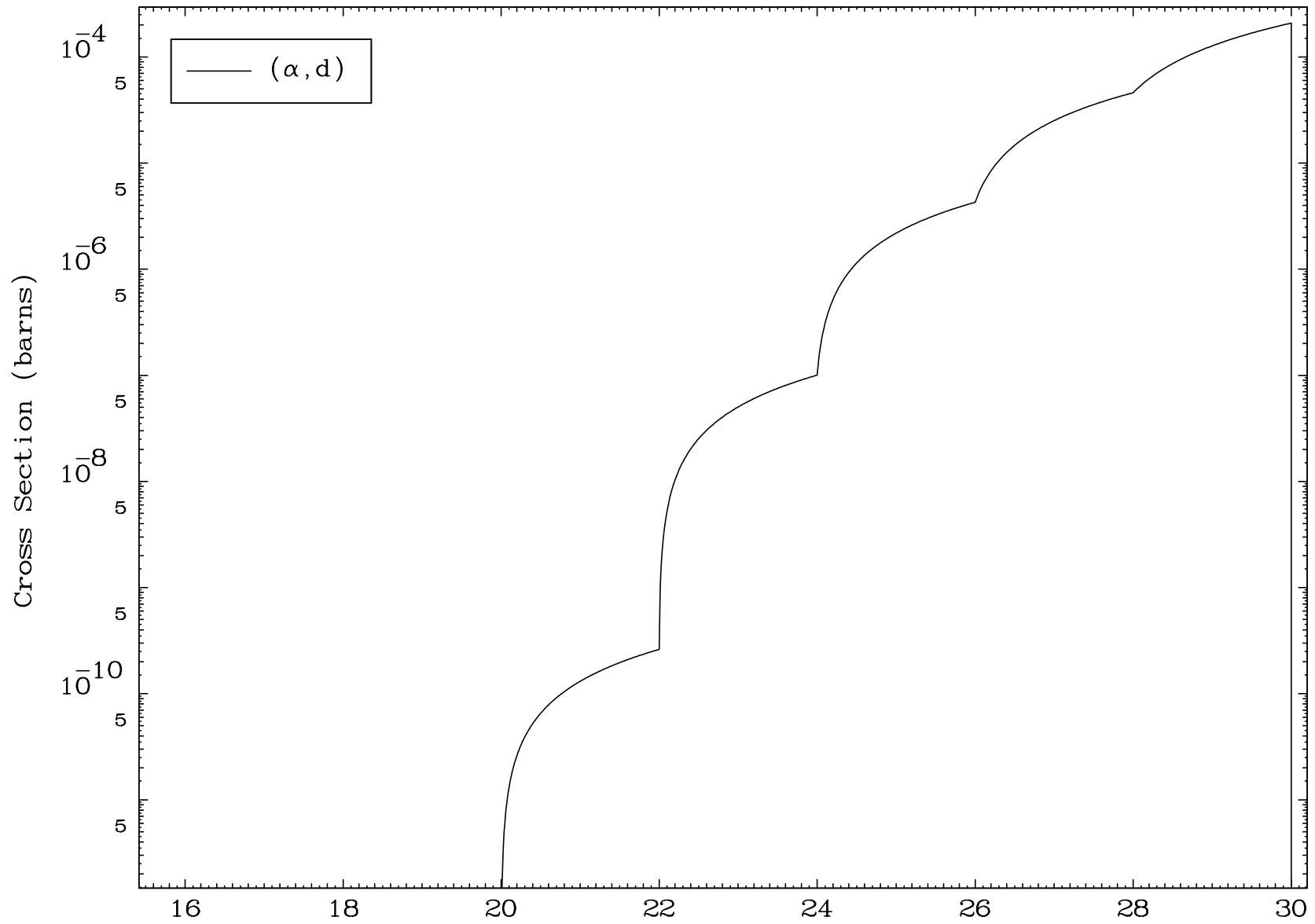


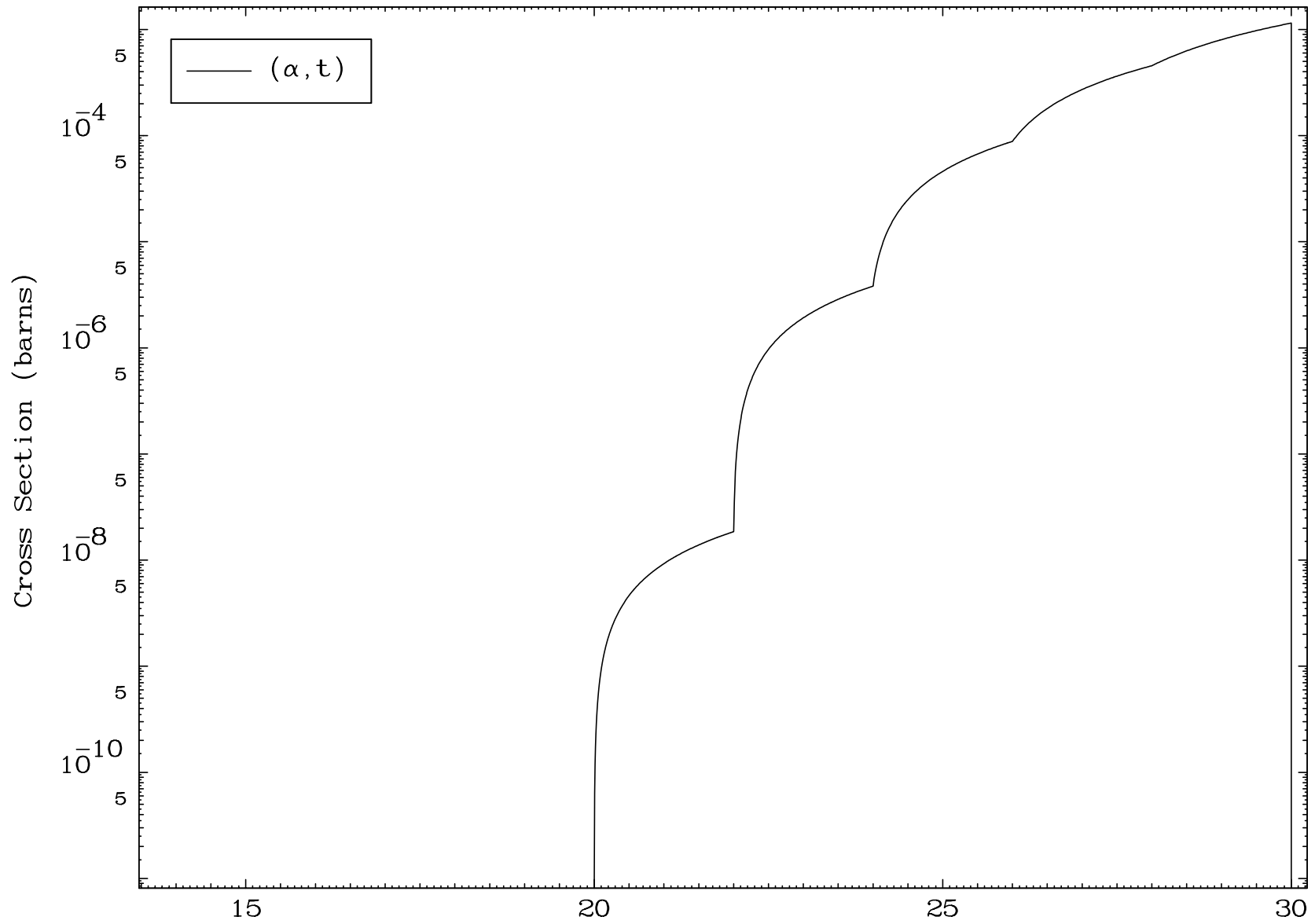
MAT 8670

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections

86-Rn-226



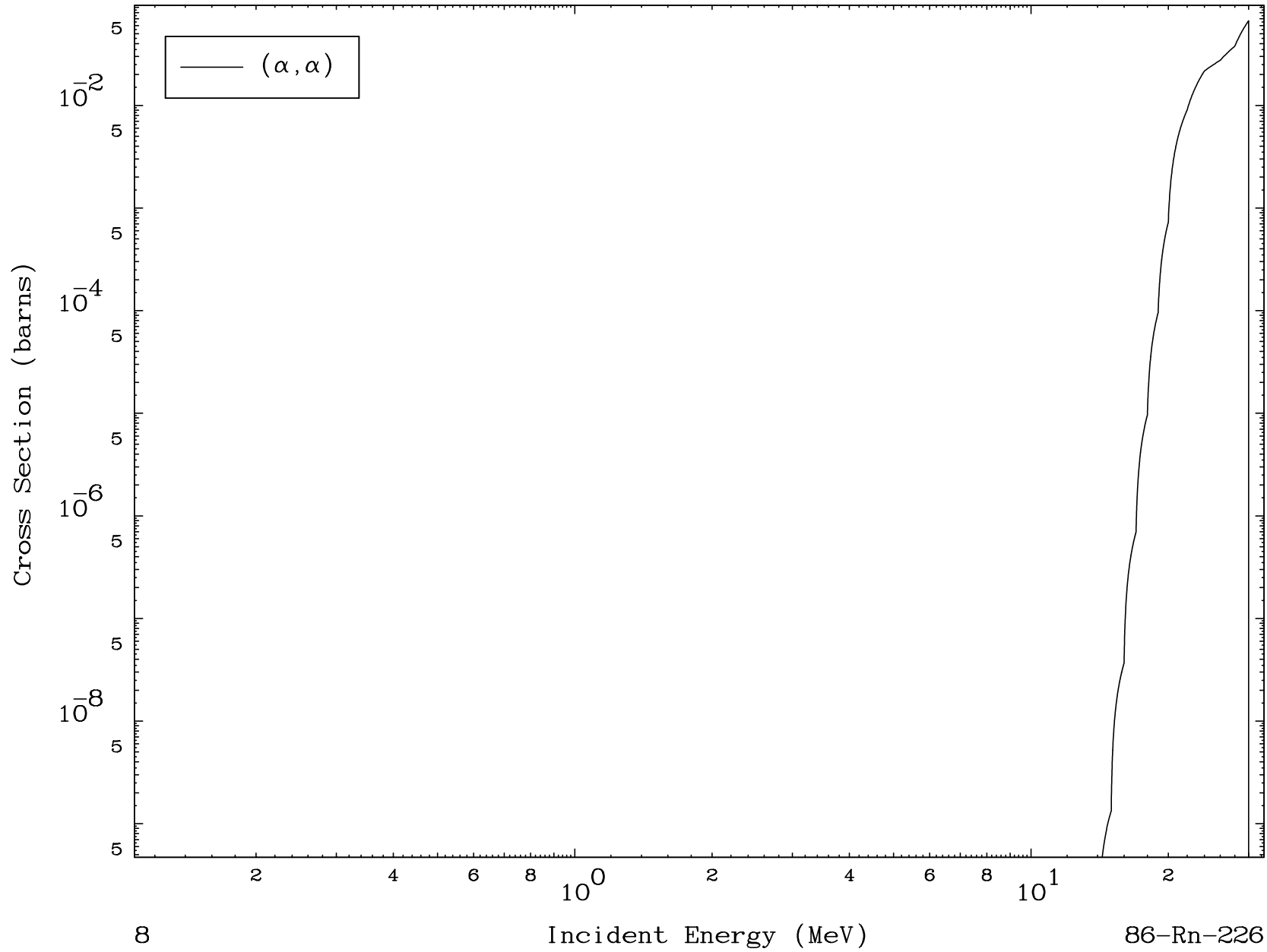




MAT 8670

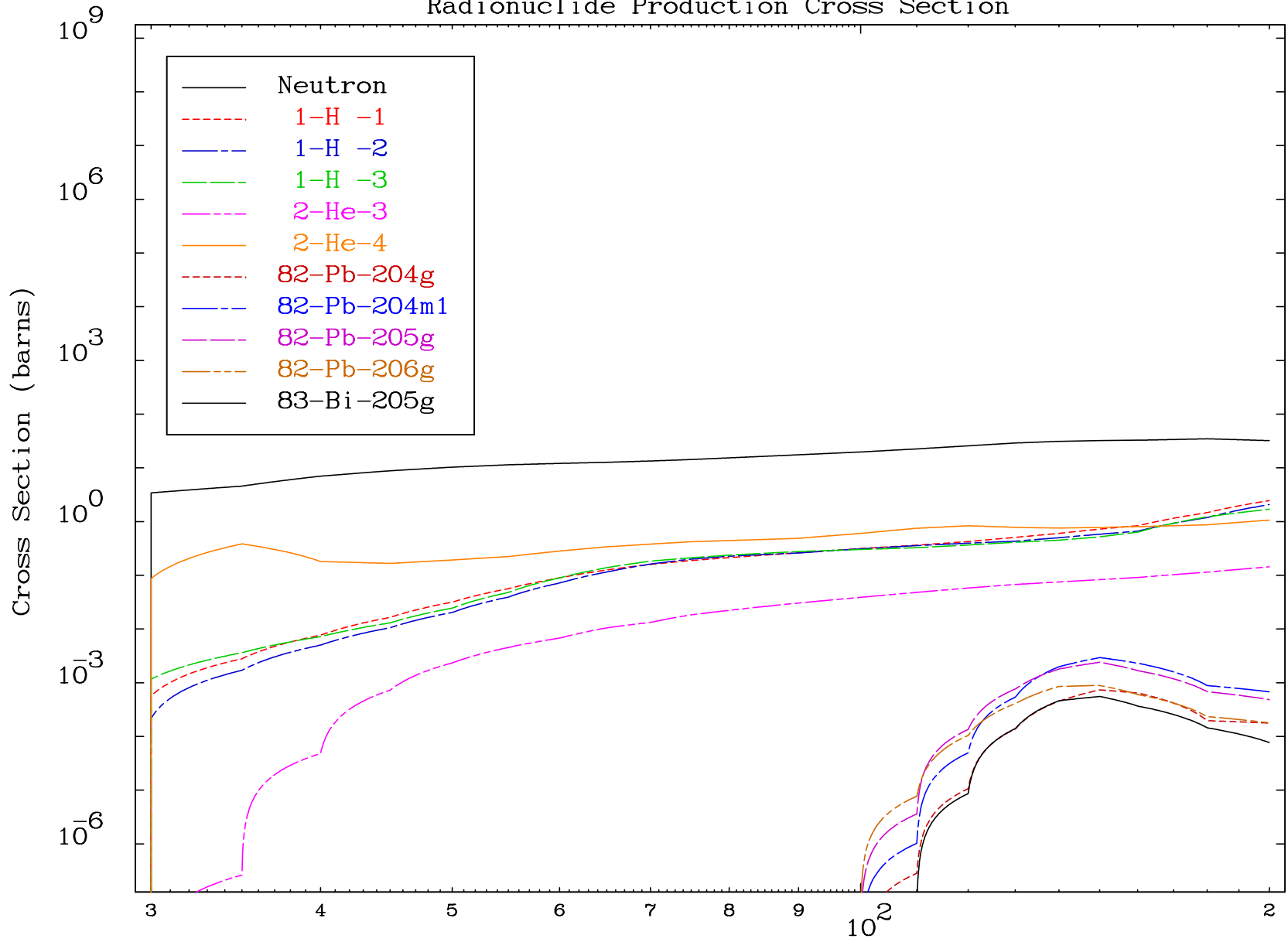
( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

86-Rn-226

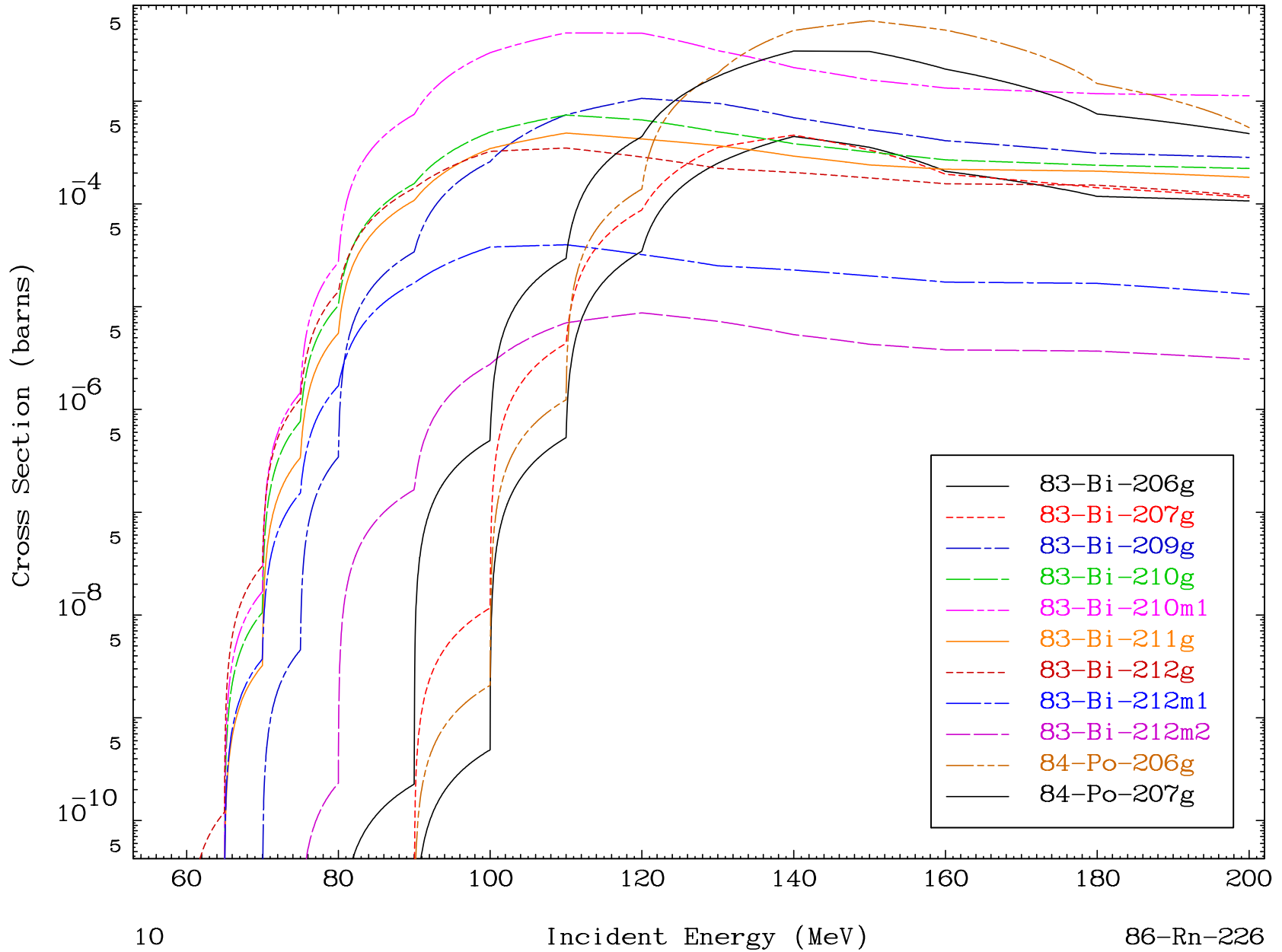




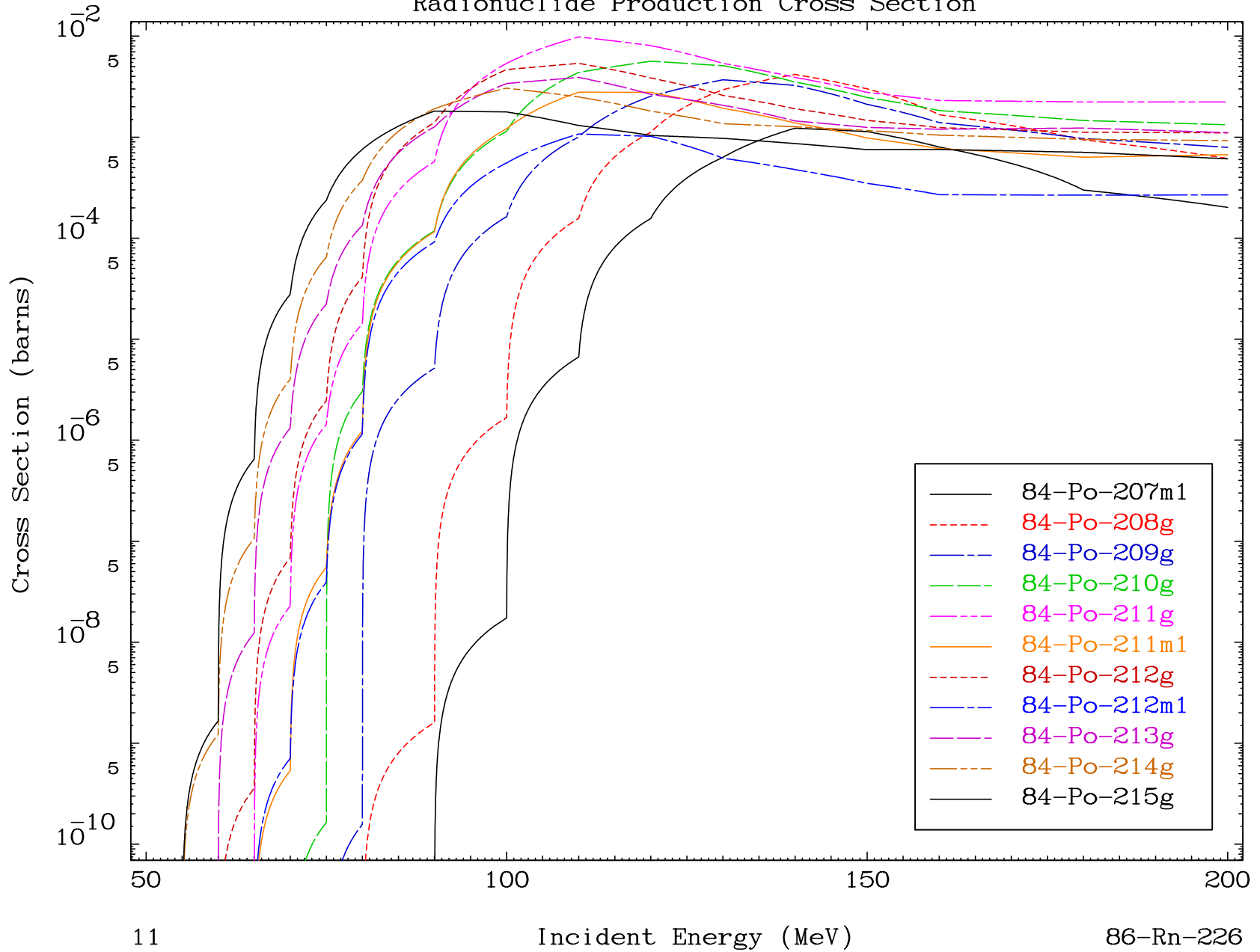
Radionuclide Production Cross Section

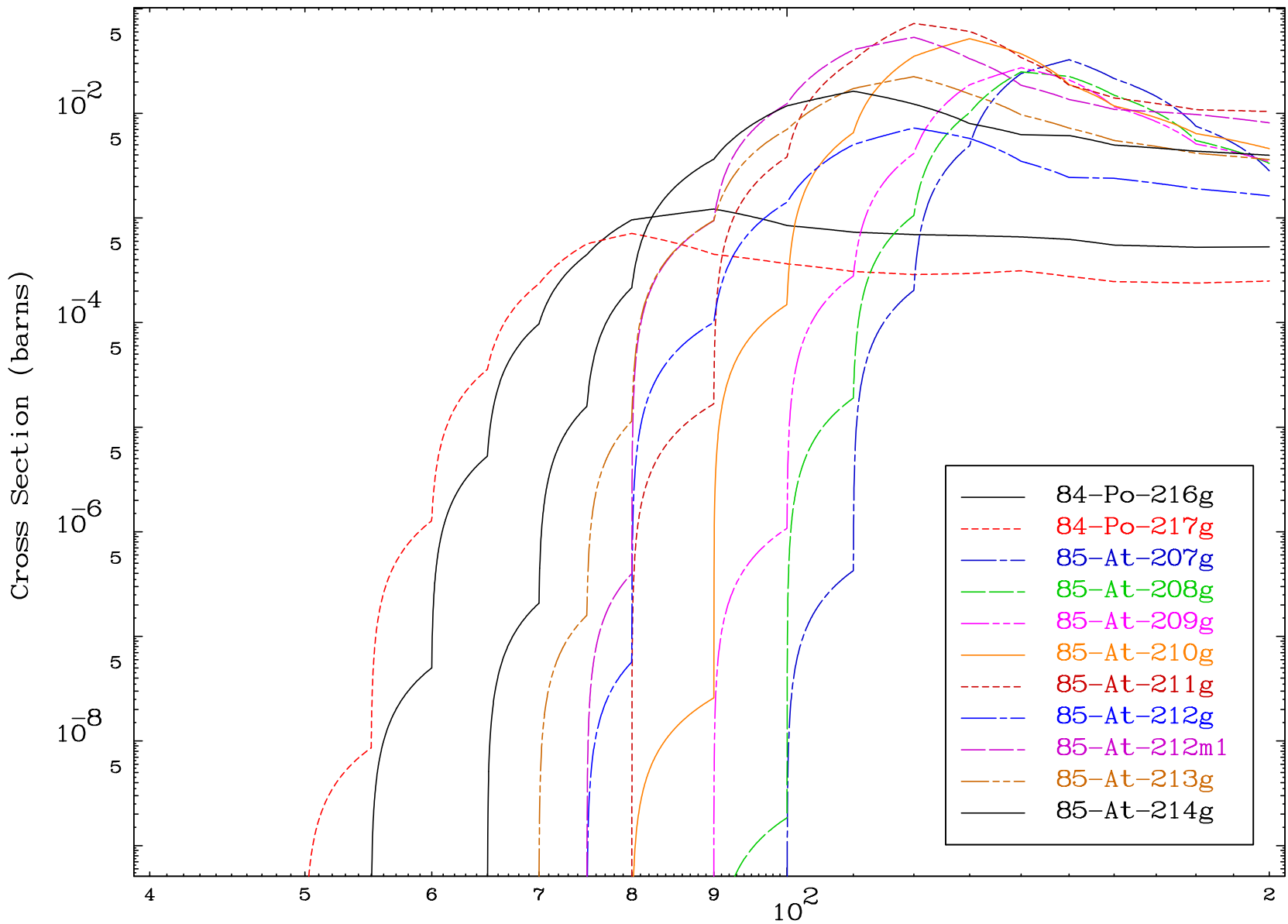


Radionuclide Production Cross Section

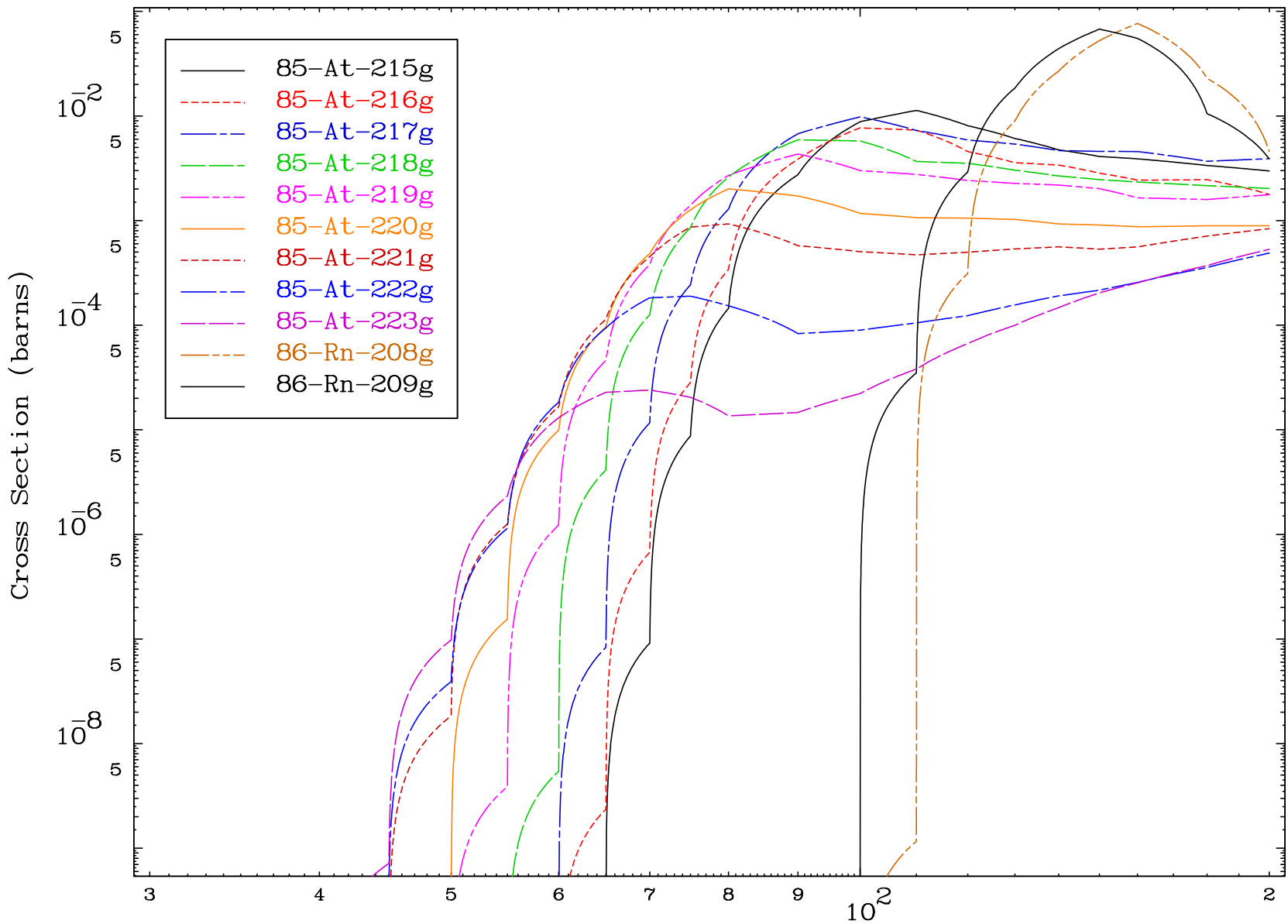


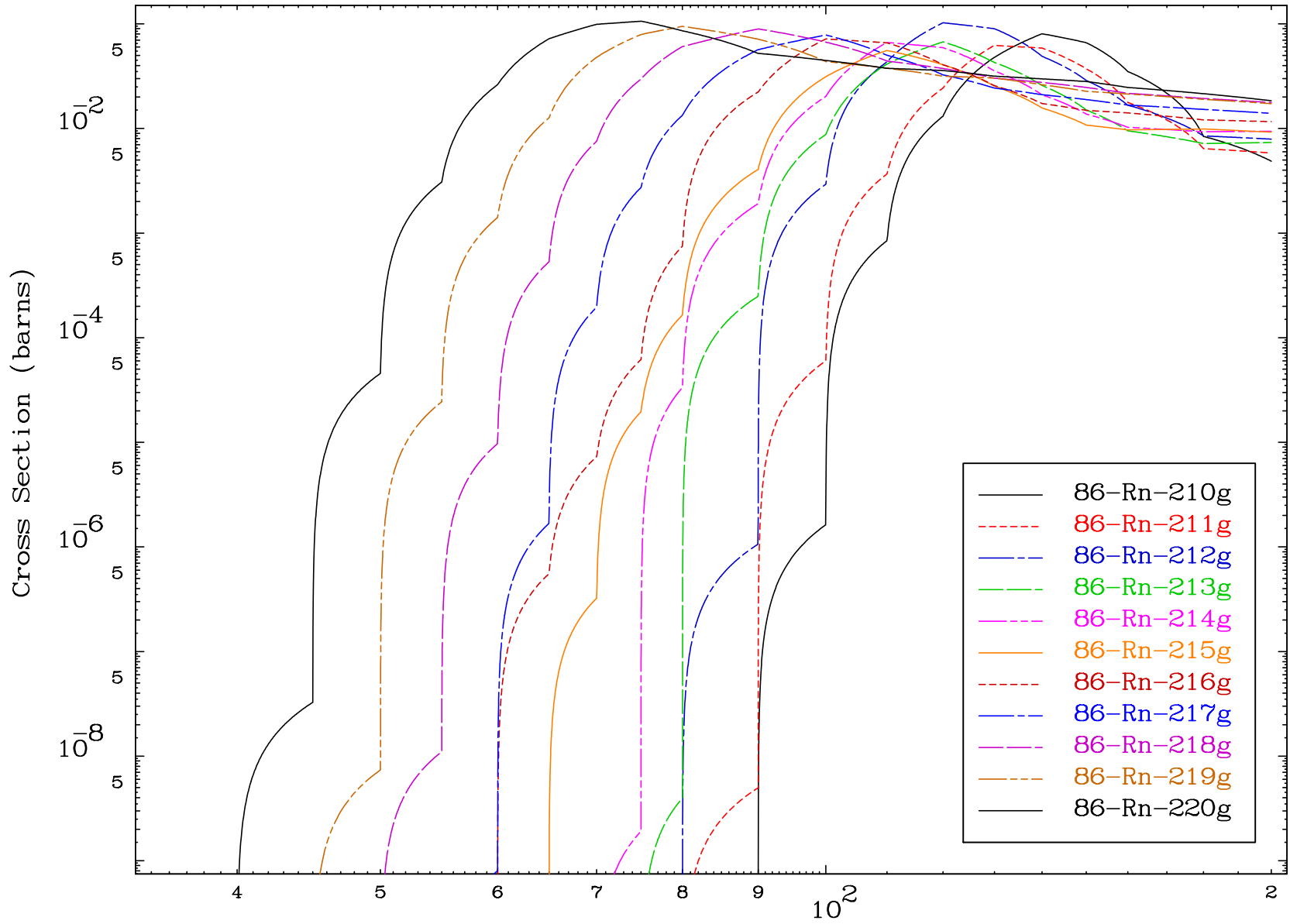
Radionuclide Production Cross Section

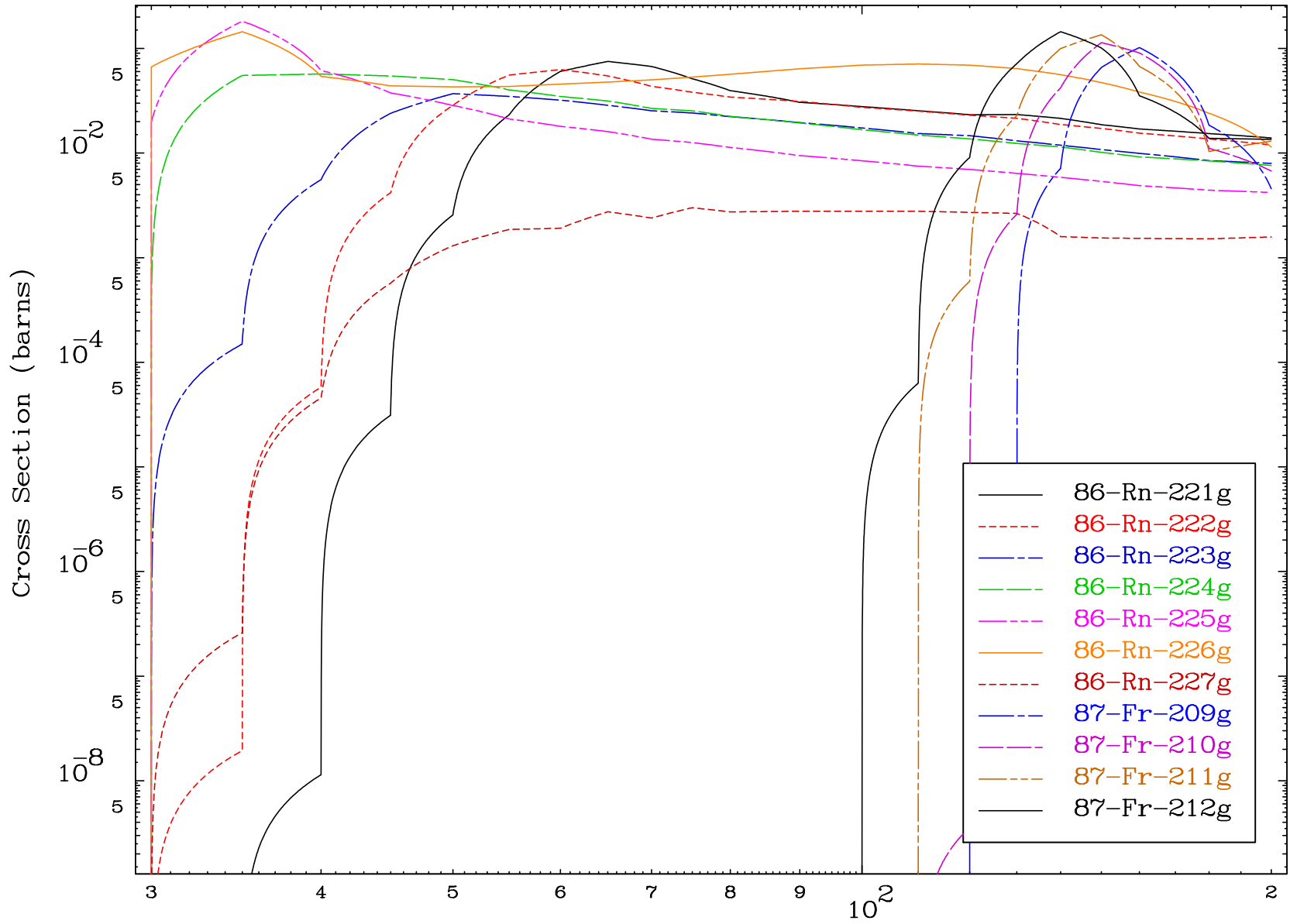


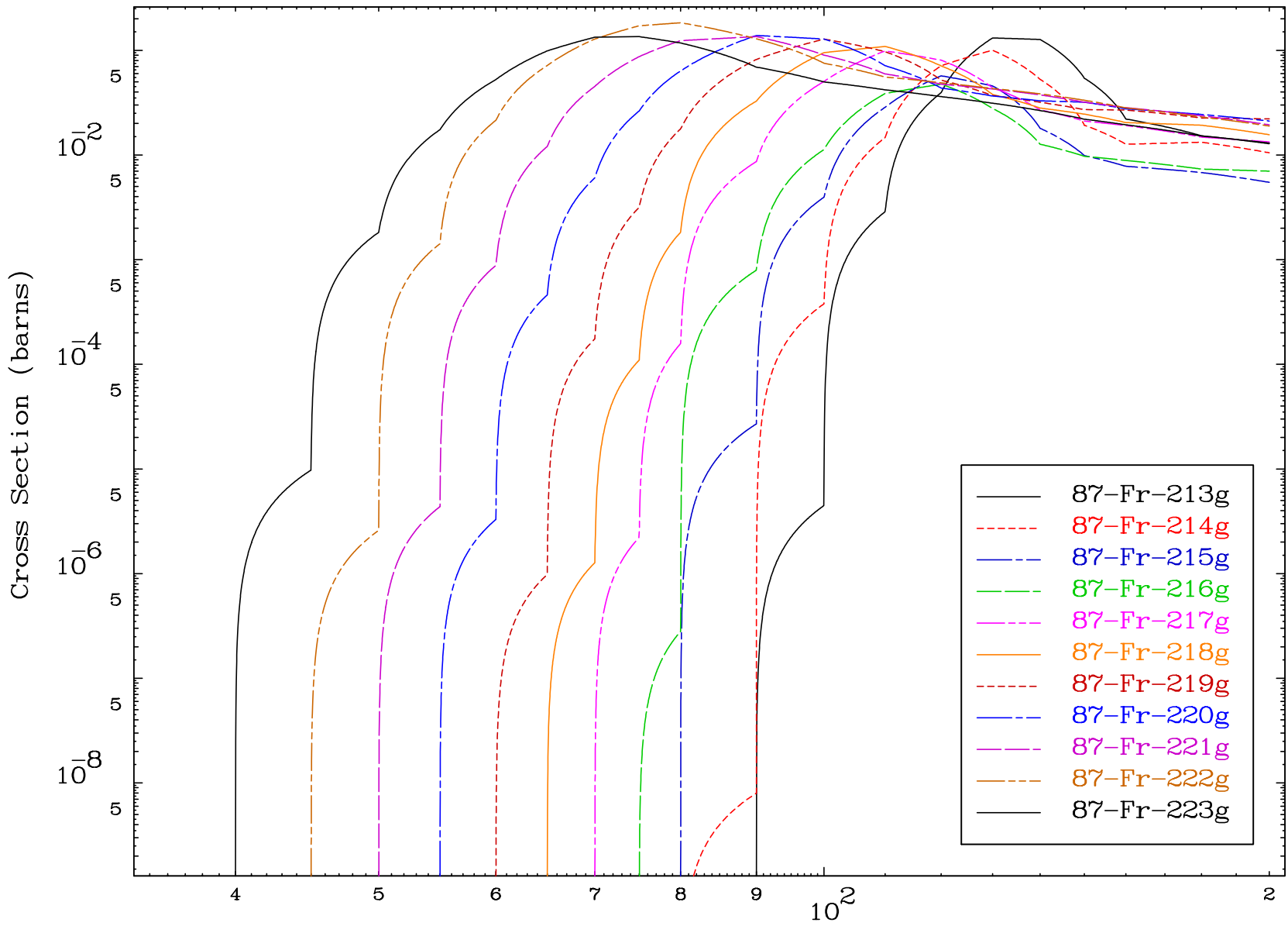


Radionuclide Production Cross Section



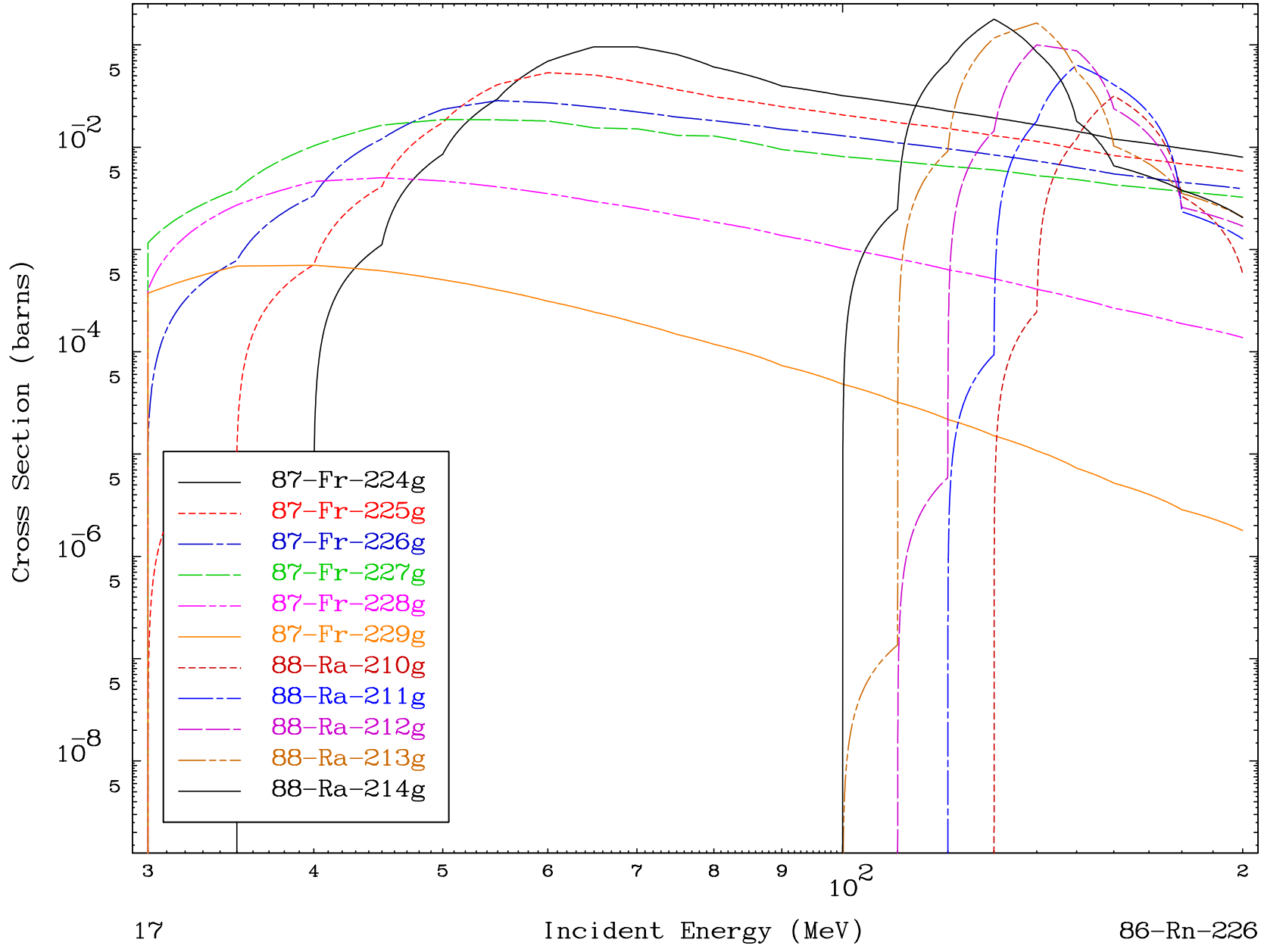








Radionuclide Production Cross Section

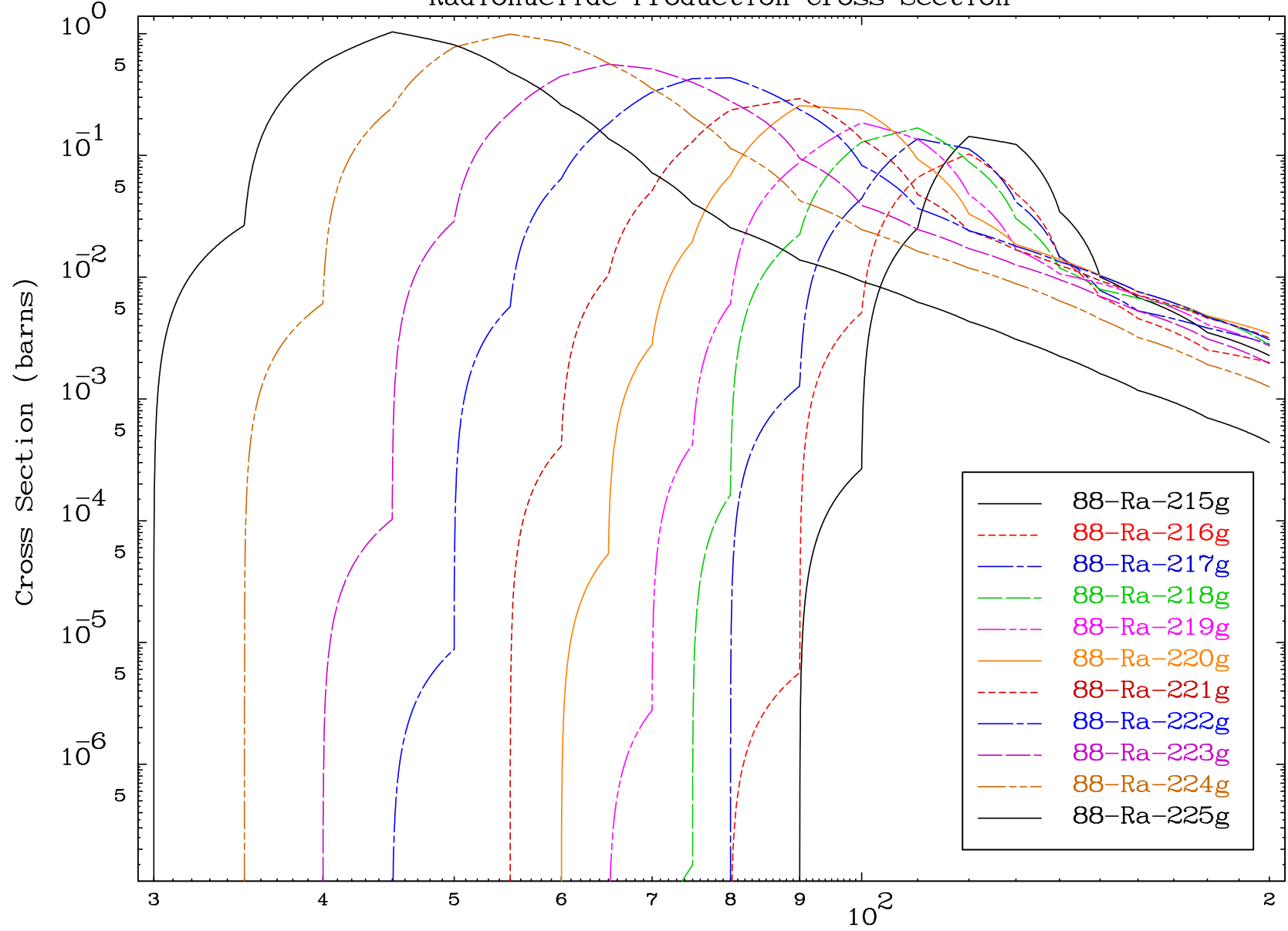


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( $\alpha$ , remainder)

86-Rn-226

### Radionuclide Production Cross Section



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Incident Energy (MeV)

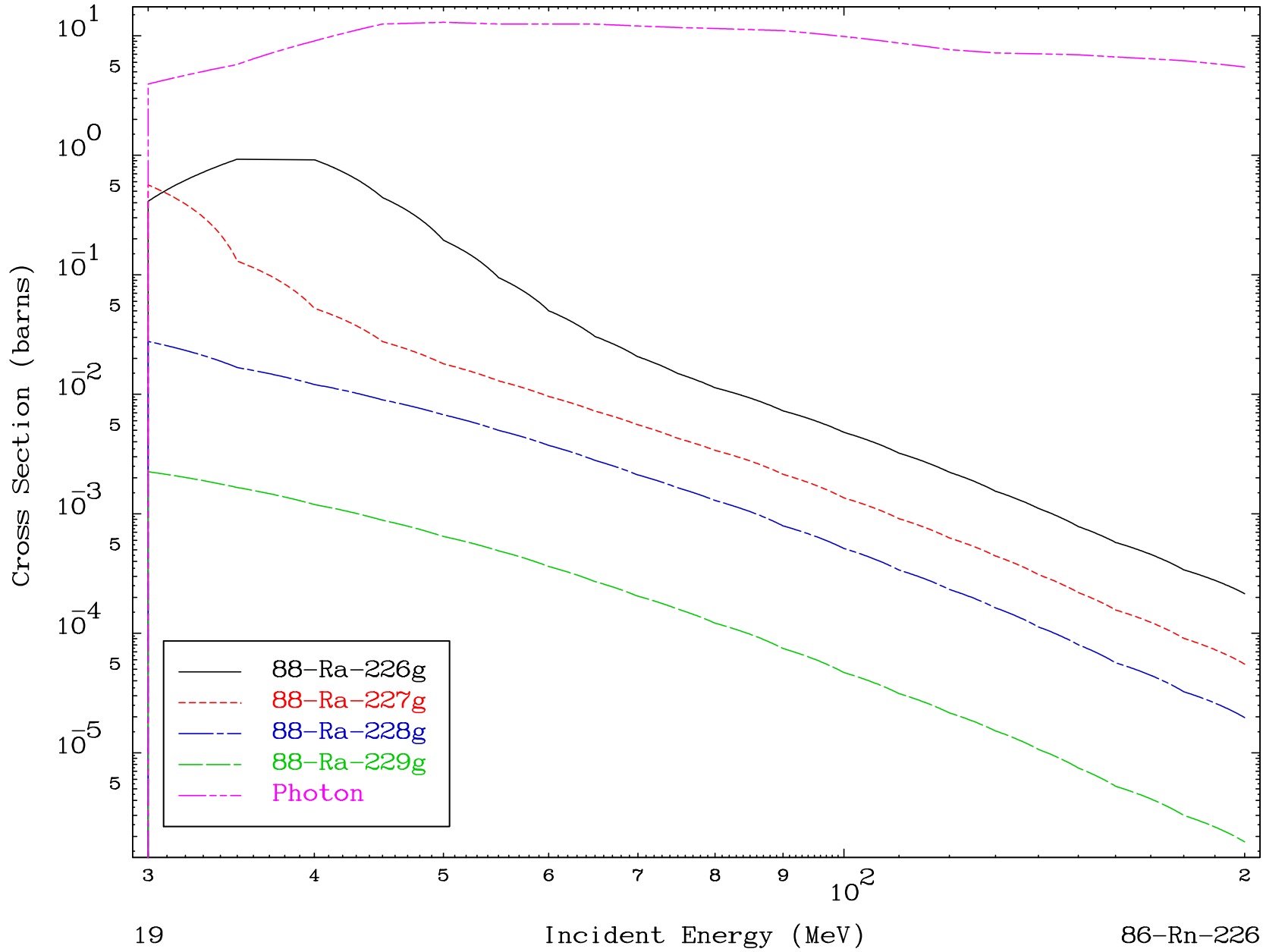
86-Rn-226

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( $\alpha$ , remainder)

86-Rn-226

Radionuclide Production Cross Section



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$\alpha$  Fission  
Radionuclide Production Cross Section

86-Rn-226

