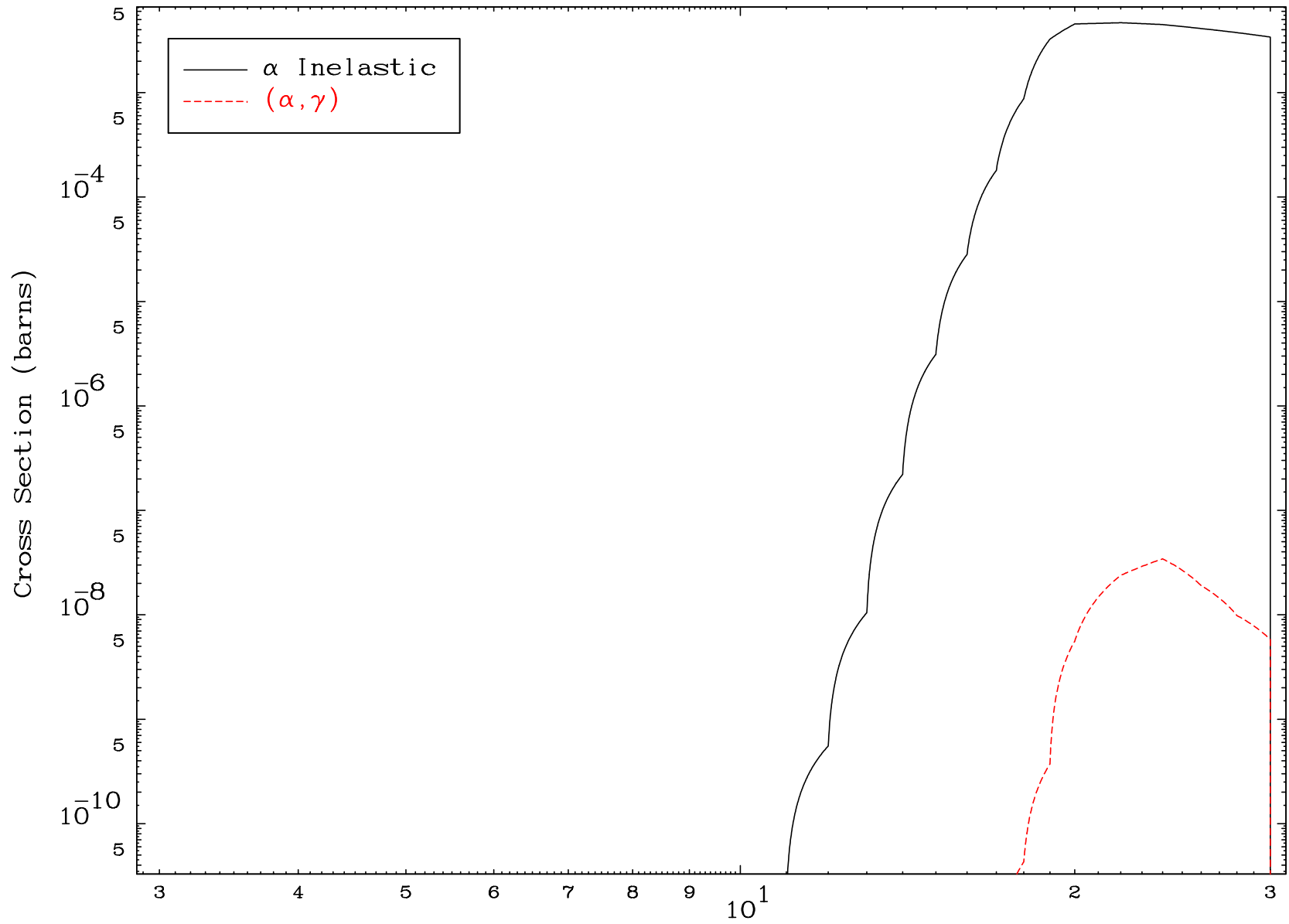


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$\alpha$  Major  
0 Kelvin Cross Sections

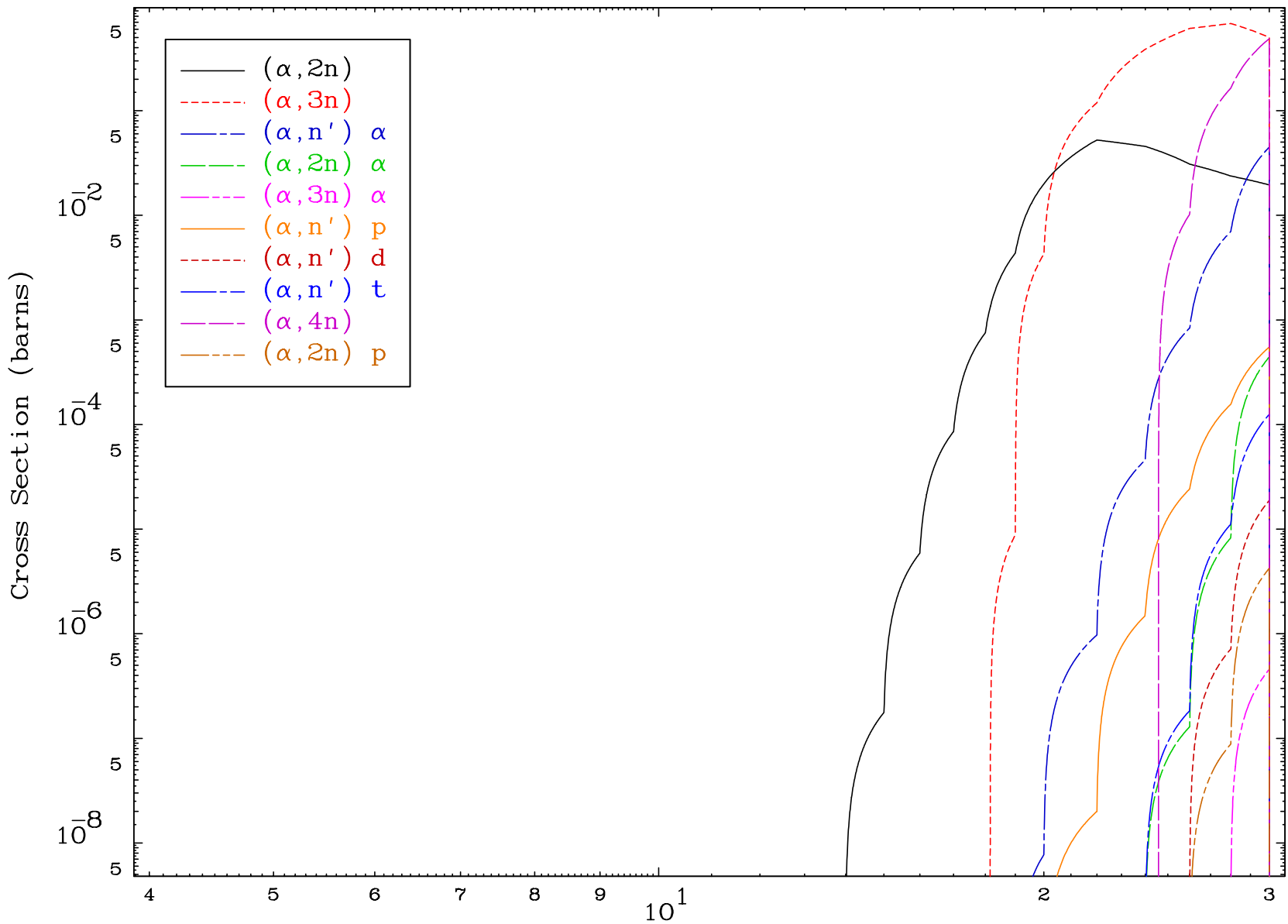
86-Rn-227

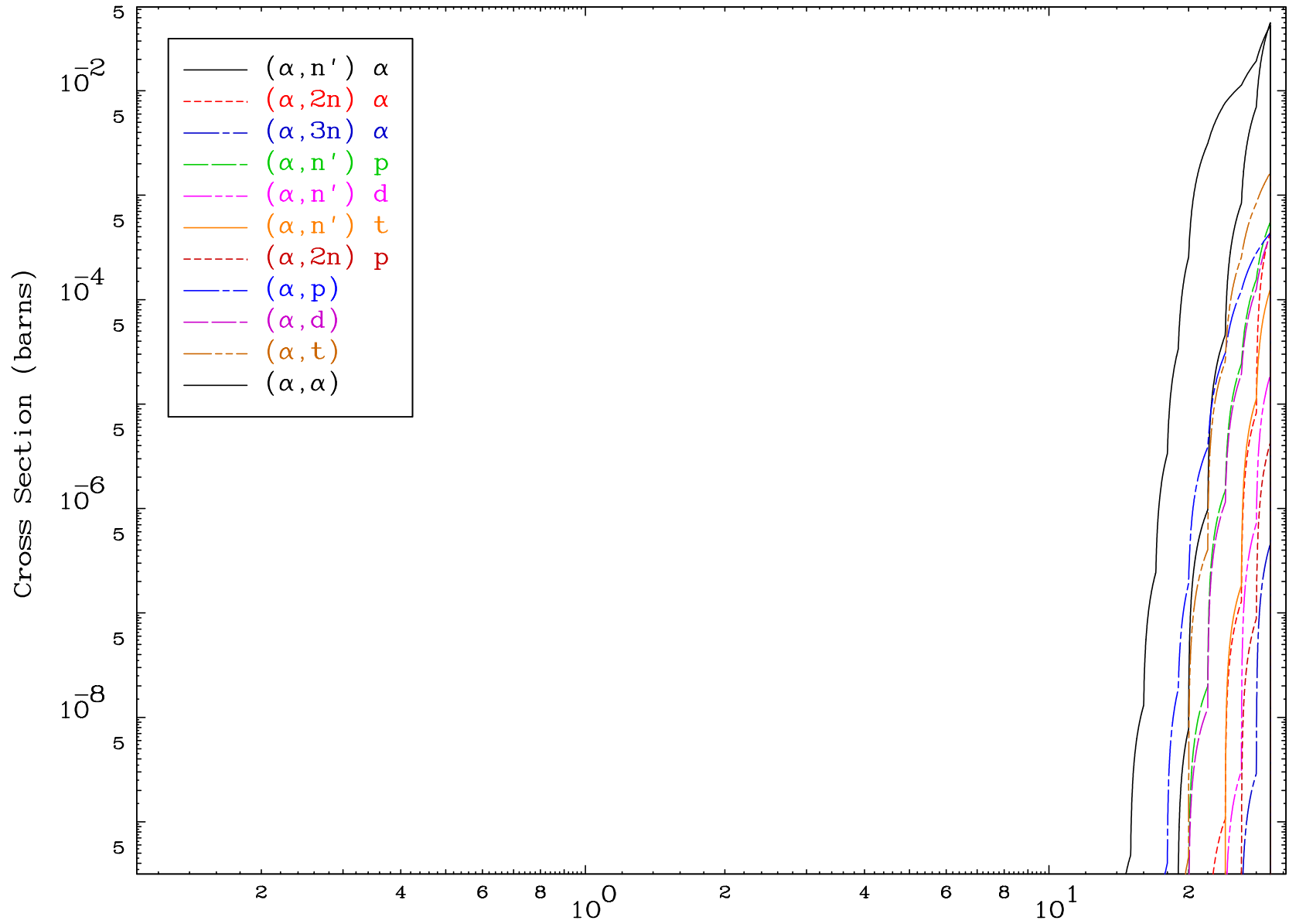


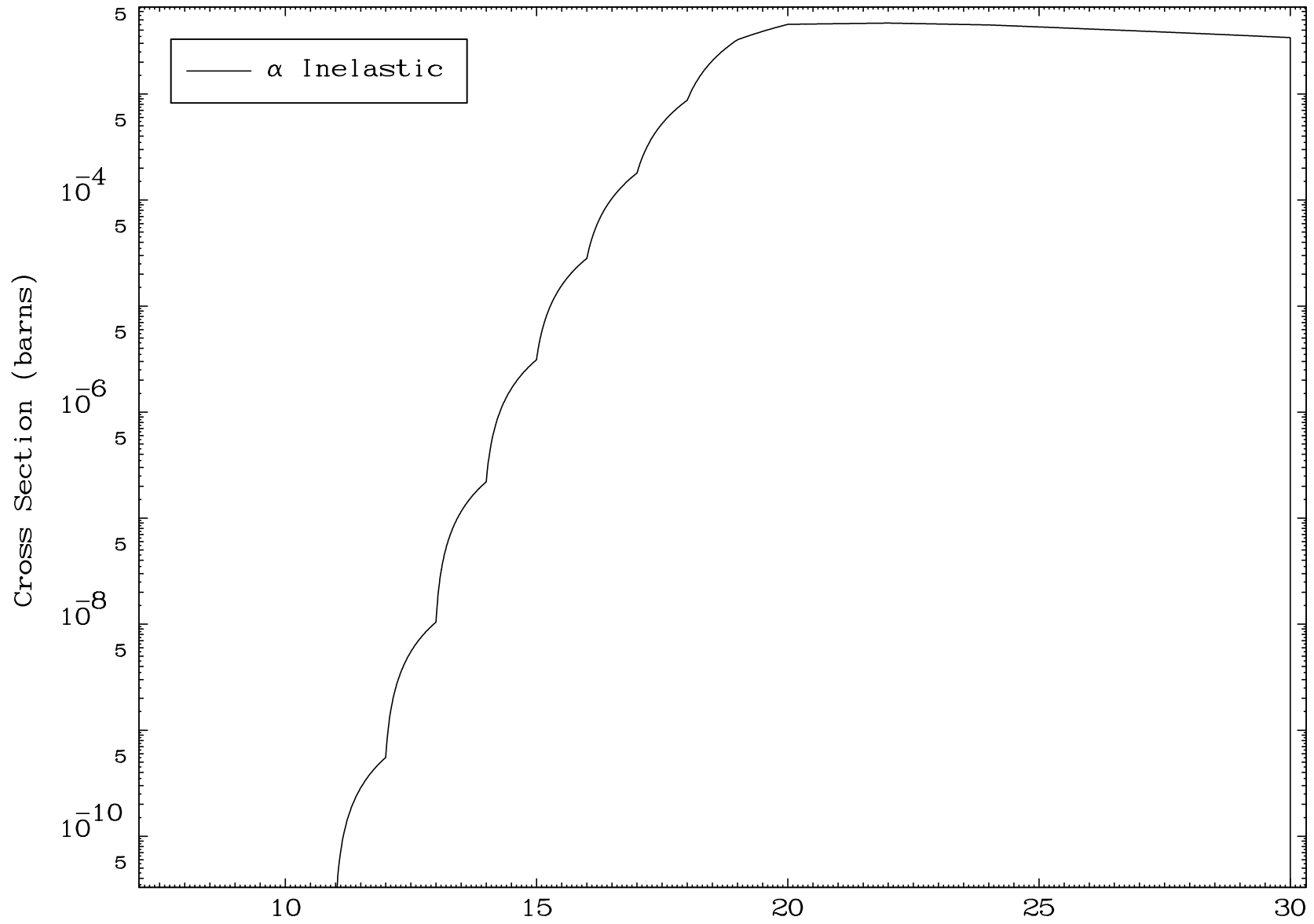
1

Incident Energy (MeV)

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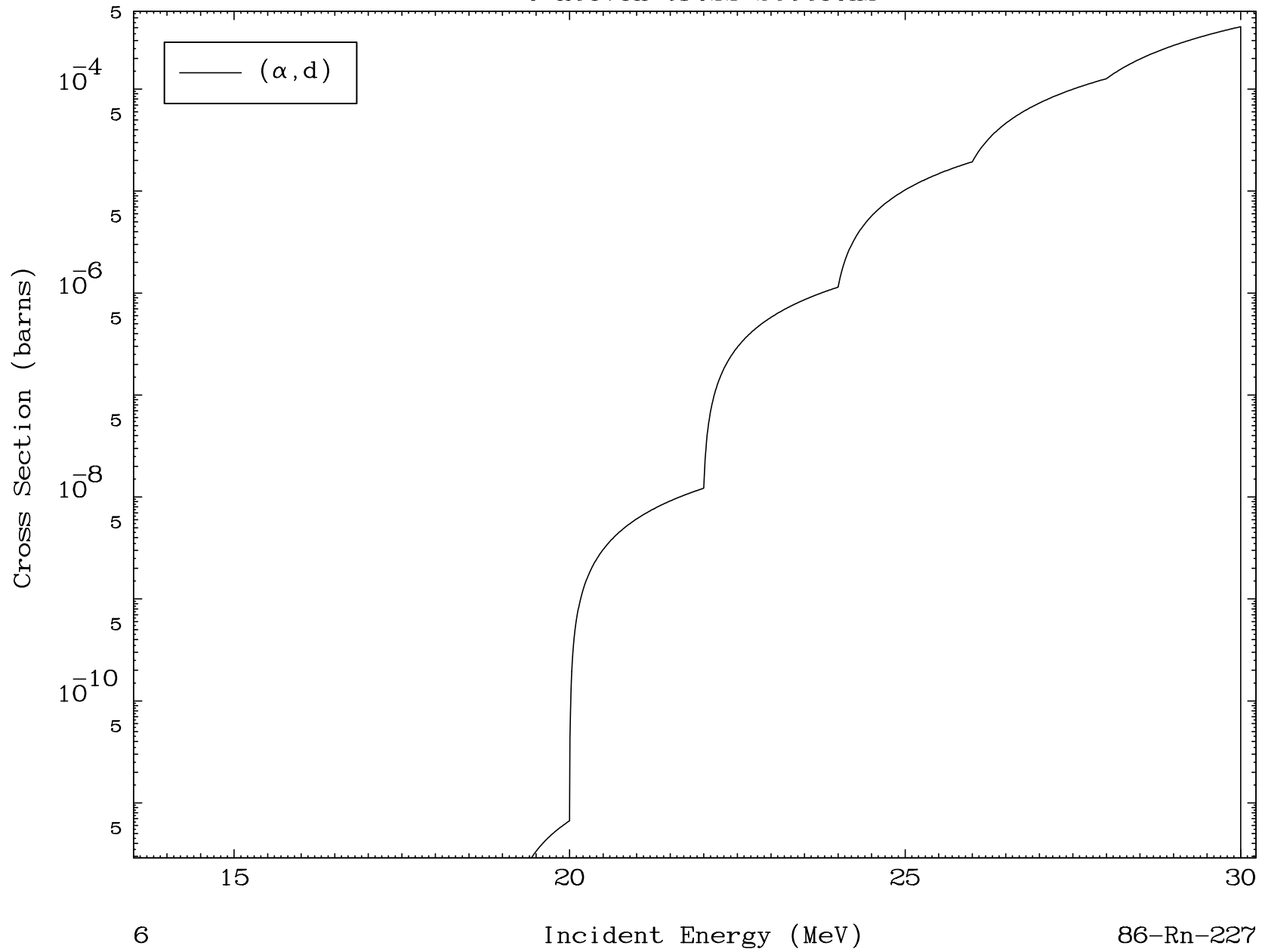




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( $\alpha$ ,d) Levels  
0 Kelvin Cross Sections

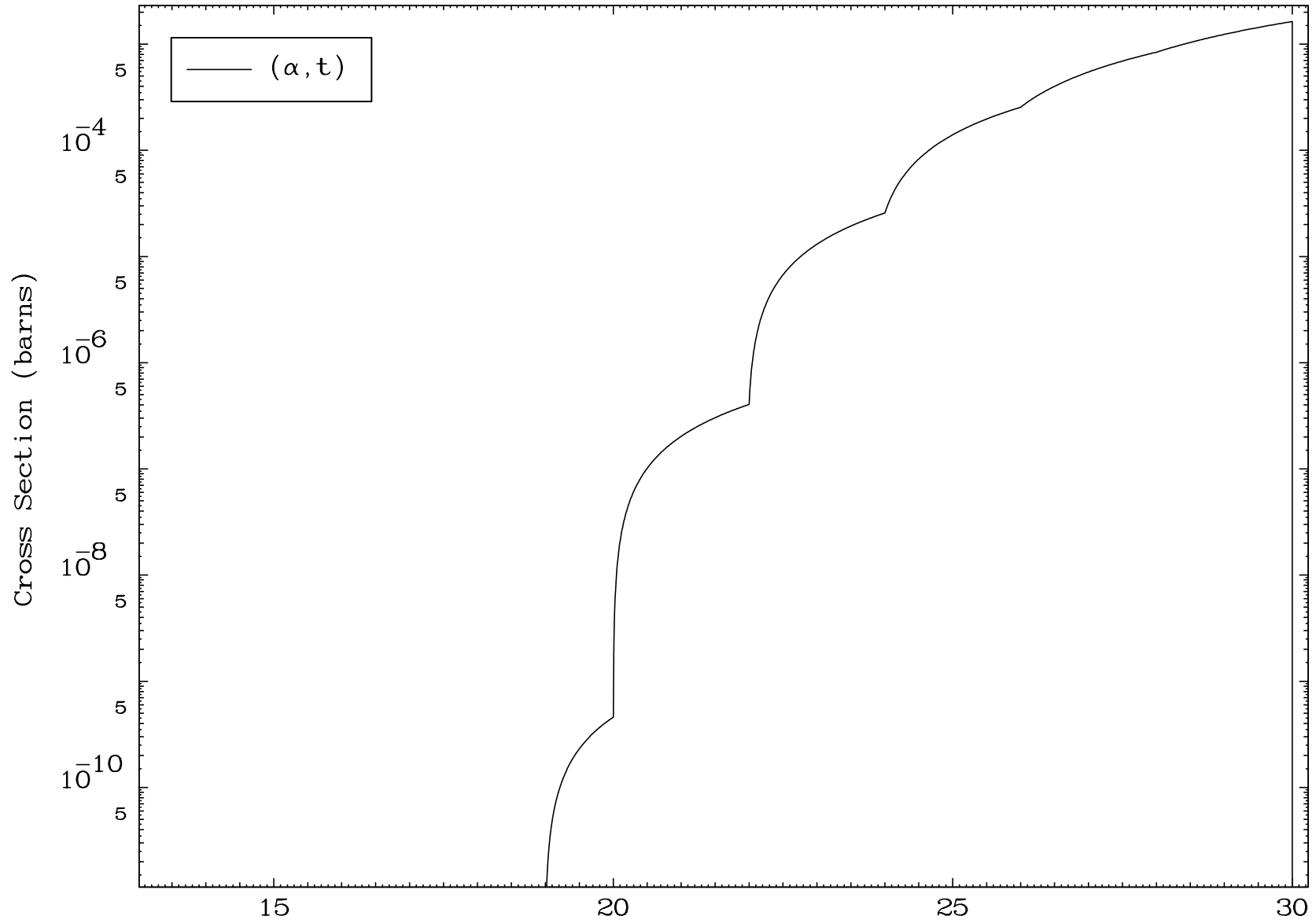
86-Rn-227



6

Incident Energy (MeV)

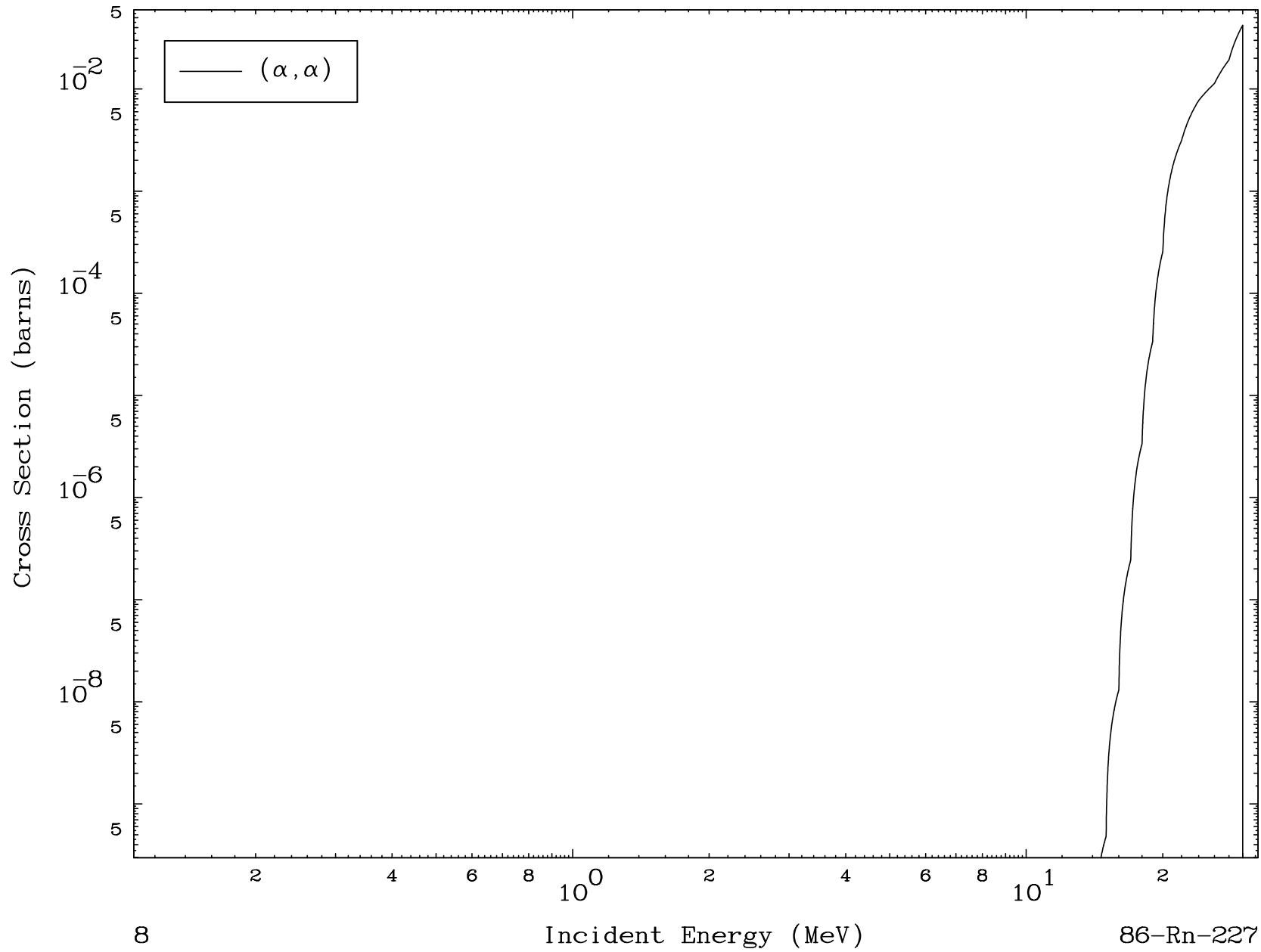
86-Rn-227



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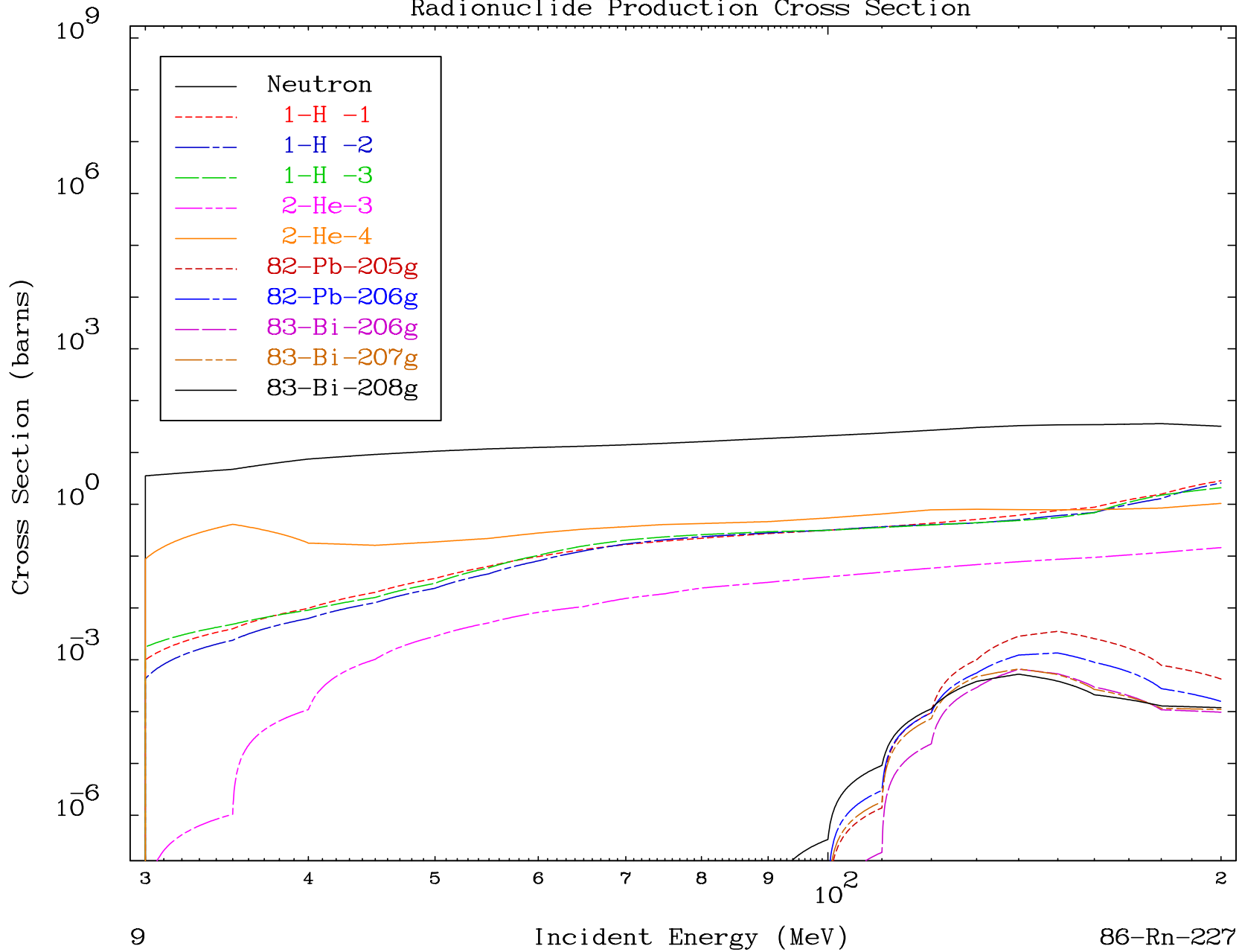
( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

86-Rn-227

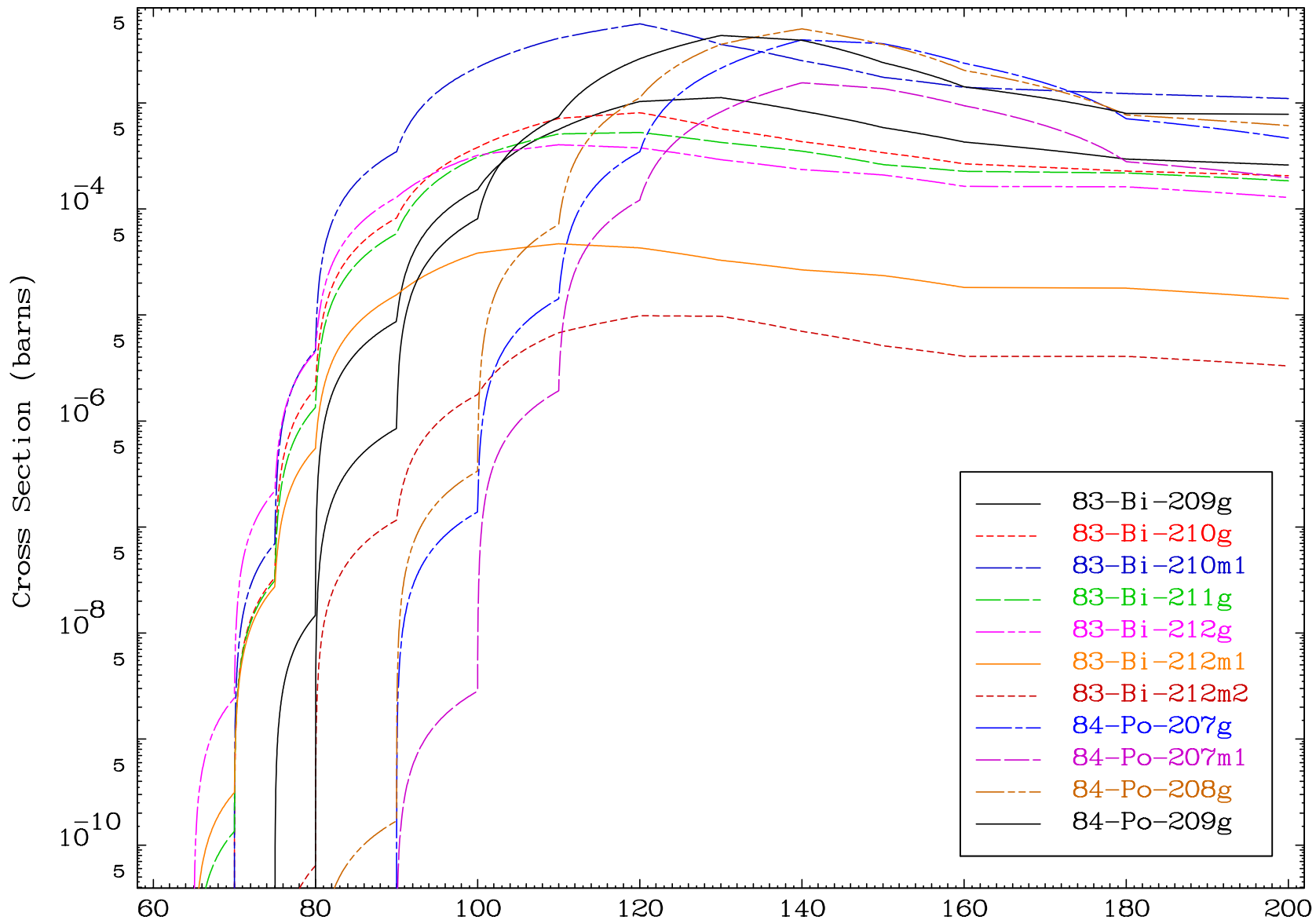




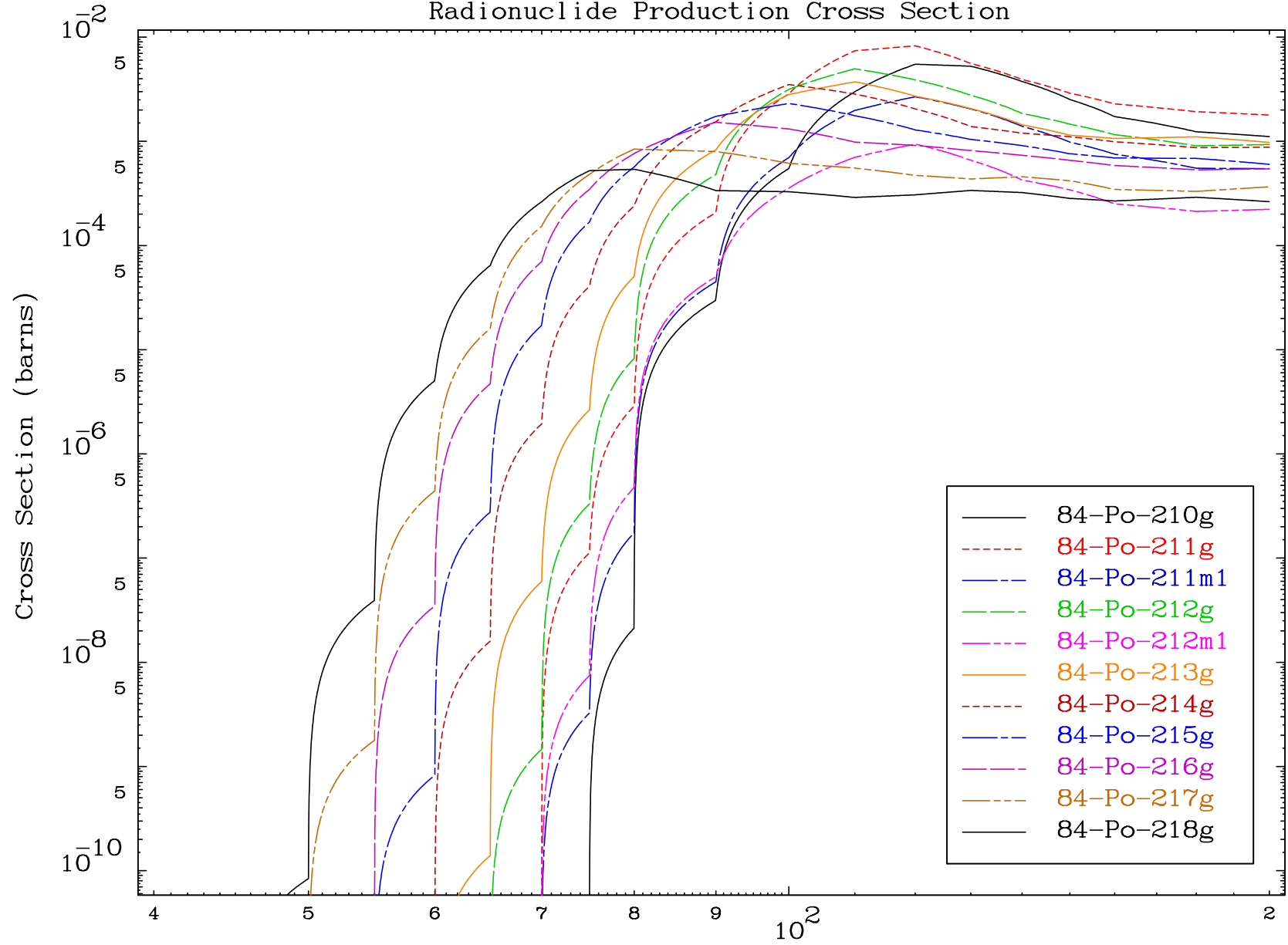
Radionuclide Production Cross Section

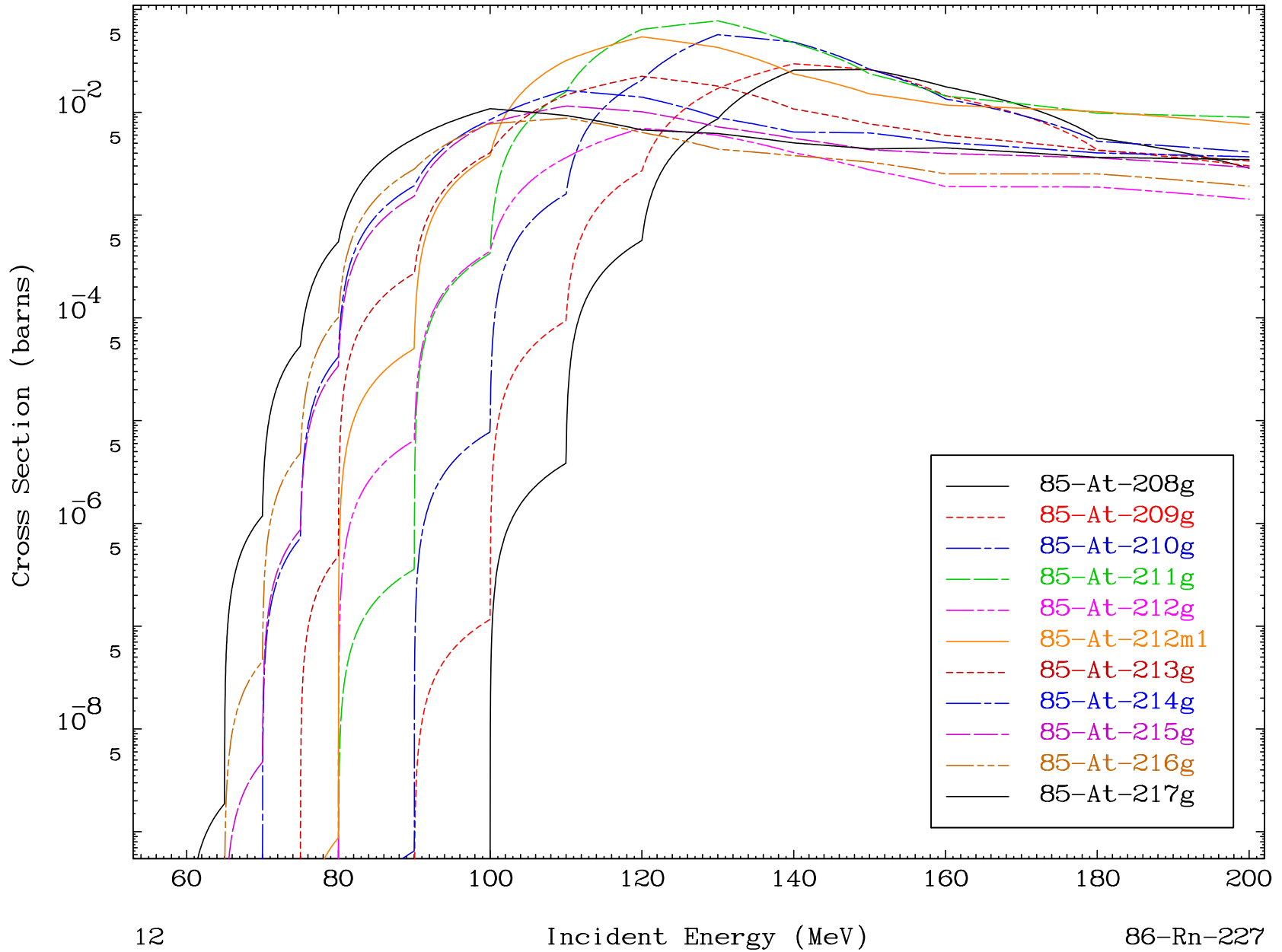


Radionuclide Production Cross Section

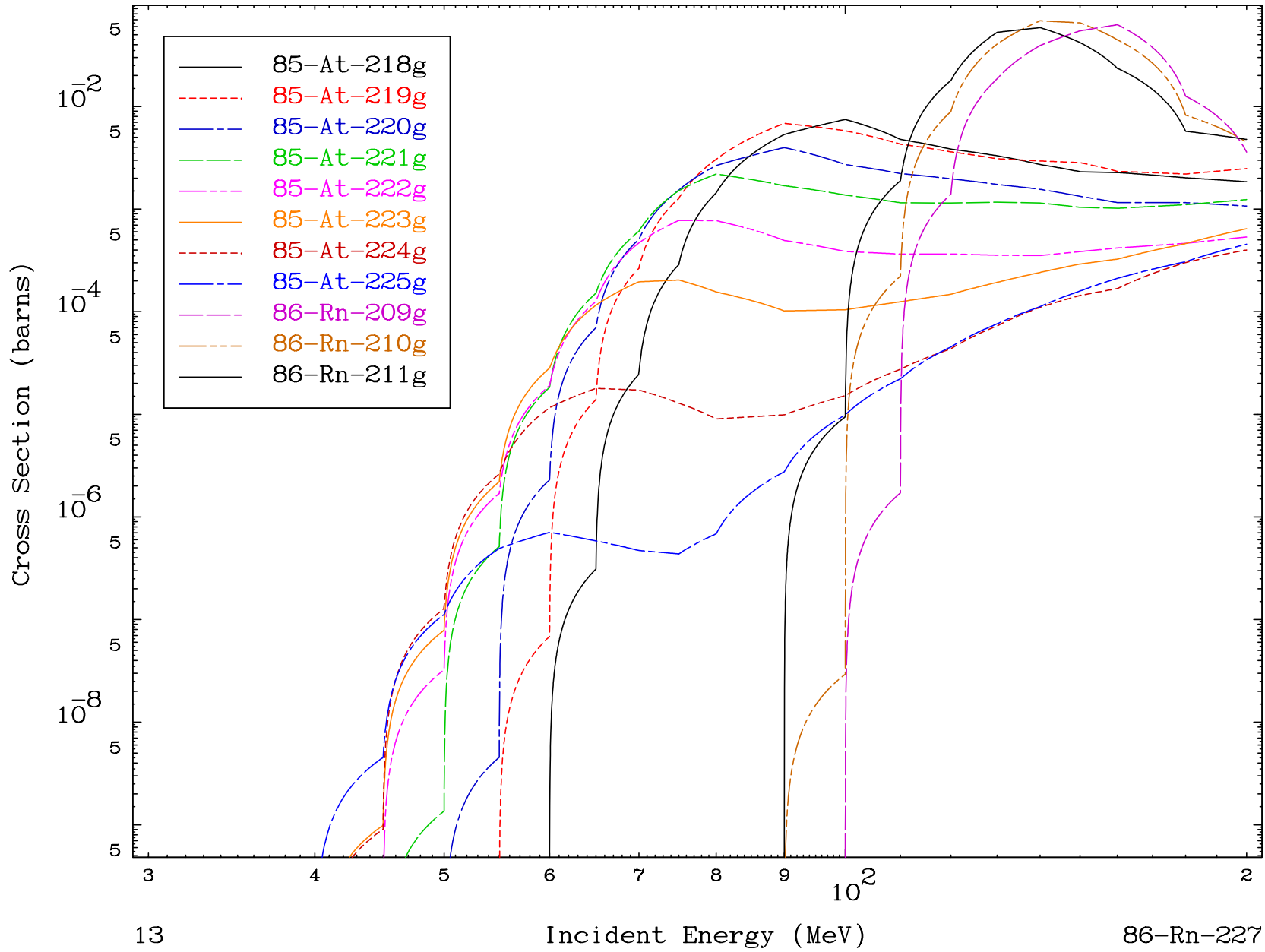


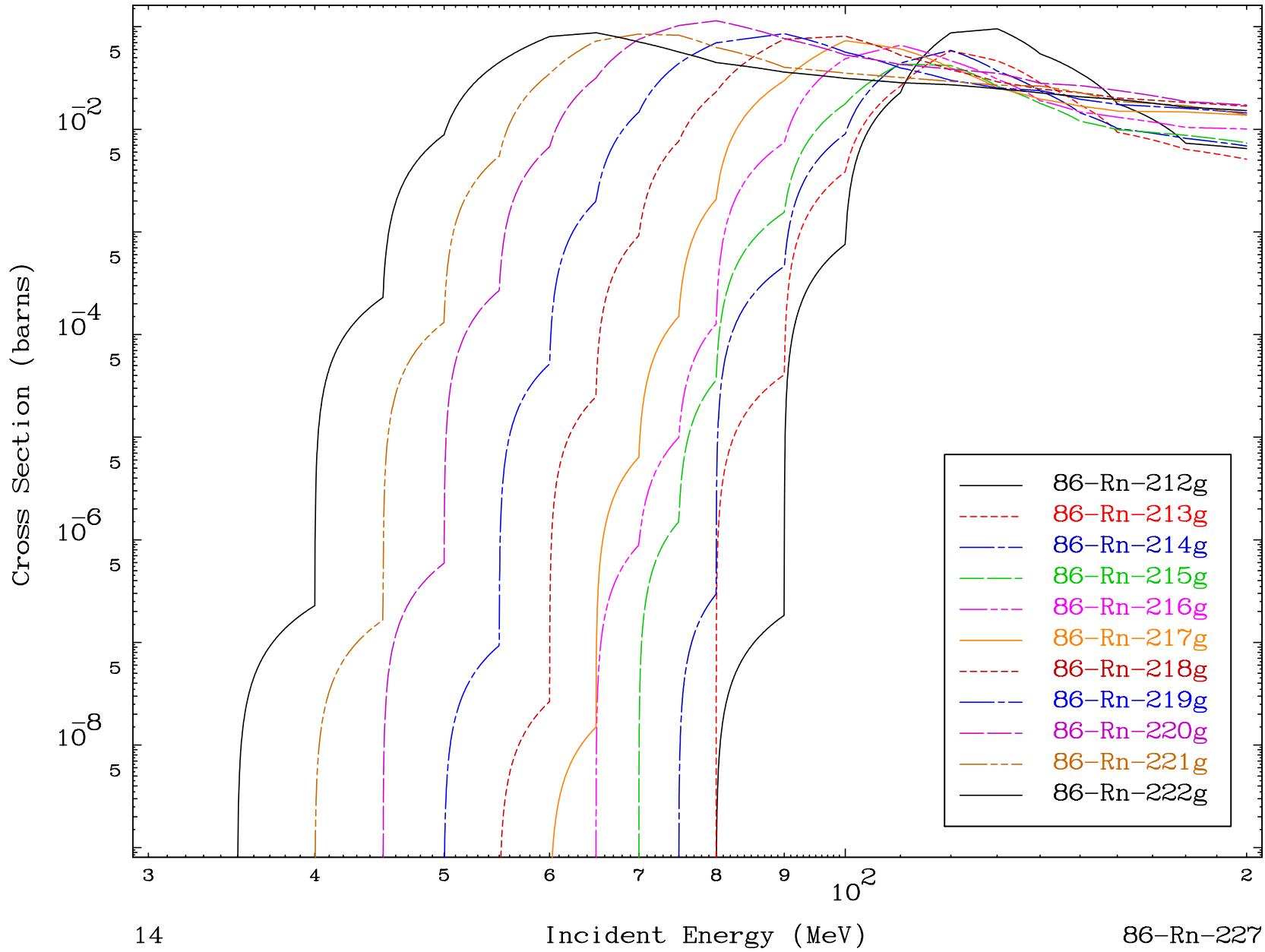
Radionuclide Production Cross Section

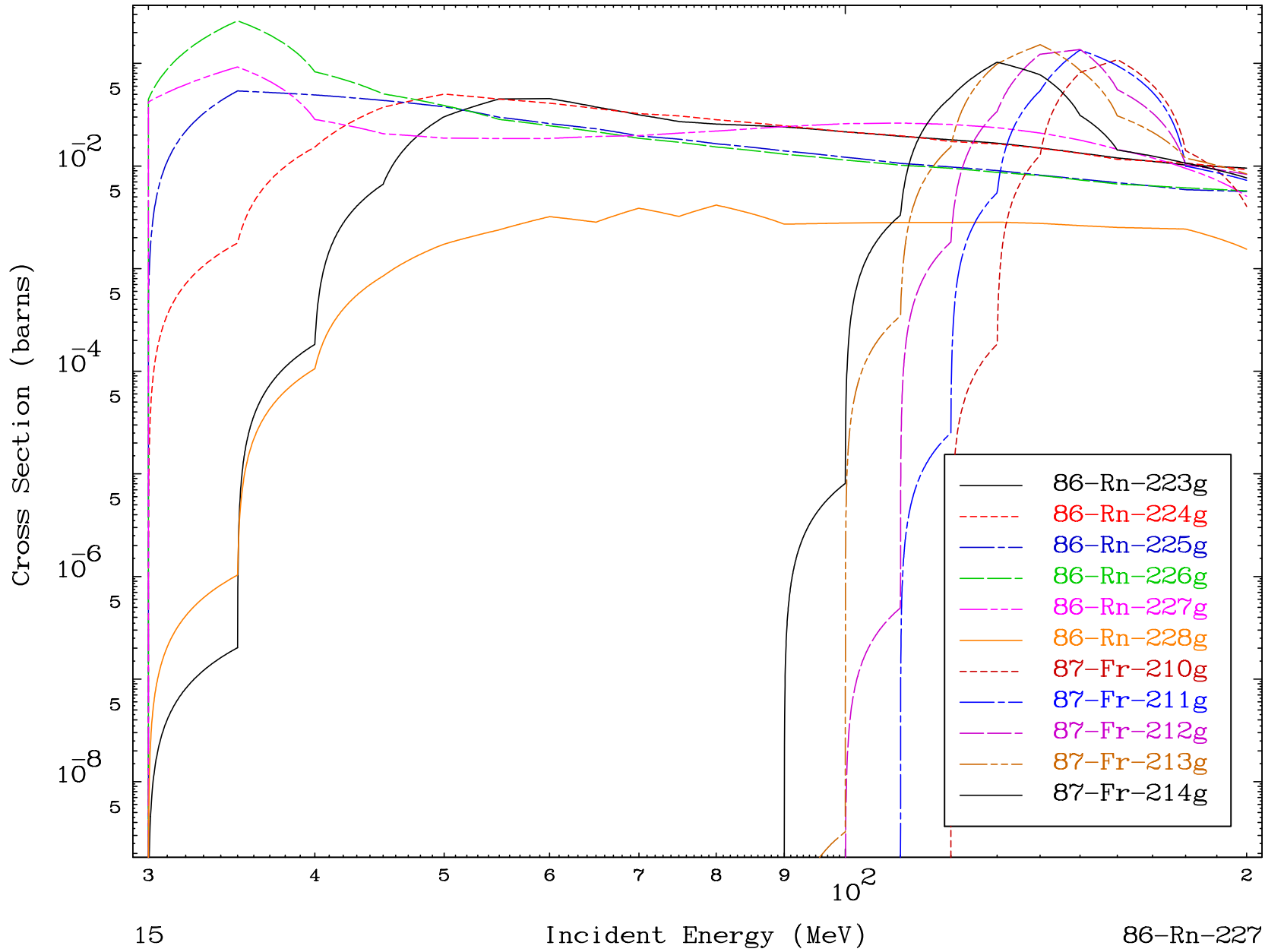


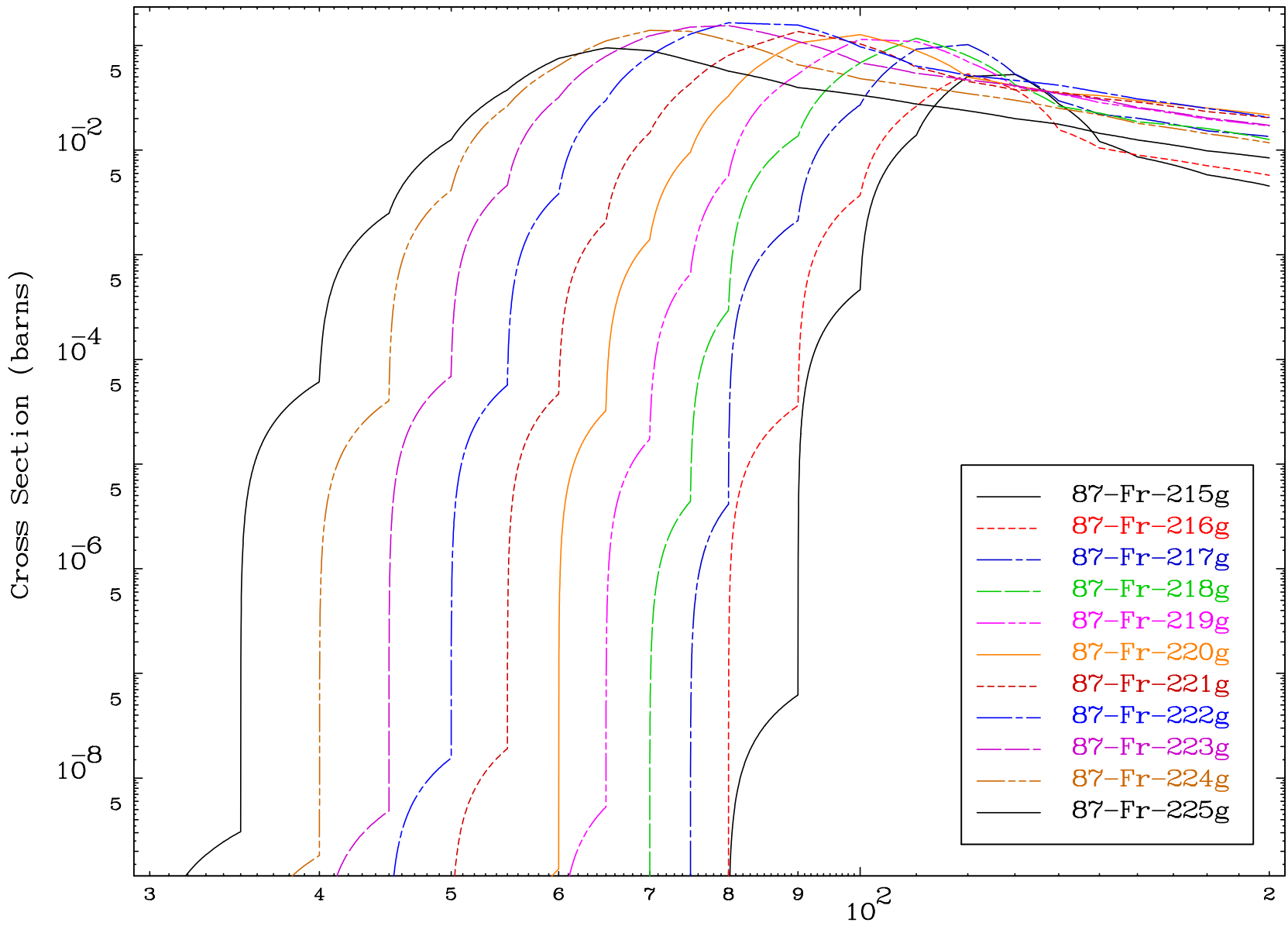


Radionuclide Production Cross Section



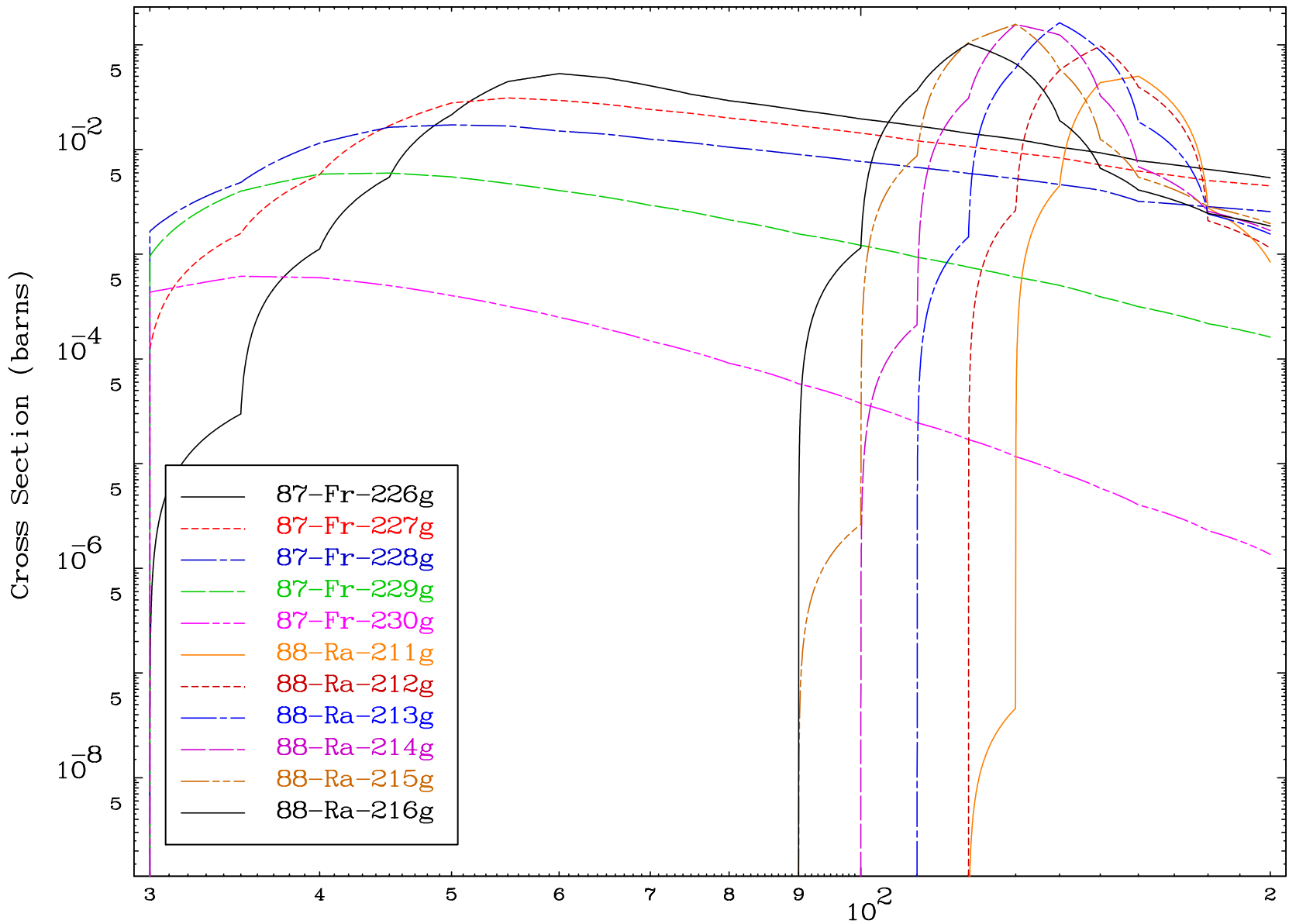








Radionuclide Production Cross Section

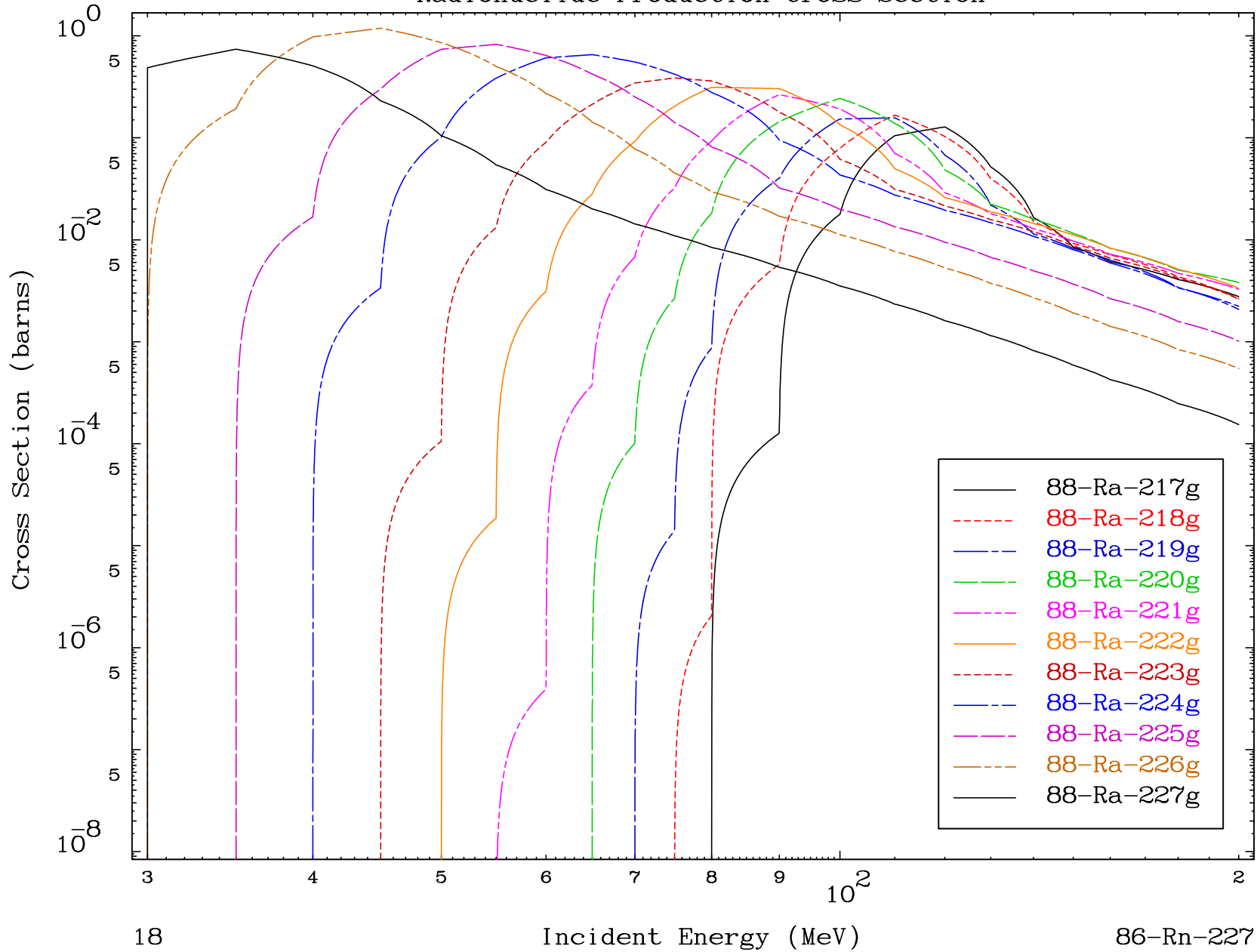


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( $\alpha$ , remainder)

86-Rn-227

### Radionuclide Production Cross Section

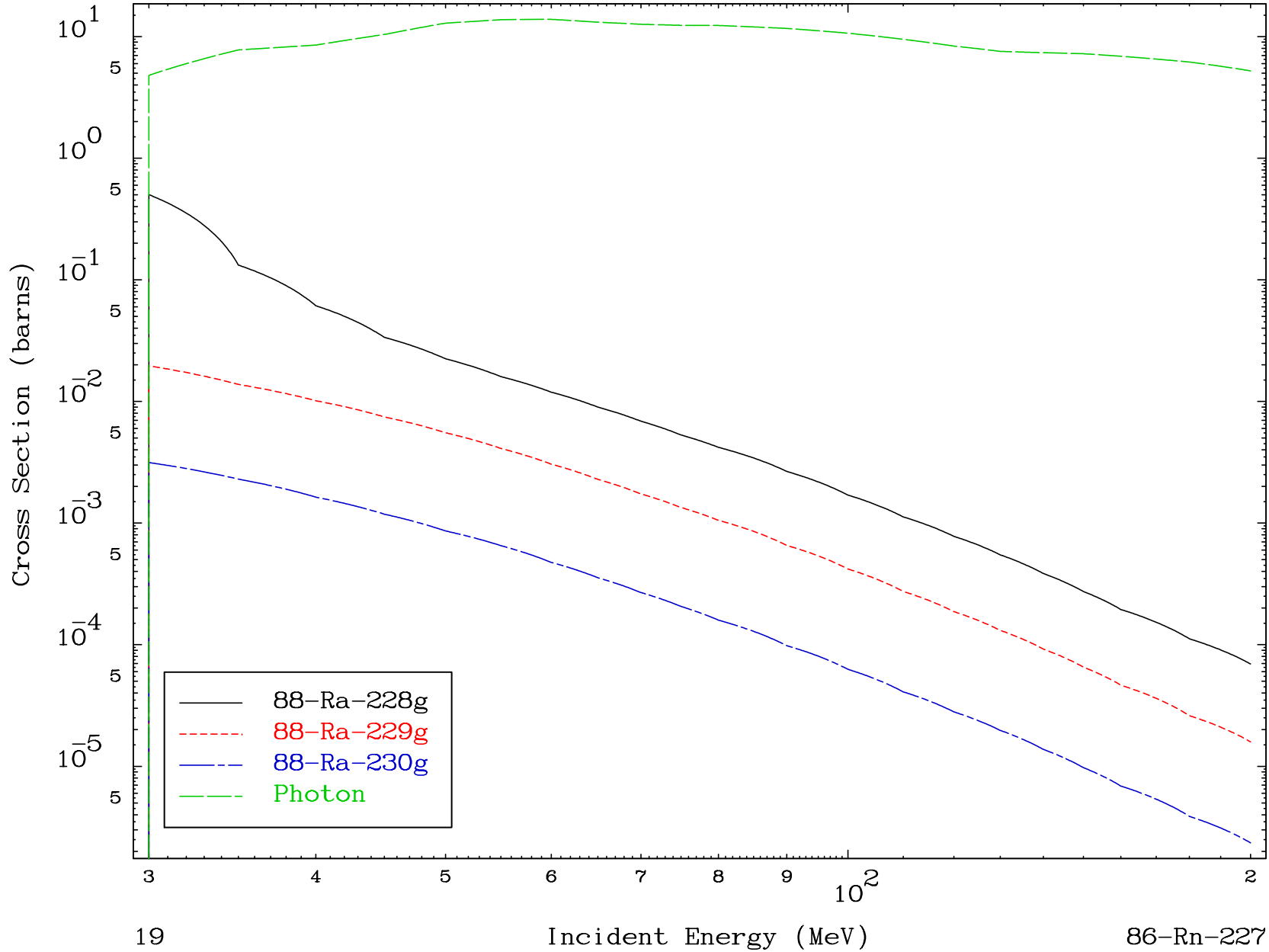


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( $\alpha$ , remainder)

86-Rn-227

### Radionuclide Production Cross Section



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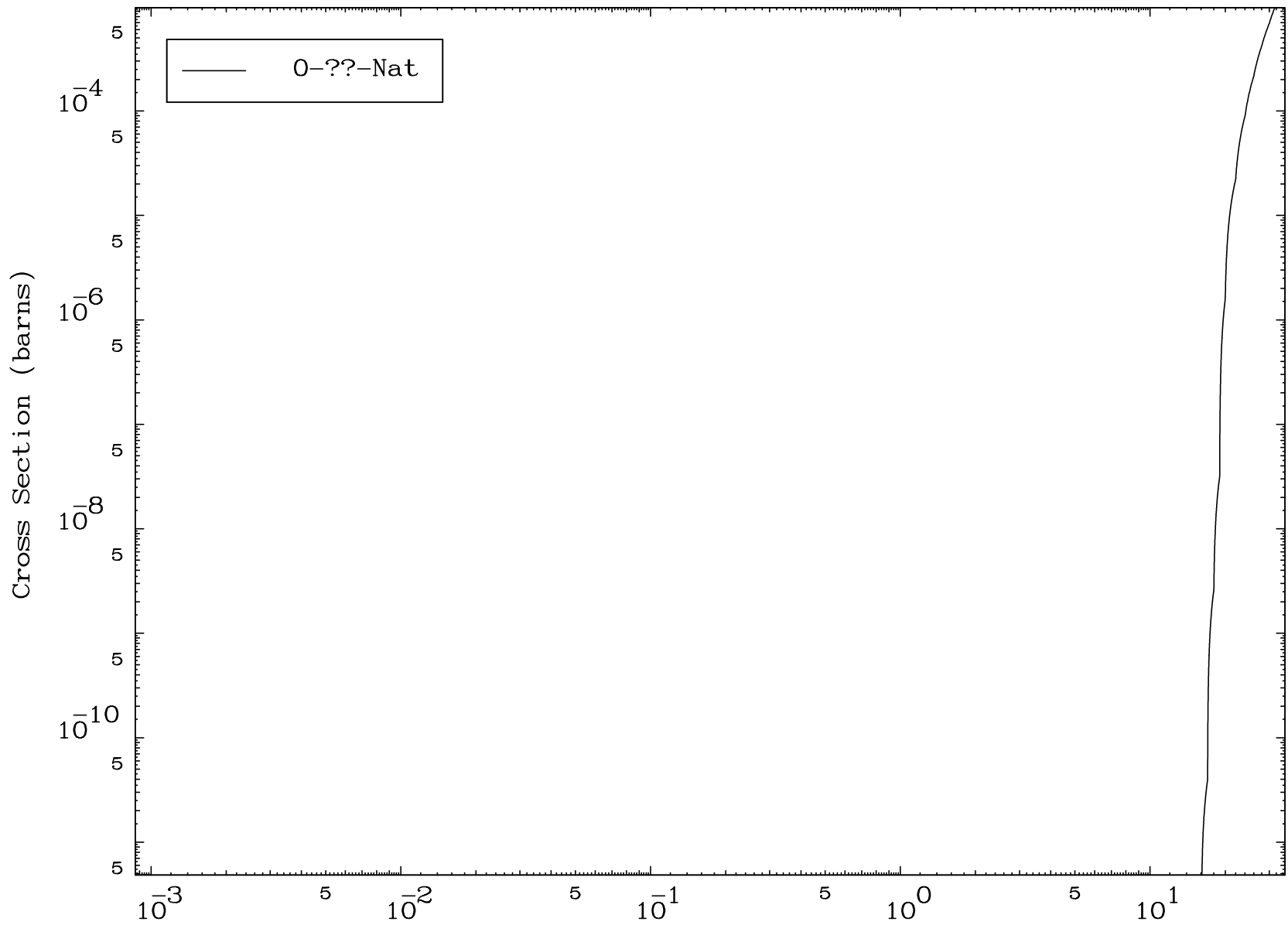
Incident Energy (MeV)

86-Rn-227

MAT 8673

$\alpha$  Fission  
Radionuclide Production Cross Section

86-Rn-227



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Incident Energy (MeV)

86-Rn-227