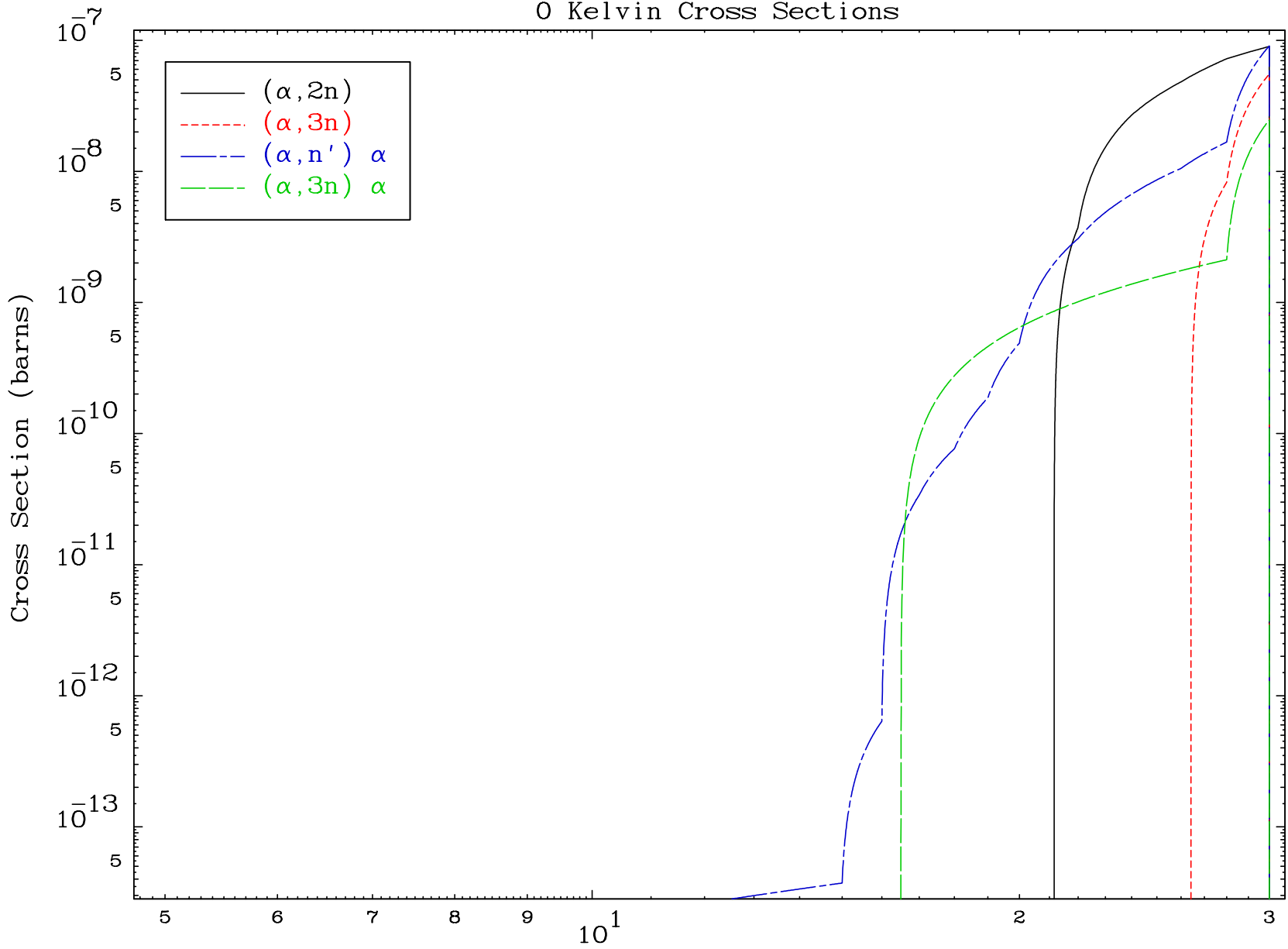


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α Neutron Production
0 Kelvin Cross Sections

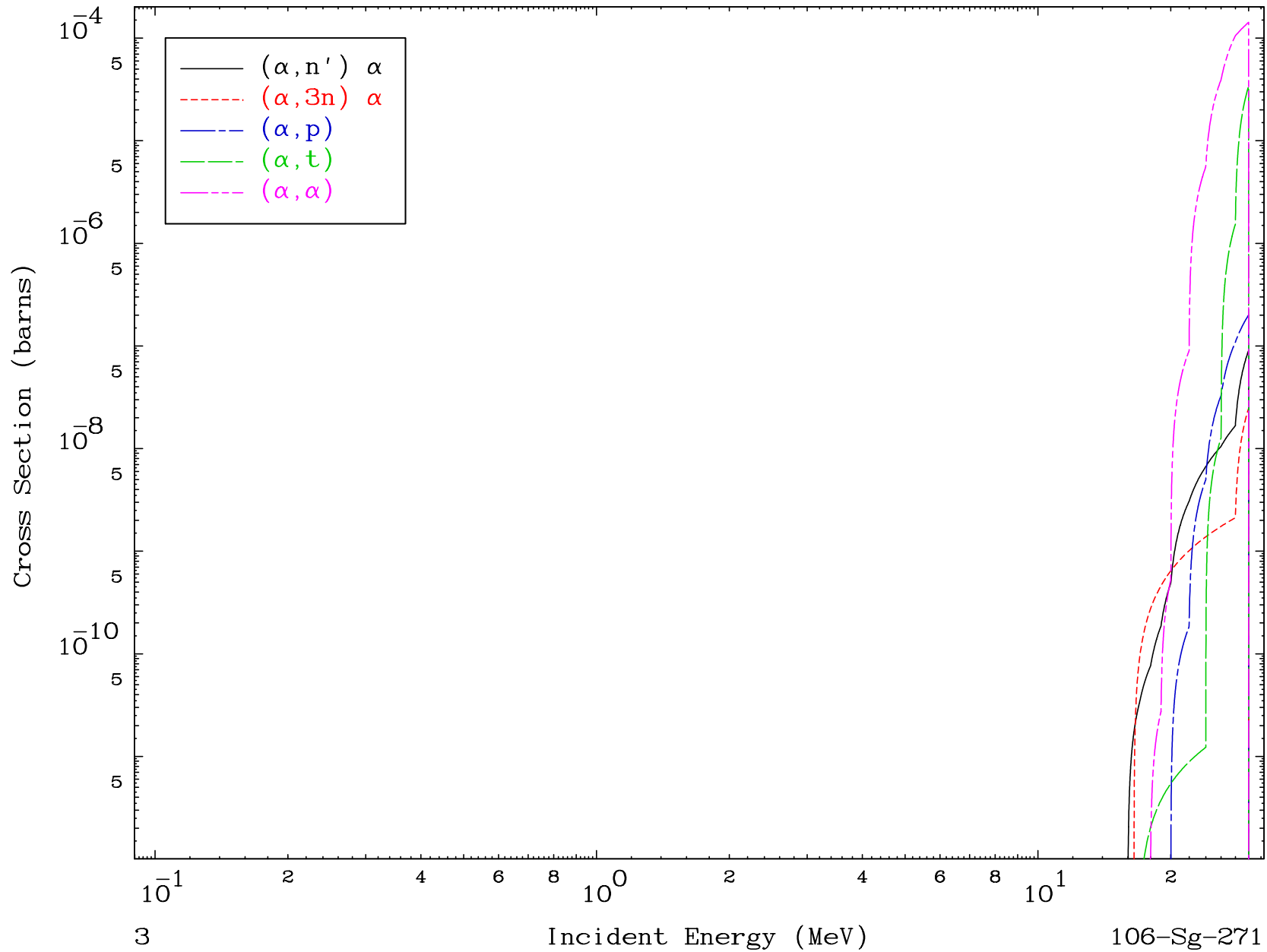
106-Sg-271

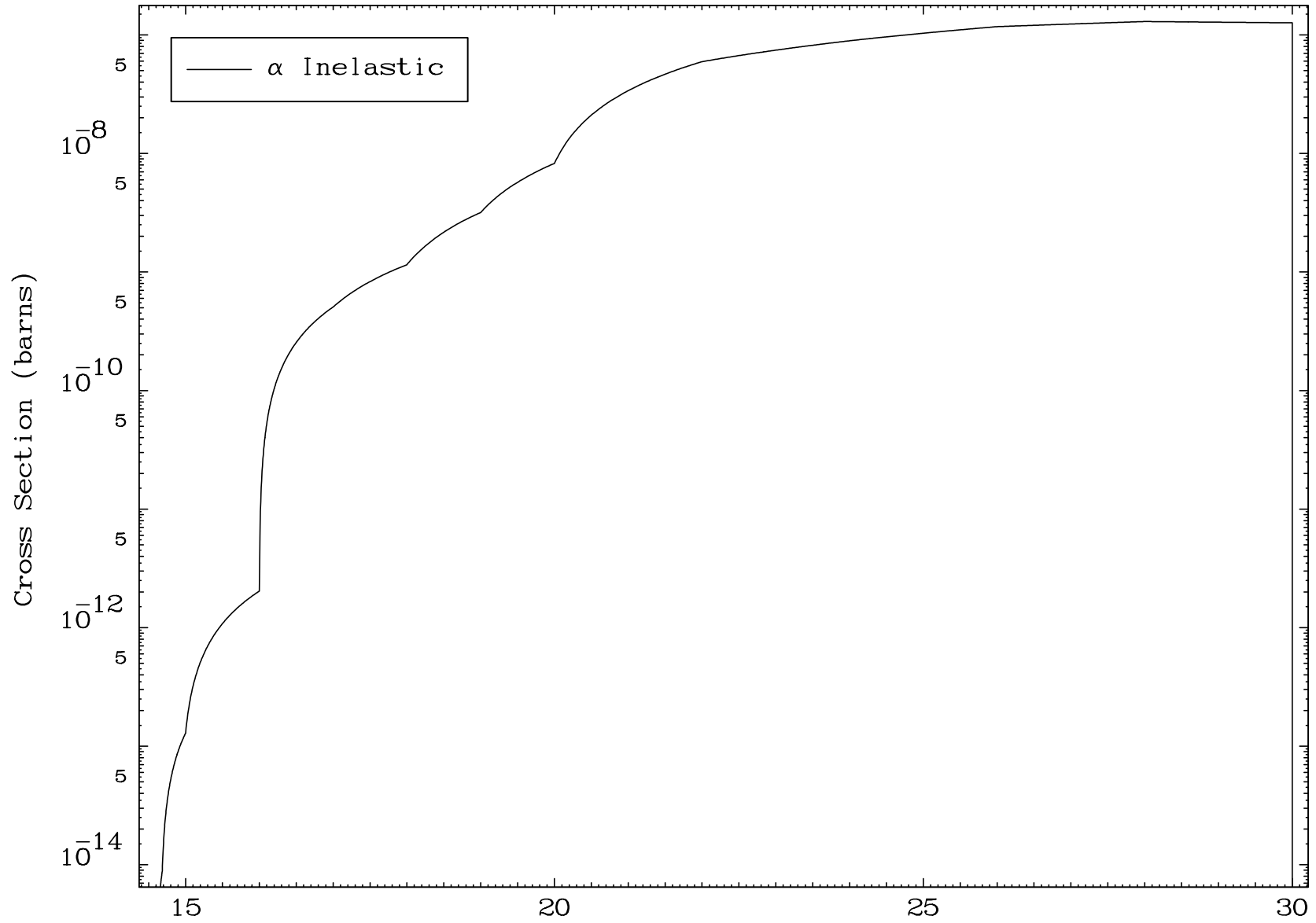


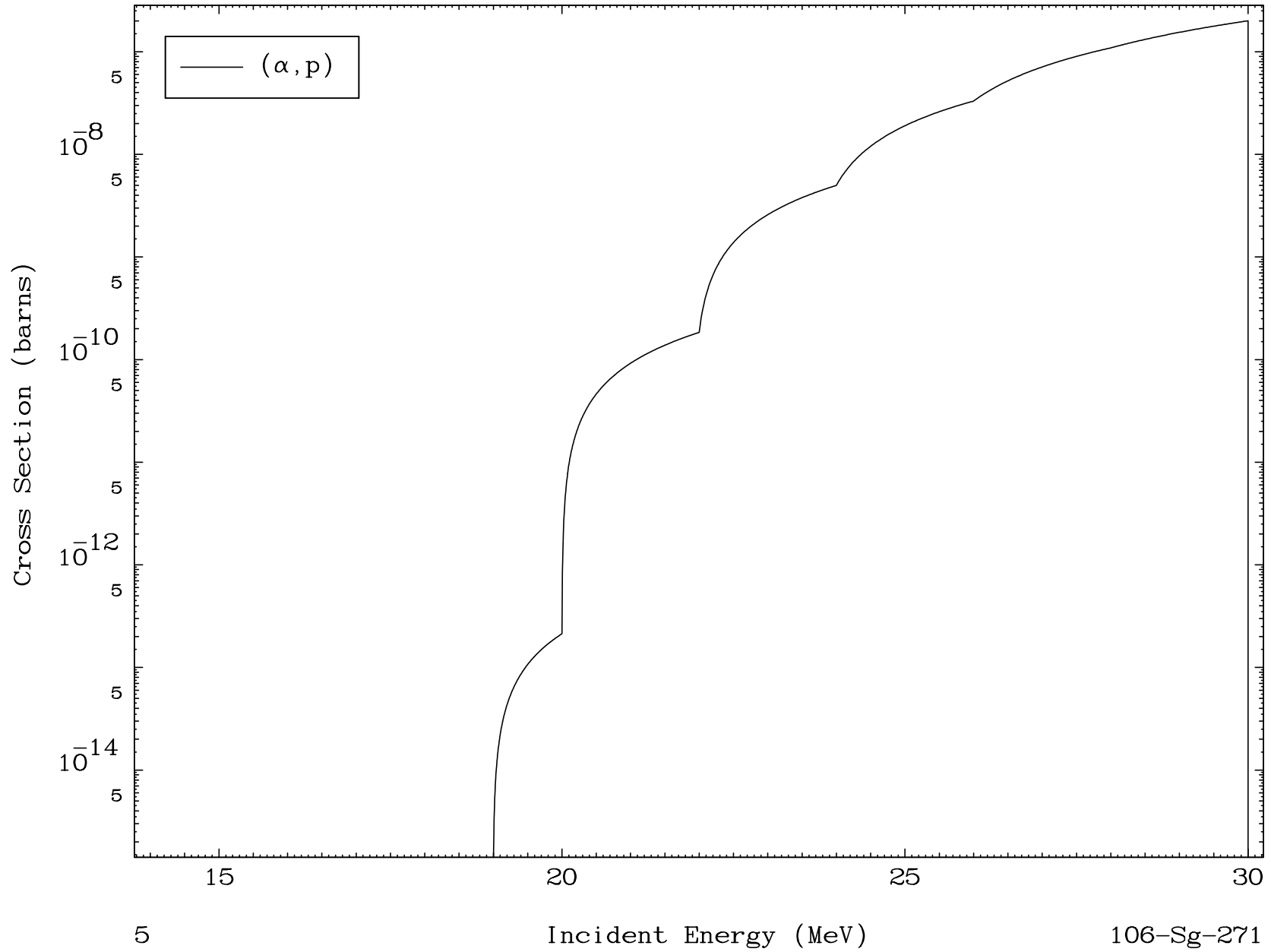
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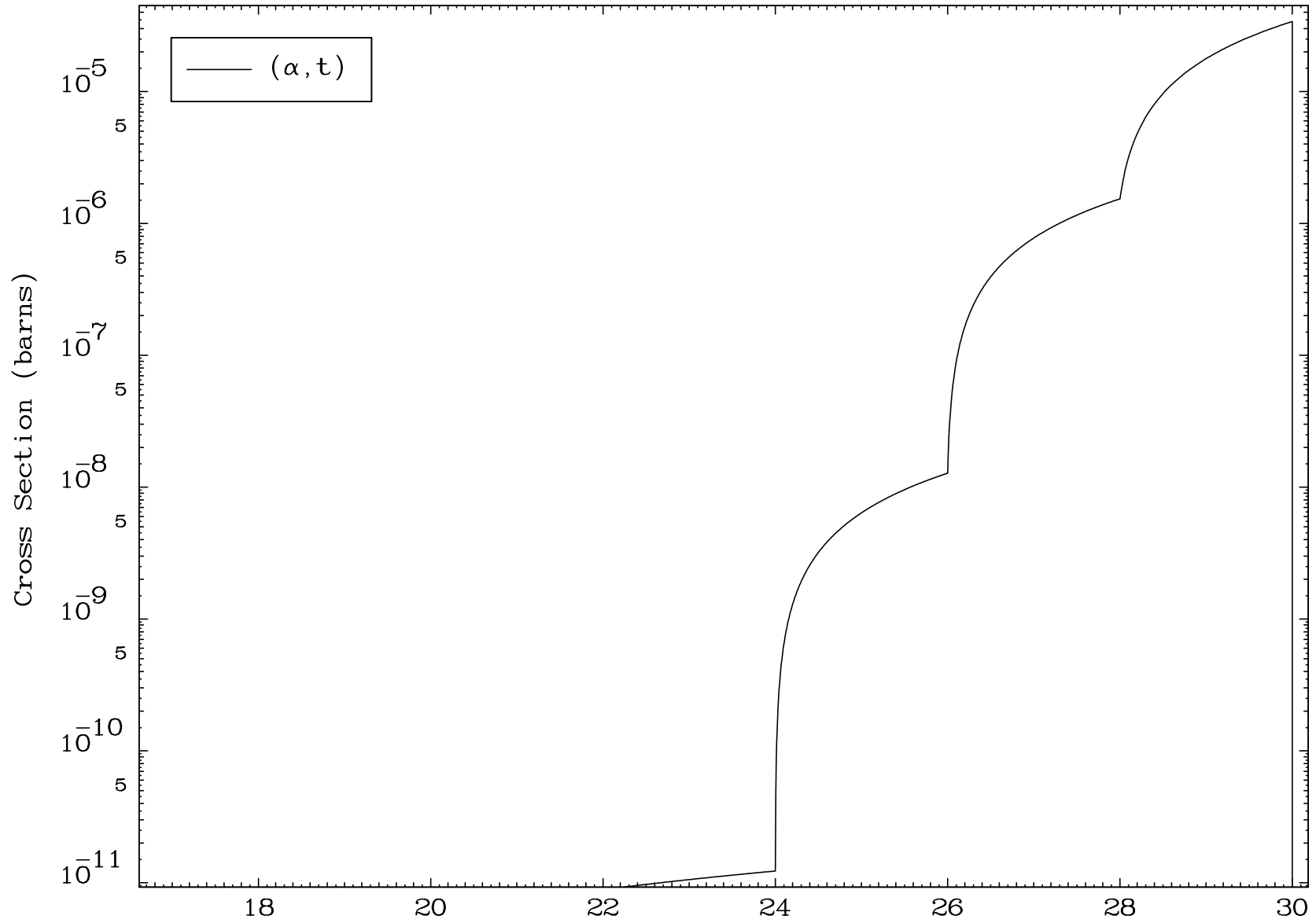
Incident Energy (MeV)

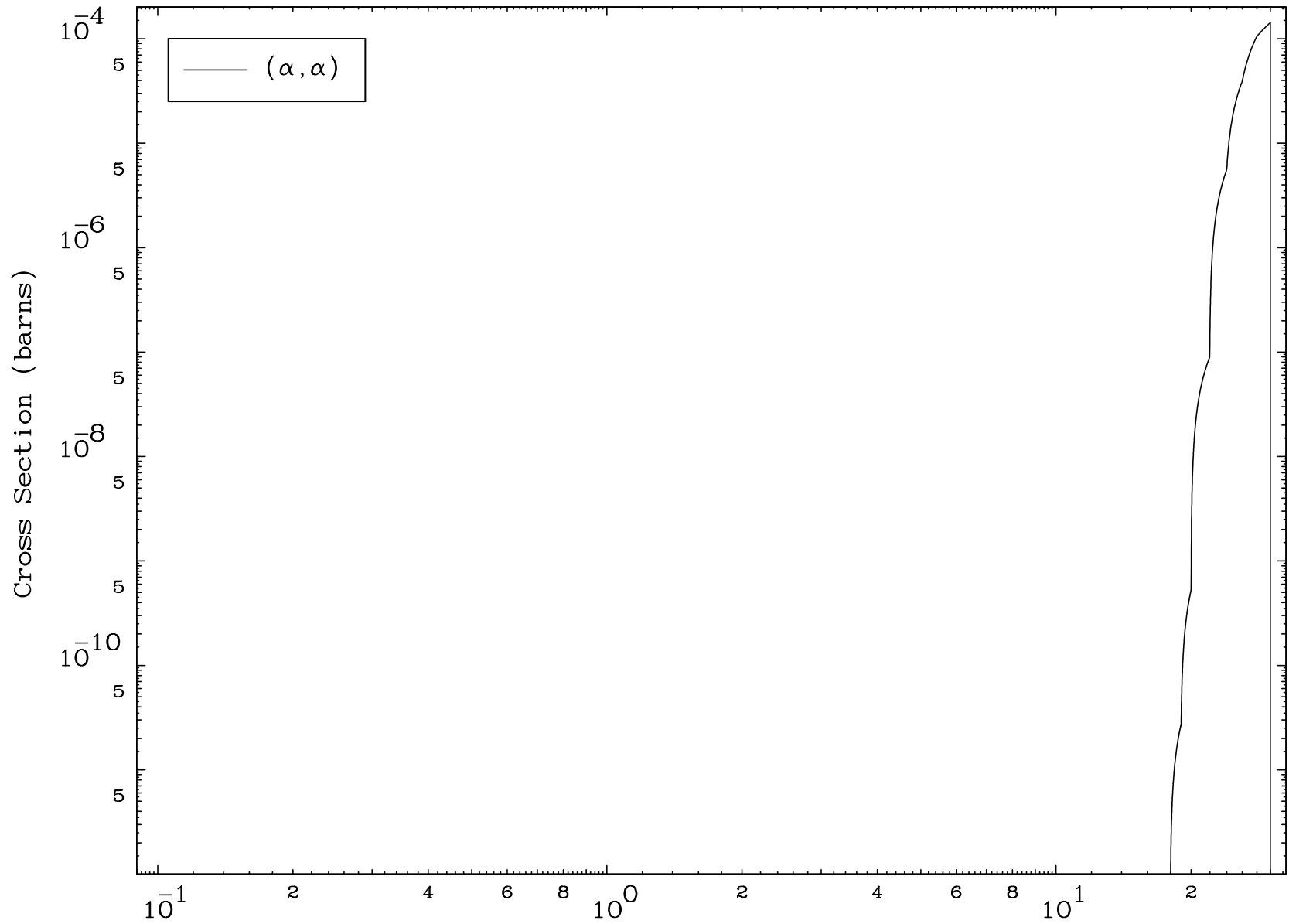
106-Sg-271



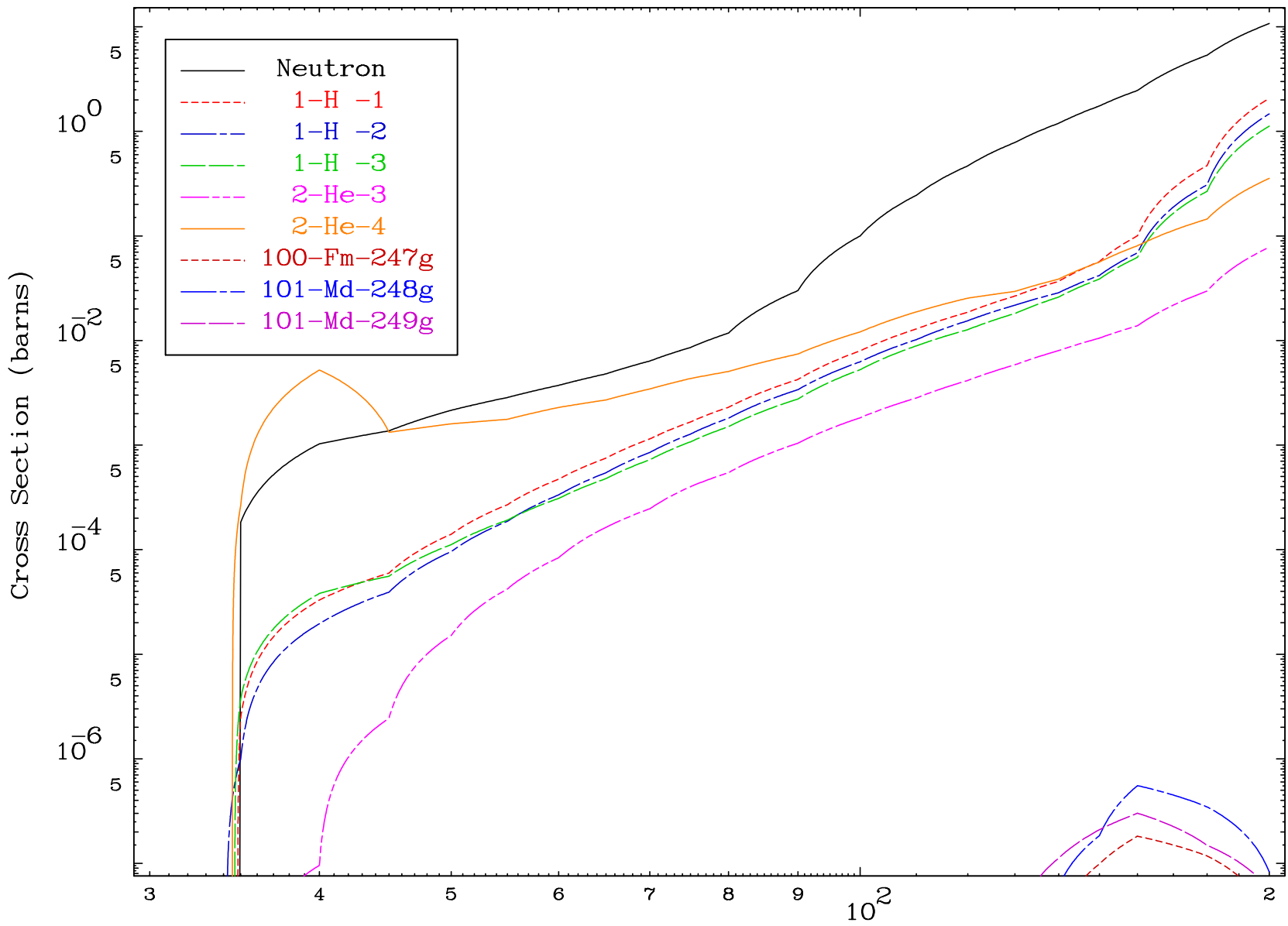




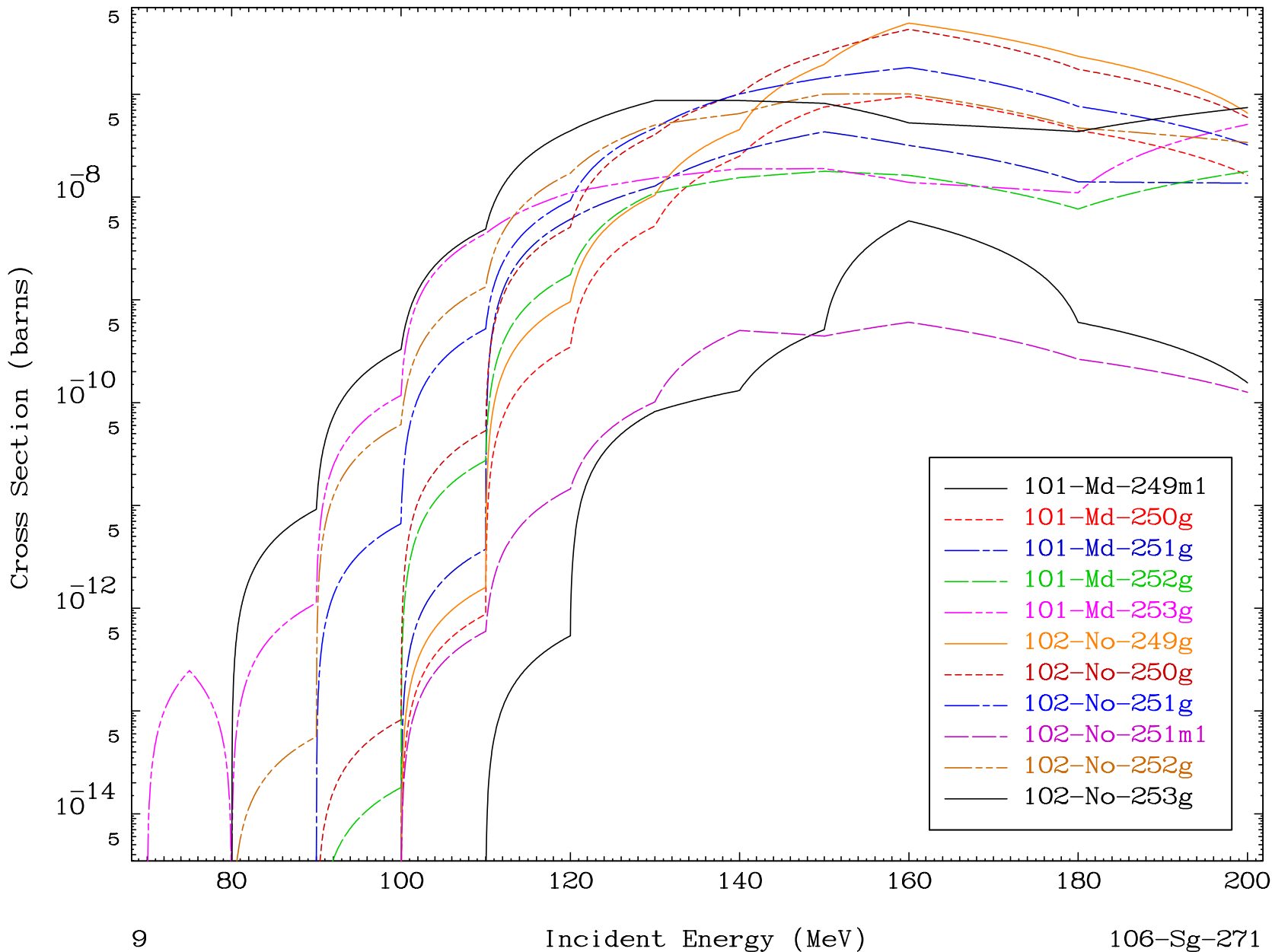




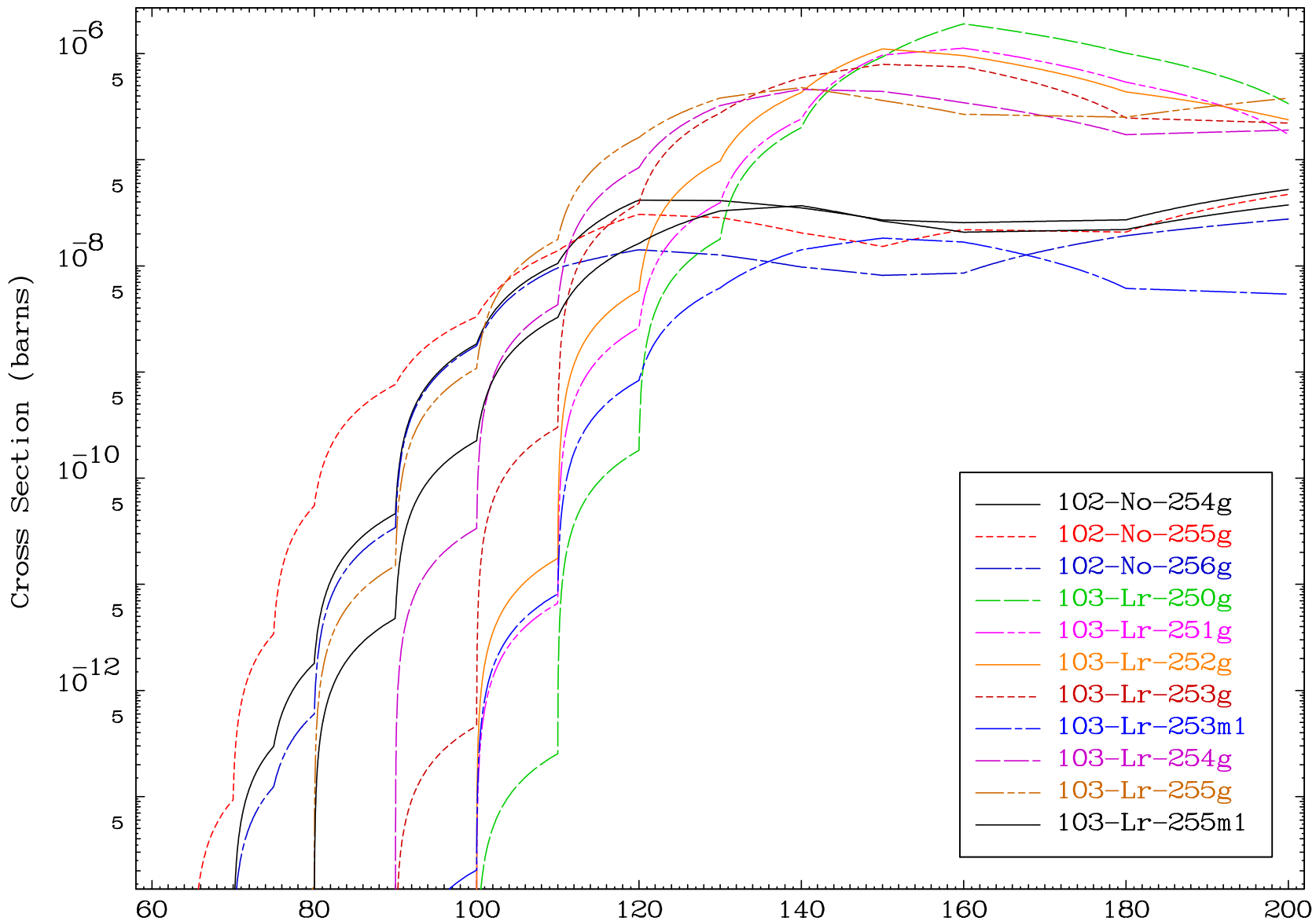
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

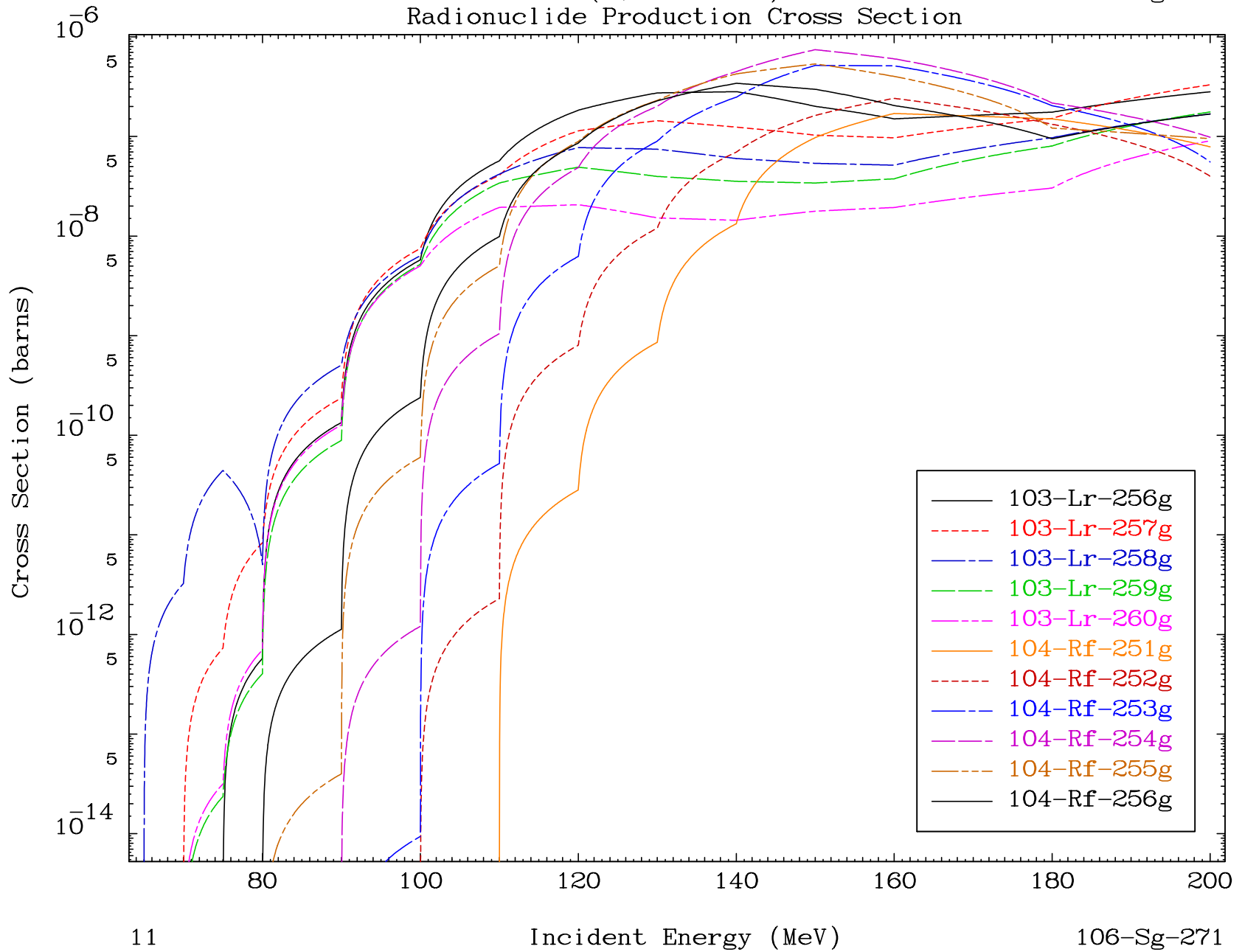


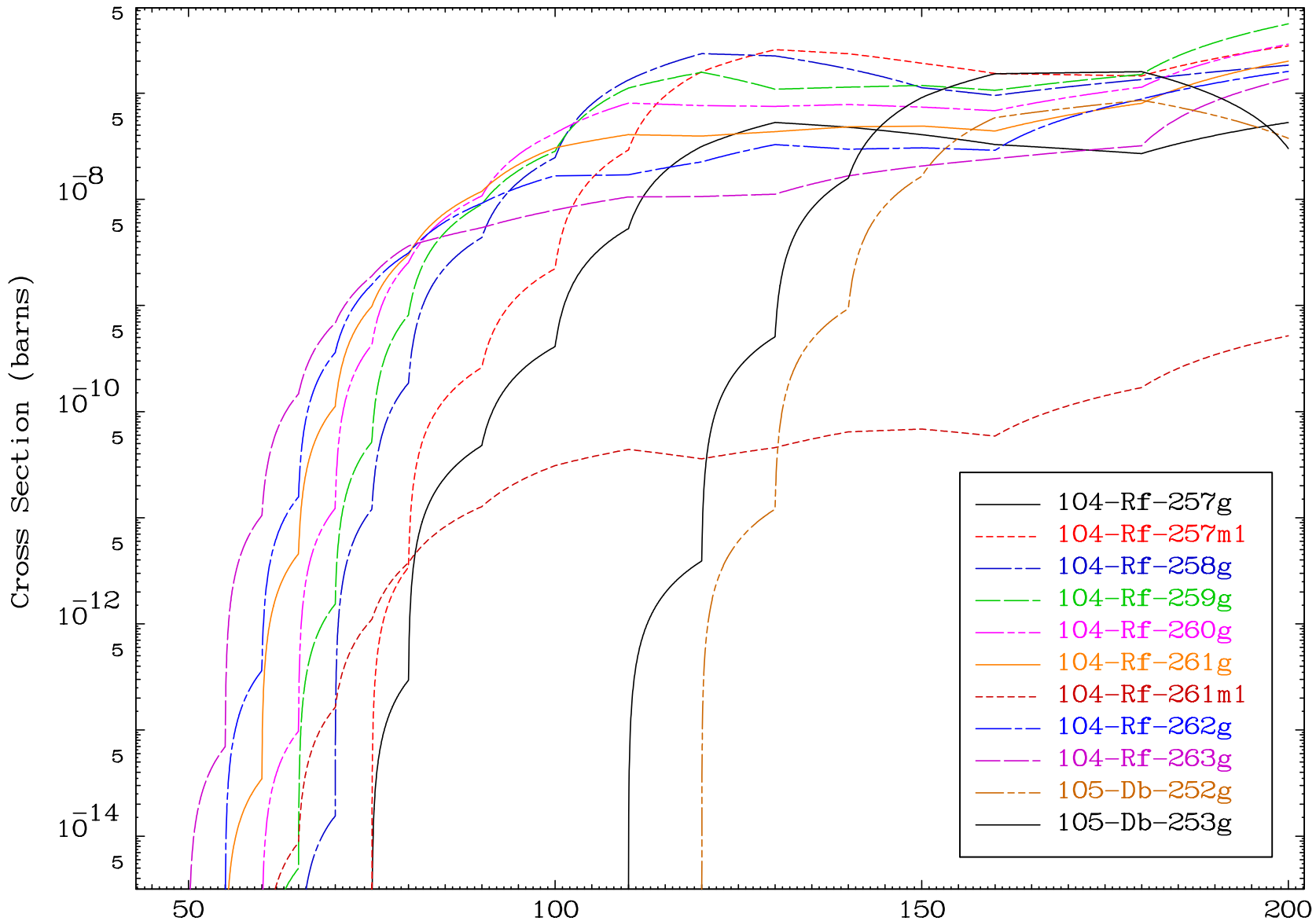
MAT 671

(α , remainder)

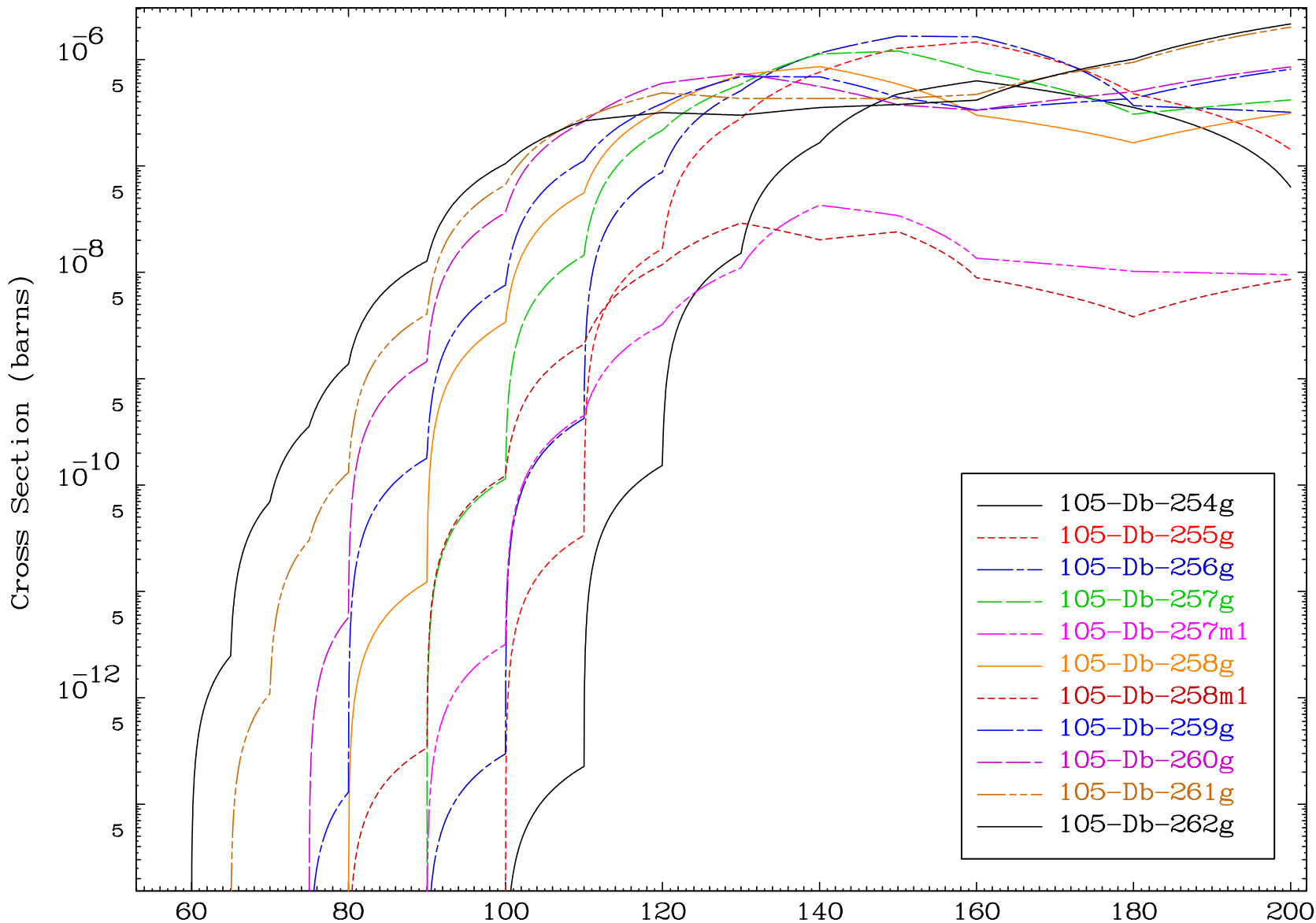
106-Sg-271

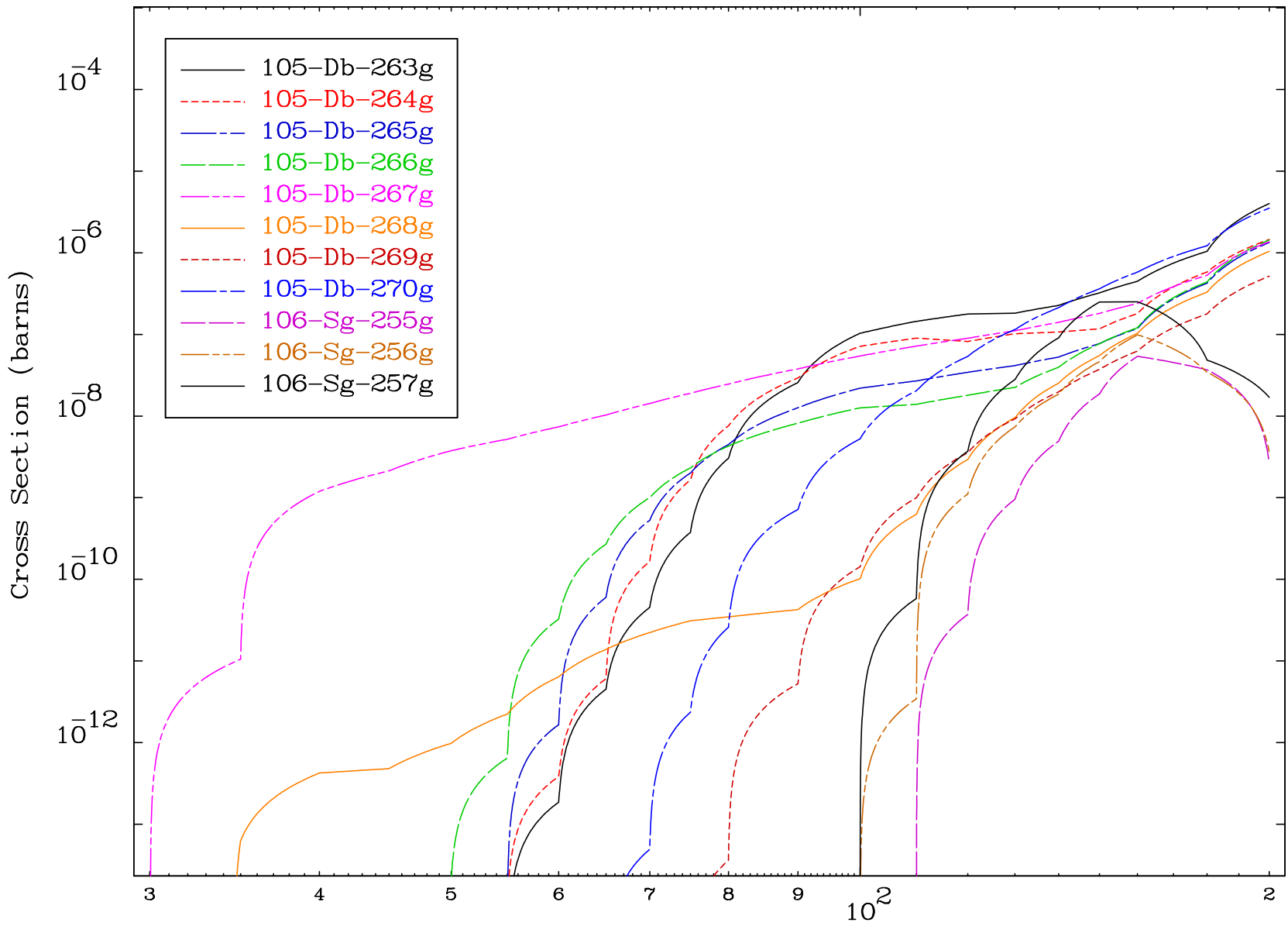
Radionuclide Production Cross Section

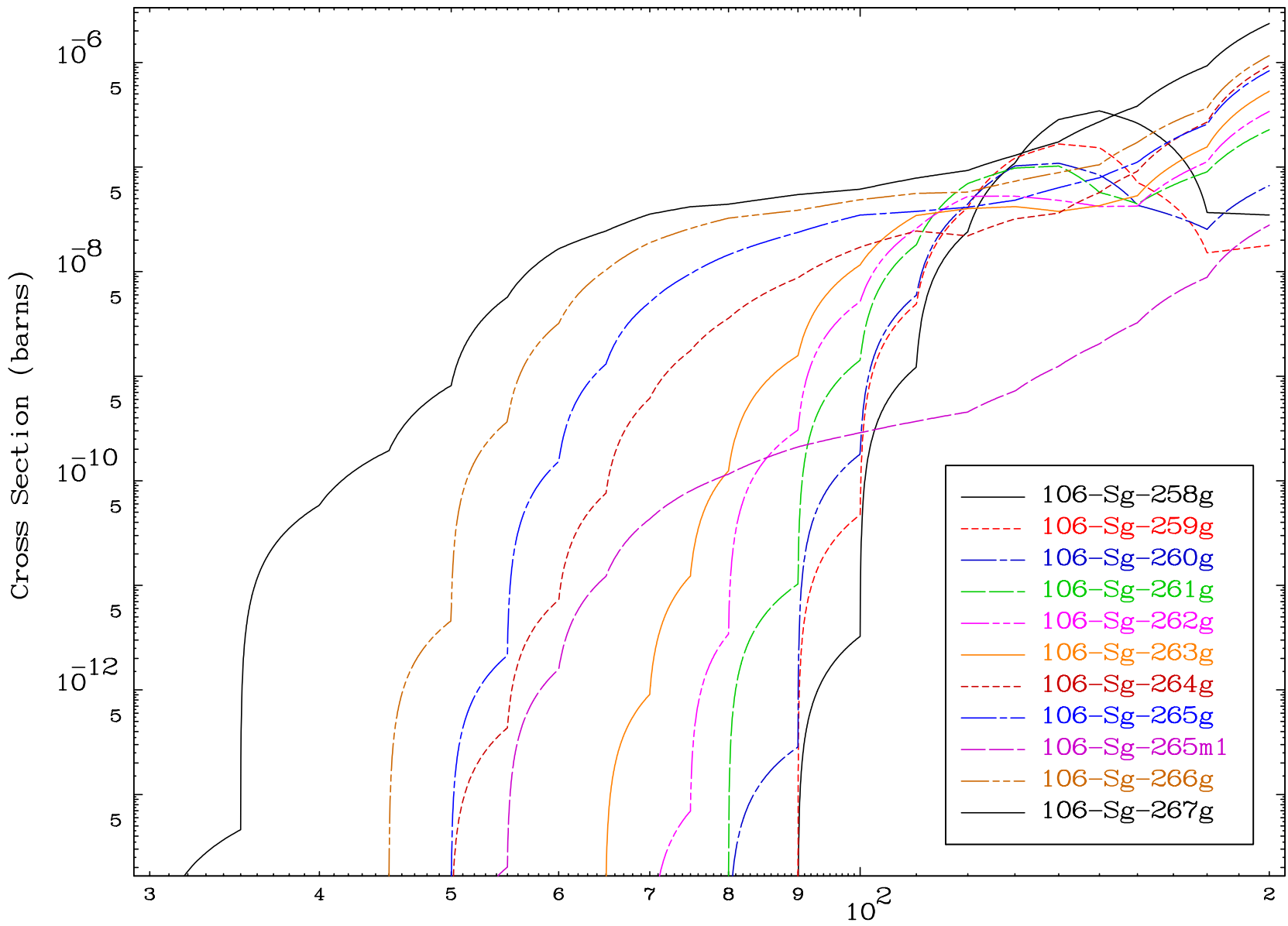




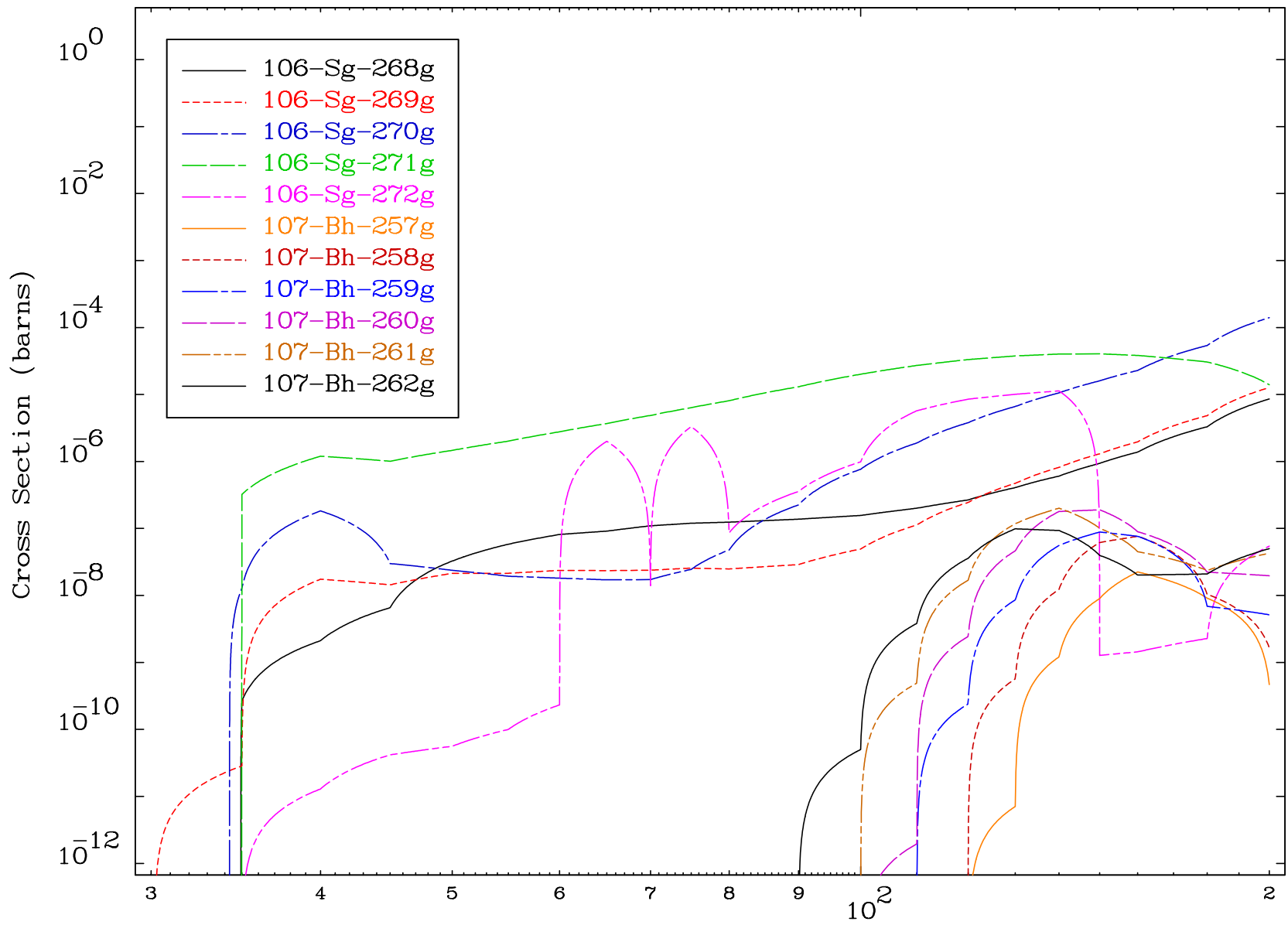
Radionuclide Production Cross Section



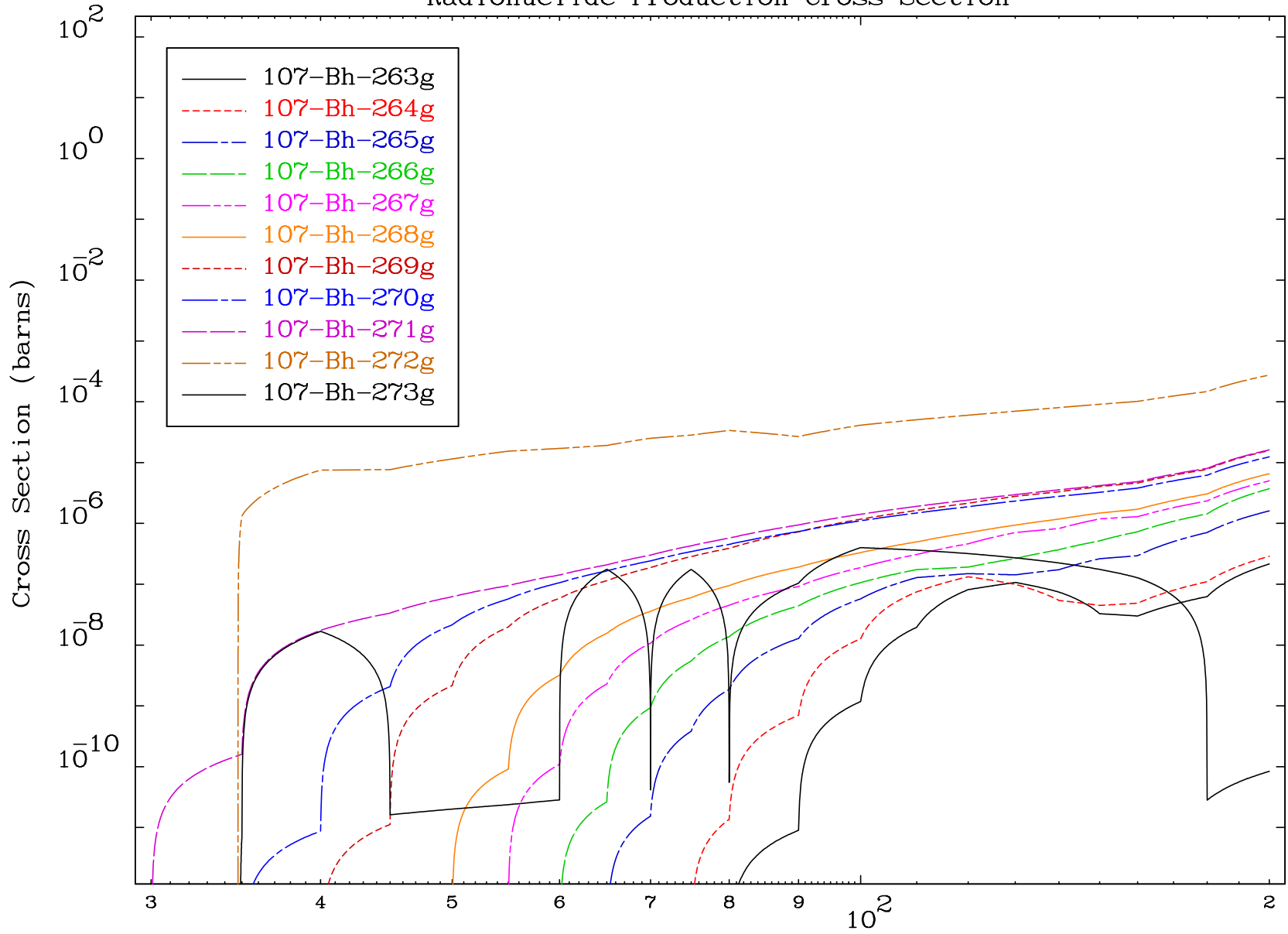




Radionuclide Production Cross Section



Radionuclide Production Cross Section



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(α , remainder)
Radionuclide Production Cross Section

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