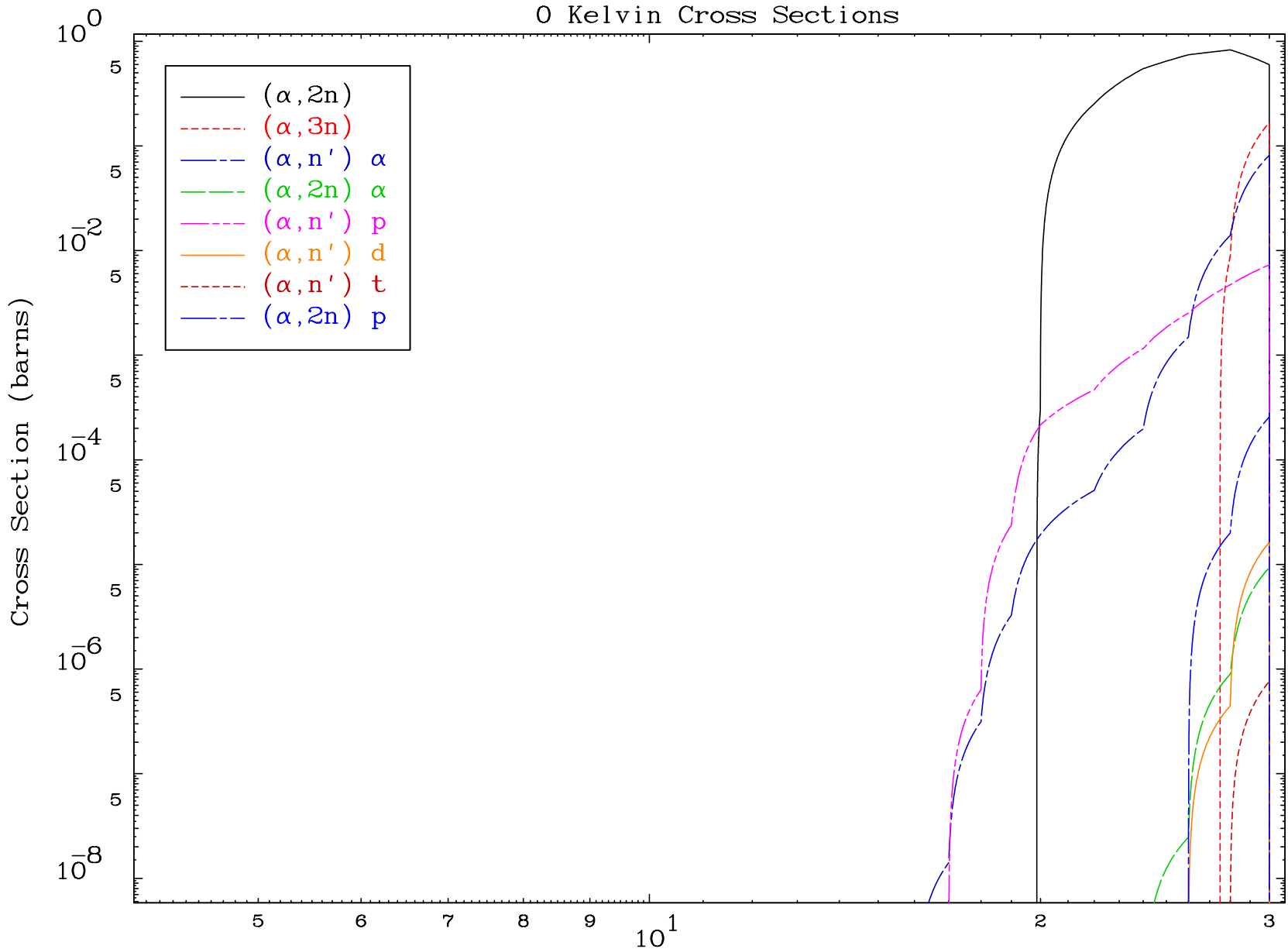


MAT 7313

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

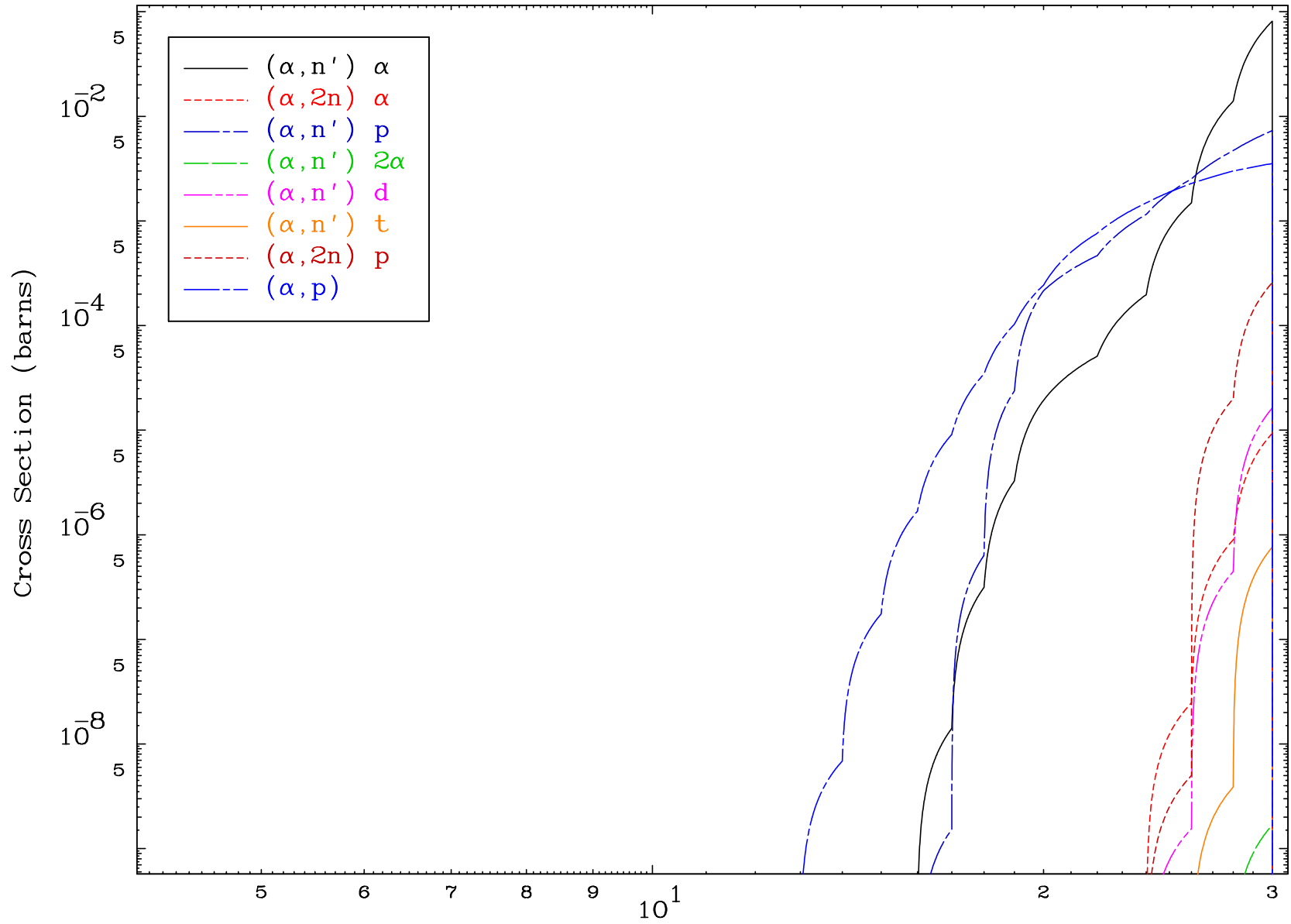
73-Ta-176

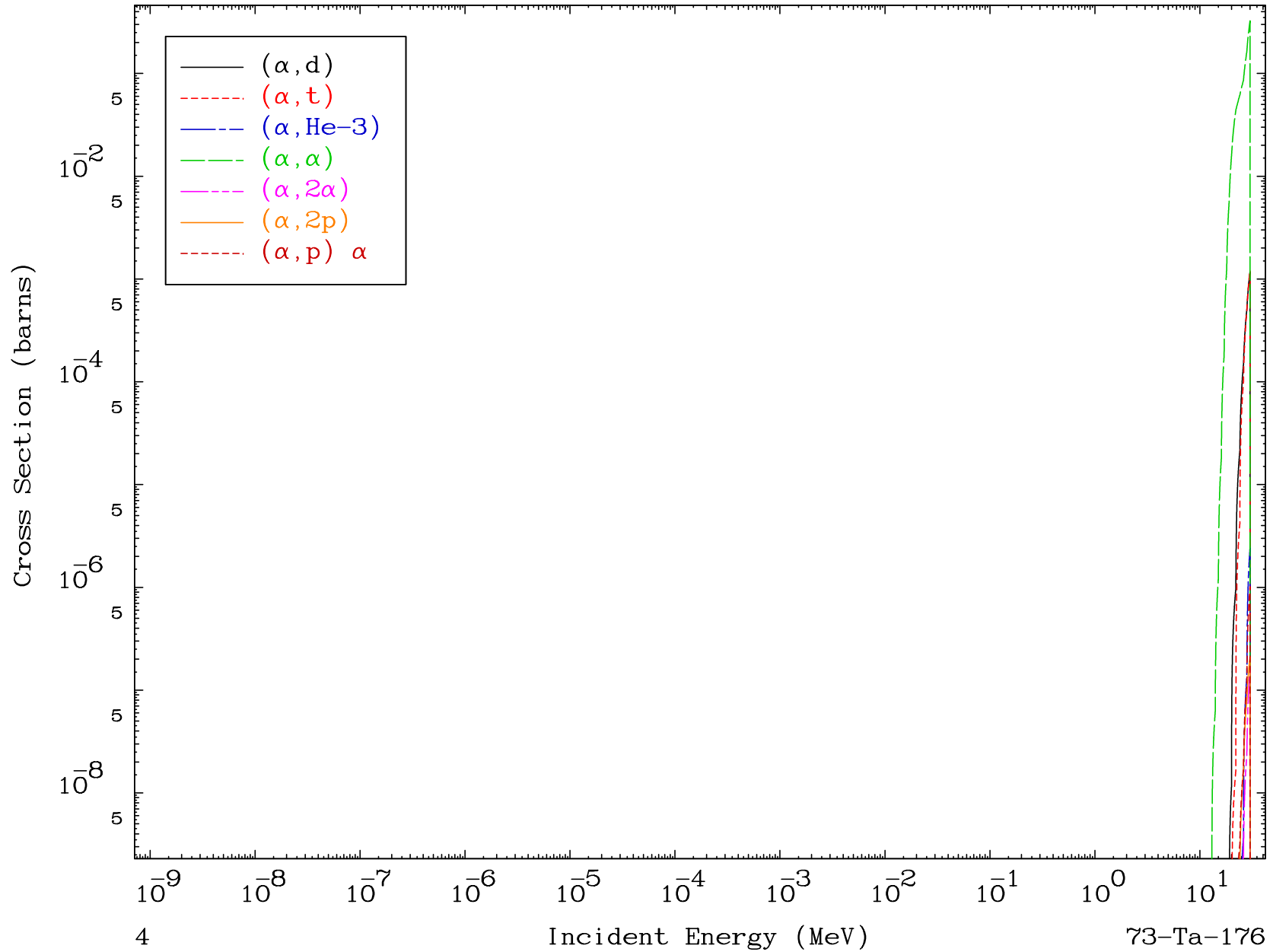


2

Incident Energy (MeV)

73-Ta-176

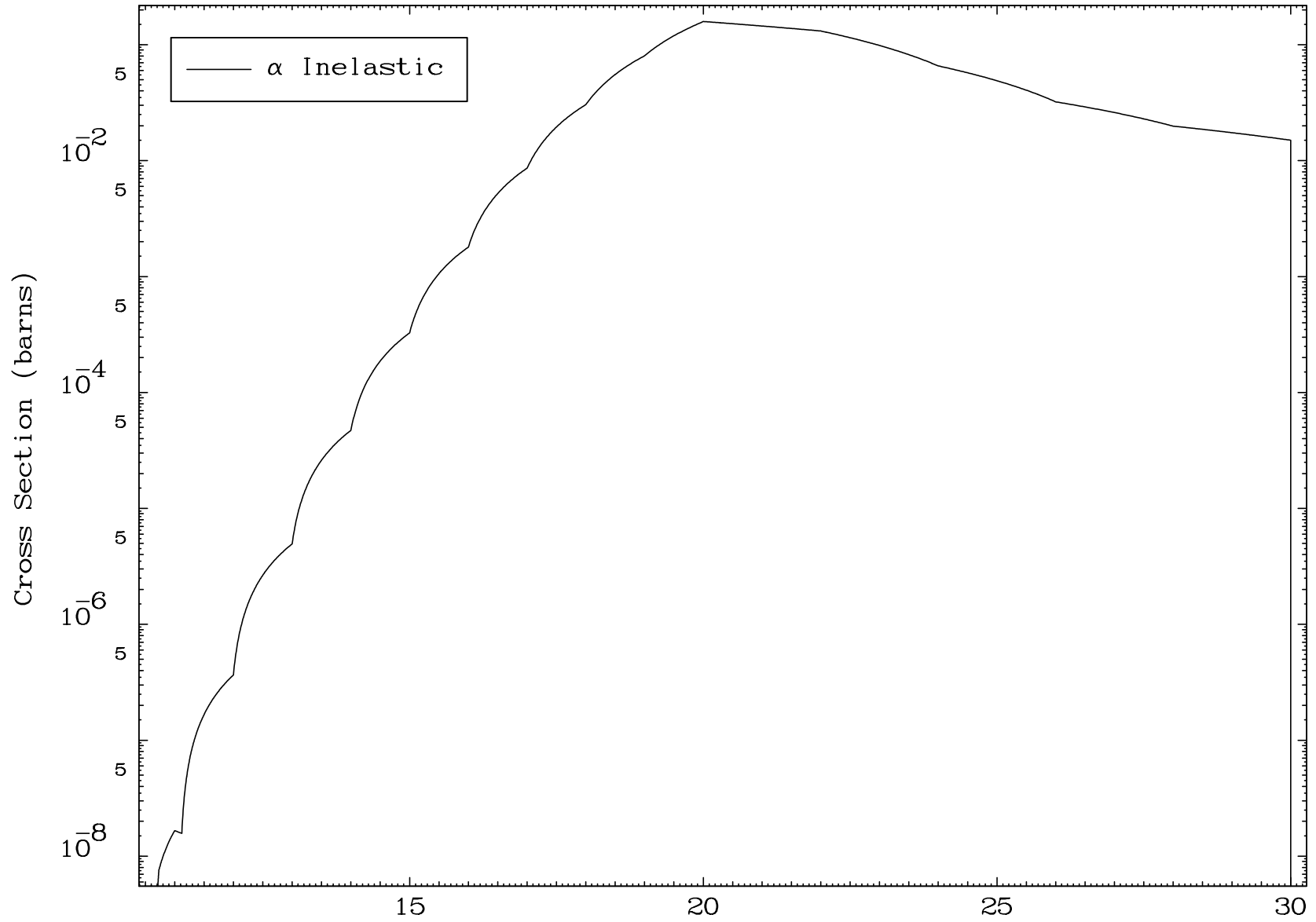




MAT 7313

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

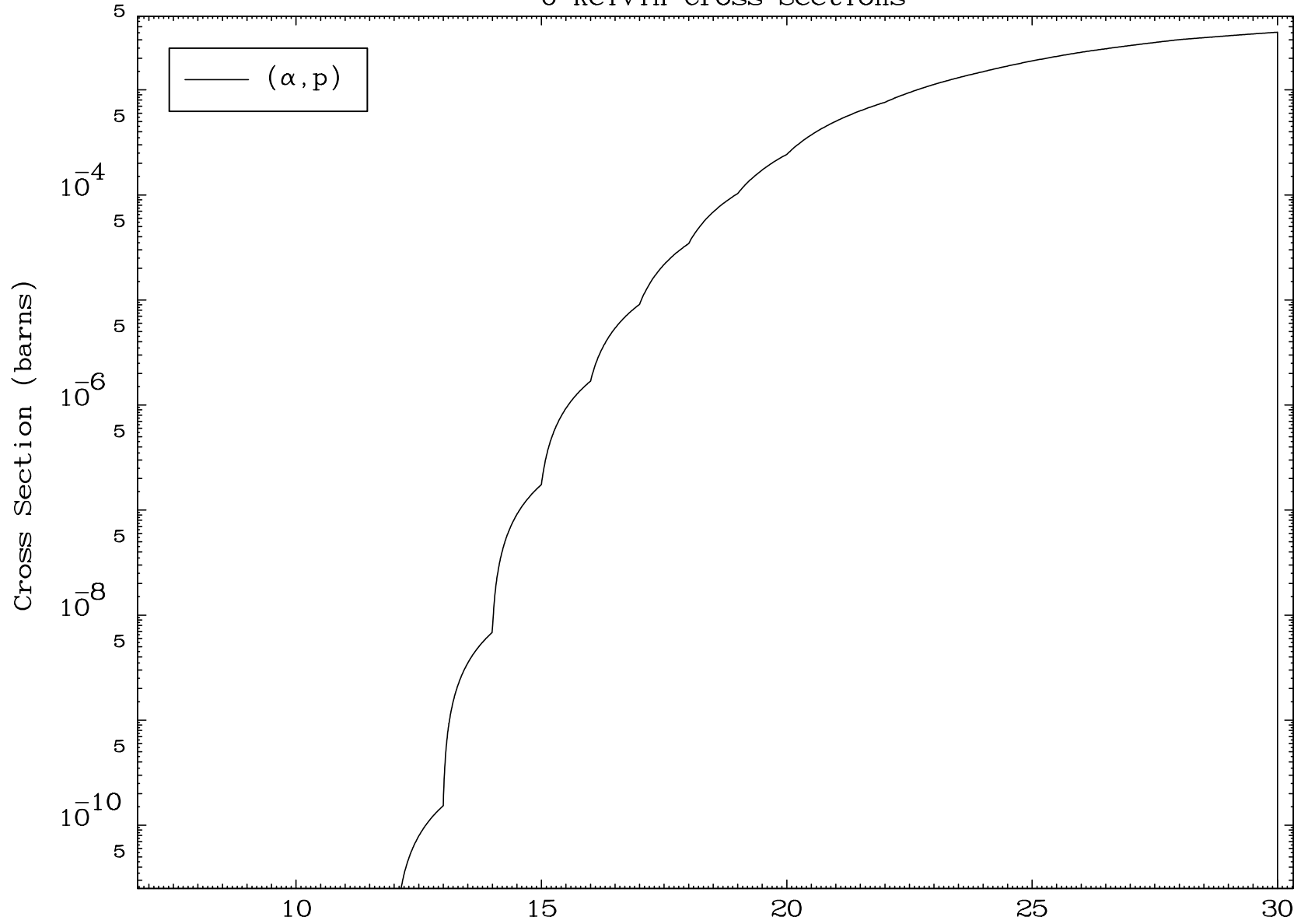
73-Ta-176

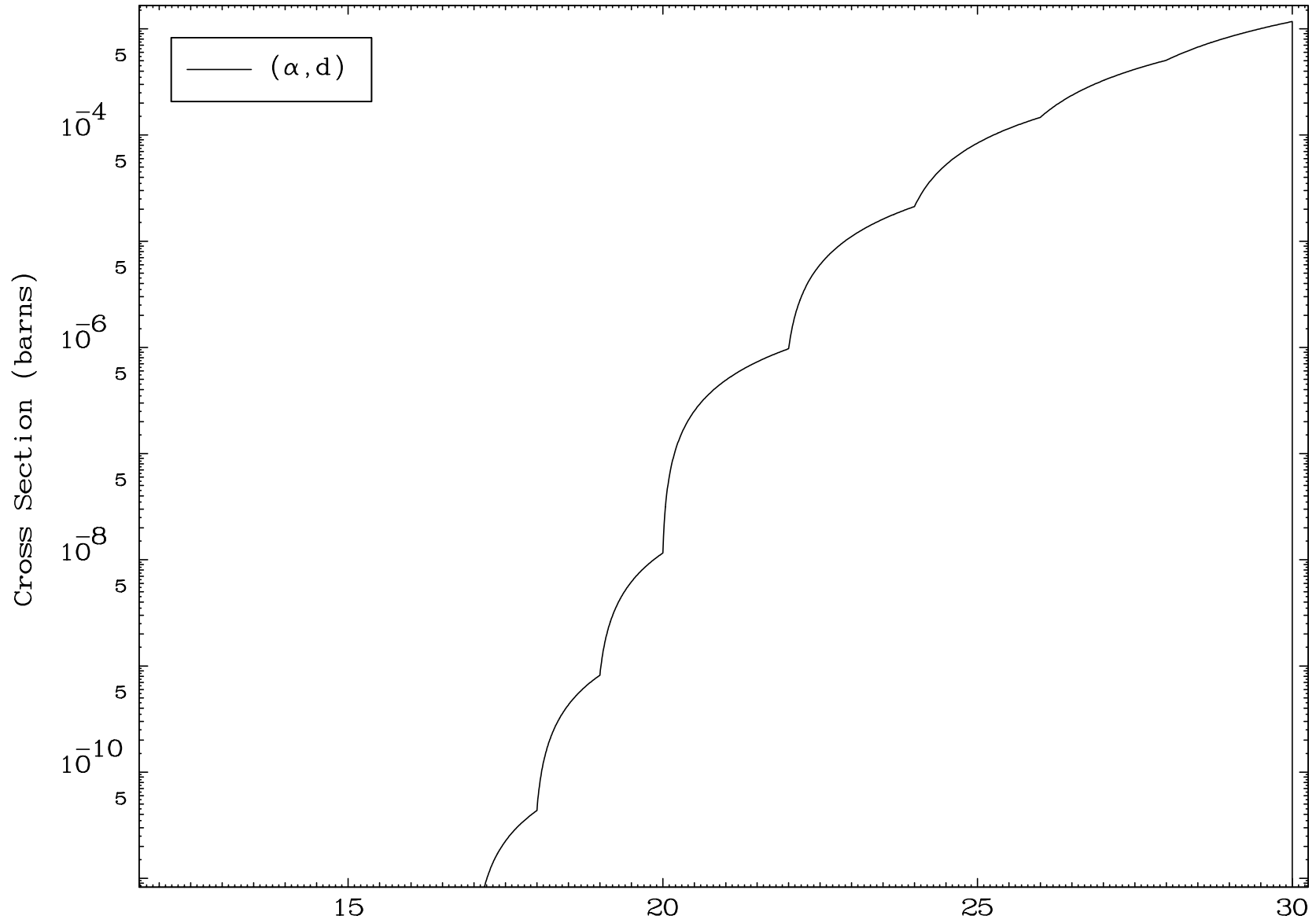


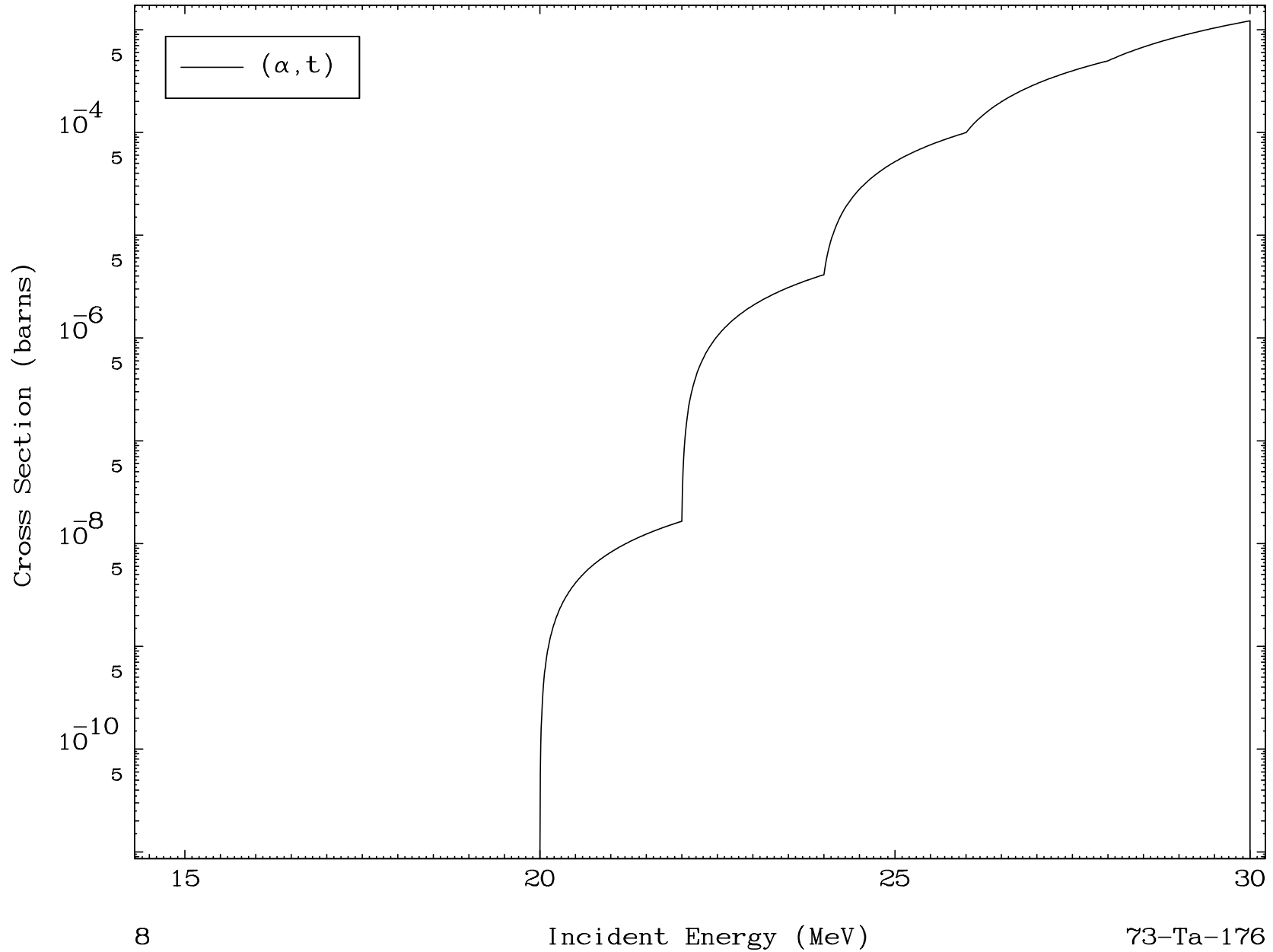
5

Incident Energy (MeV)

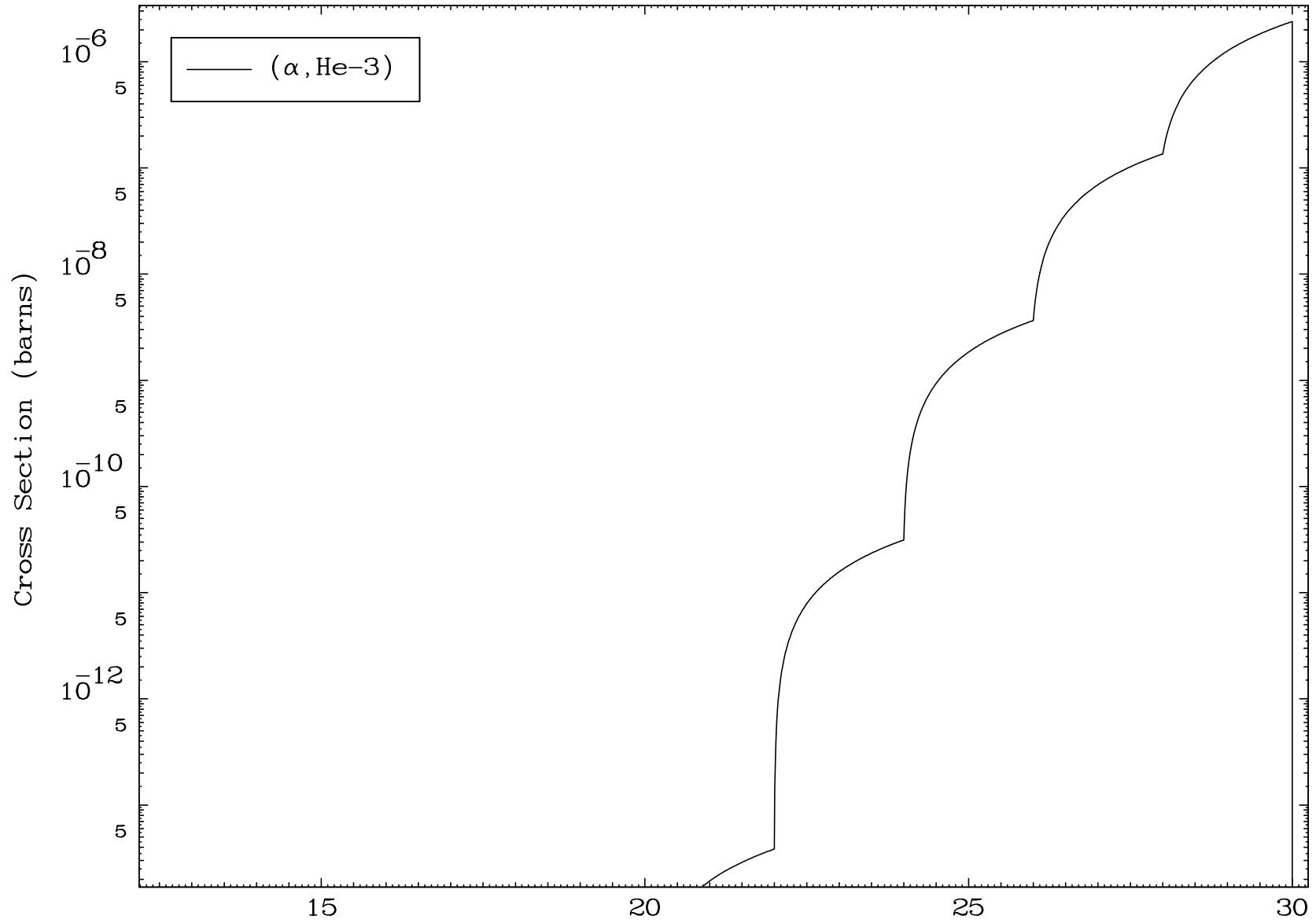
73-Ta-176







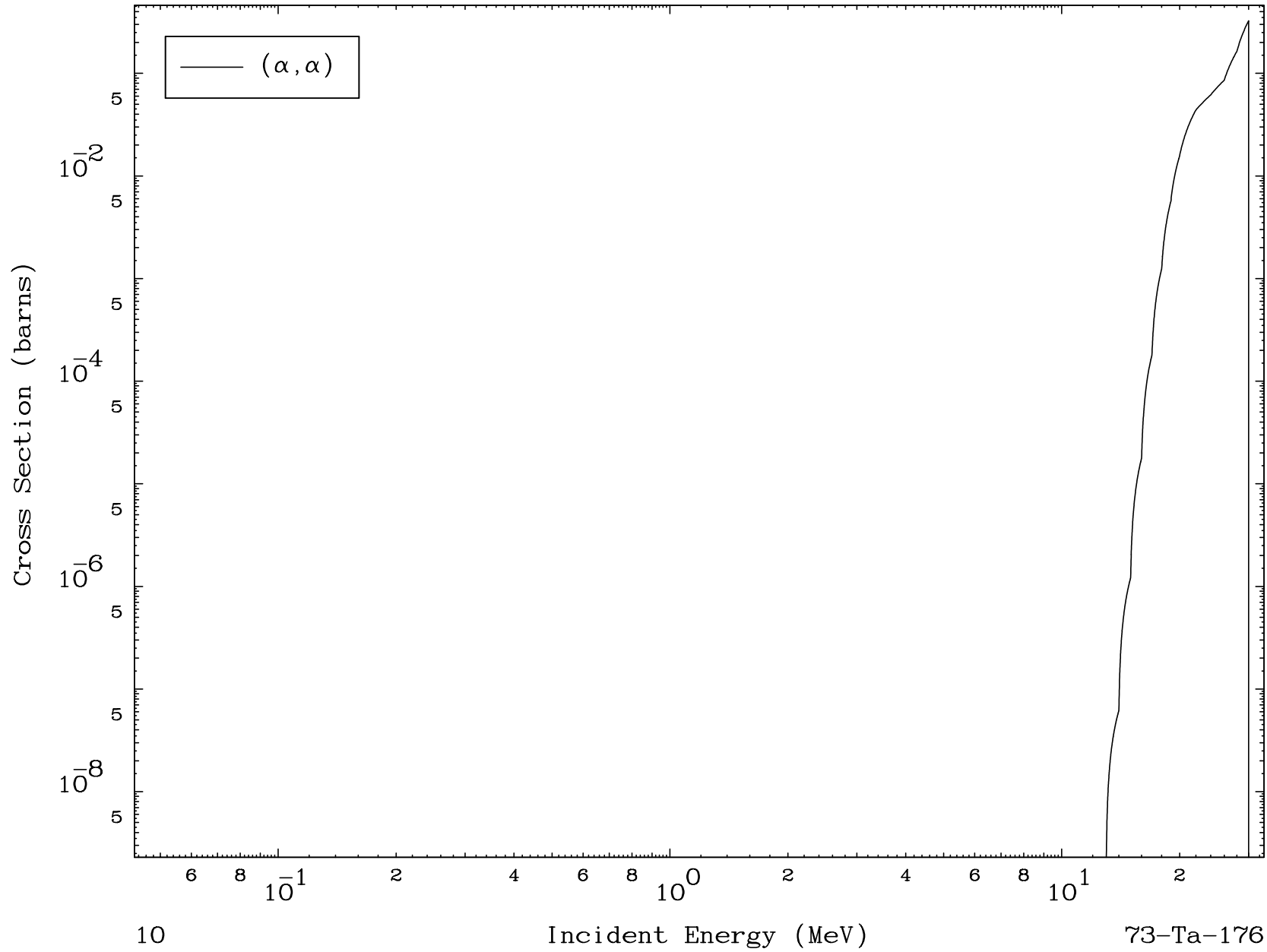




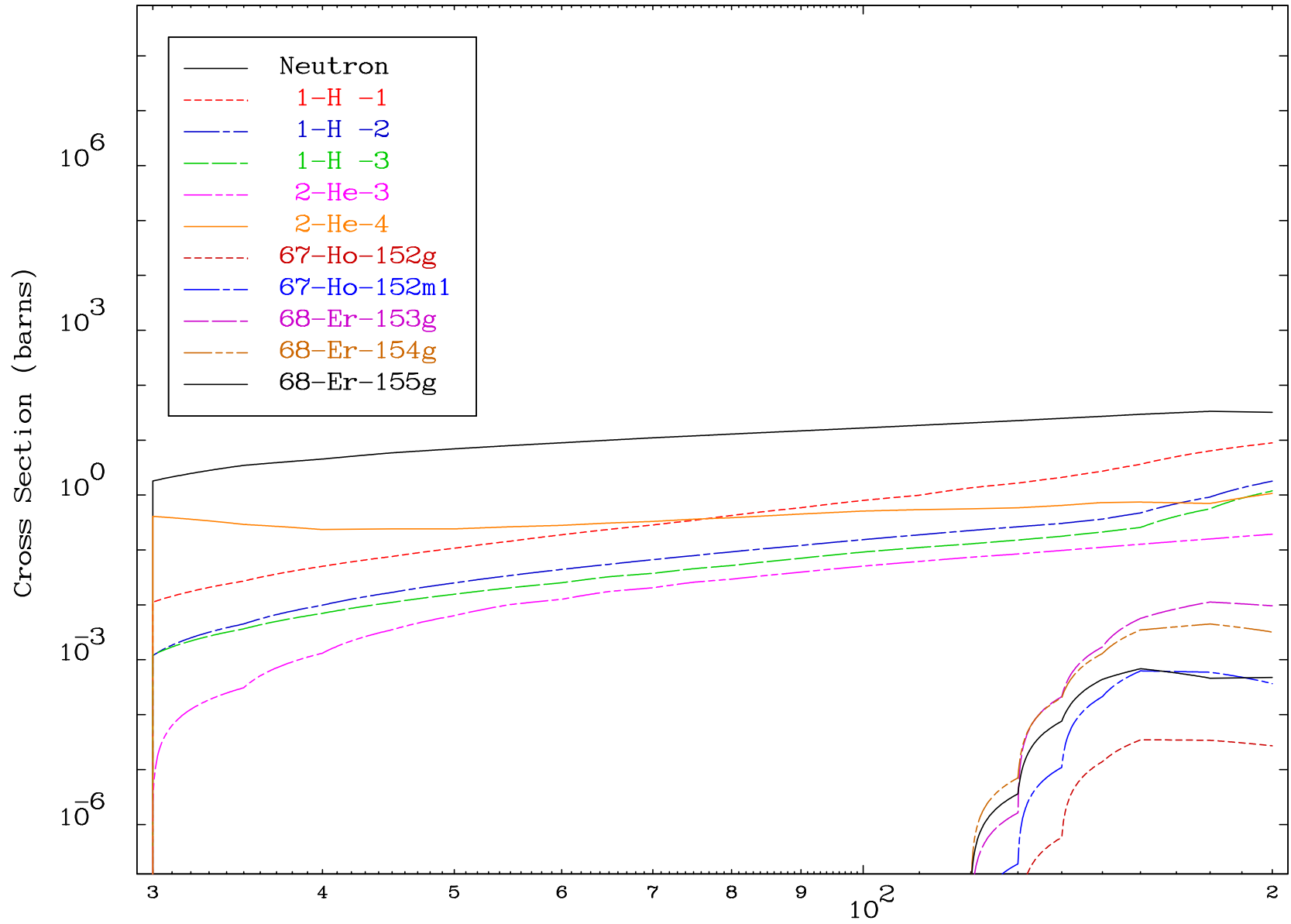
MAT 7313

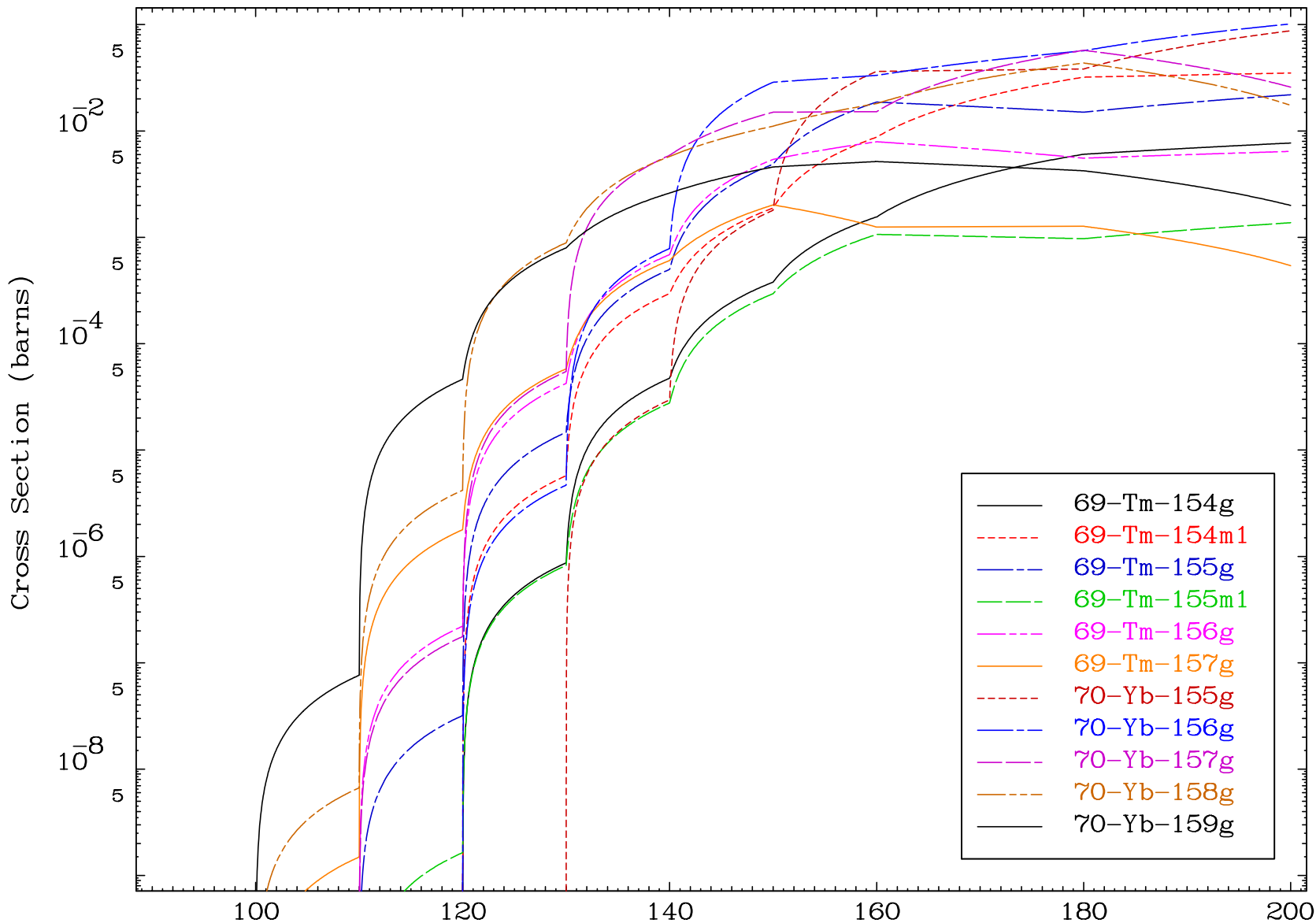
( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

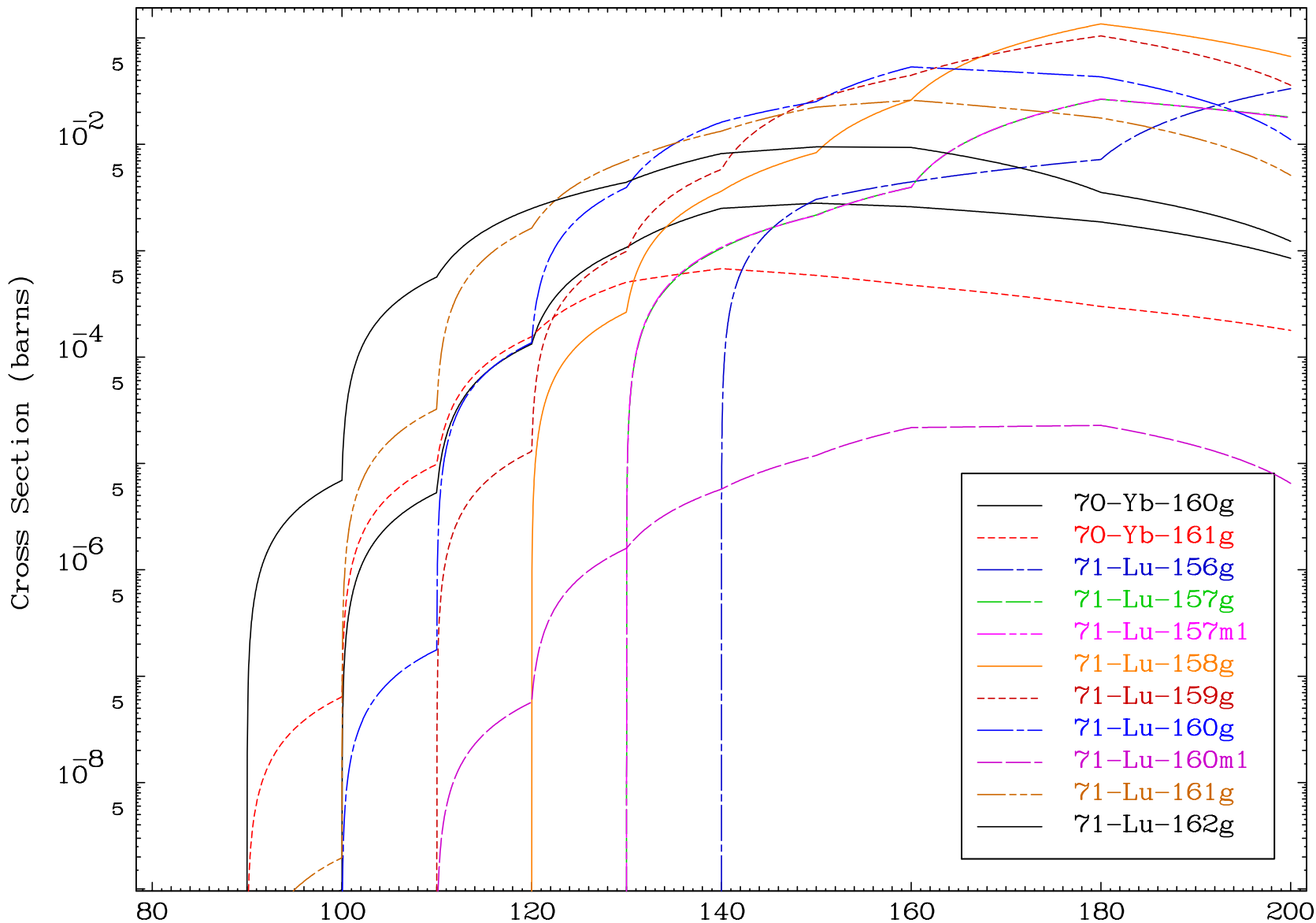
73-Ta-176

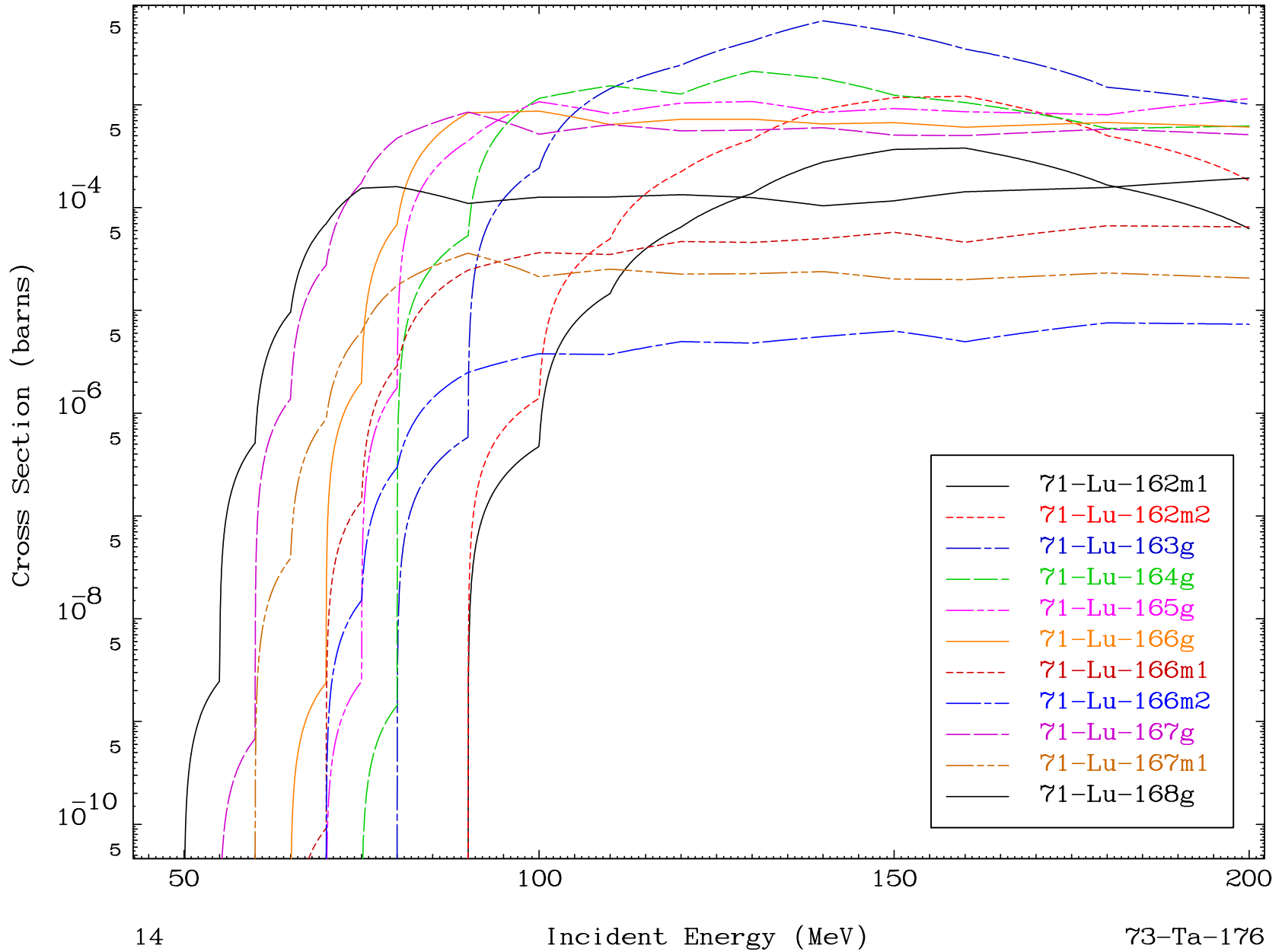


Radionuclide Production Cross Section

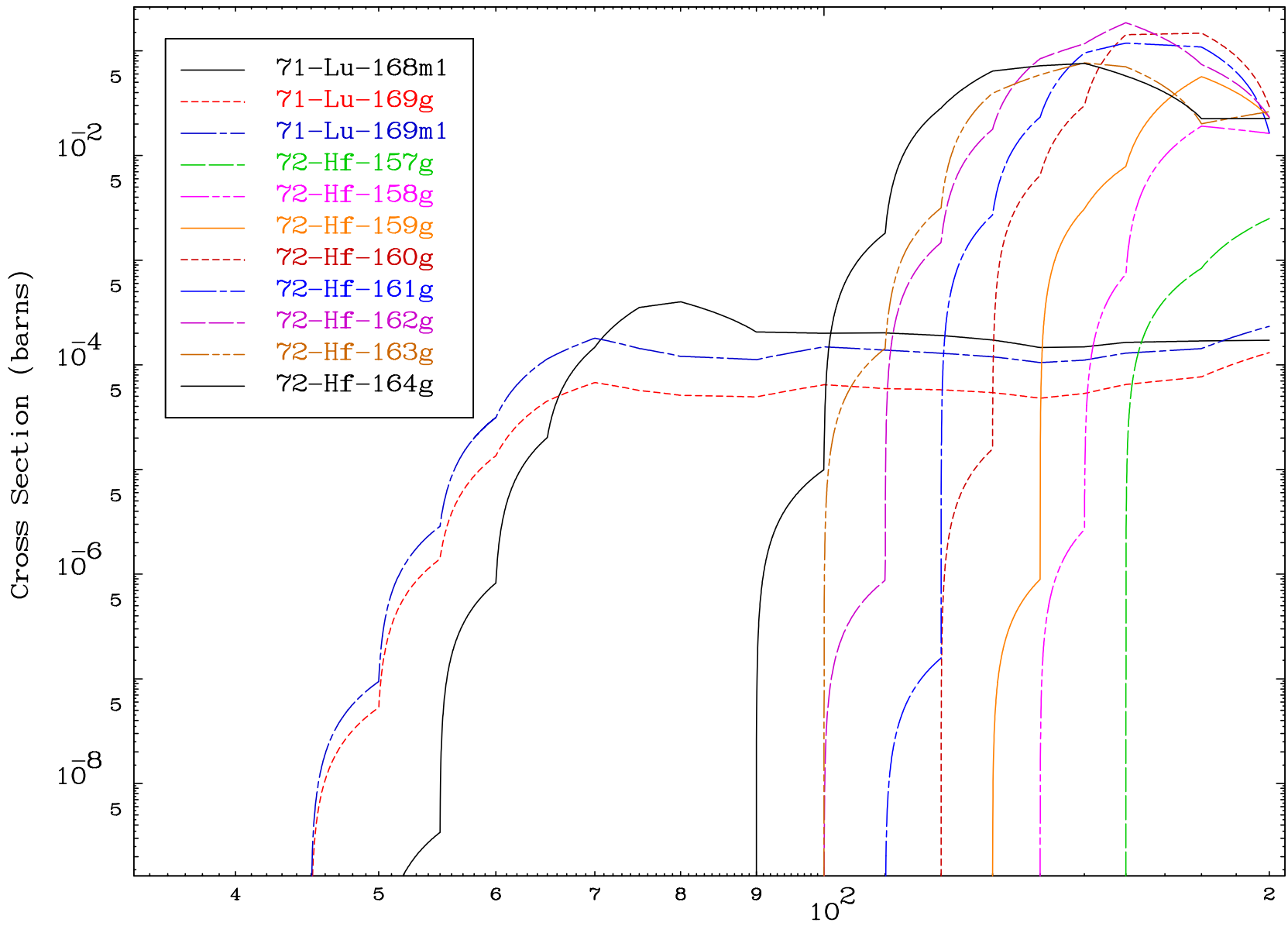


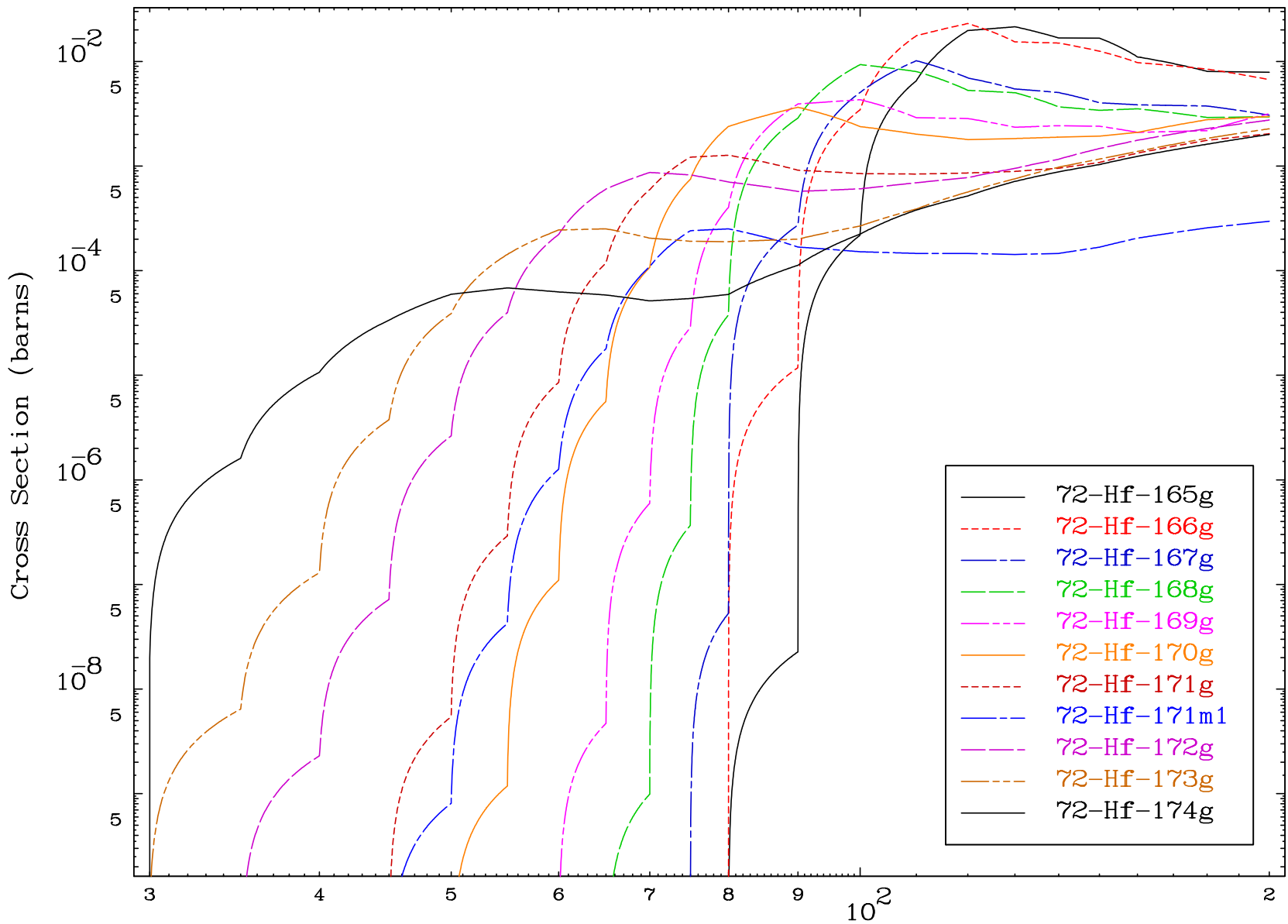






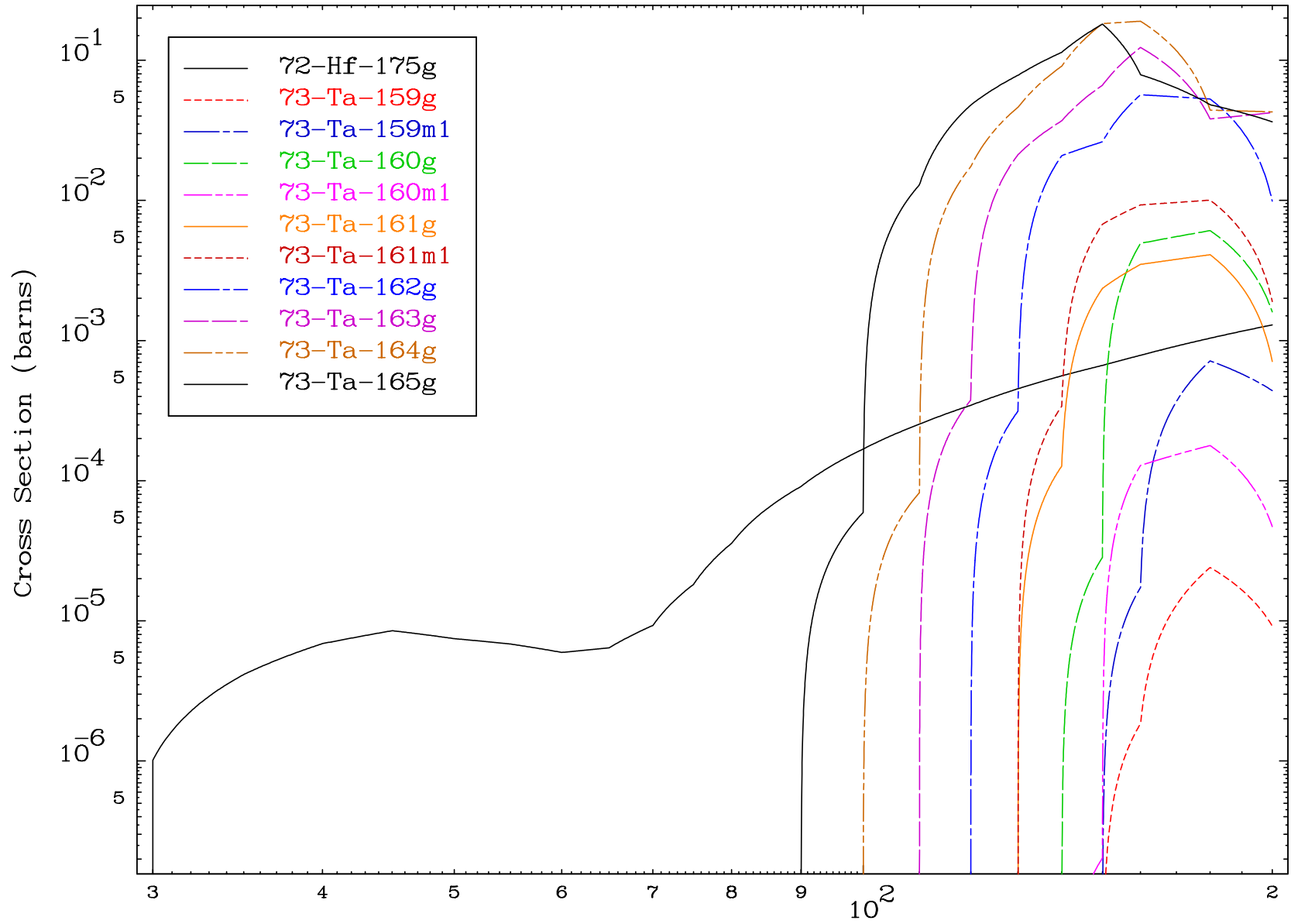
Radionuclide Production Cross Section

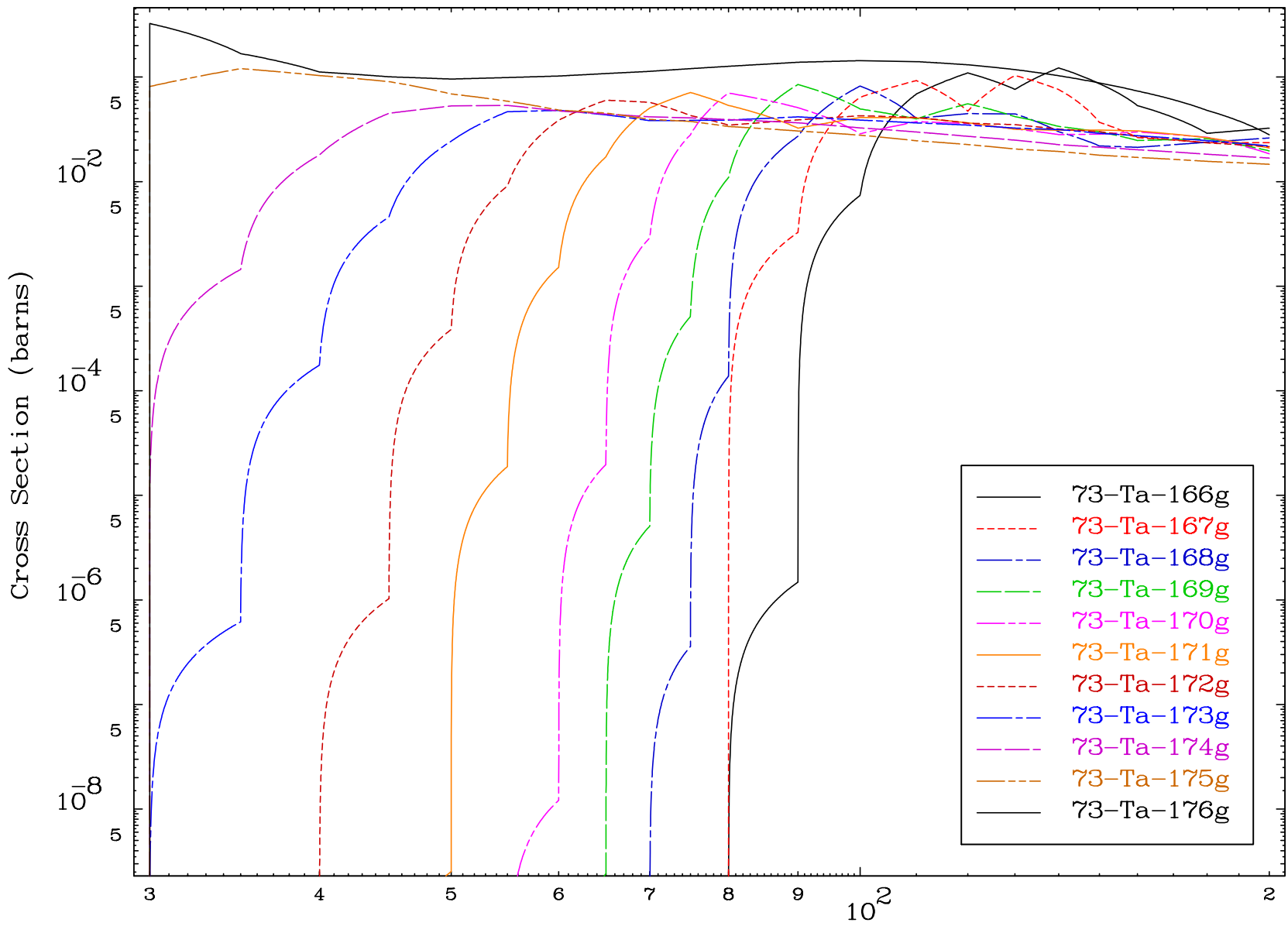




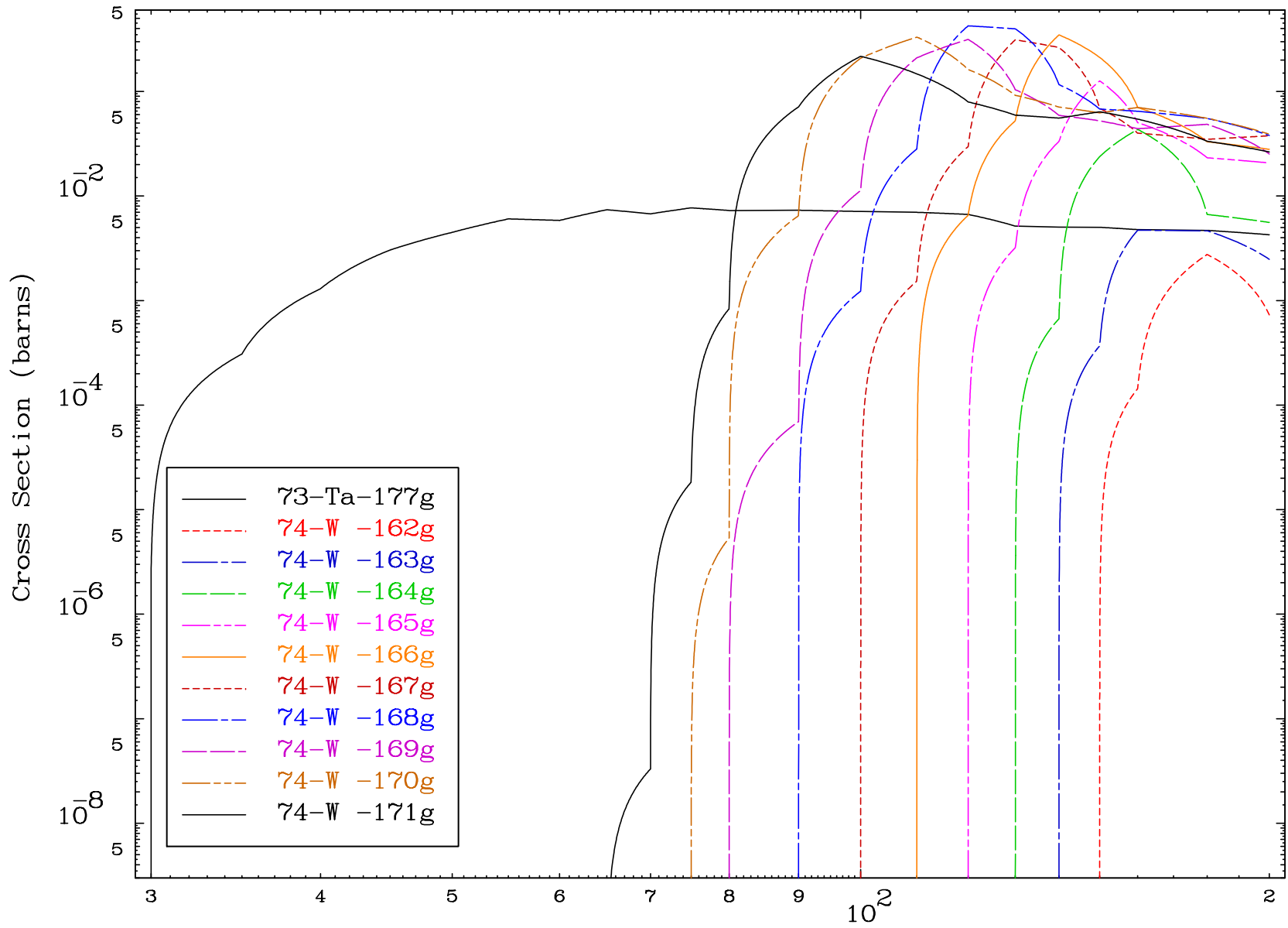


Radionuclide Production Cross Section

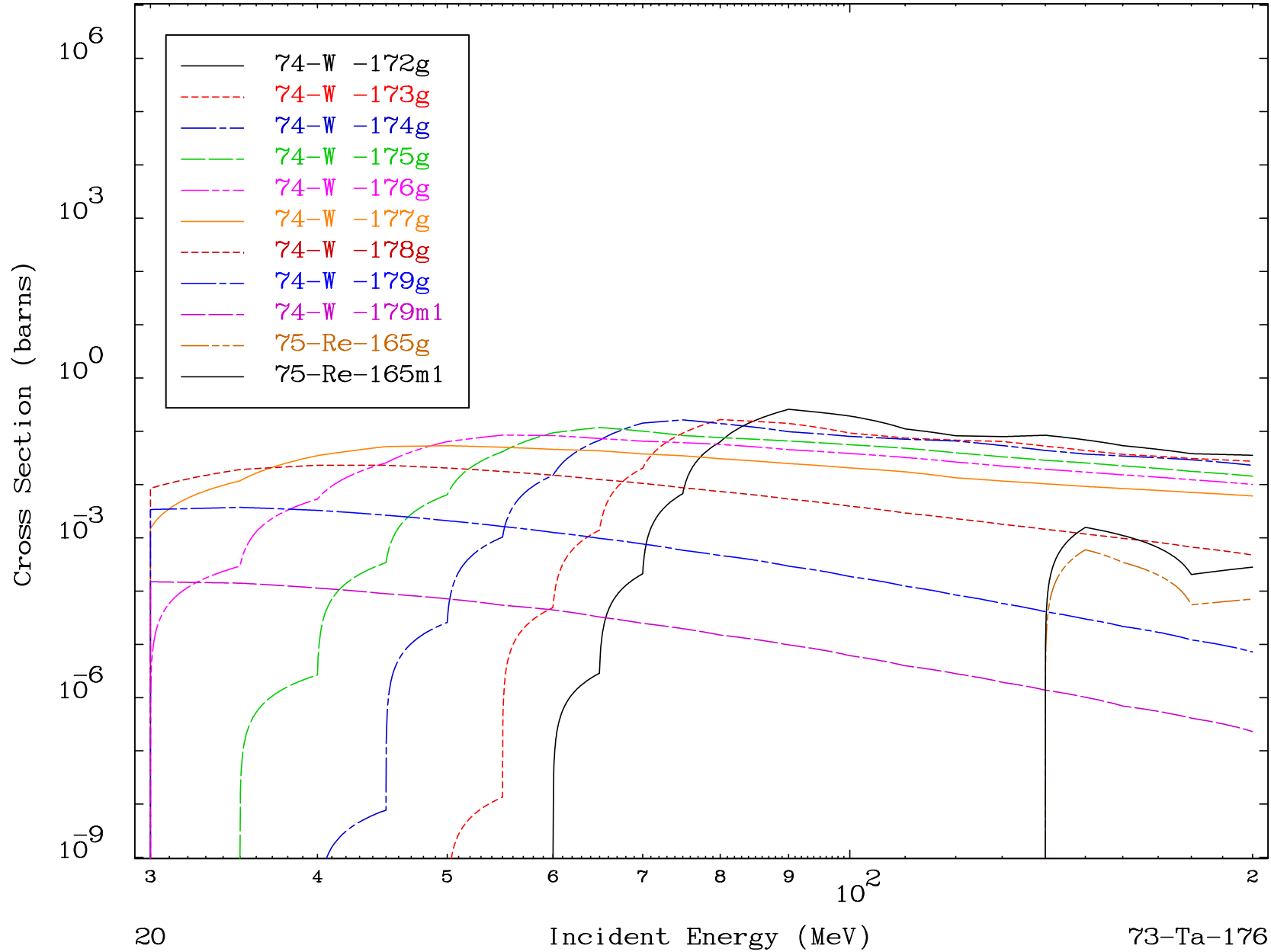




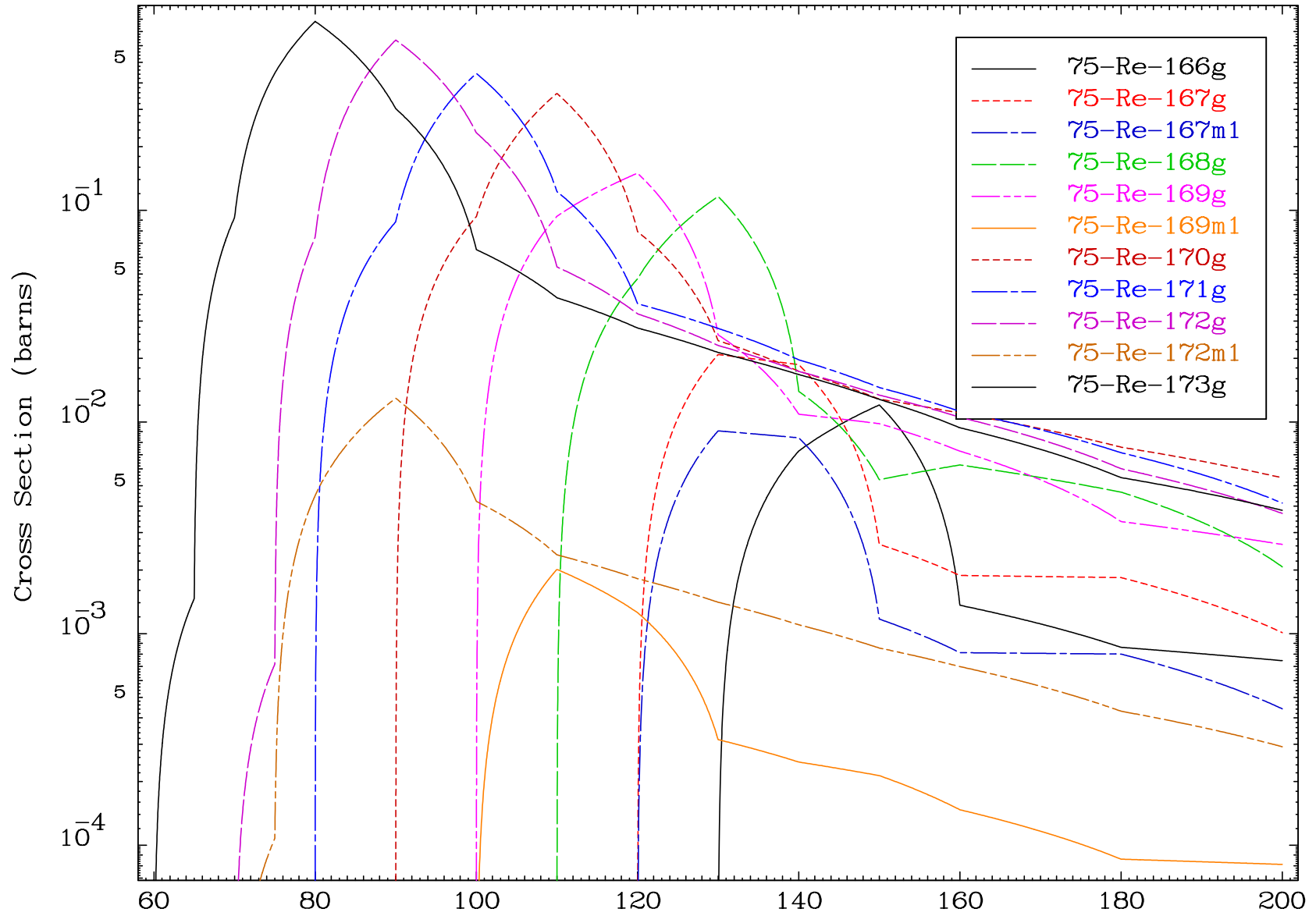
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

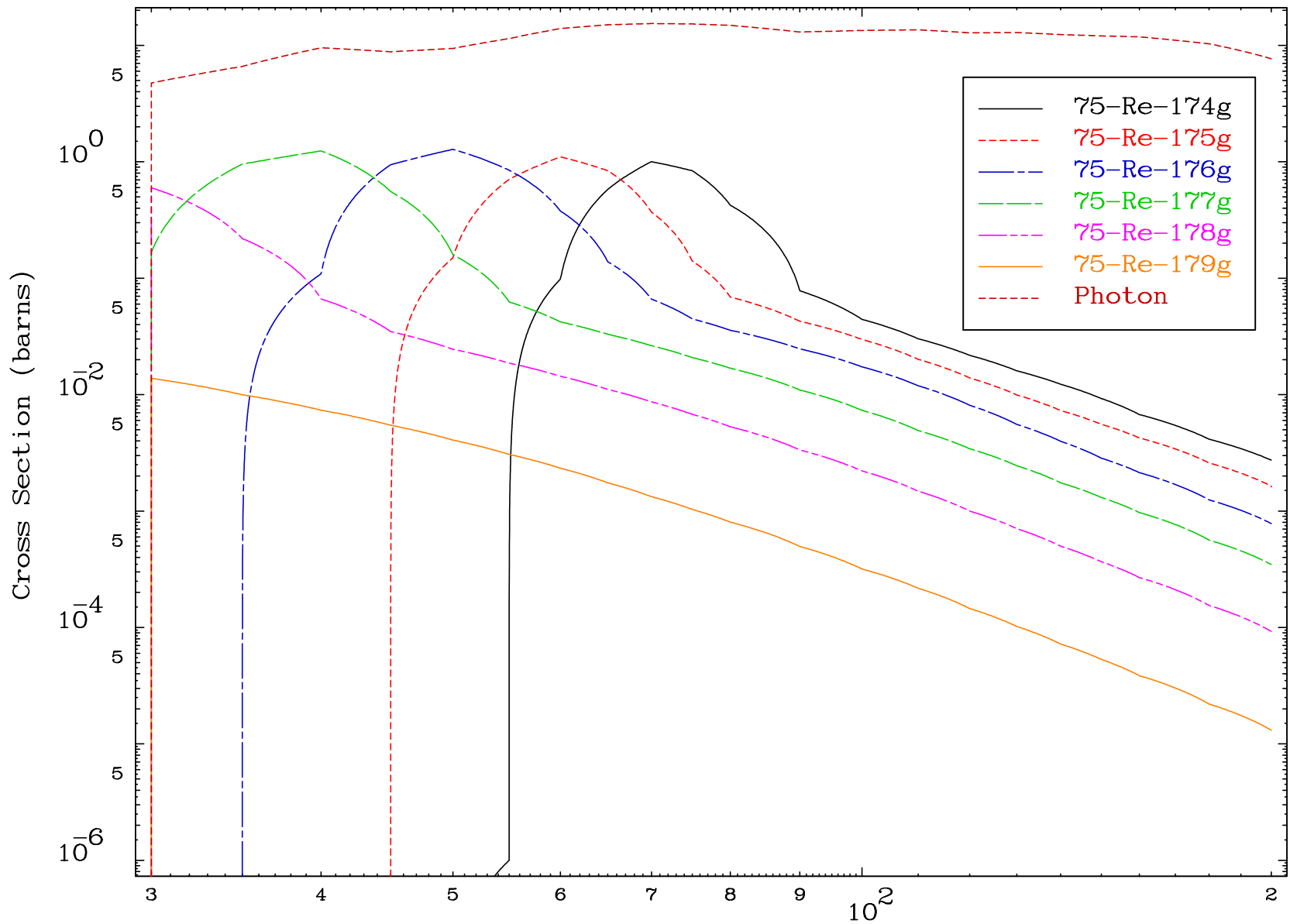


MAT 7313

( $\alpha$ , remainder)

73-Ta-176

### Radionuclide Production Cross Section



22

Incident Energy (MeV)

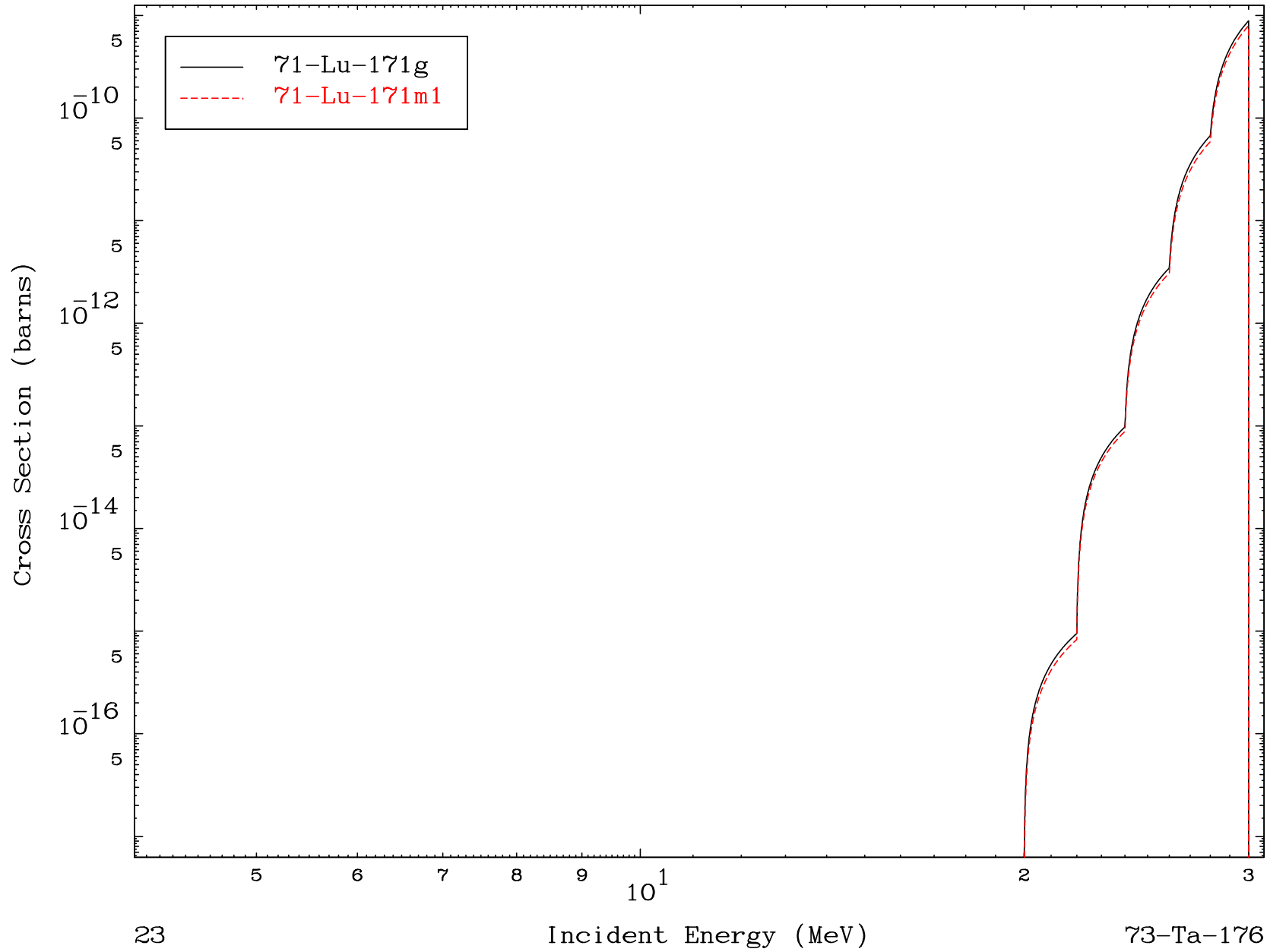
73-Ta-176

MAT 7313

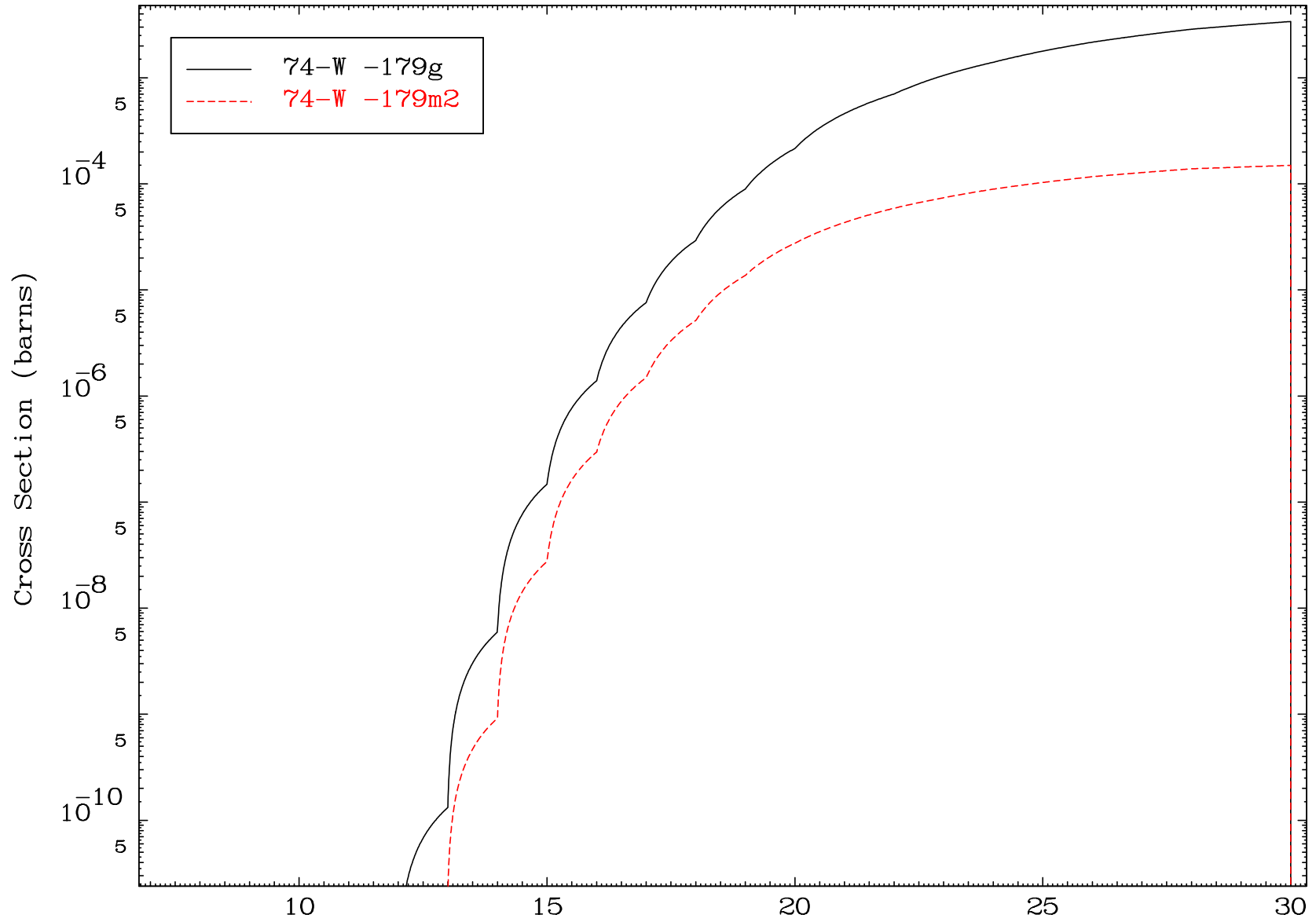
$(\alpha, n')$   $2\alpha$

73-Ta-176

Radionuclide Production Cross Section



Radionuclide Production Cross Section



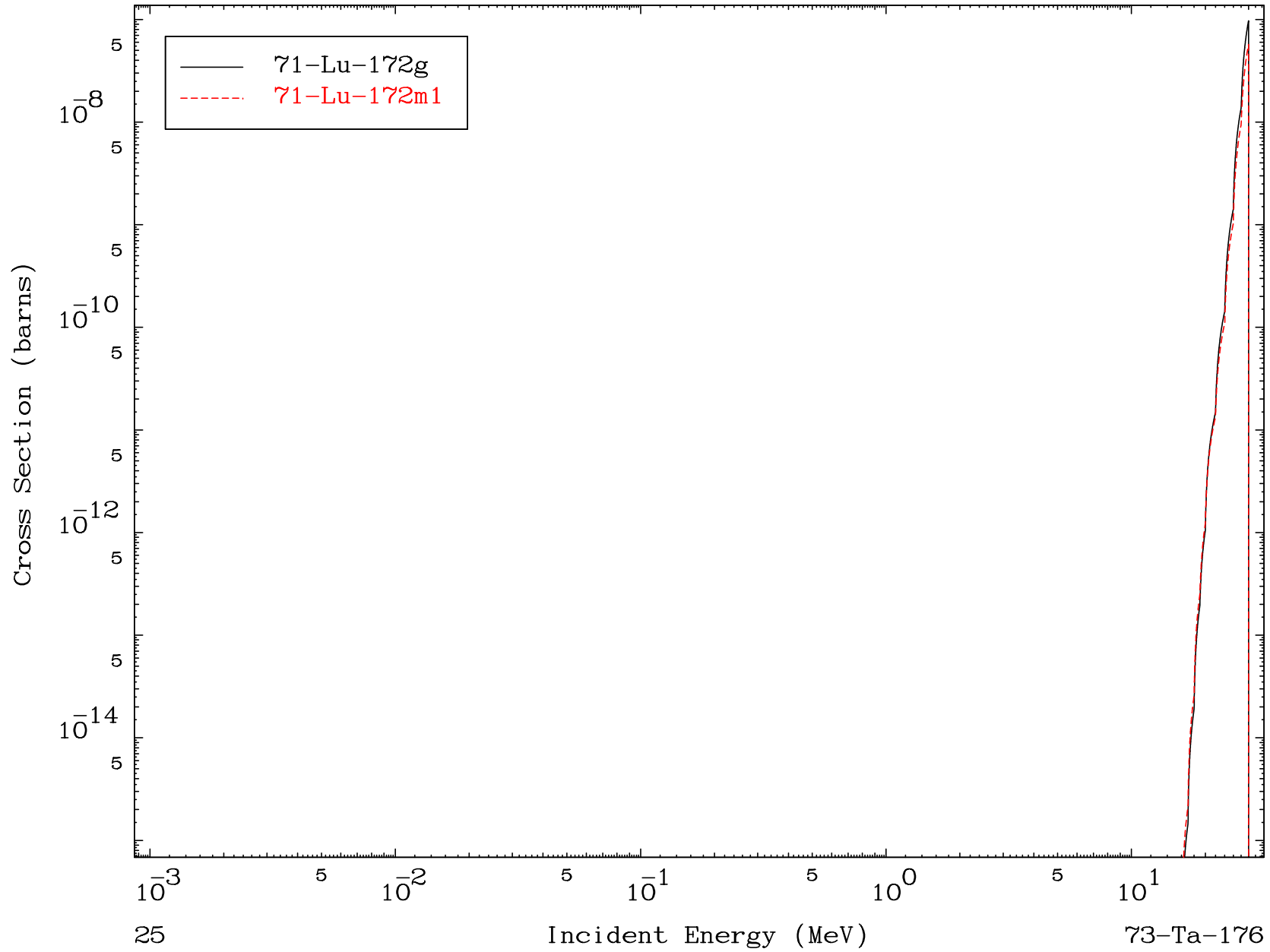


MAT 7313

( $\alpha, 2\alpha$ )

73-Ta-176

Radionuclide Production Cross Section



MAT 7313

( $\alpha, 2p$ )

73-Ta-176

Radionuclide Production Cross Section

