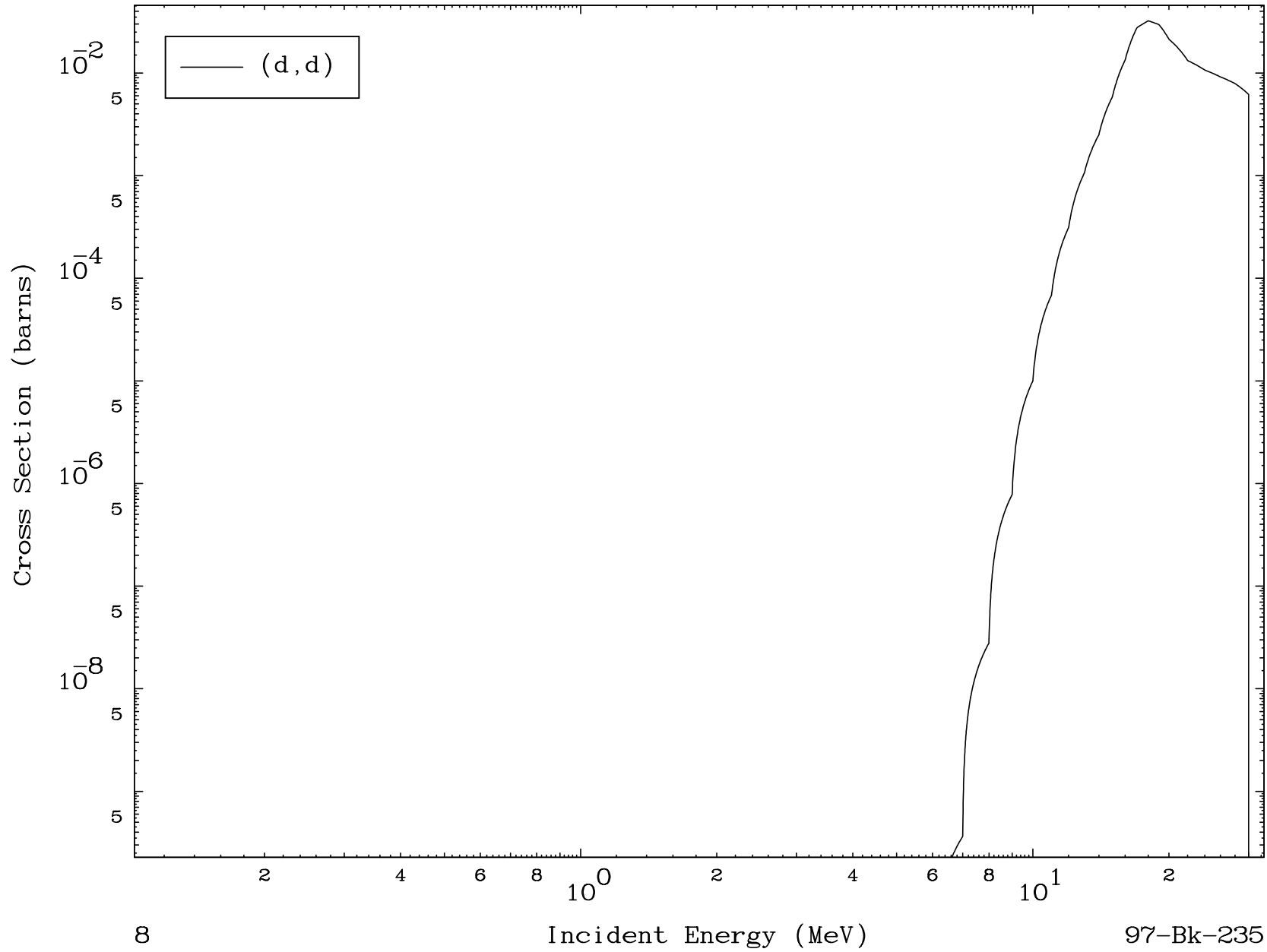
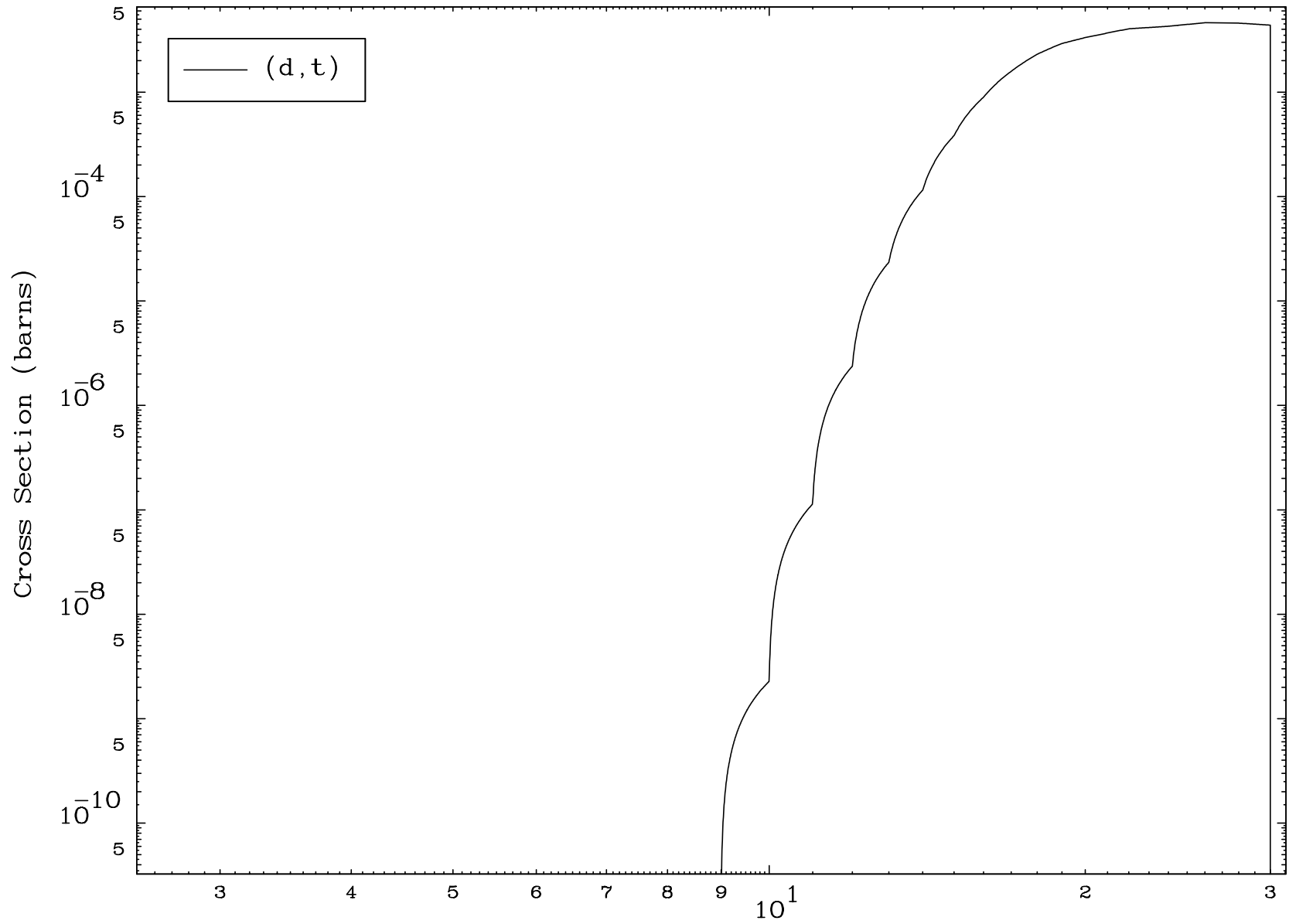


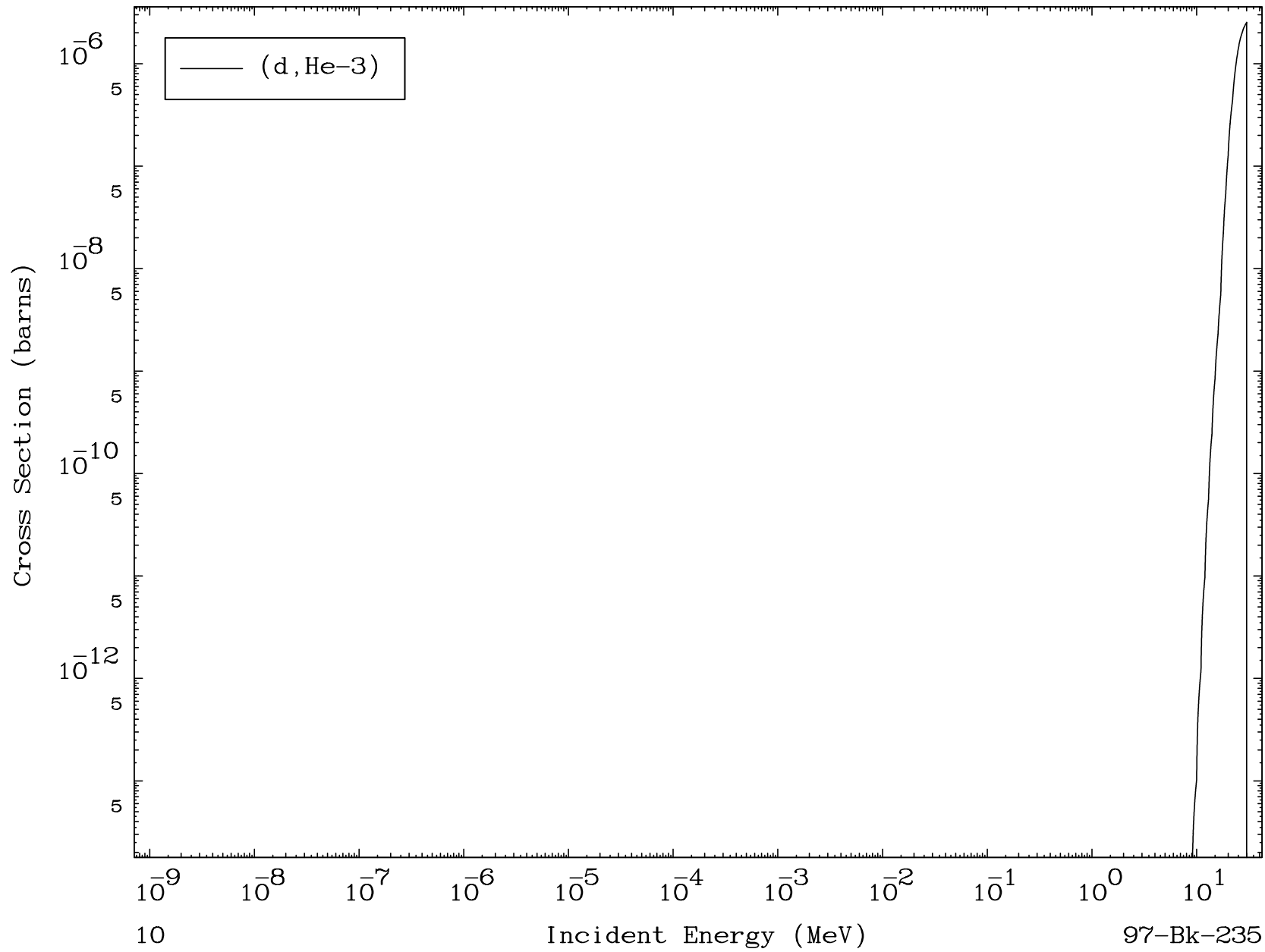
MAT 9710

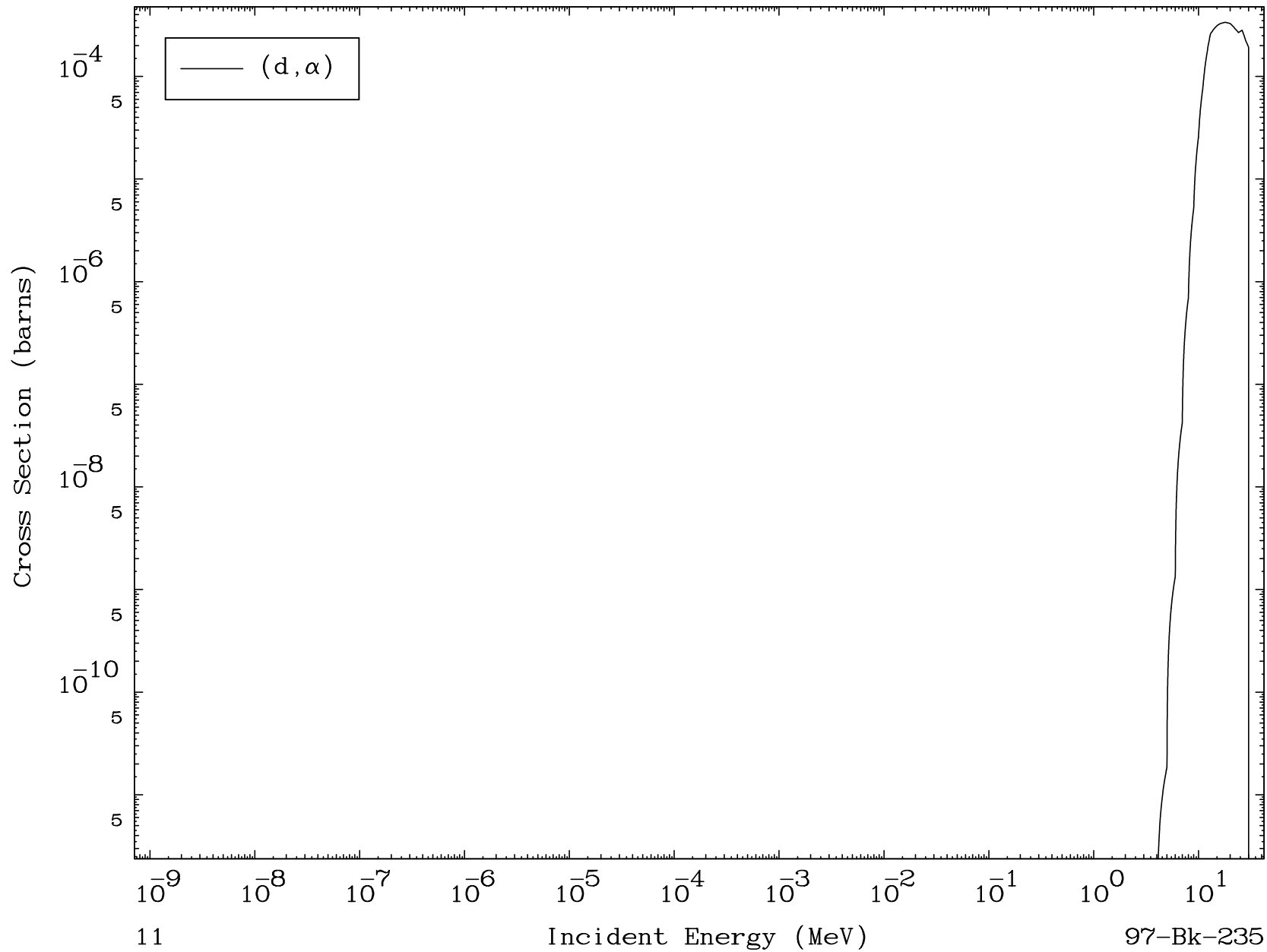
(d,d) Levels
0 Kelvin Cross Sections

97-Bk-235







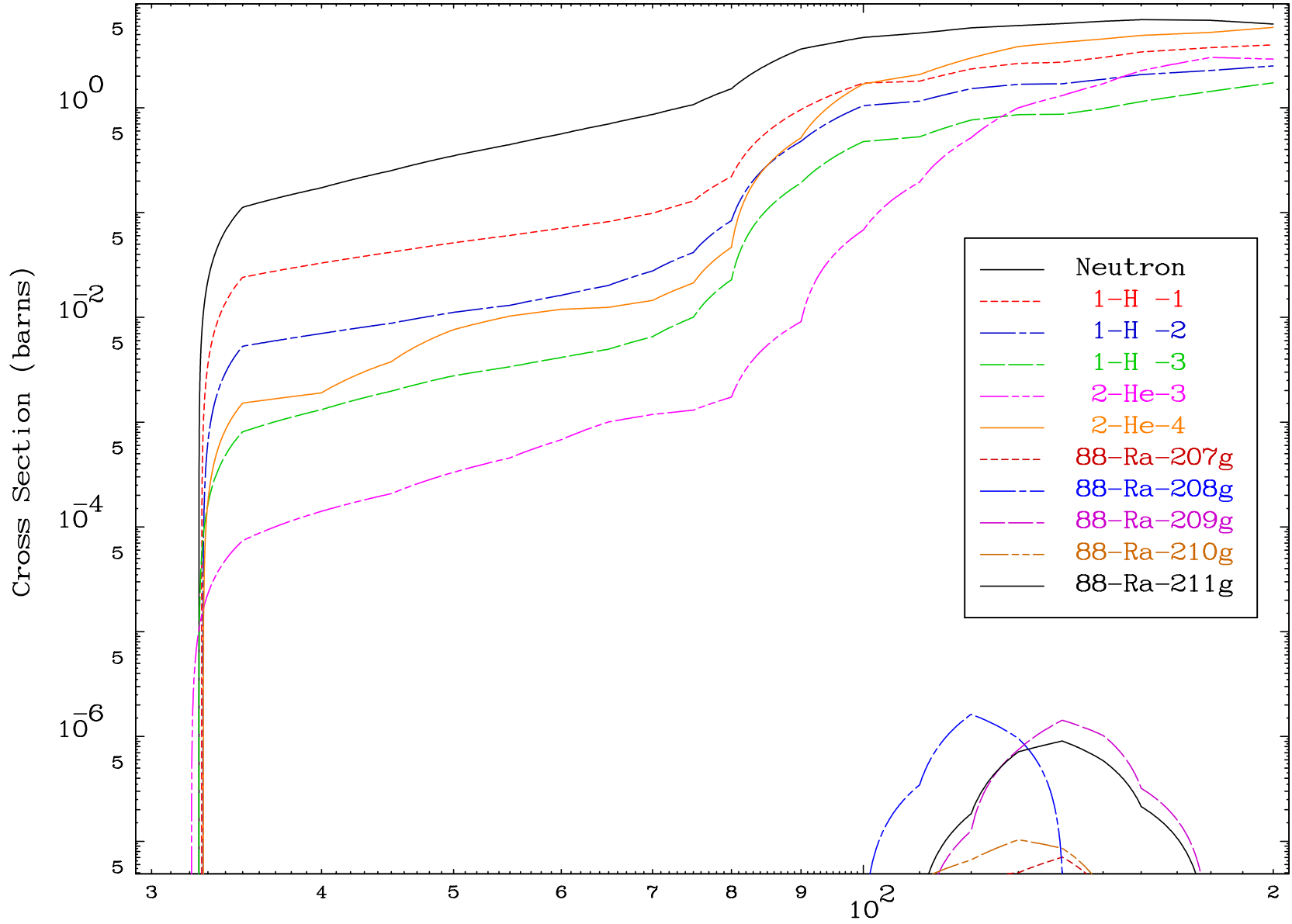


MAT 9710

(d,remainder)

97-Bk-235

Radionuclide Production Cross Section

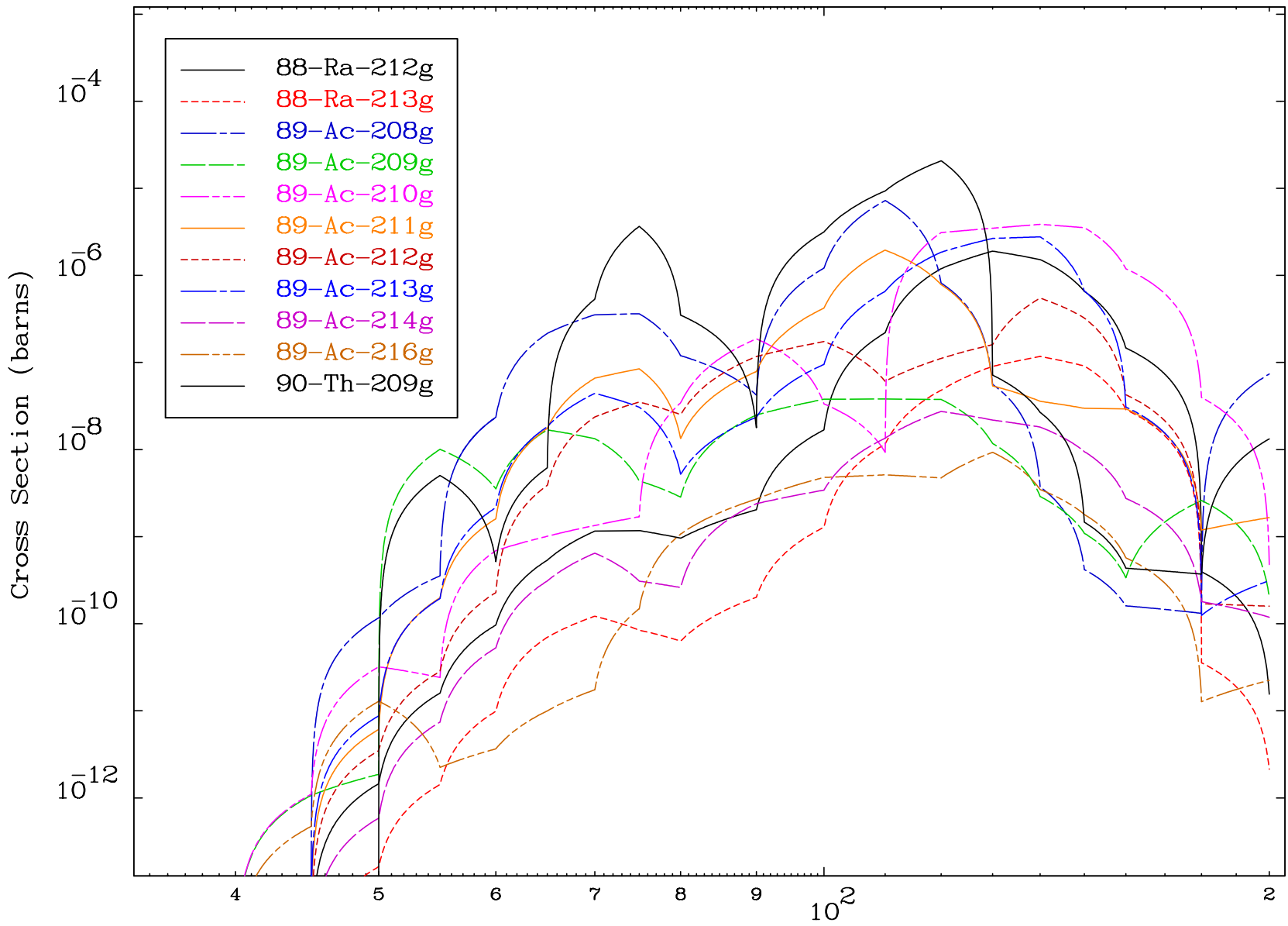


12

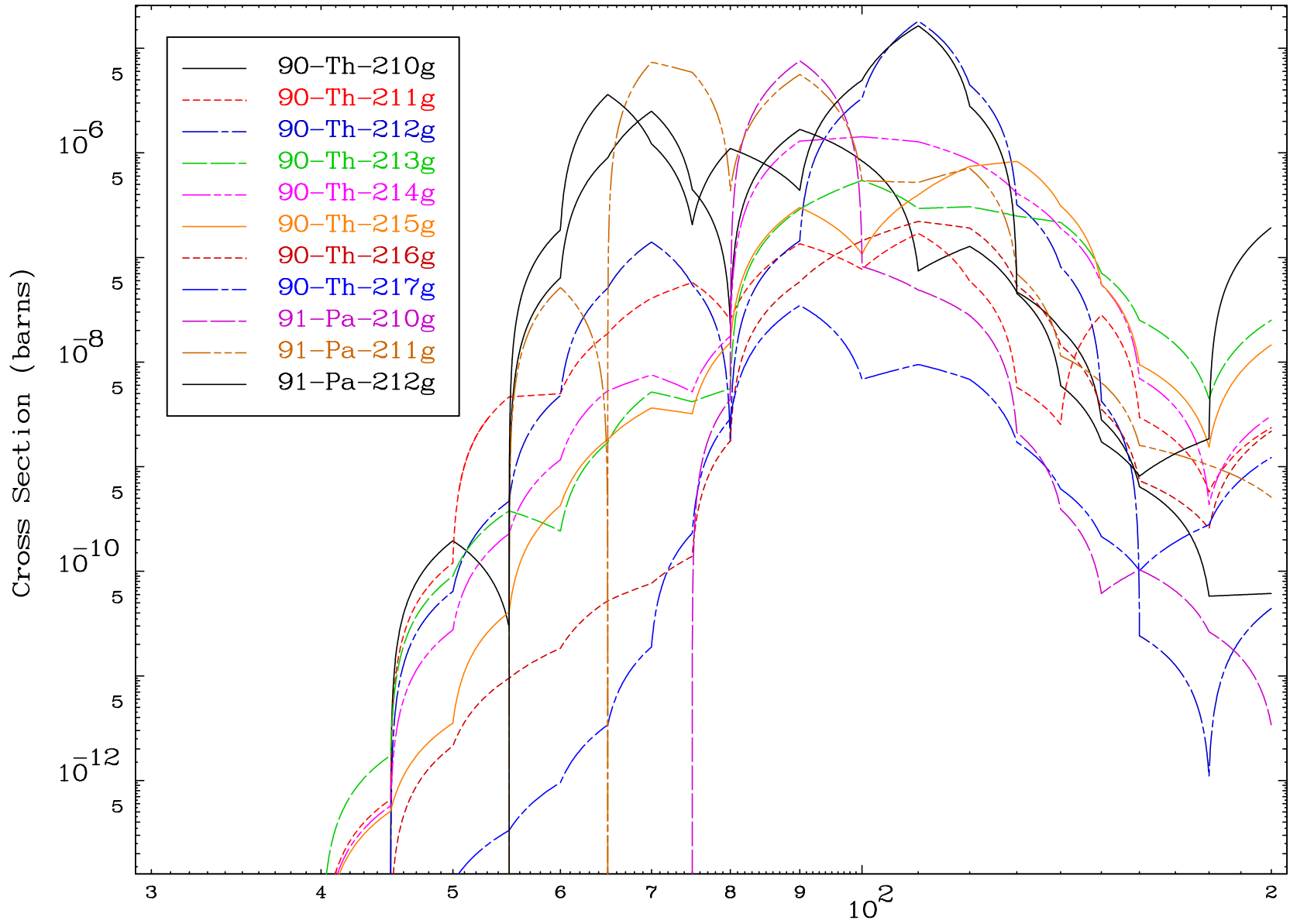
Incident Energy (MeV)

97-Bk-235

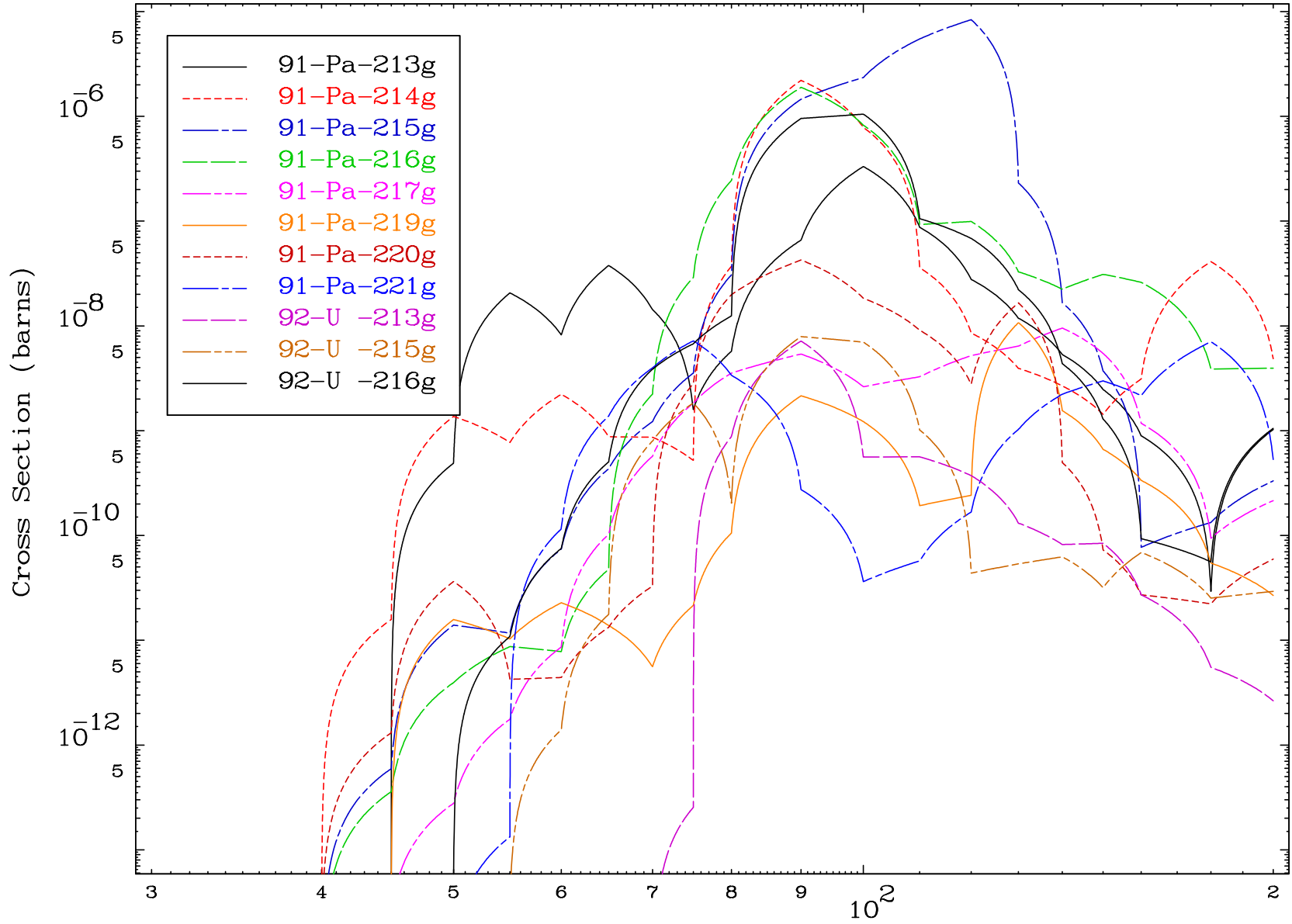
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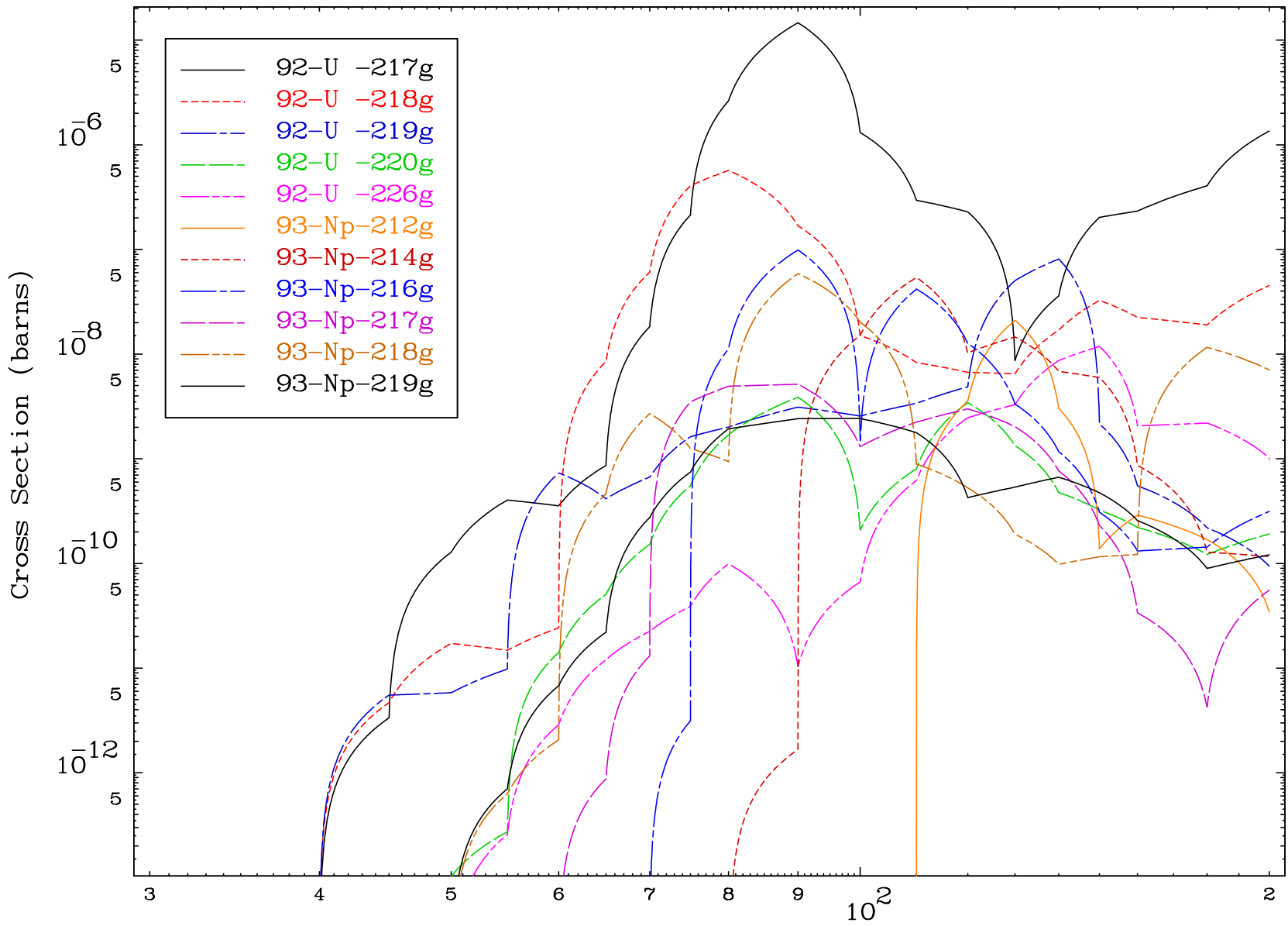
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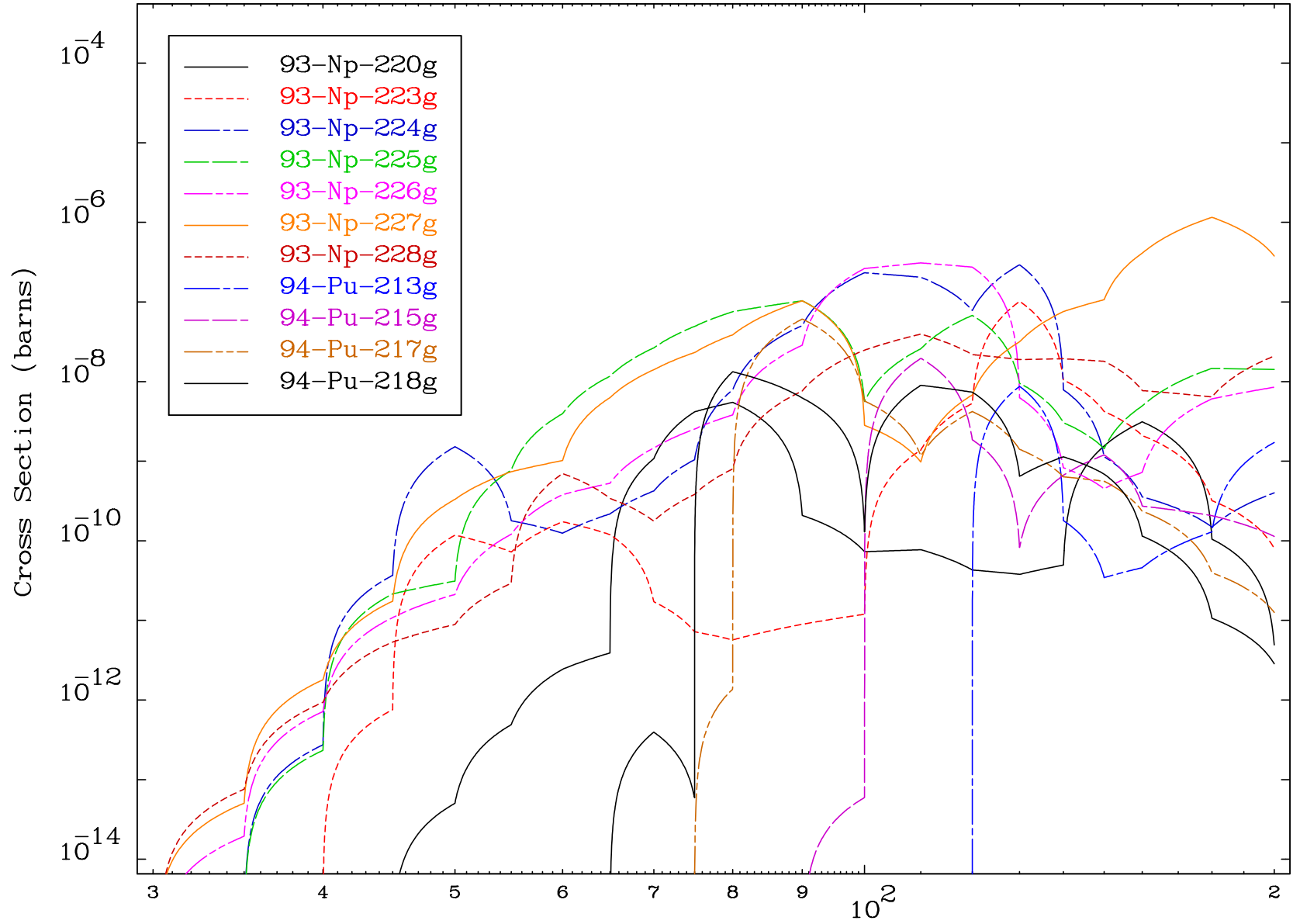
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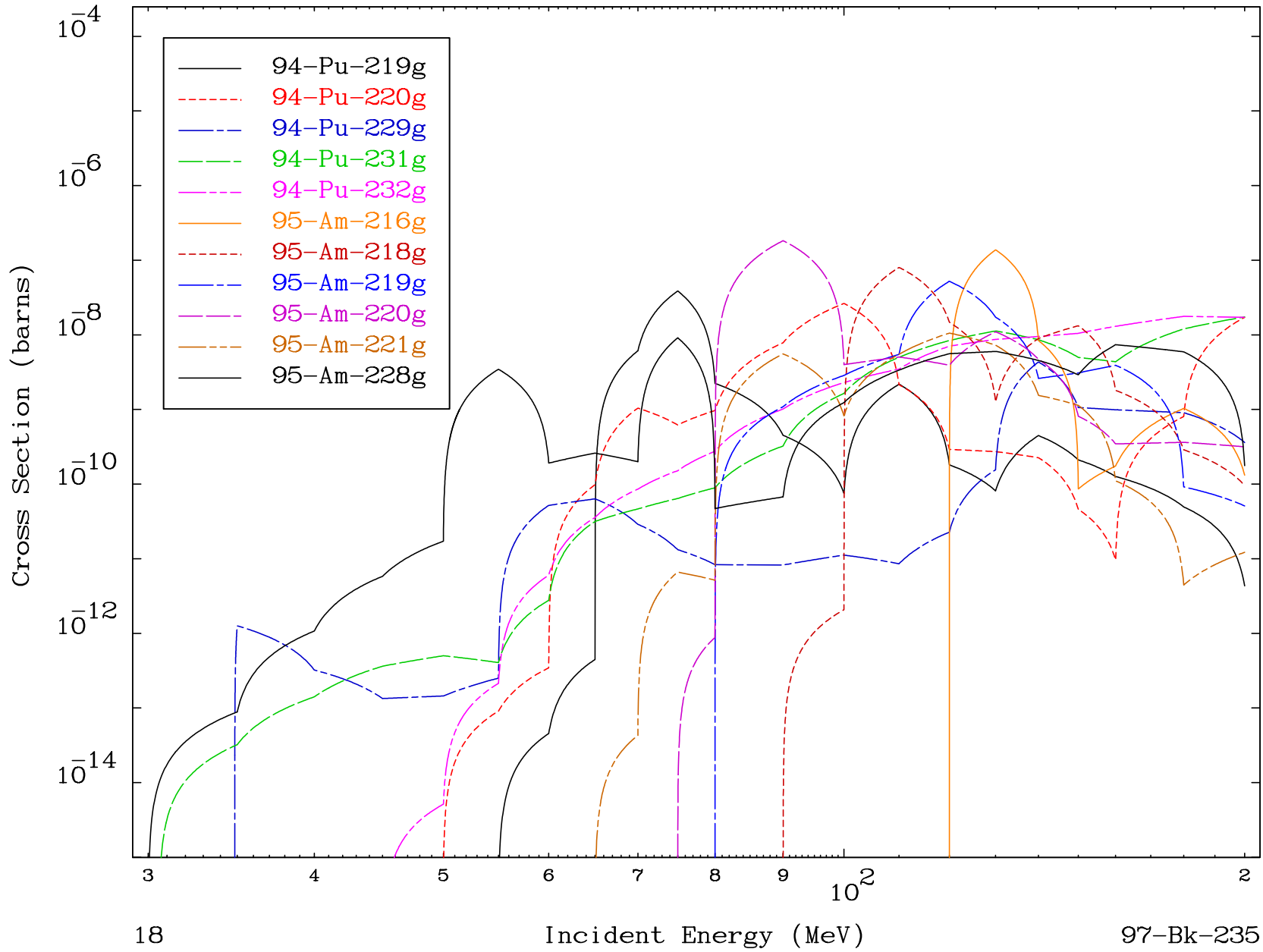
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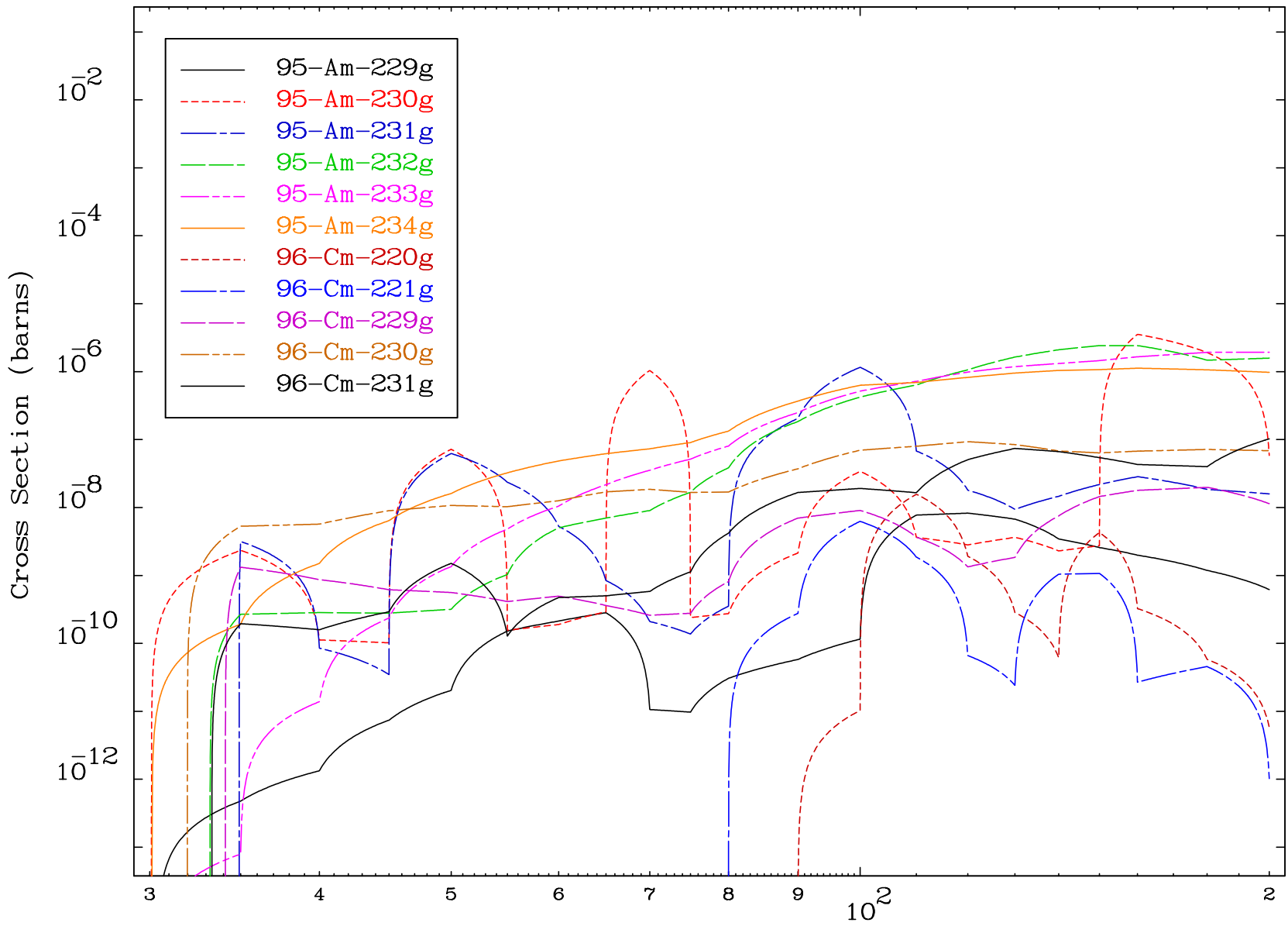
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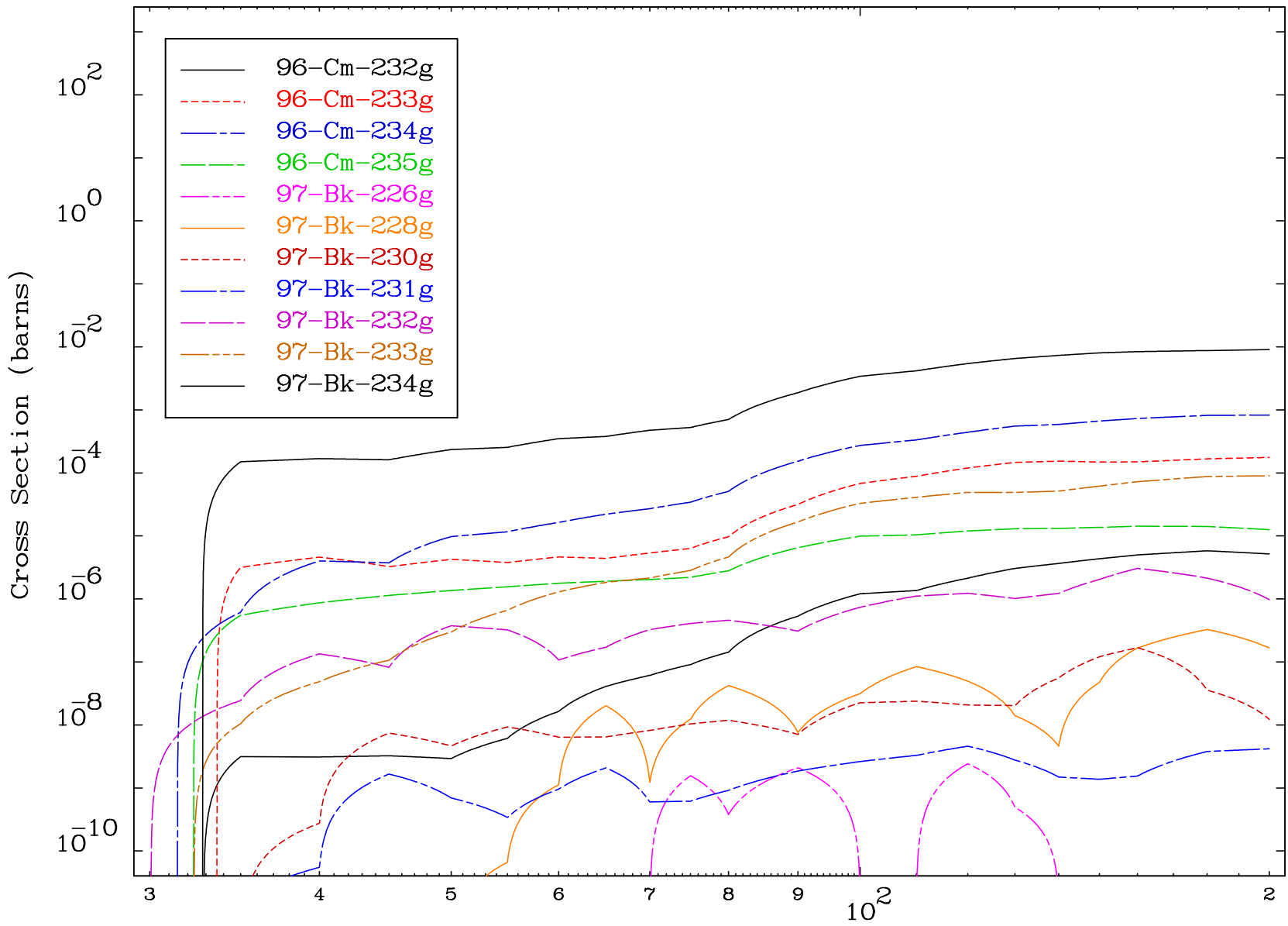
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

