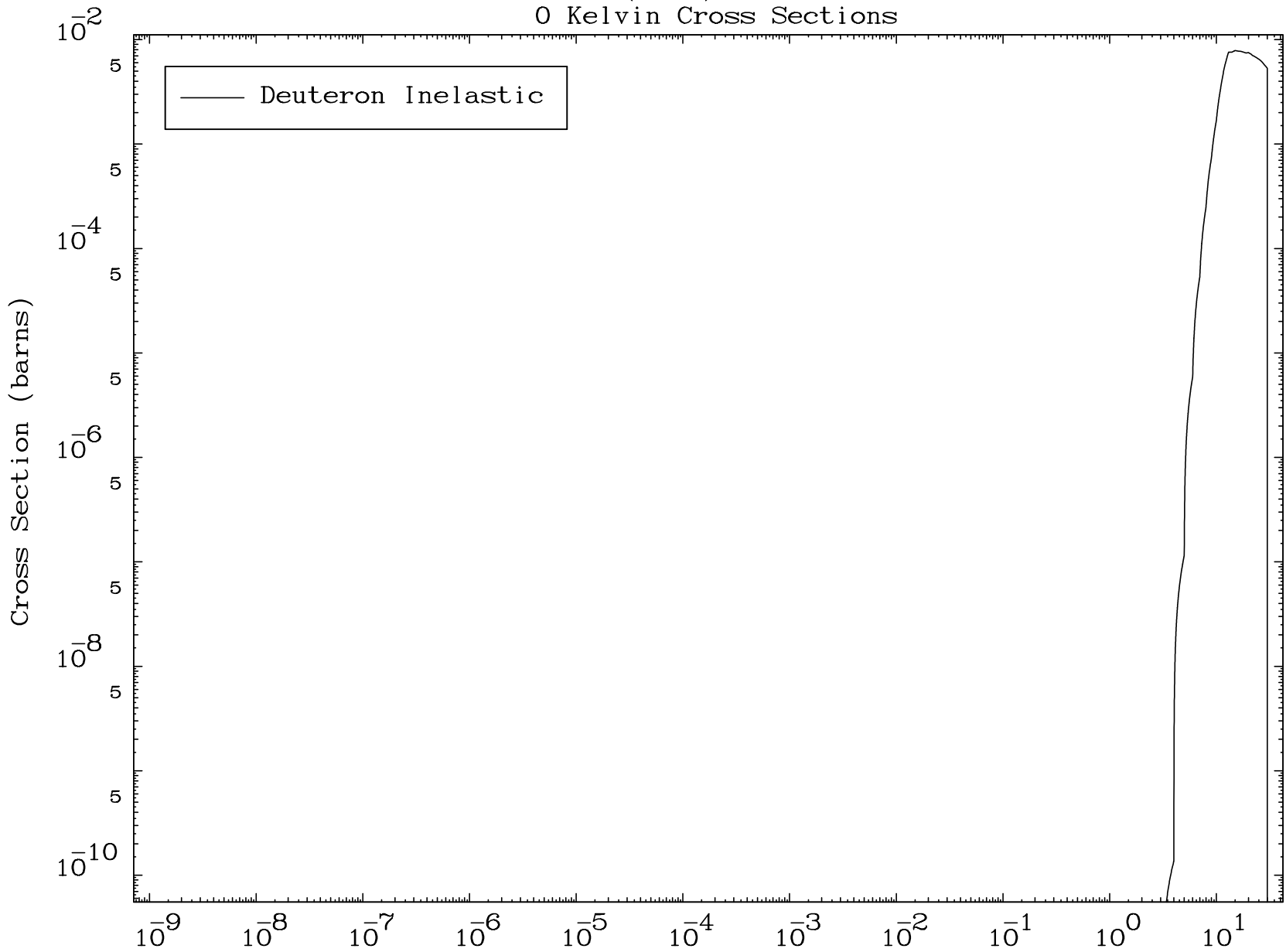


MAT 9990

(d,n') Level
0 Kelvin Cross Sections

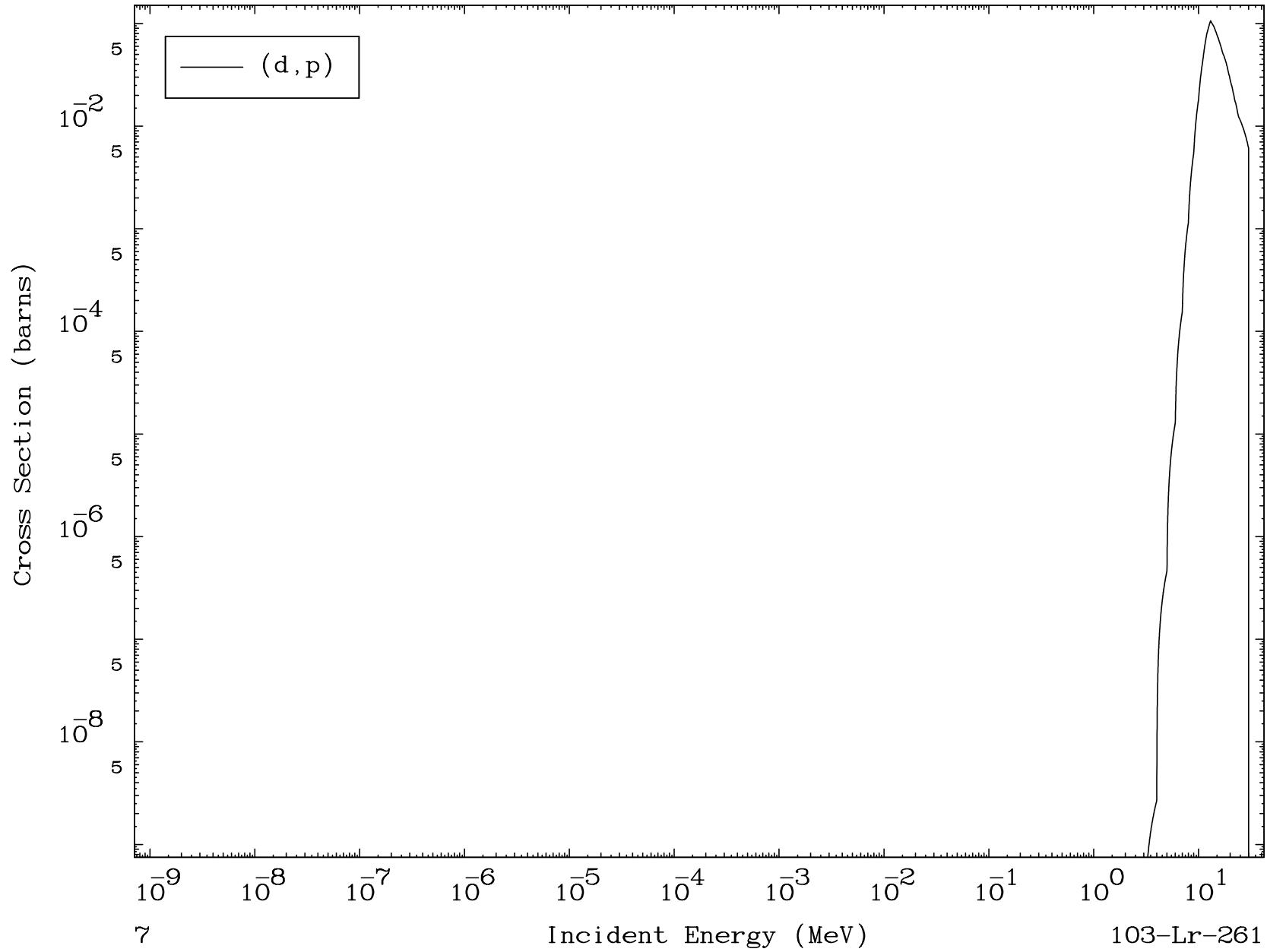
103-Lr-261

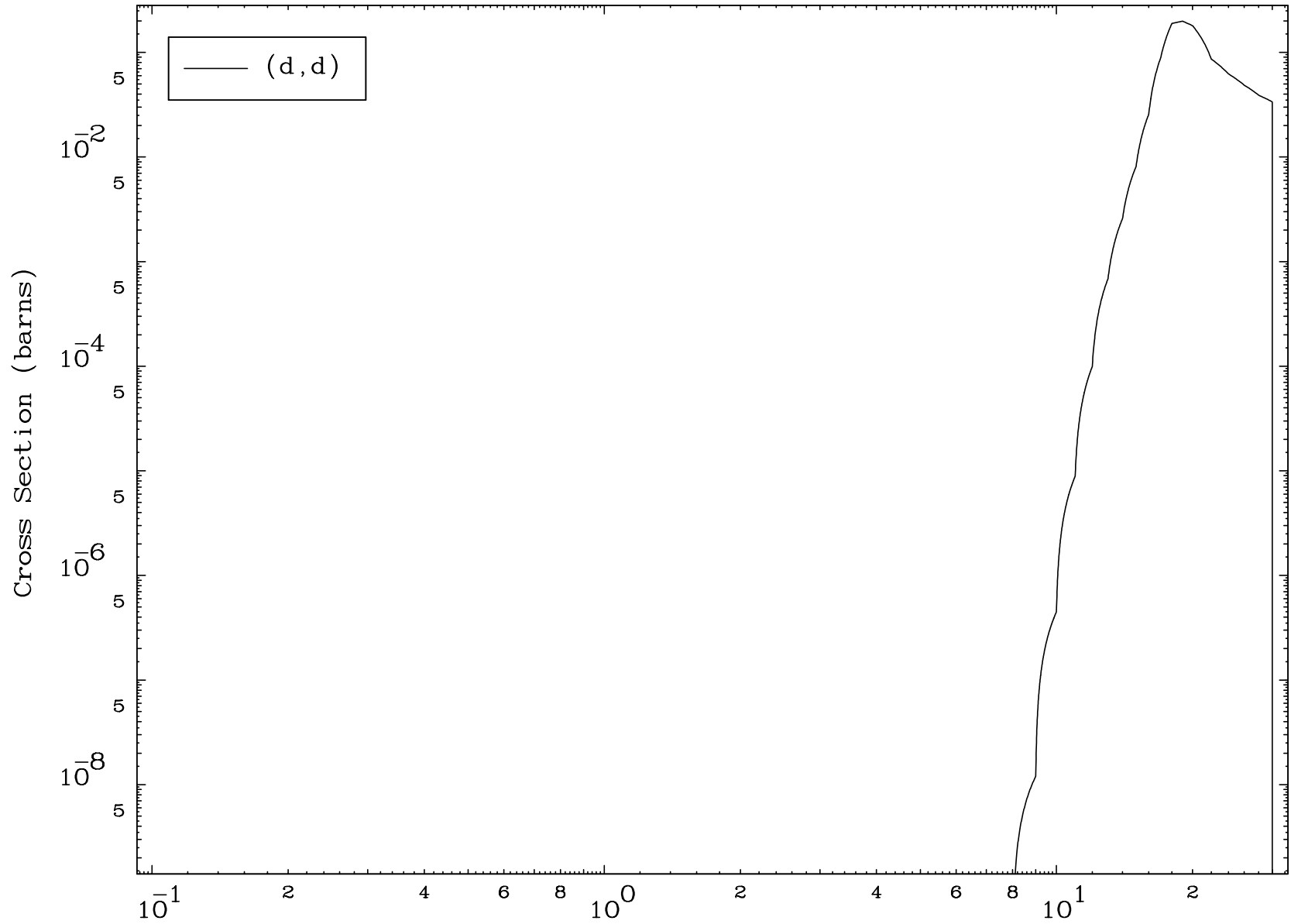


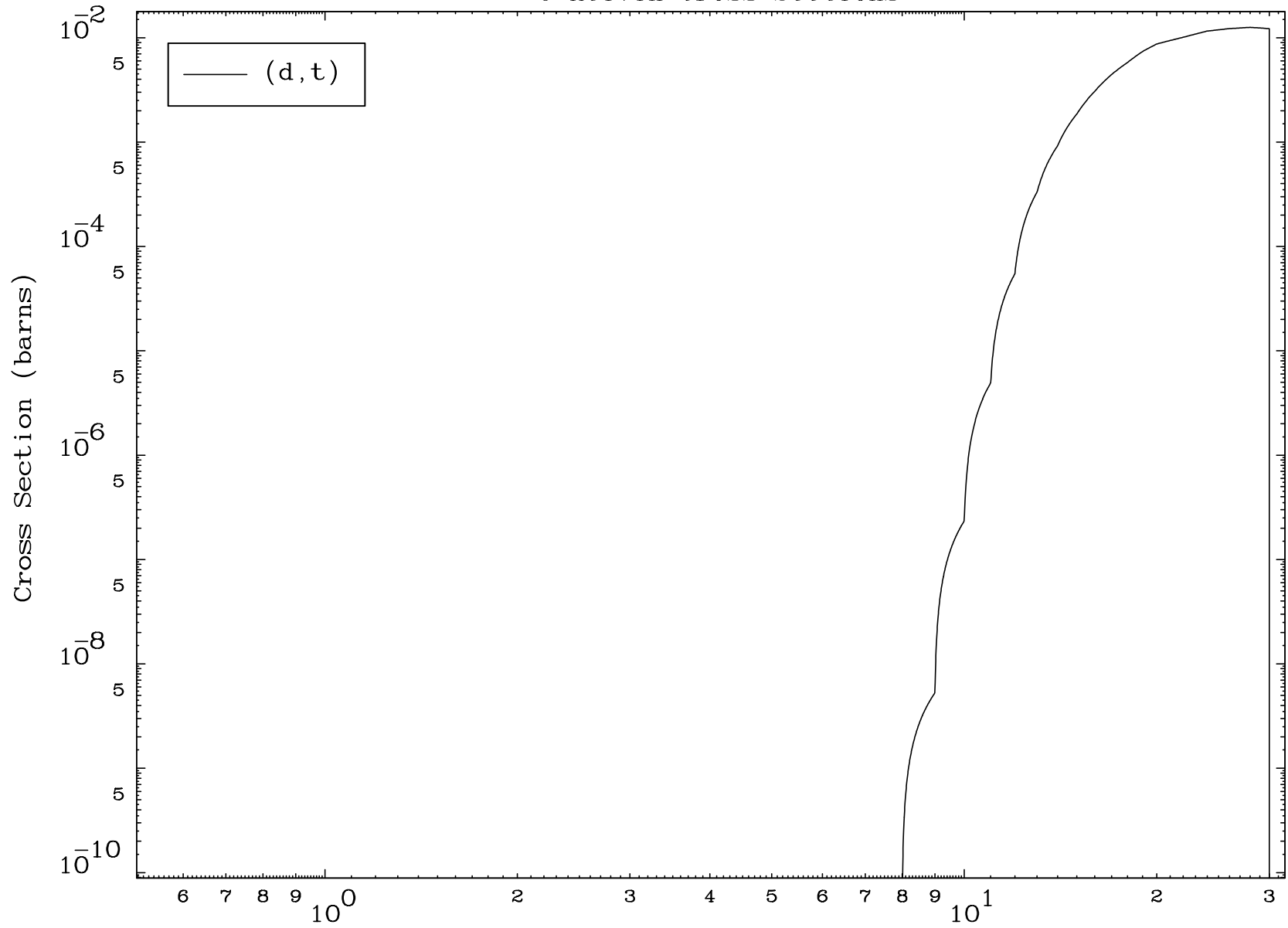
6

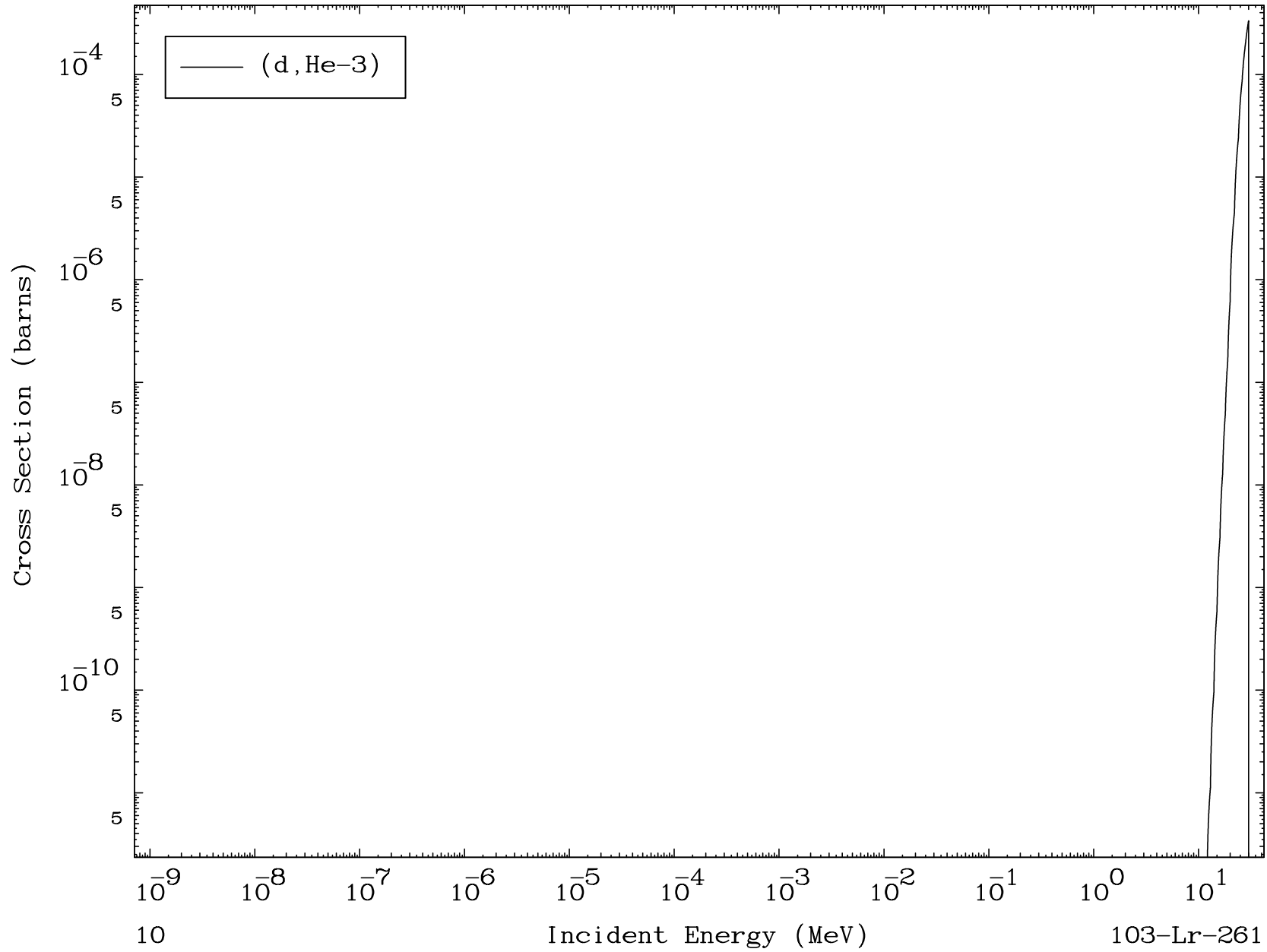
Incident Energy (MeV)

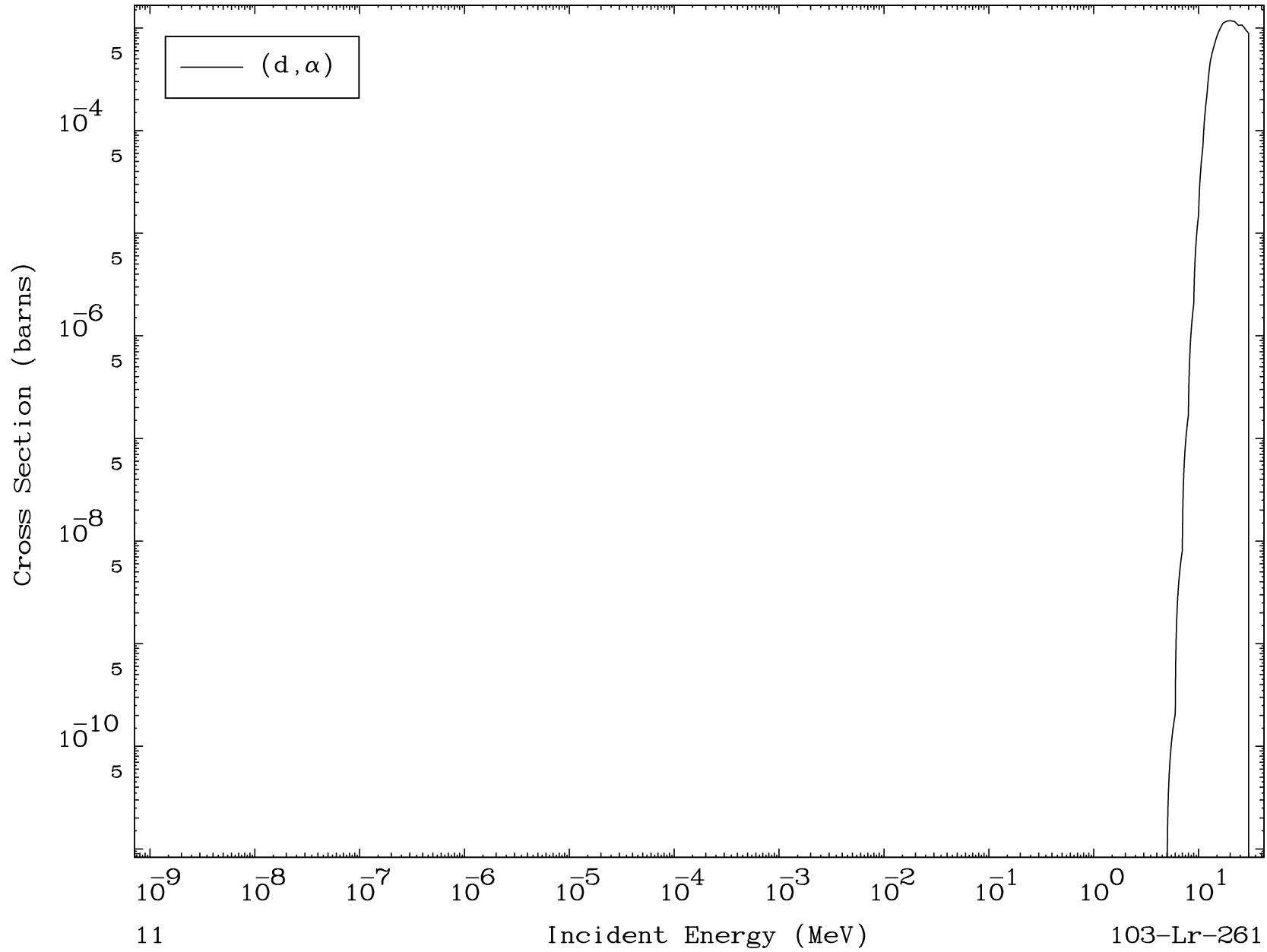
103-Lr-261



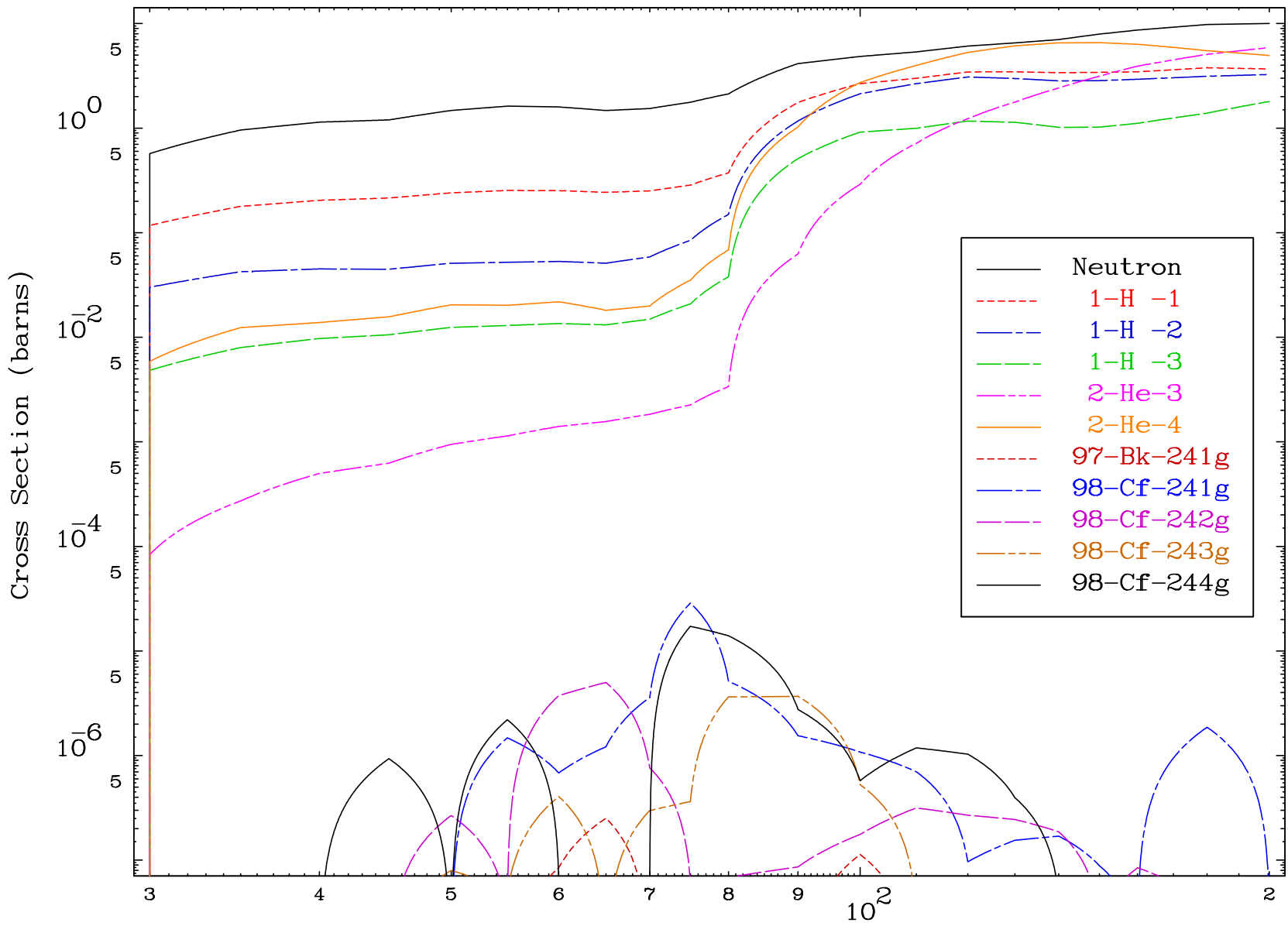




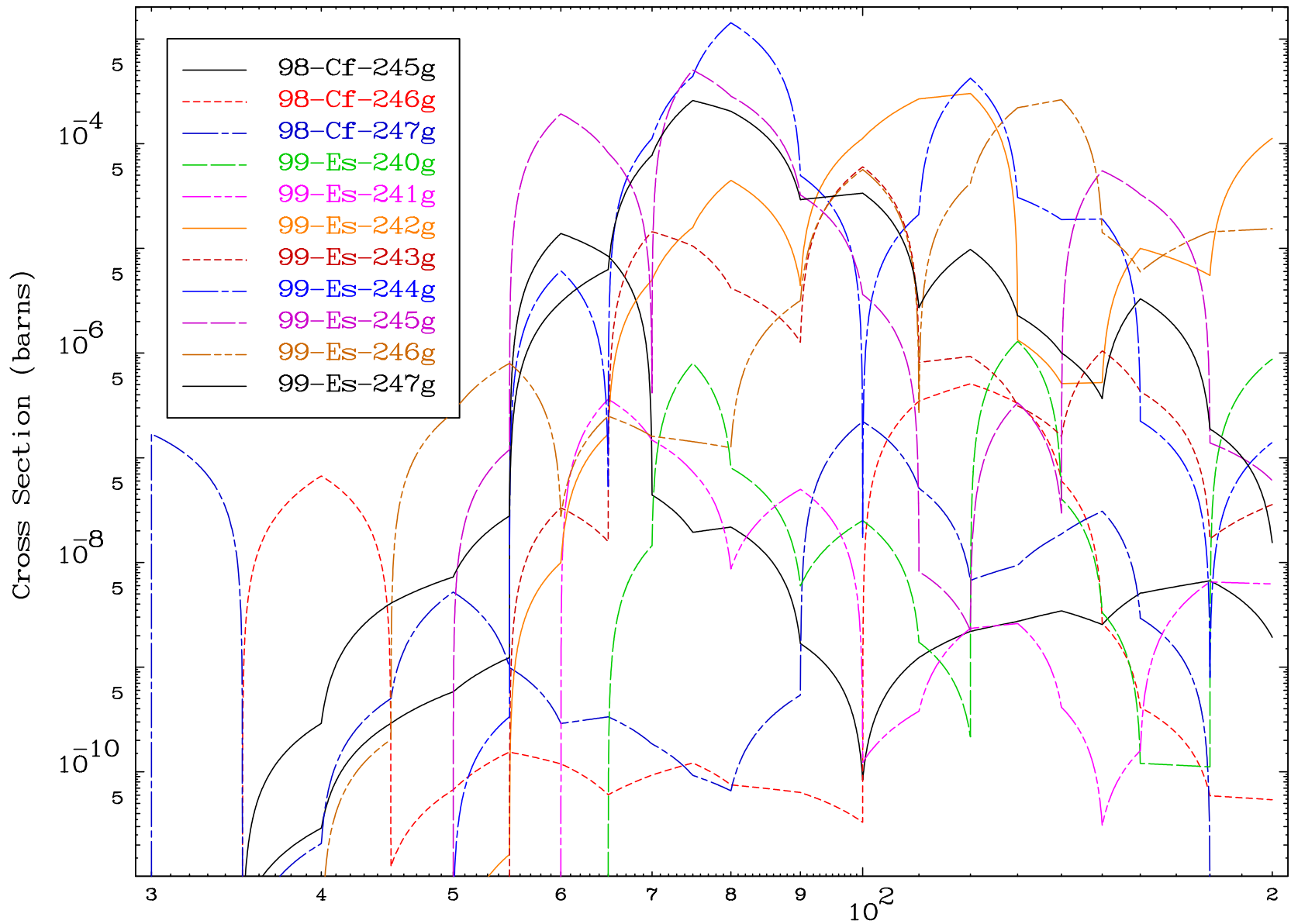




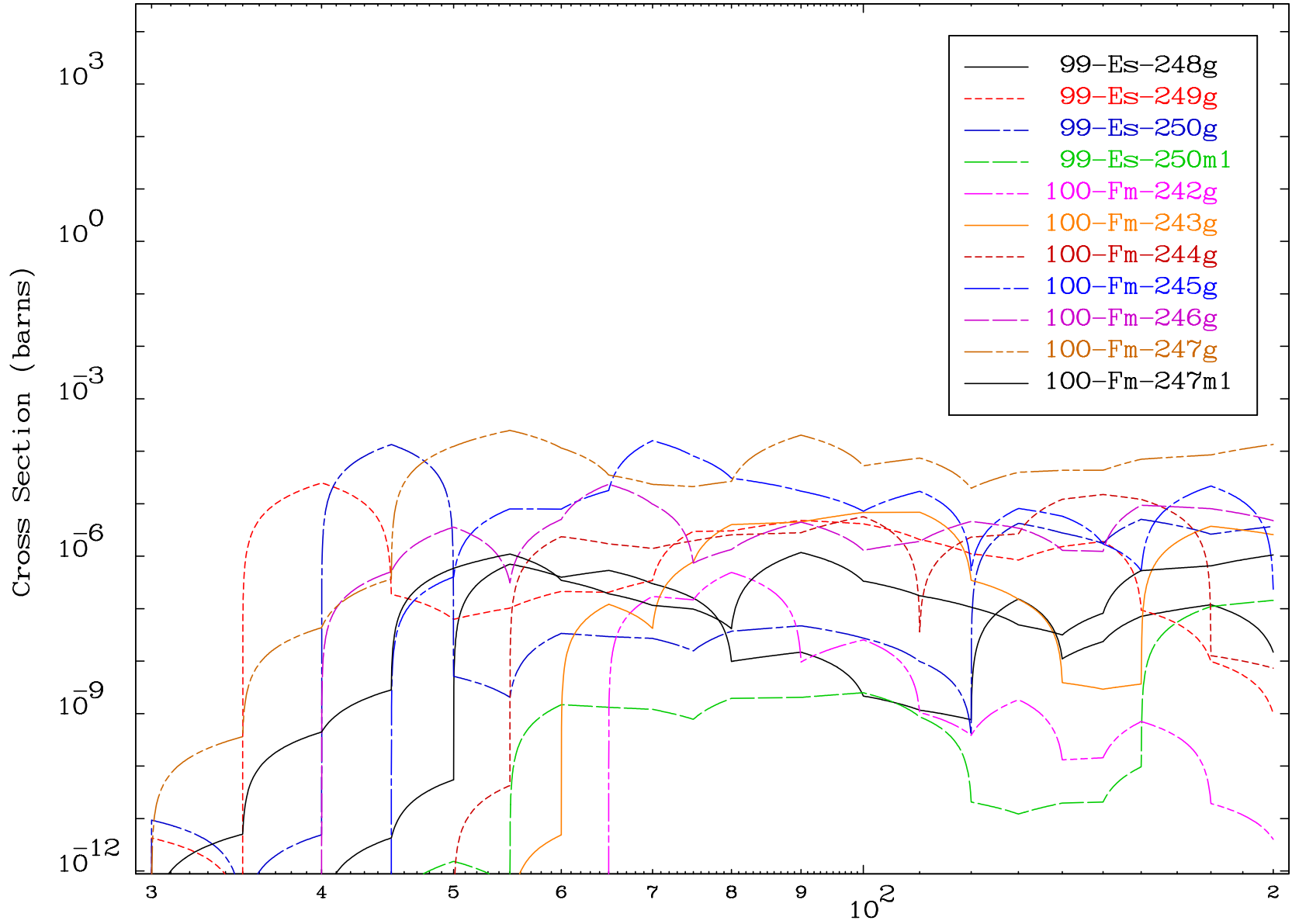
Radionuclide Production Cross Section



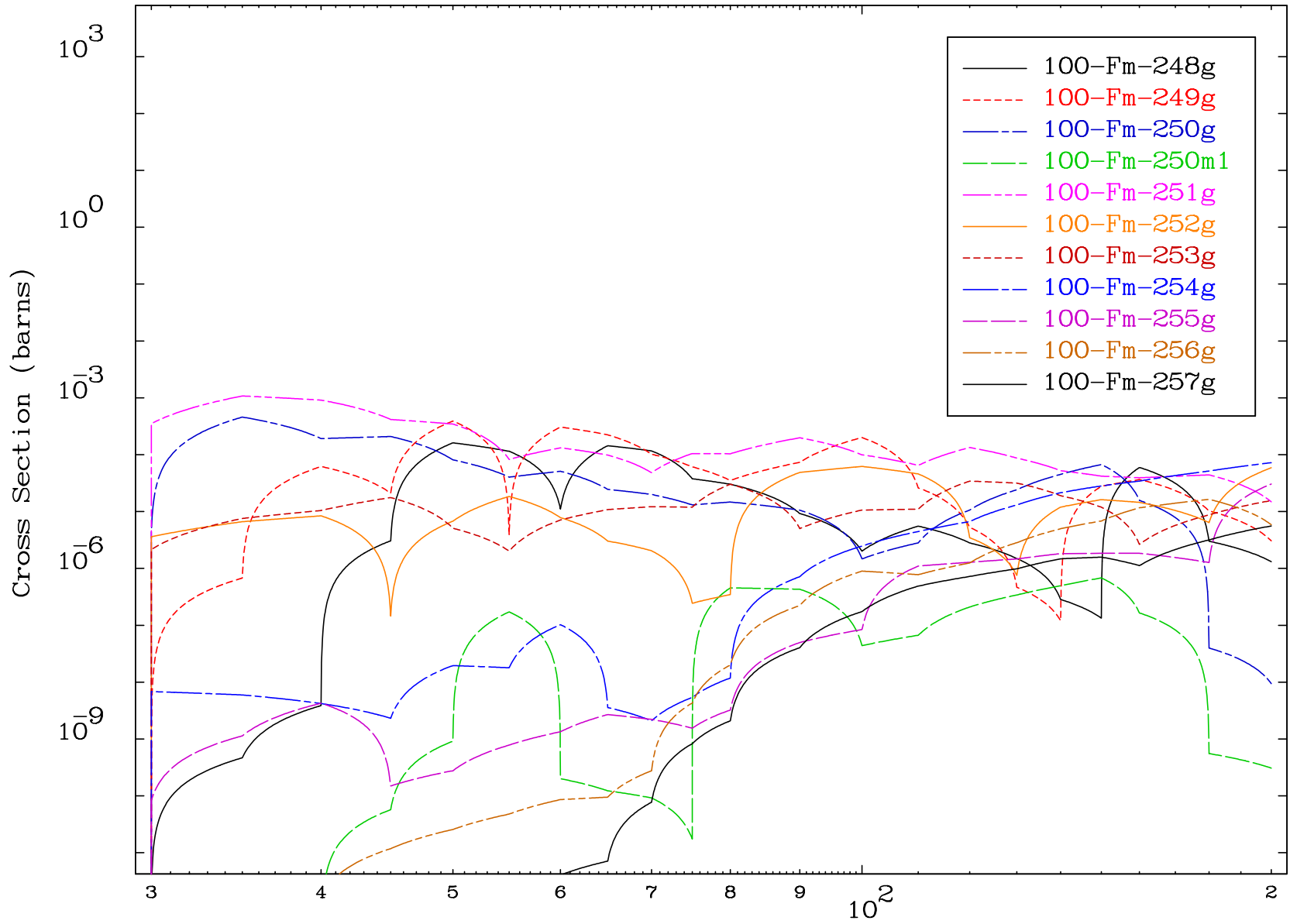
Radionuclide Production Cross Section



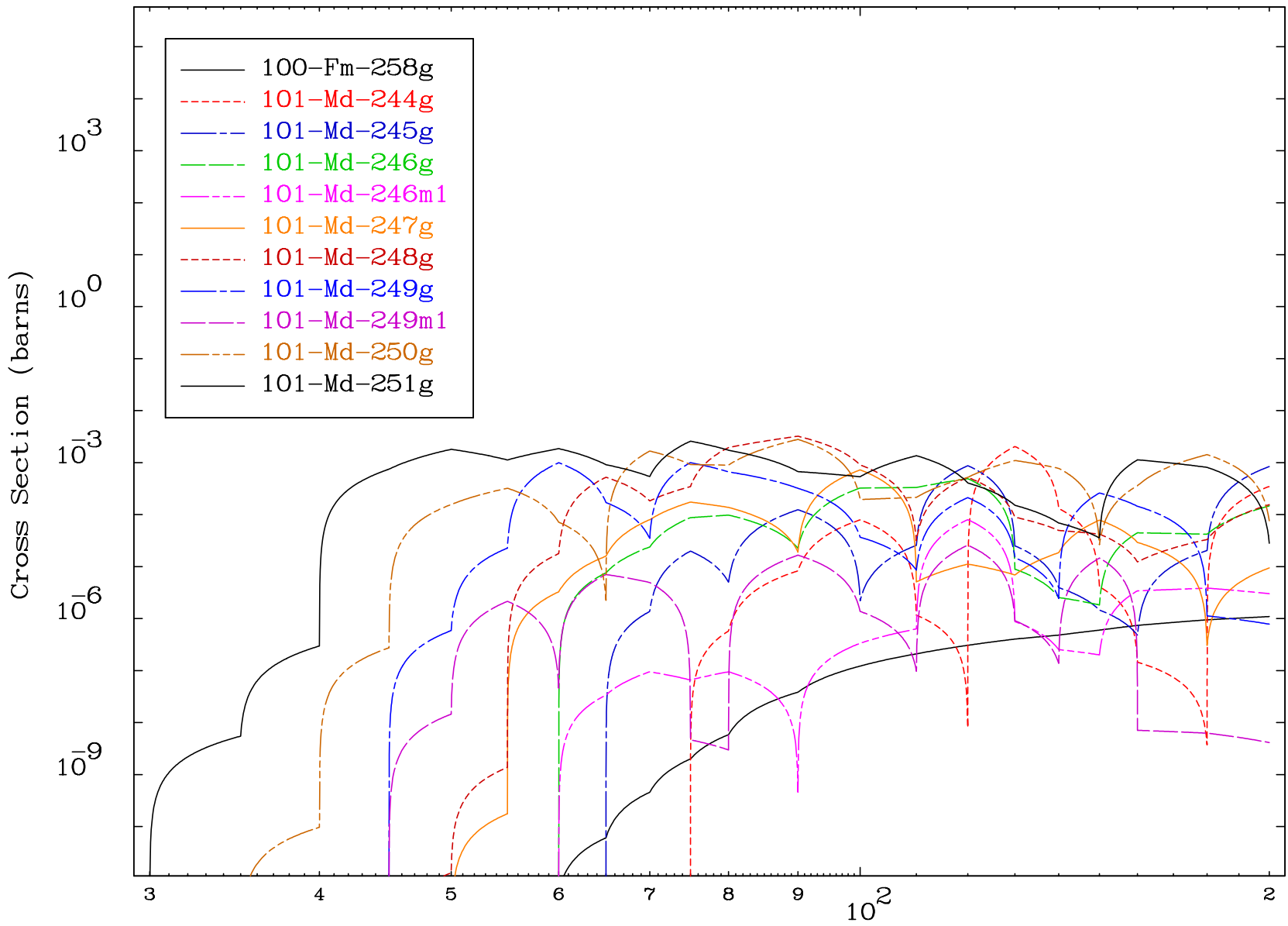
Radionuclide Production Cross Section



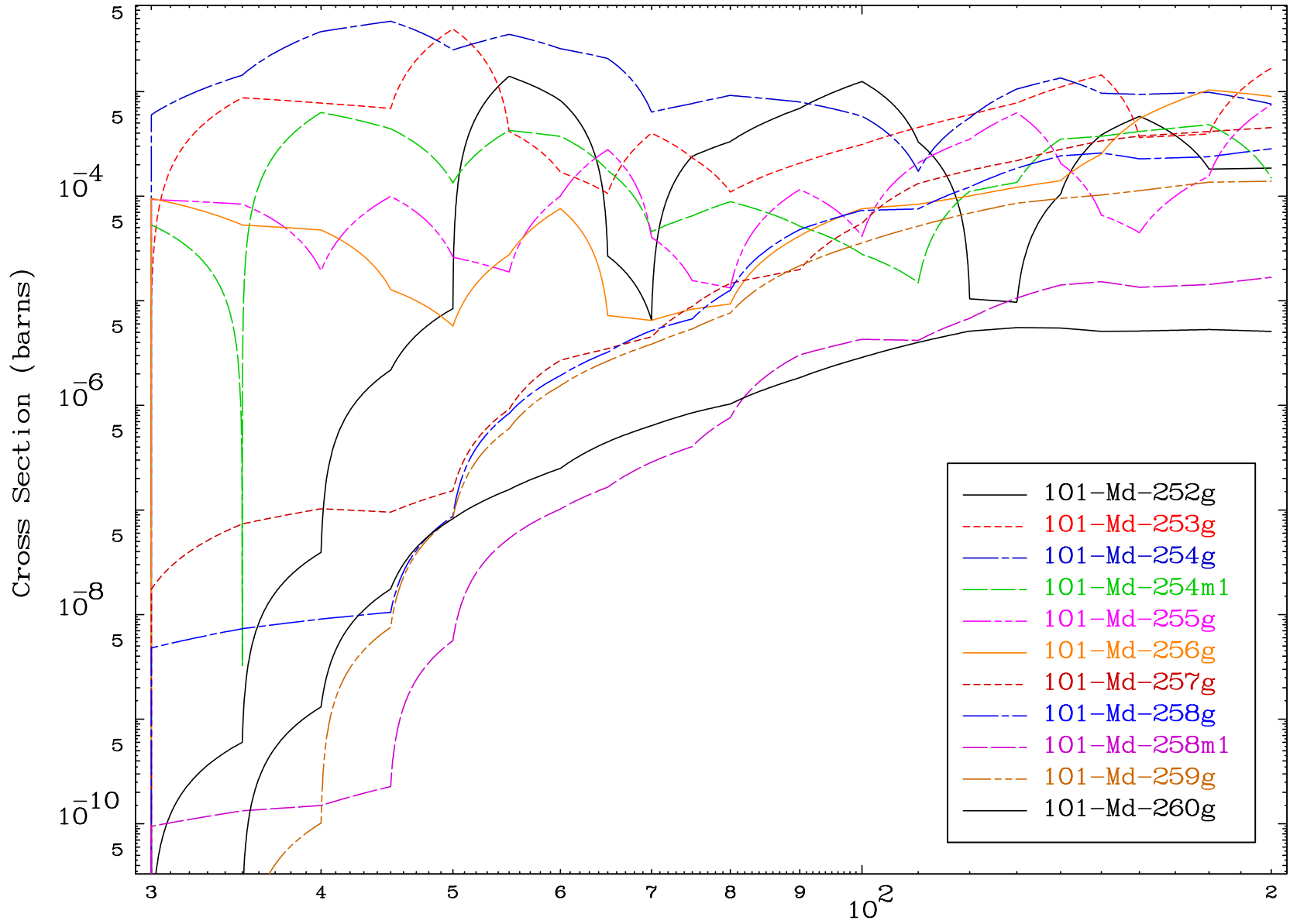
Radionuclide Production Cross Section



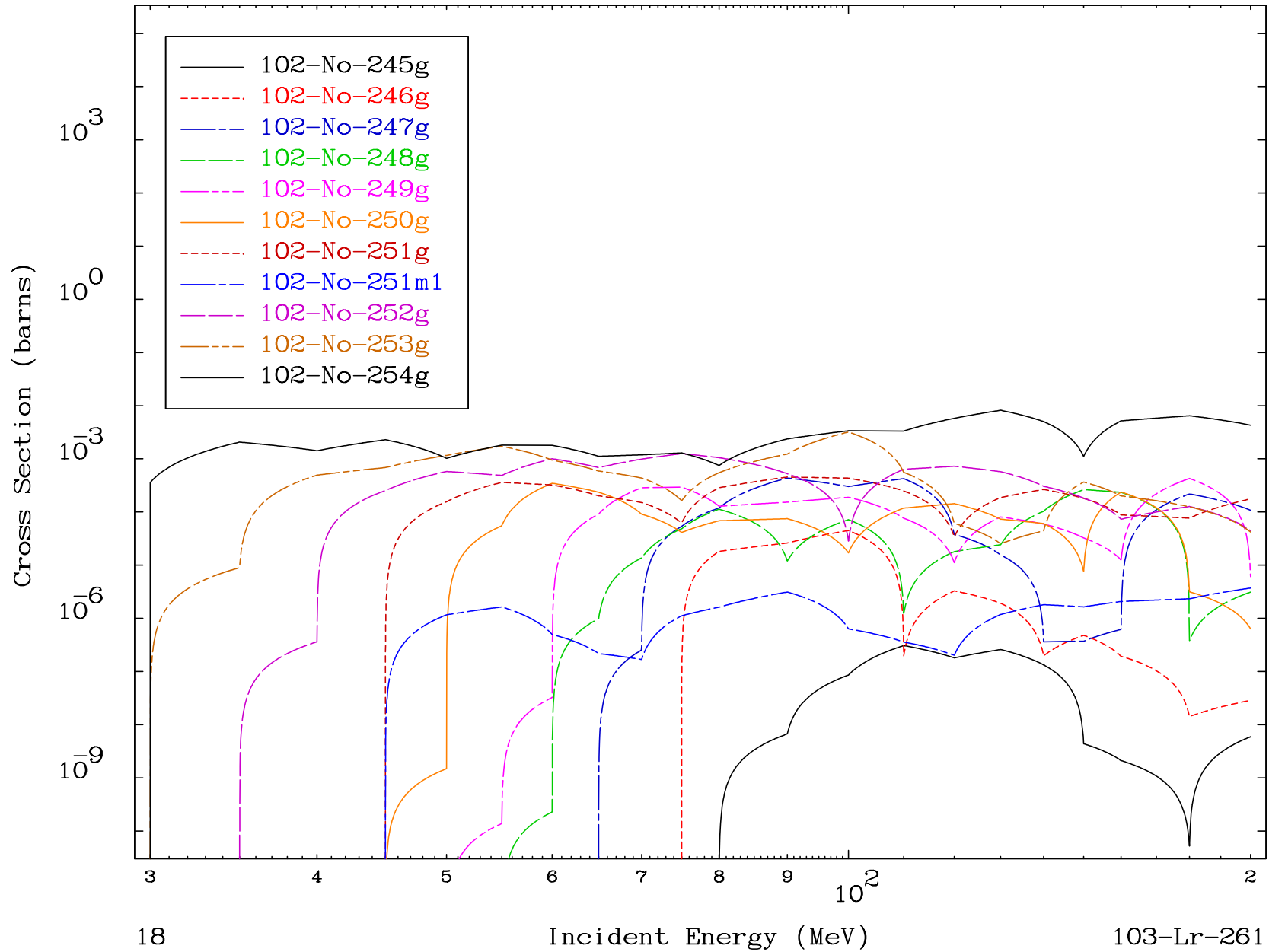
Radionuclide Production Cross Section



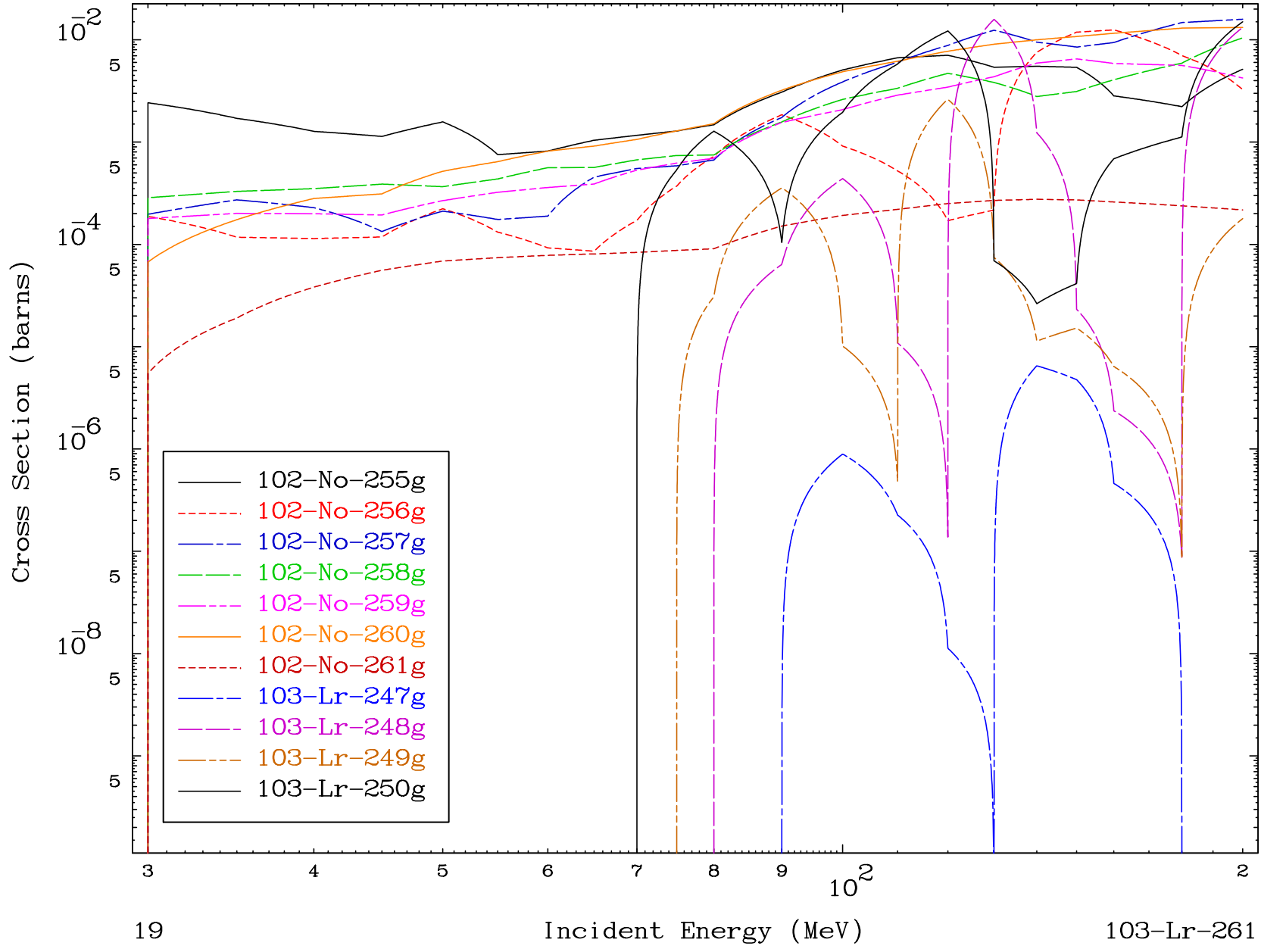
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

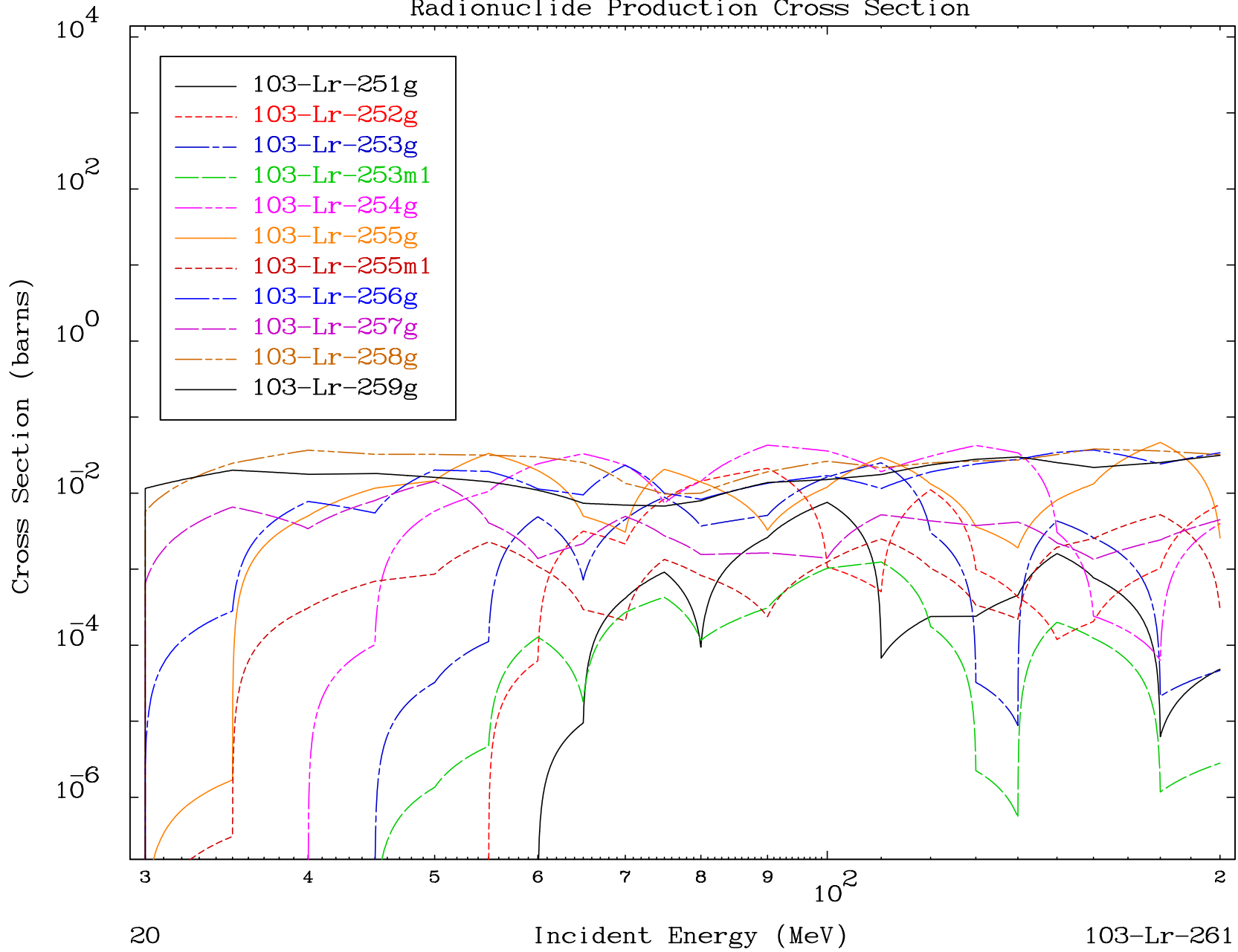


MAT 9990

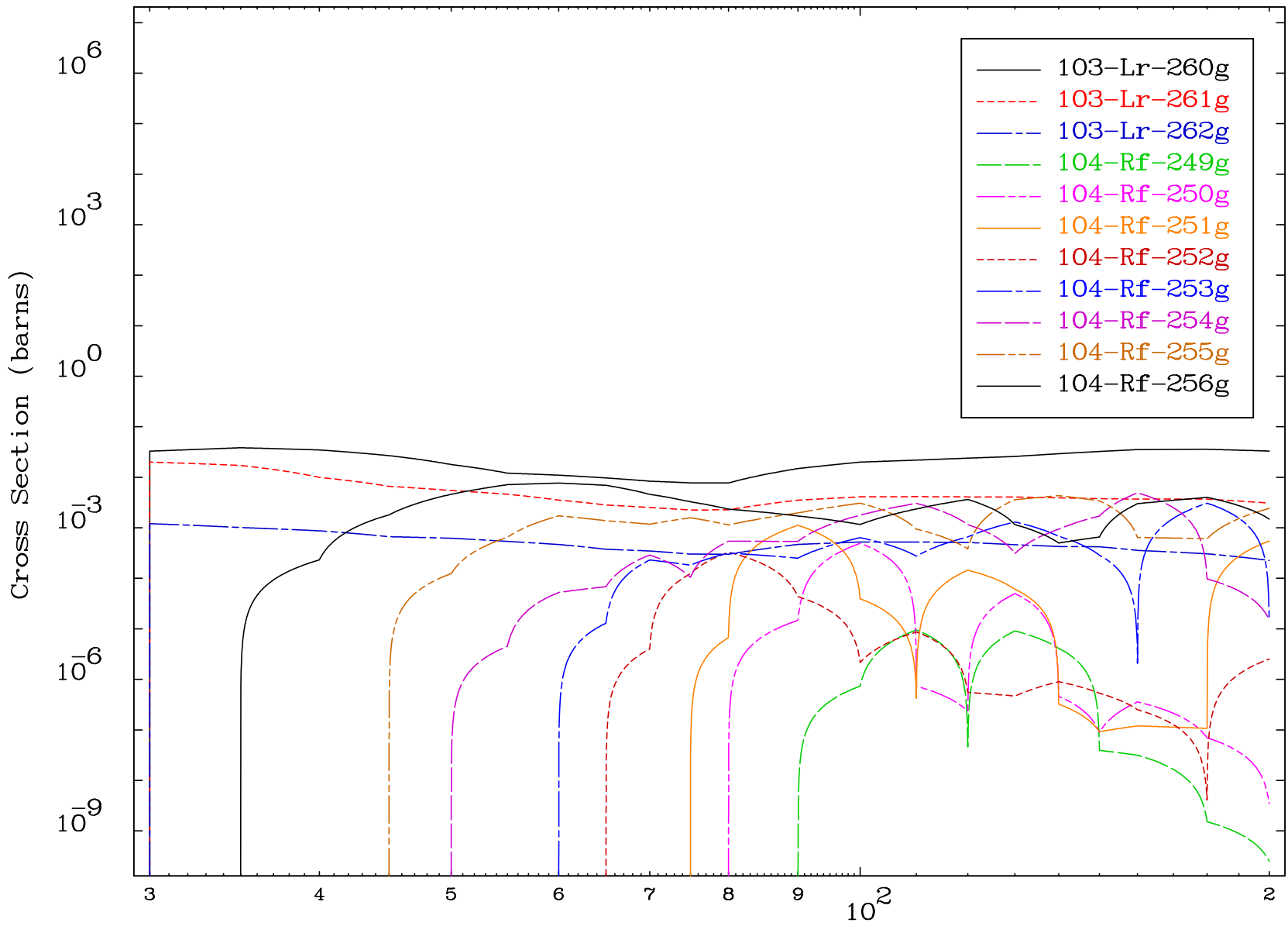
(d,remainder)

103-Lr-261

Radionuclide Production Cross Section



Radionuclide Production Cross Section

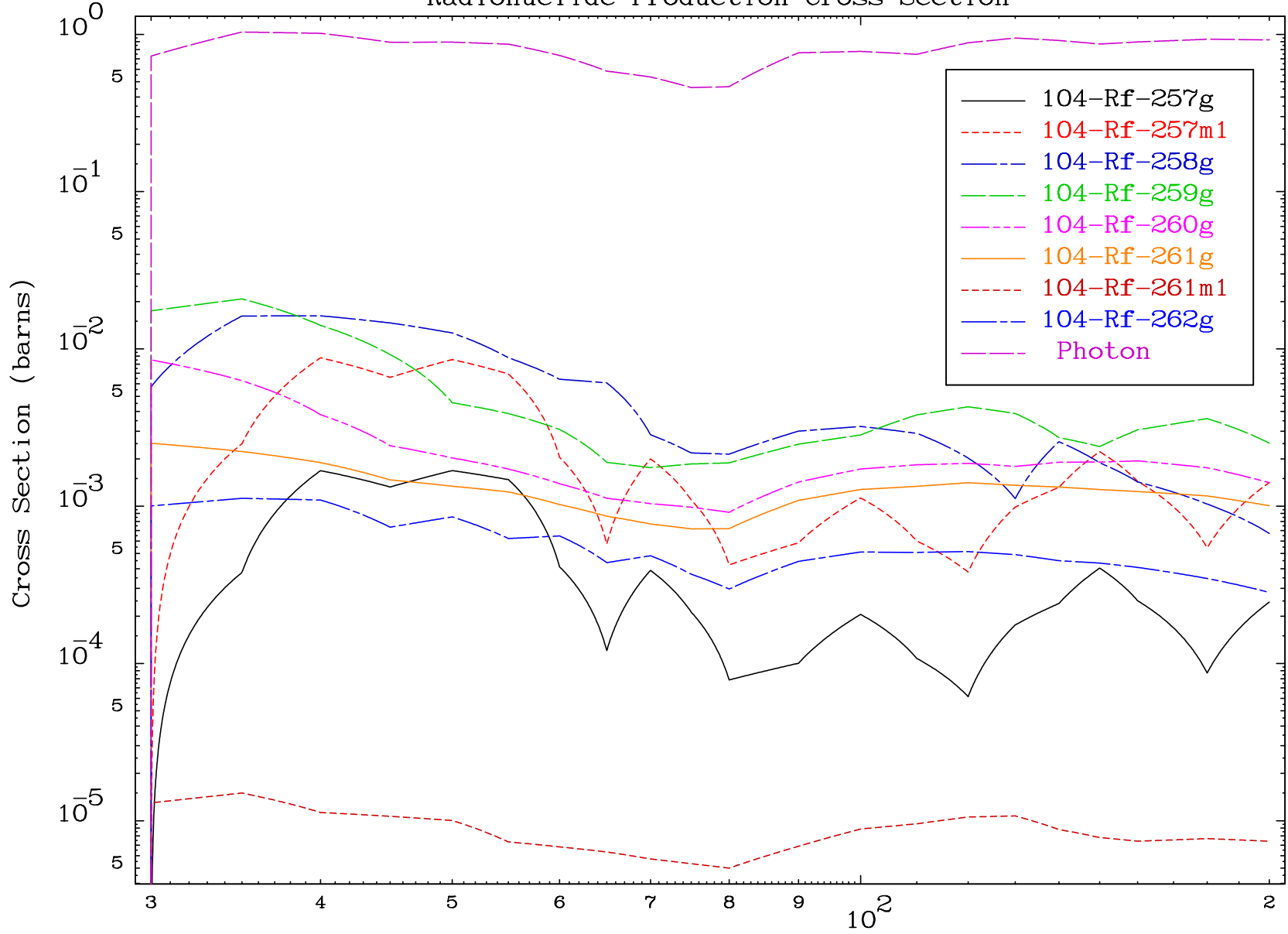


MAT 9990

(d,remainder)

103-Lr-261

Radionuclide Production Cross Section



22

Incident Energy (MeV)

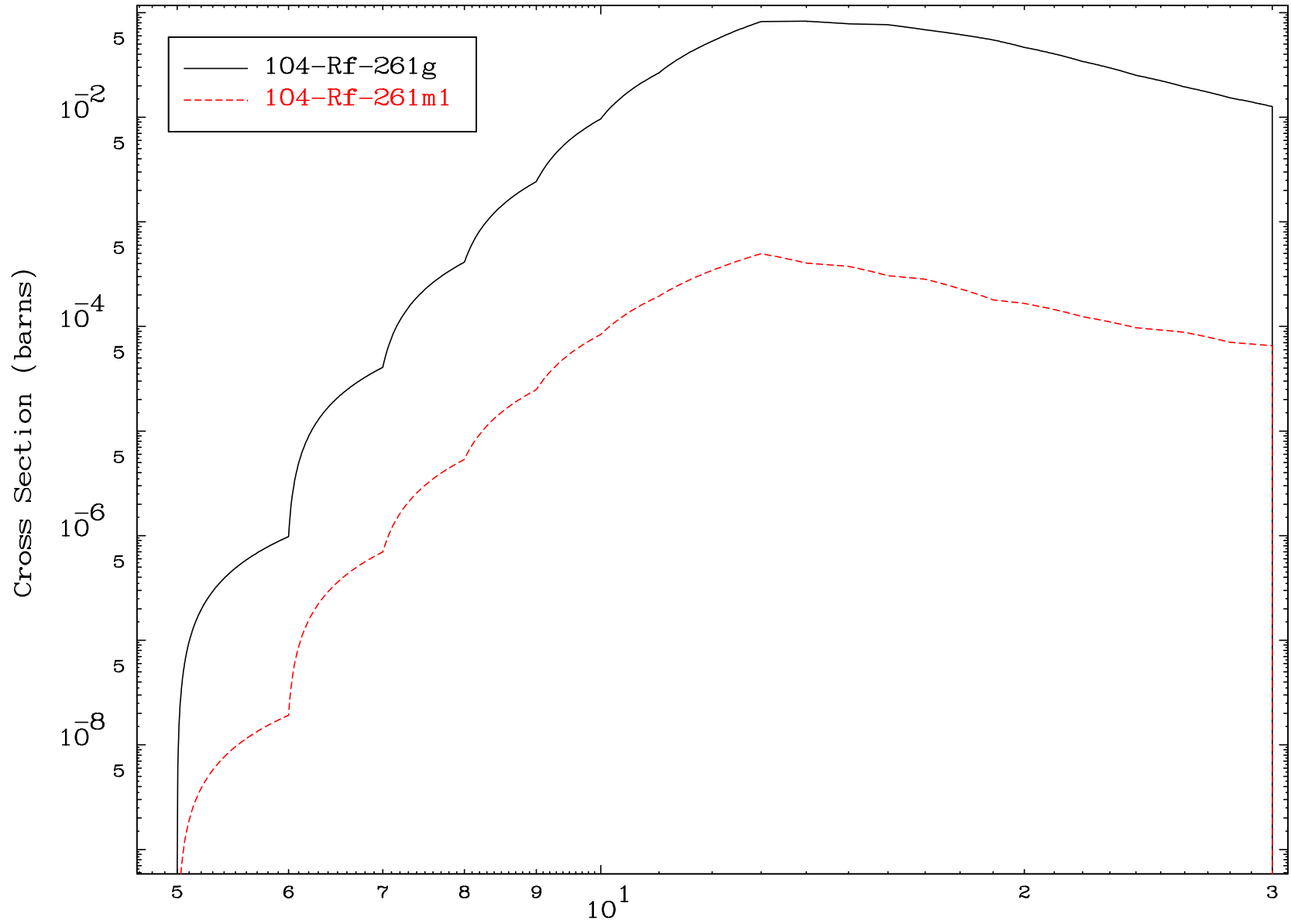
103-Lr-261

MAT 9990

(d,2n)

103-Lr-261

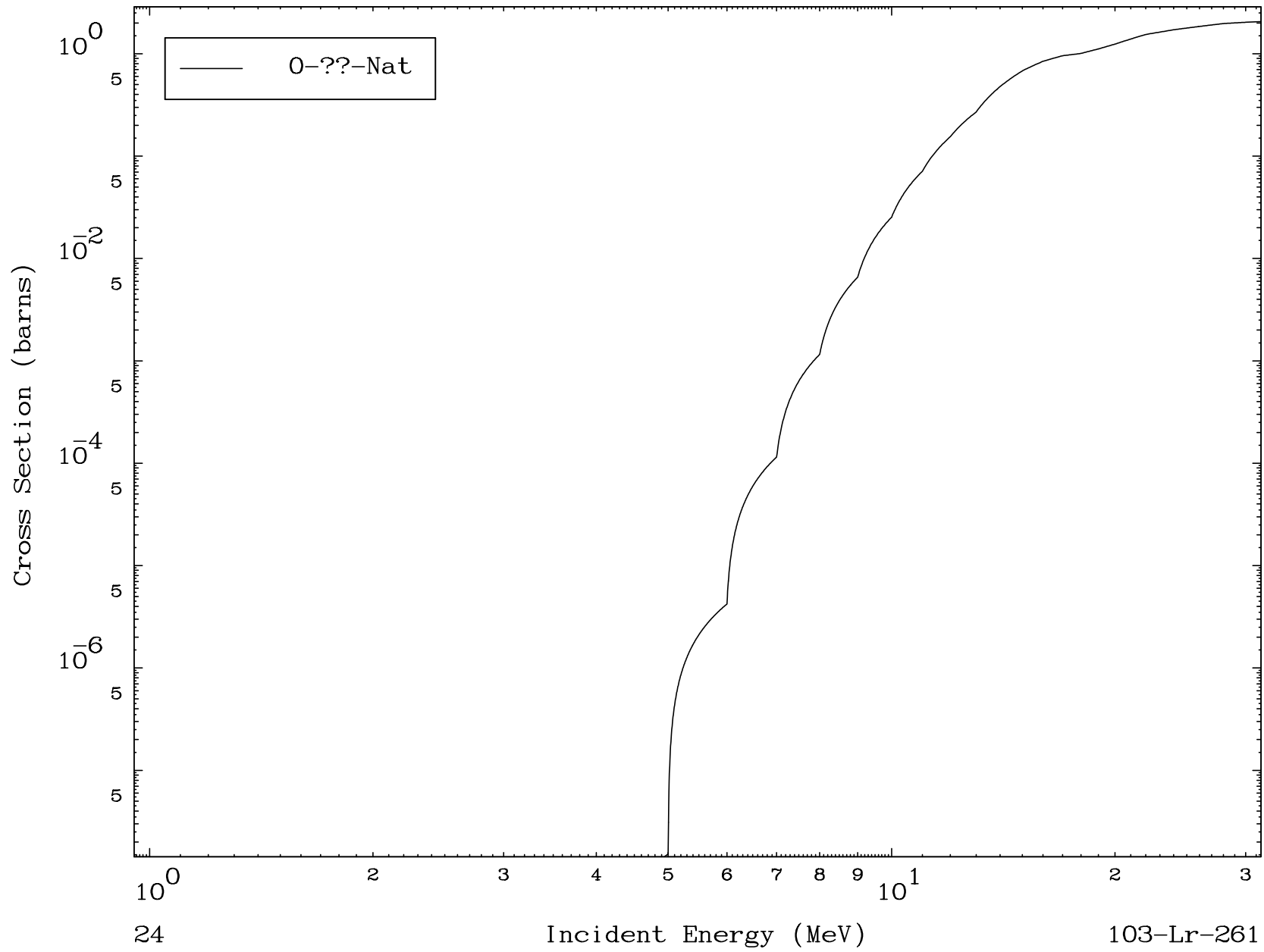
Radionuclide Production Cross Section



23

Incident Energy (MeV)

103-Lr-261



MAT 9990

(d,p) α

103-Lr-261

Radionuclide Production Cross Section

