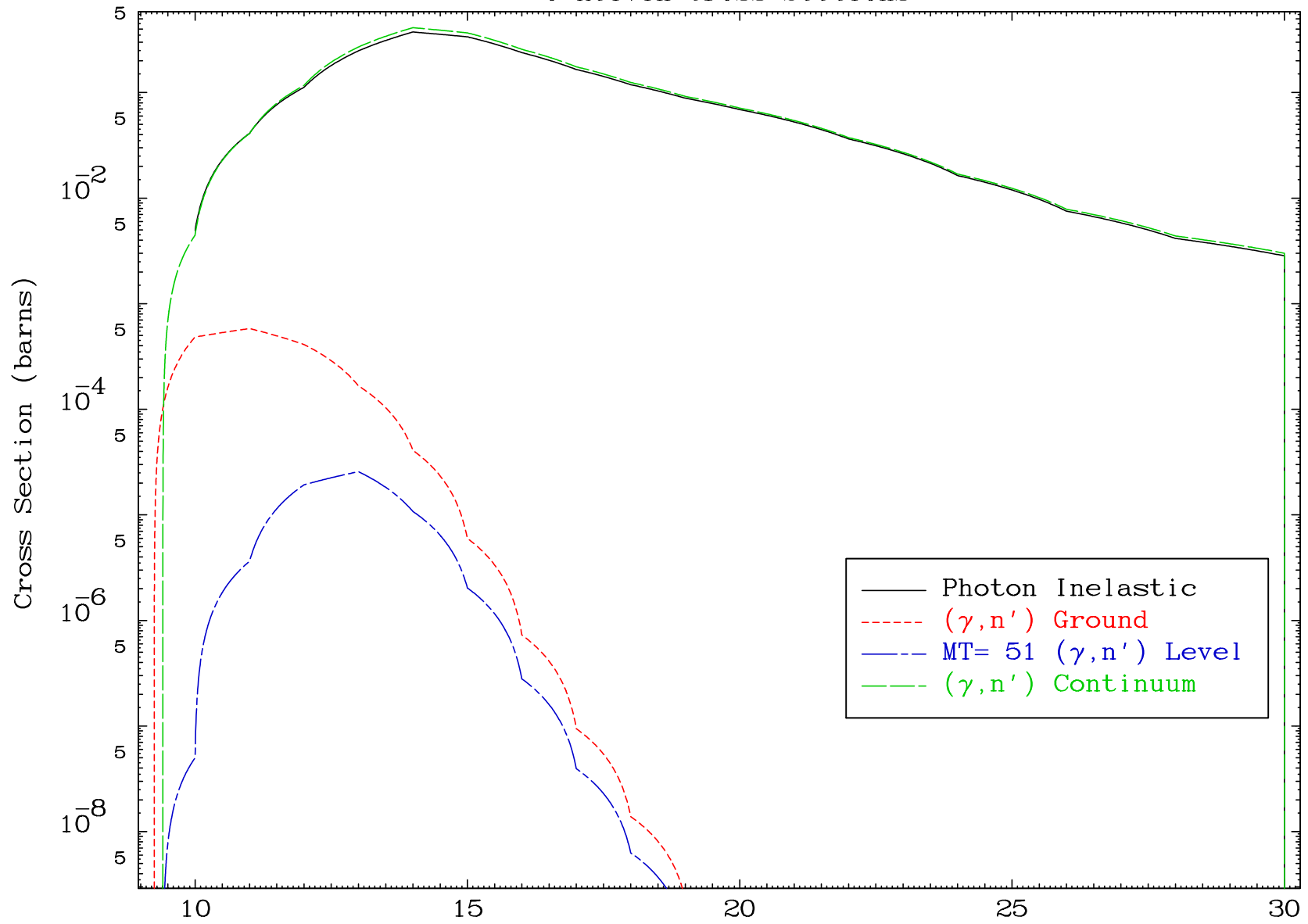


MAT 8290

$(\gamma, n')$  Level  
0 Kelvin Cross Sections

83-Bi-197



5

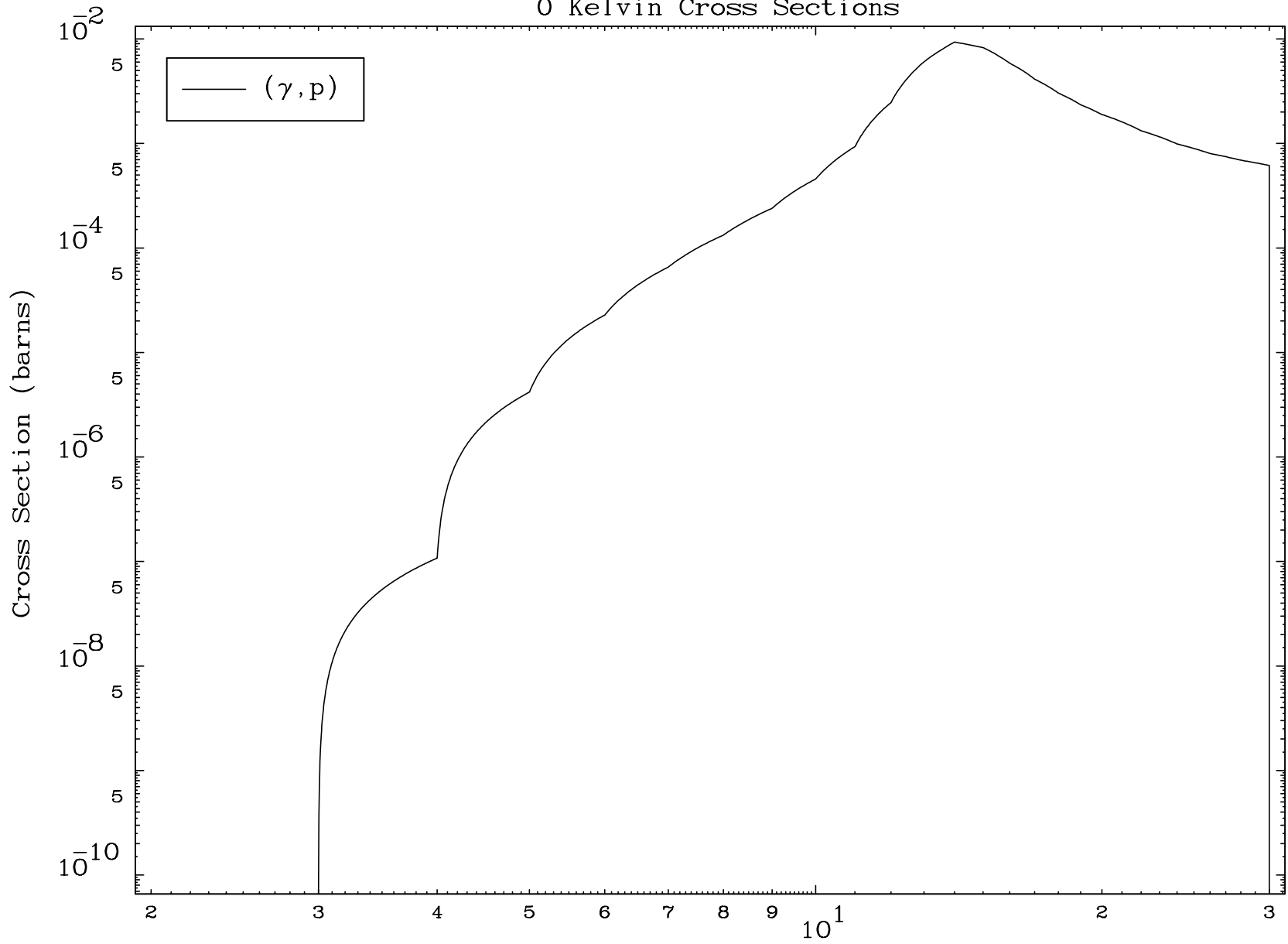
Incident Energy (MeV)

83-Bi-197

MAT 8290

( $\gamma$ ,p) Levels  
0 Kelvin Cross Sections

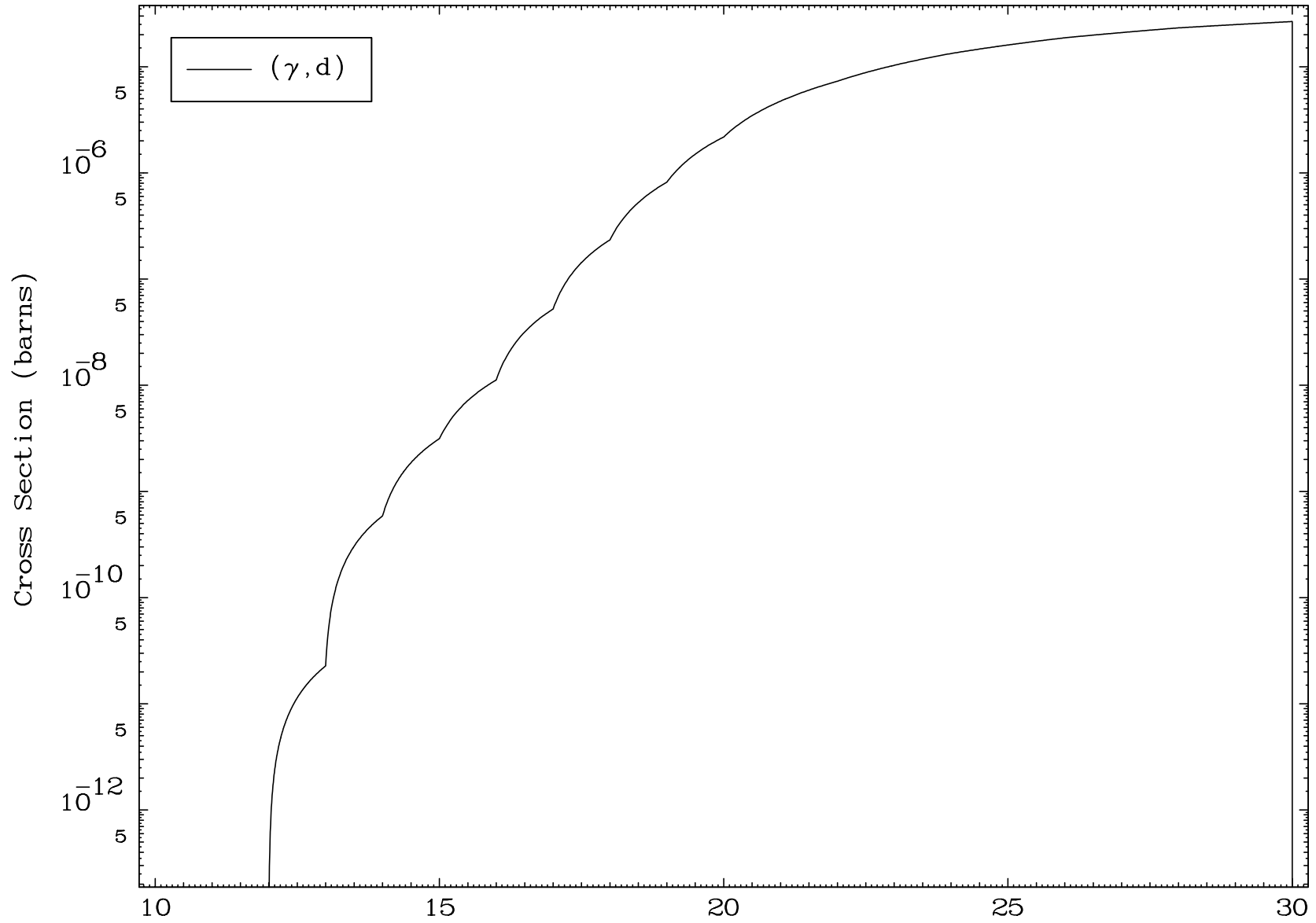
83-Bi-197

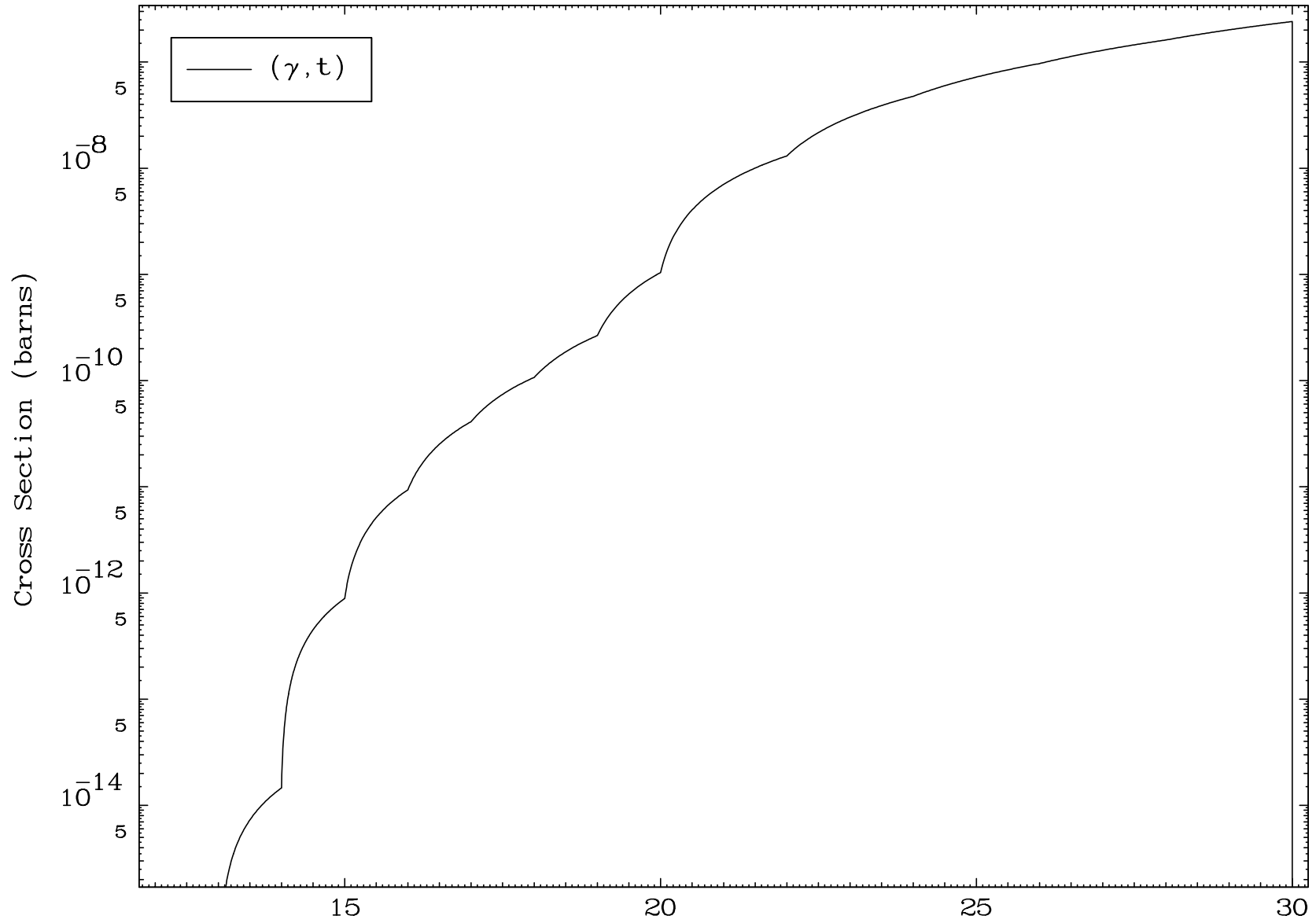


6

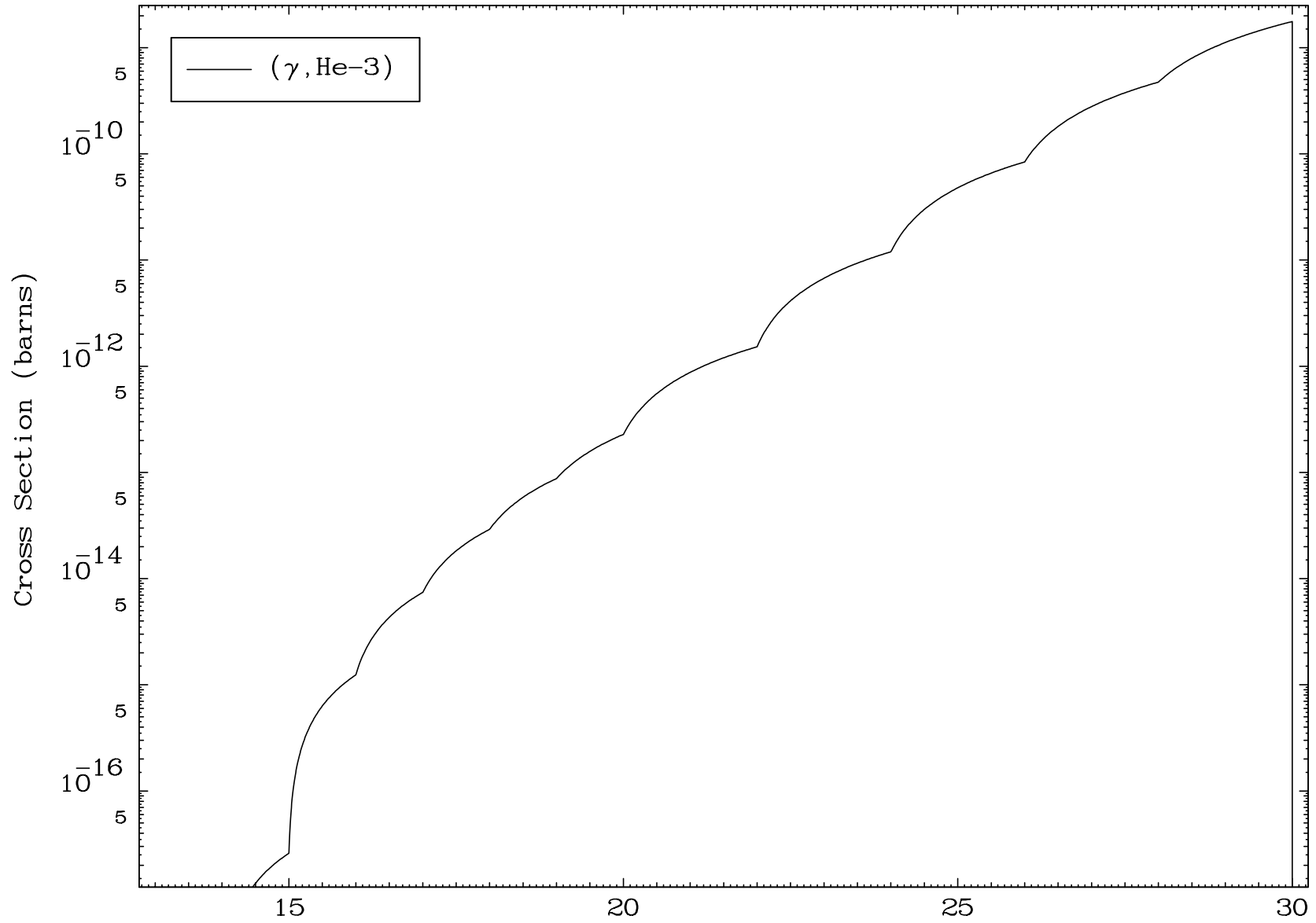
Incident Energy (MeV)

83-Bi-197





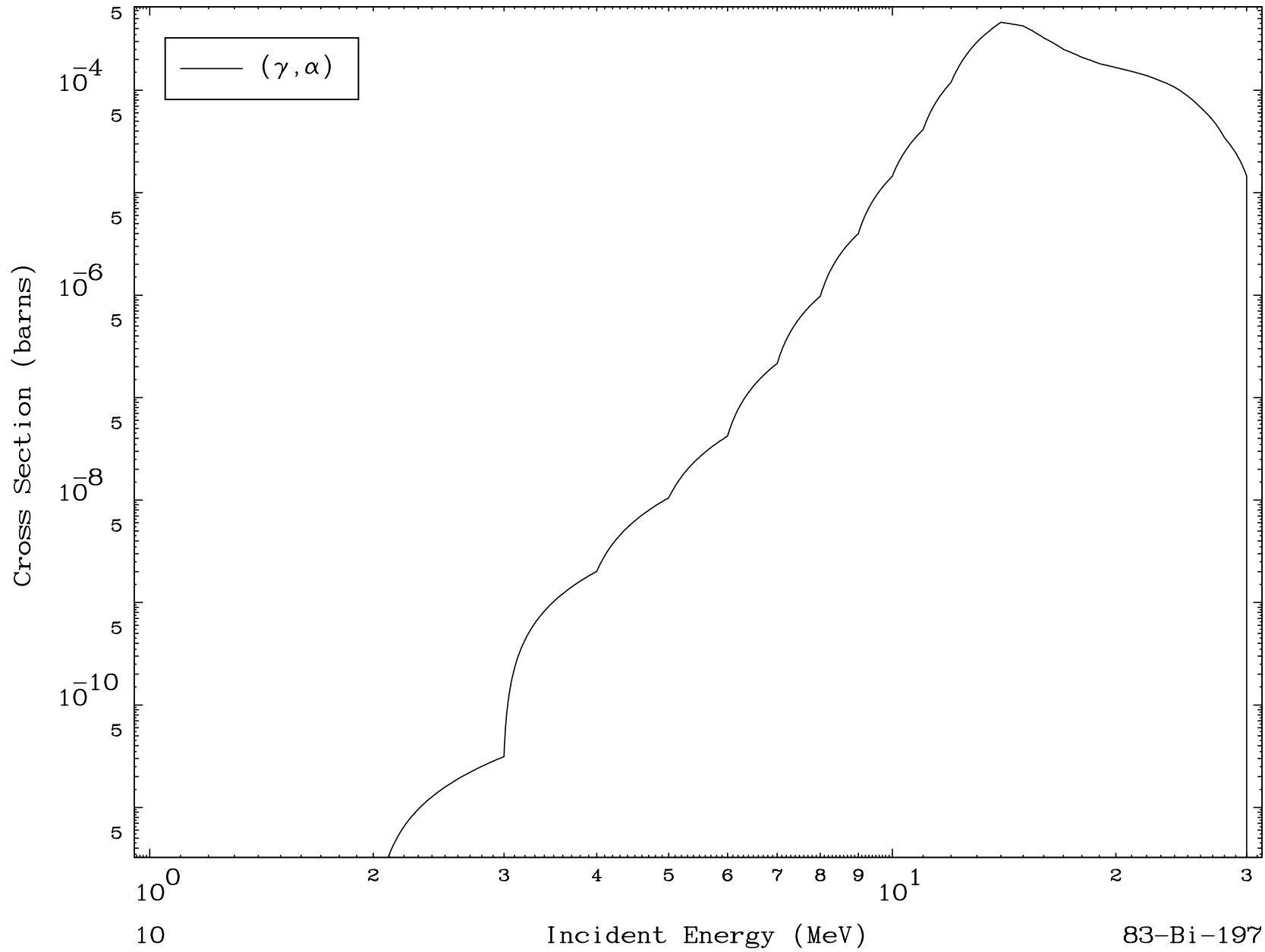


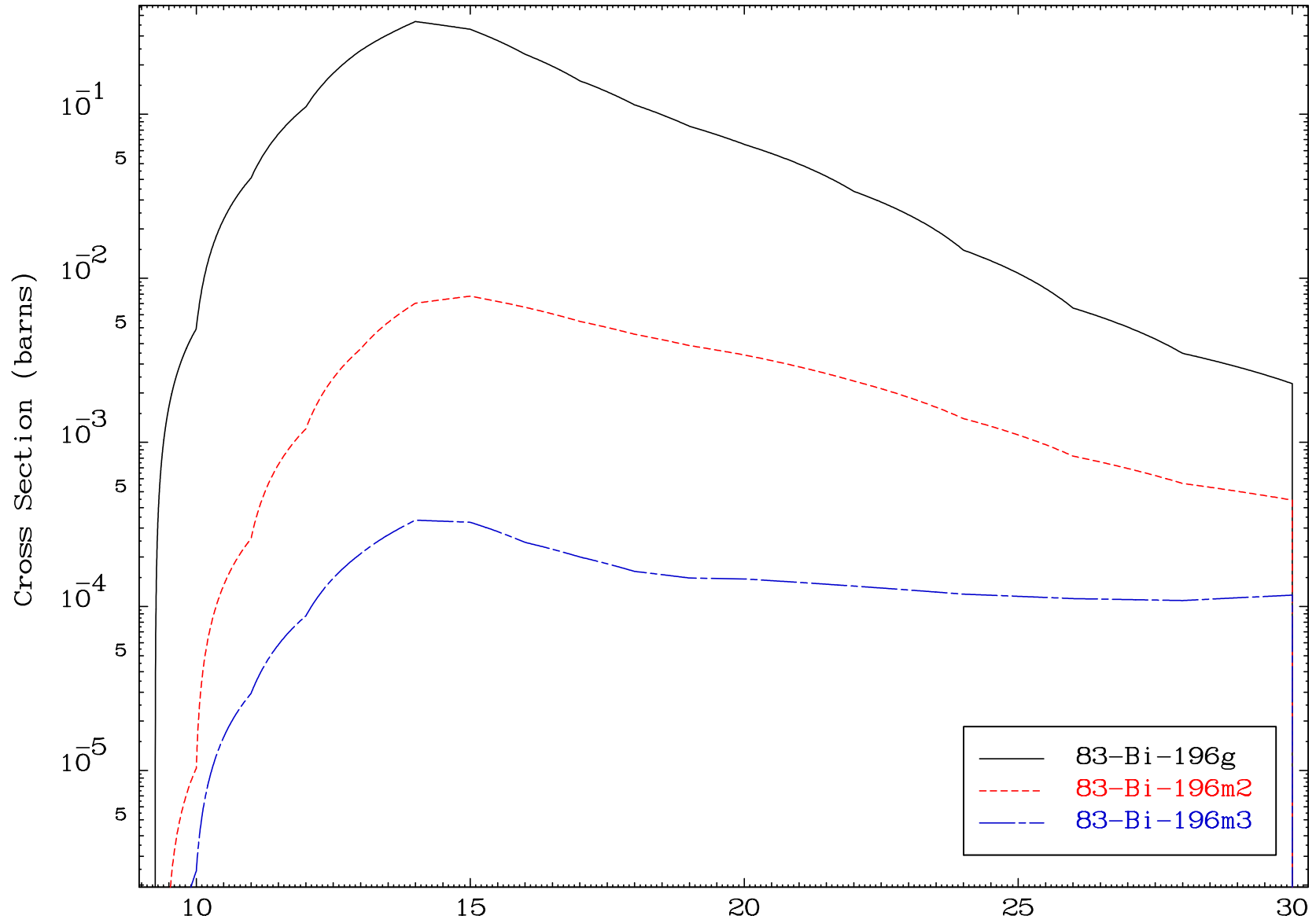


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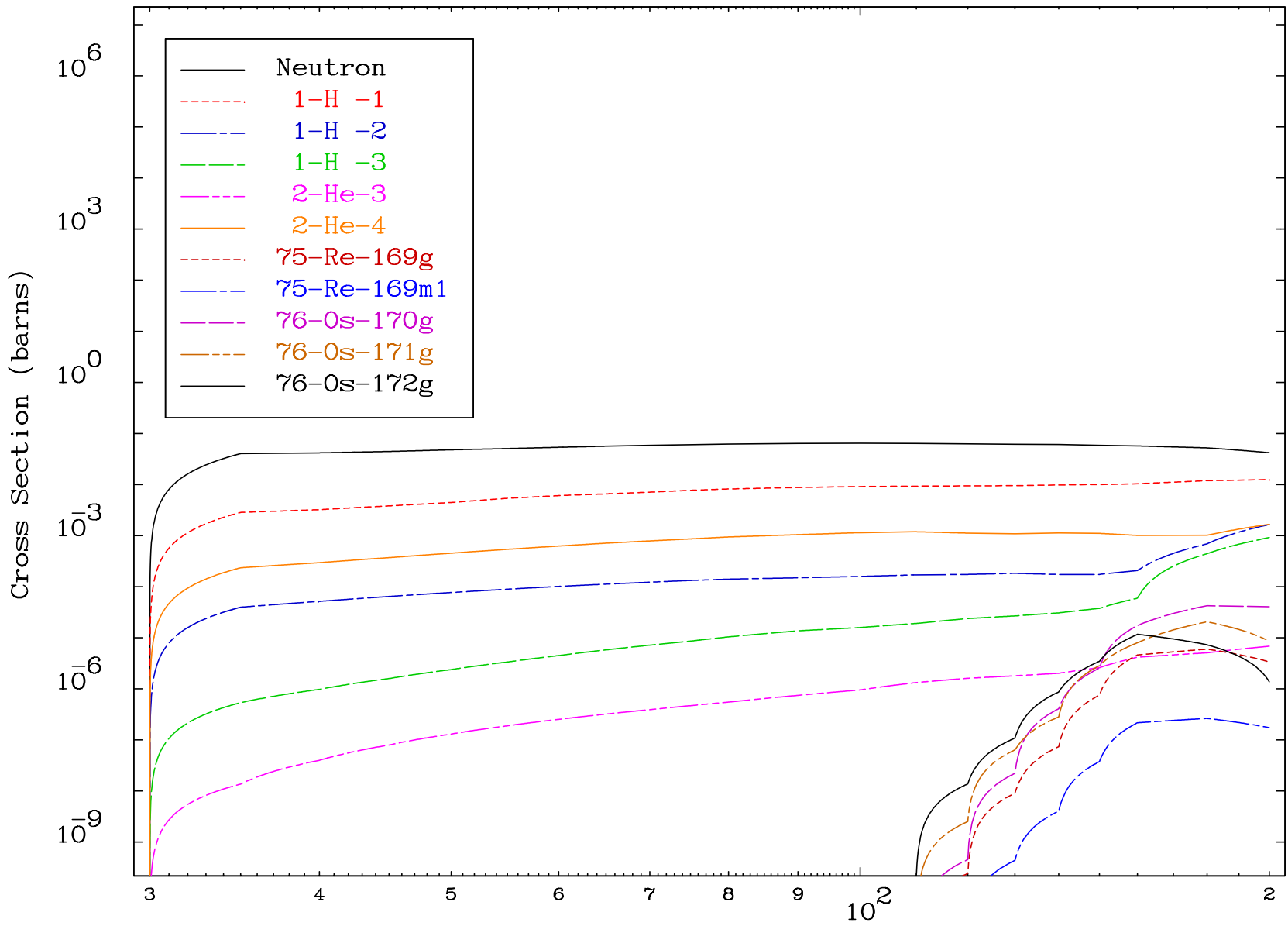
$(\gamma, \alpha)$  Levels  
0 Kelvin Cross Sections

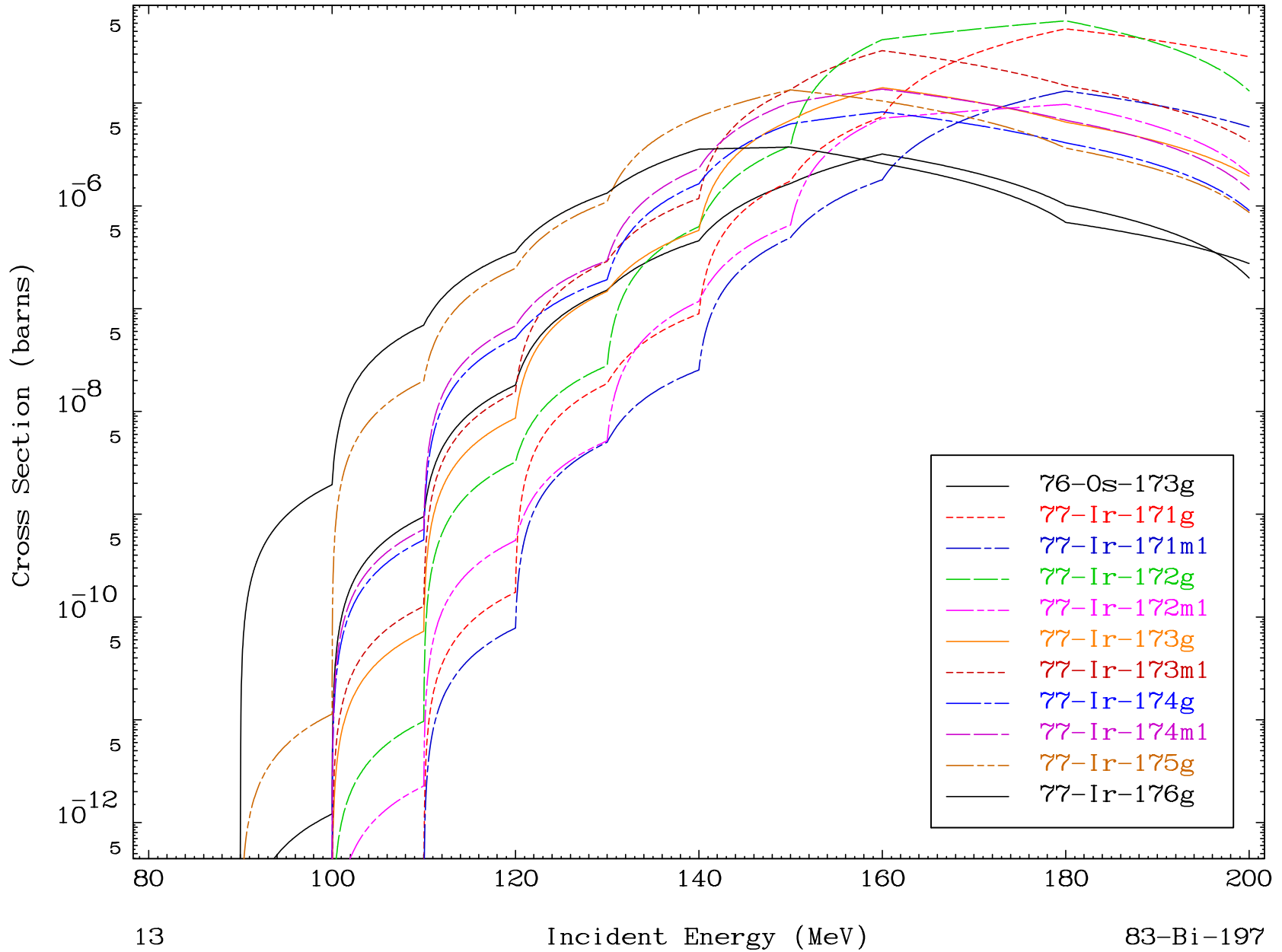
83-Bi-197

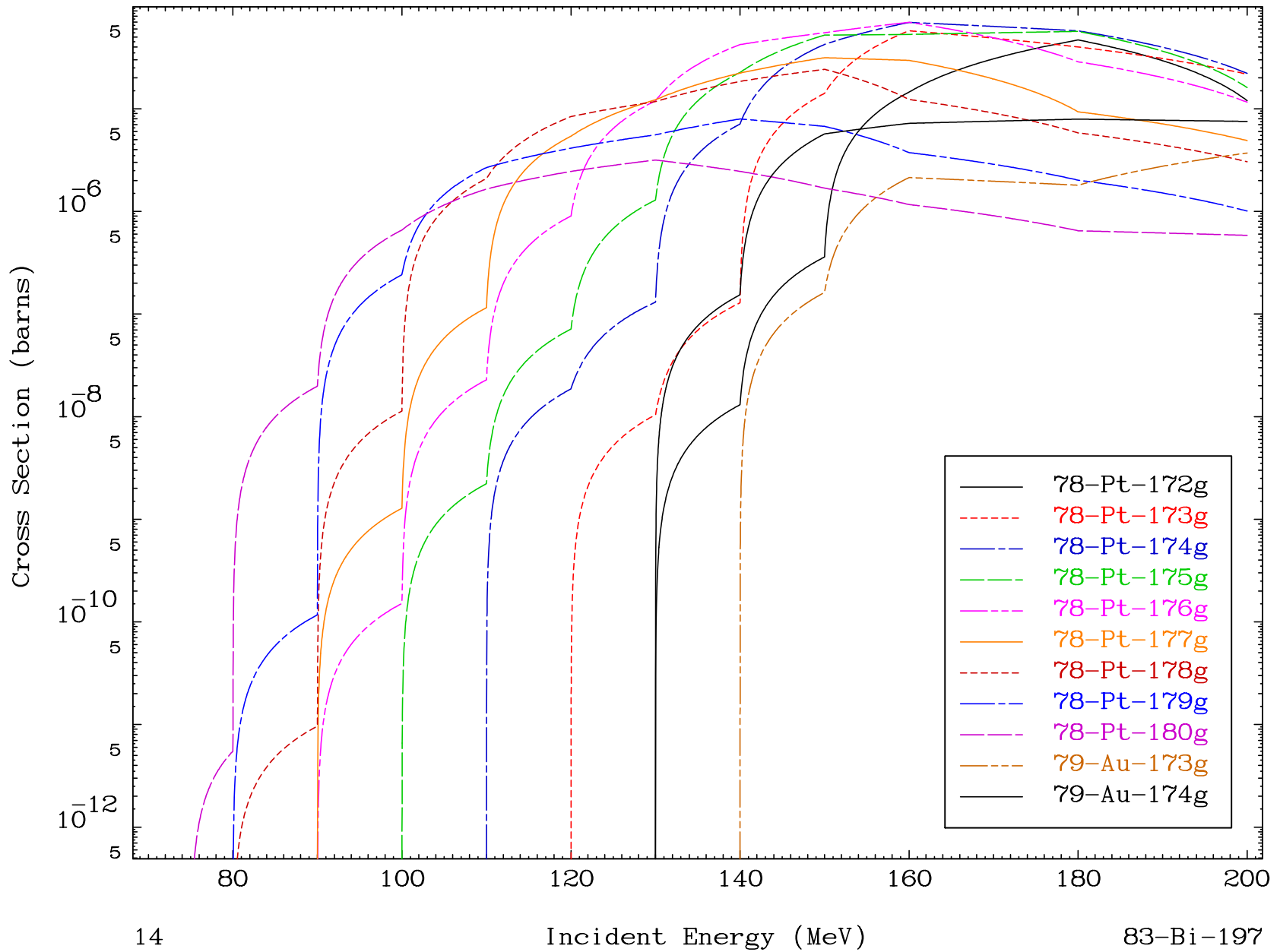




Radionuclide Production Cross Section





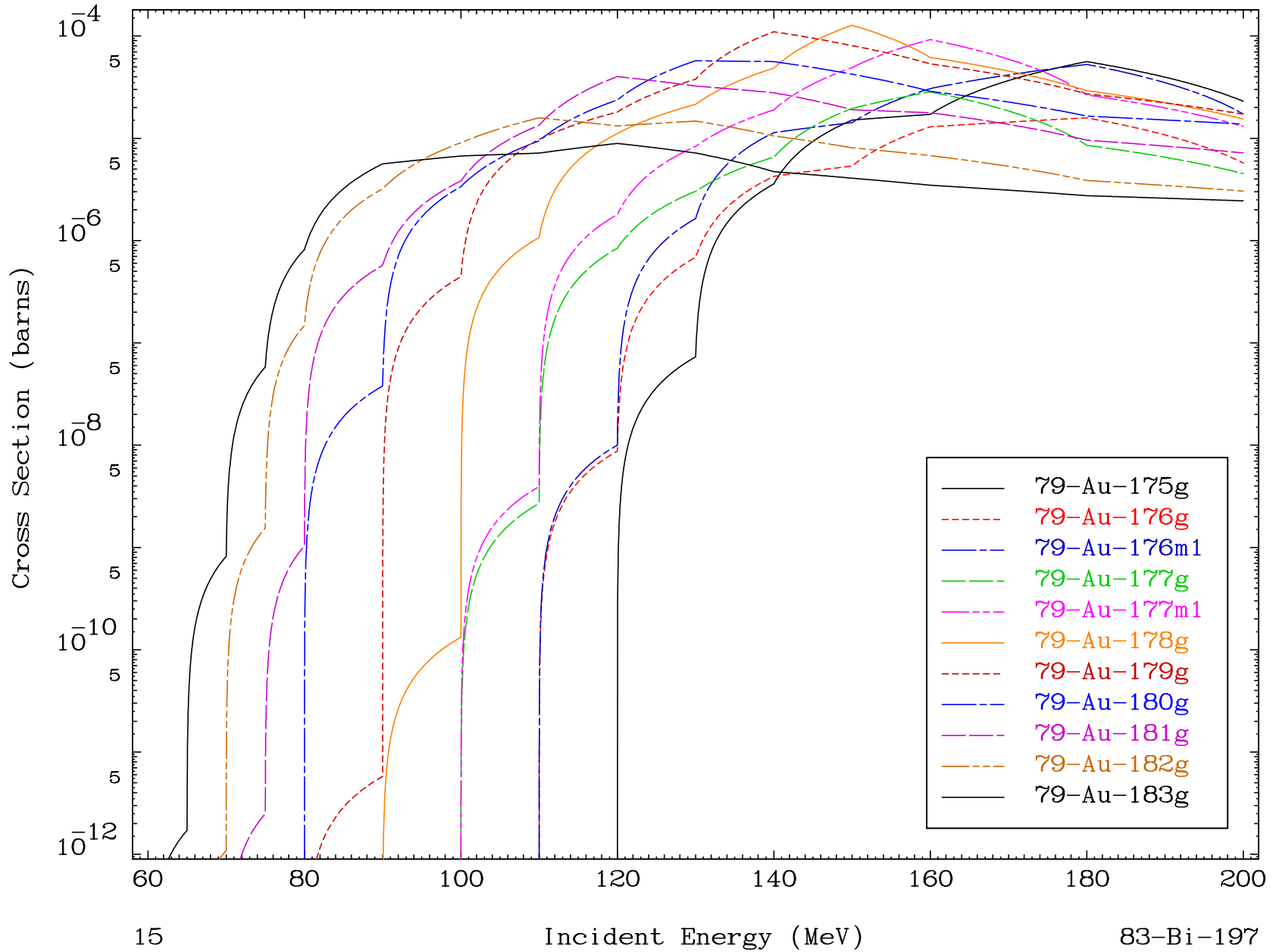


MAT 8290

( $\gamma$ , remainder)

83-Bi-197

### Radionuclide Production Cross Section



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Incident Energy (MeV)

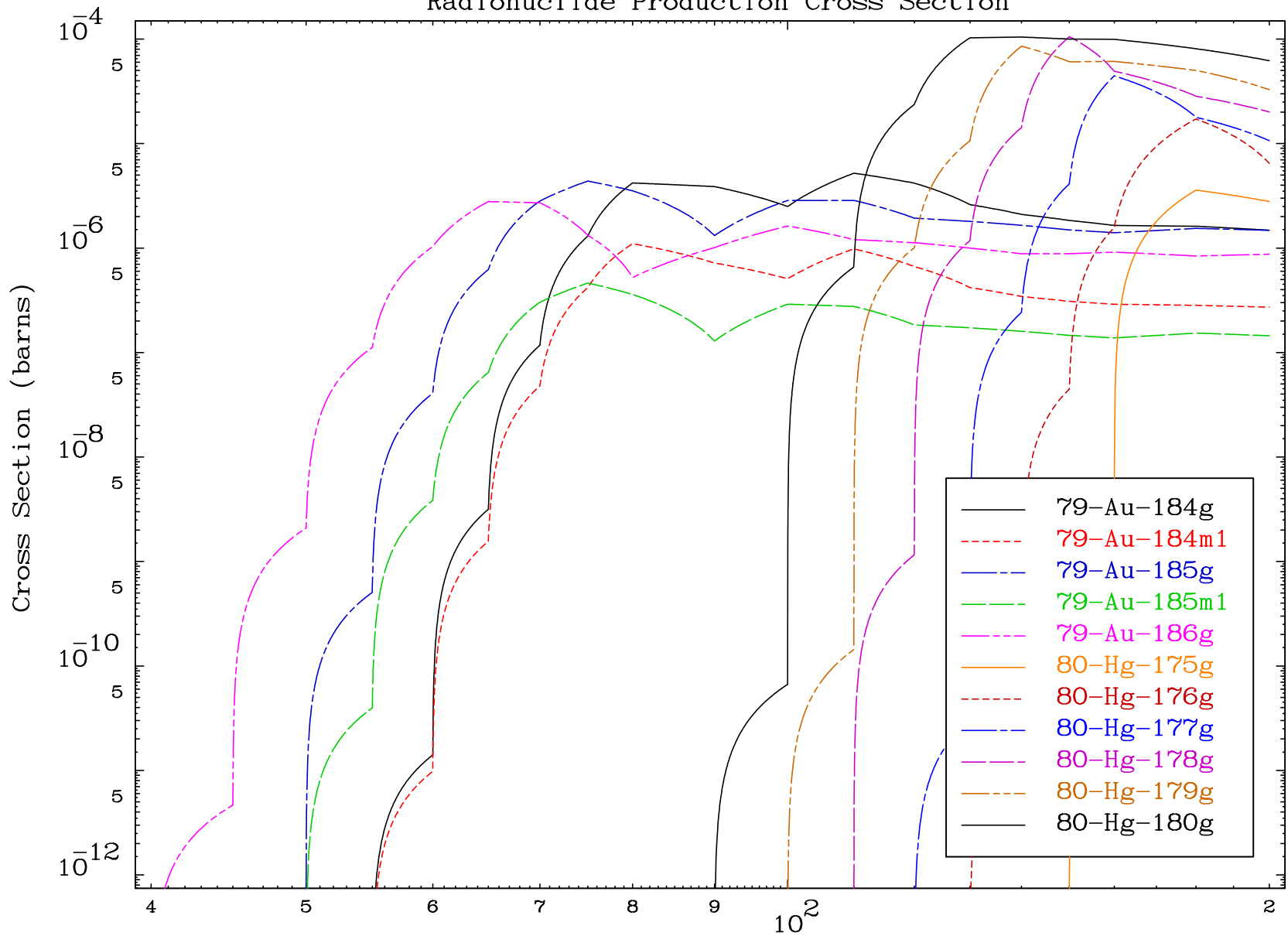
83-Bi-197

MAT 8290

( $\gamma$ , remainder)

83-Bi-197

### Radionuclide Production Cross Section

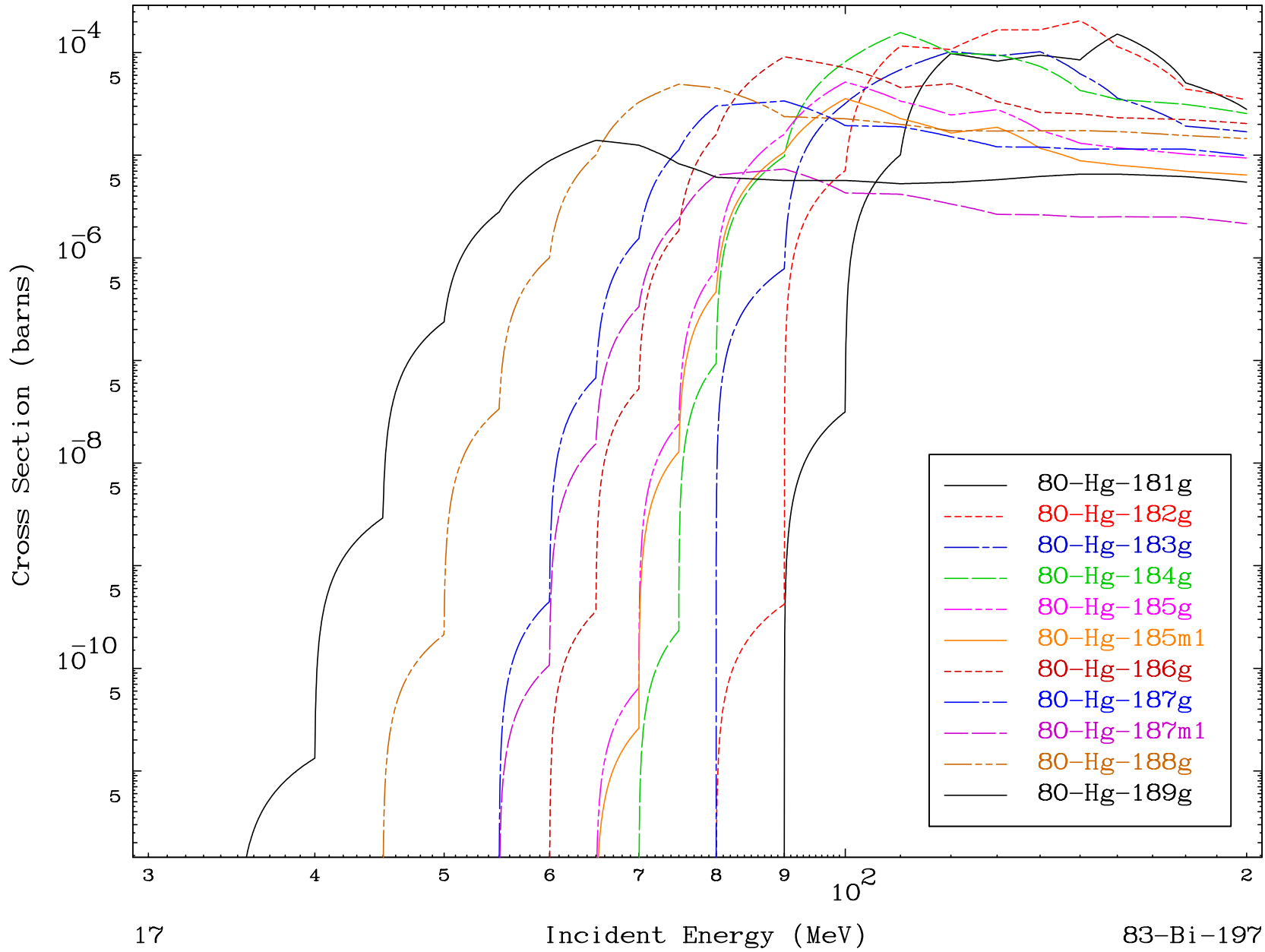


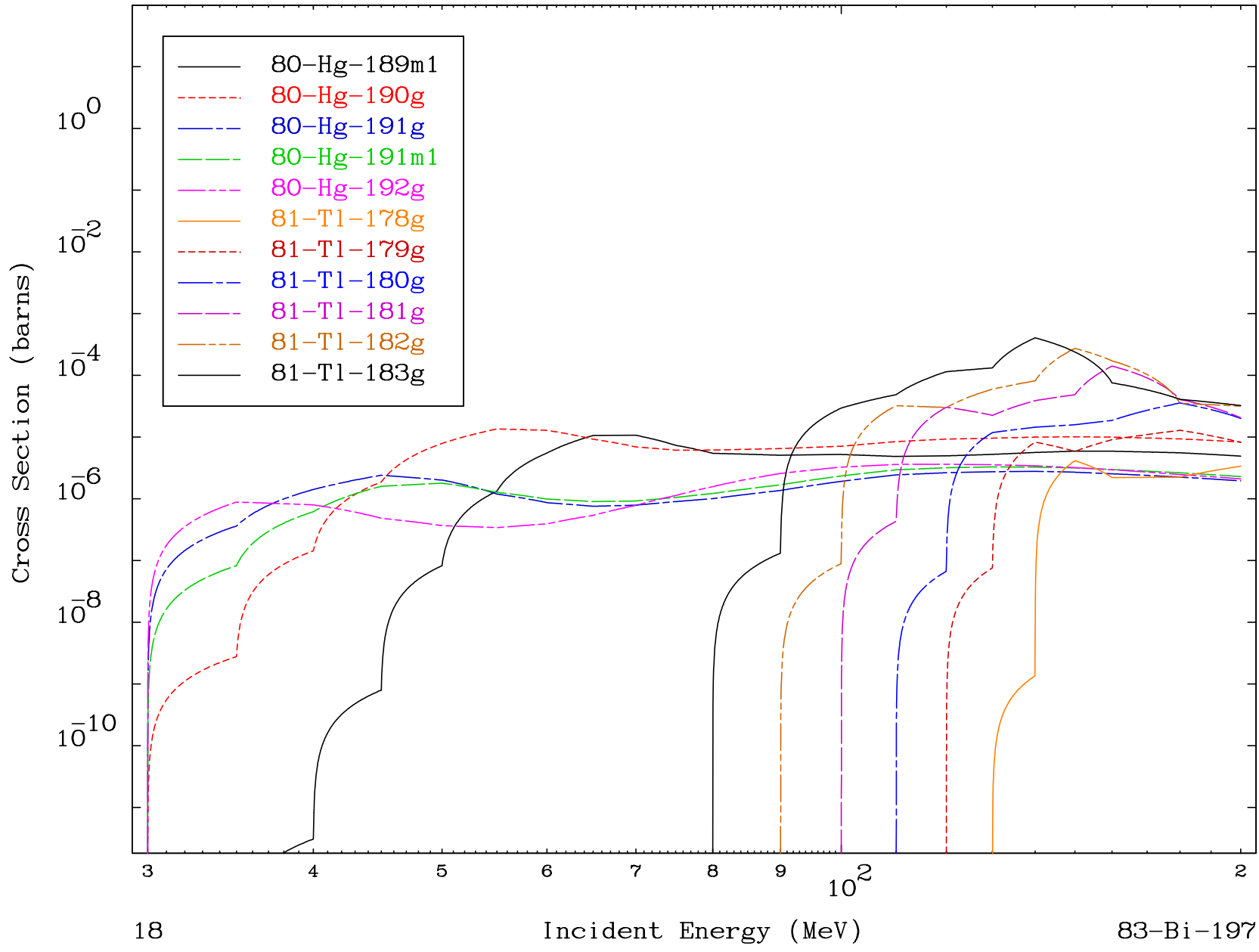
16

Incident Energy (MeV)

83-Bi-197





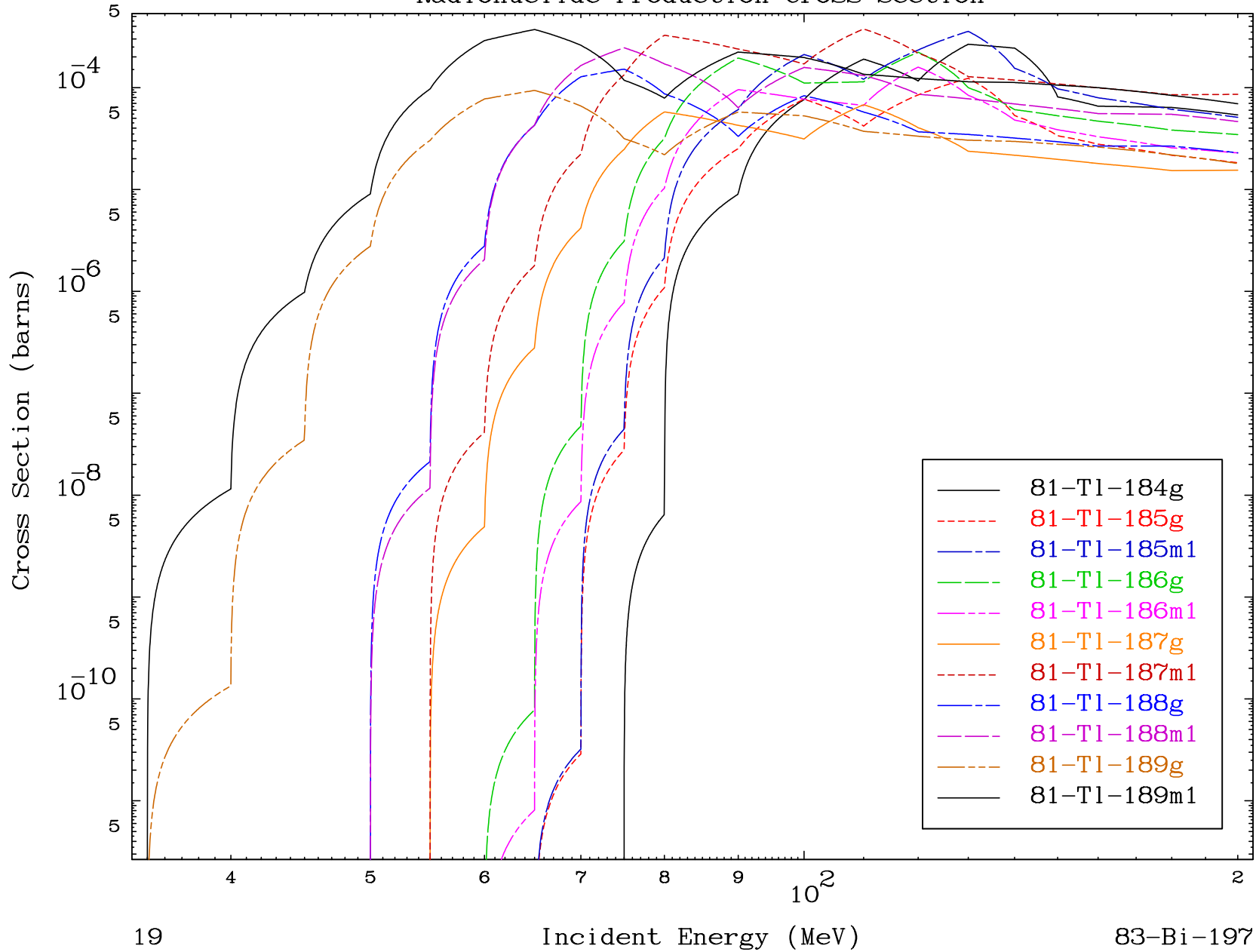


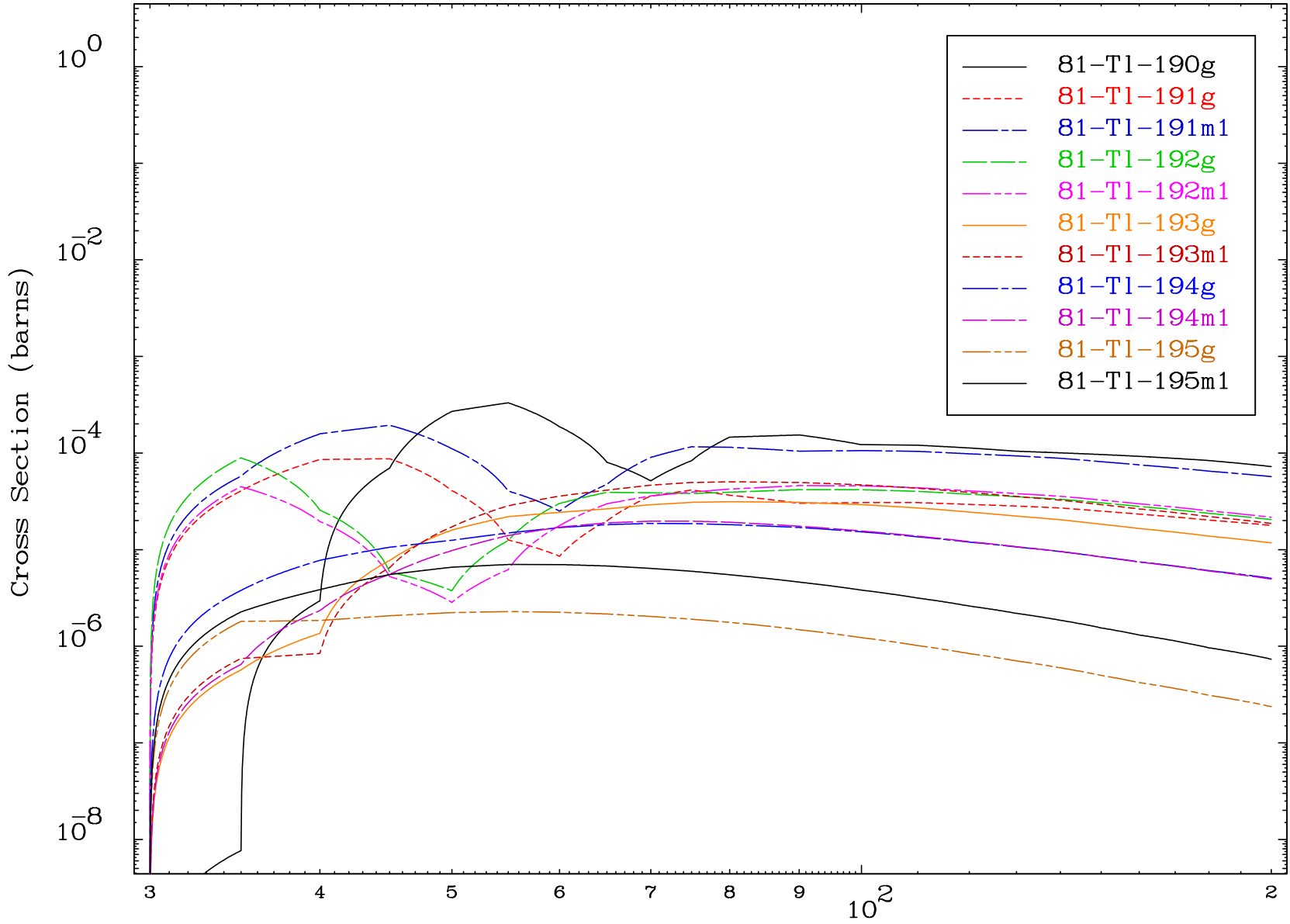
MAT 8290

( $\gamma$ , remainder)

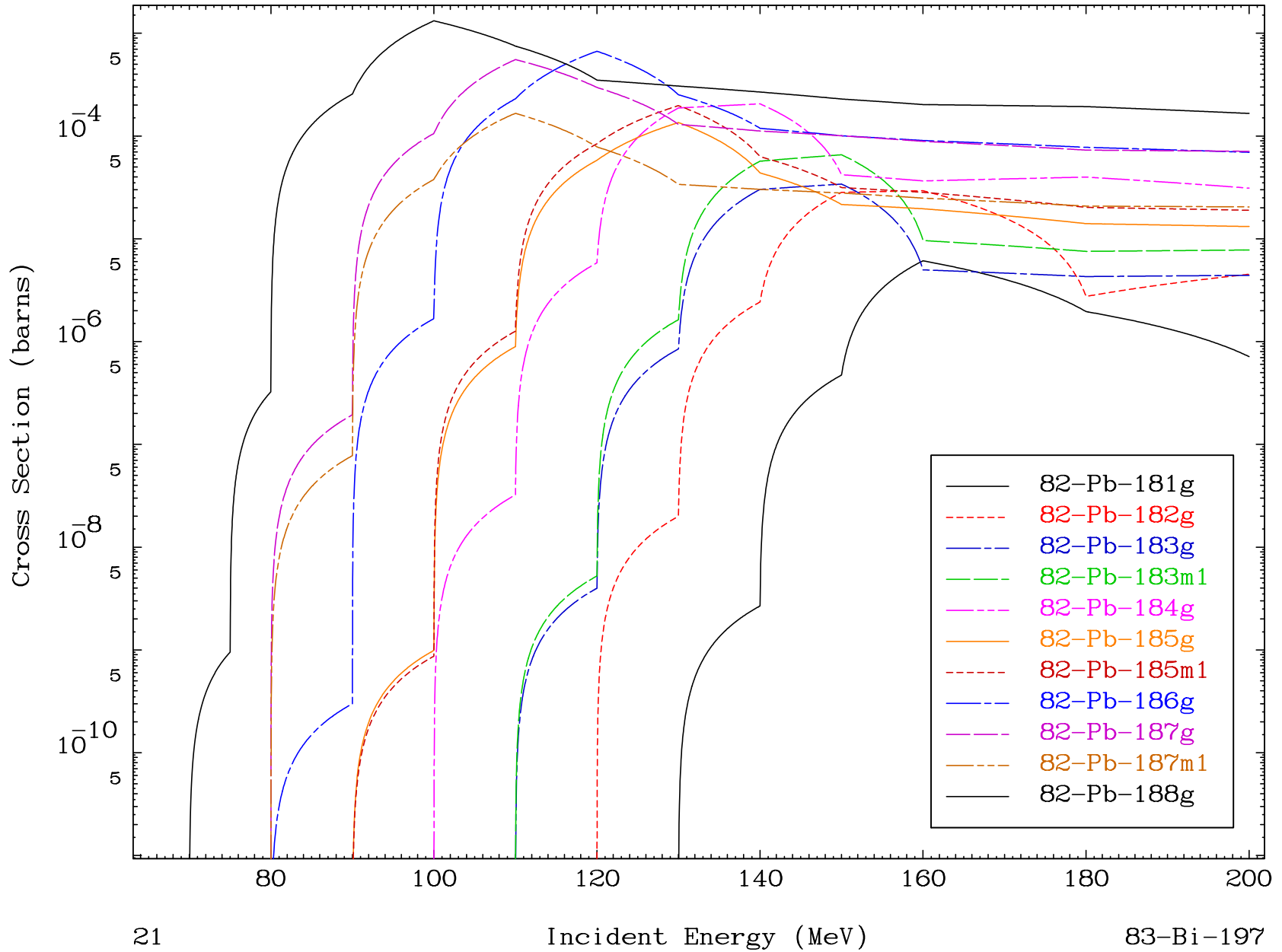
83-Bi-197

### Radionuclide Production Cross Section

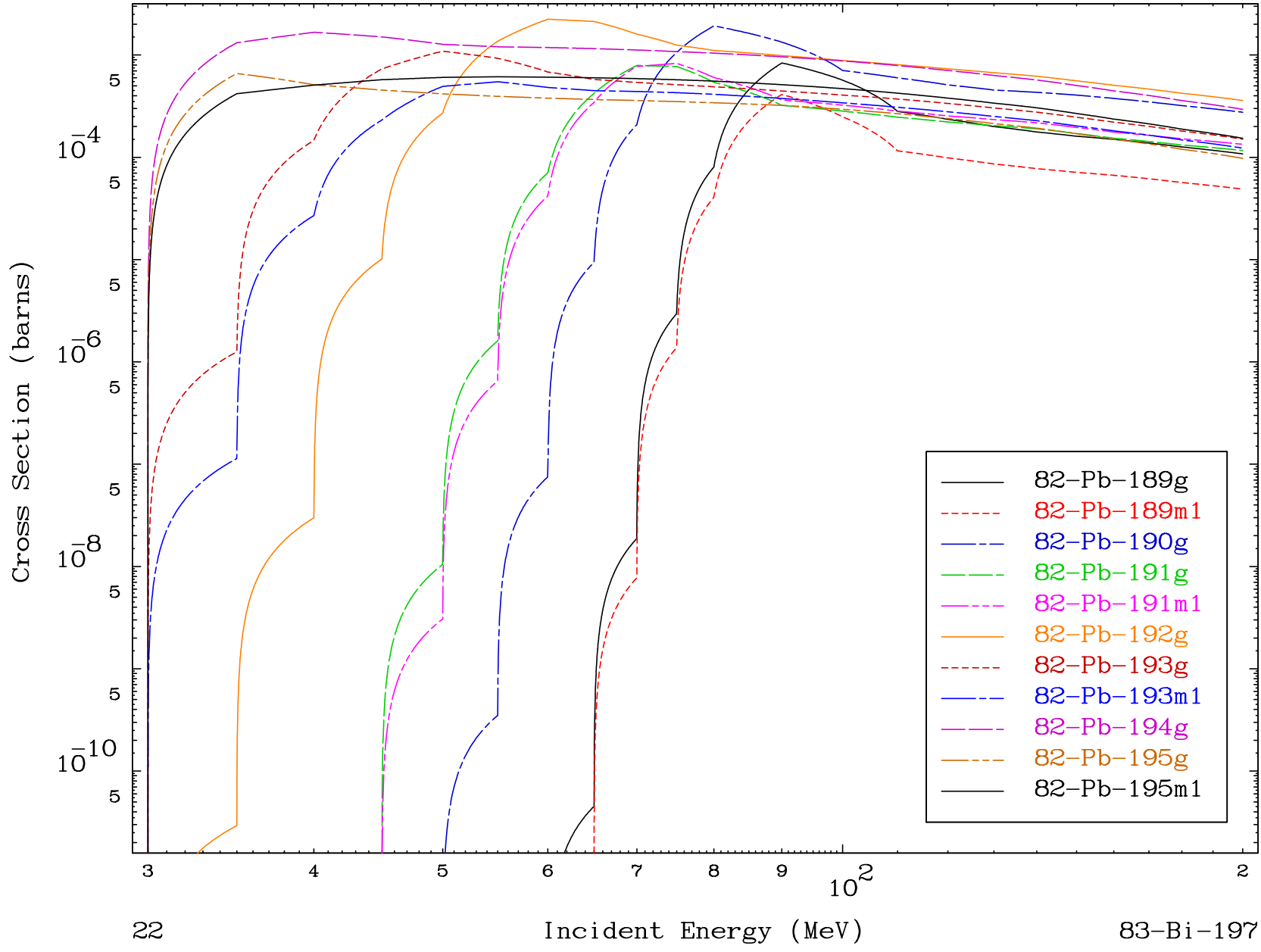


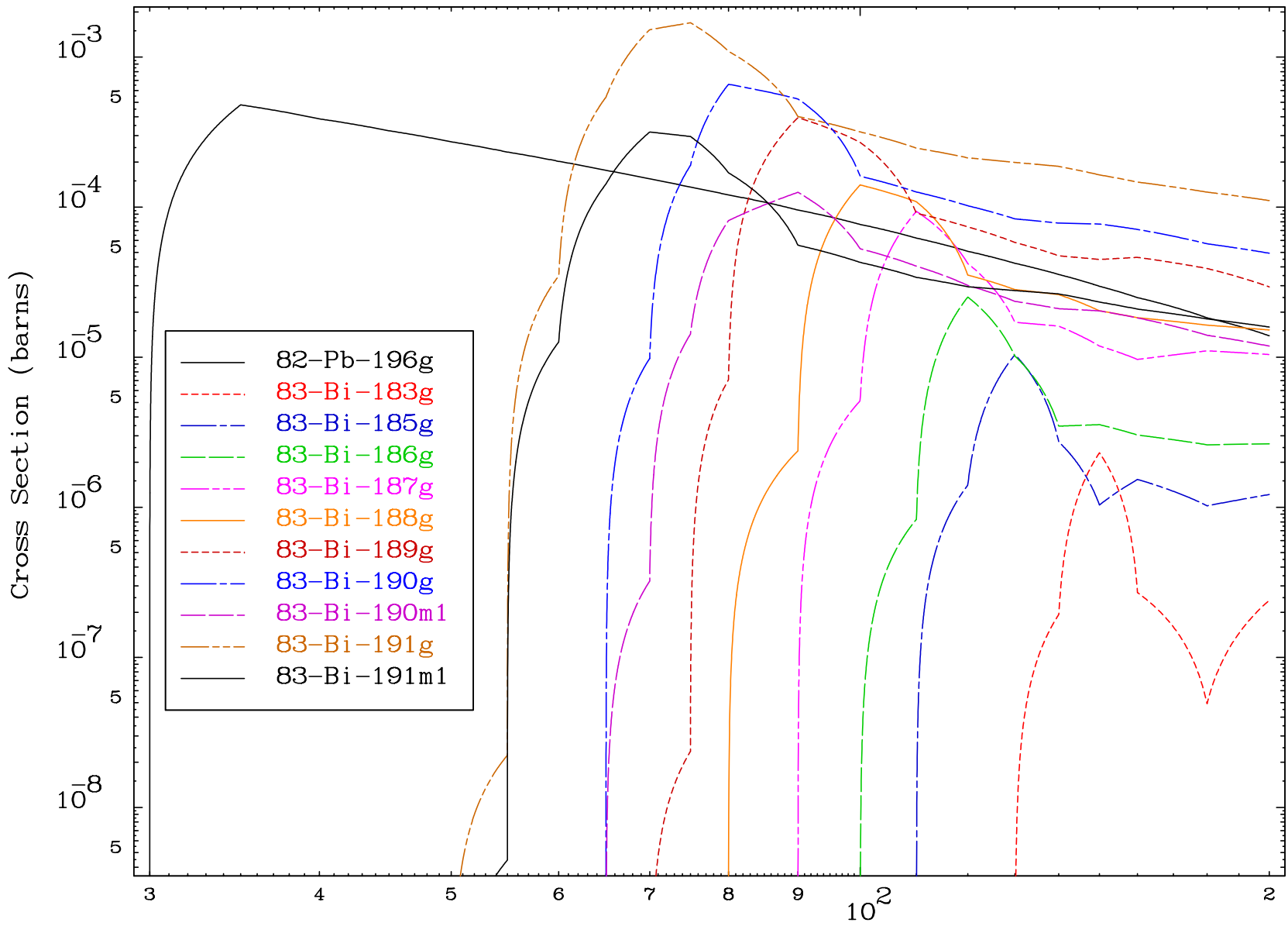


Radionuclide Production Cross Section

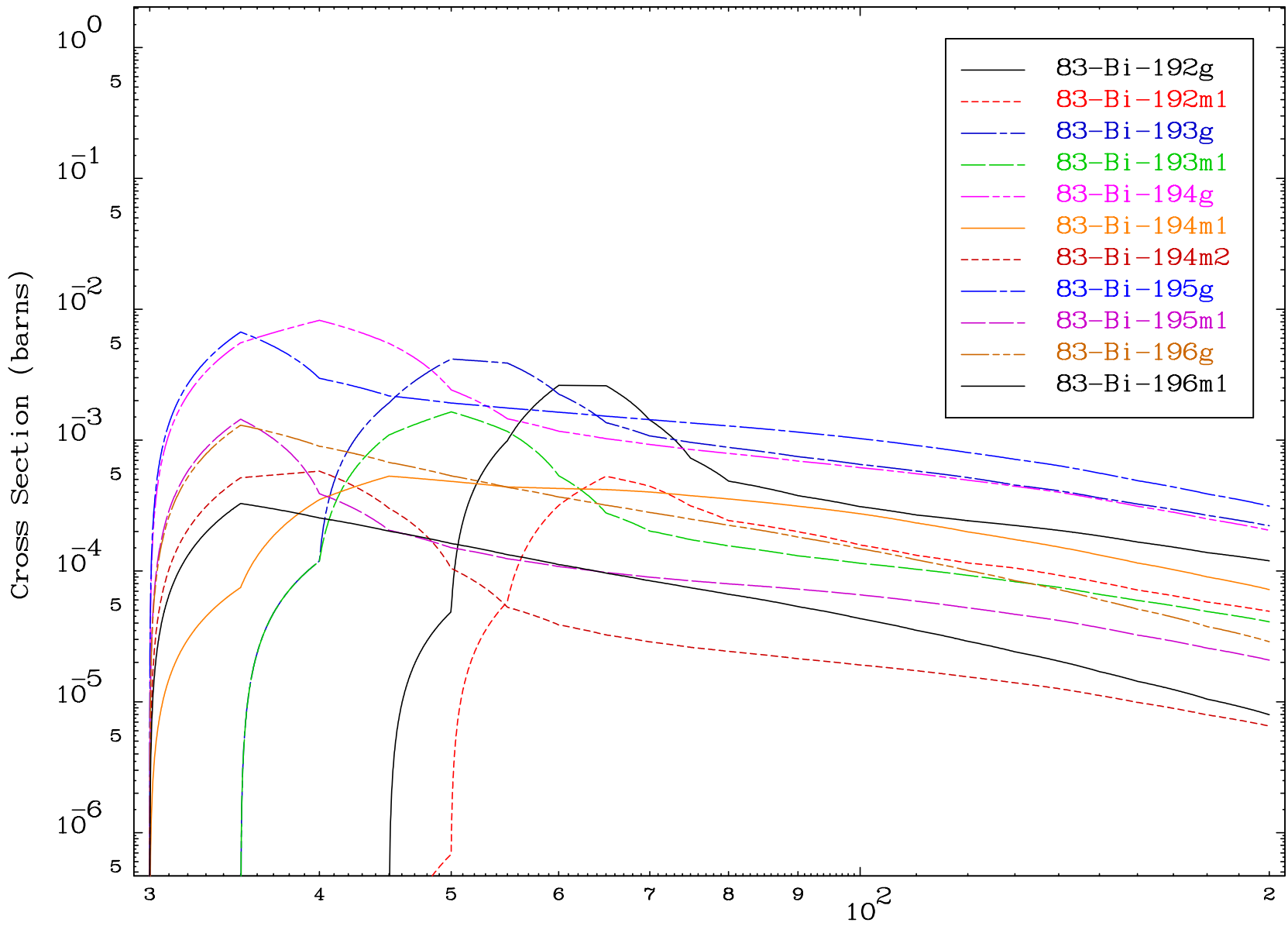


Radionuclide Production Cross Section





Radionuclide Production Cross Section



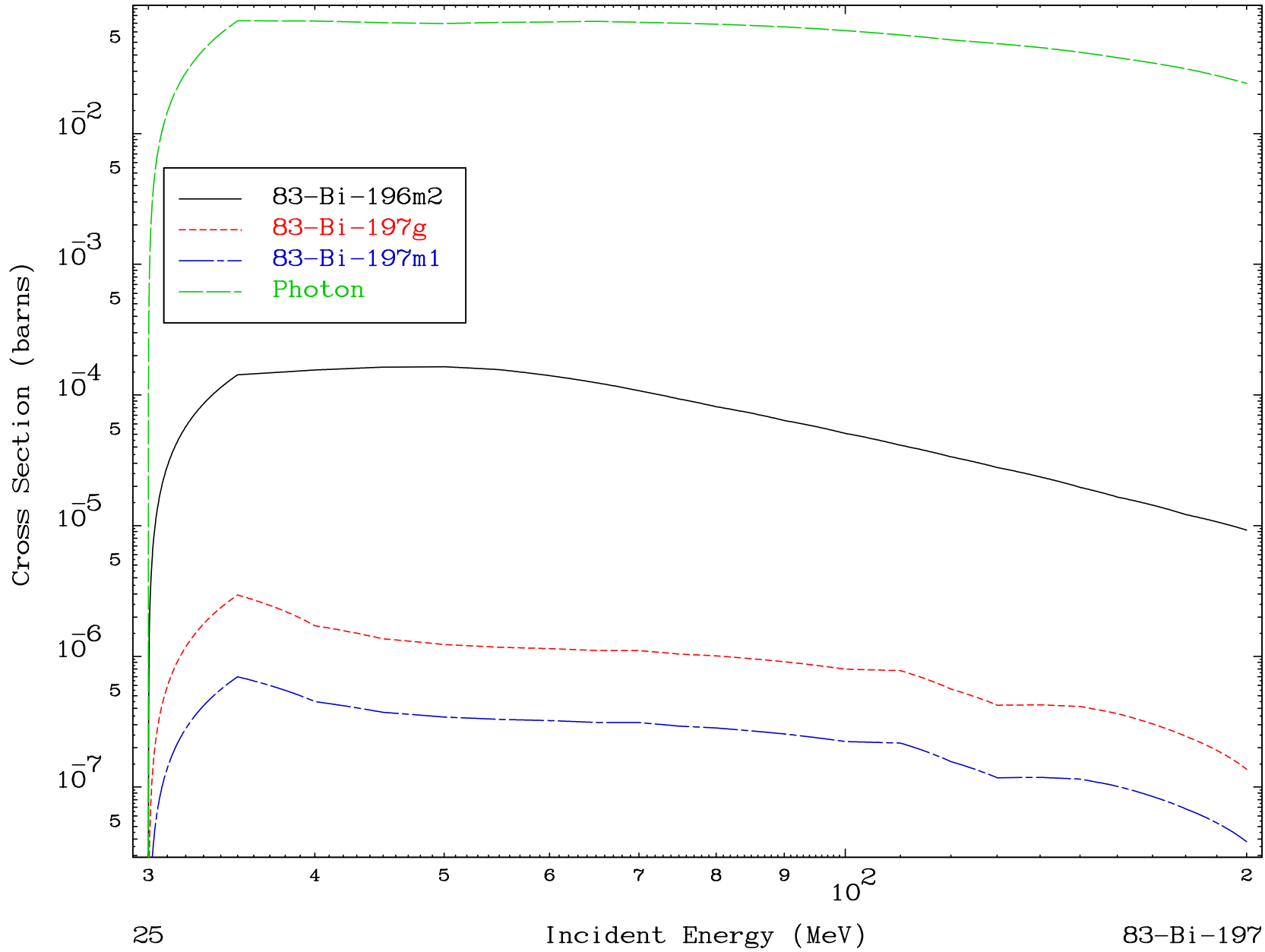


MAT 8290

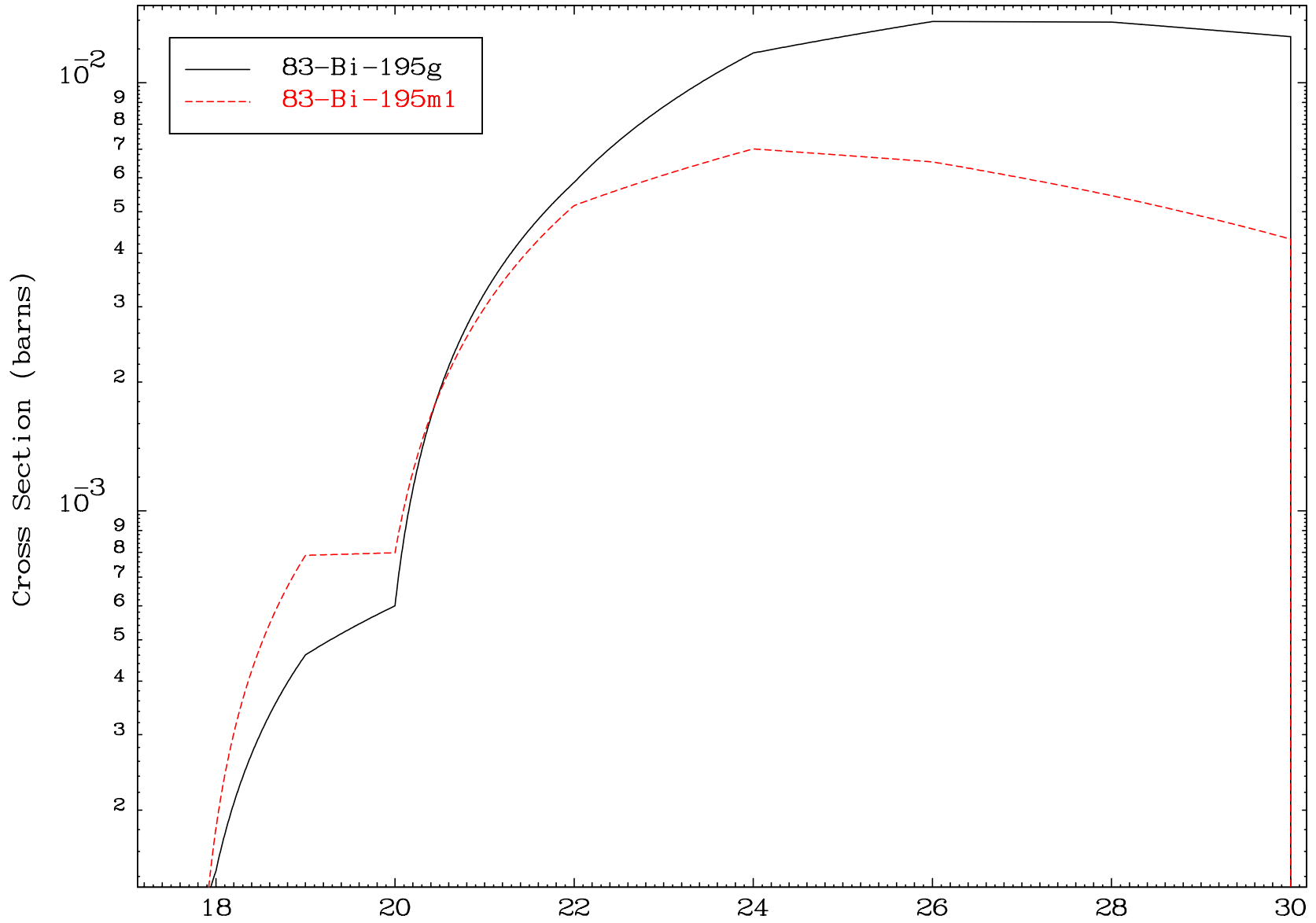
( $\gamma$ , remainder)

83-Bi-197

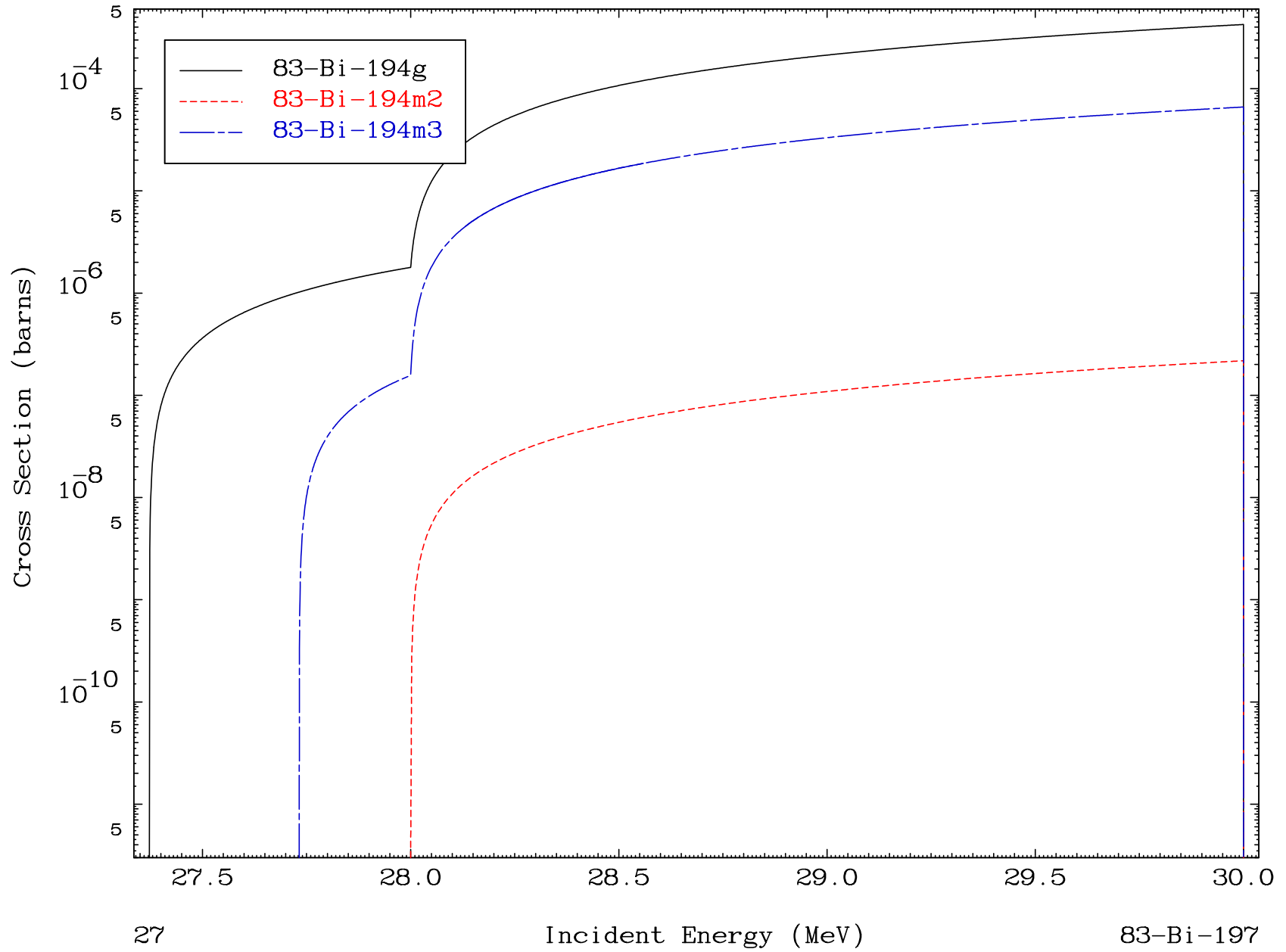
Radionuclide Production Cross Section

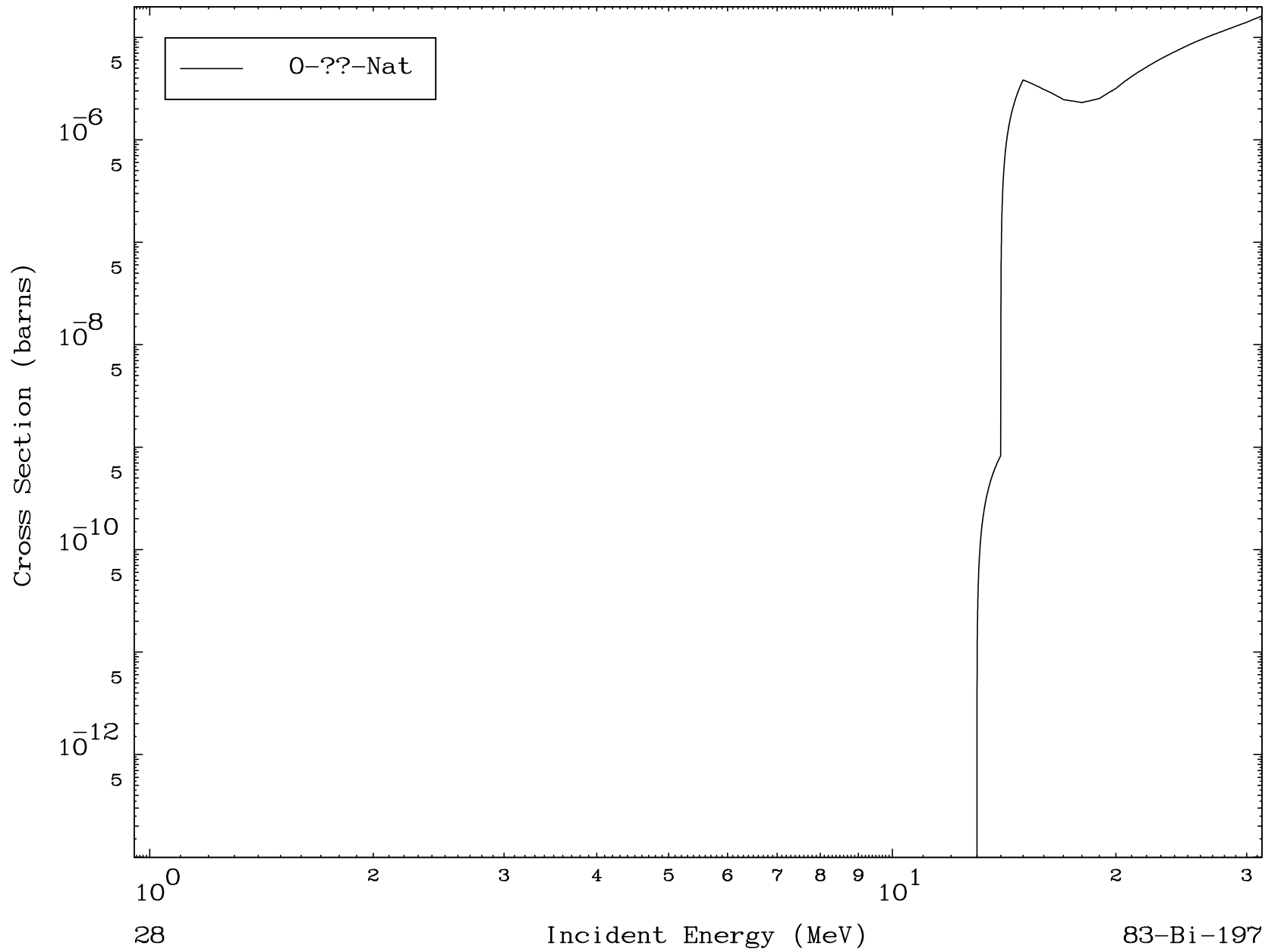


Radionuclide Production Cross Section



Radionuclide Production Cross Section



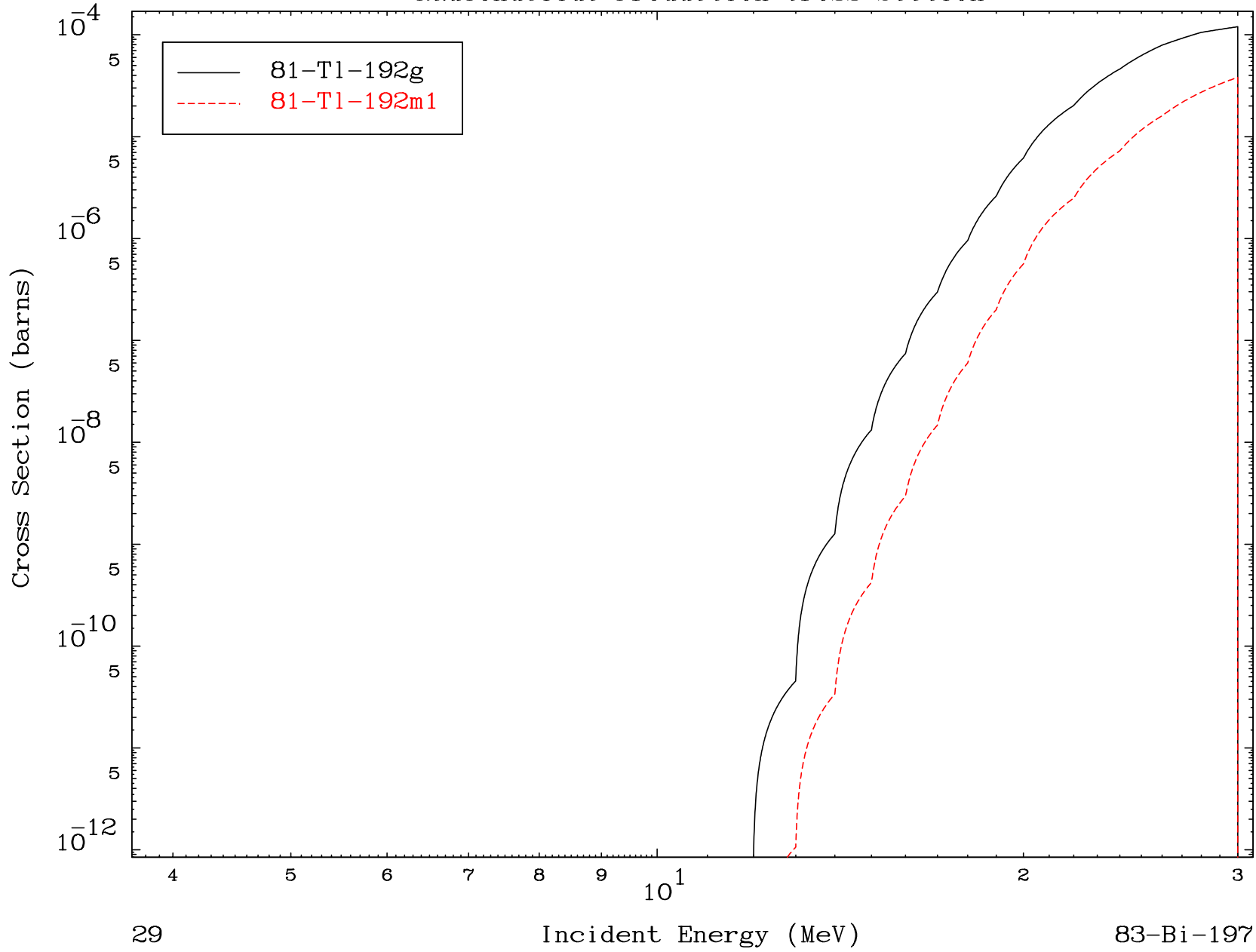


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$(\gamma, n')$   $\alpha$

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Radionuclide Production Cross Section

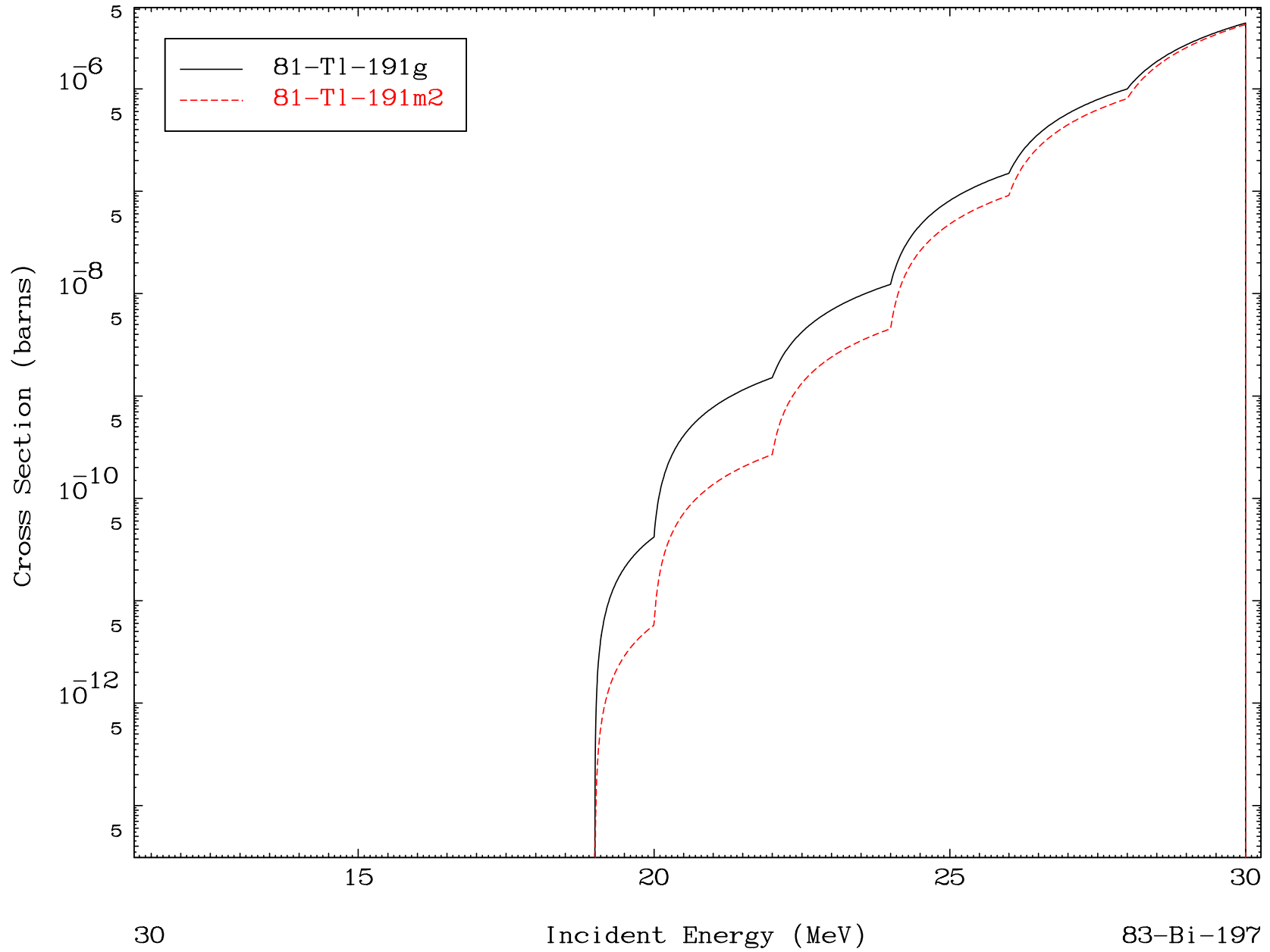


MAT 8290

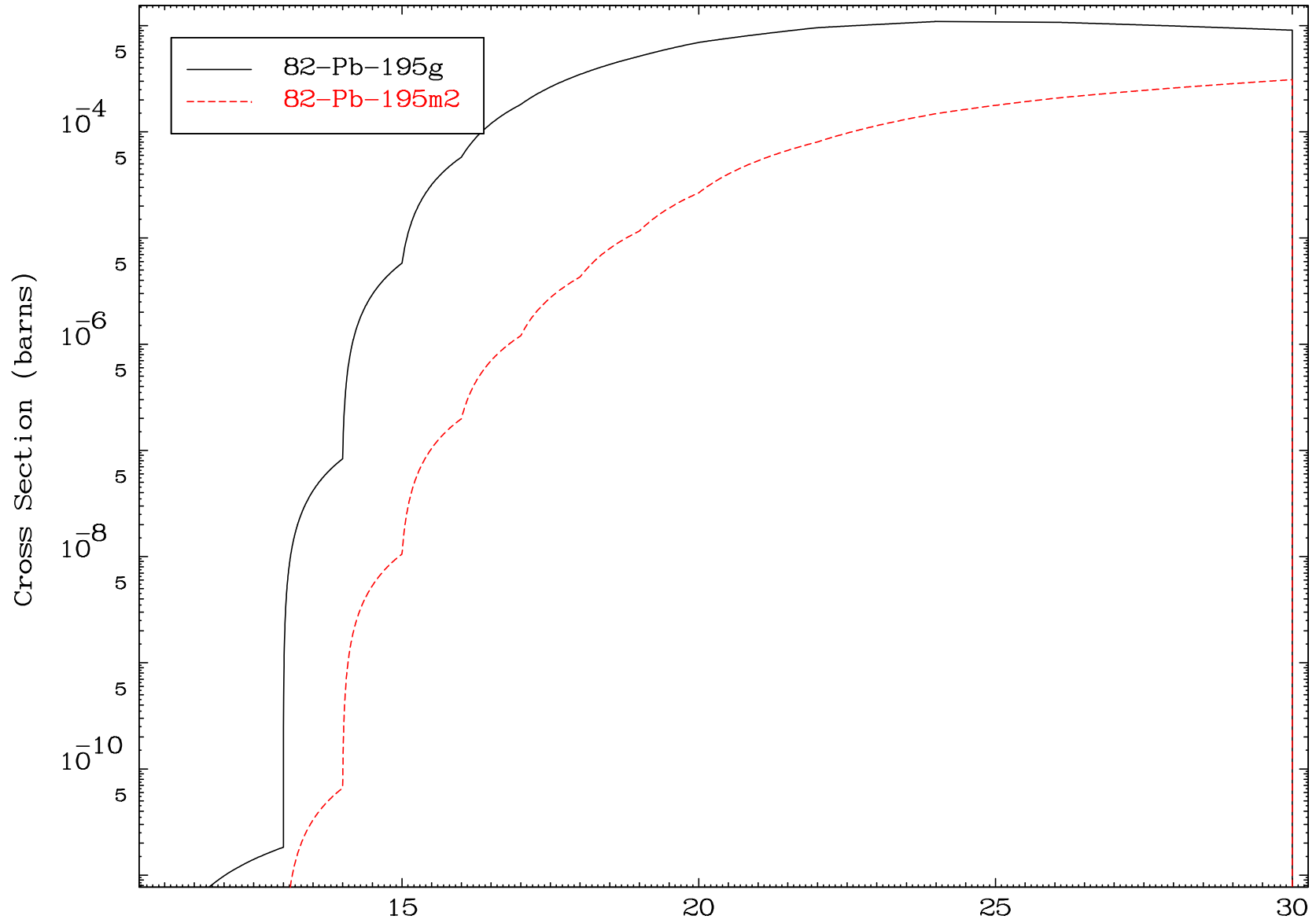
$(\gamma, 2n) \alpha$

83-Bi-197

Radionuclide Production Cross Section



Radionuclide Production Cross Section

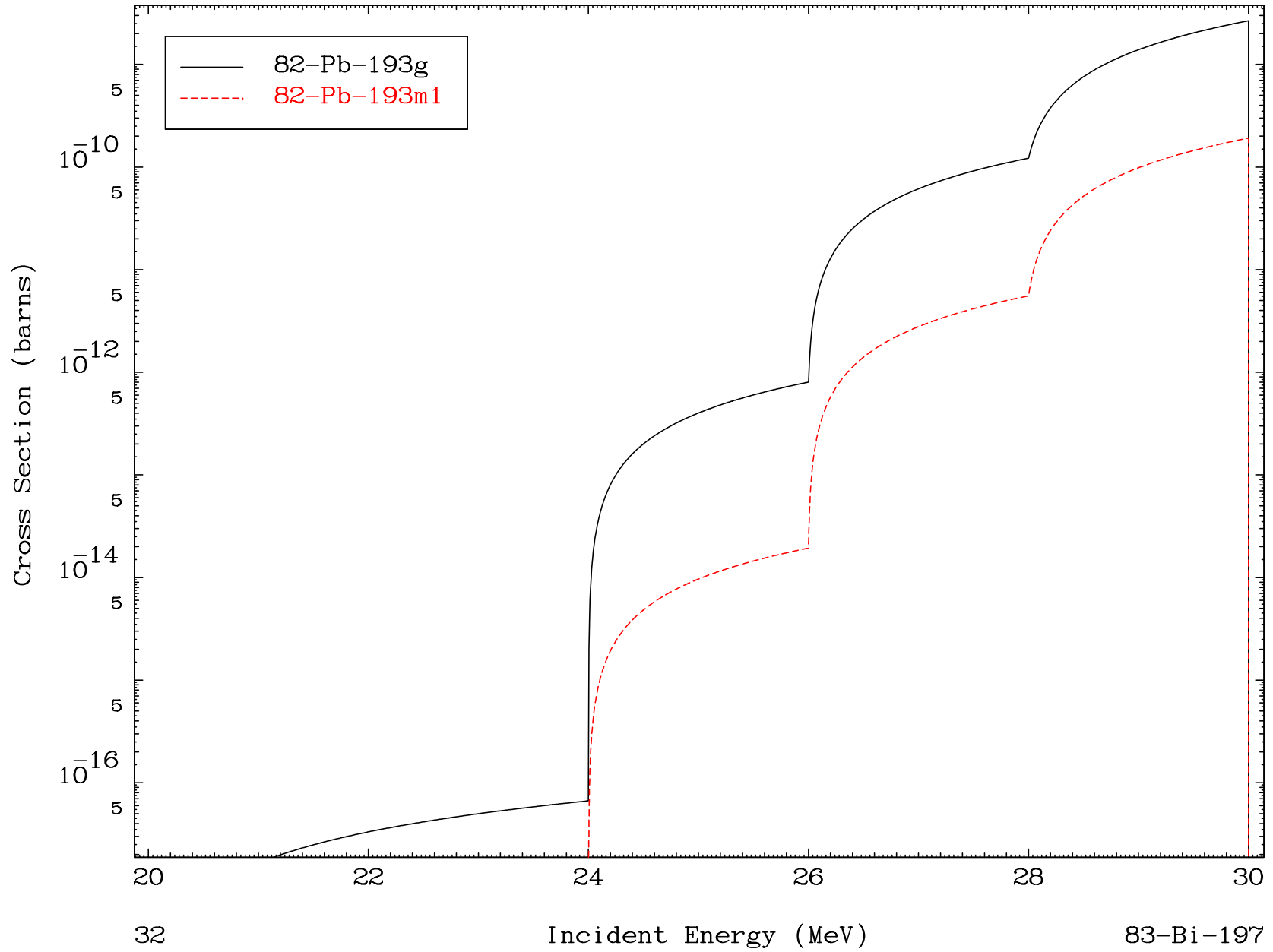


MAT 8290

( $\gamma, n'$ ) t

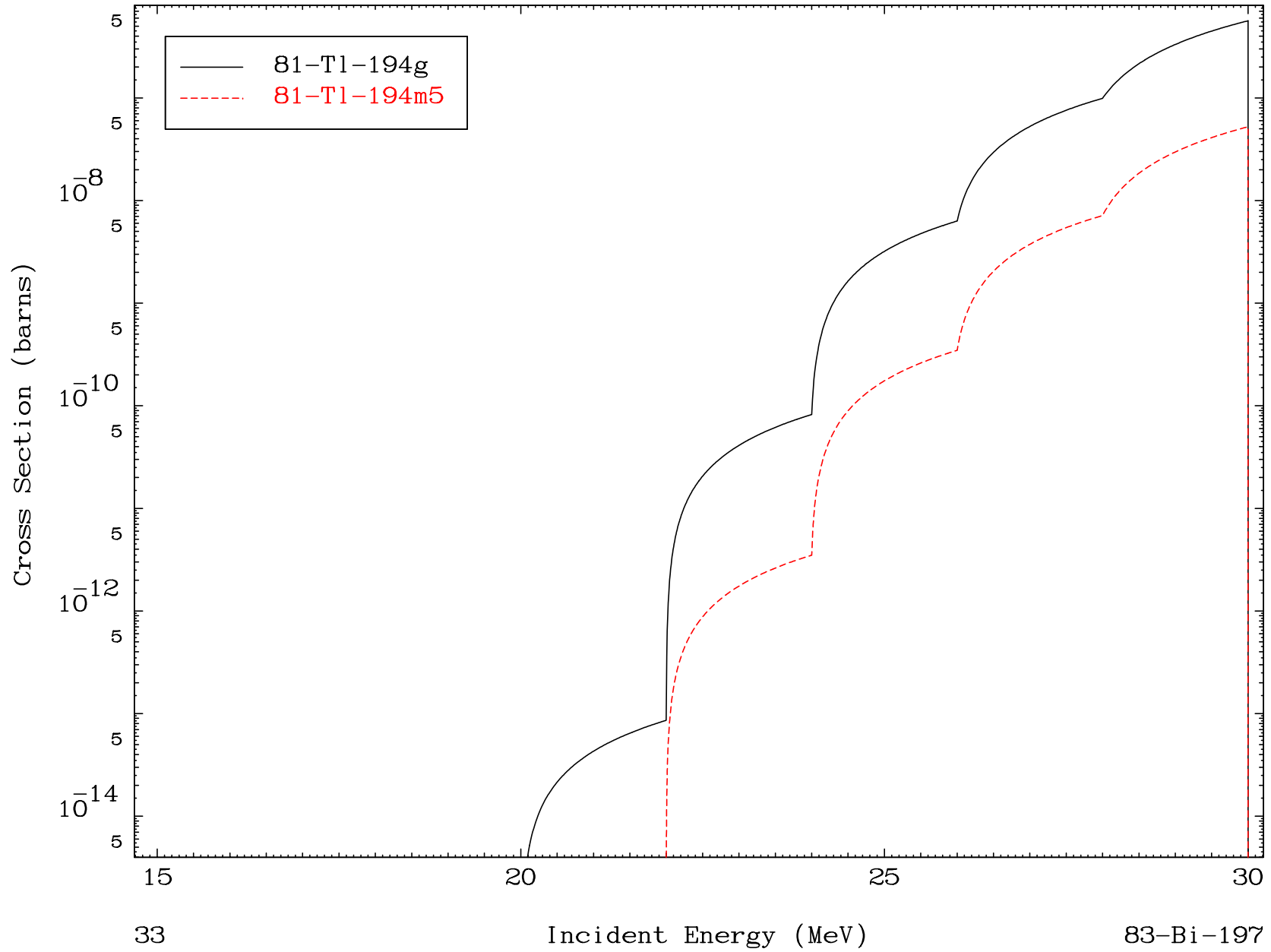
83-Bi-197

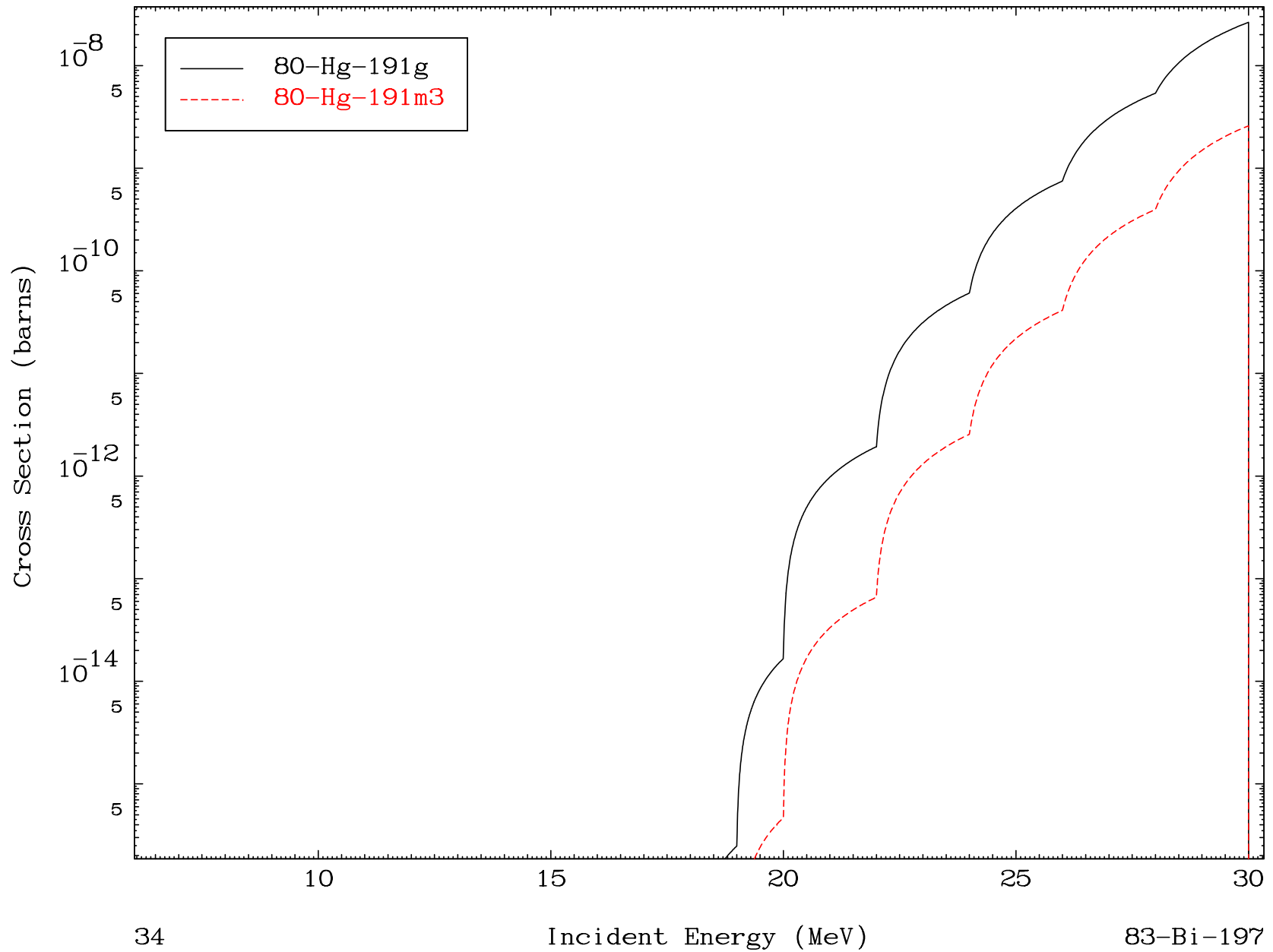
Radionuclide Production Cross Section





Radionuclide Production Cross Section



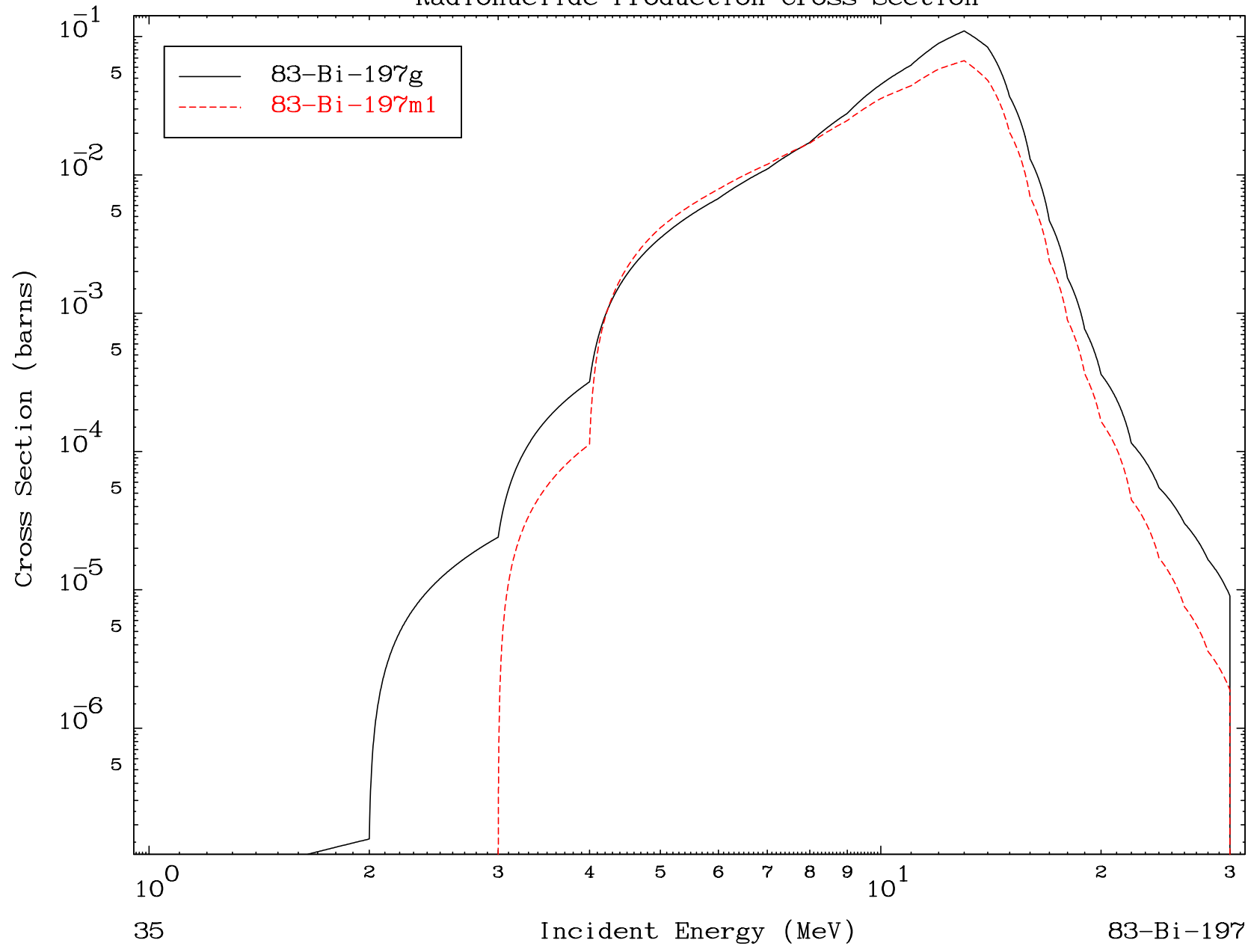


MAT 8290

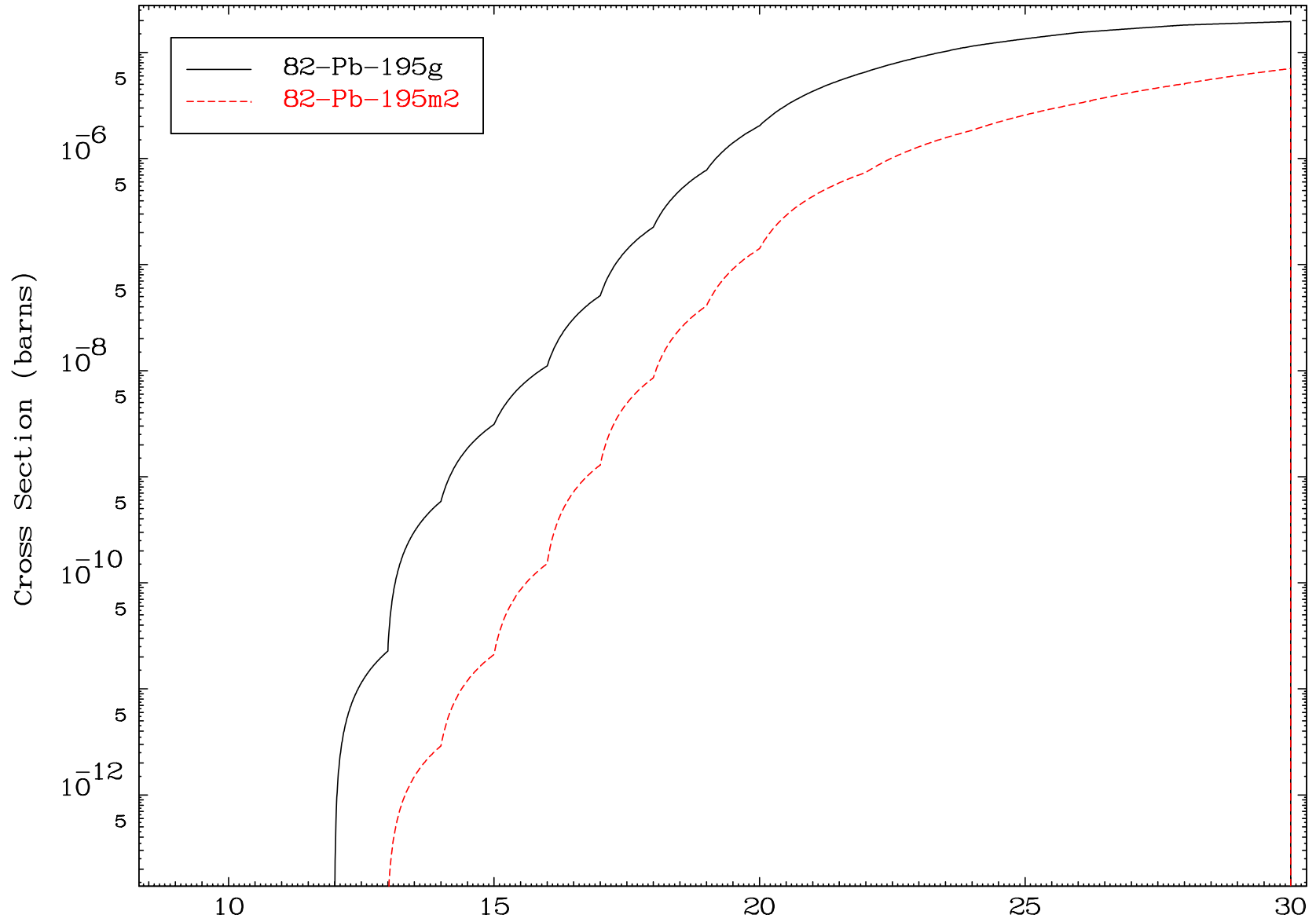
( $\gamma, \gamma$ )

83-Bi-197

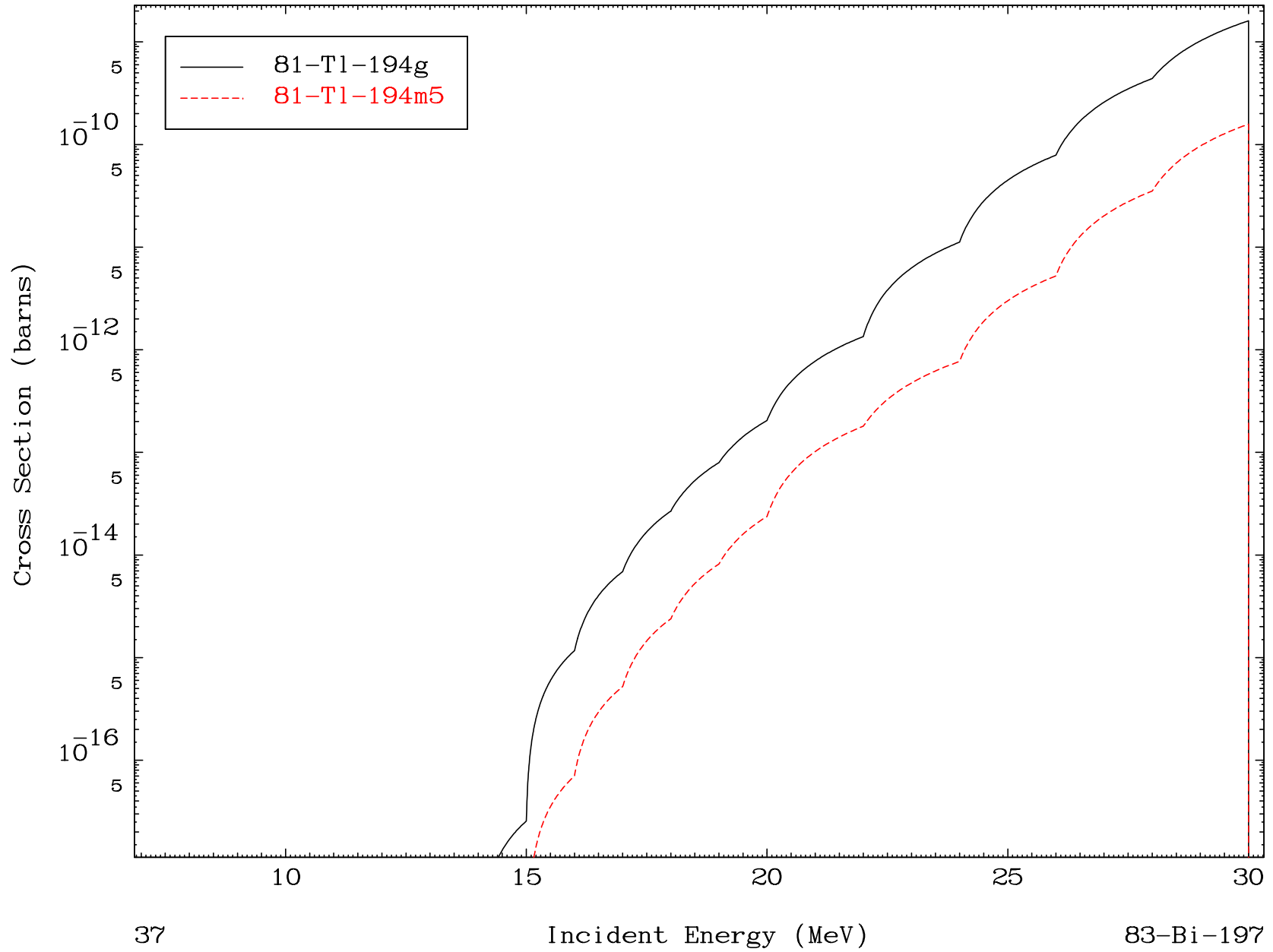
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

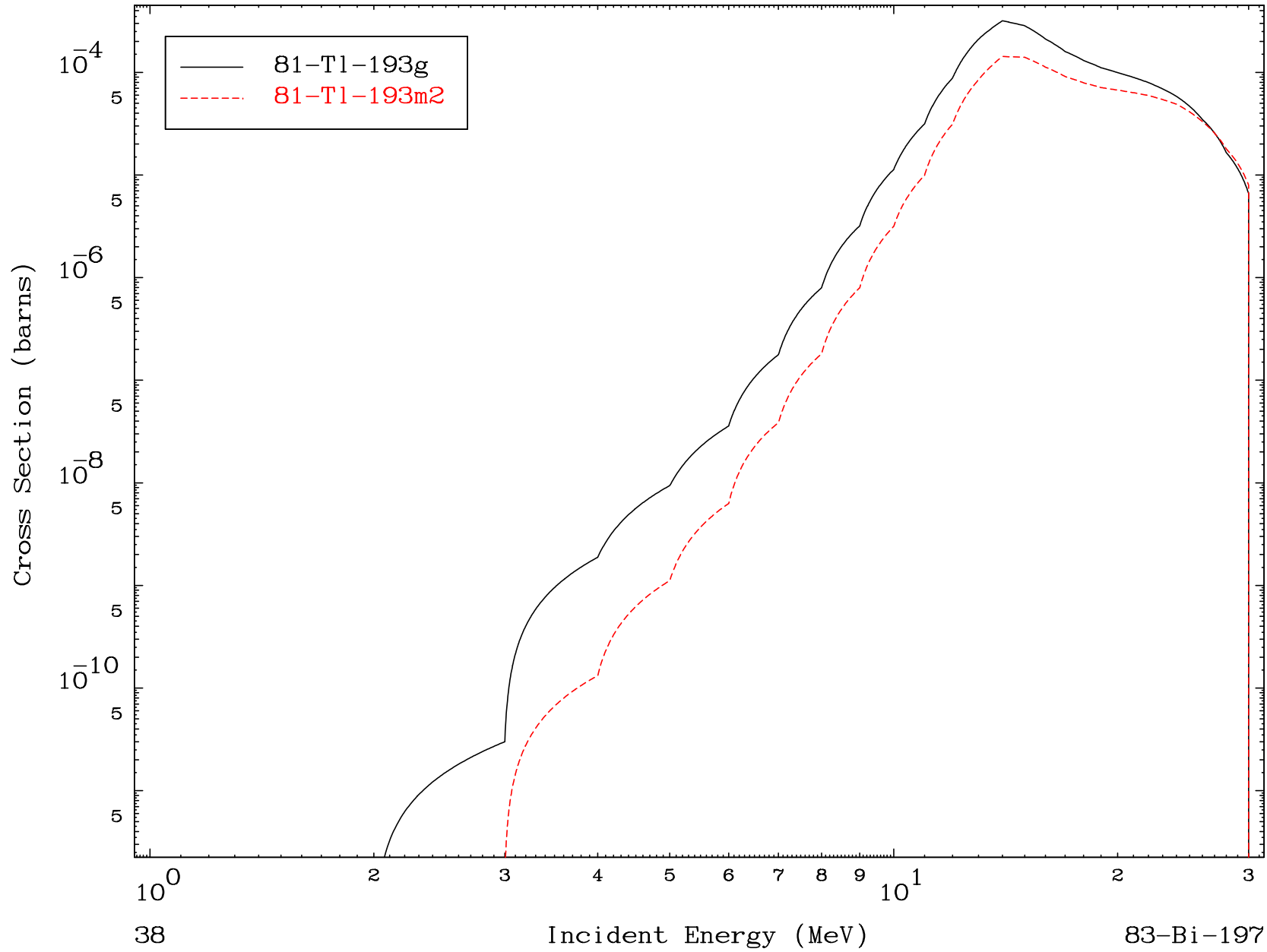


MAT 8290

( $\gamma, \alpha$ )

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Radionuclide Production Cross Section

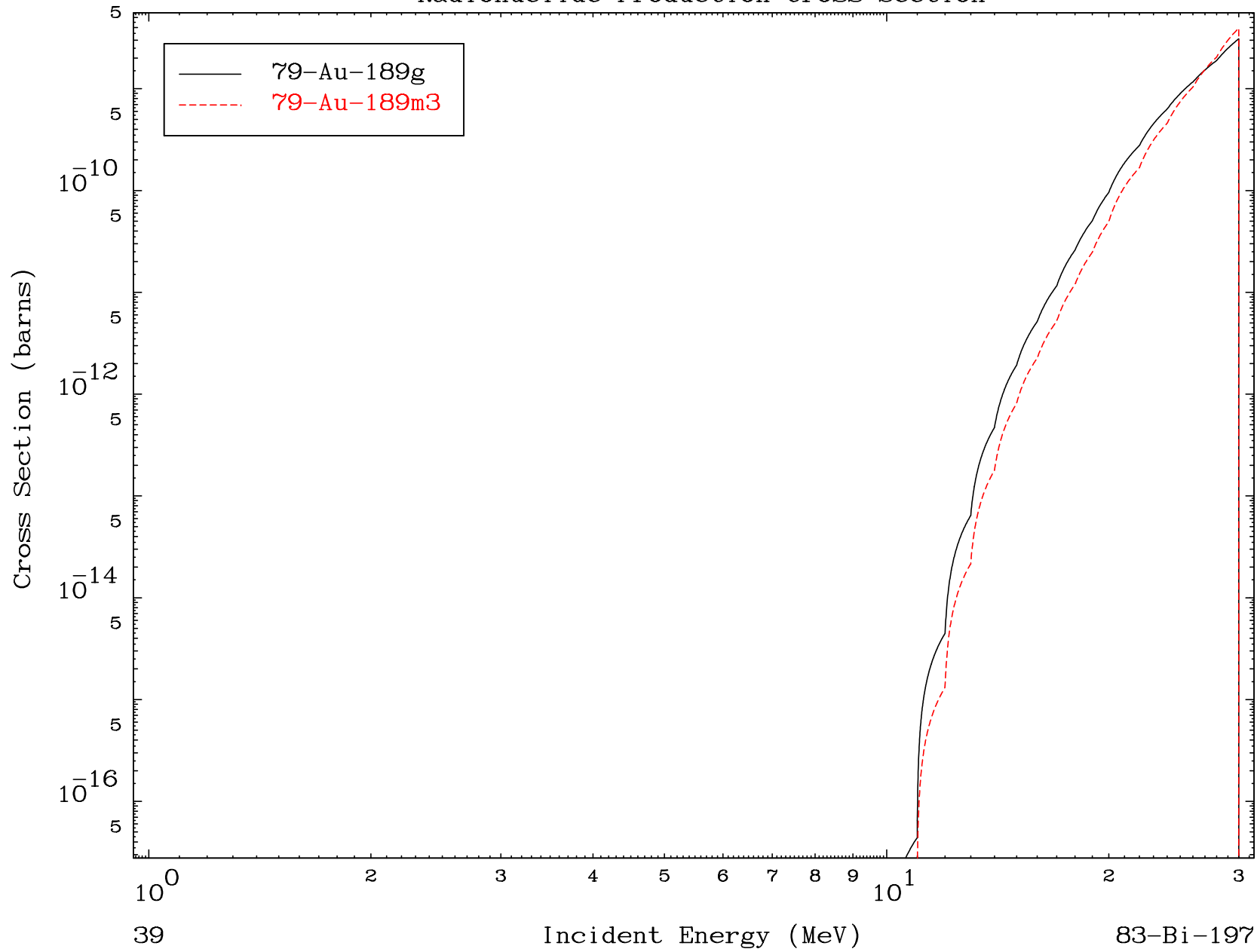


MAT 8290

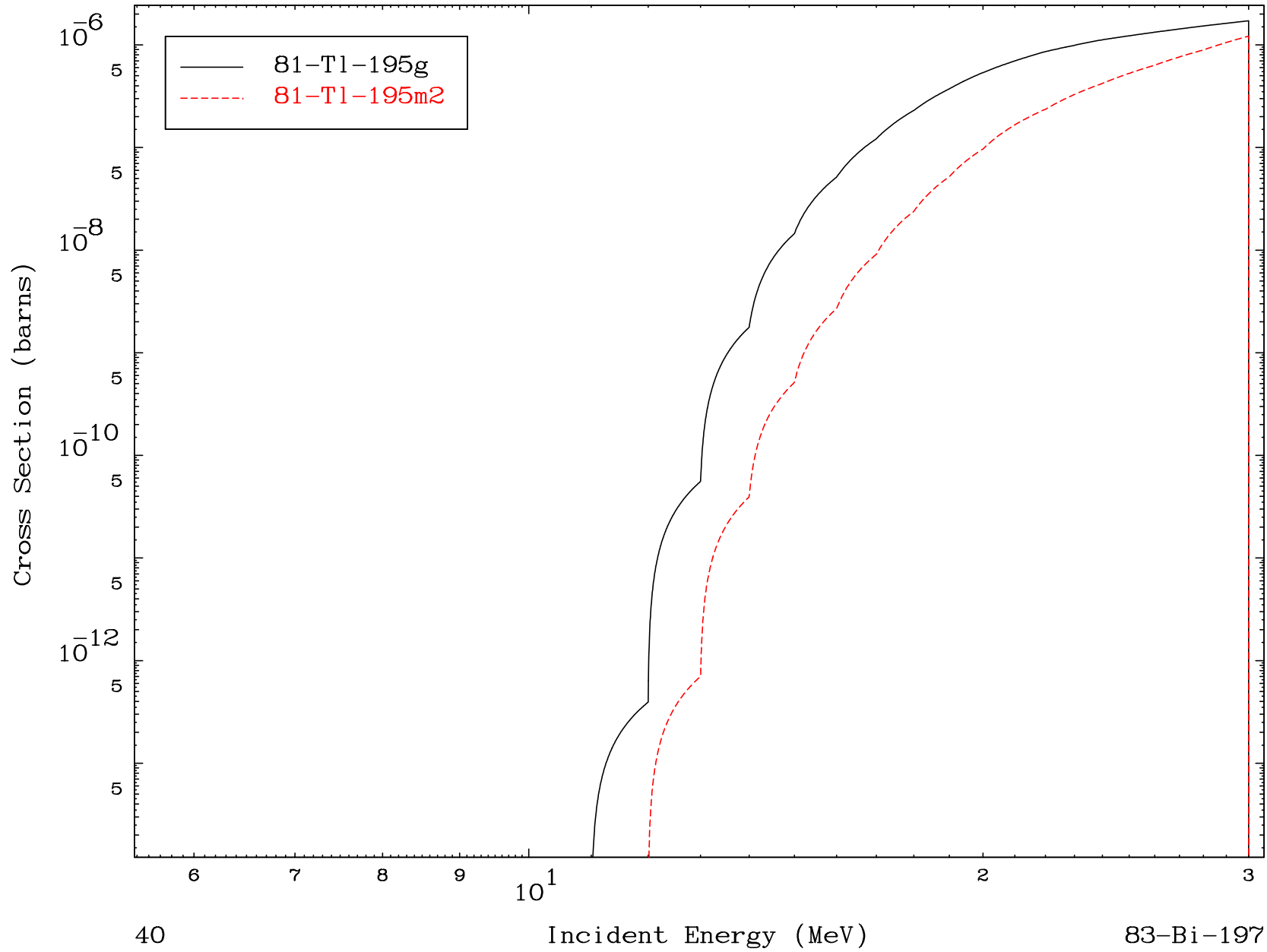
( $\gamma, 2\alpha$ )

83-Bi-197

### Radionuclide Production Cross Section



Radionuclide Production Cross Section





Radionuclide Production Cross Section

