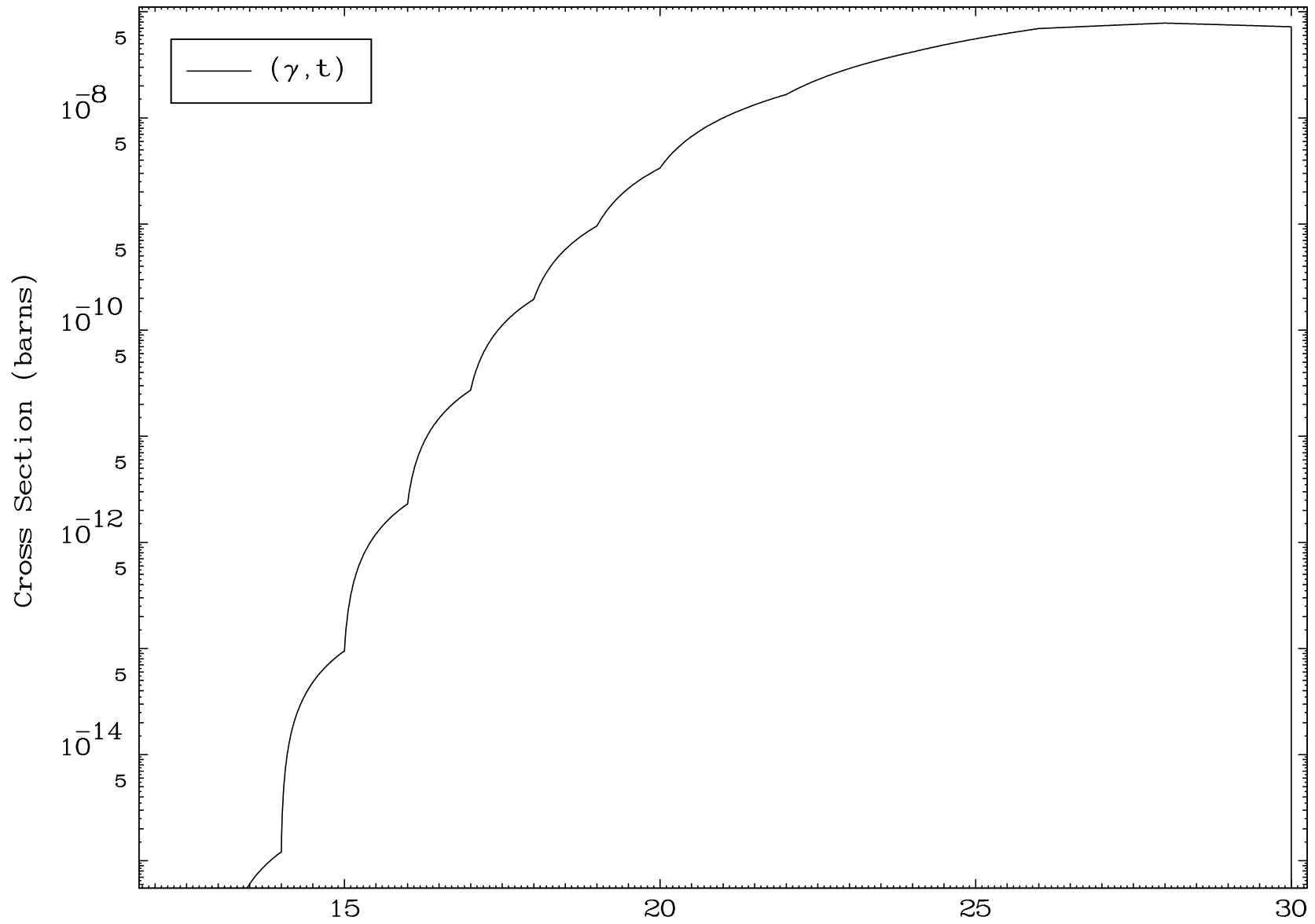


MAT 9873

( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

98-Cf-256

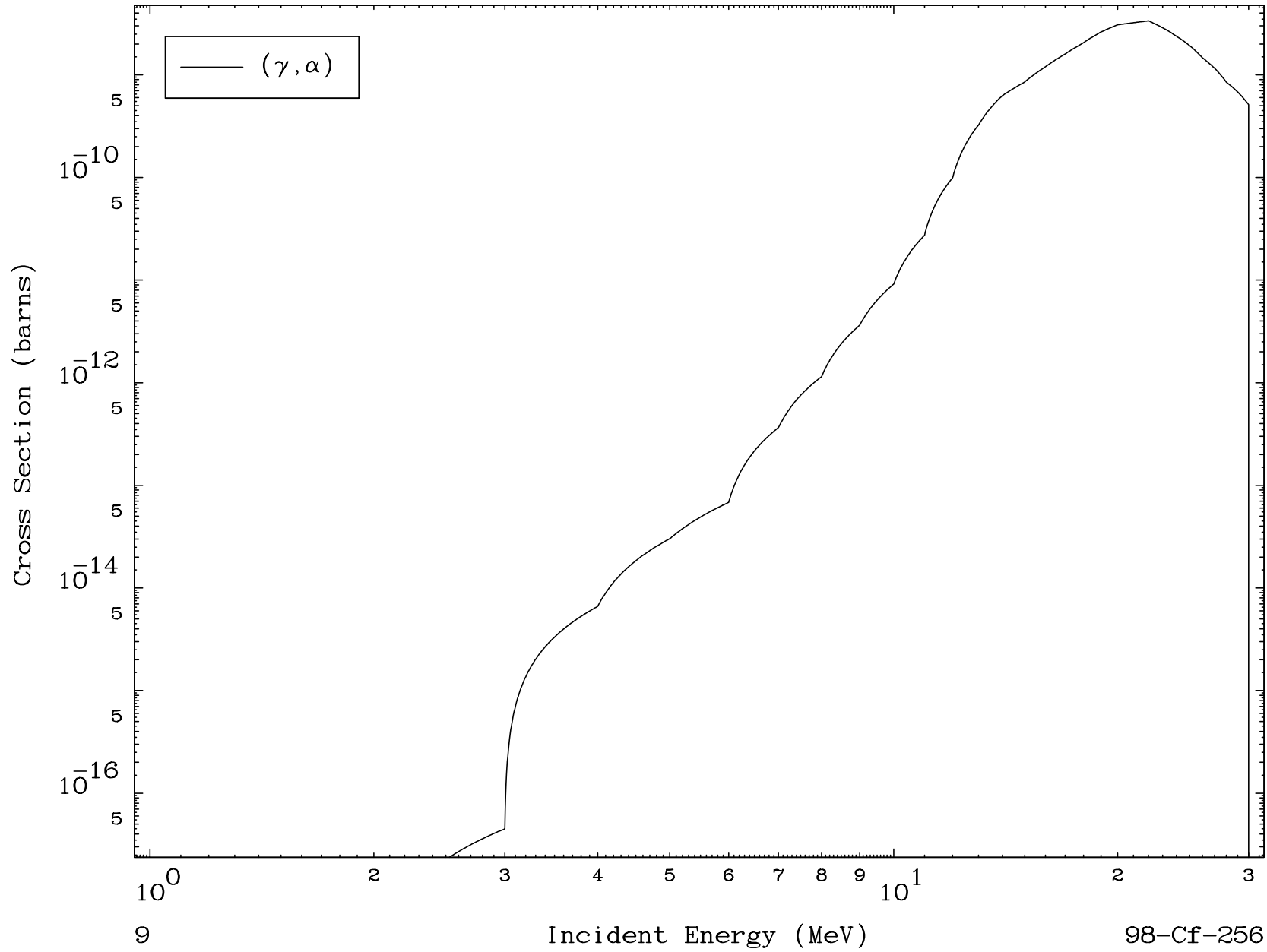


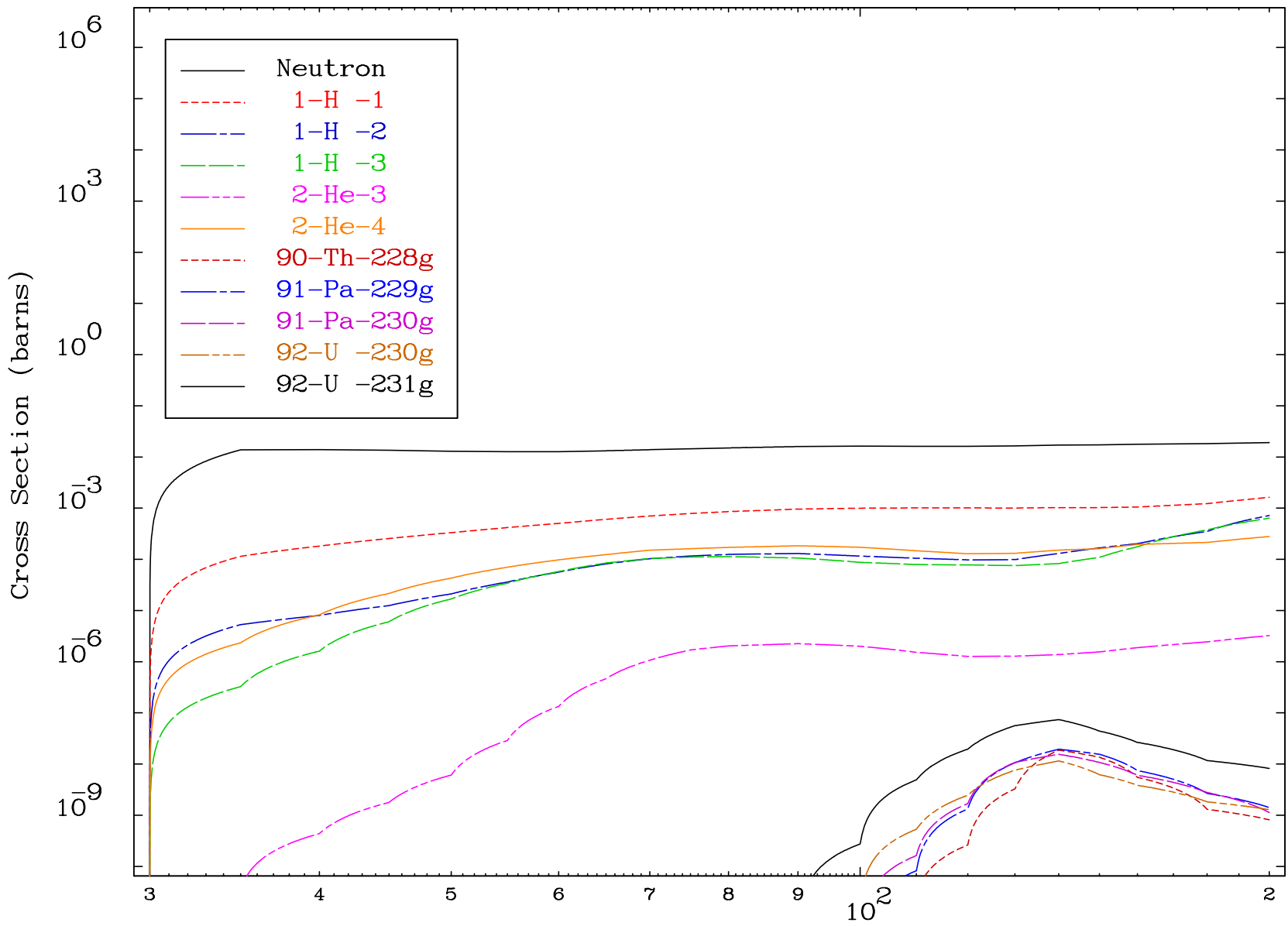
8

Incident Energy (MeV)

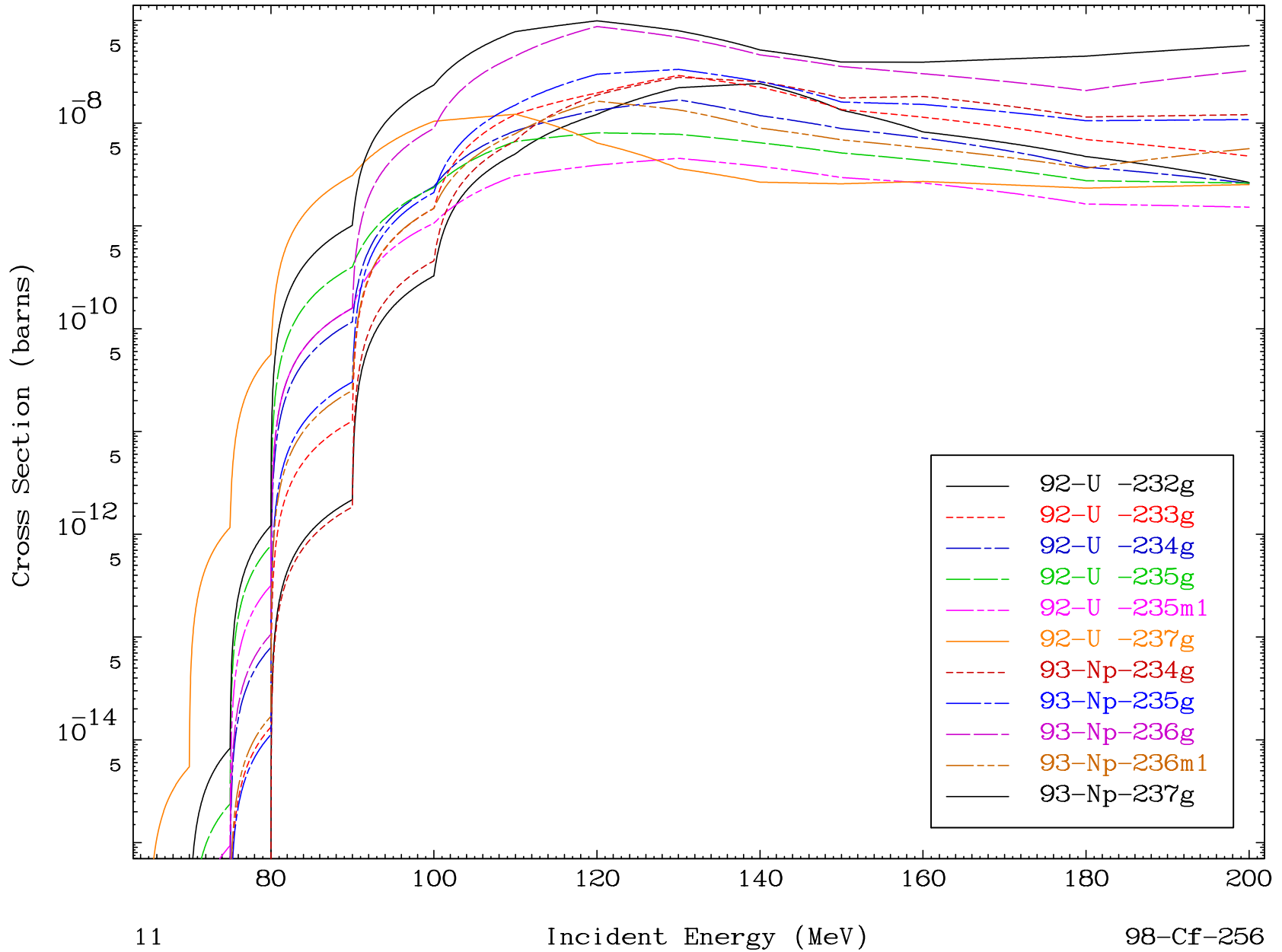
98-Cf-256



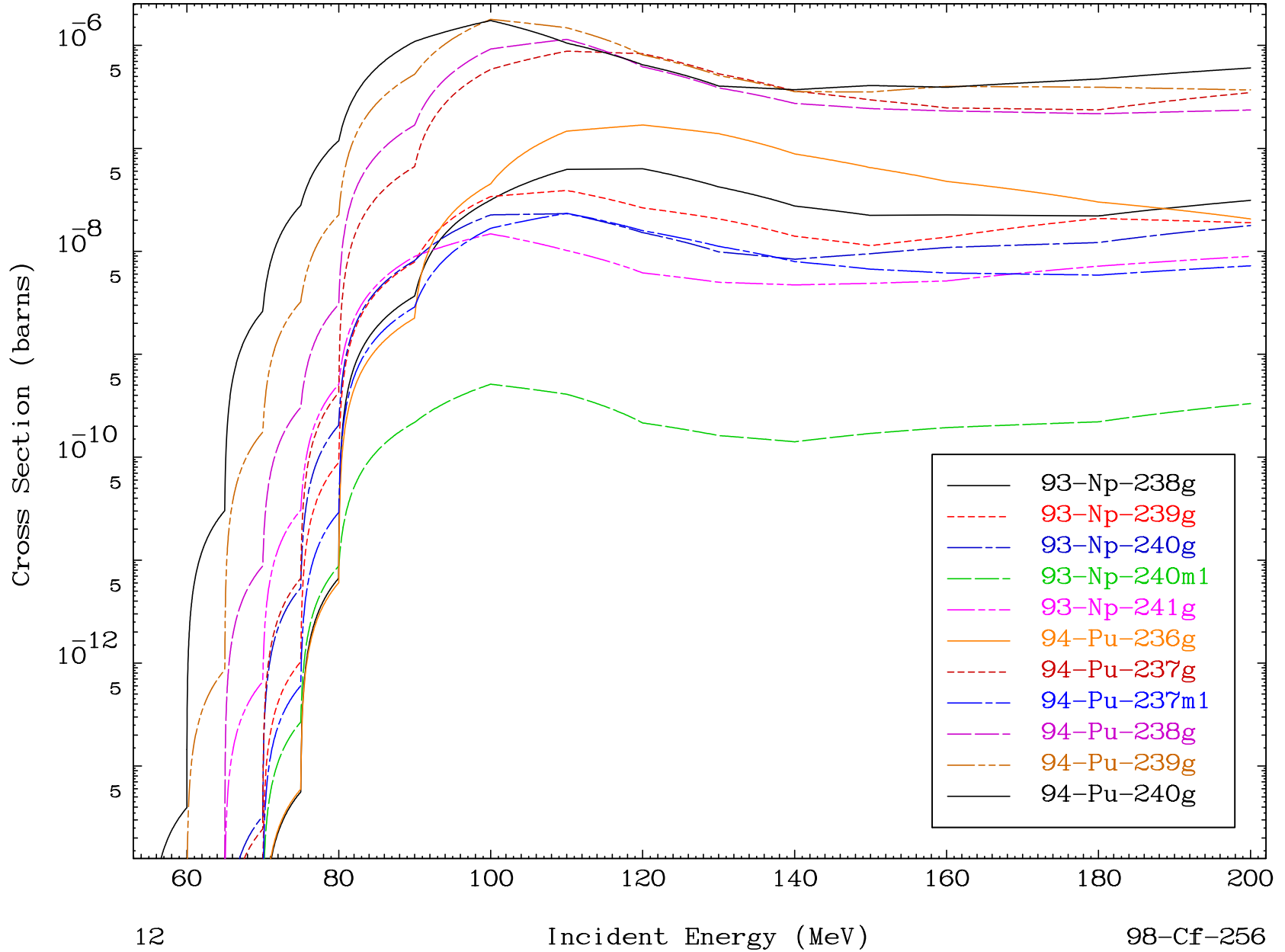




Radionuclide Production Cross Section



Radionuclide Production Cross Section

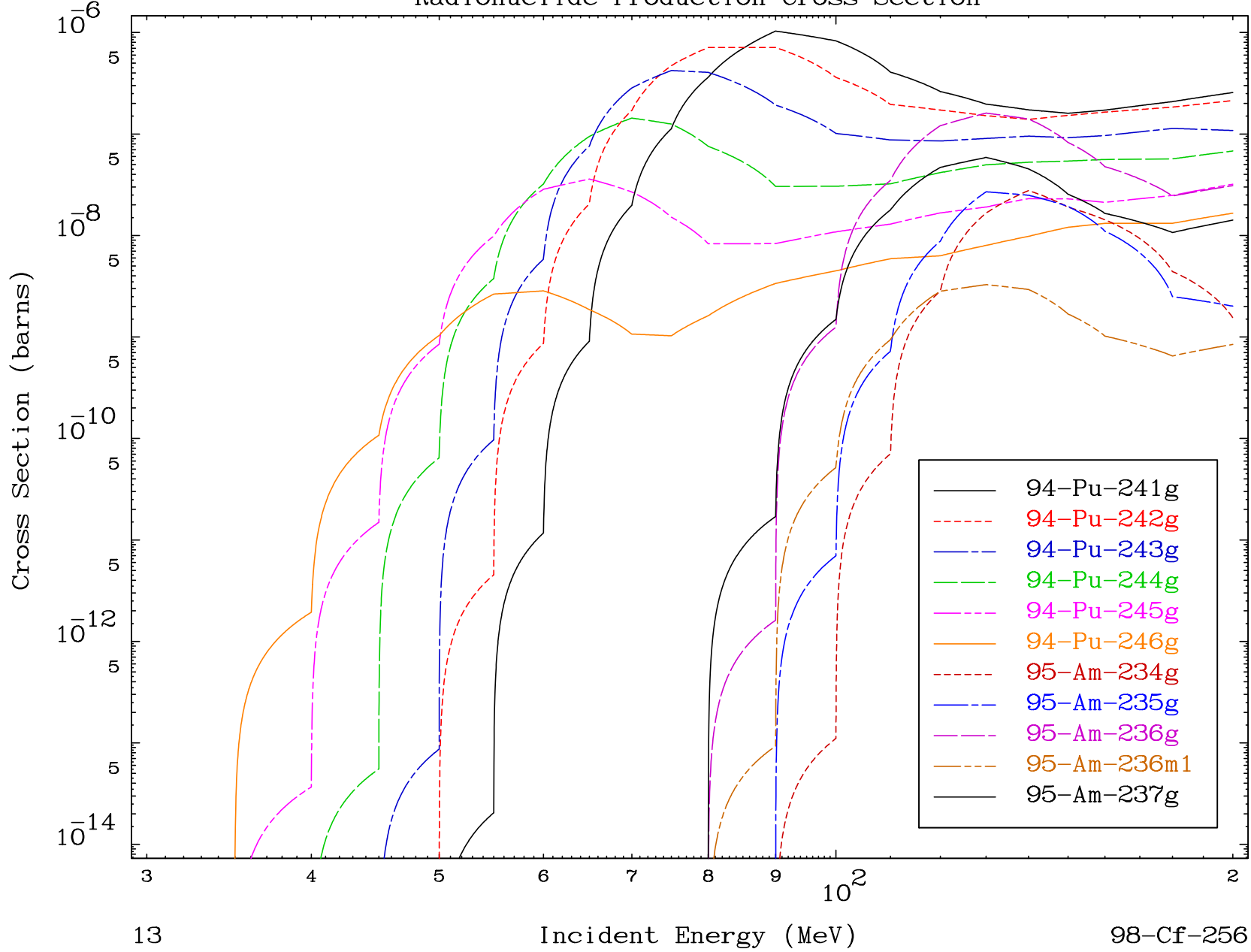


MAT 9873

( $\gamma$ , remainder)

98-Cf-256

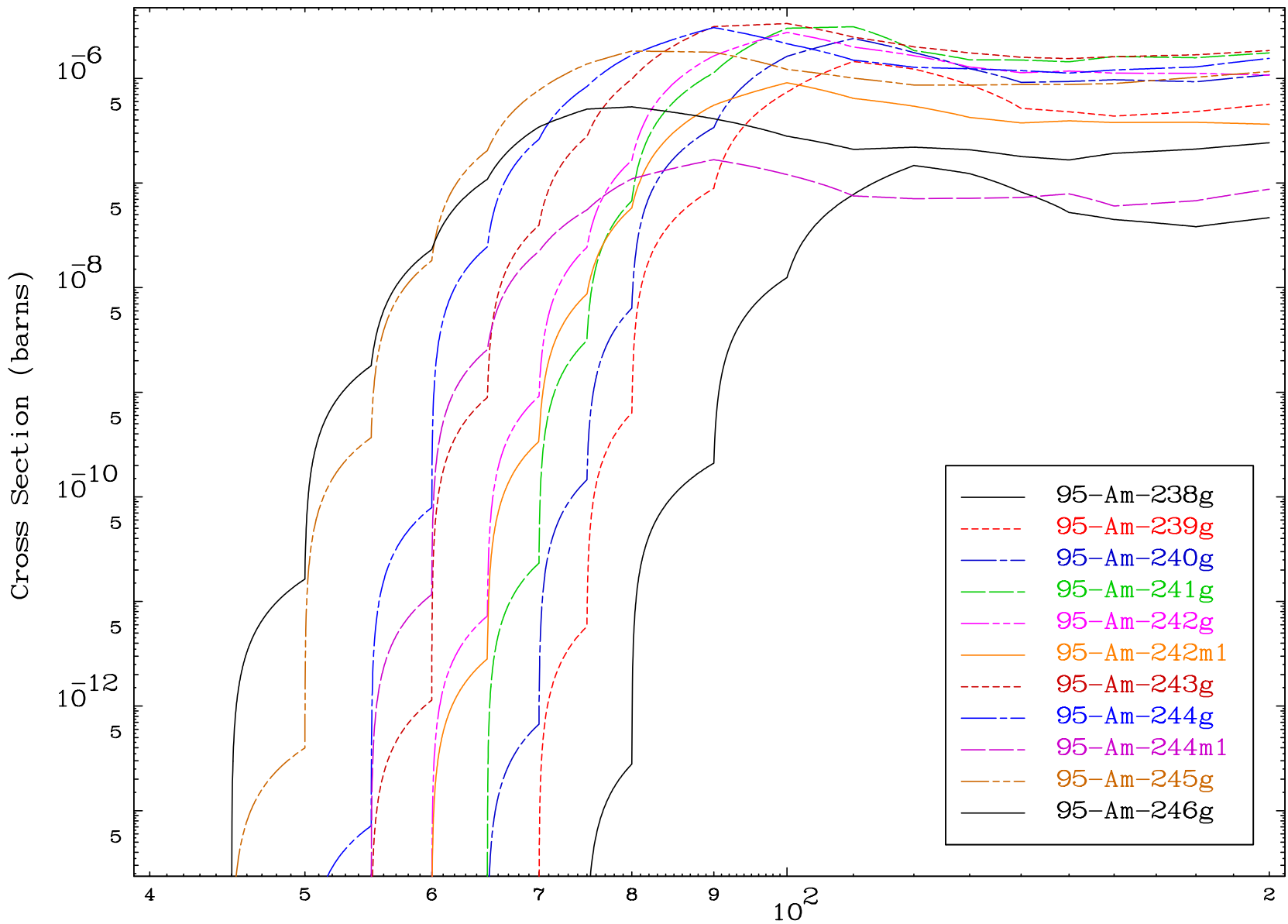
### Radionuclide Production Cross Section

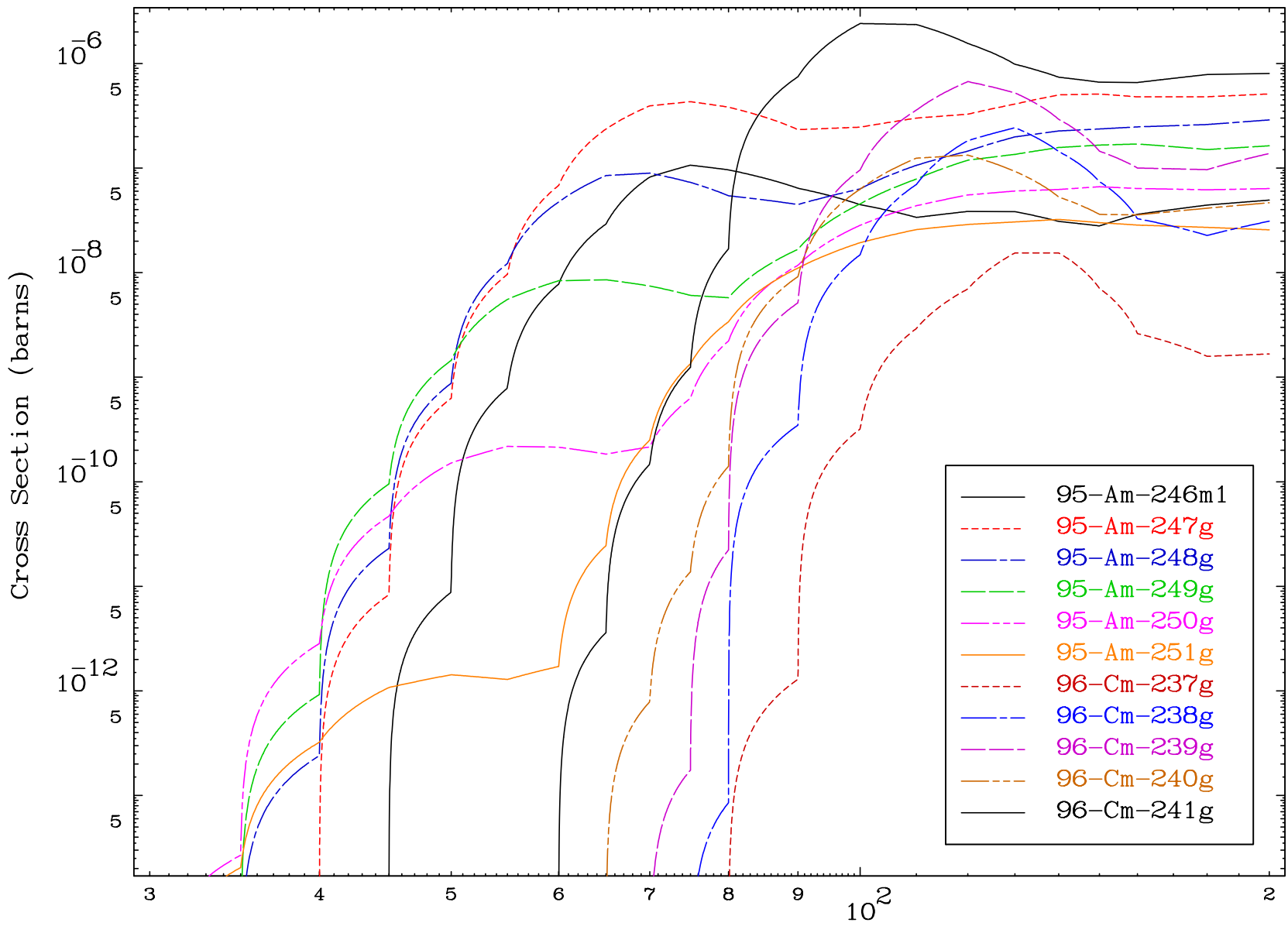


13

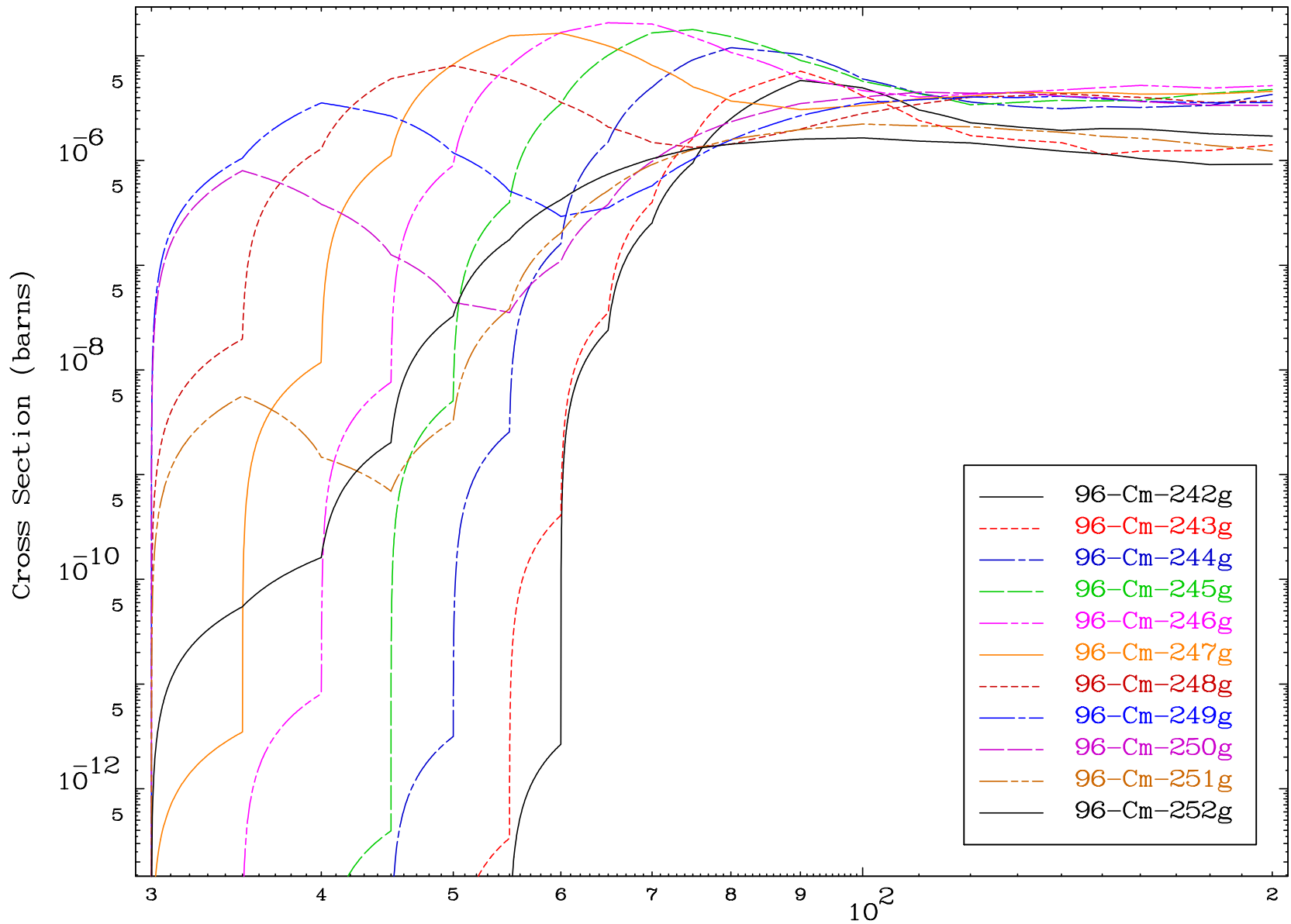
Incident Energy (MeV)

98-Cf-256

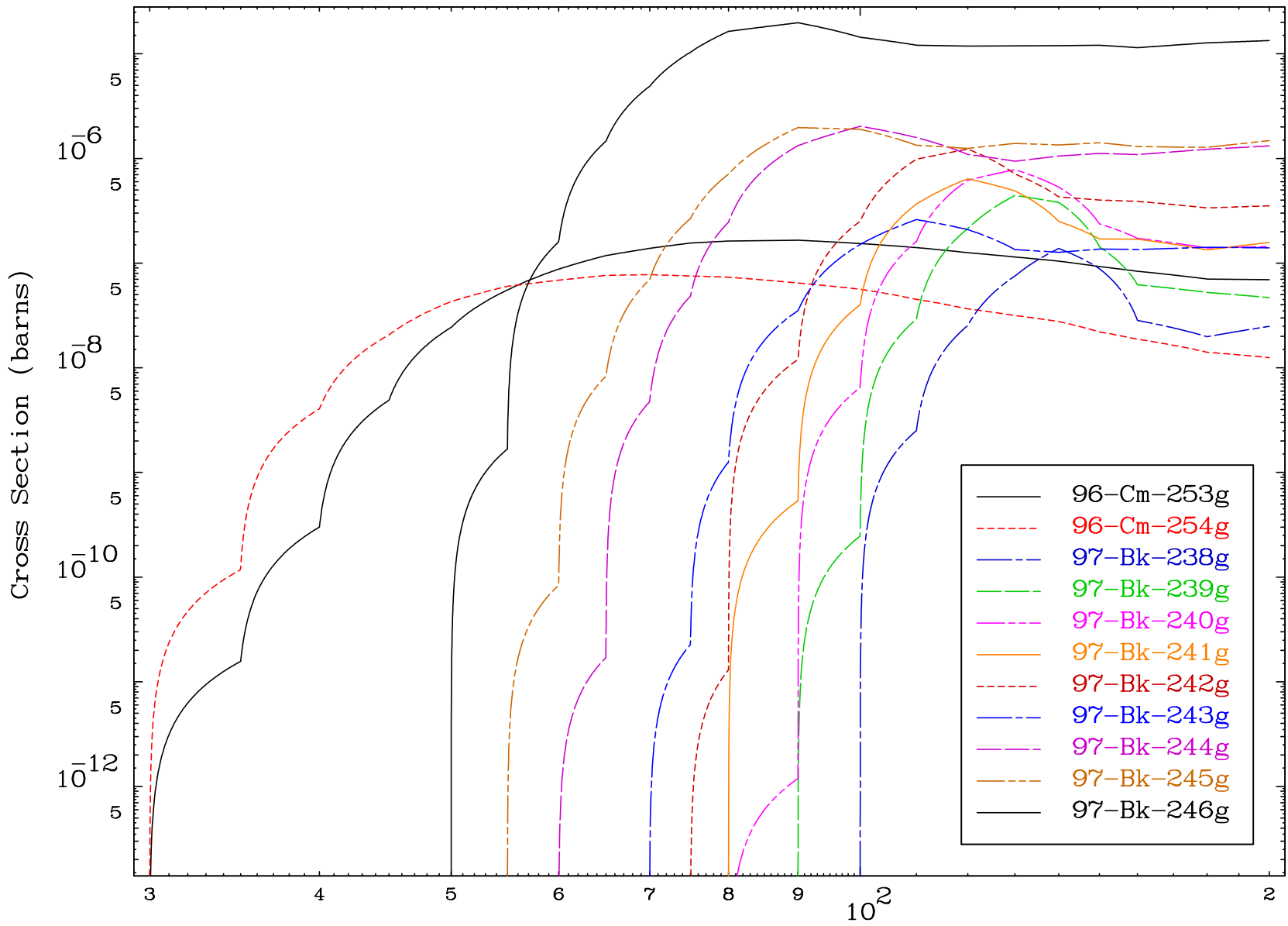


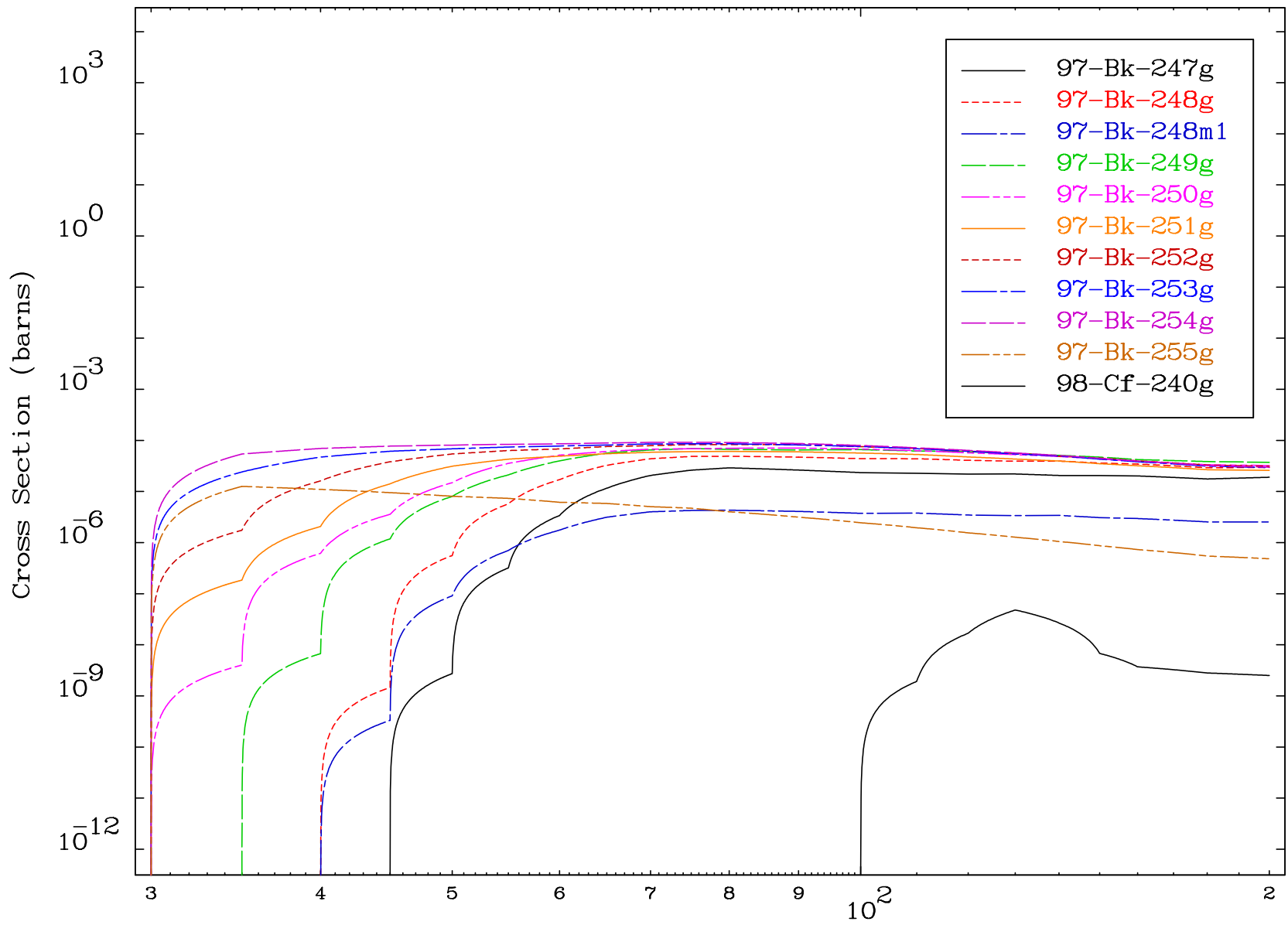


Radionuclide Production Cross Section

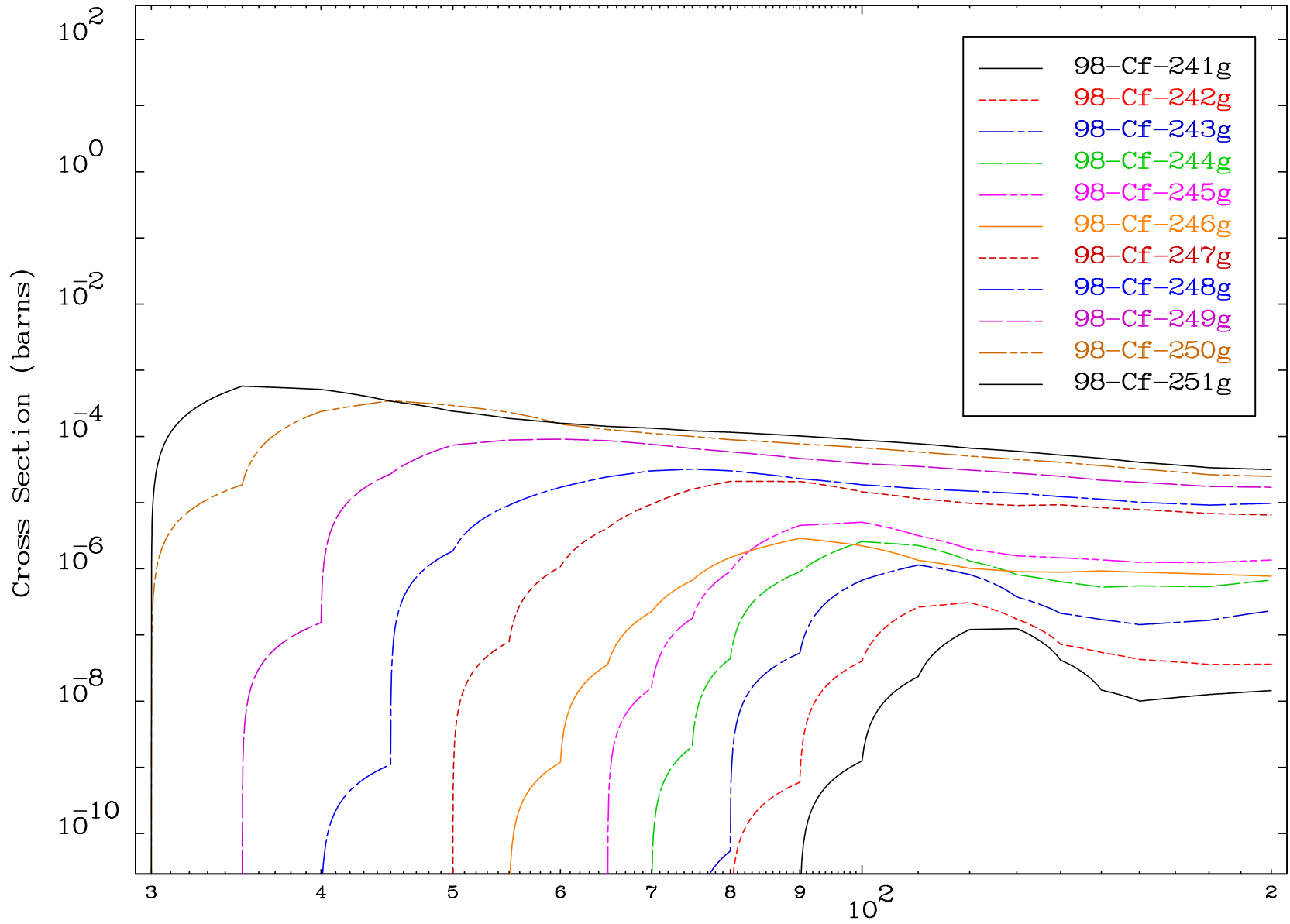








Radionuclide Production Cross Section



MAT 9873

( $\gamma$ , remainder)

98-Cf-256

Radionuclide Production Cross Section

