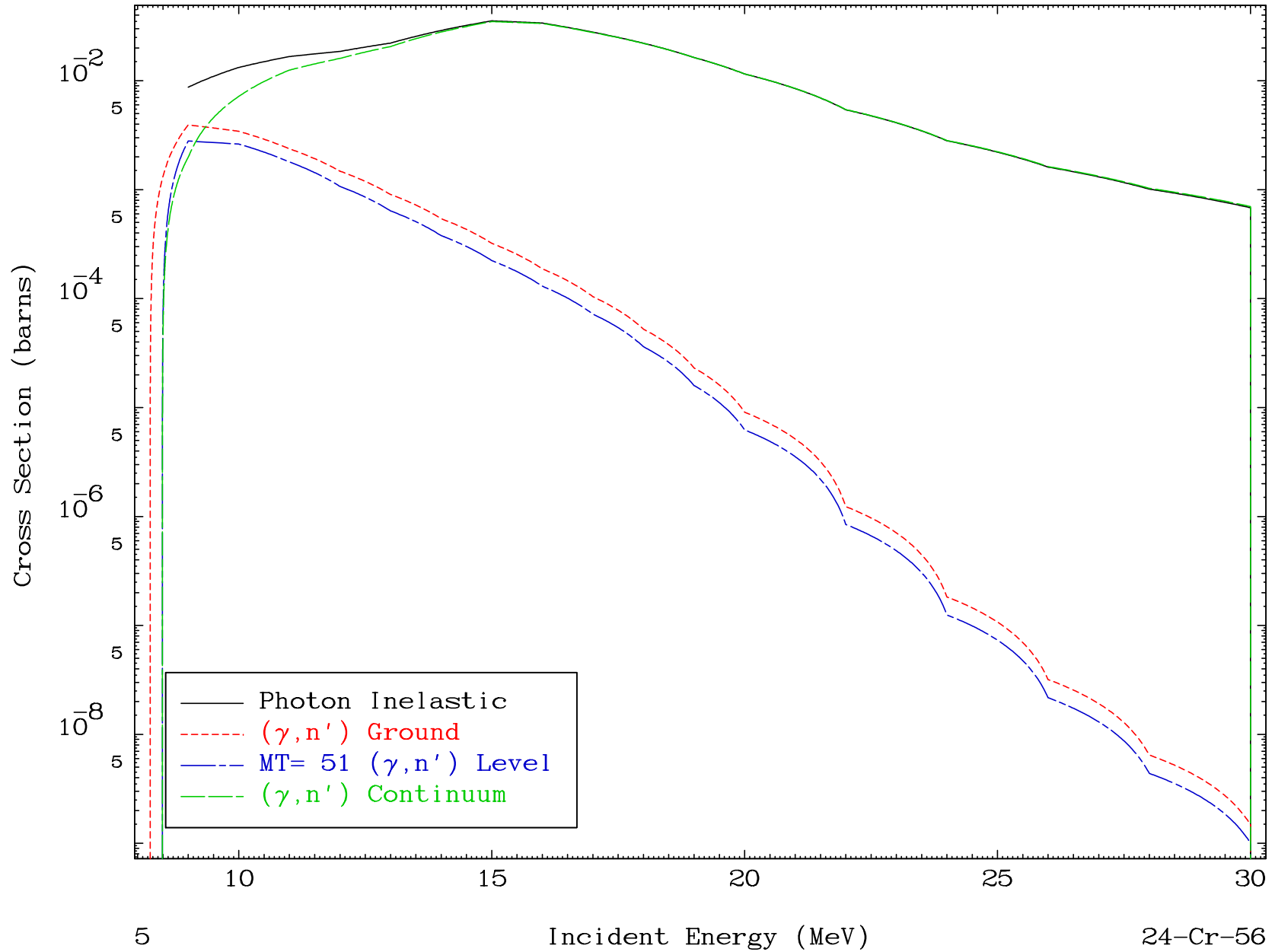
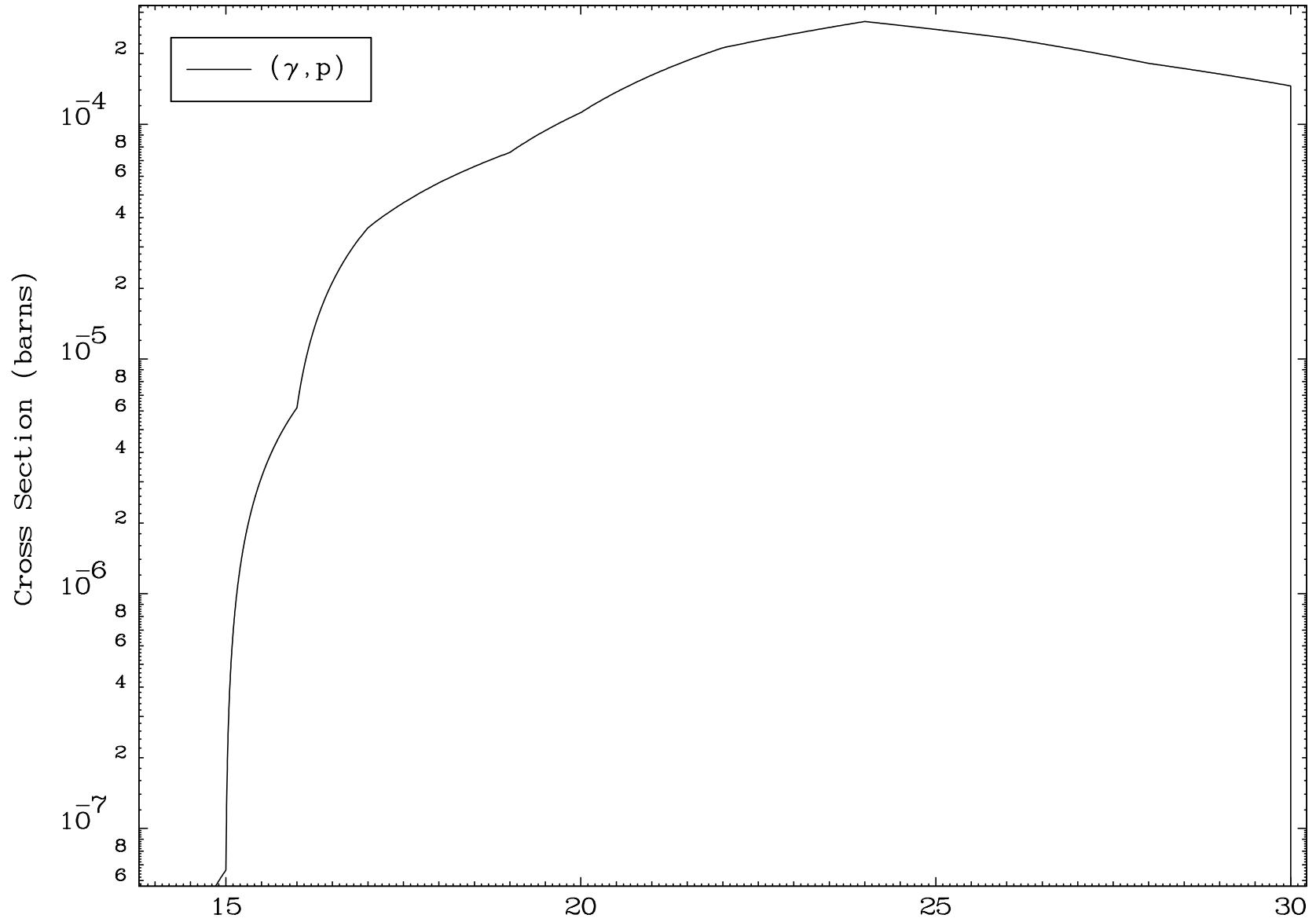


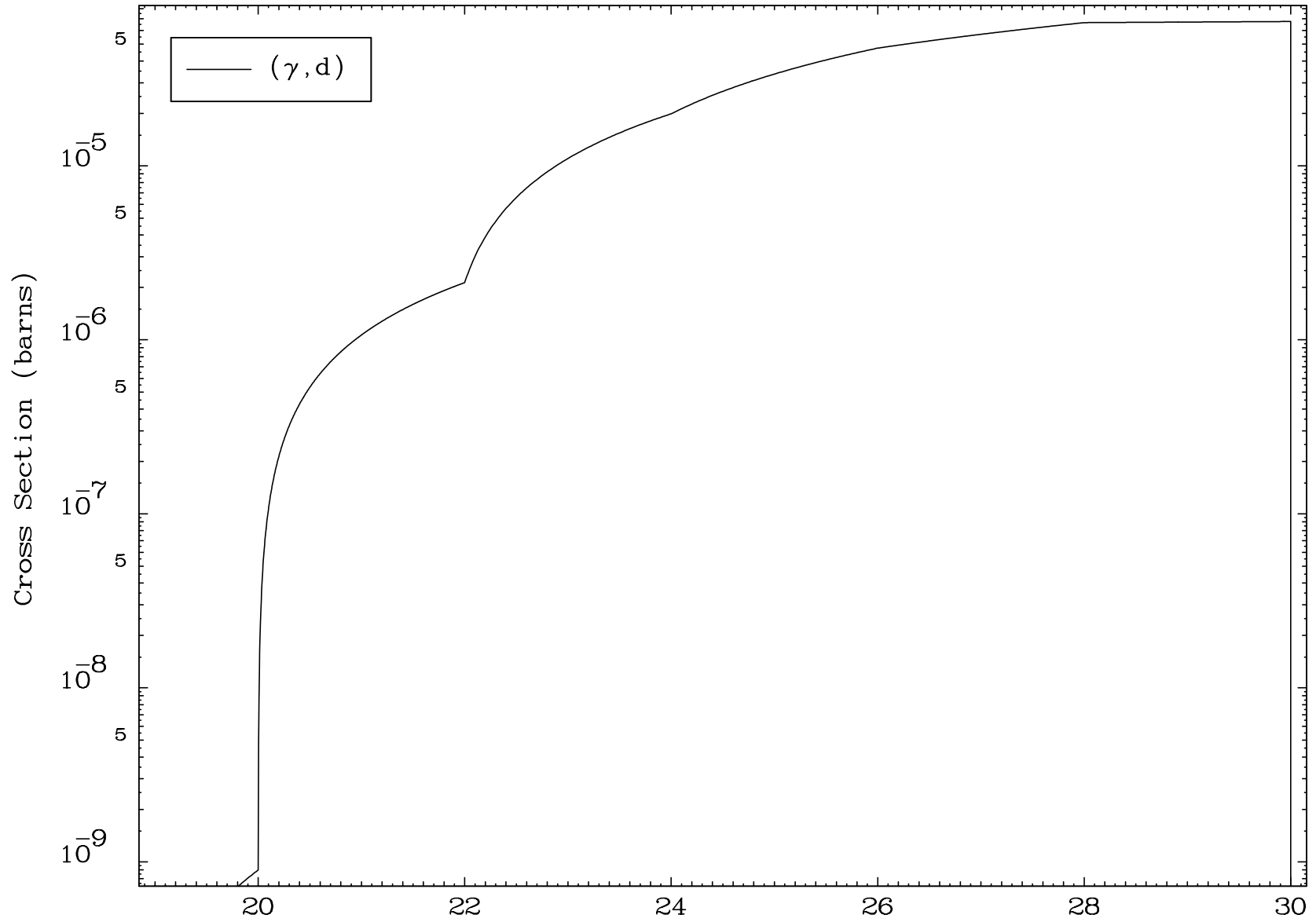
MAT 2443

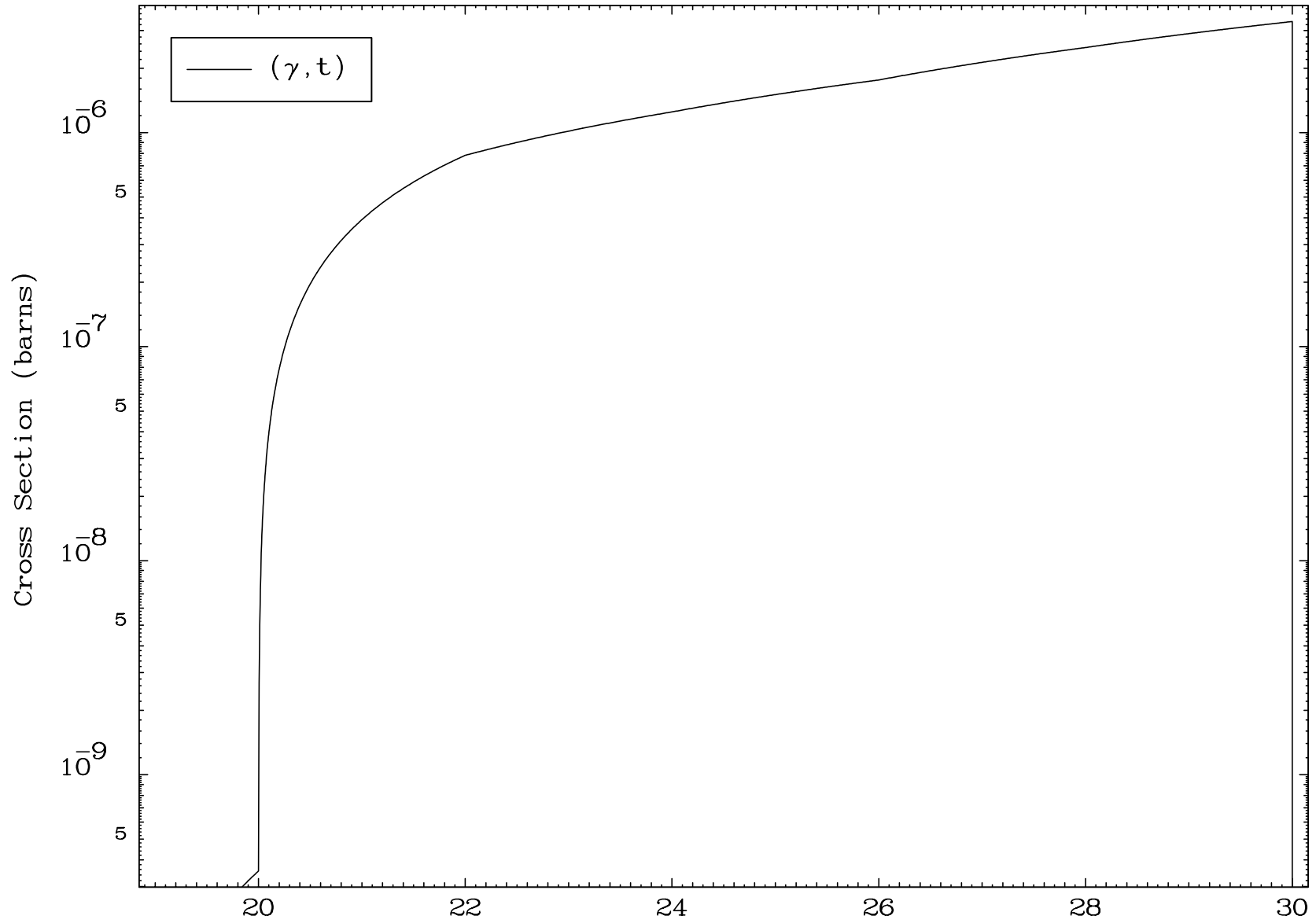
( $\gamma, n'$ ) Level  
0 Kelvin Cross Sections

24-Cr-56







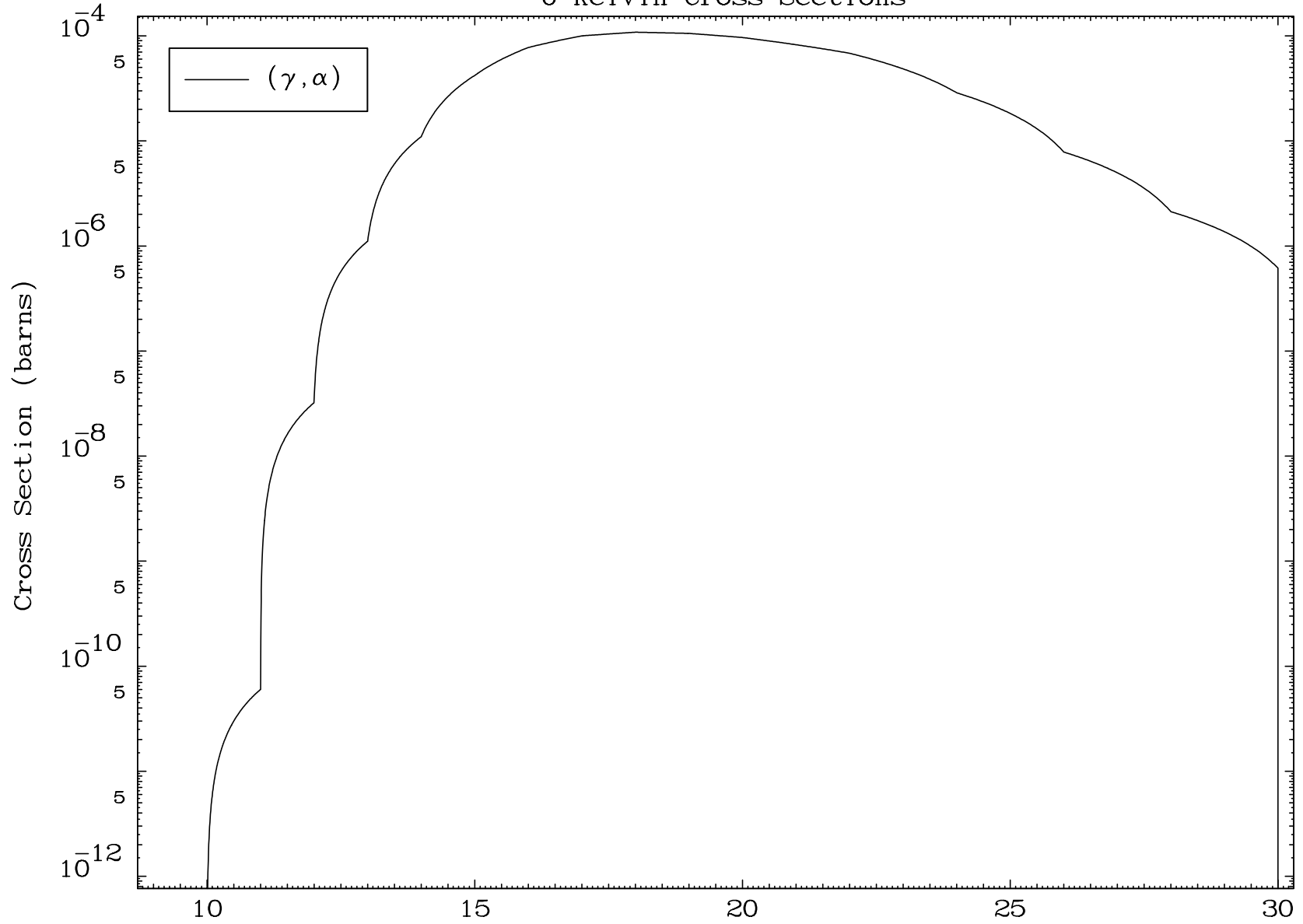




MAT 2443

$(\gamma, \alpha)$  Levels  
0 Kelvin Cross Sections

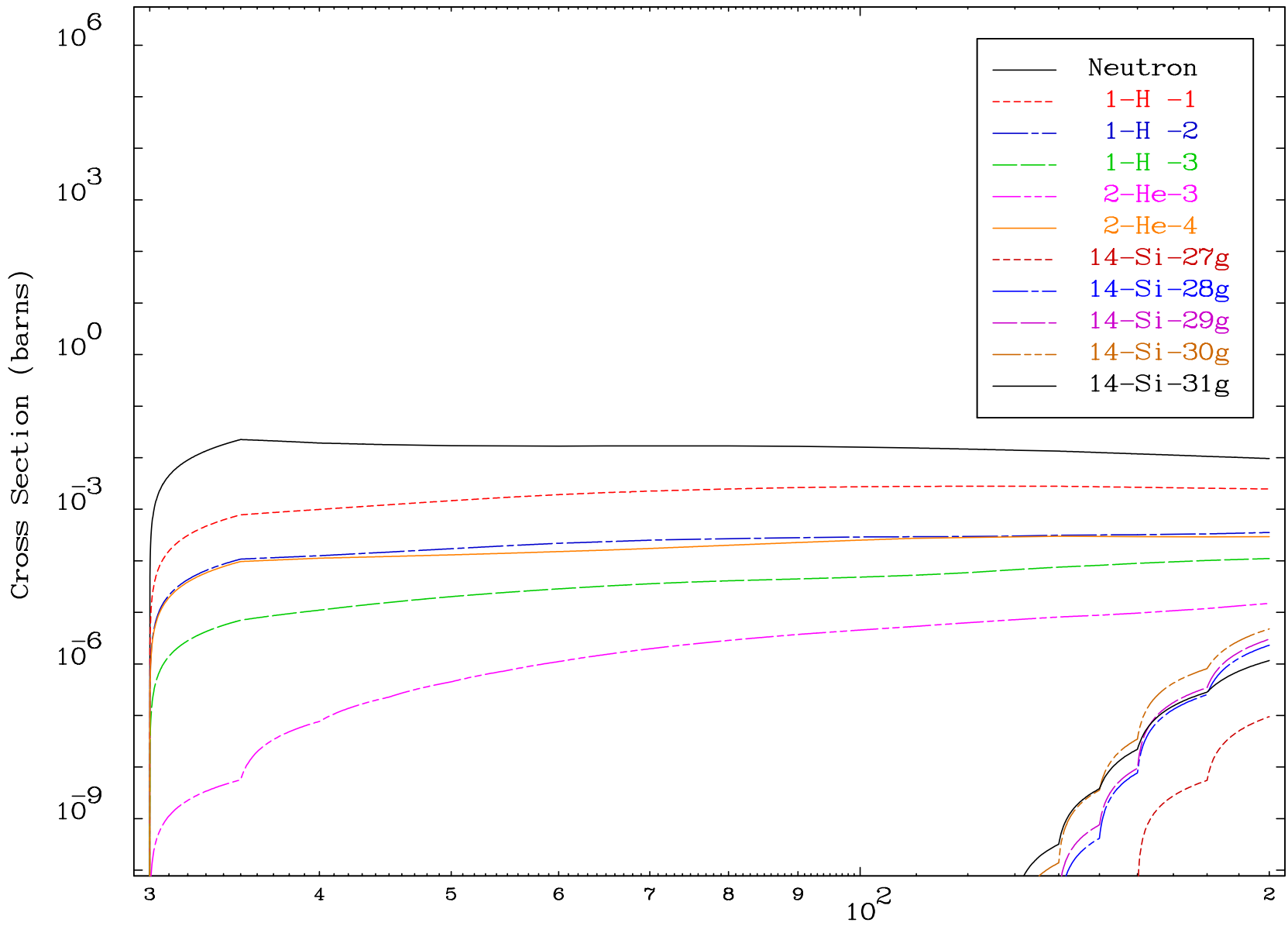
24-Cr-56

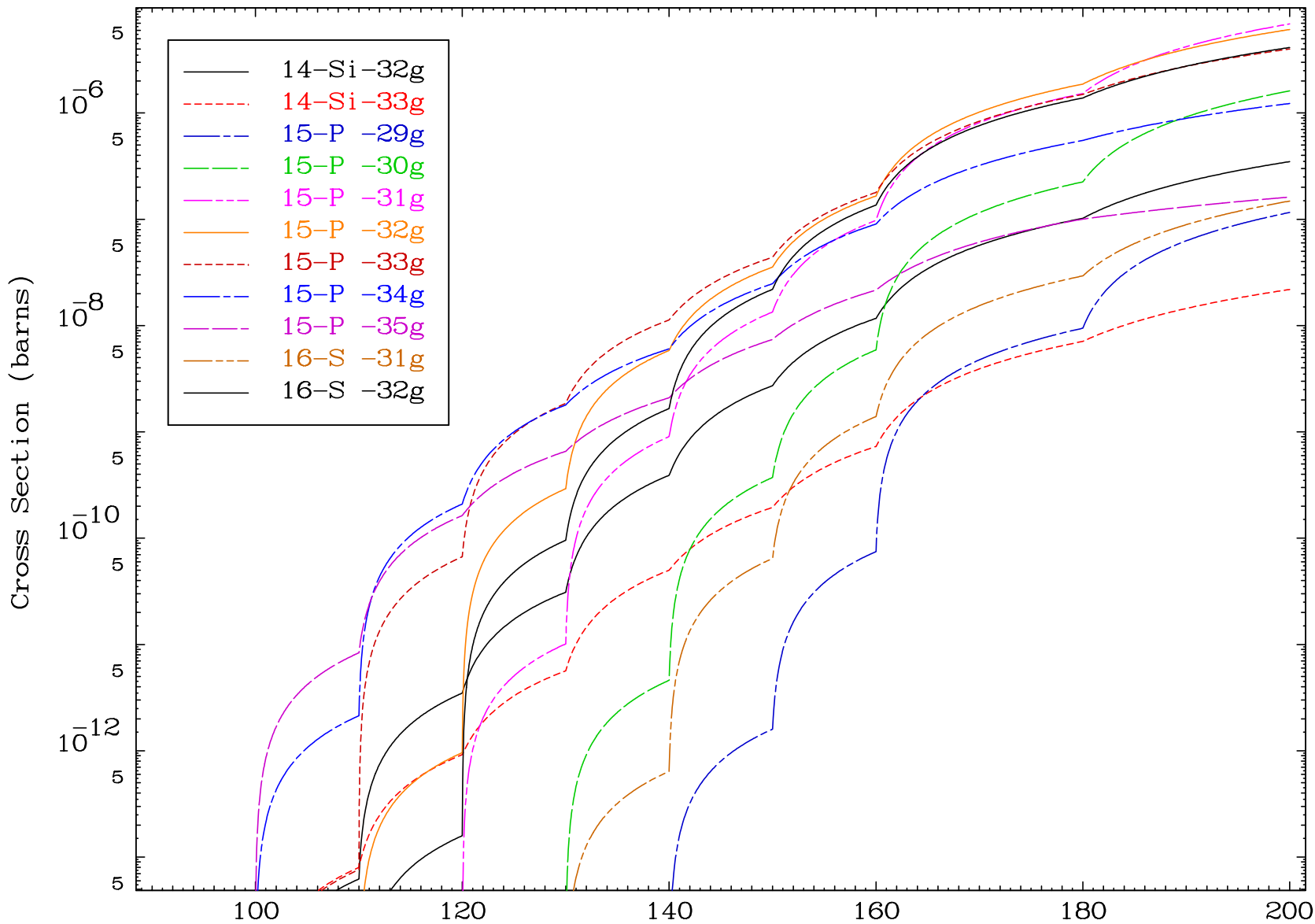


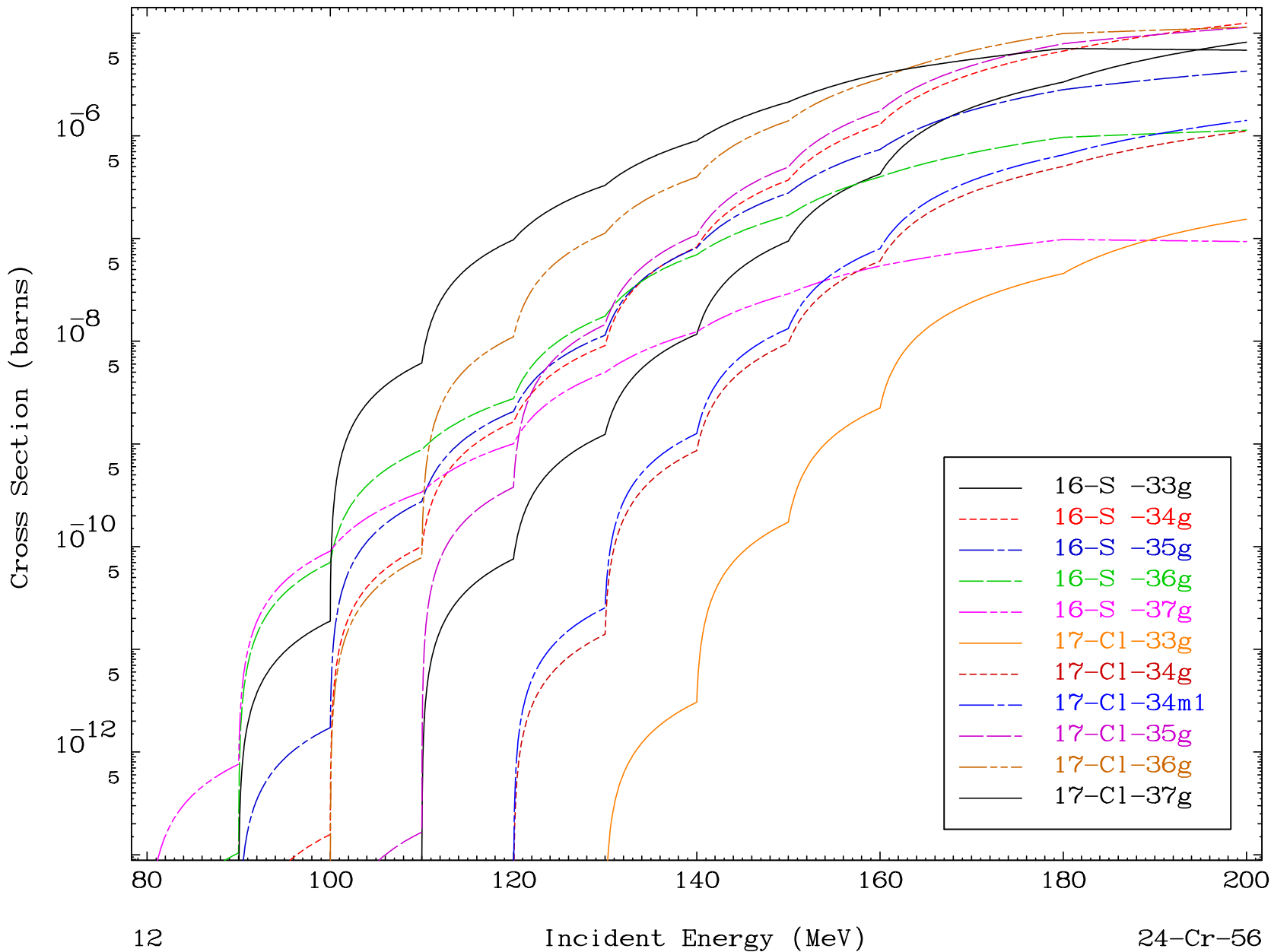
9

Incident Energy (MeV)

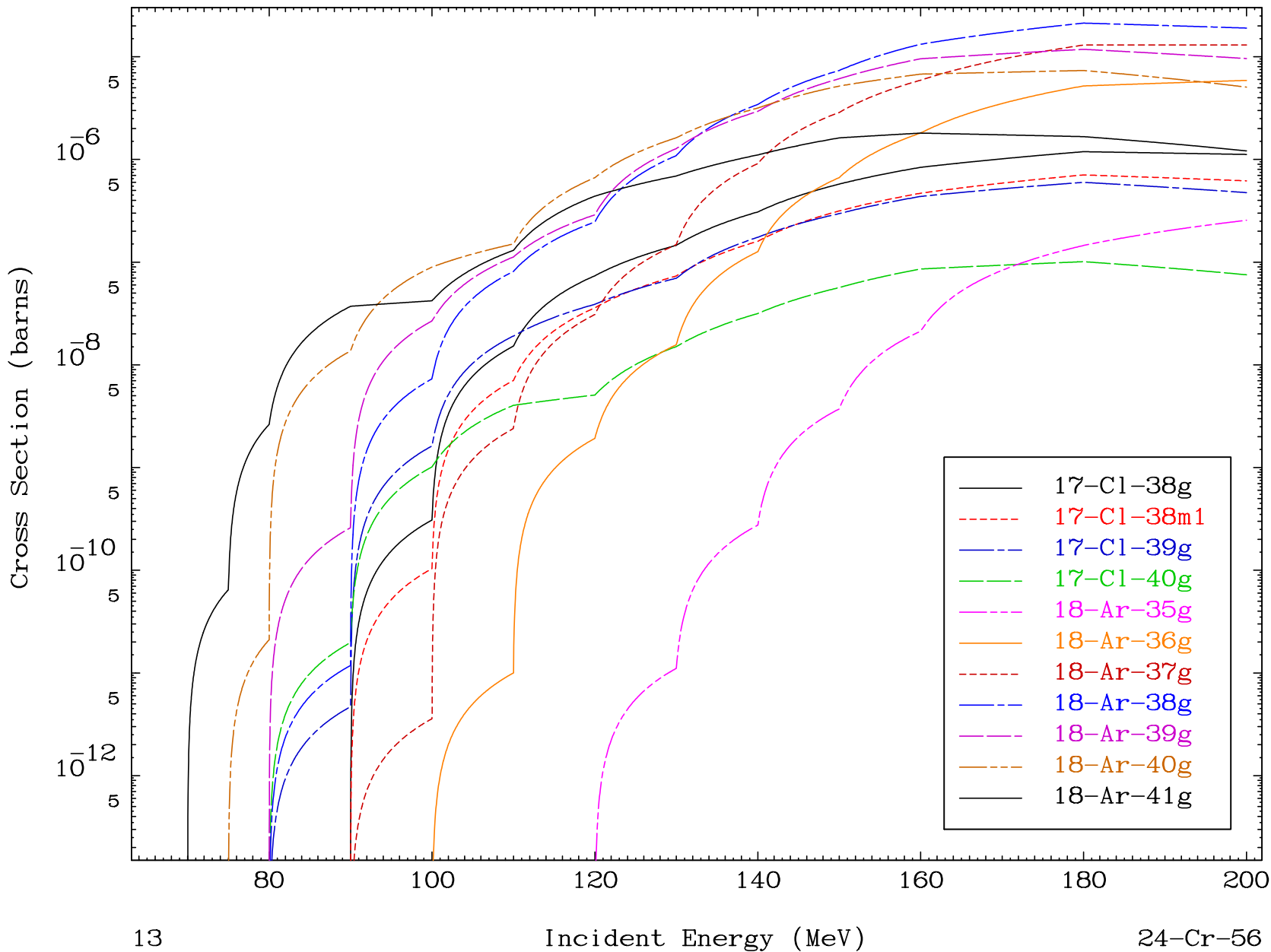
24-Cr-56







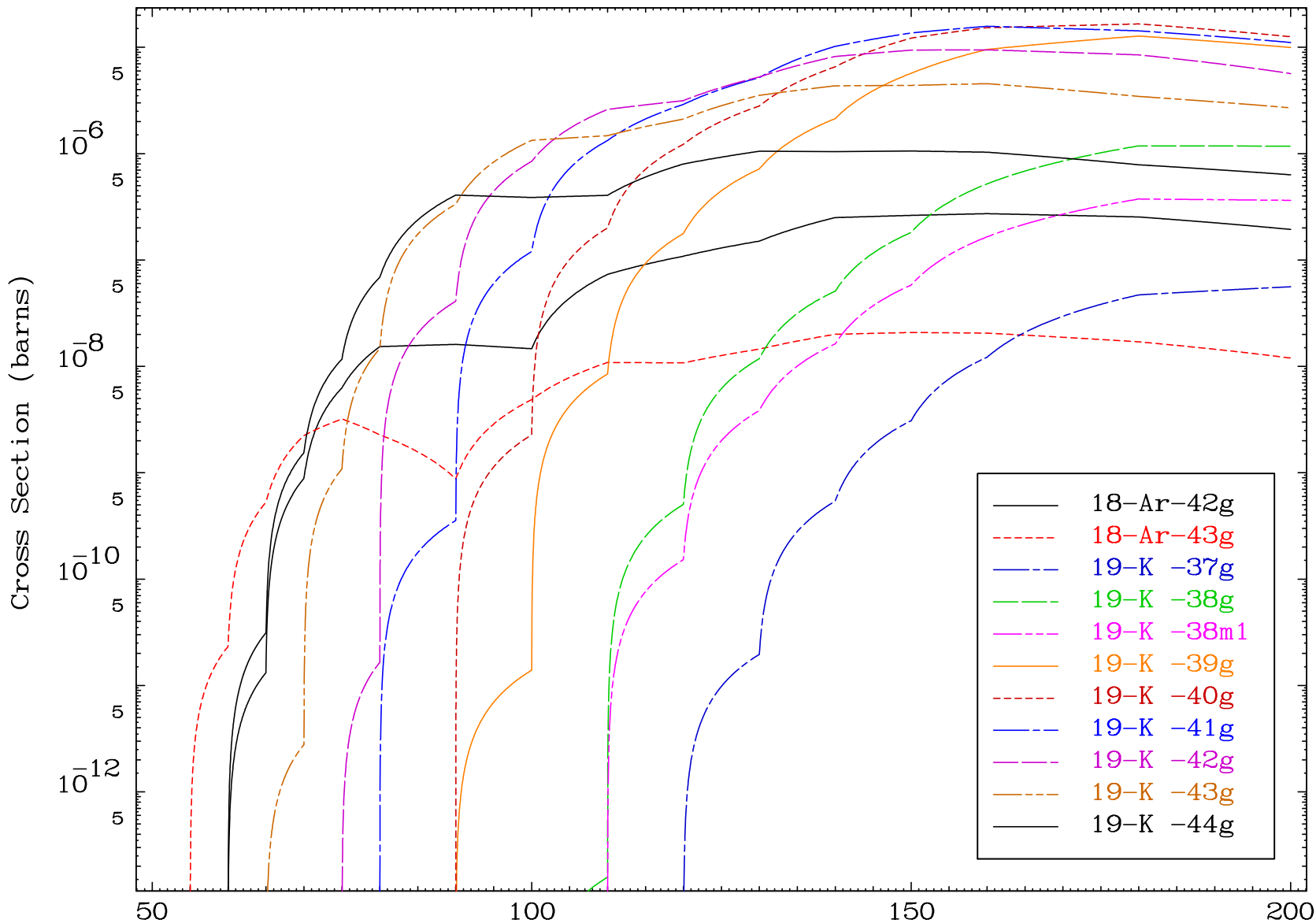
Radionuclide Production Cross Section



MAT 2443

( $\gamma$ , remainder)  
Radionuclide Production Cross Section

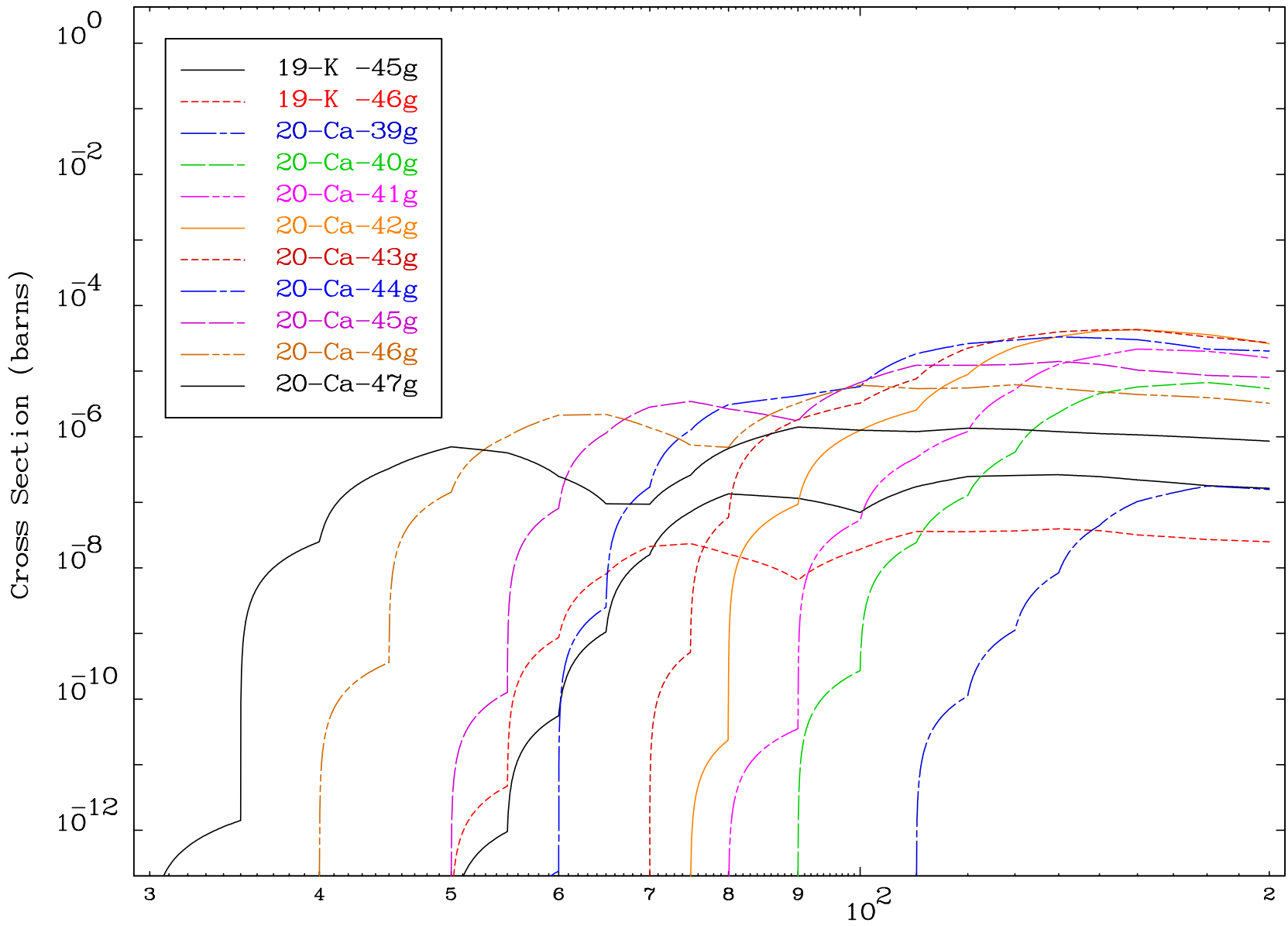
24-Cr-56



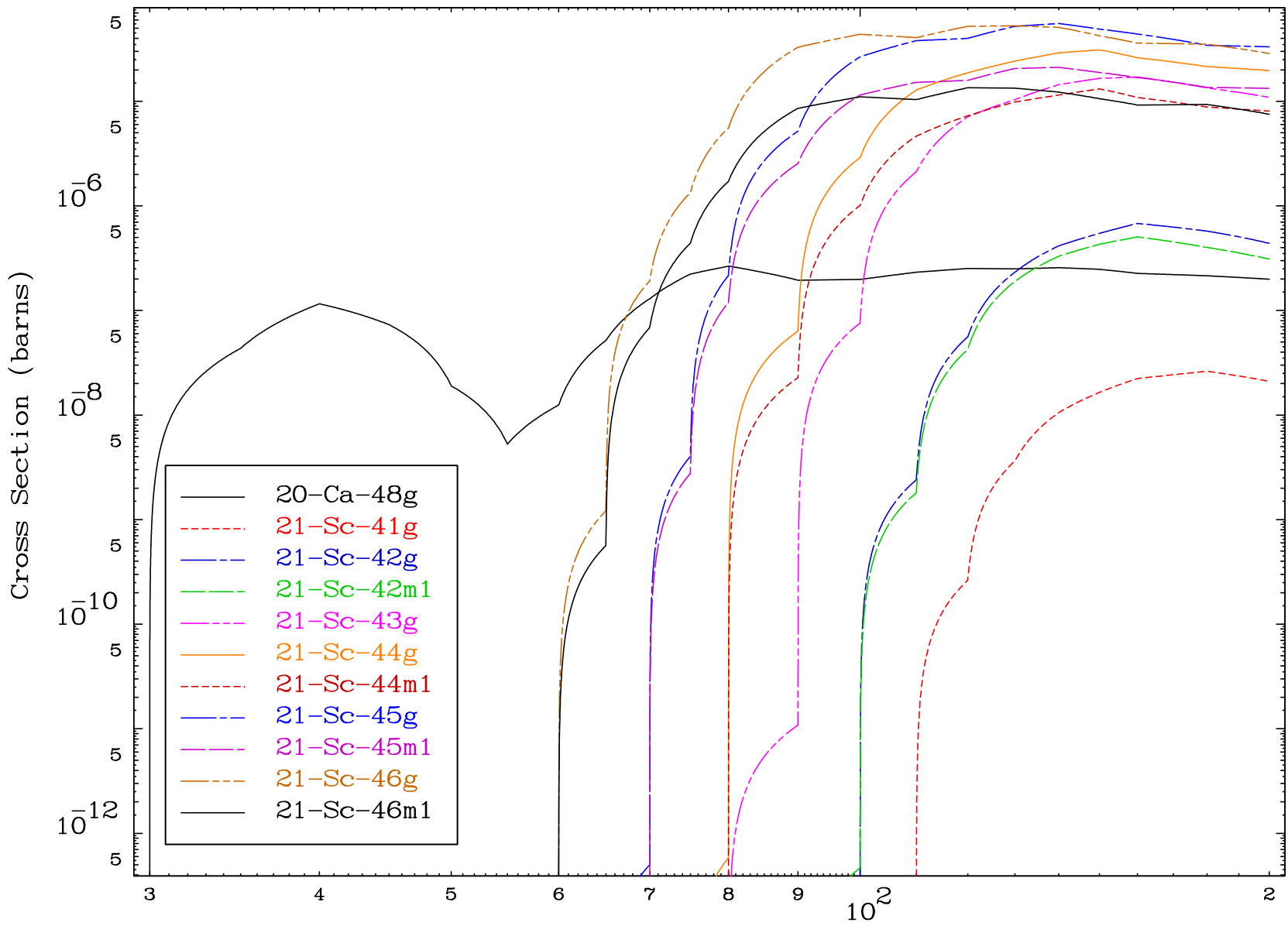
14

Incident Energy (MeV)

24-Cr-56



Radionuclide Production Cross Section

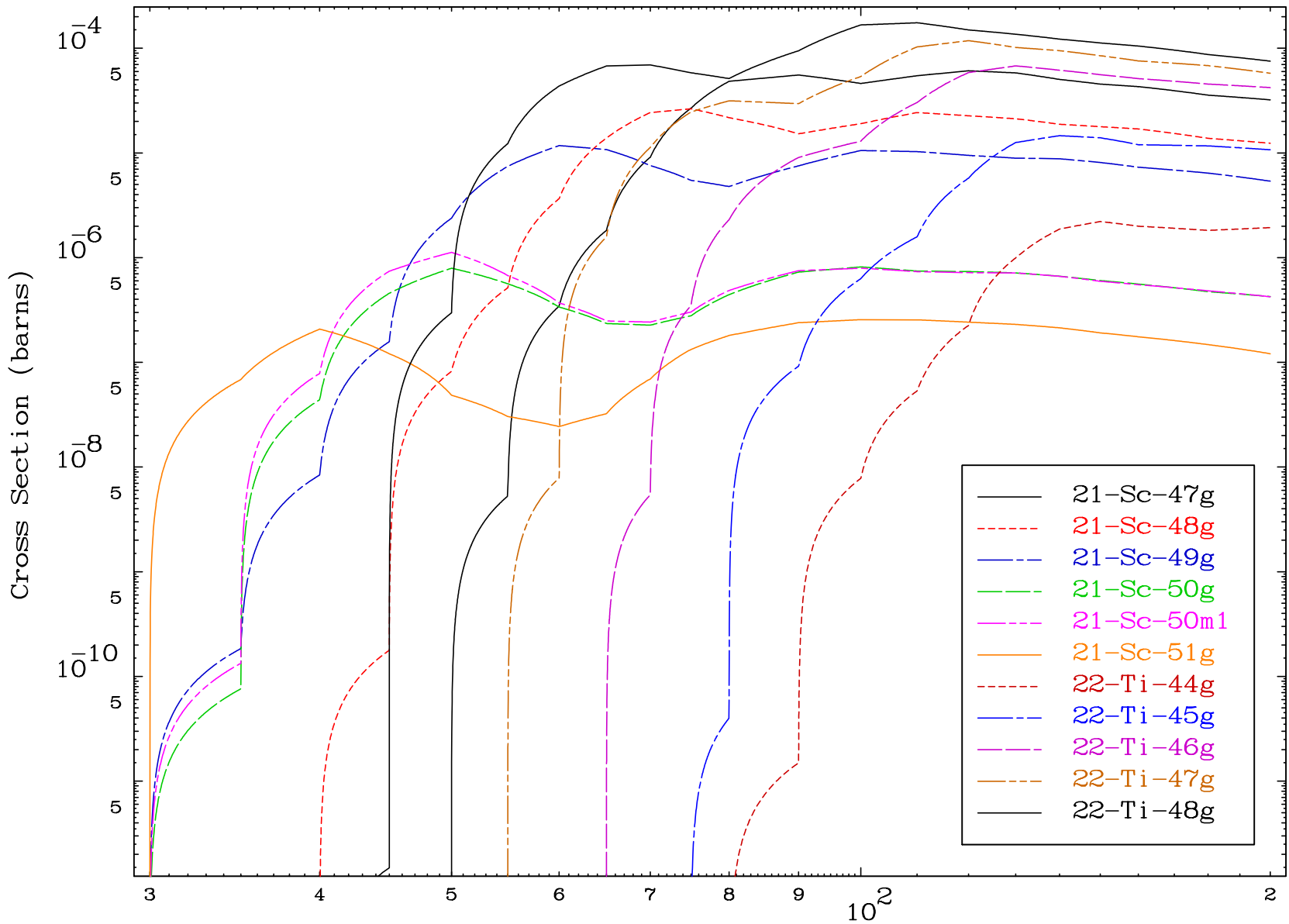




MAT 2443

( $\gamma$ , remainder)  
Radionuclide Production Cross Section

24-Cr-56



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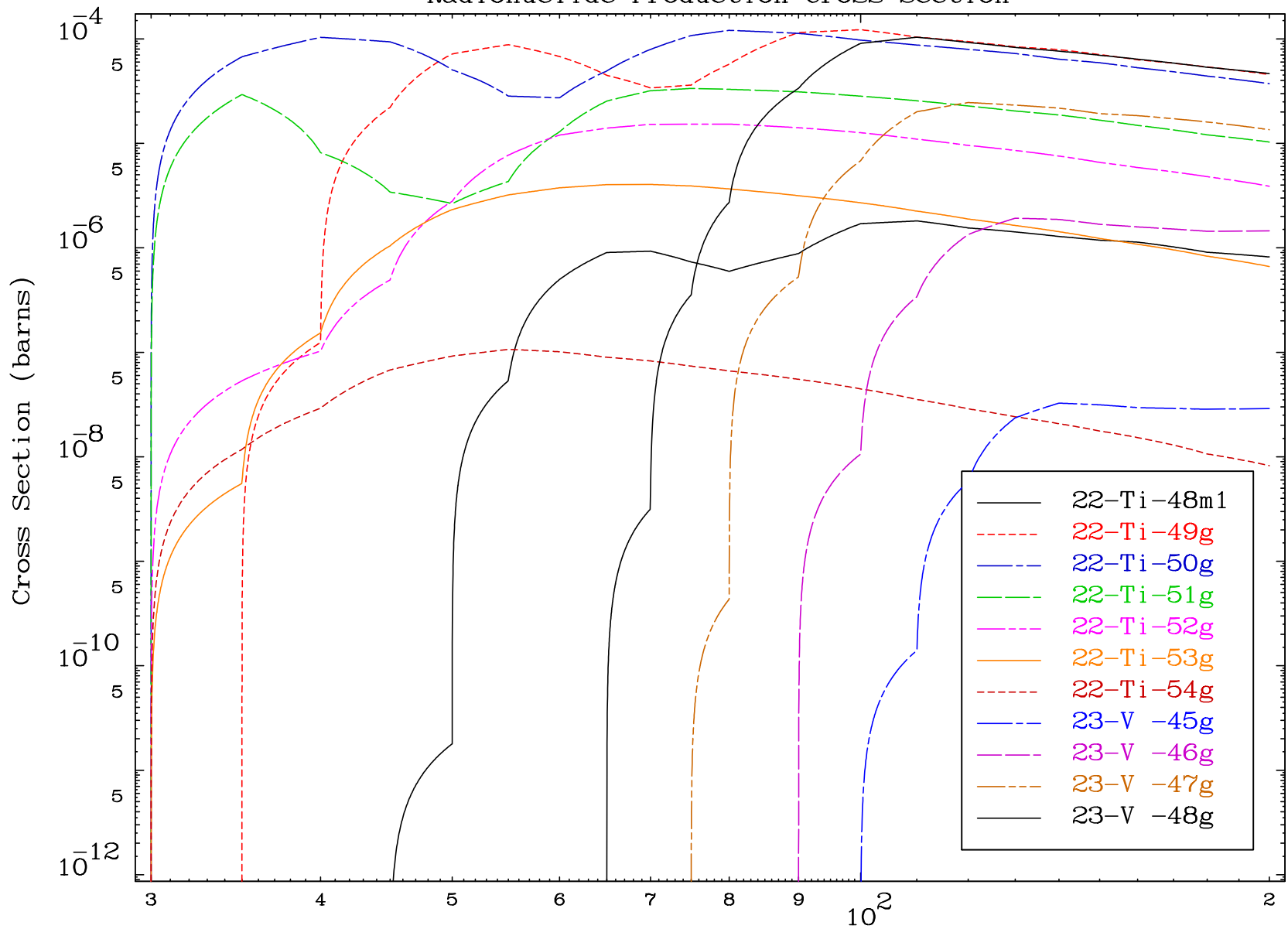
Incident Energy (MeV)

24-Cr-56

MAT 2443

( $\gamma$ , remainder)  
Radionuclide Production Cross Section

24-Cr-56



18

Incident Energy (MeV)

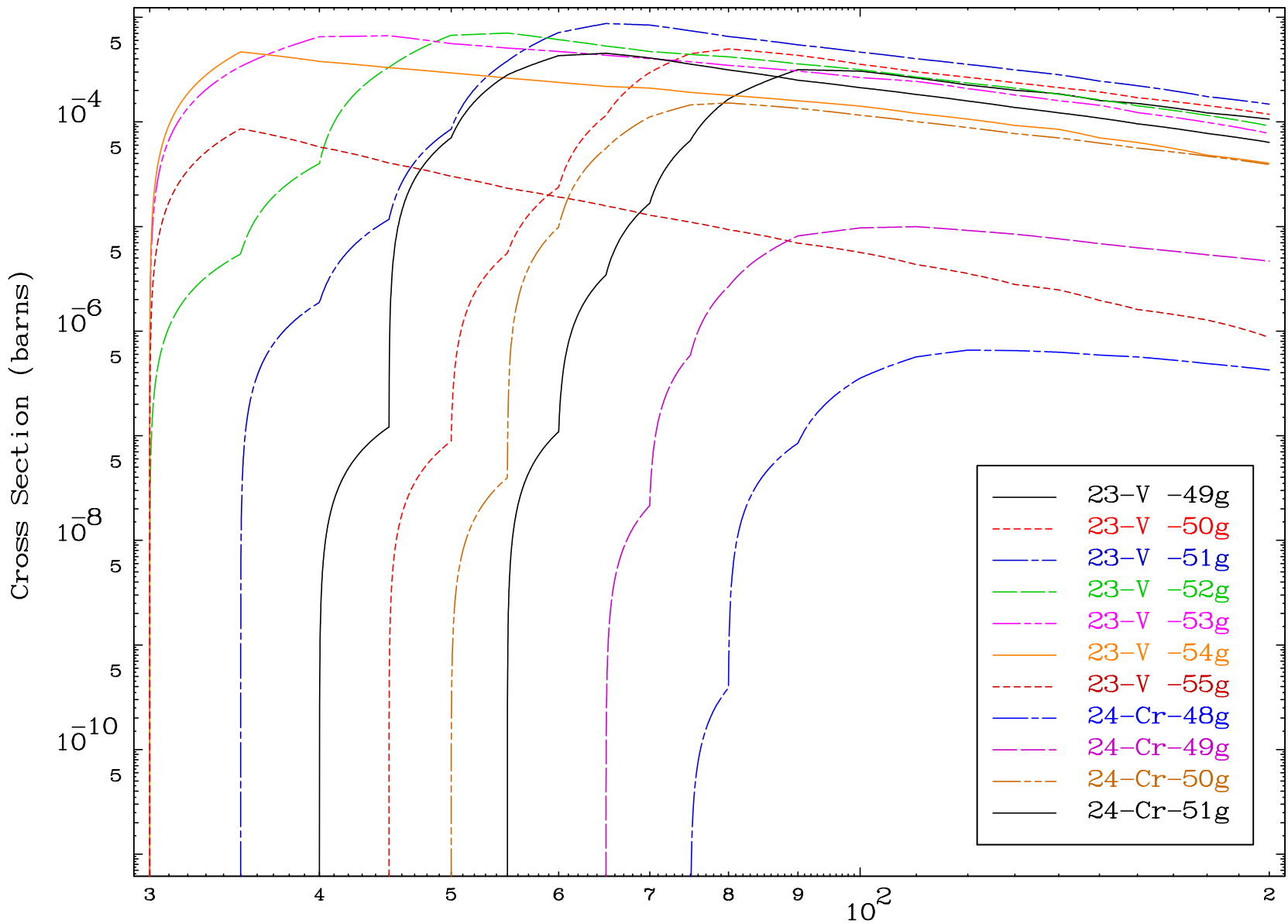
24-Cr-56

MAT 2443

( $\gamma$ , remainder)

24-Cr-56

### Radionuclide Production Cross Section



19

Incident Energy (MeV)

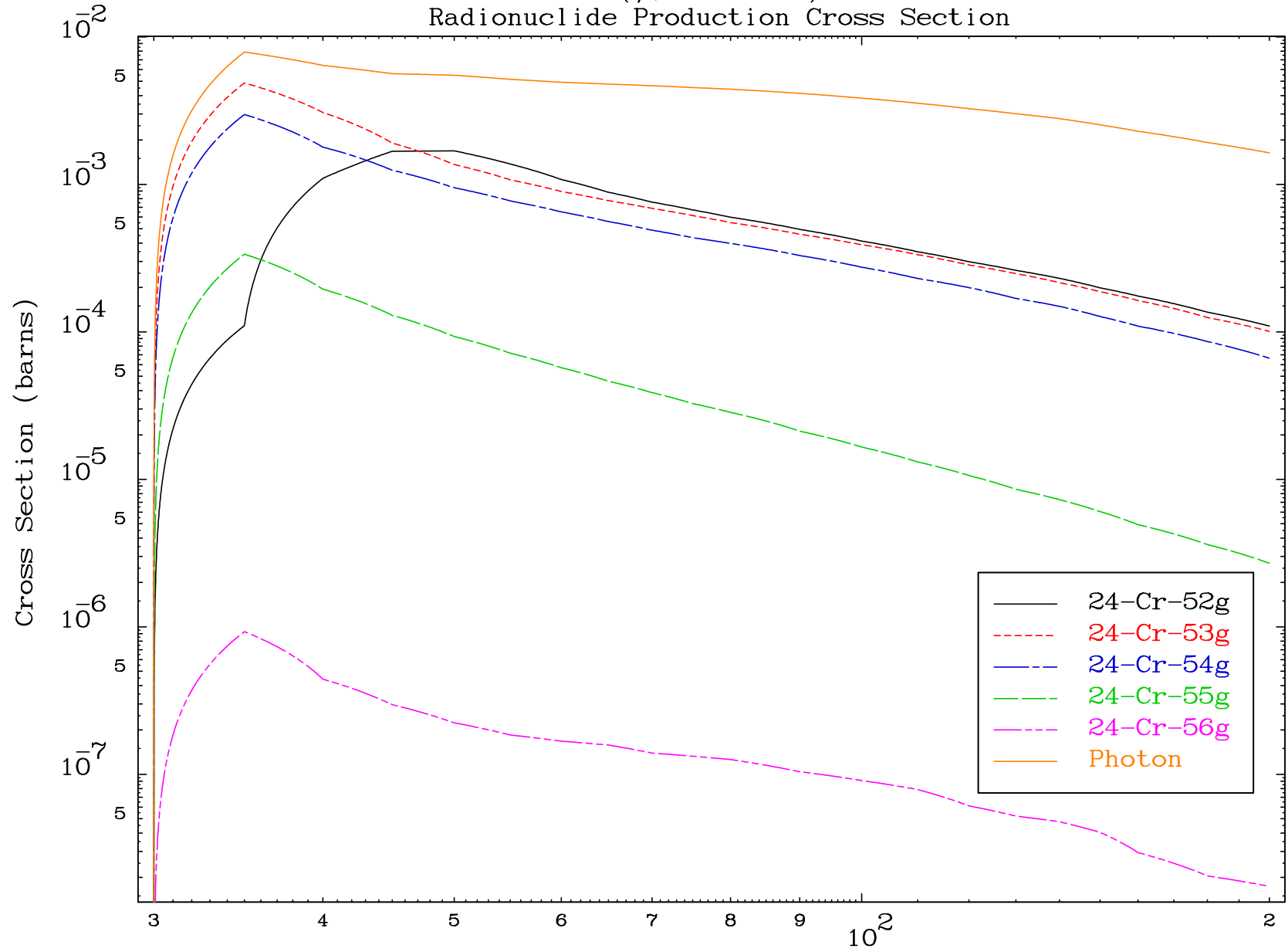
24-Cr-56

MAT 2443

( $\gamma$ , remainder)

24-Cr-56

### Radionuclide Production Cross Section



20

Incident Energy (MeV)

24-Cr-56