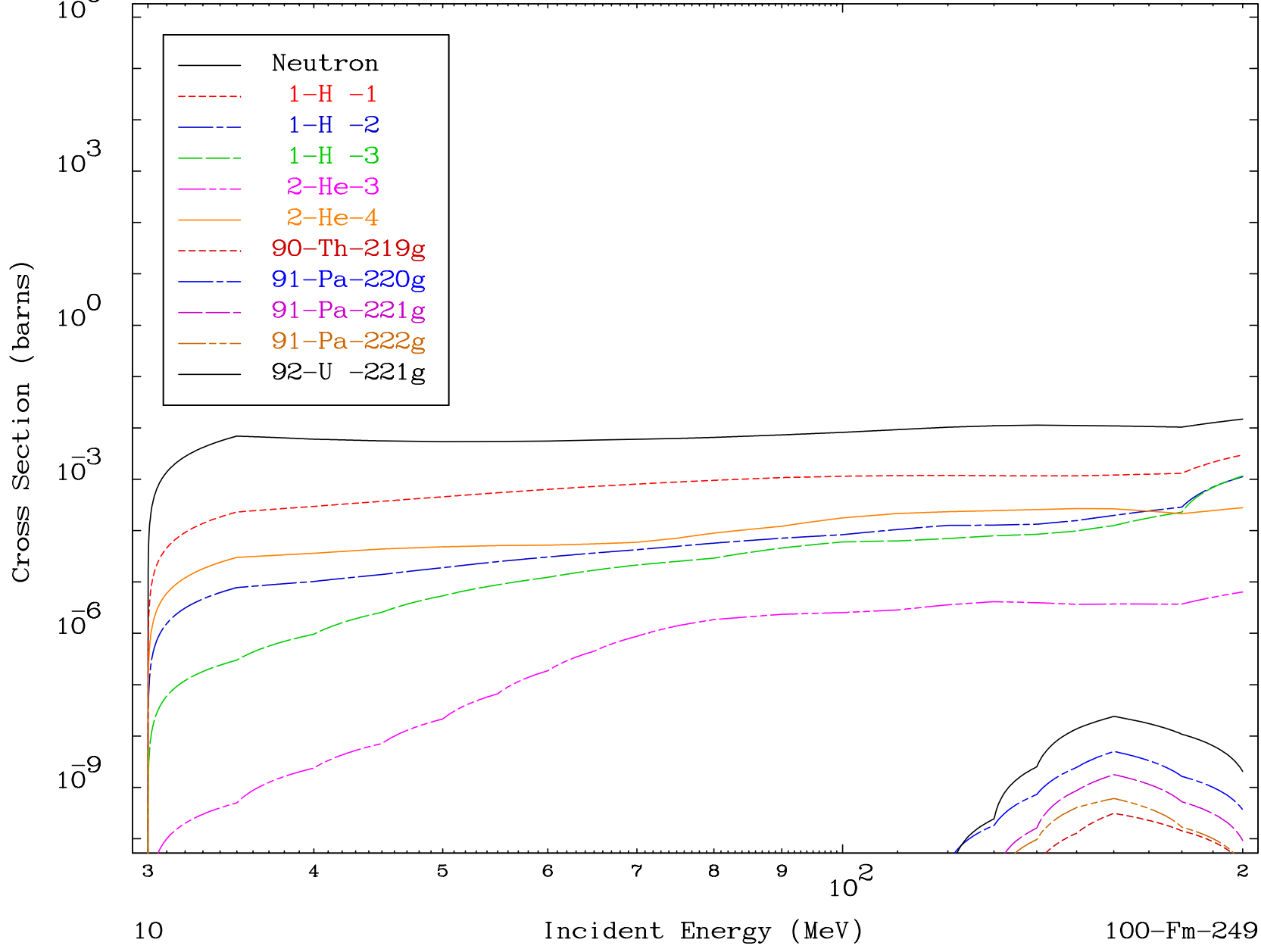
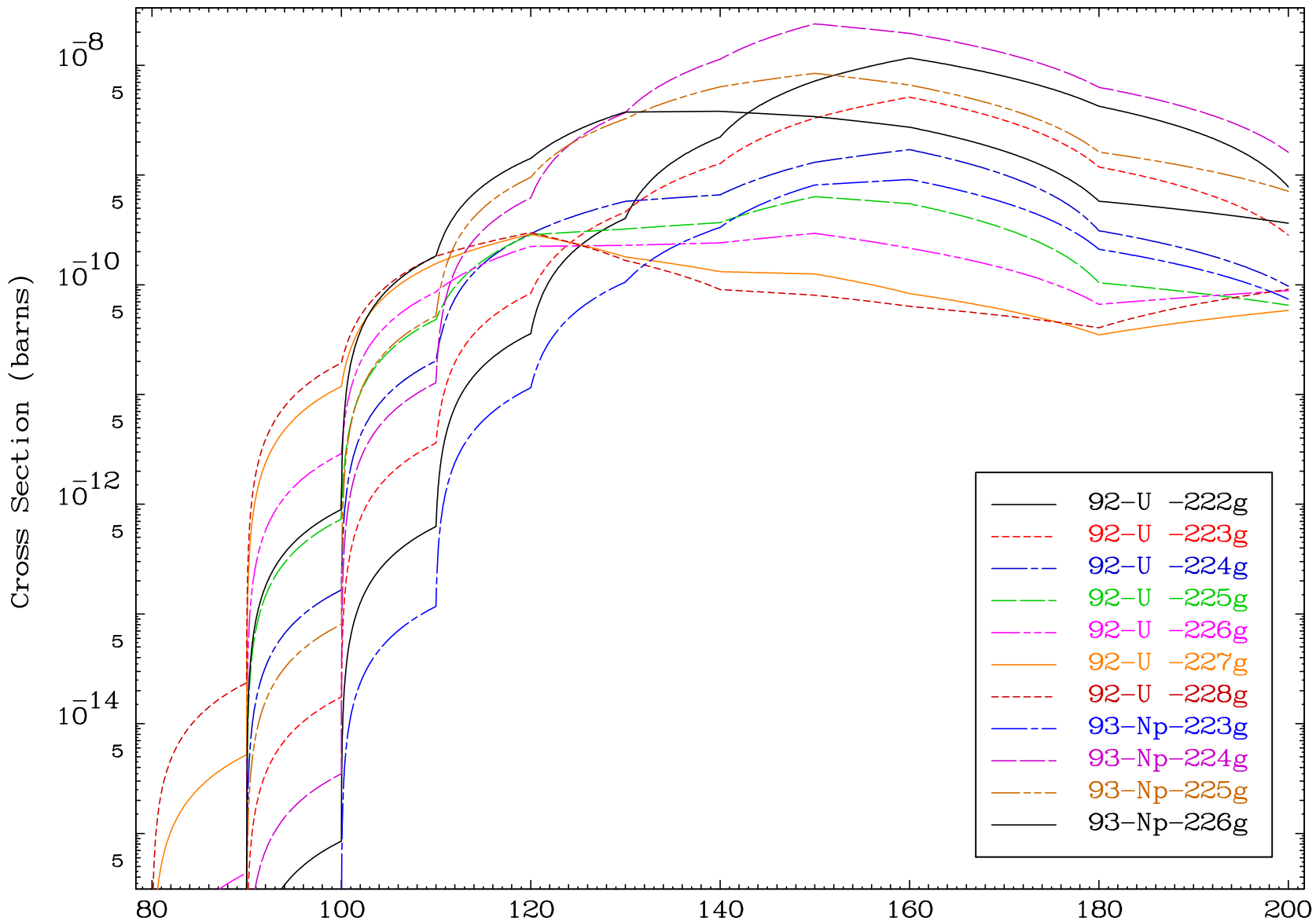
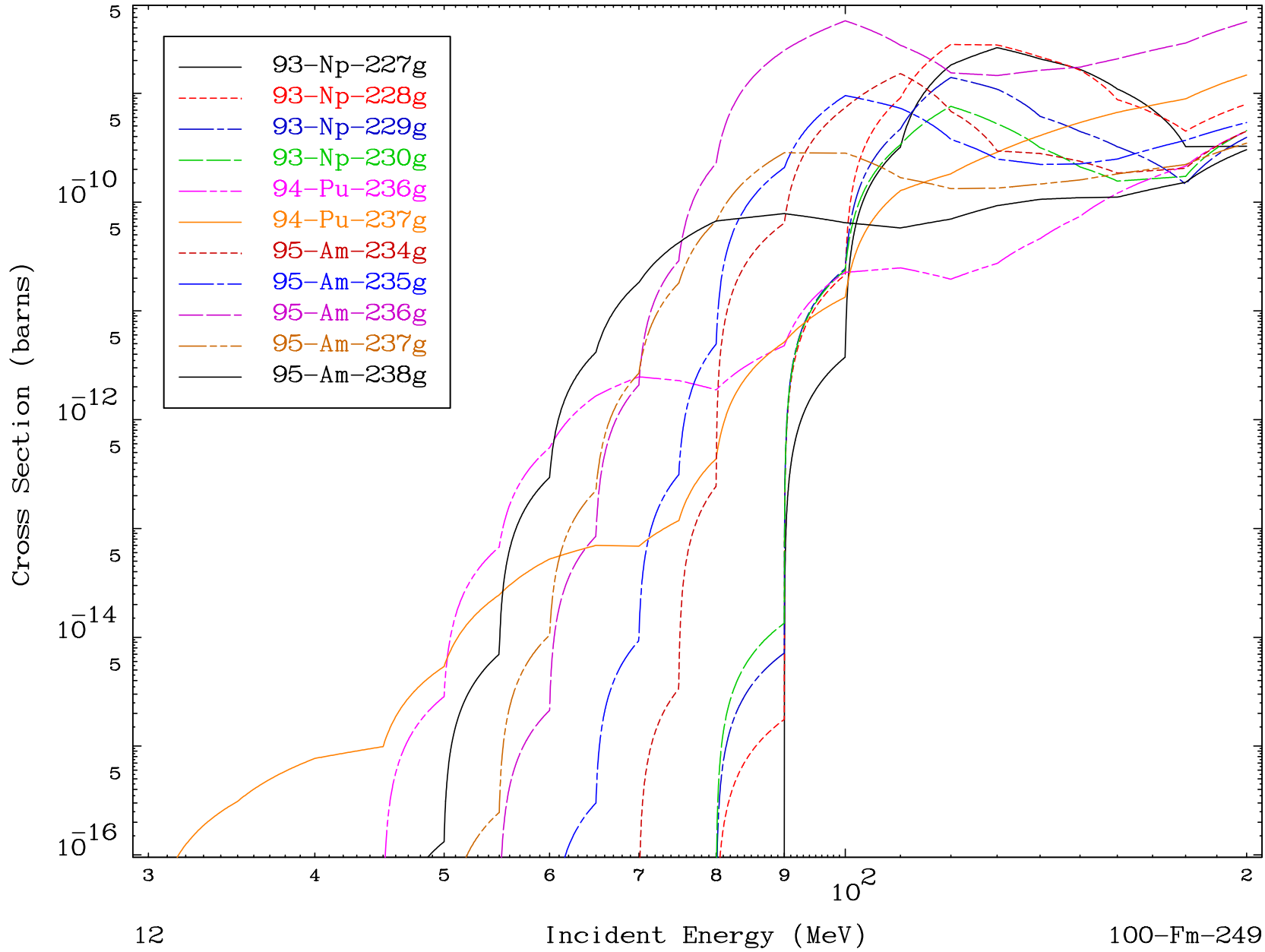


Radionuclide Production Cross Section





Radionuclide Production Cross Section

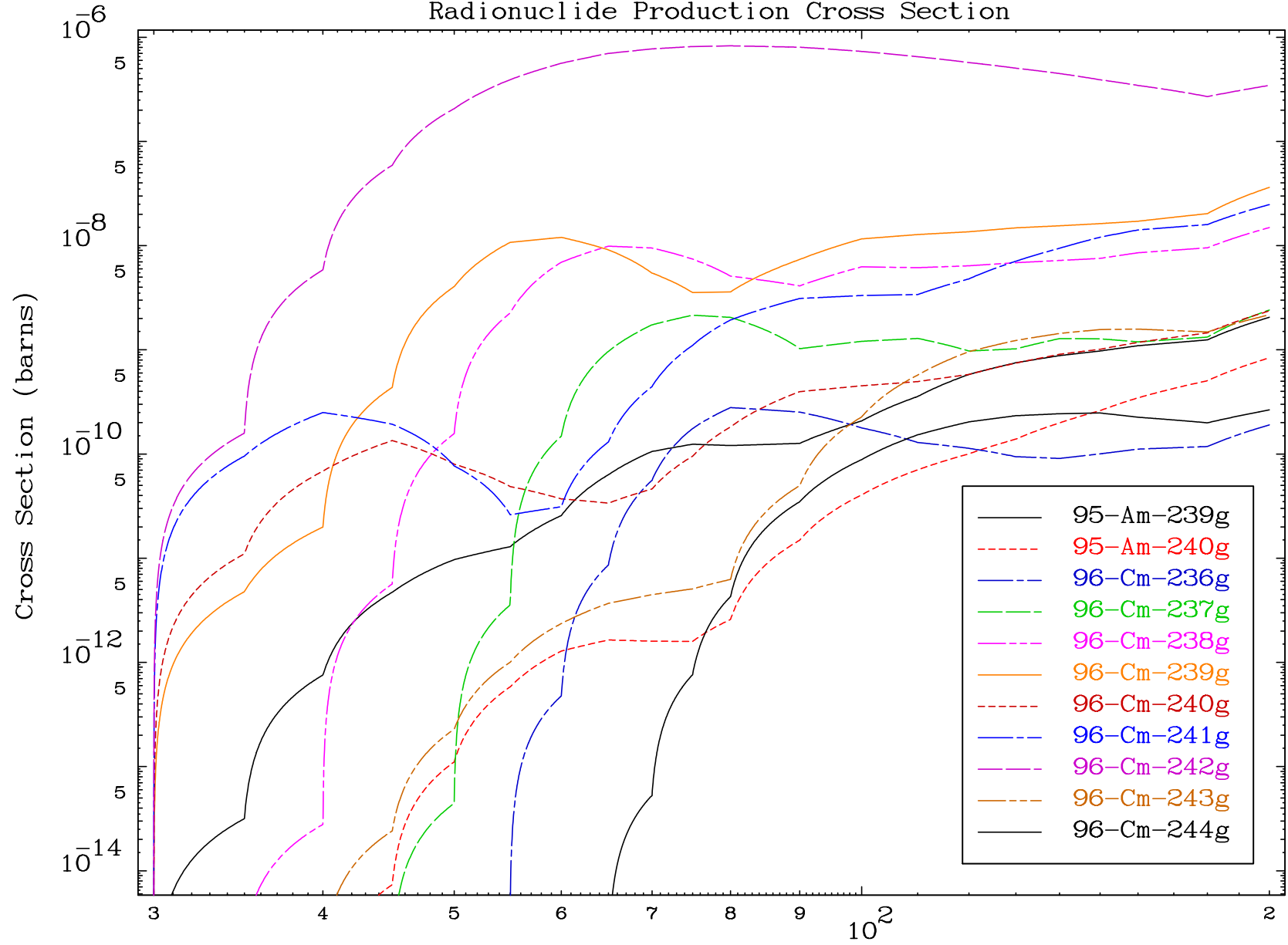


MAT 9930

(γ , remainder)

100-Fm-249

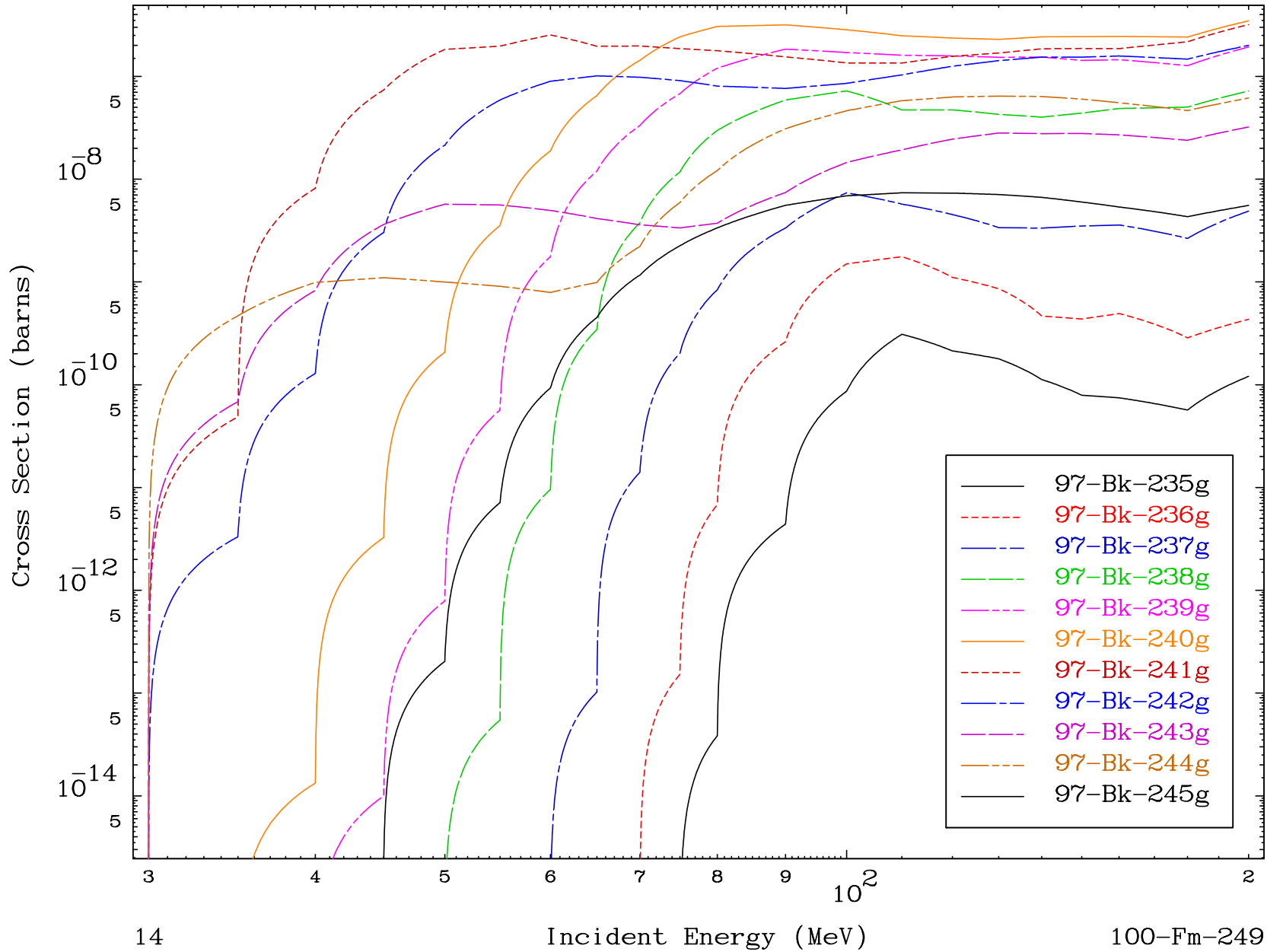
Radionuclide Production Cross Section



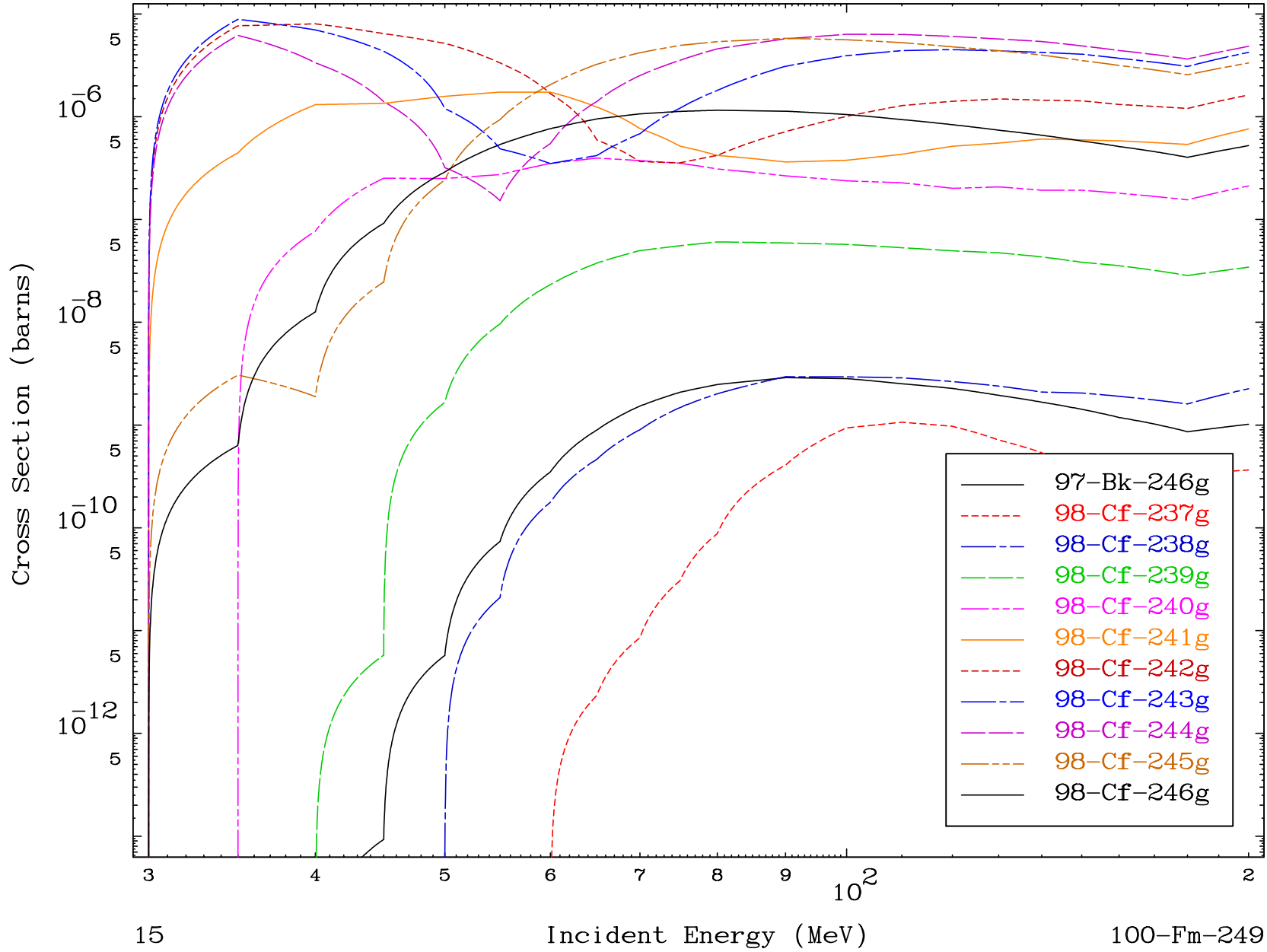
13

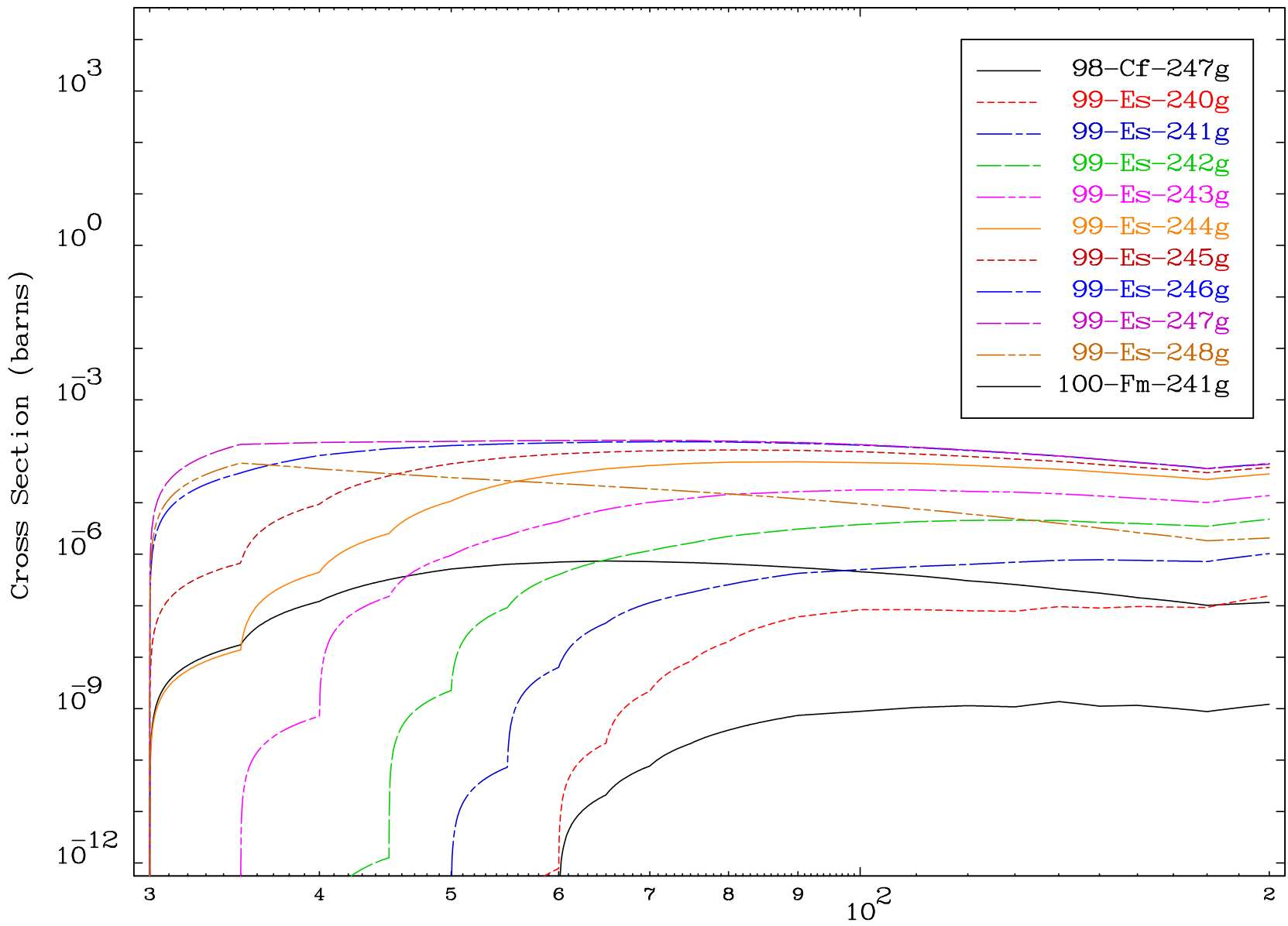
Incident Energy (MeV)

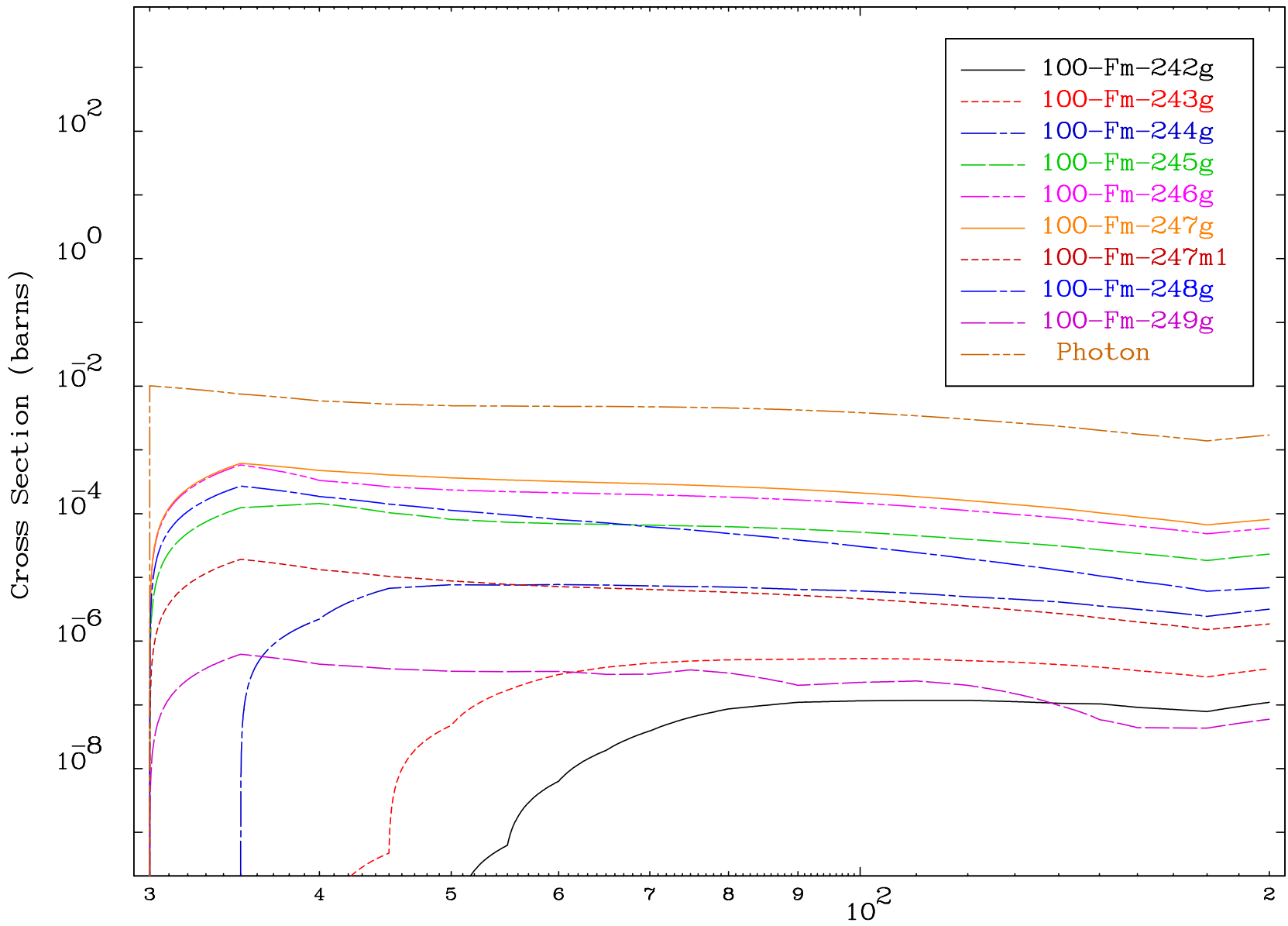
100-Fm-249



Radionuclide Production Cross Section







MAT 9930

($\gamma, 2n$)

100-Fm-249

Radionuclide Production Cross Section

