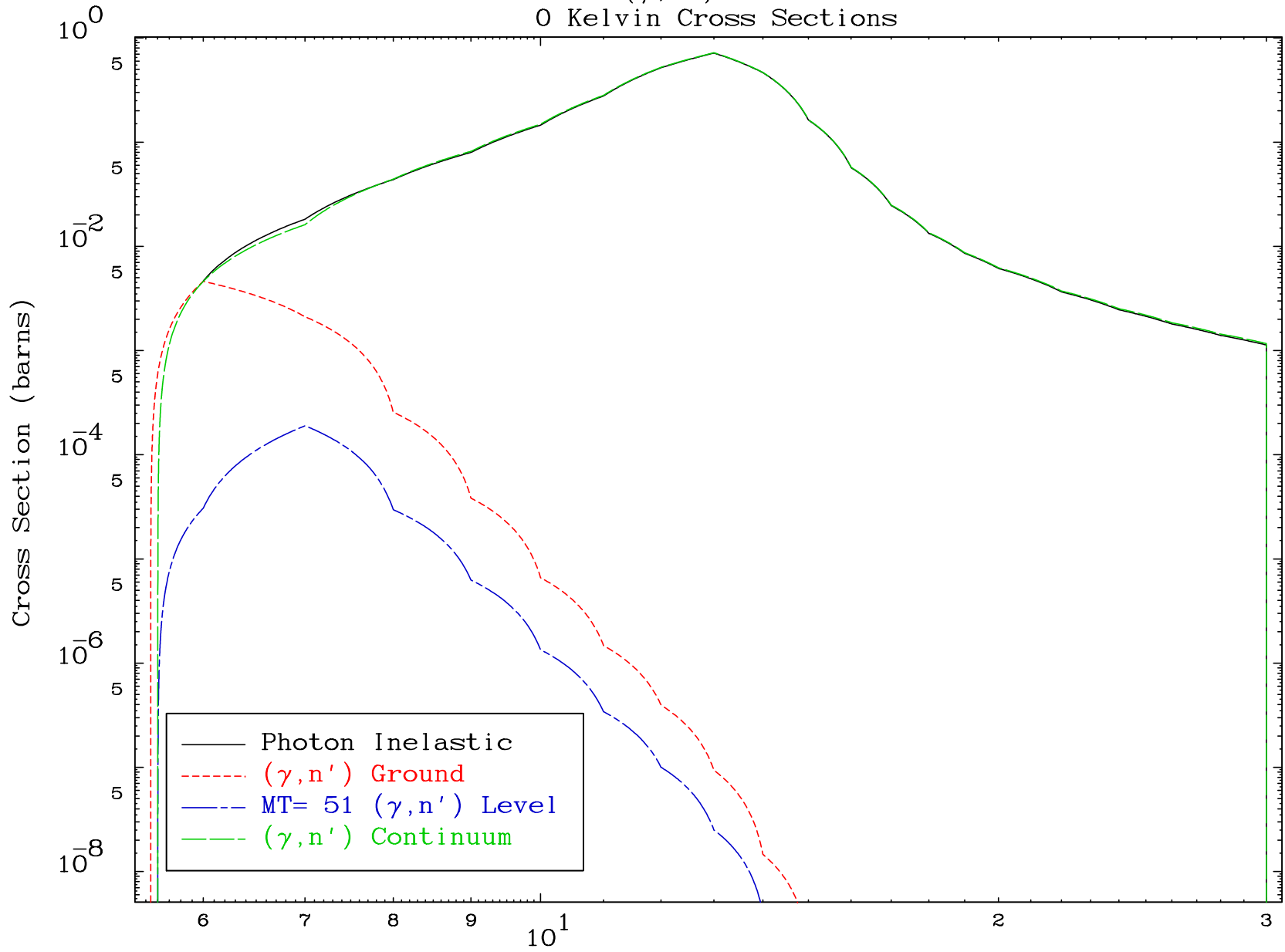


MAT 9960

( $\gamma, n'$ ) Level  
0 Kelvin Cross Sections

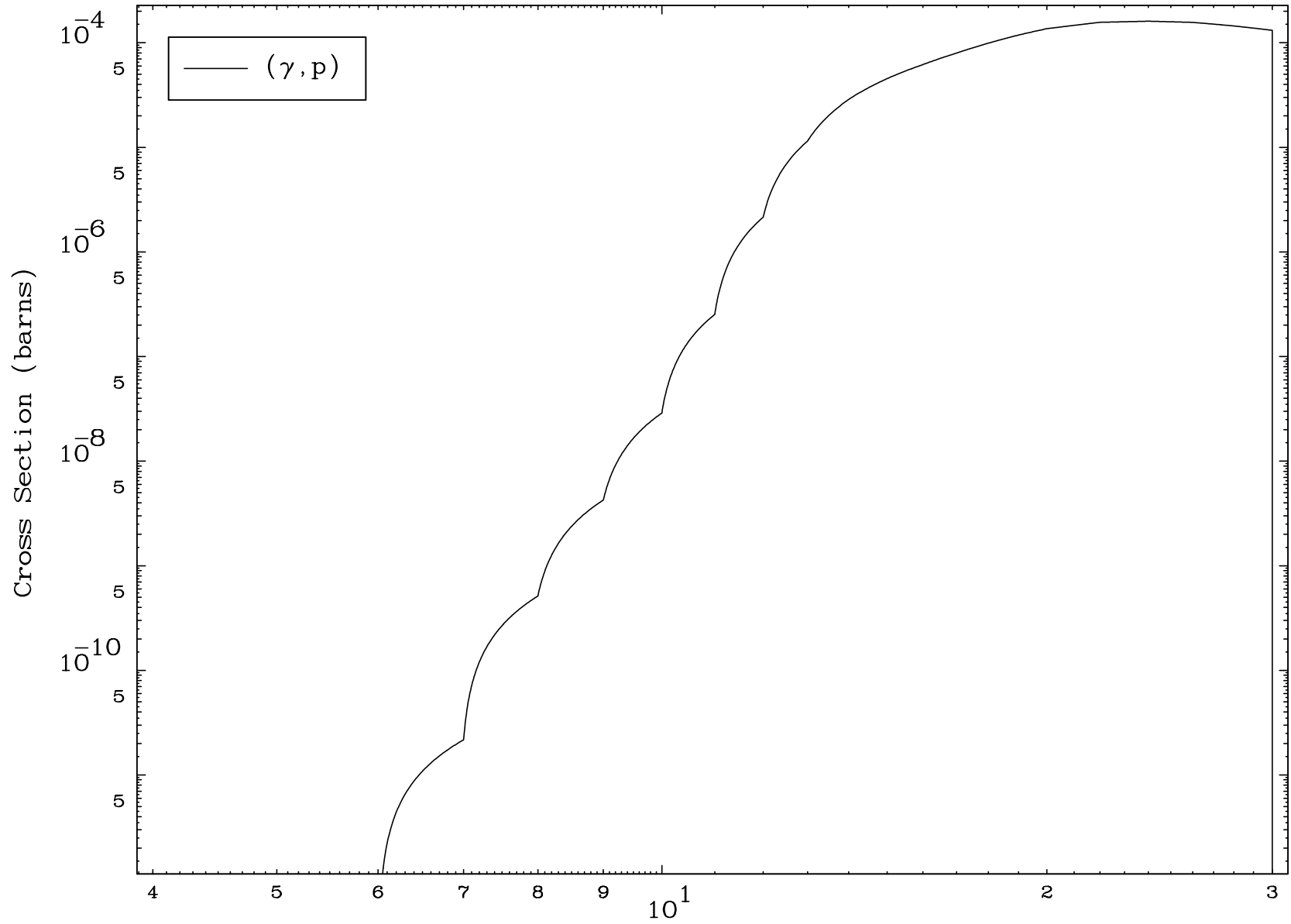
101-Md-254

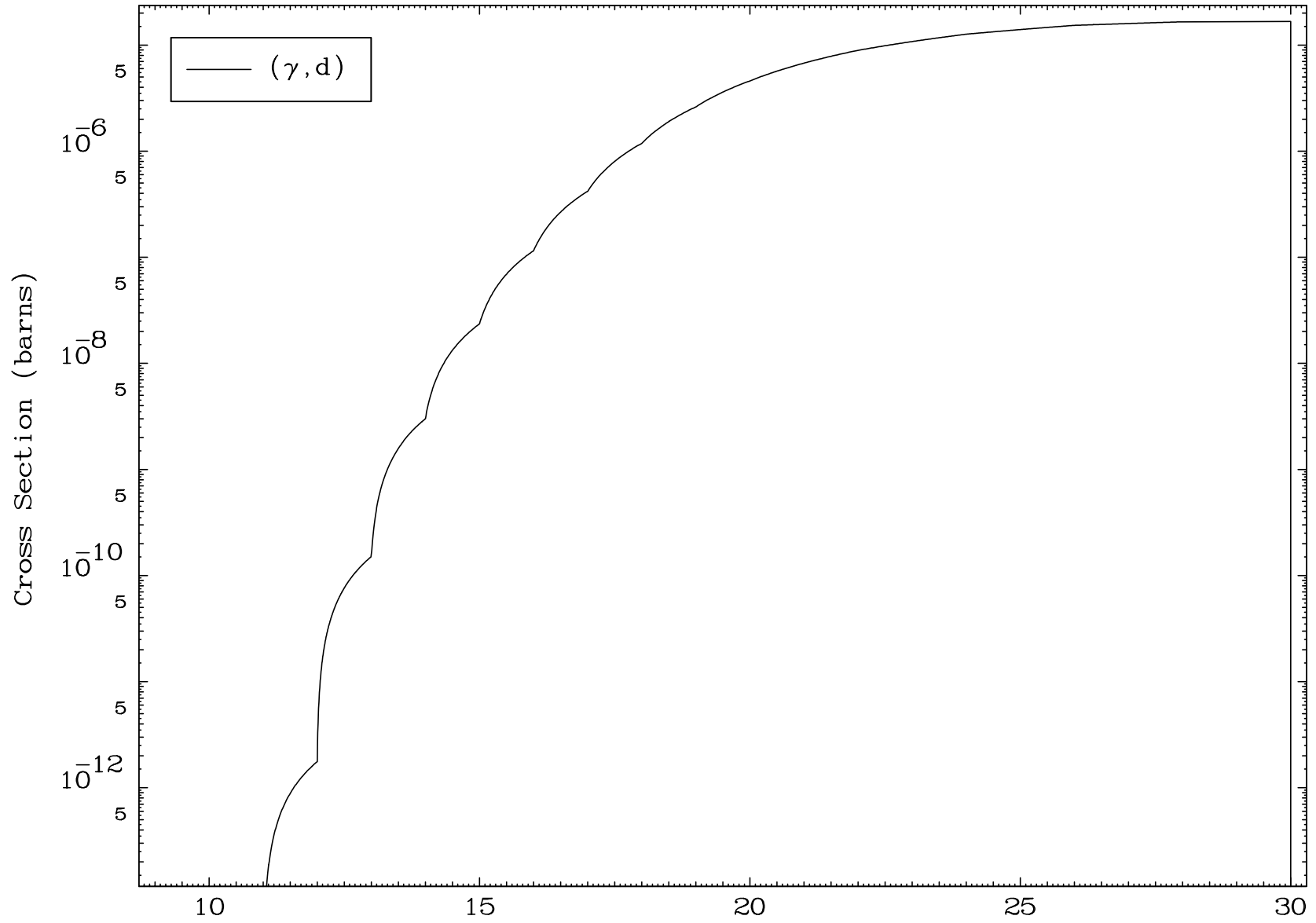


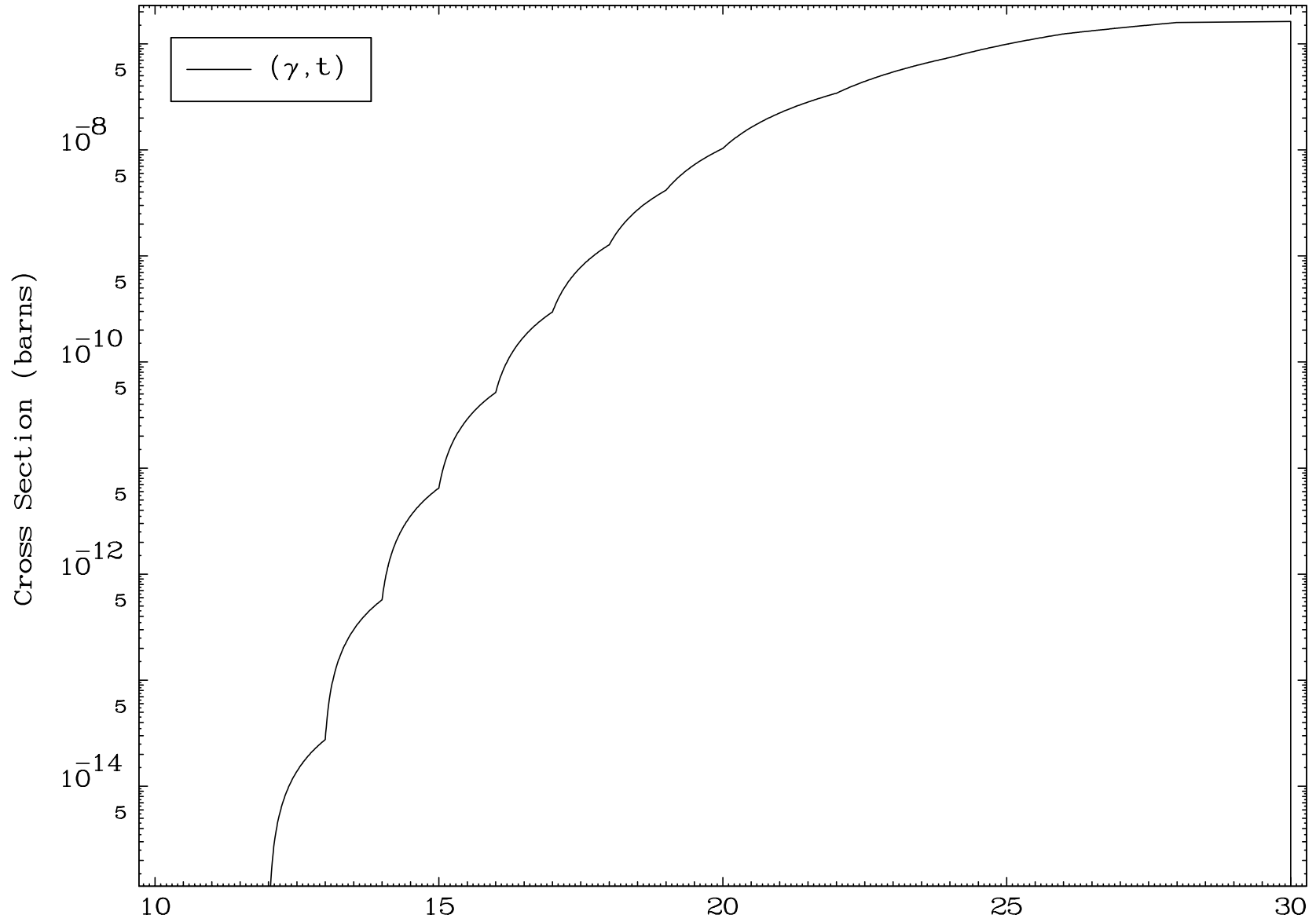
5

Incident Energy (MeV)

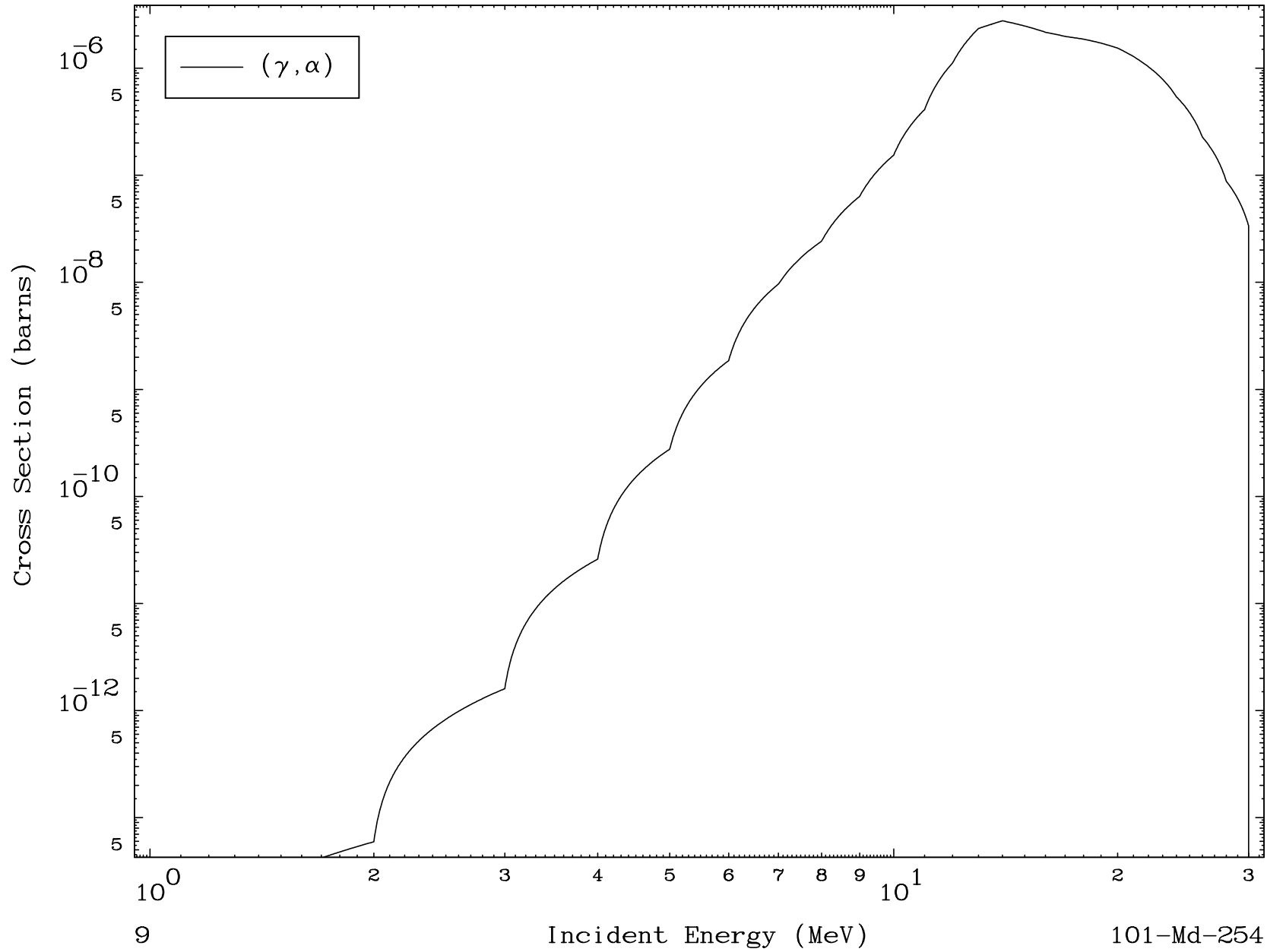
101-Md-254



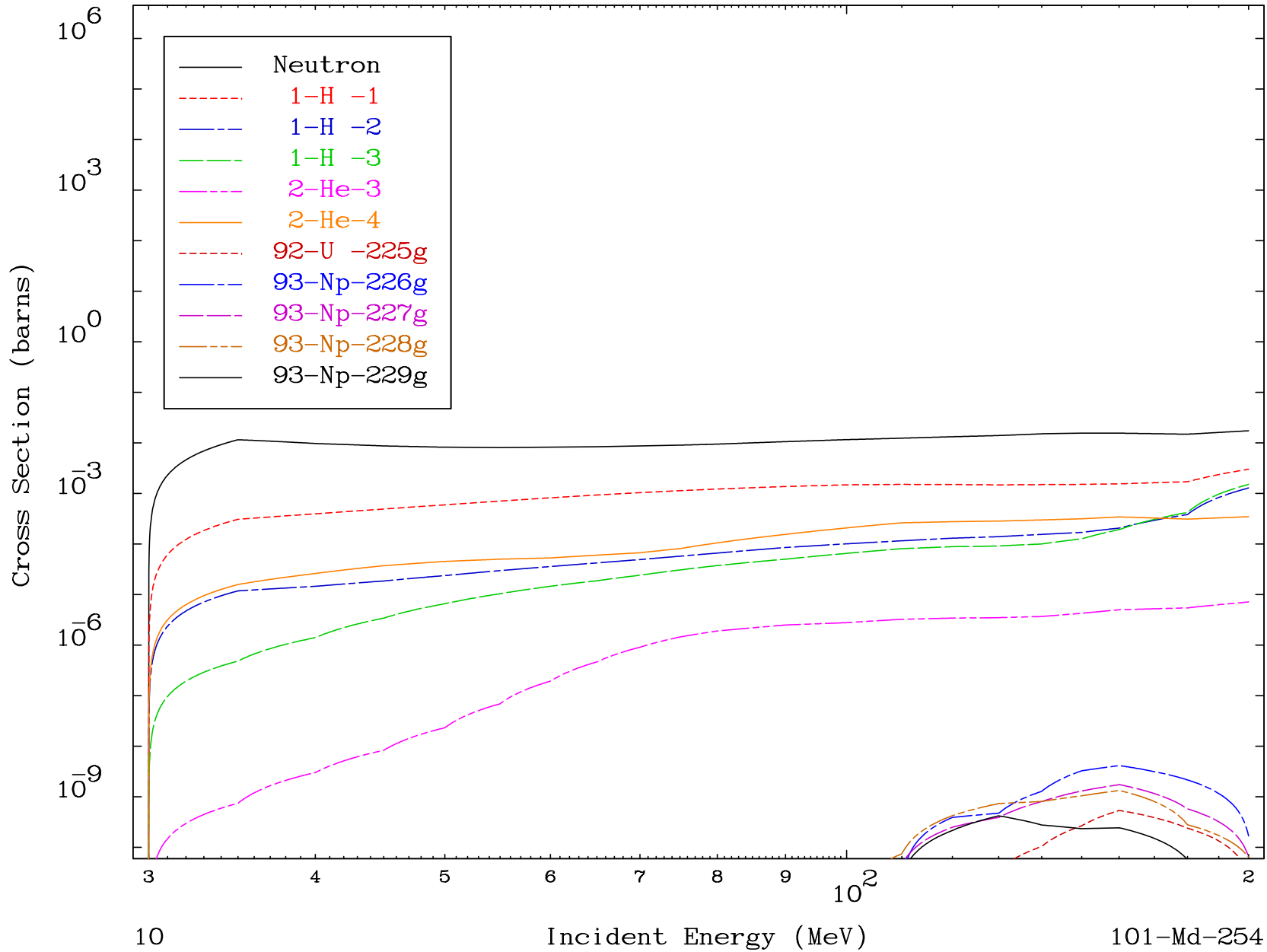




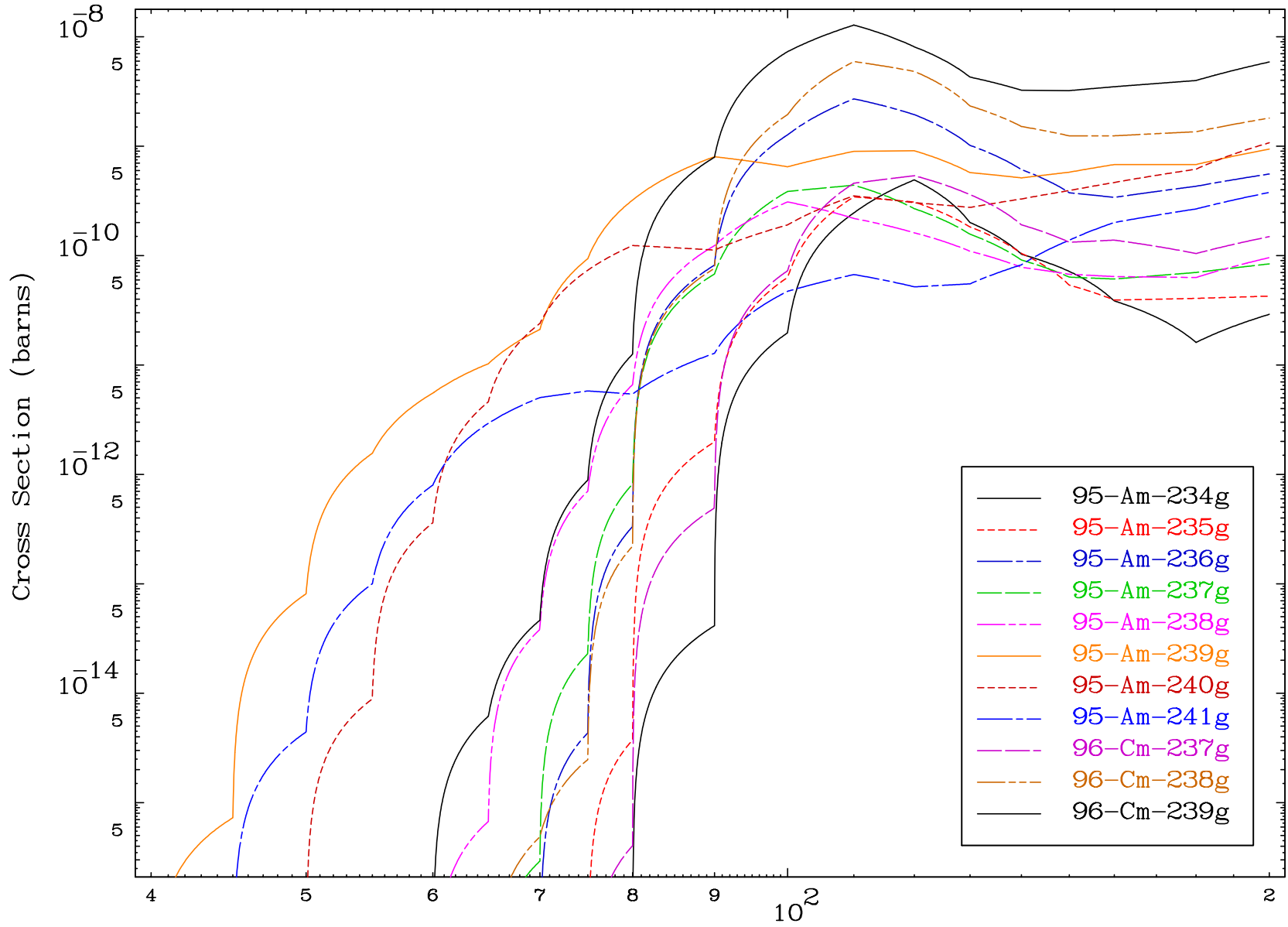


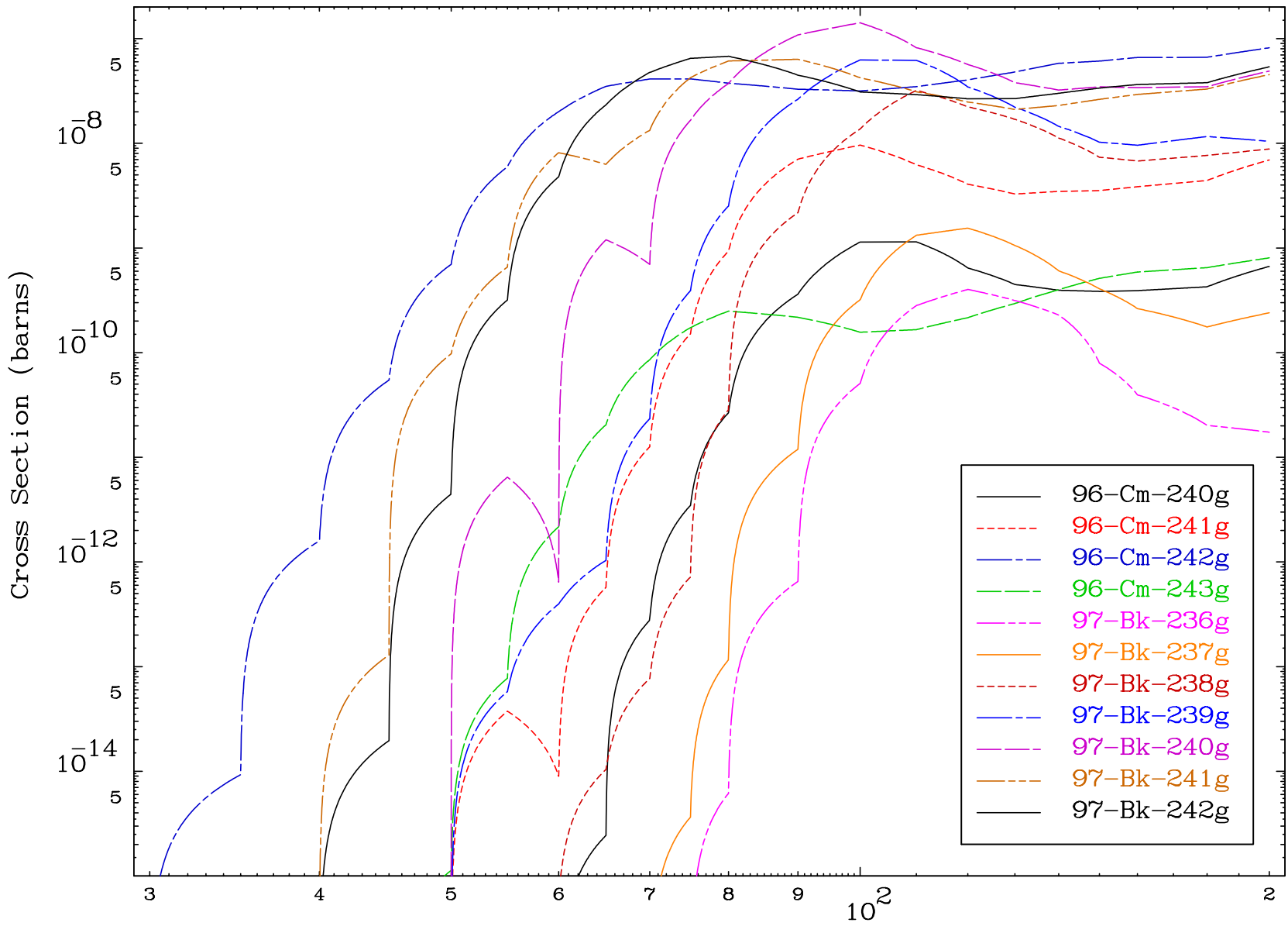


Radionuclide Production Cross Section

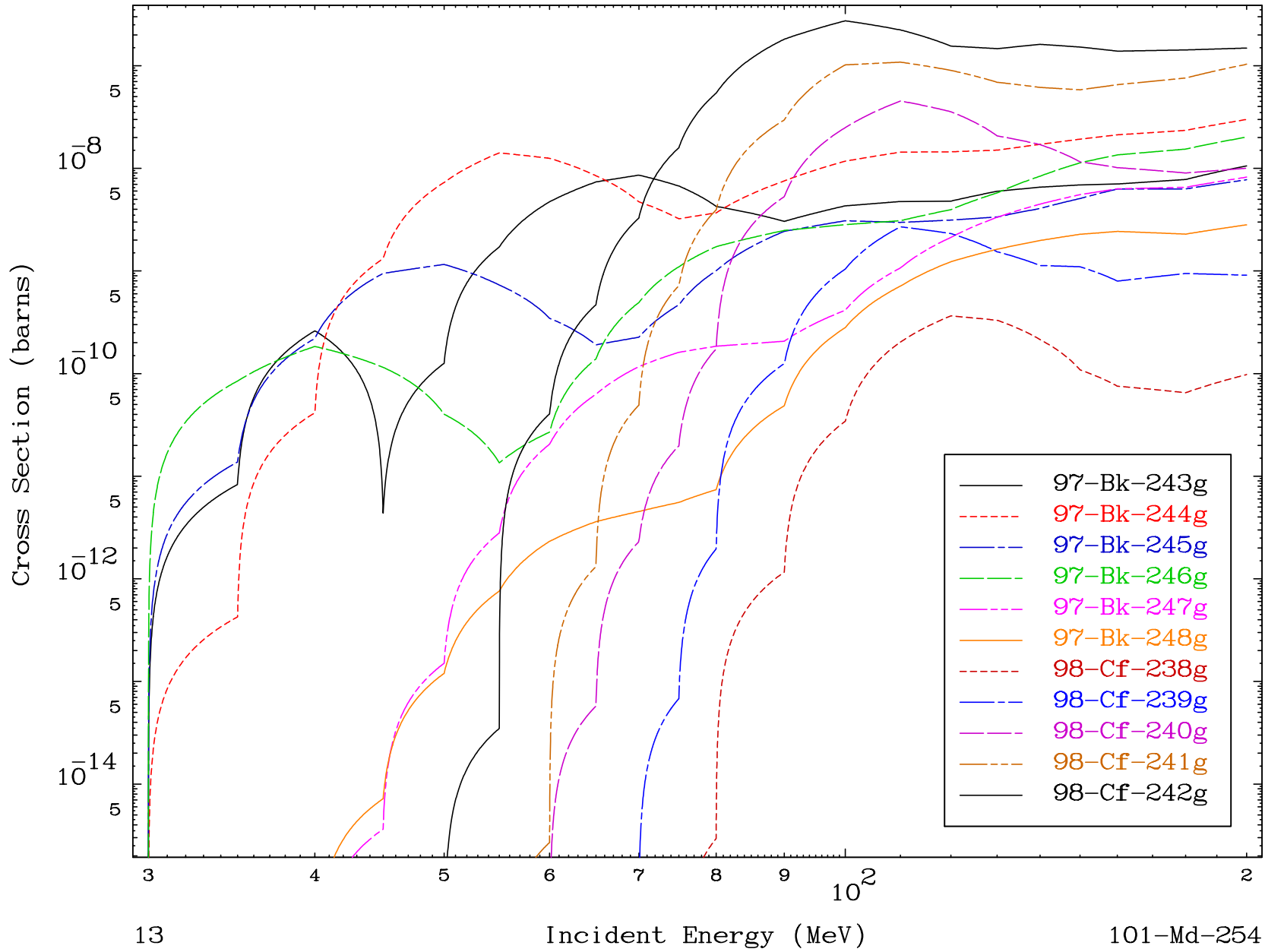


Radionuclide Production Cross Section





Radionuclide Production Cross Section

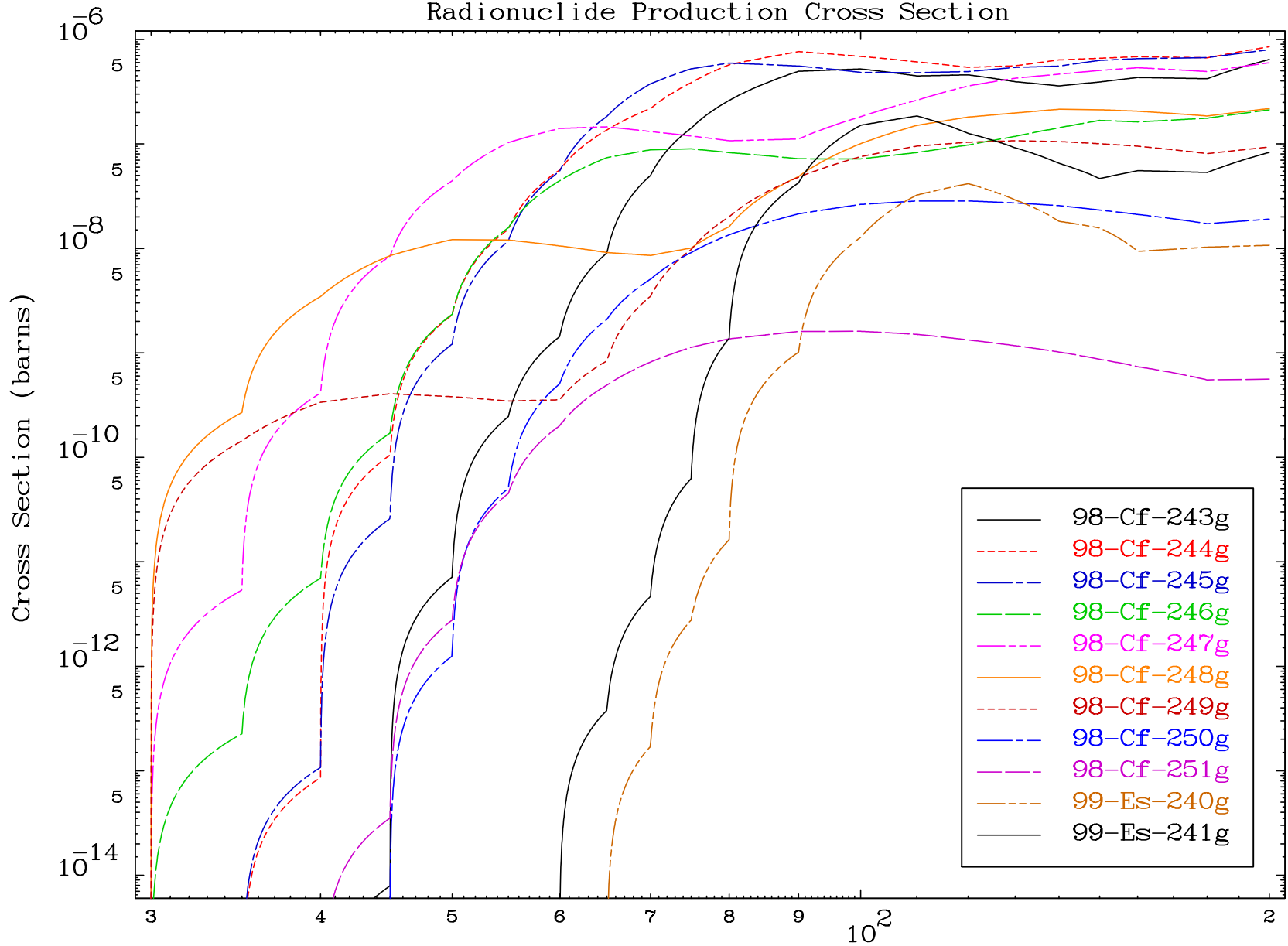


MAT 9960

( $\gamma$ , remainder)

101-Md-254

### Radionuclide Production Cross Section

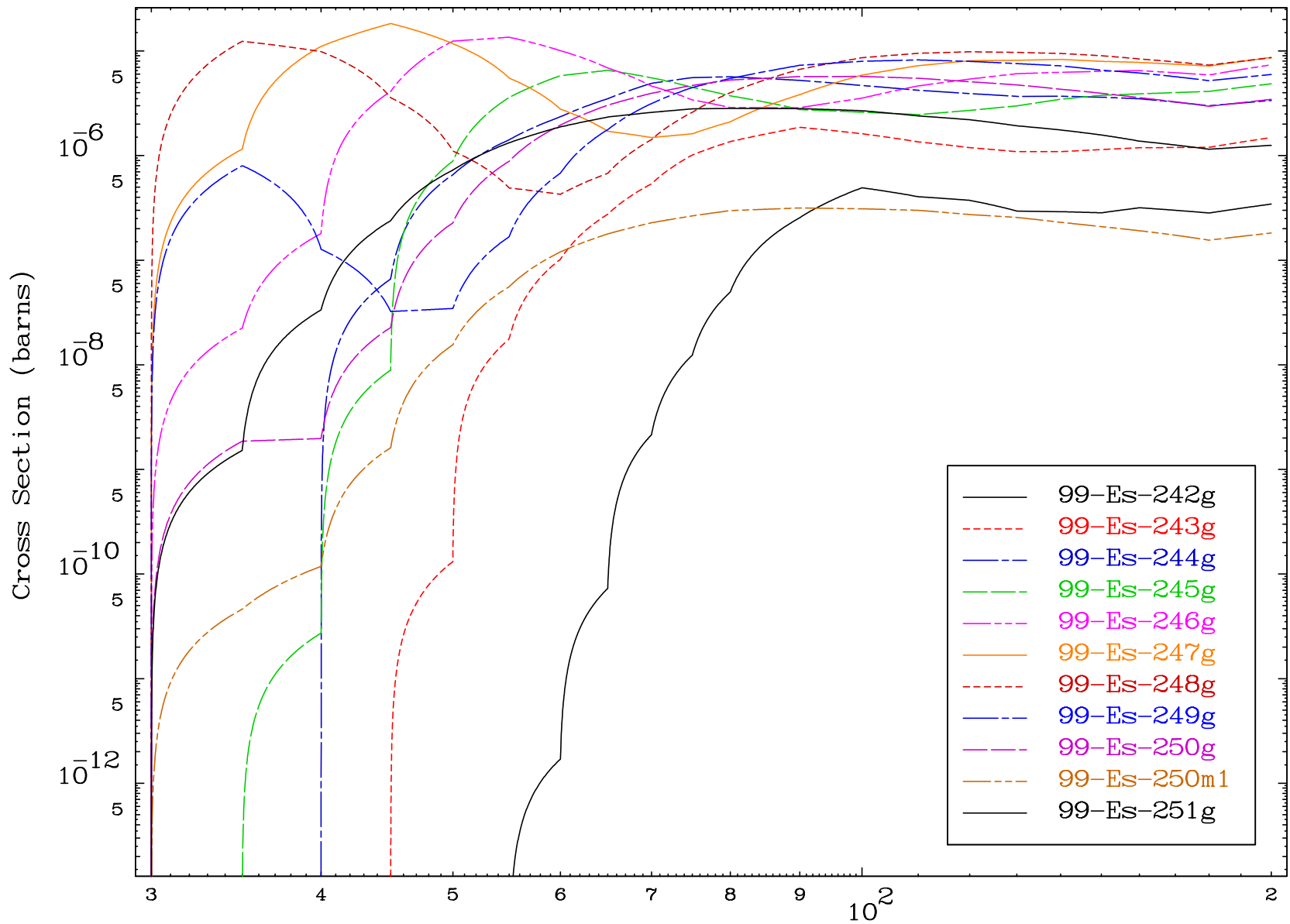


14

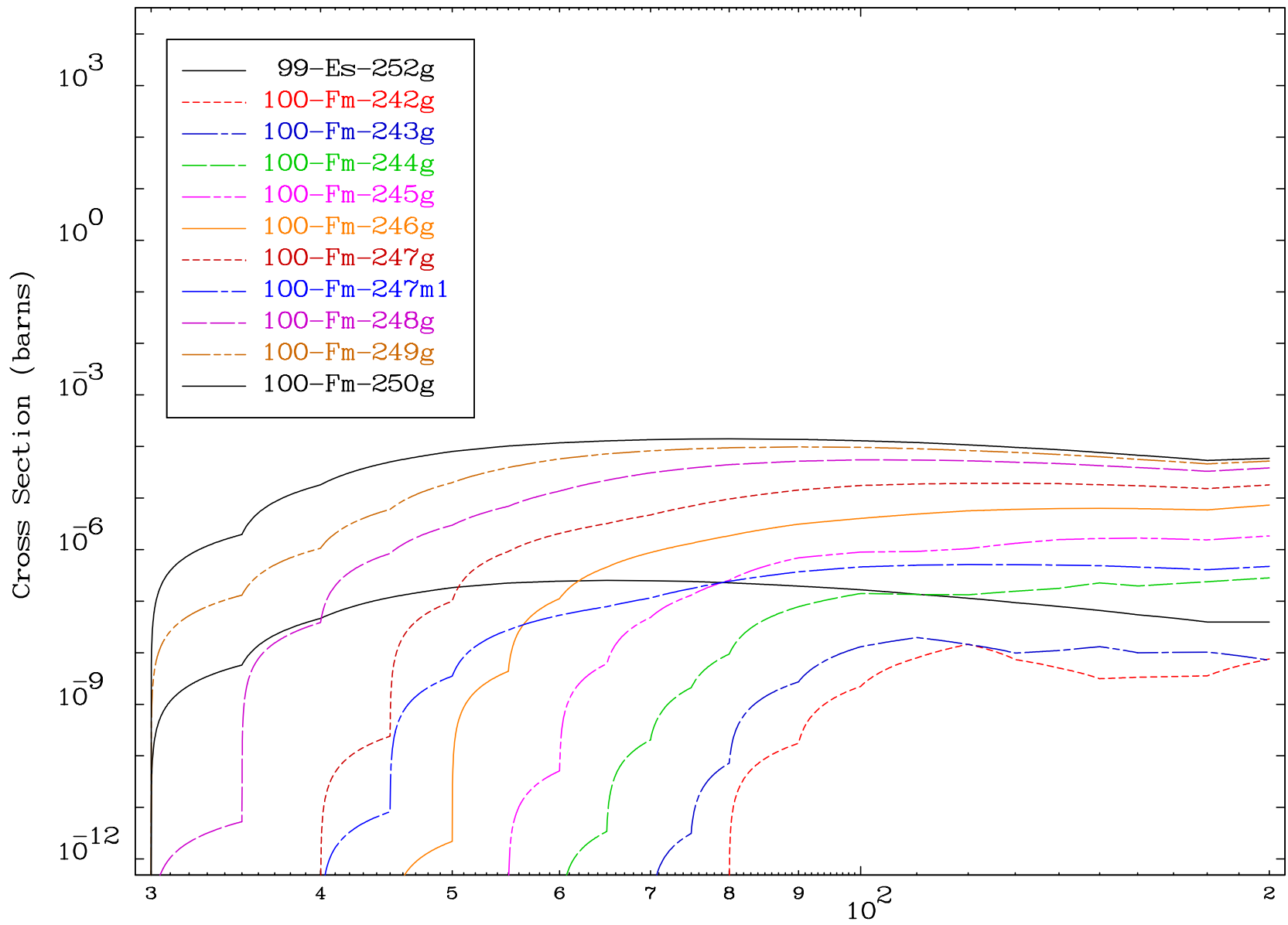
Incident Energy (MeV)

101-Md-254

Radionuclide Production Cross Section

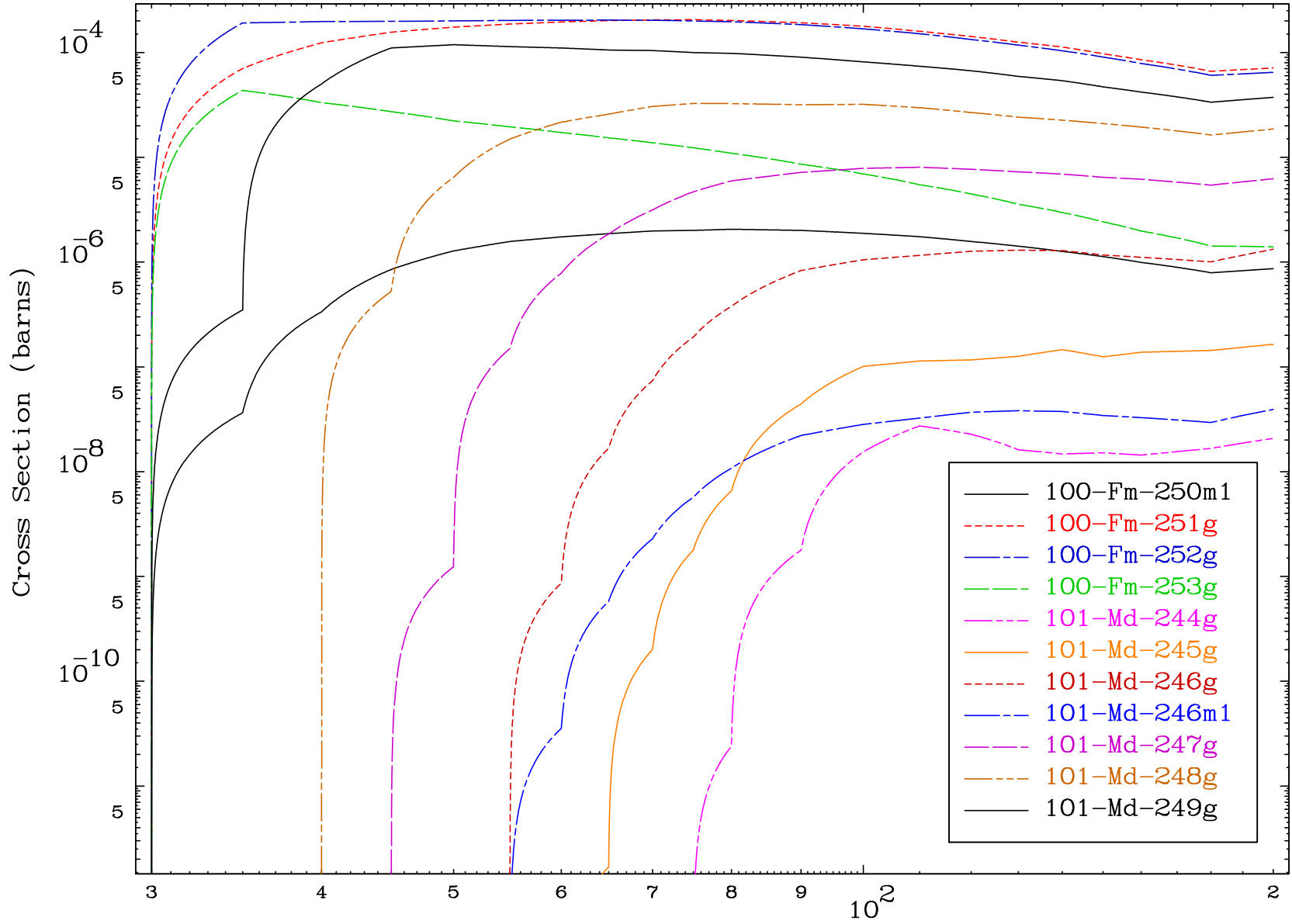


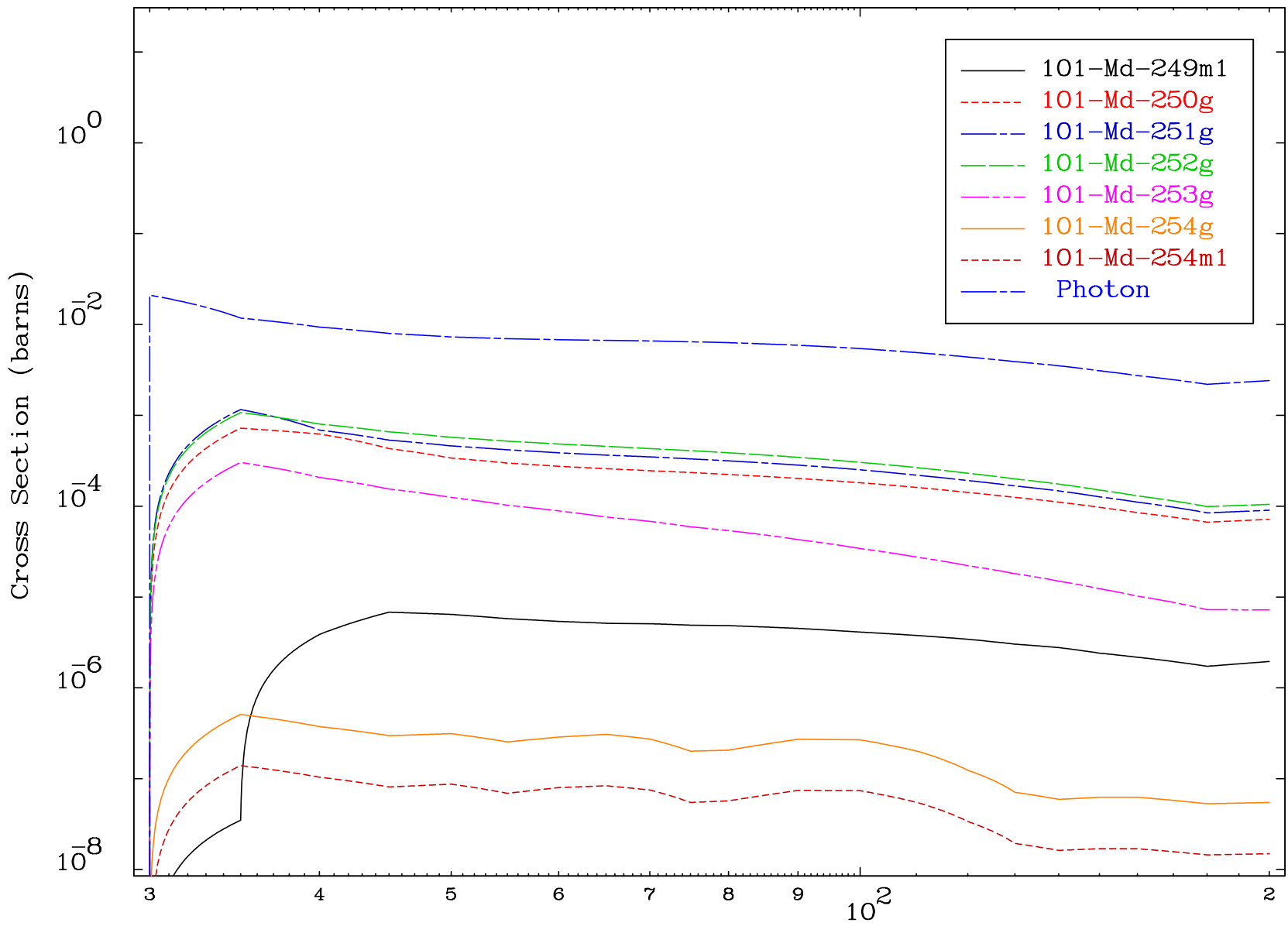
Radionuclide Production Cross Section



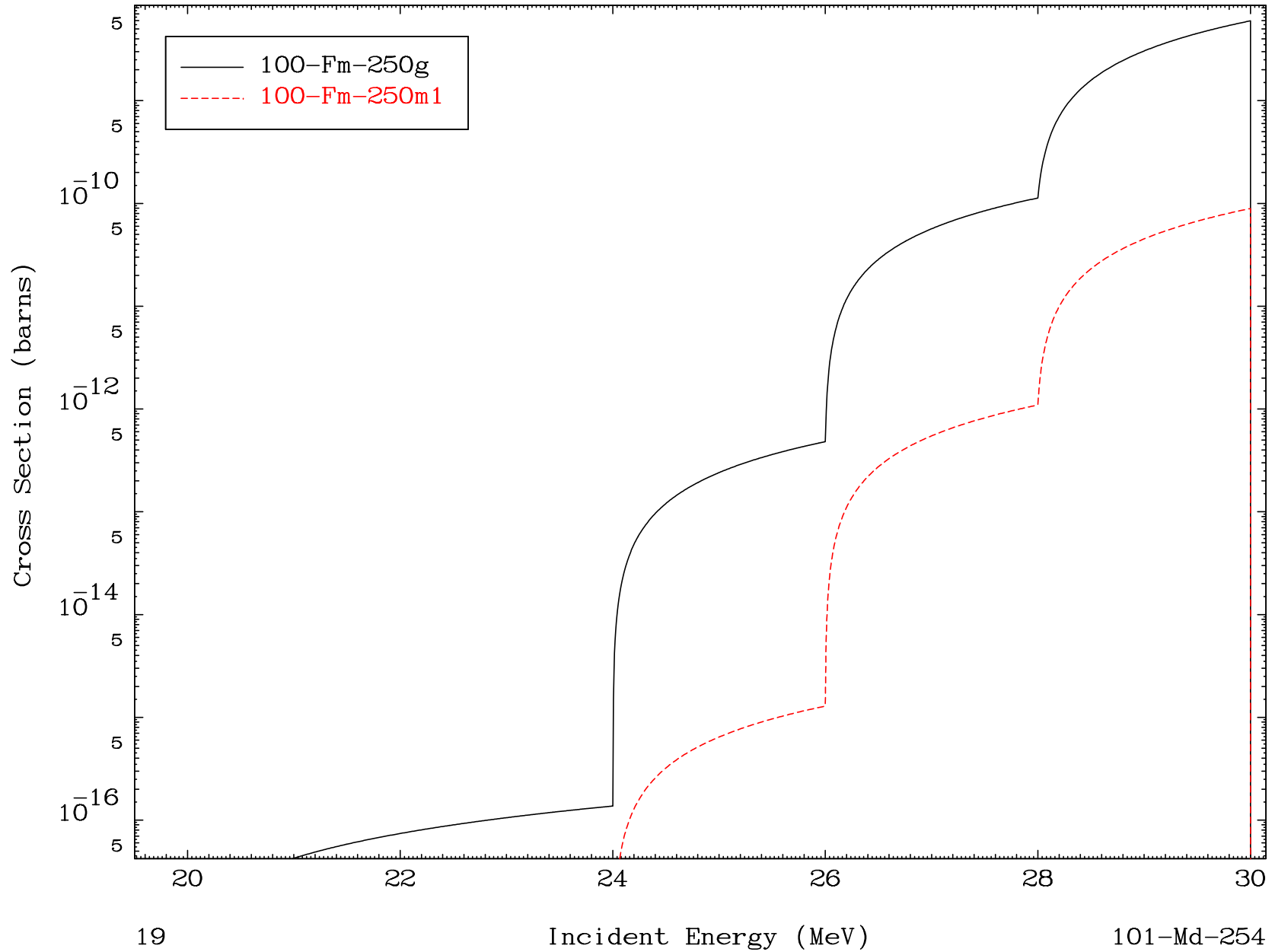


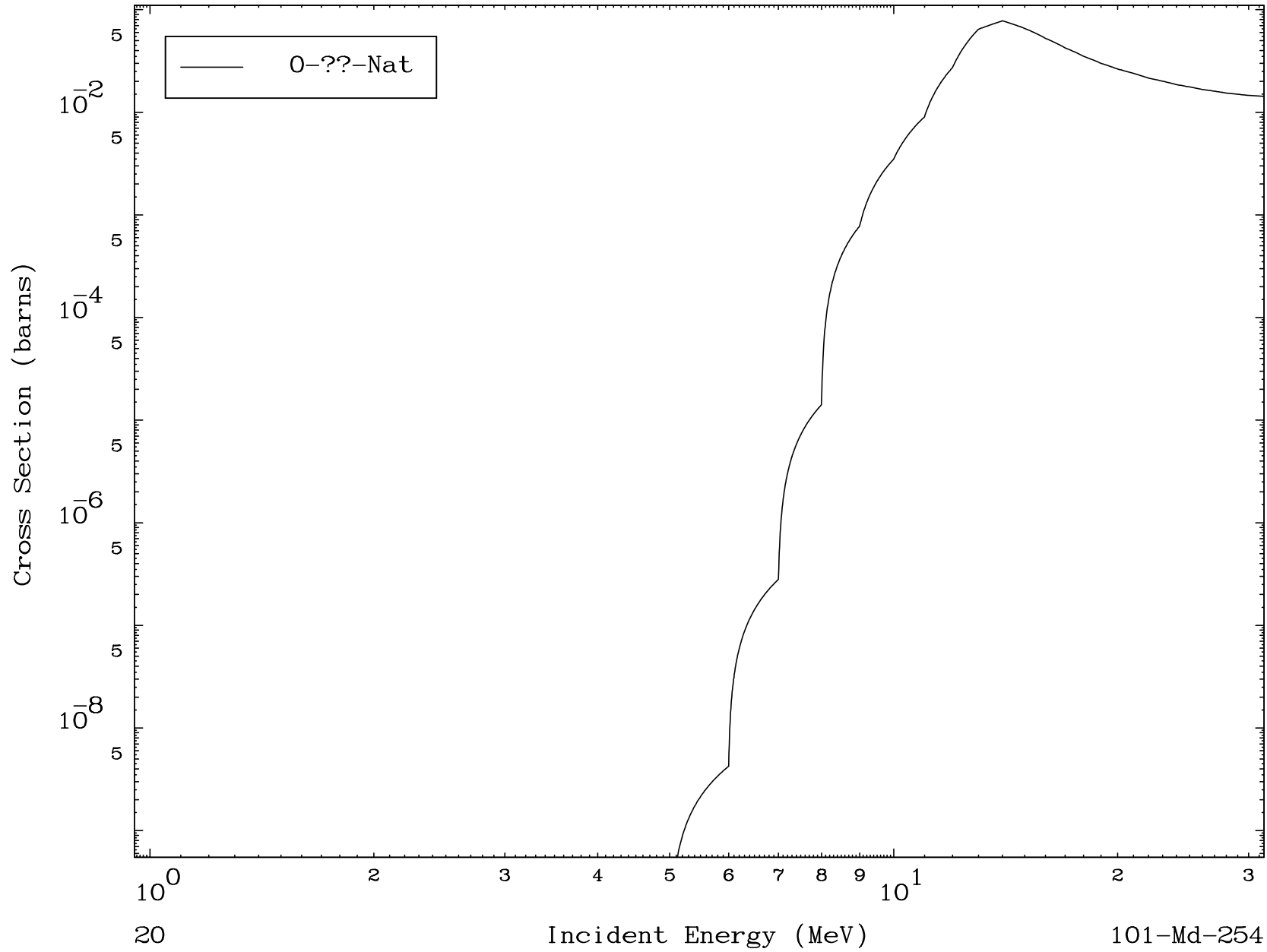
Radionuclide Production Cross Section



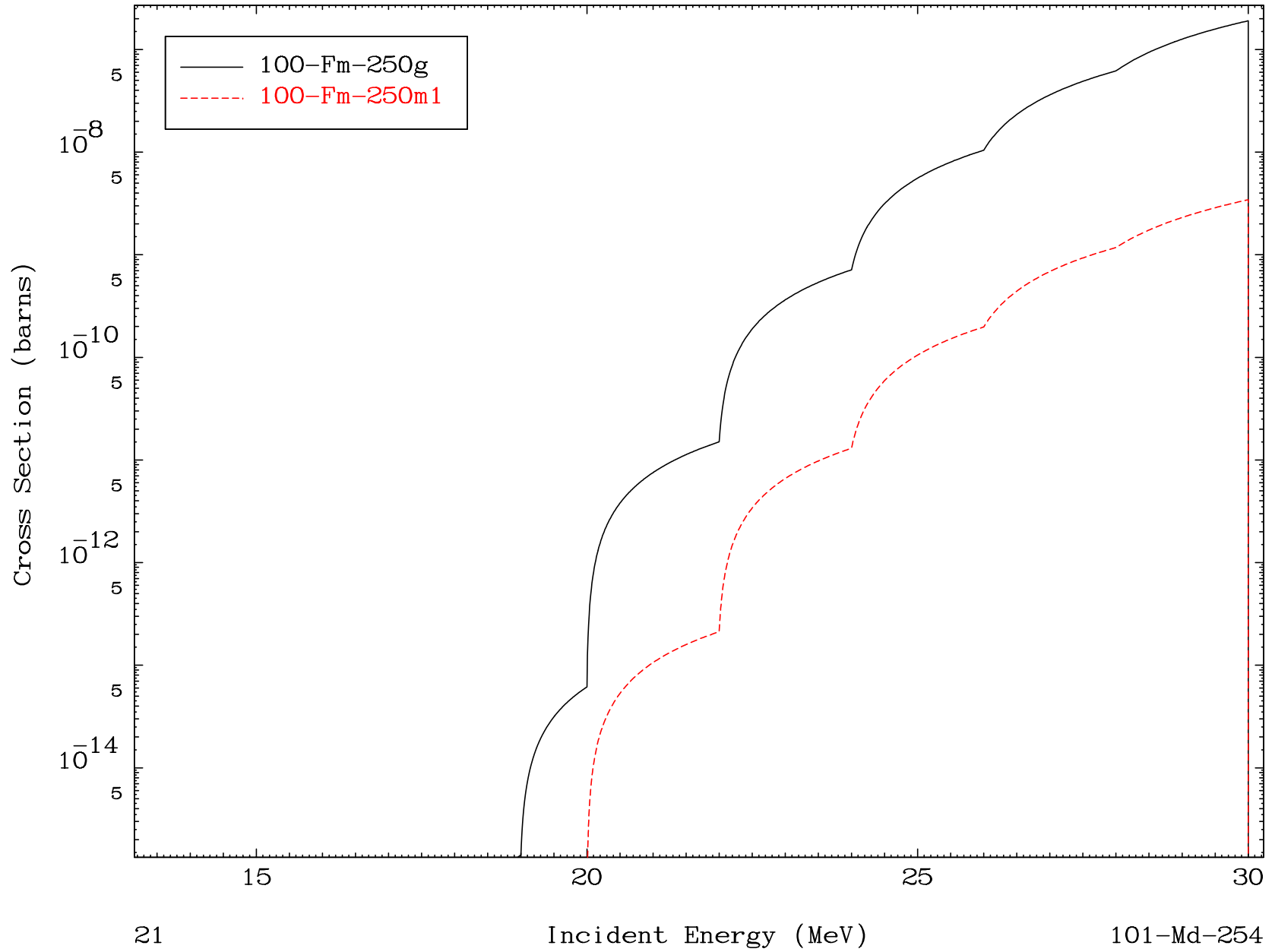


Radionuclide Production Cross Section





Radionuclide Production Cross Section

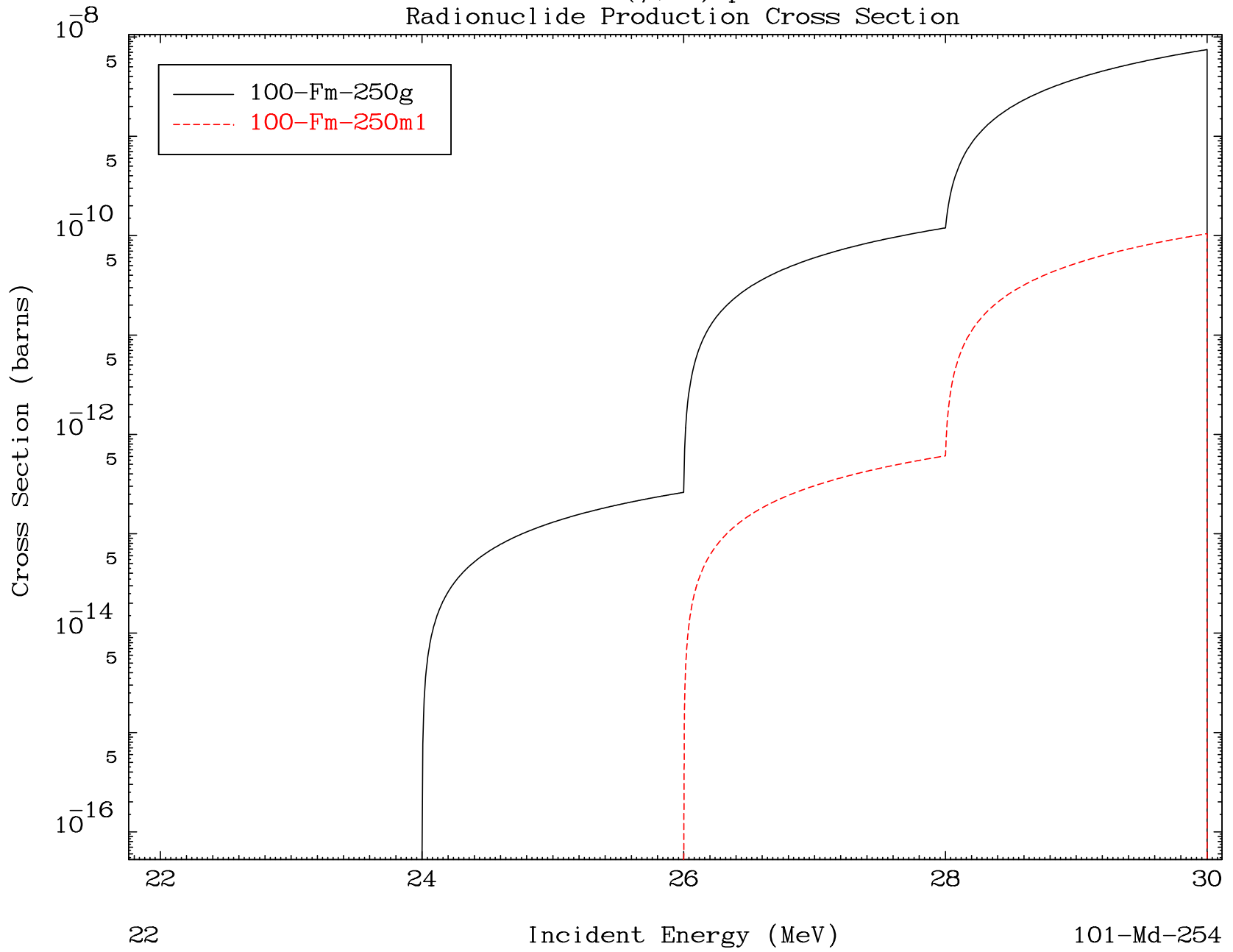


MAT 9960

$(\gamma, 3n) p$

101-Md-254

Radionuclide Production Cross Section

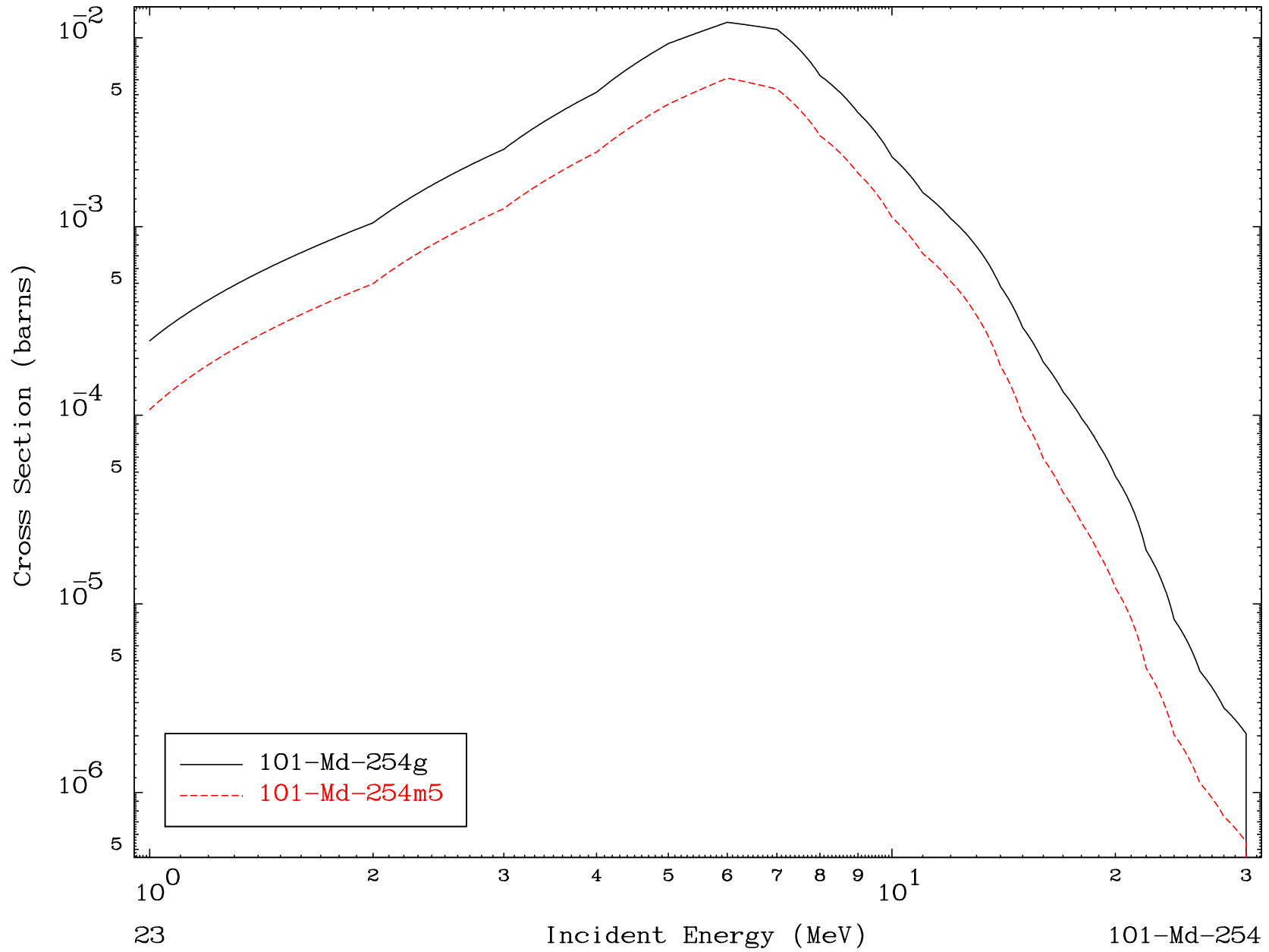


MAT 9960

( $\gamma, \gamma$ )

101-Md-254

Radionuclide Production Cross Section



Radionuclide Production Cross Section

