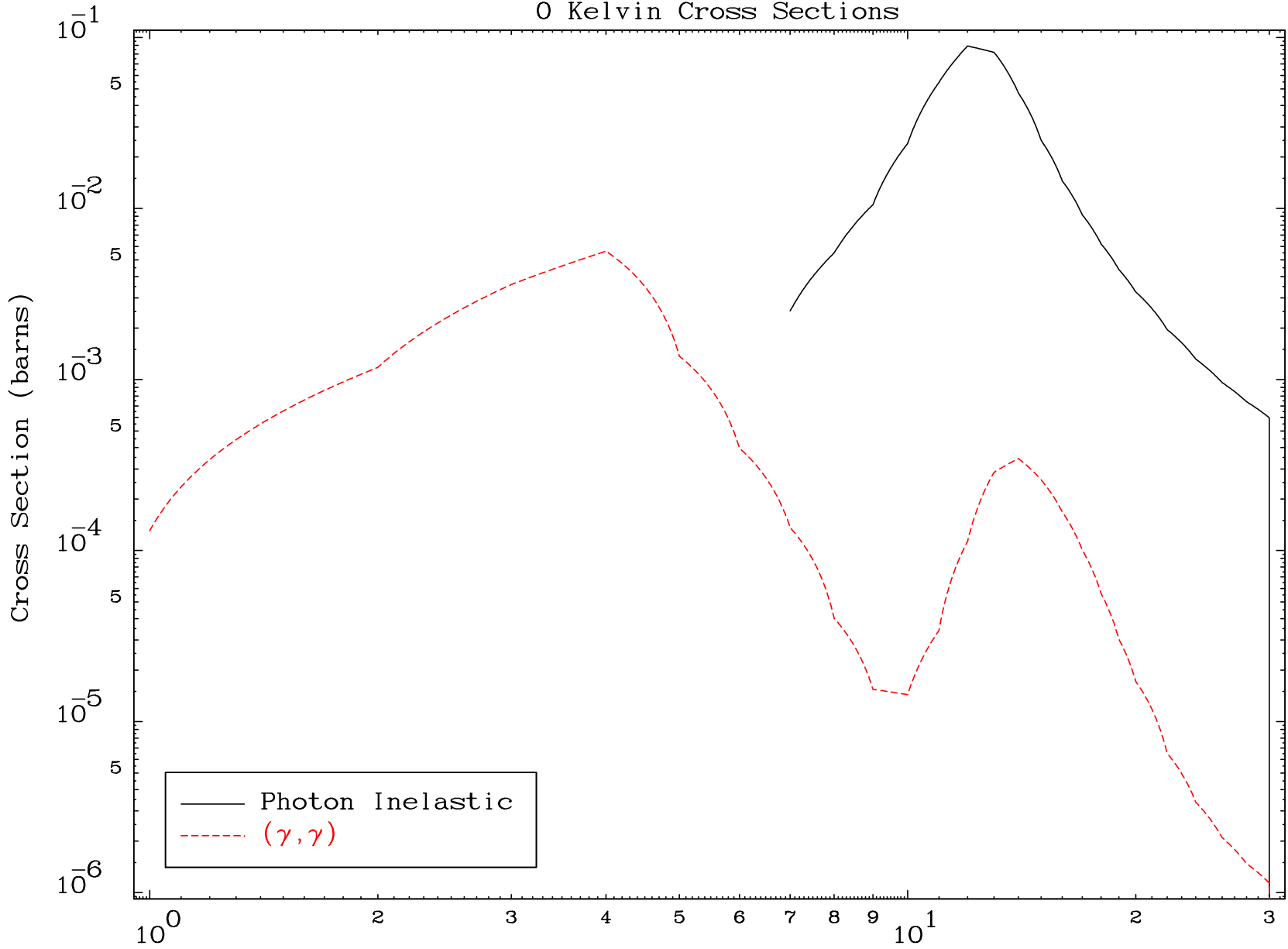


MAT 9962

Photon Major
0 Kelvin Cross Sections

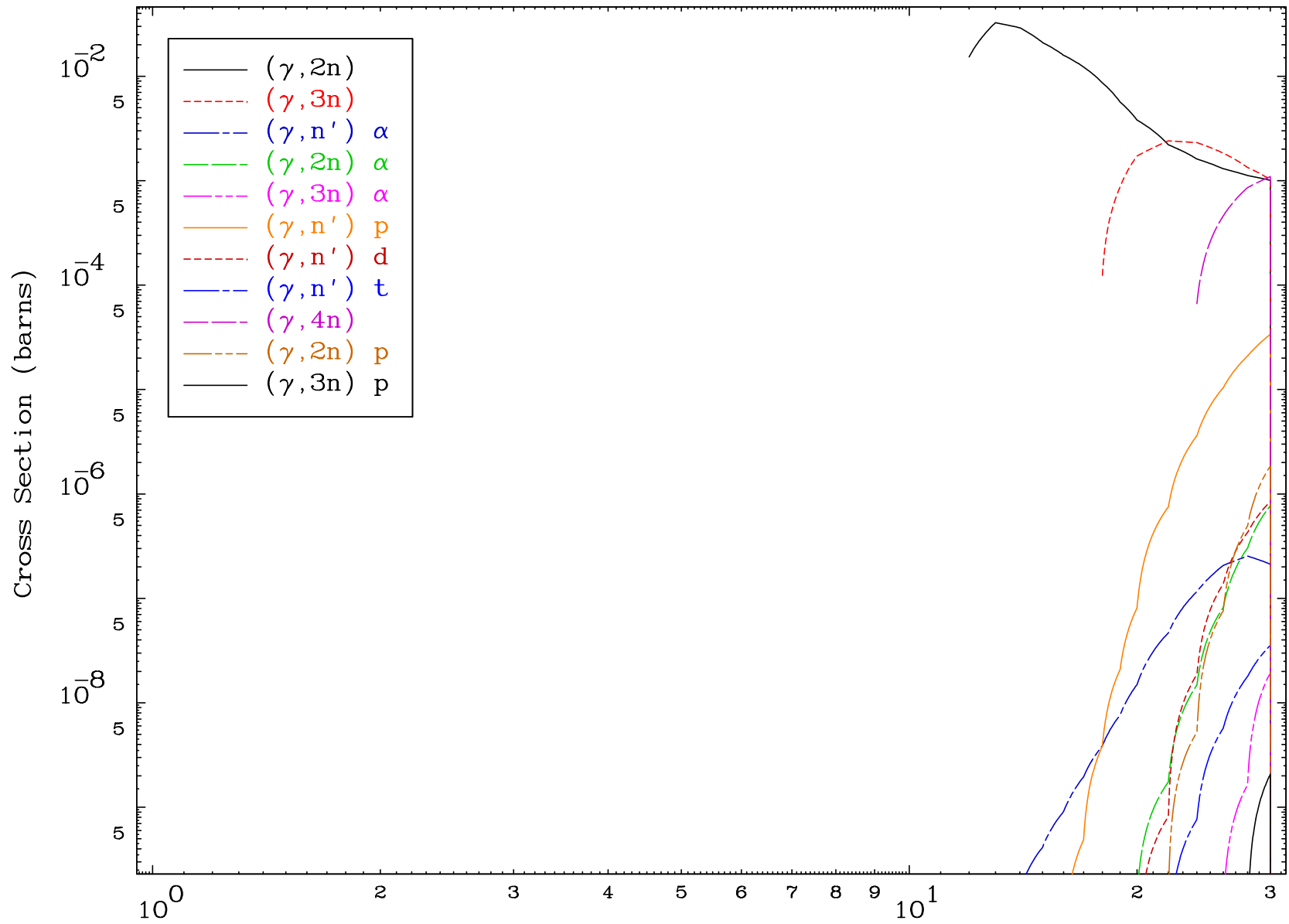
101-Md-261

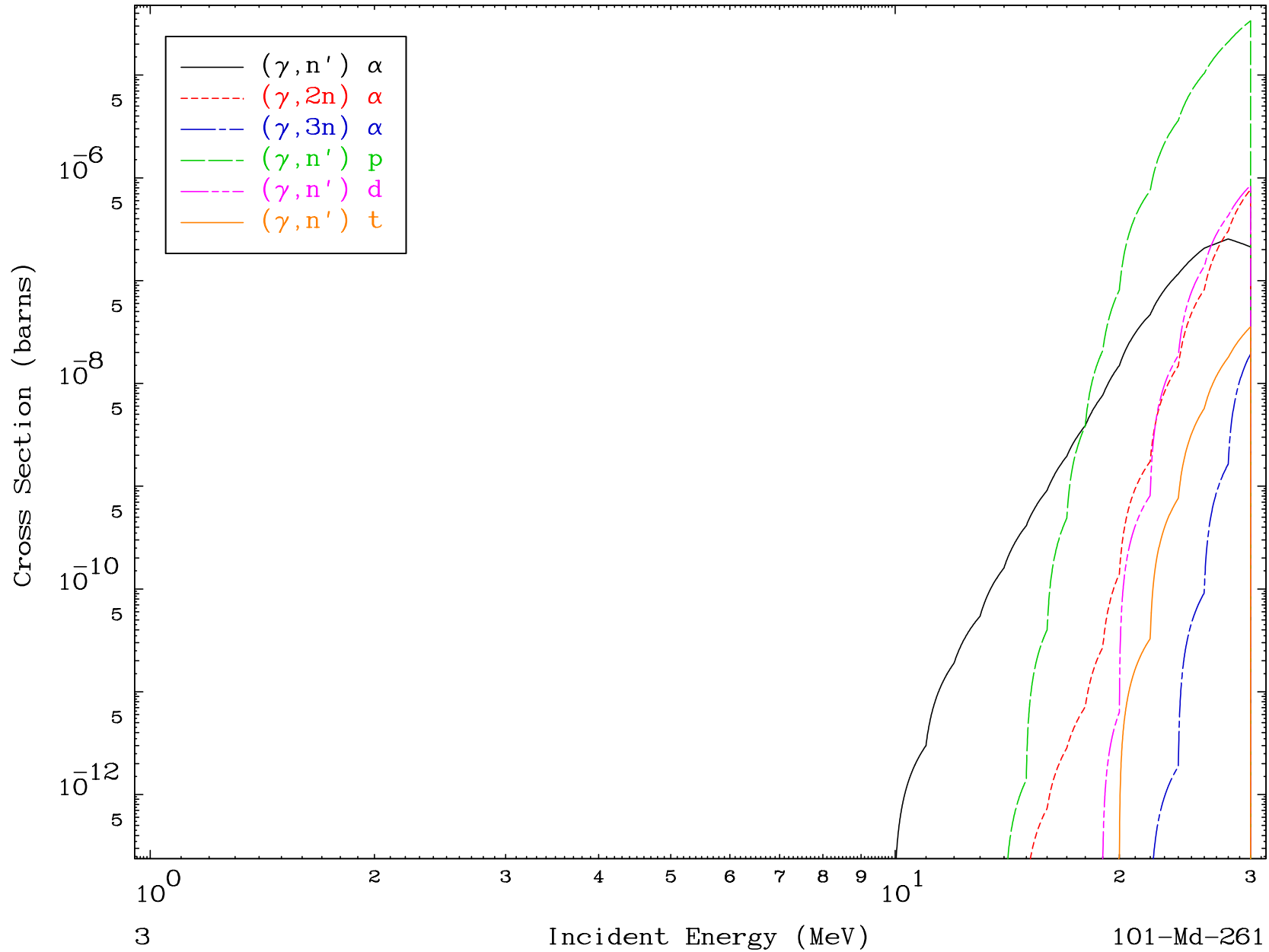


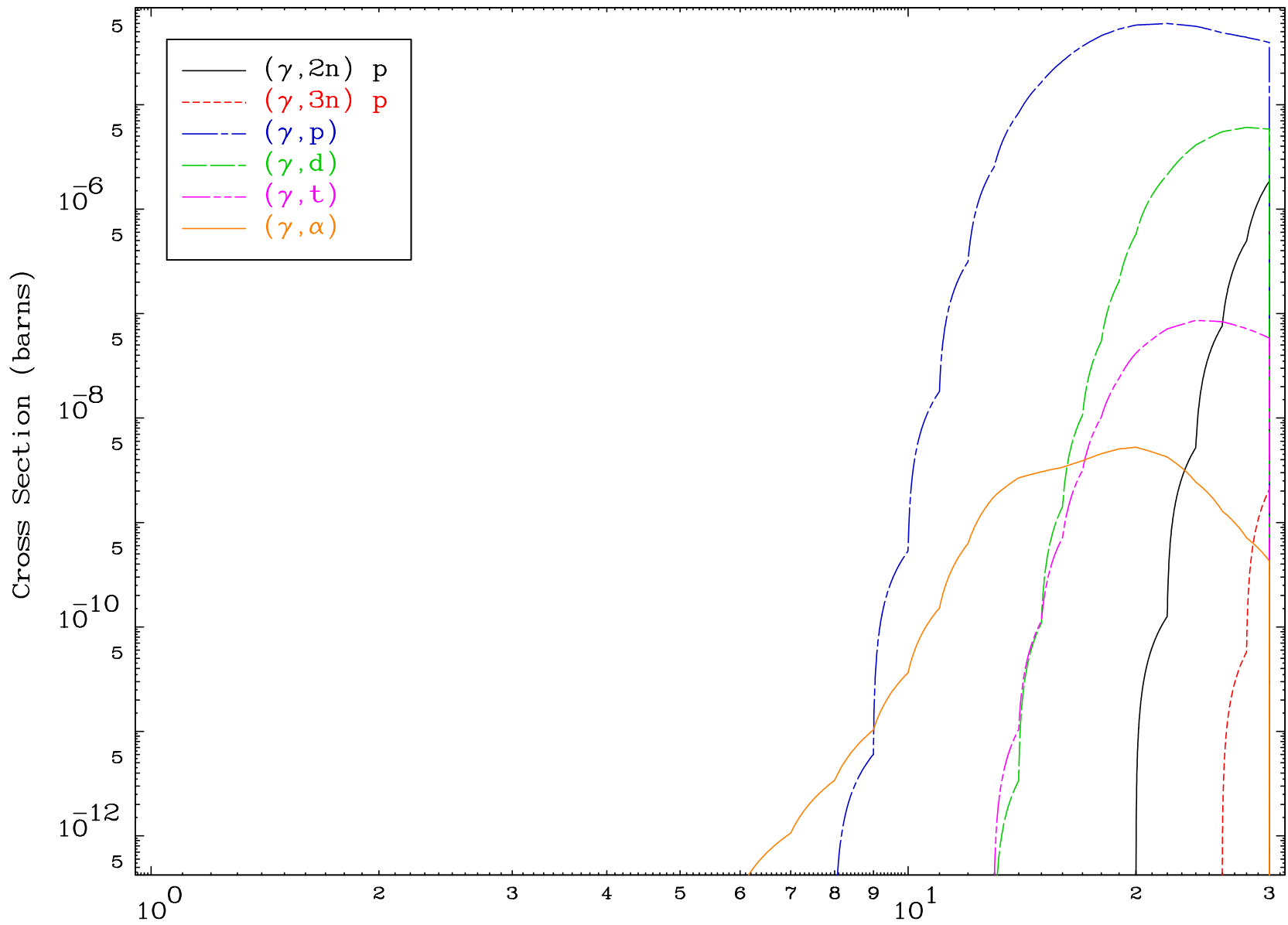
1

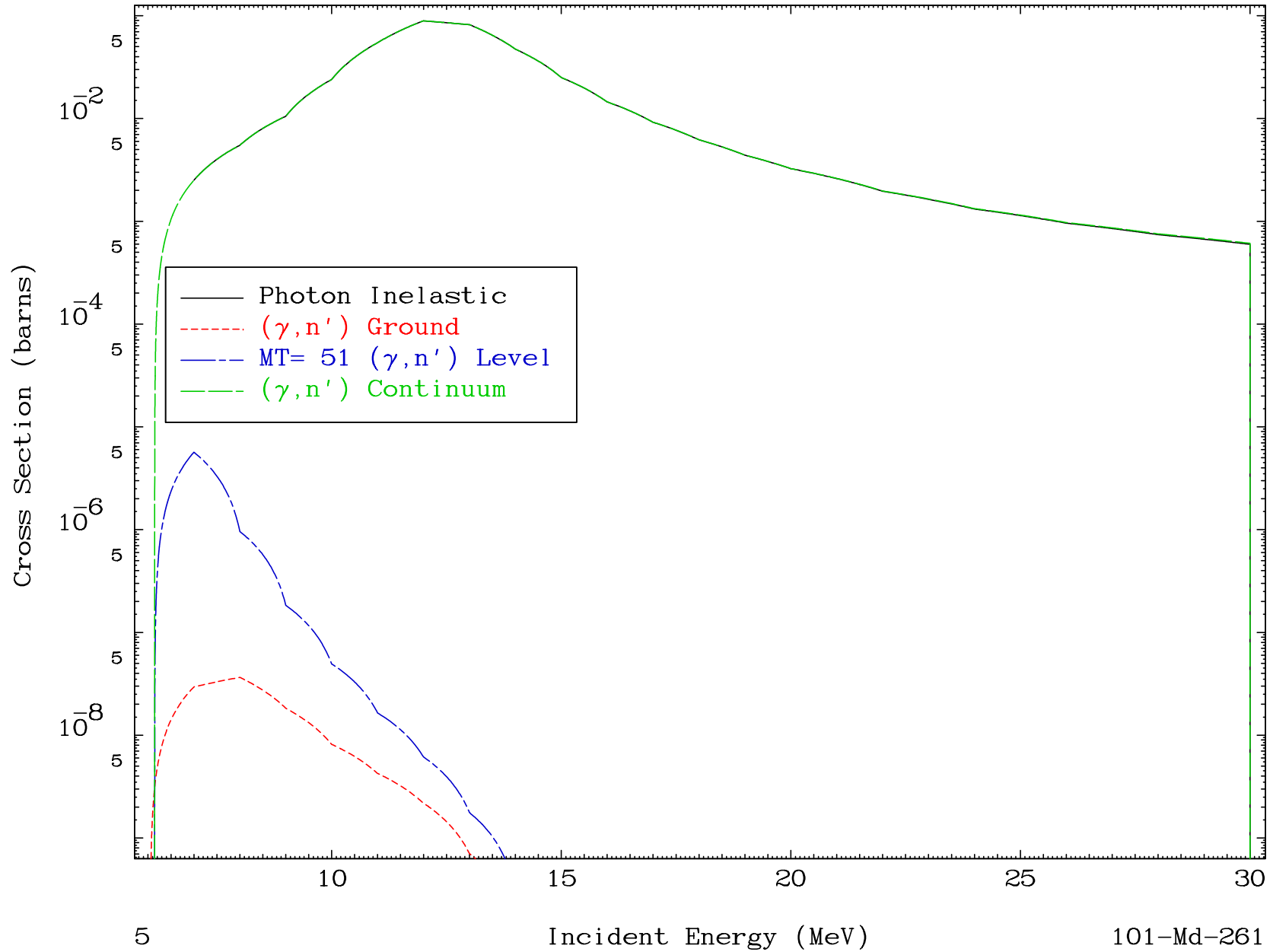
Incident Energy (MeV)

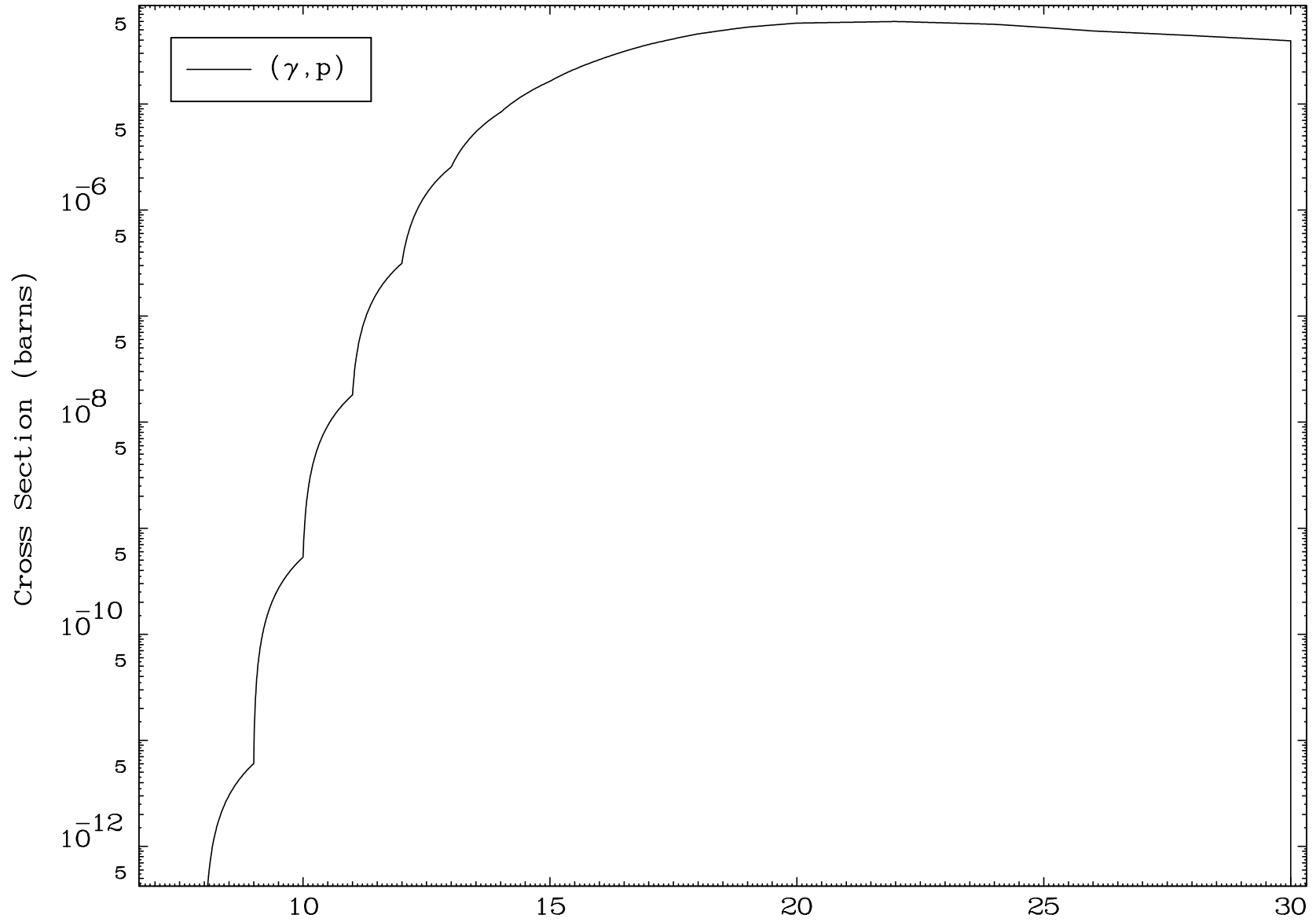
101-Md-261

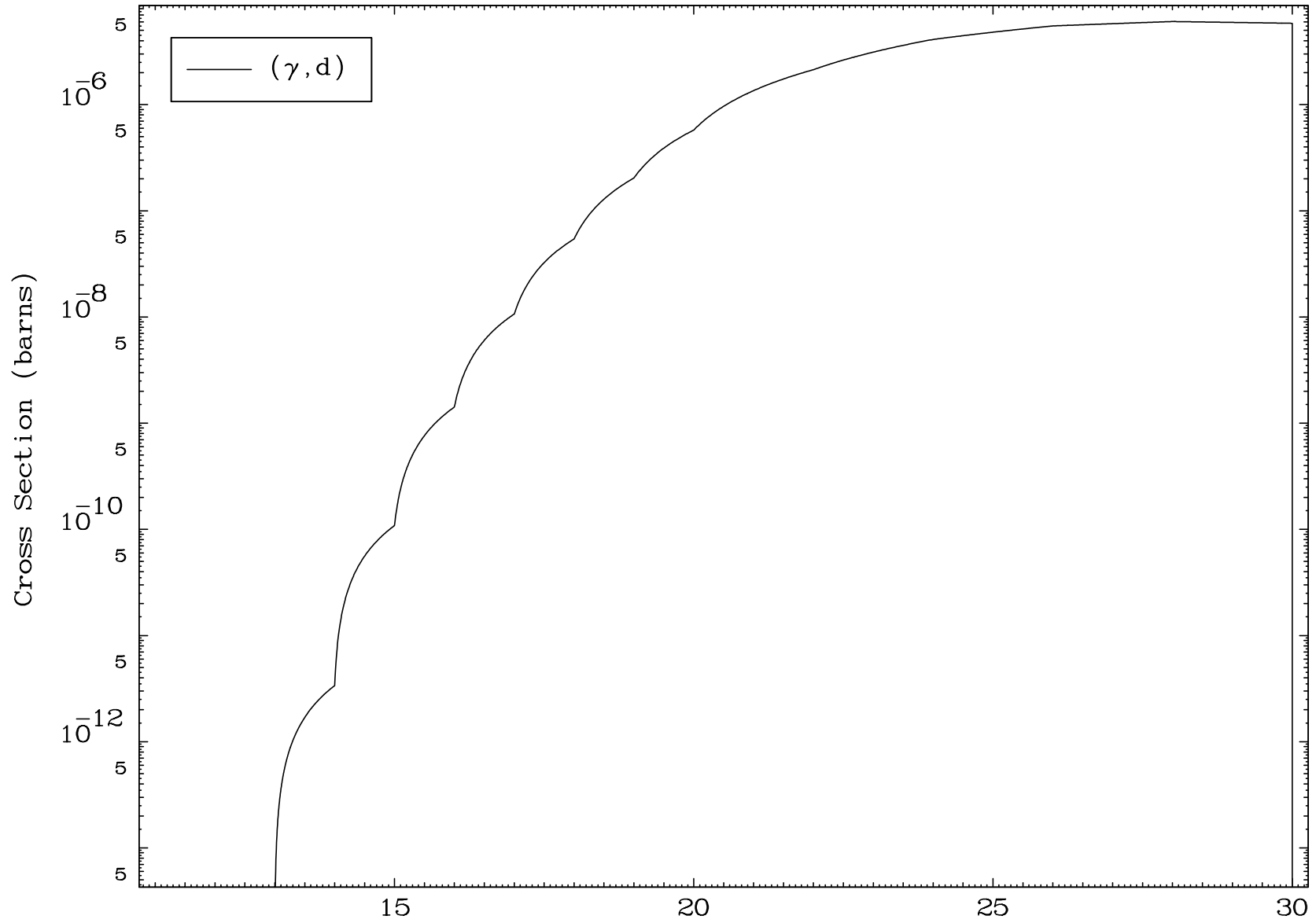


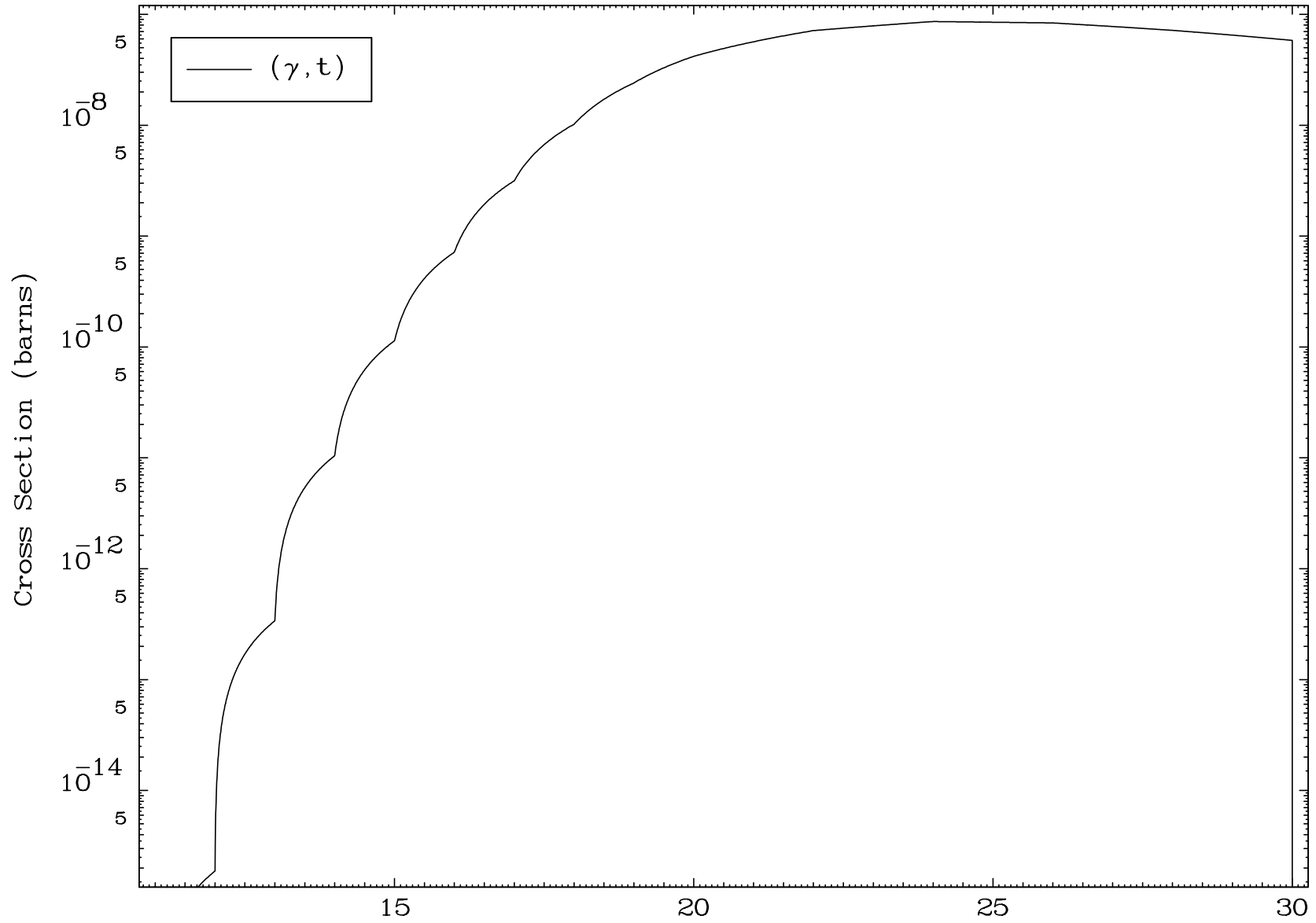


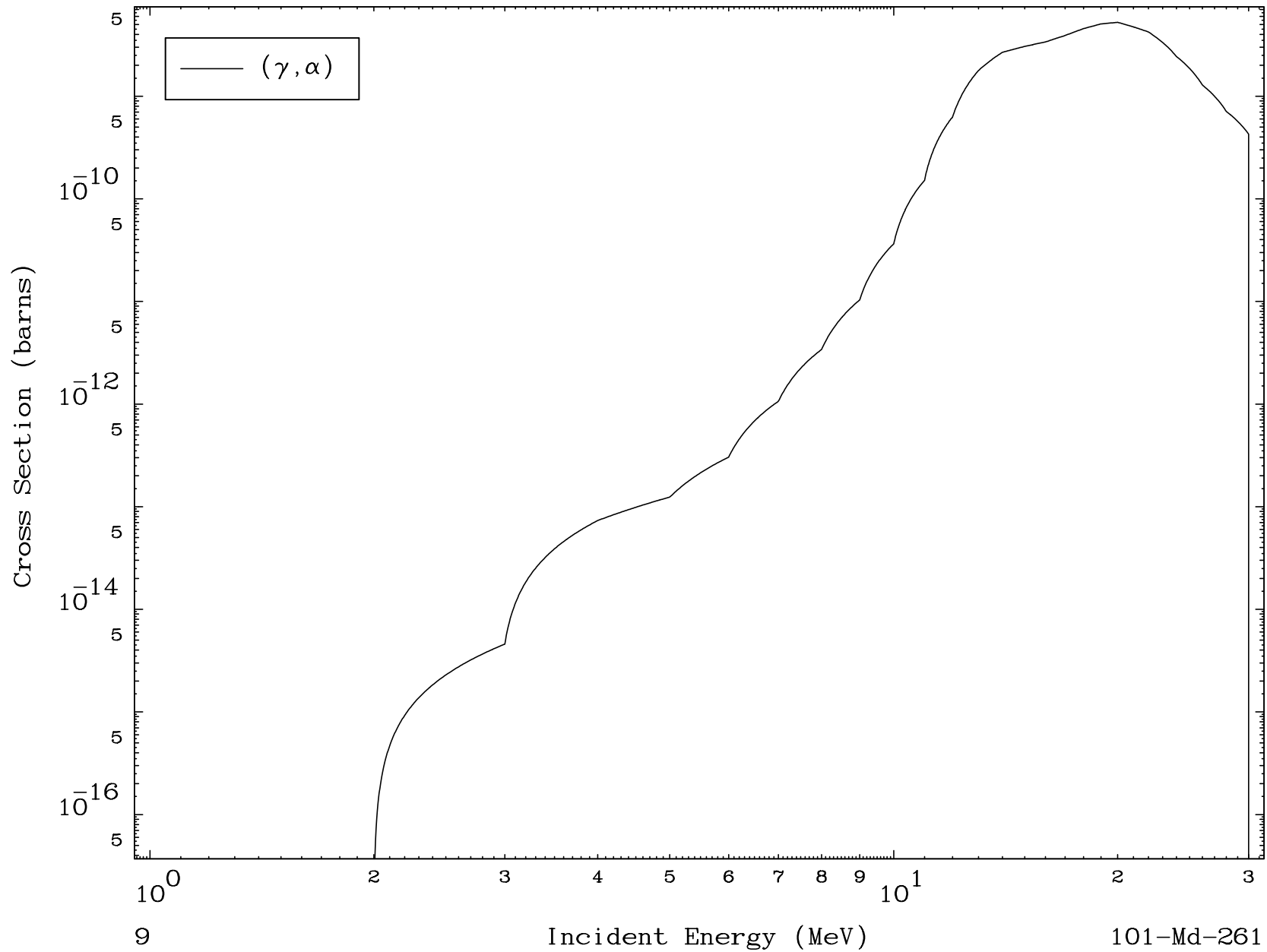


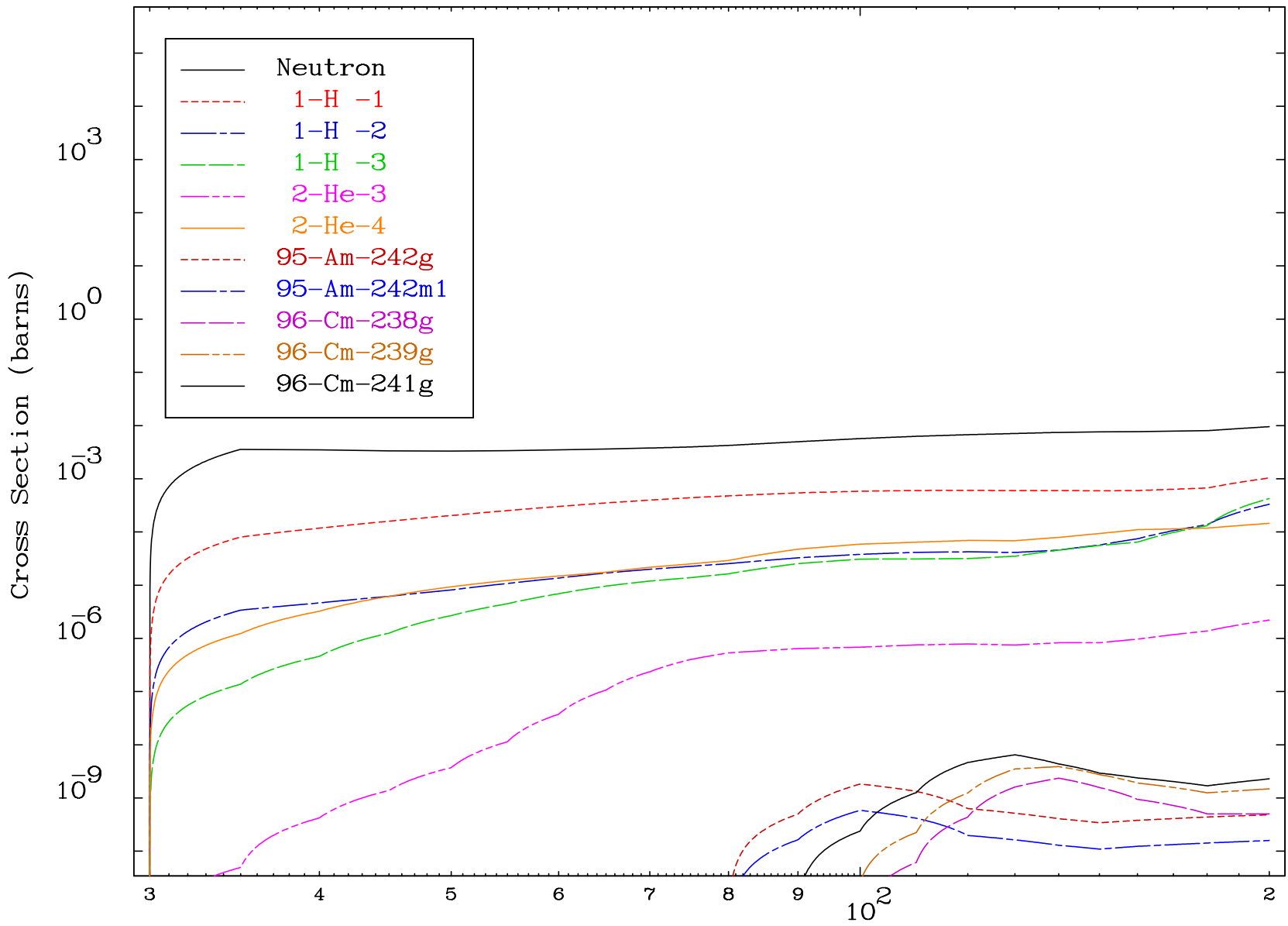




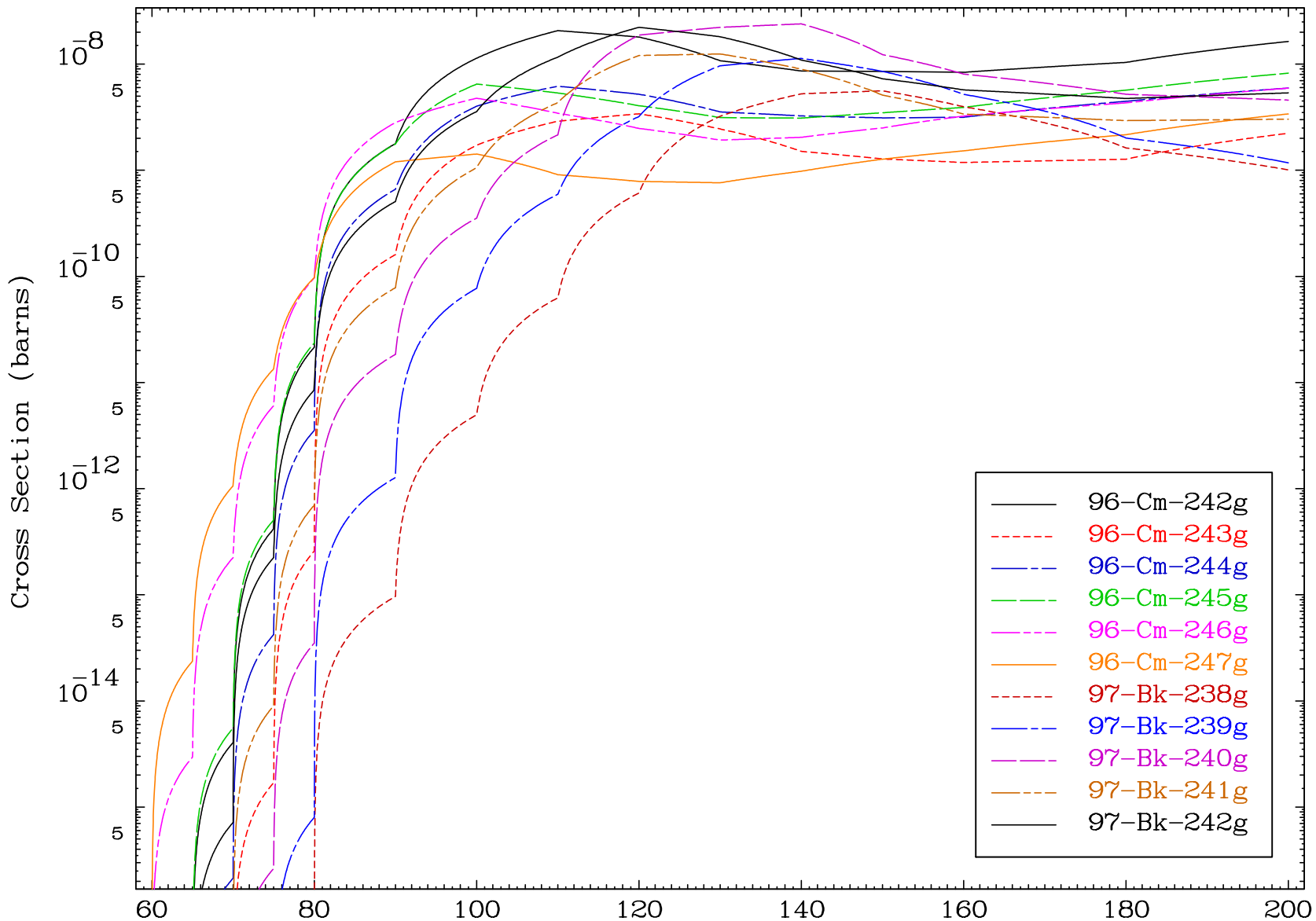


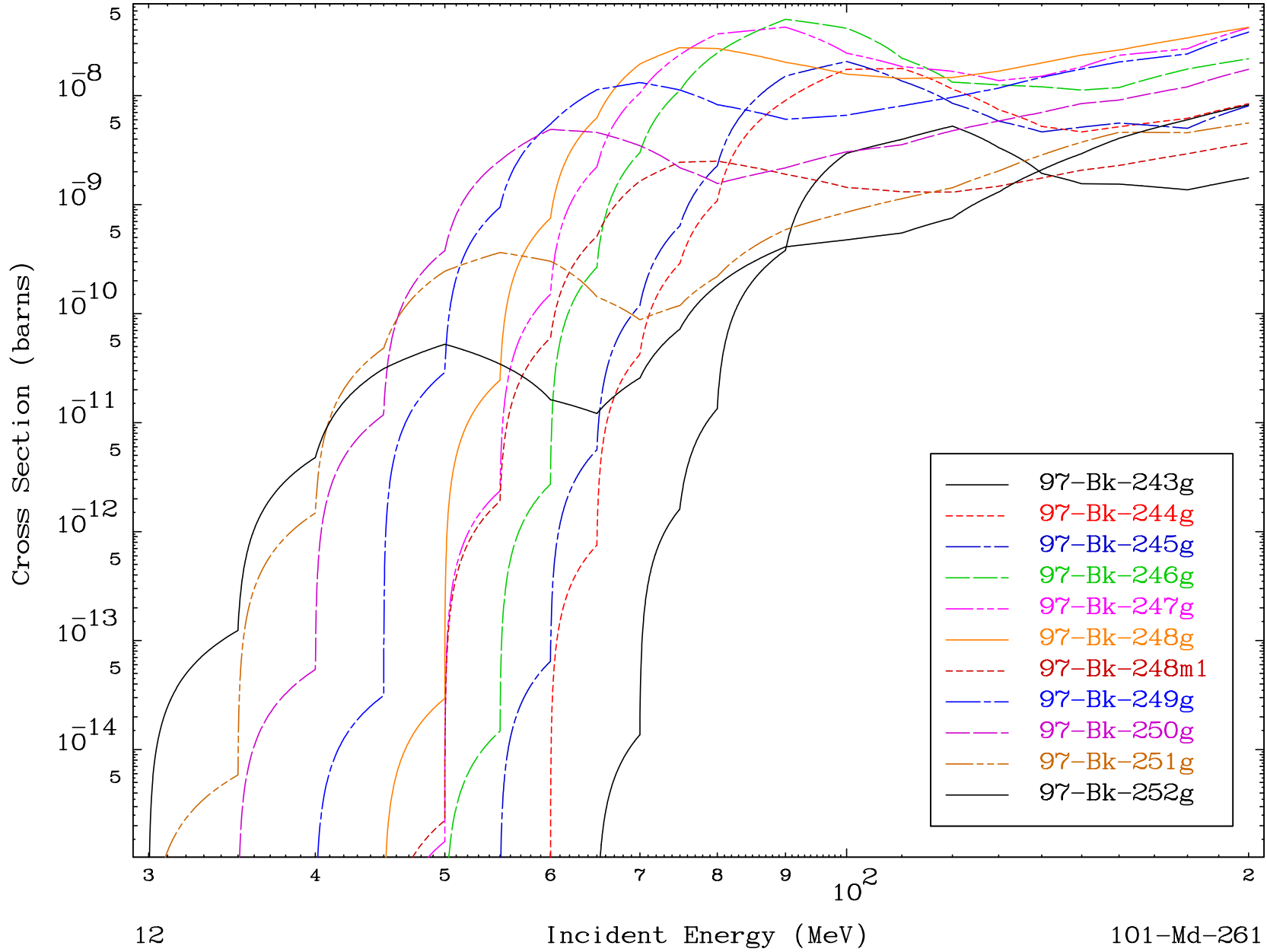




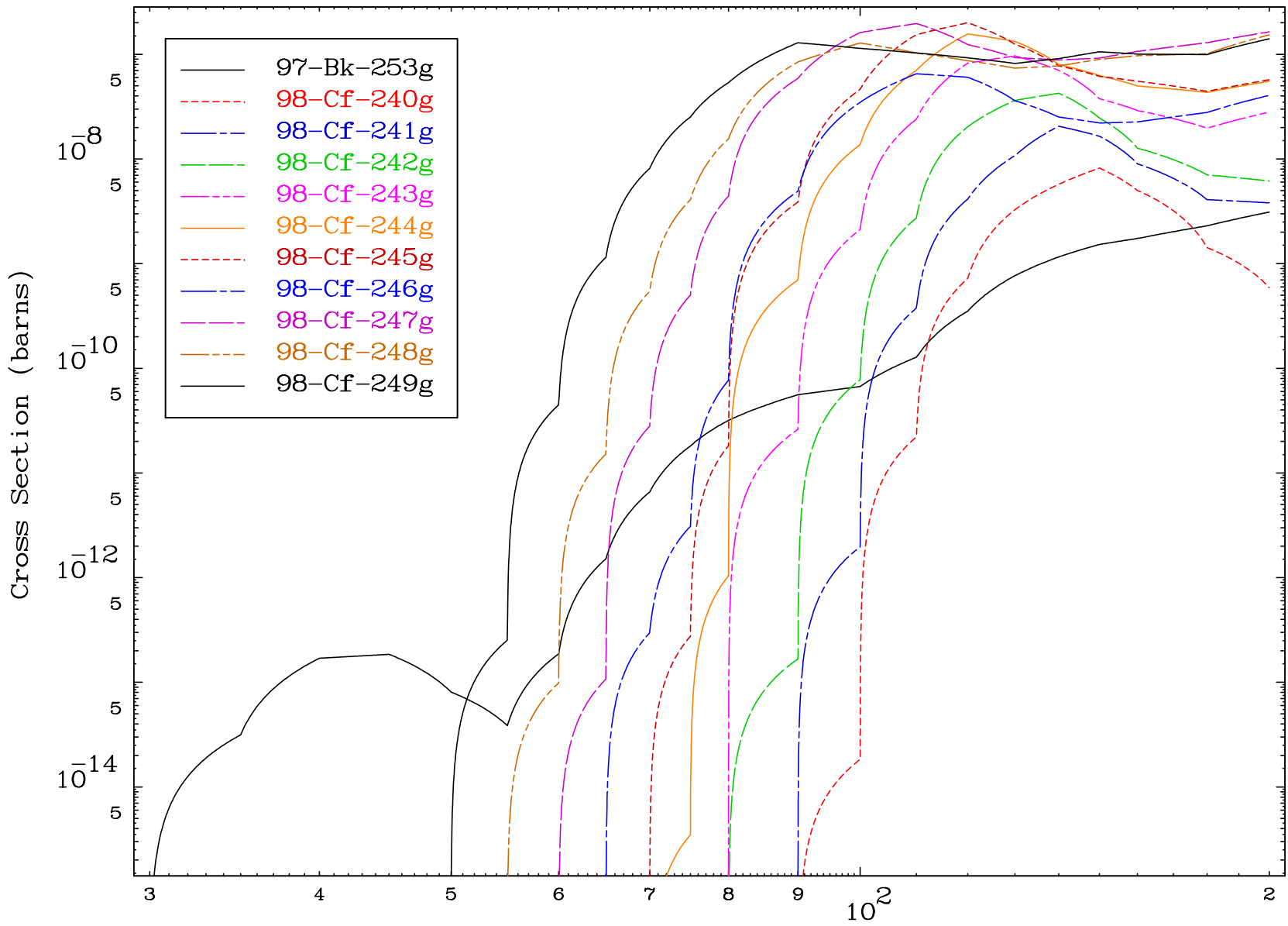


Radionuclide Production Cross Section

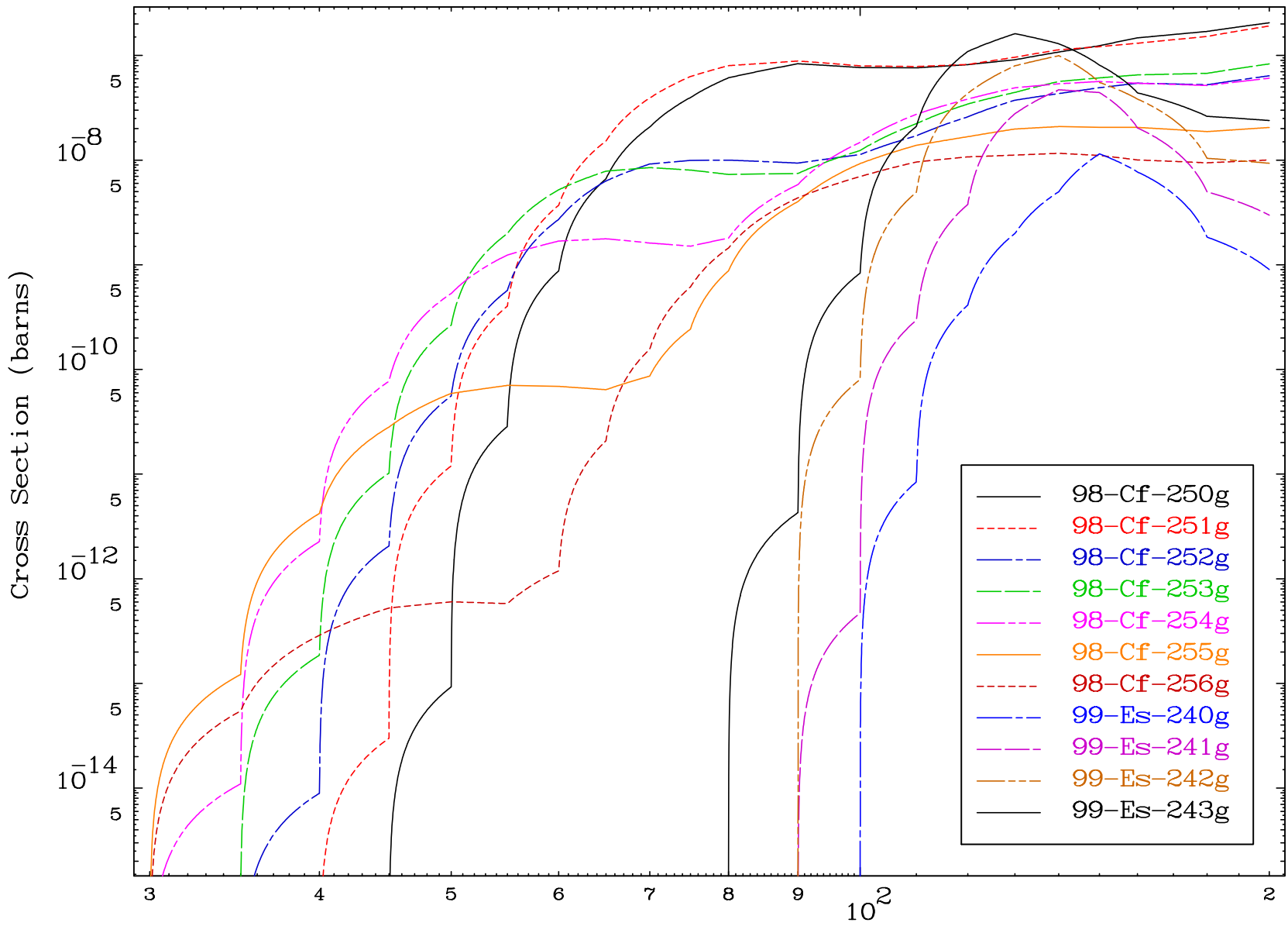




Radionuclide Production Cross Section



Radionuclide Production Cross Section

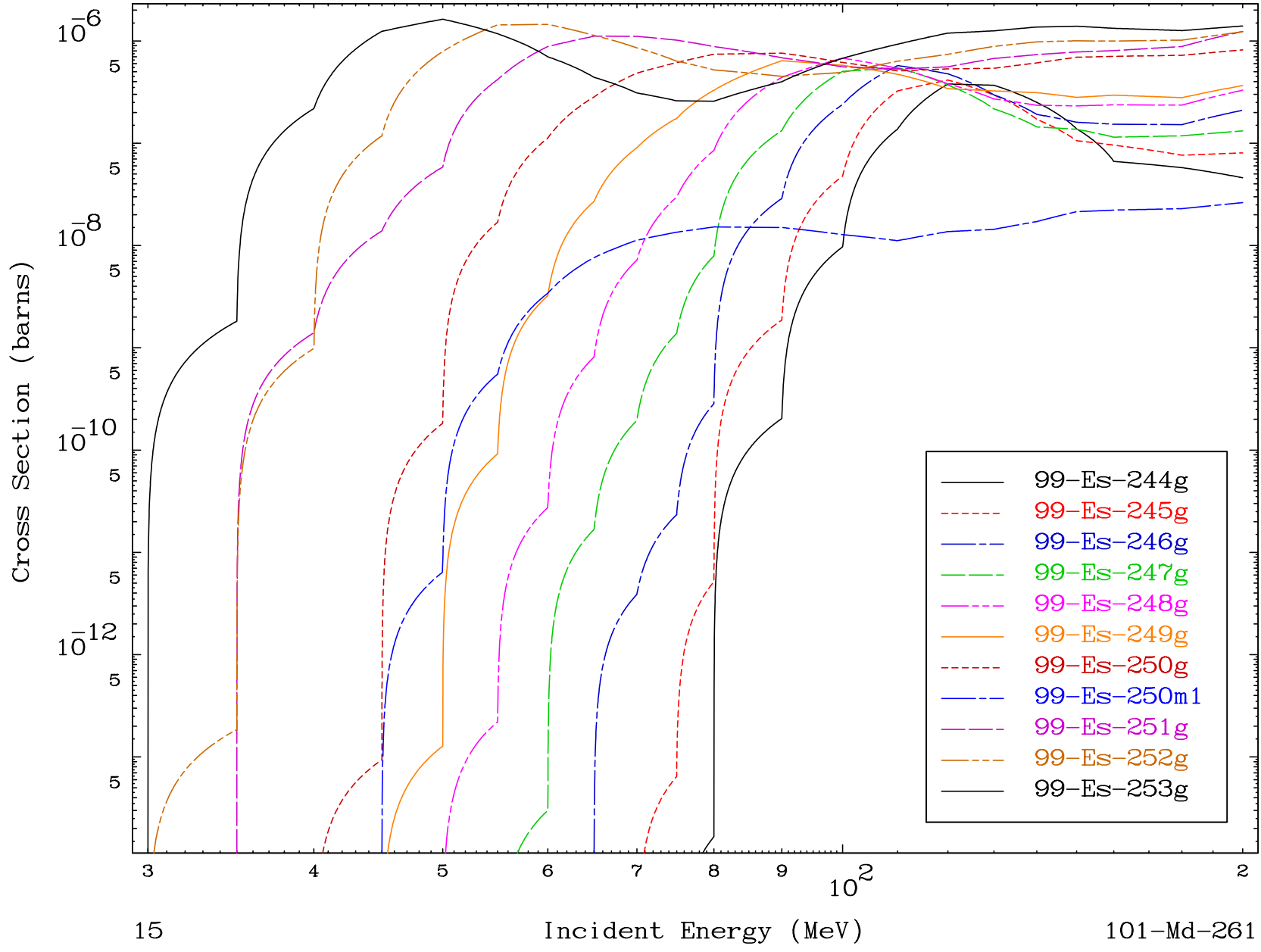


MAT 9962

(γ , remainder)

101-Md-261

Radionuclide Production Cross Section

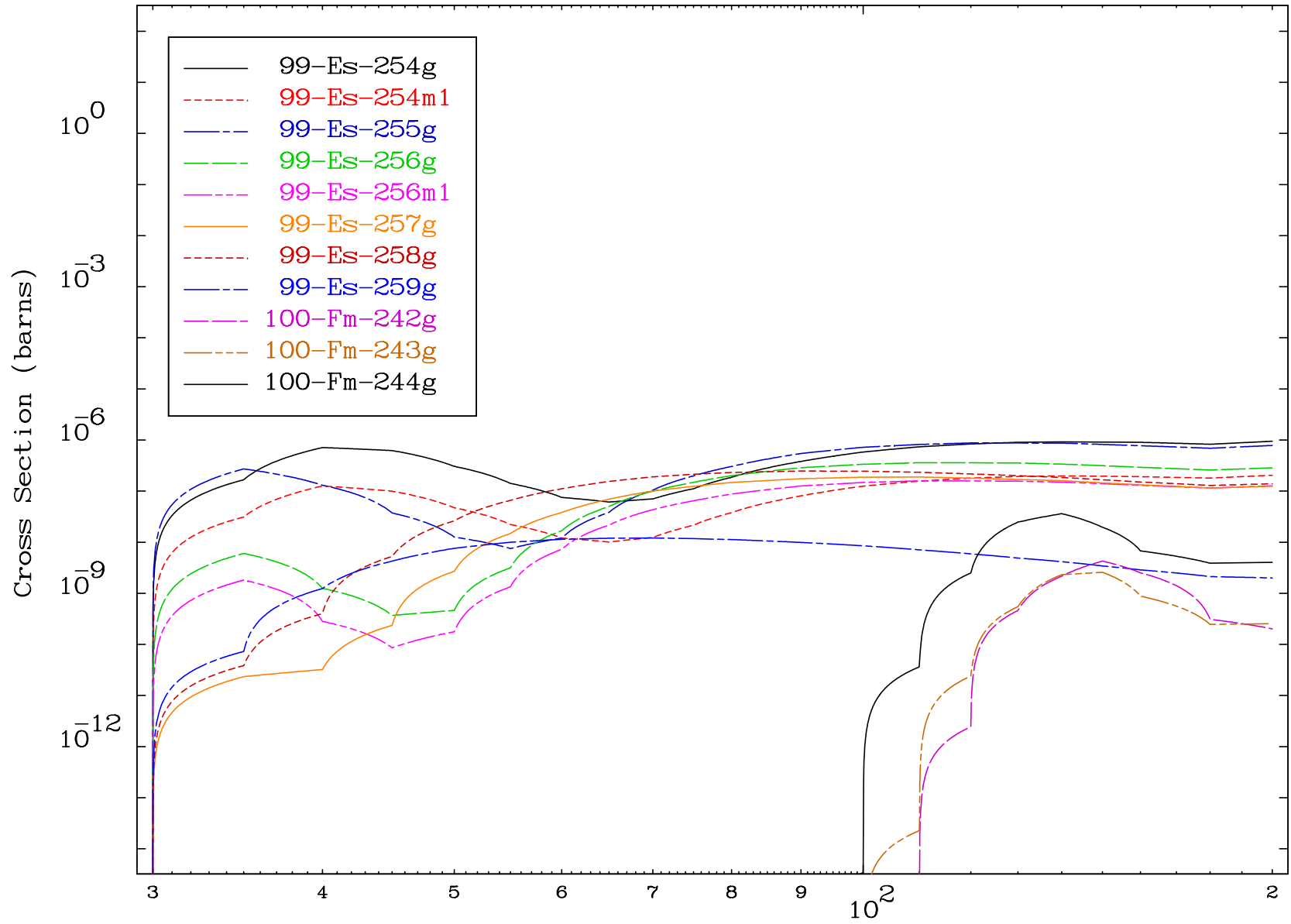


15

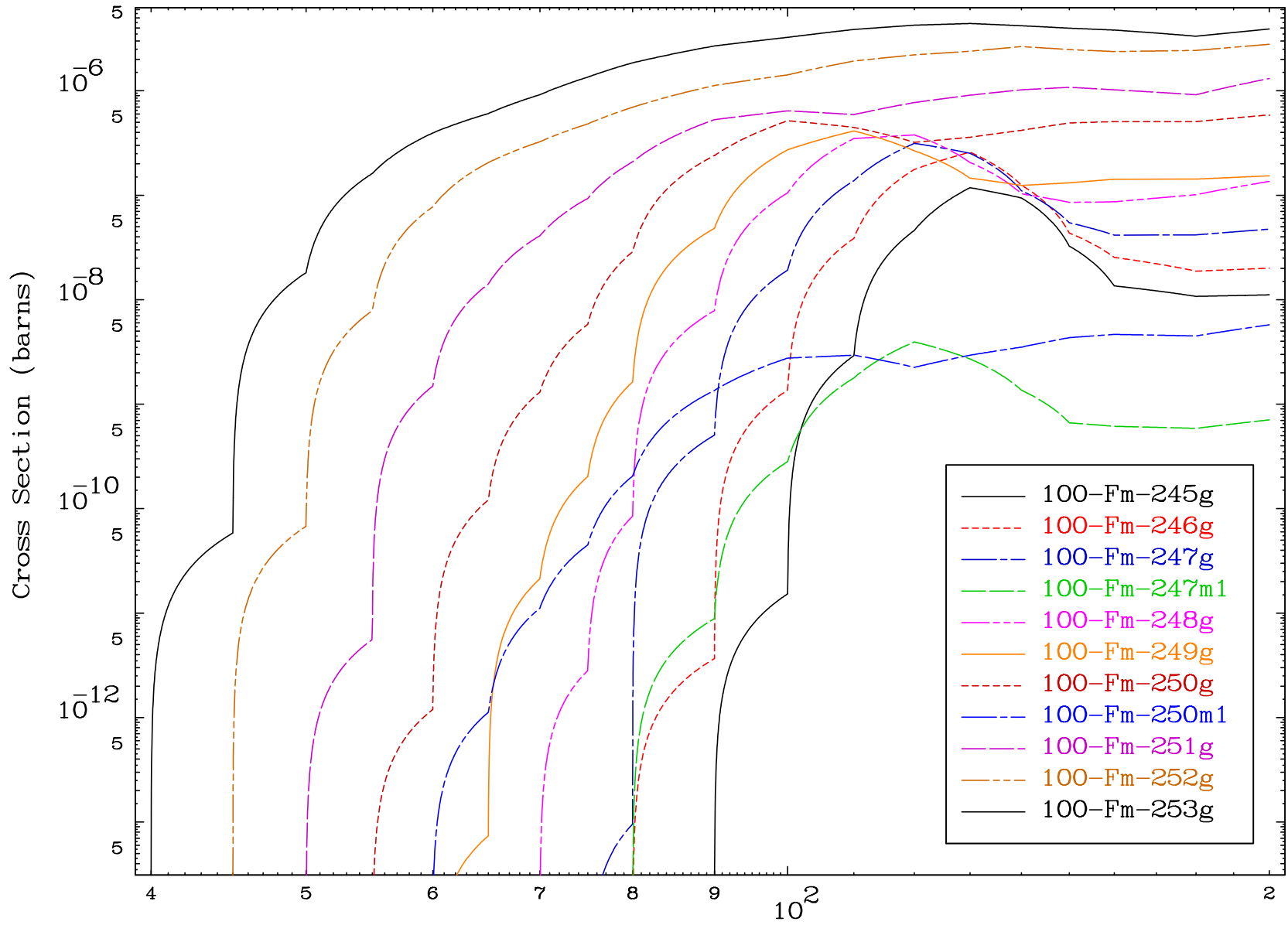
Incident Energy (MeV)

101-Md-261

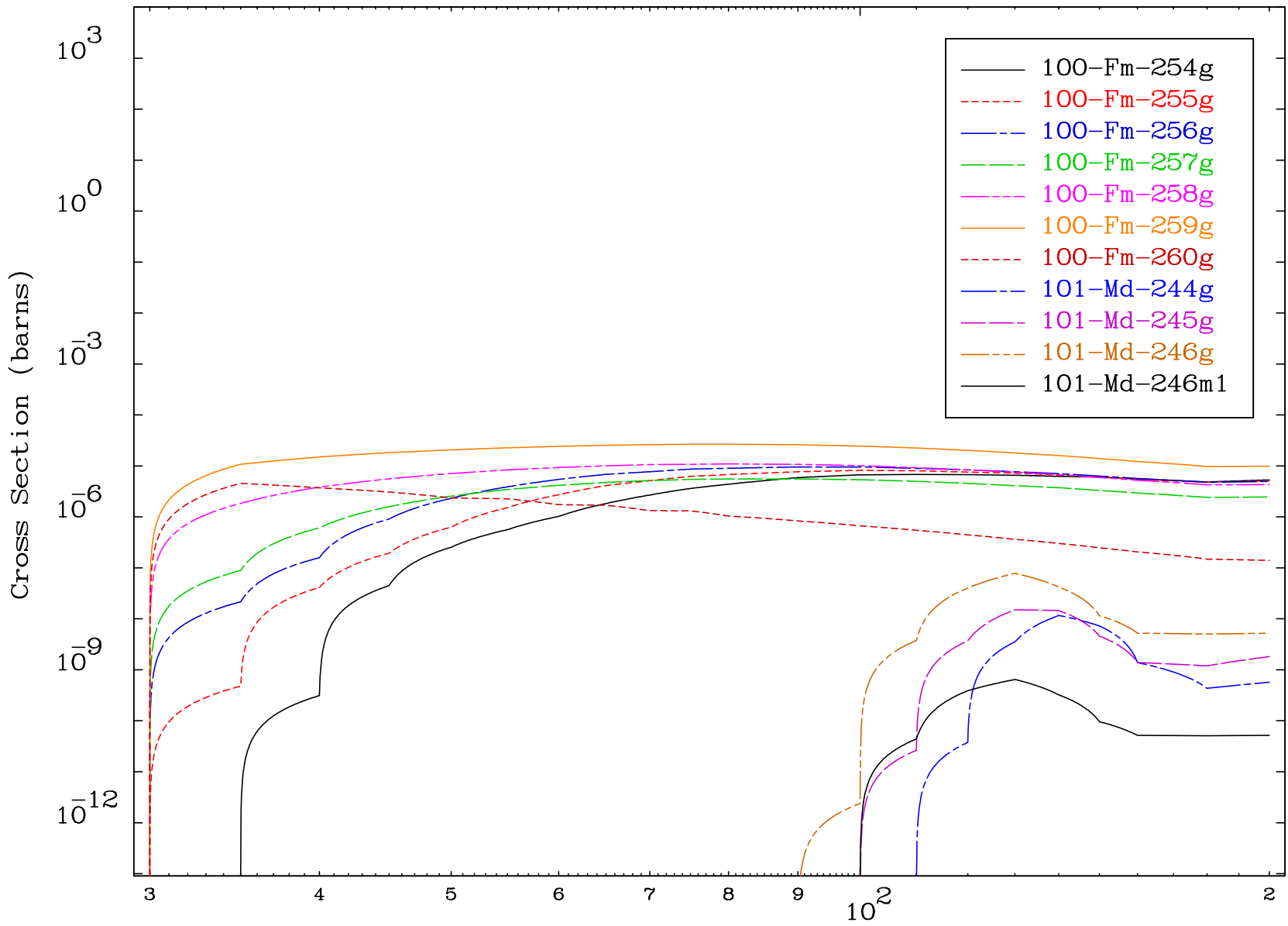
Radionuclide Production Cross Section

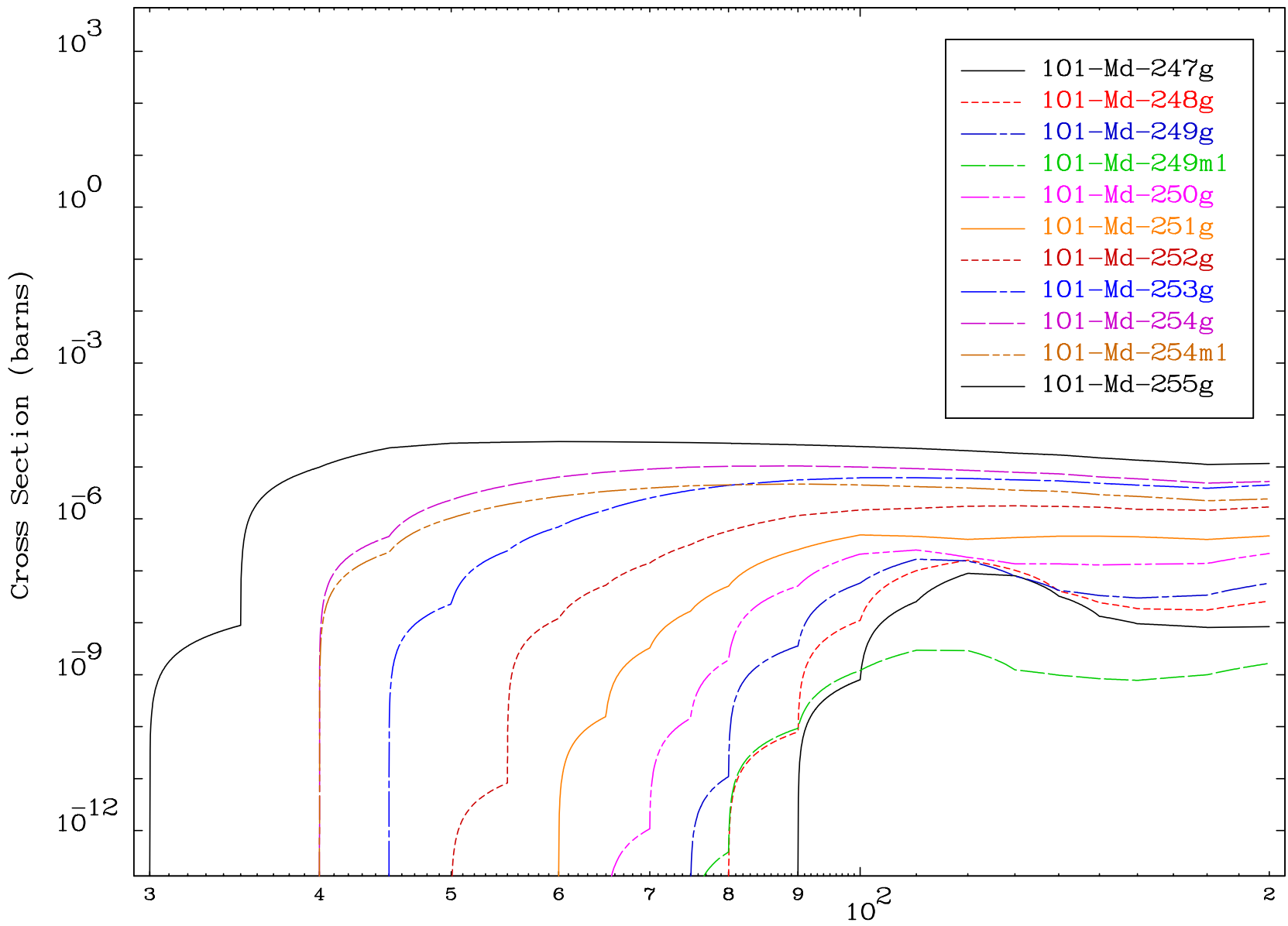


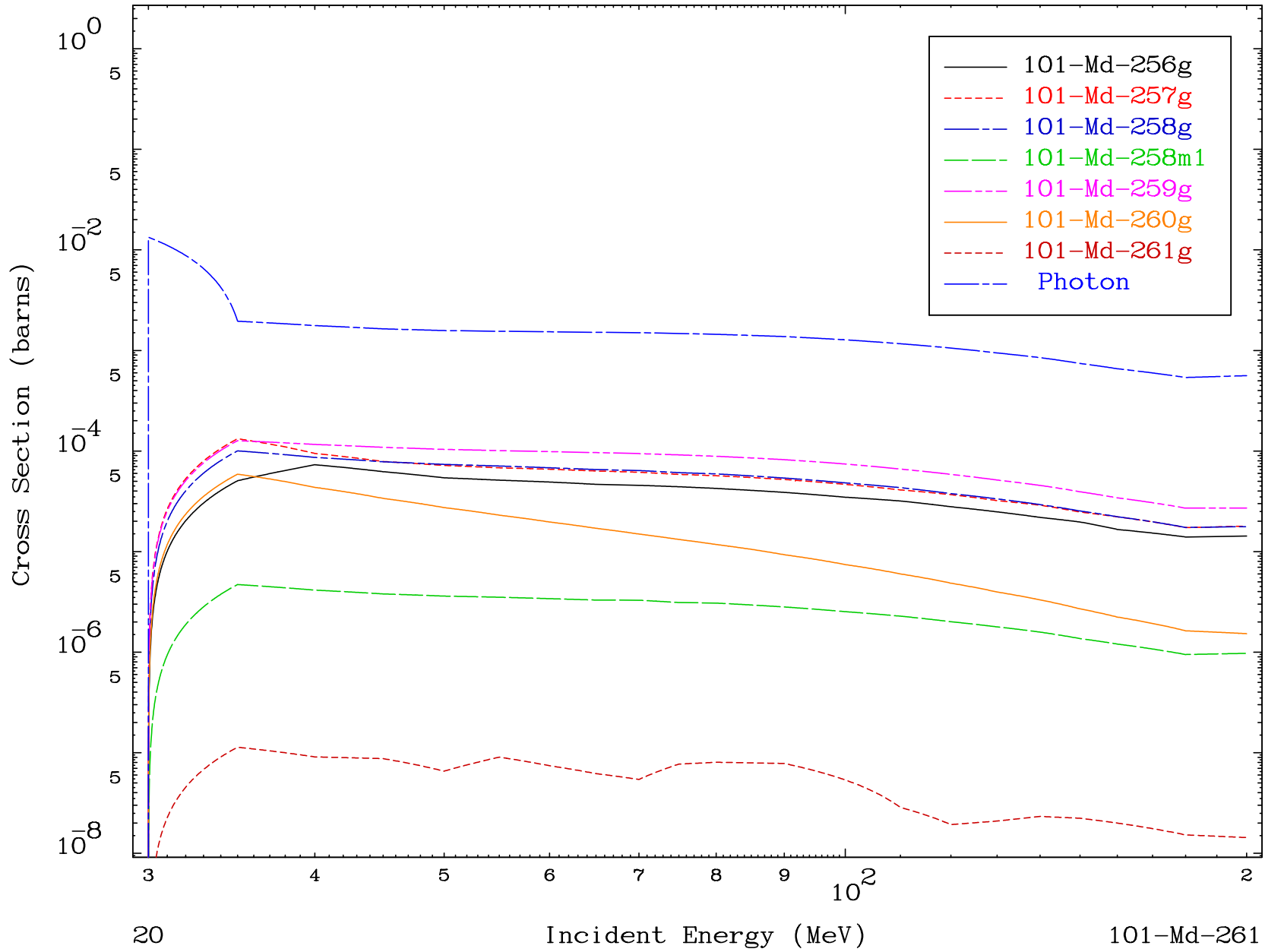
Radionuclide Production Cross Section



Radionuclide Production Cross Section





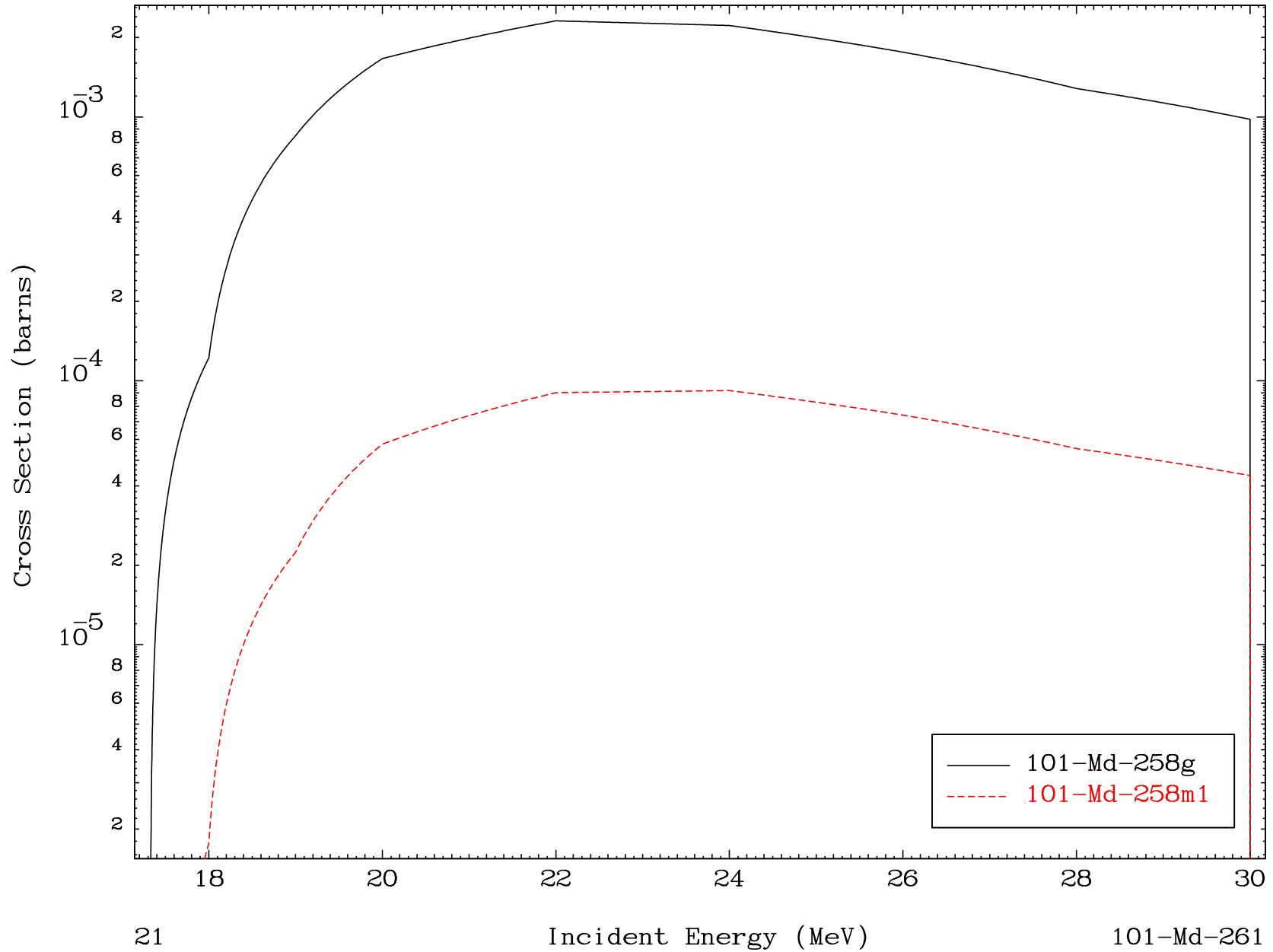


MAT 9962

($\gamma, 3n$)

101-Md-261

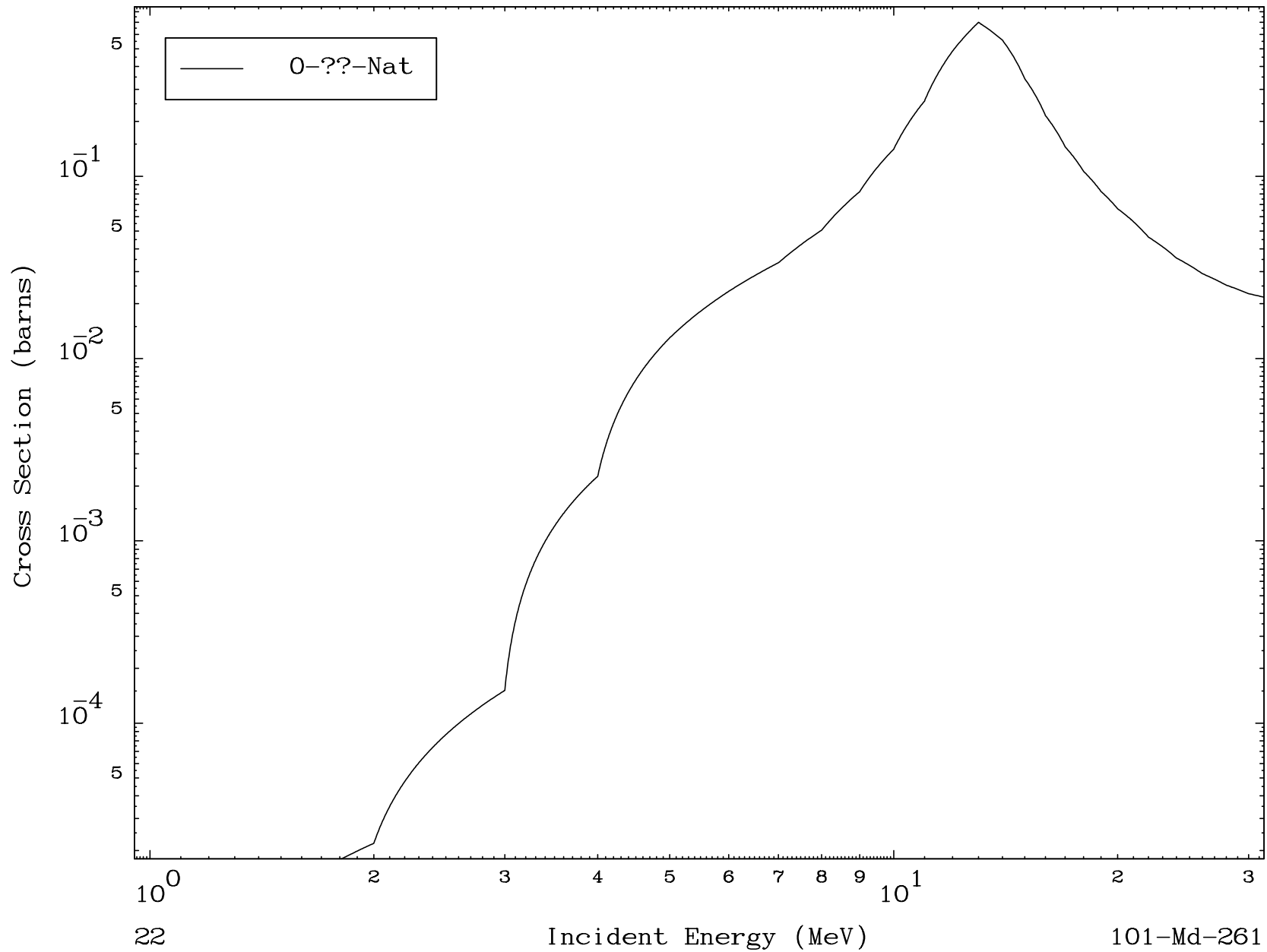
Radionuclide Production Cross Section



21

Incident Energy (MeV)

101-Md-261

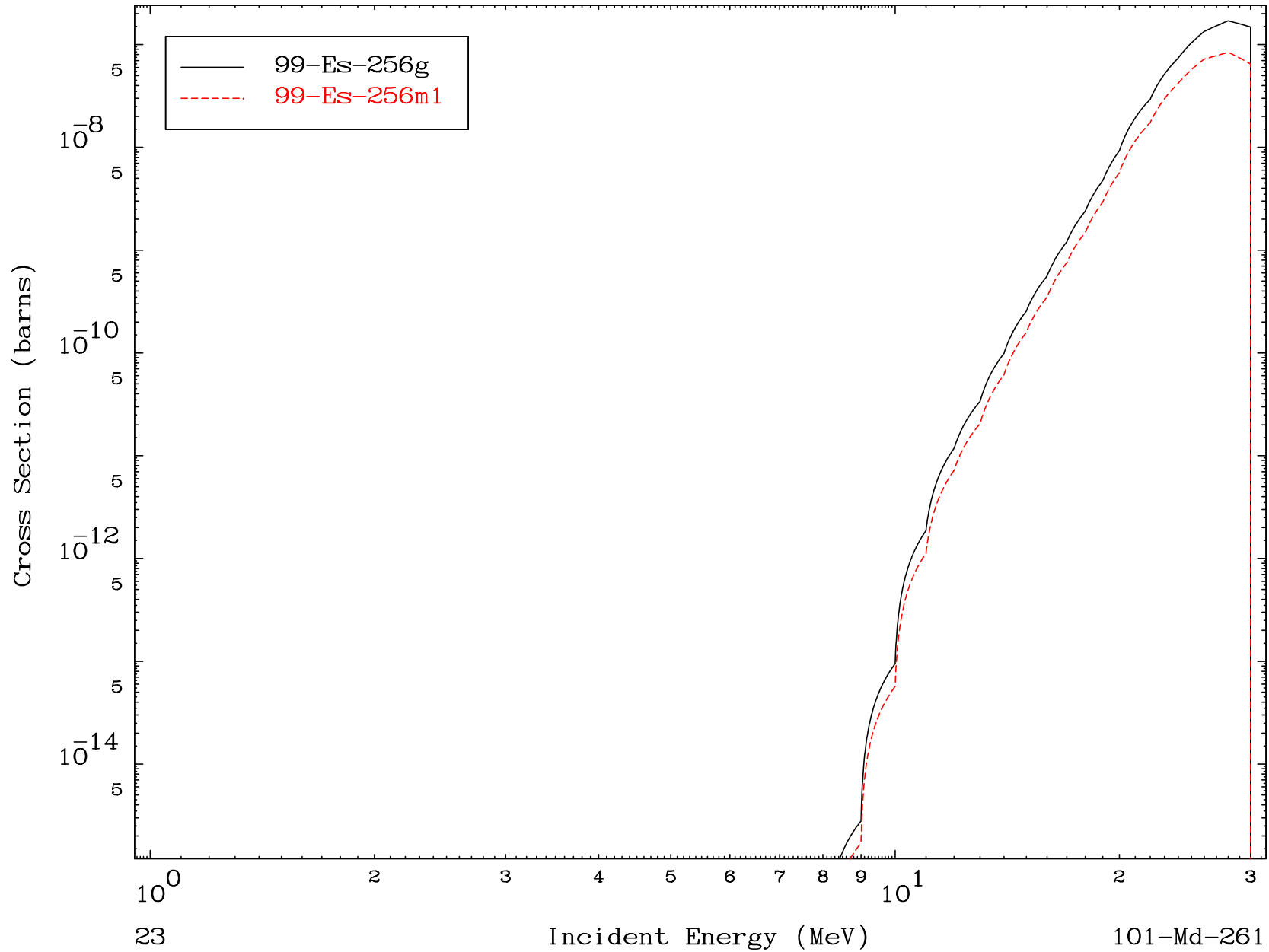


MAT 9962

(γ, n') α

101-Md-261

Radionuclide Production Cross Section



MAT 9962

$(\gamma, 3n) \alpha$

101-Md-261

Radionuclide Production Cross Section

