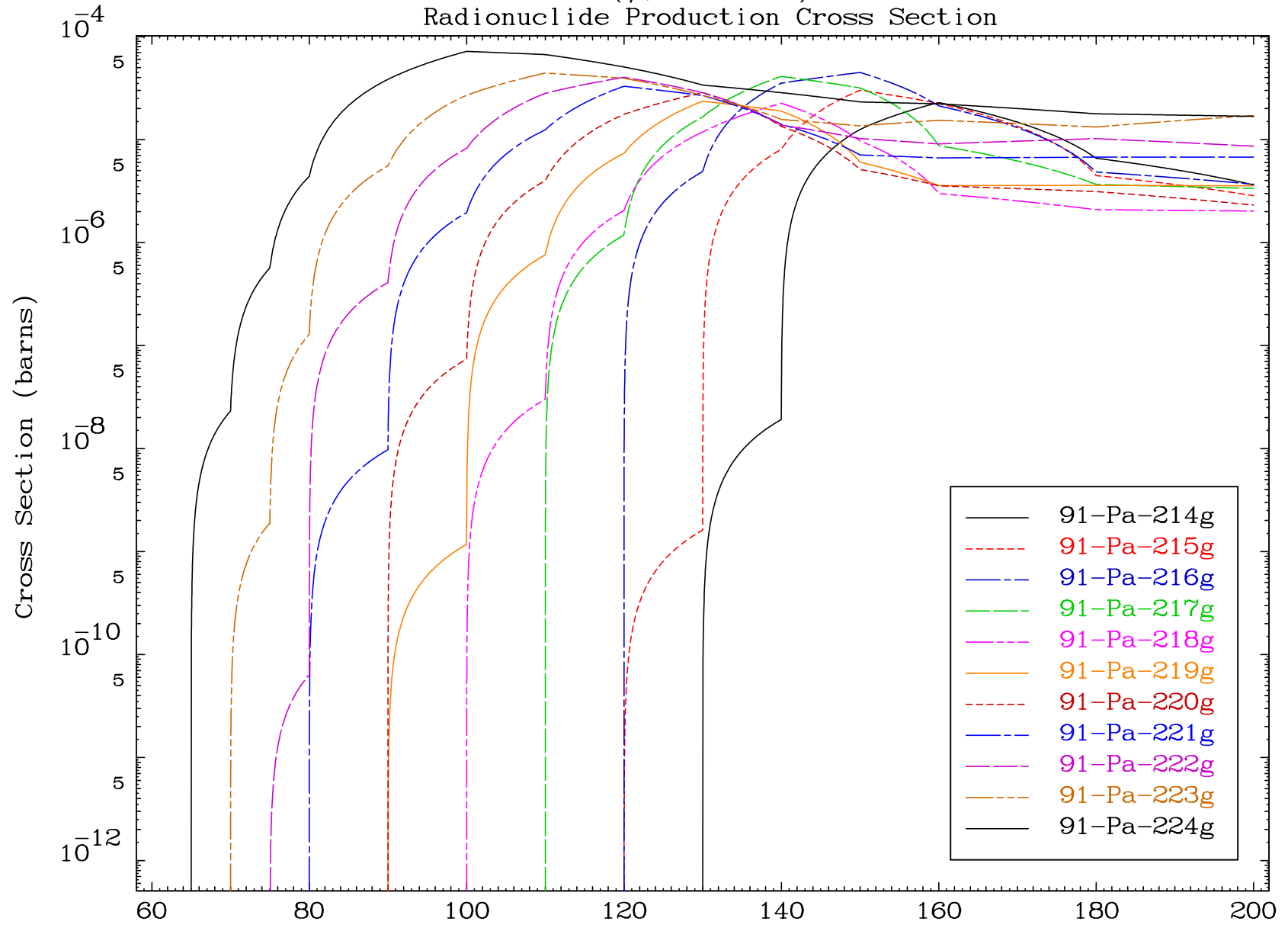


MAT 9229

(γ , remainder)

92-U -235

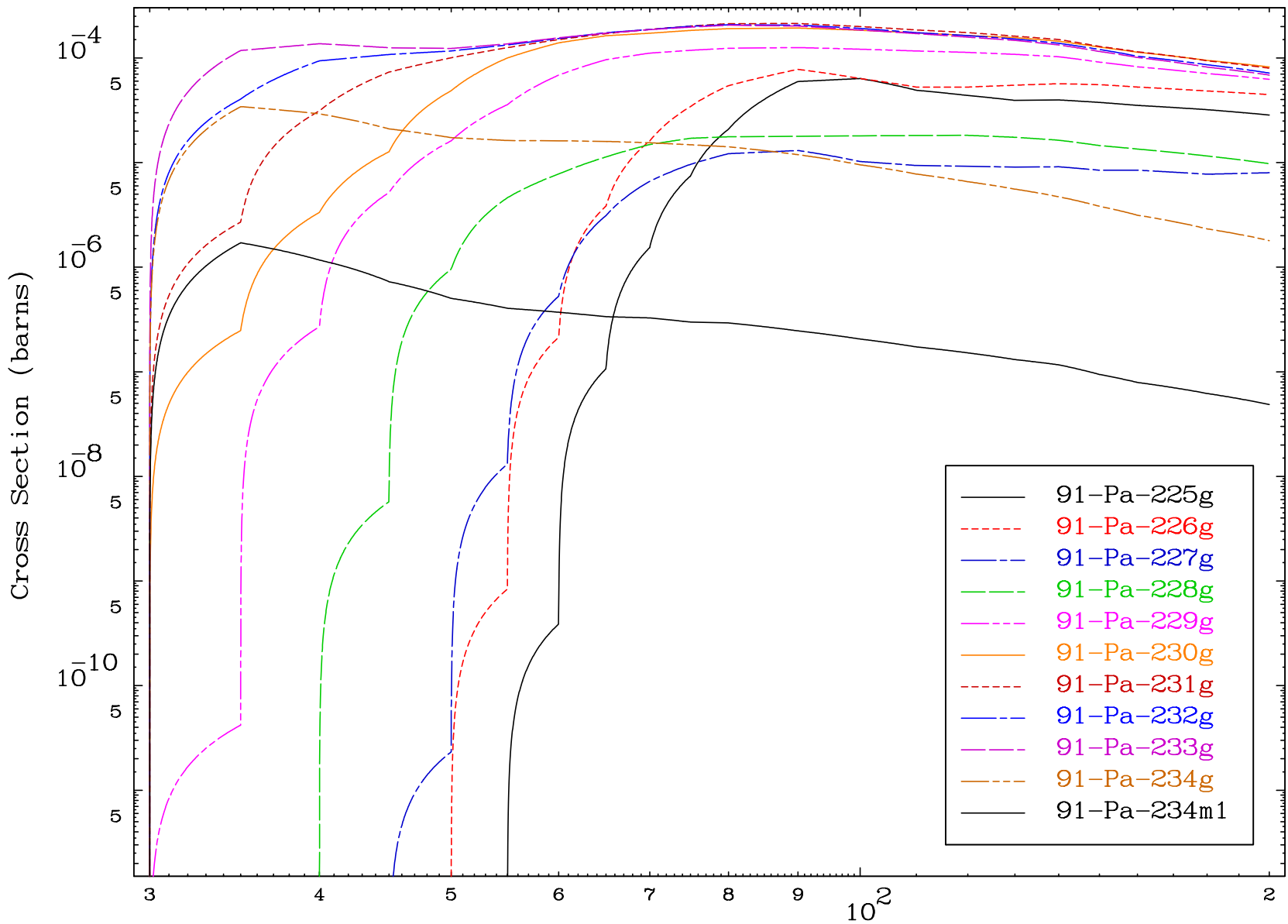
Radionuclide Production Cross Section



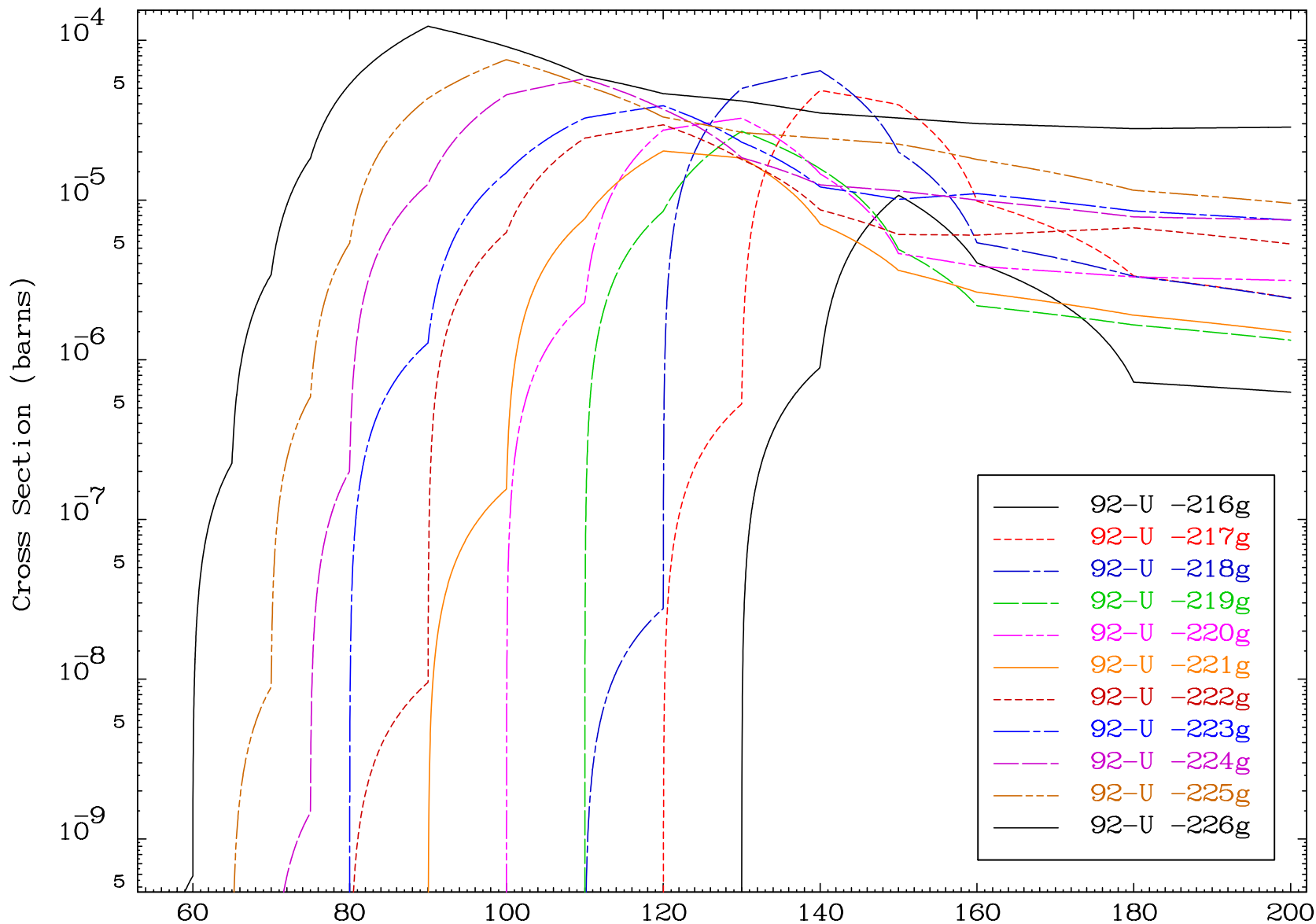
16

Incident Energy (MeV)

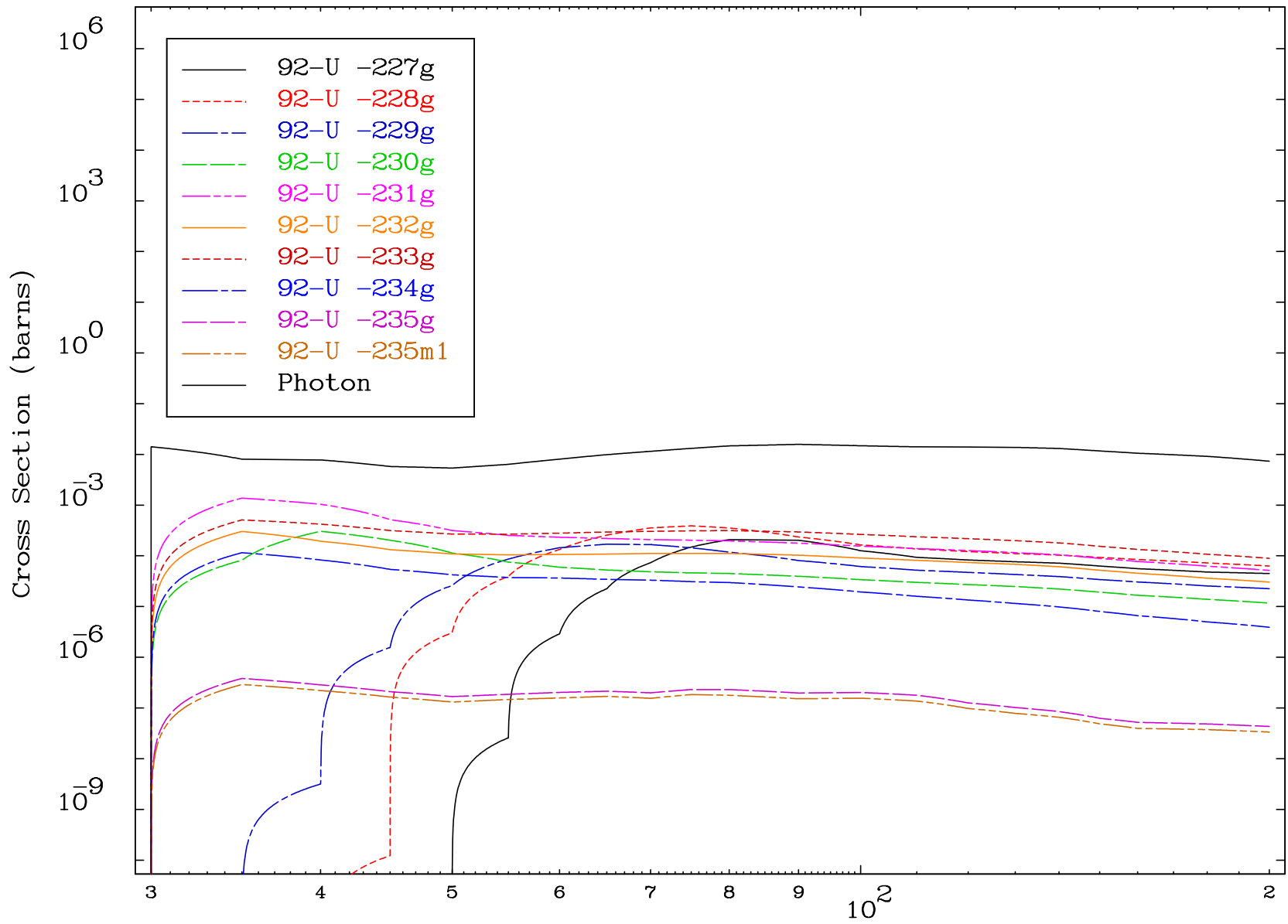
92-U -235

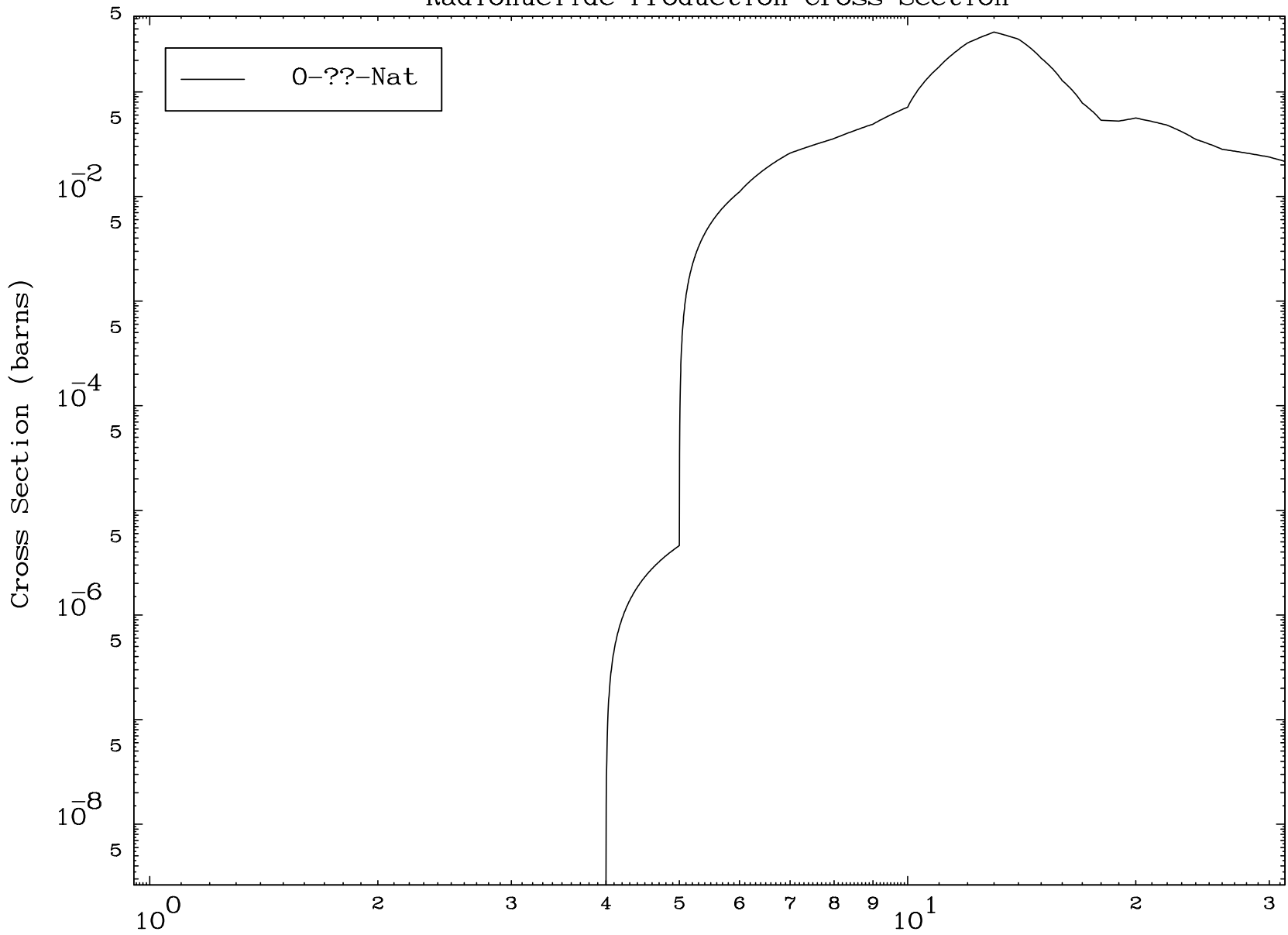


Radionuclide Production Cross Section



Radionuclide Production Cross Section

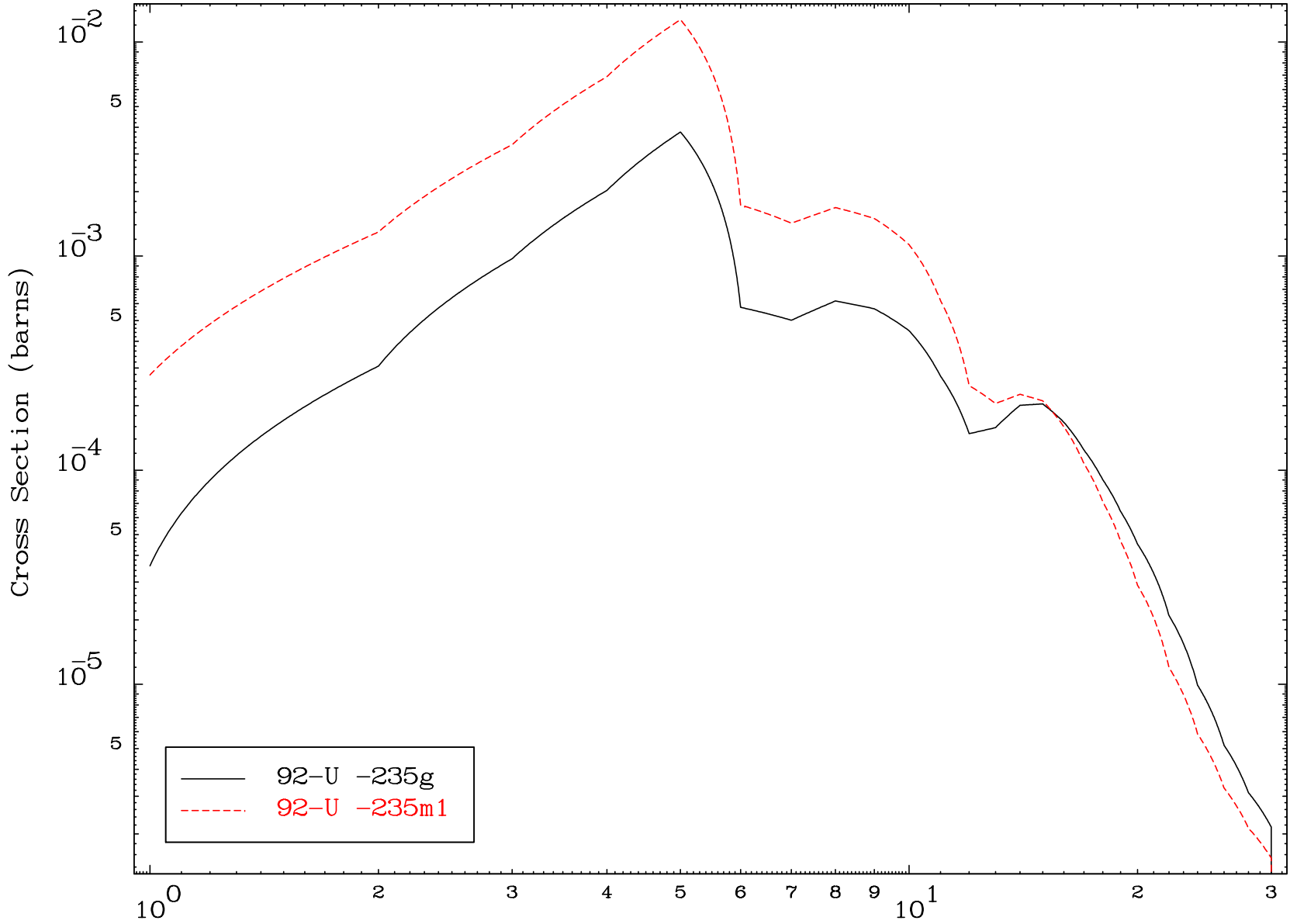




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(γ, γ)
Radionuclide Production Cross Section

92-U -235



21

Incident Energy (MeV)

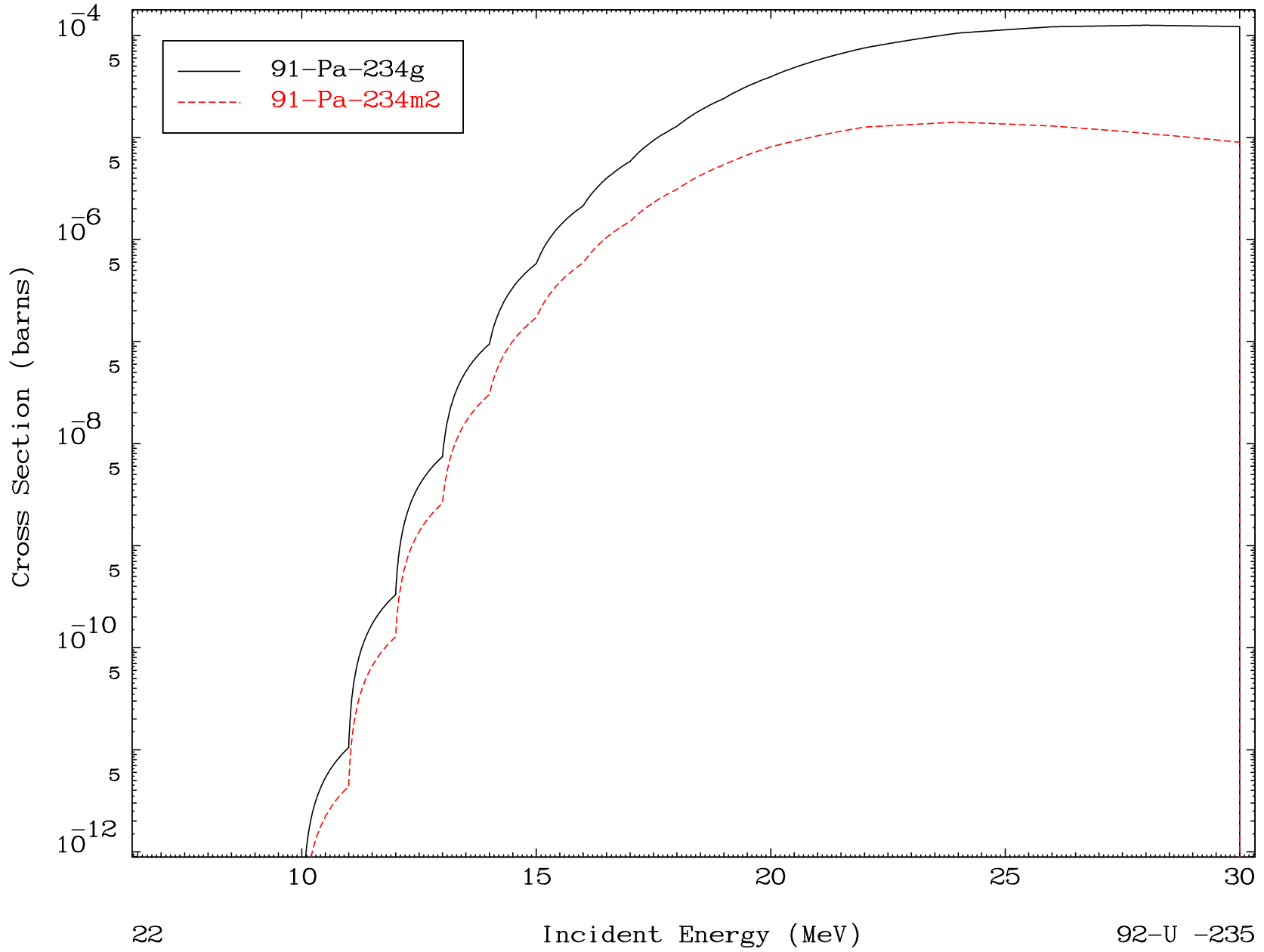
92-U -235

MAT 9229

(γ, p)

92-U -235

Radionuclide Production Cross Section



22

Incident Energy (MeV)

92-U -235