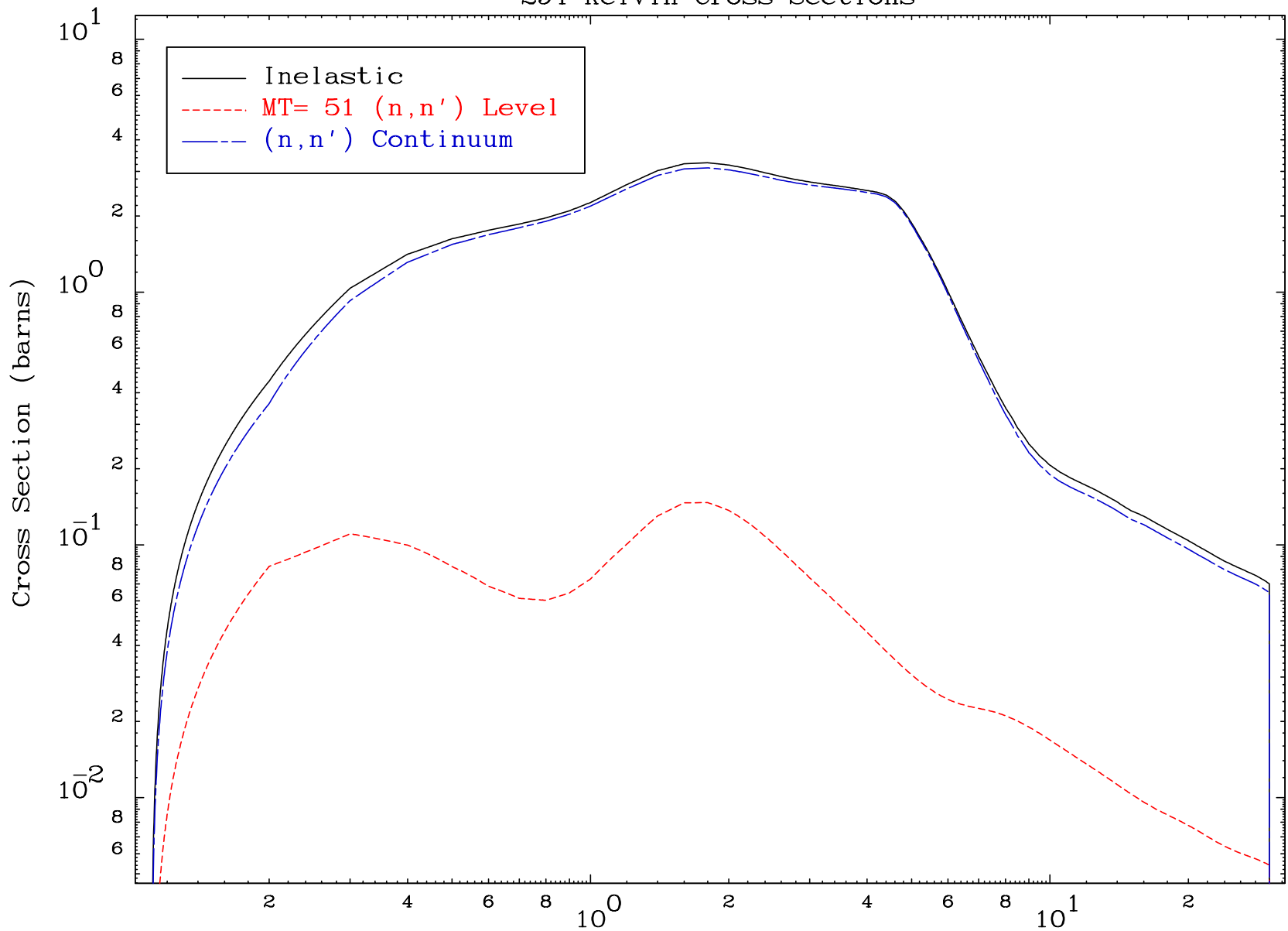


MAT 8958

(n,n') Level  
294 Kelvin Cross Sections

89-Ac-236

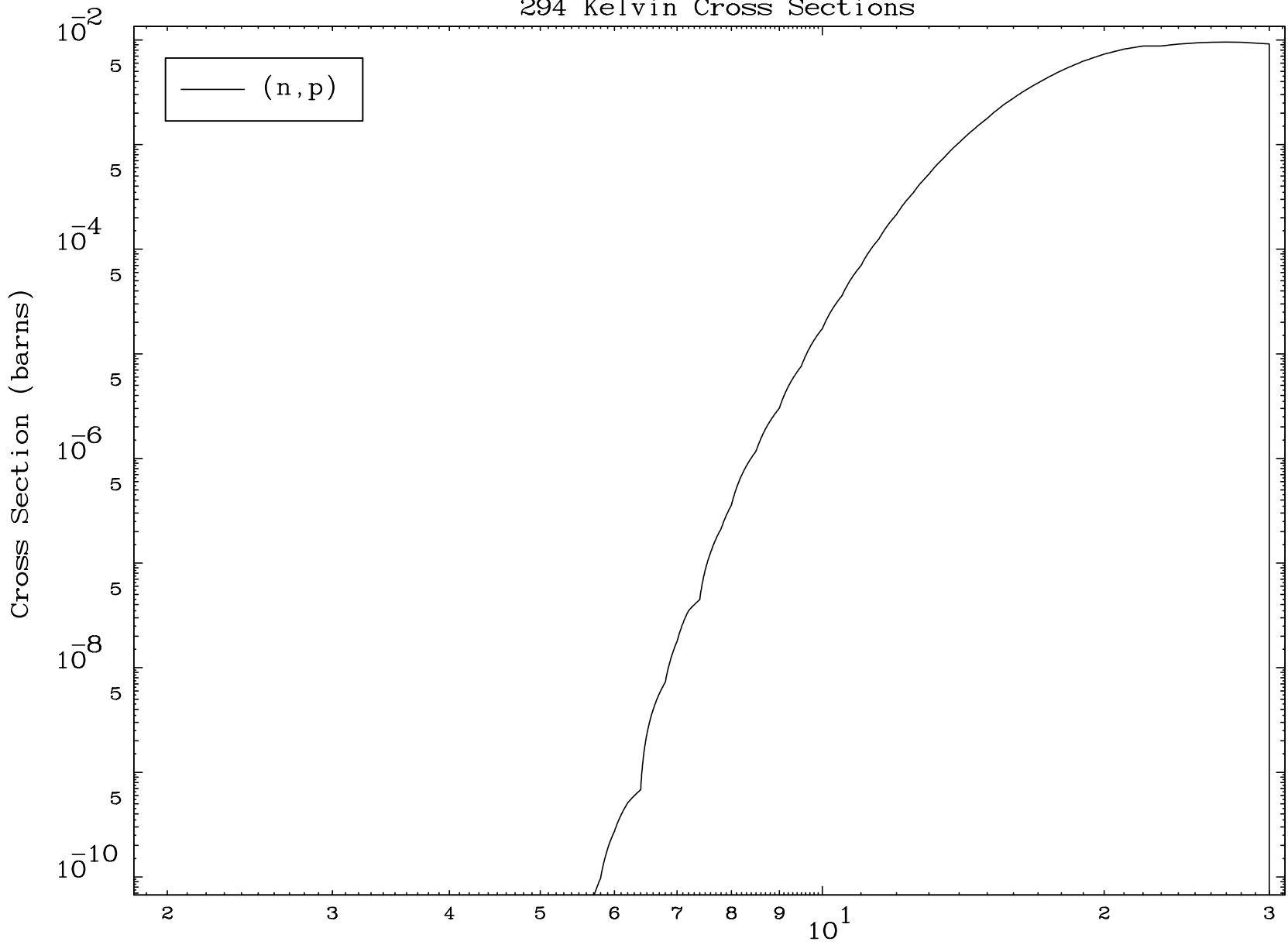


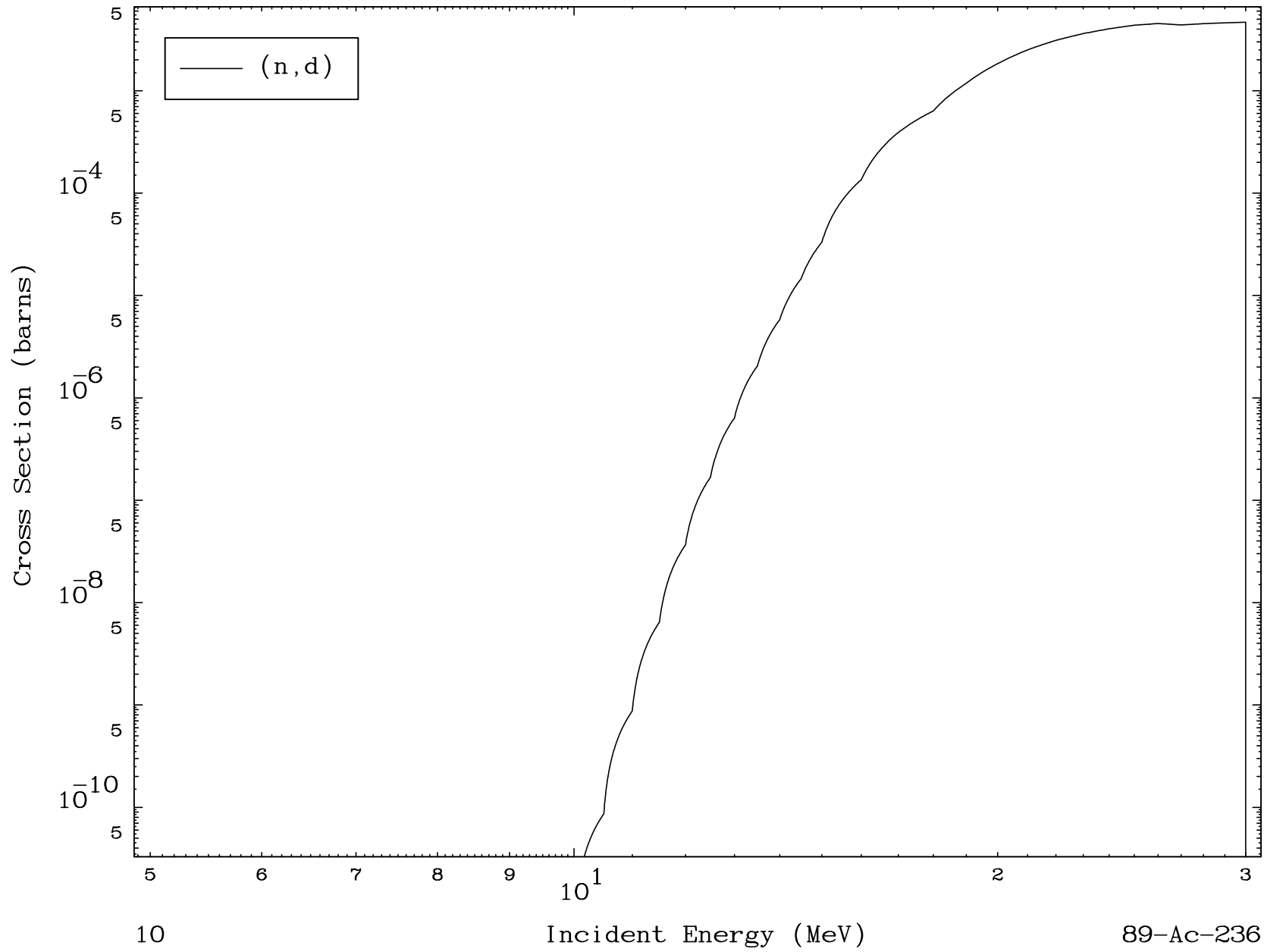
8

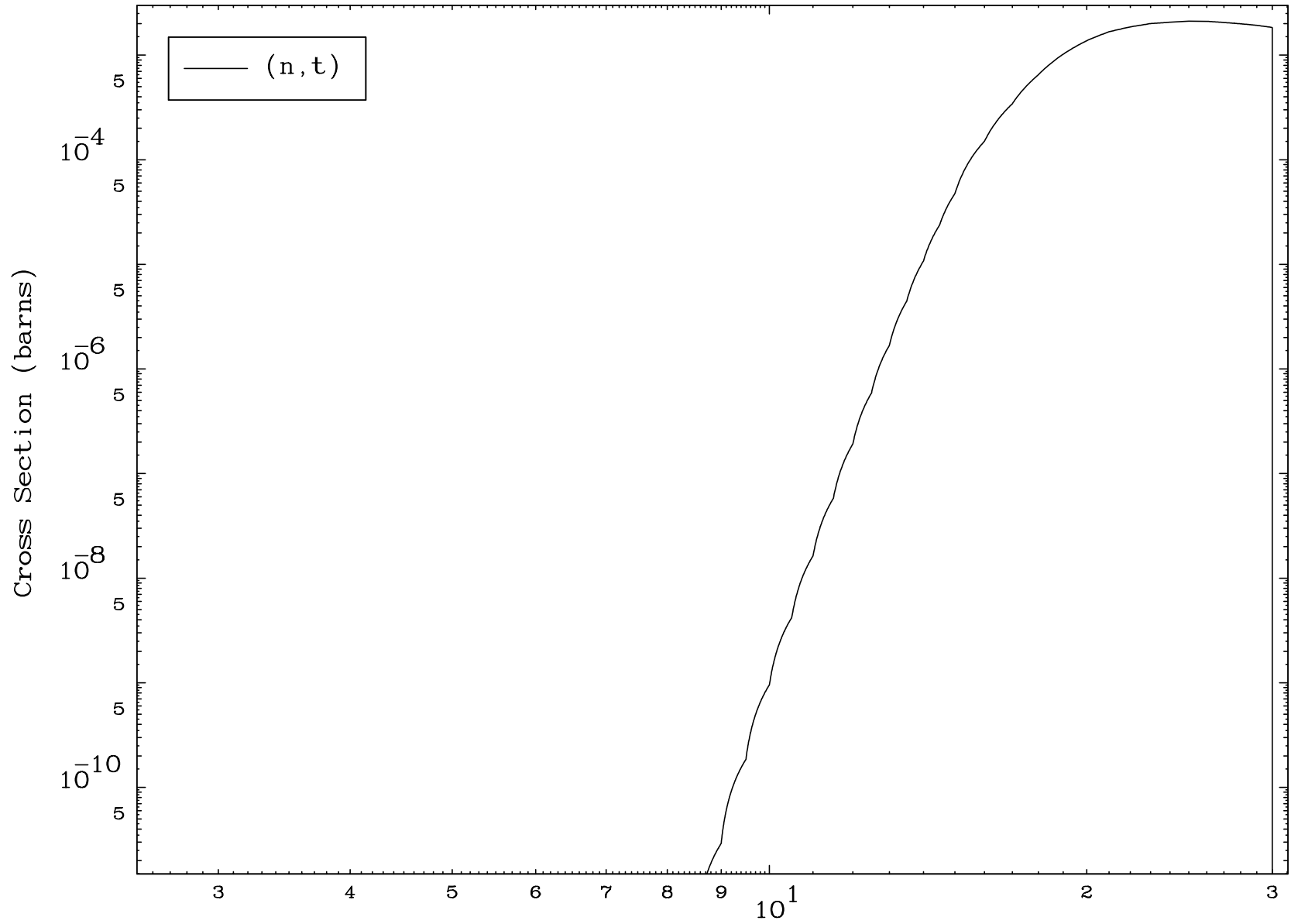
Incident Energy (MeV)

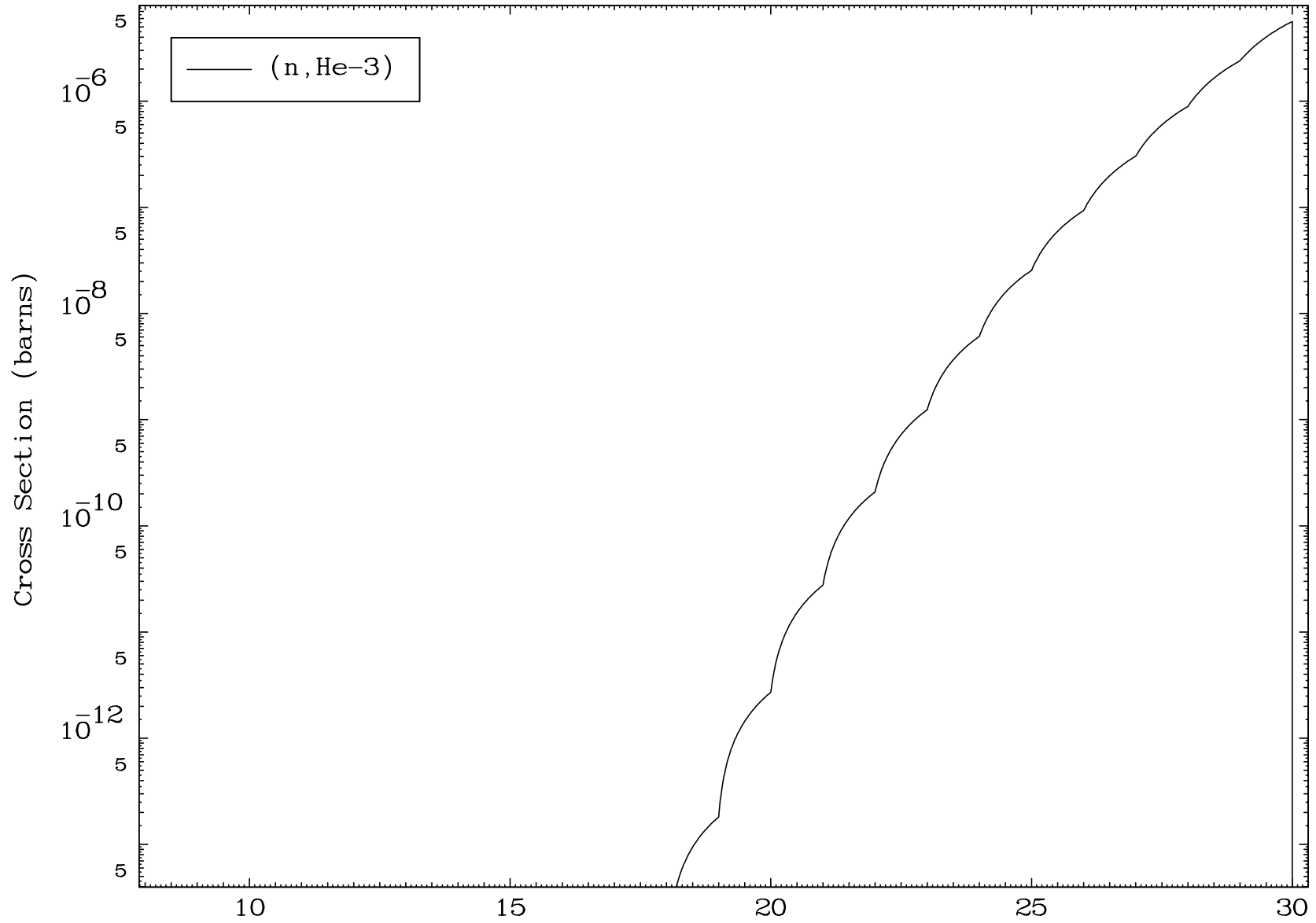
89-Ac-236







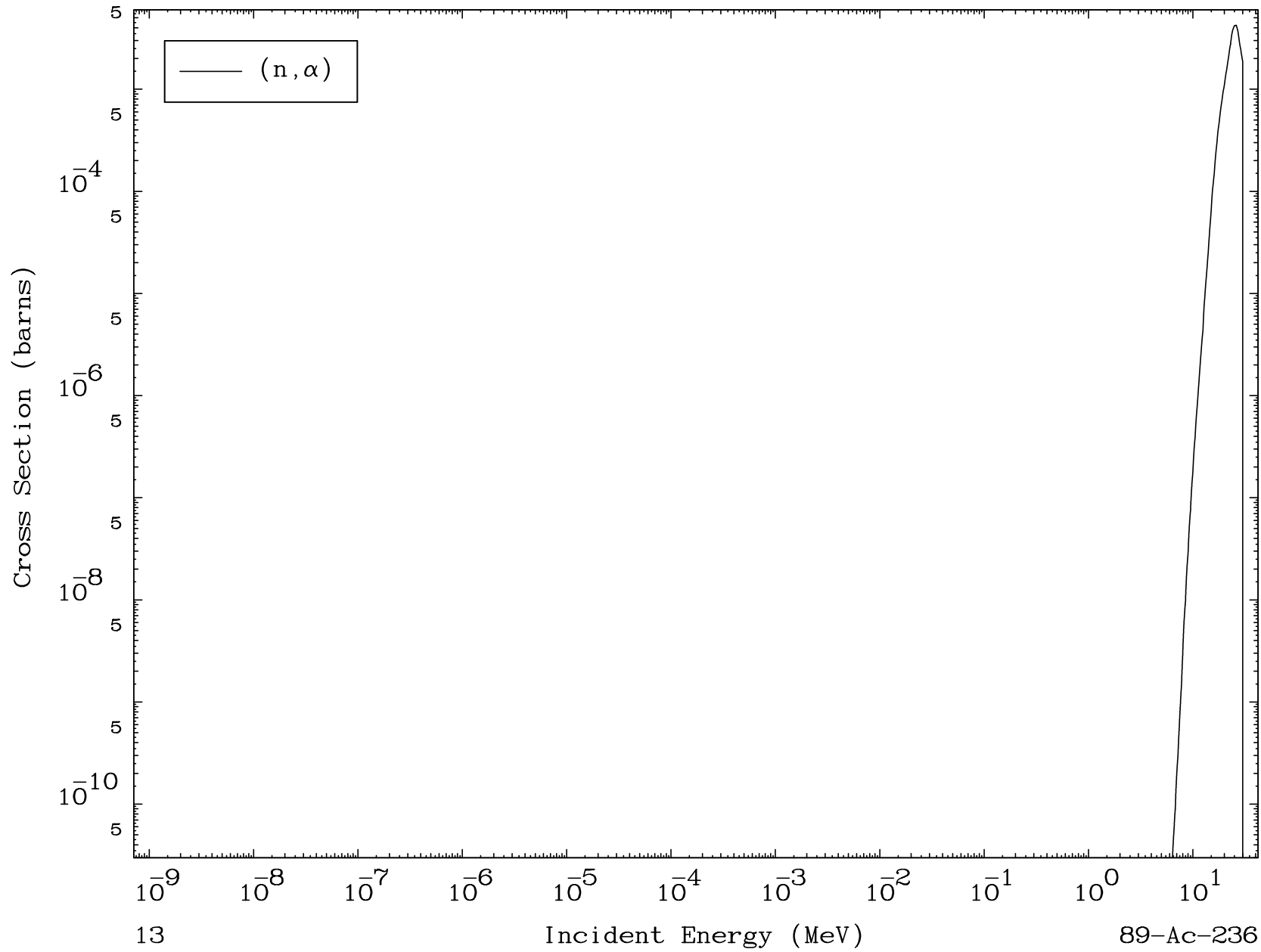




MAT 8958

(n,  $\alpha$ ) Levels  
294 Kelvin Cross Sections

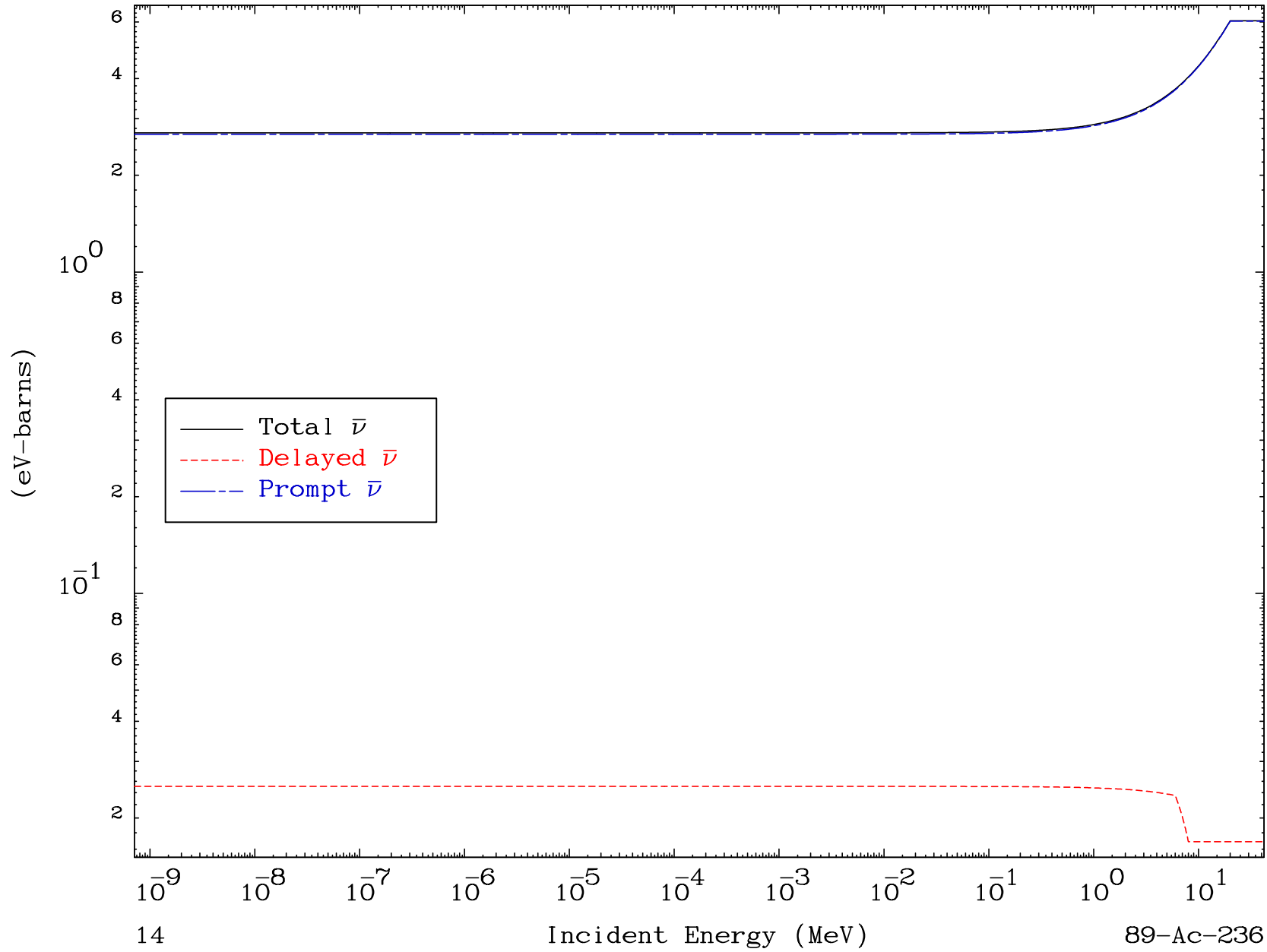
89-Ac-236

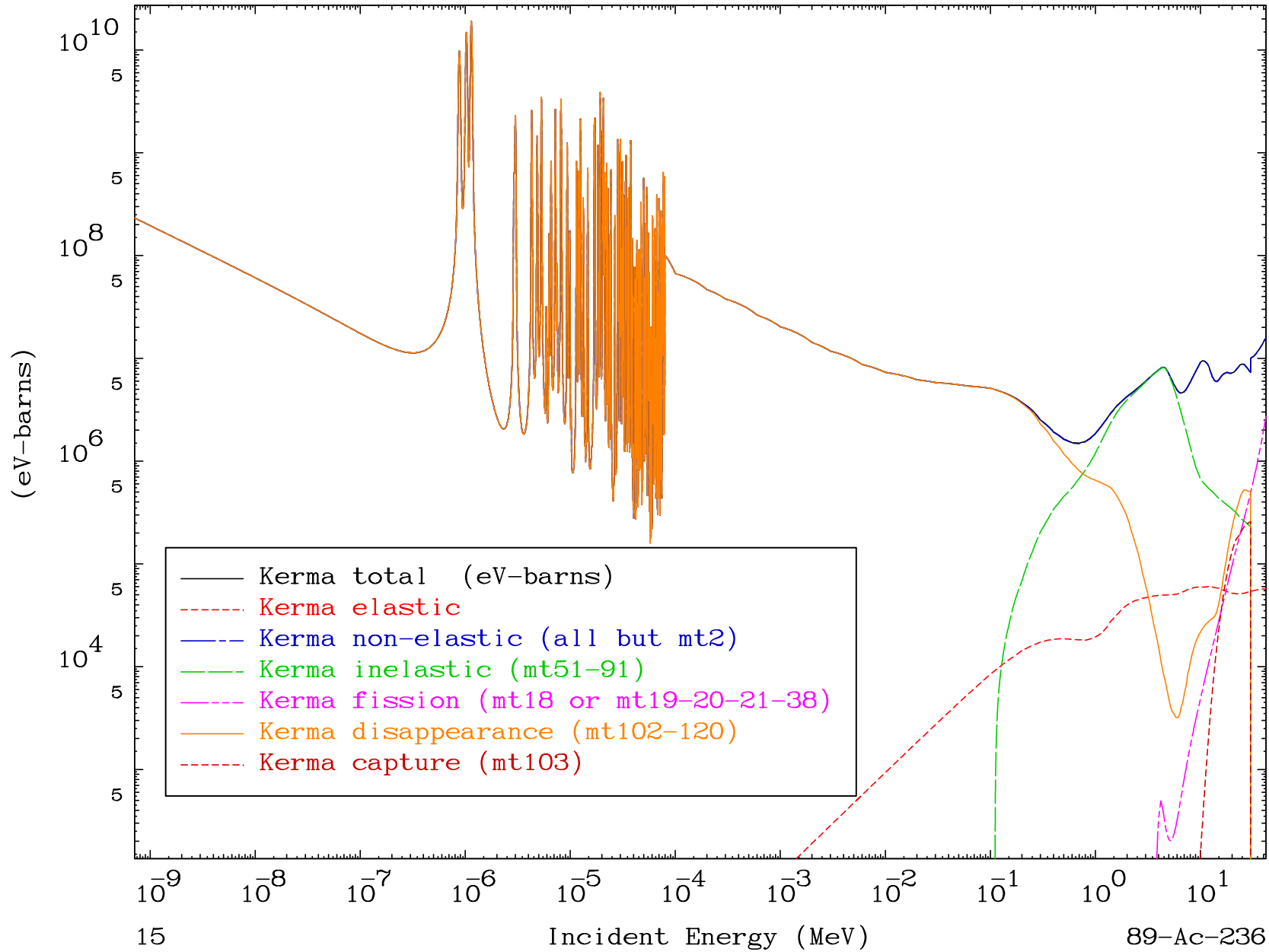


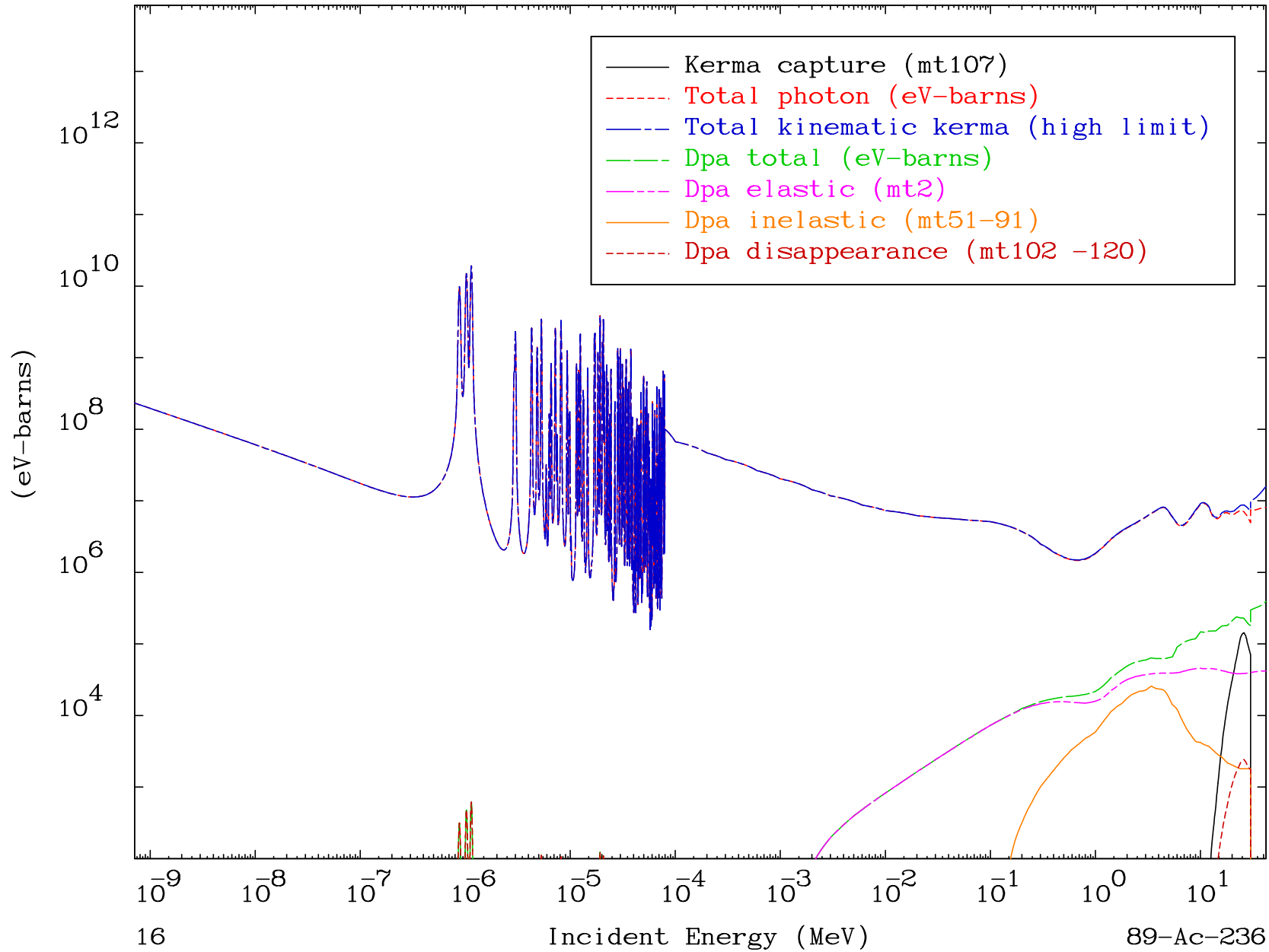
13

Incident Energy (MeV)

89-Ac-236

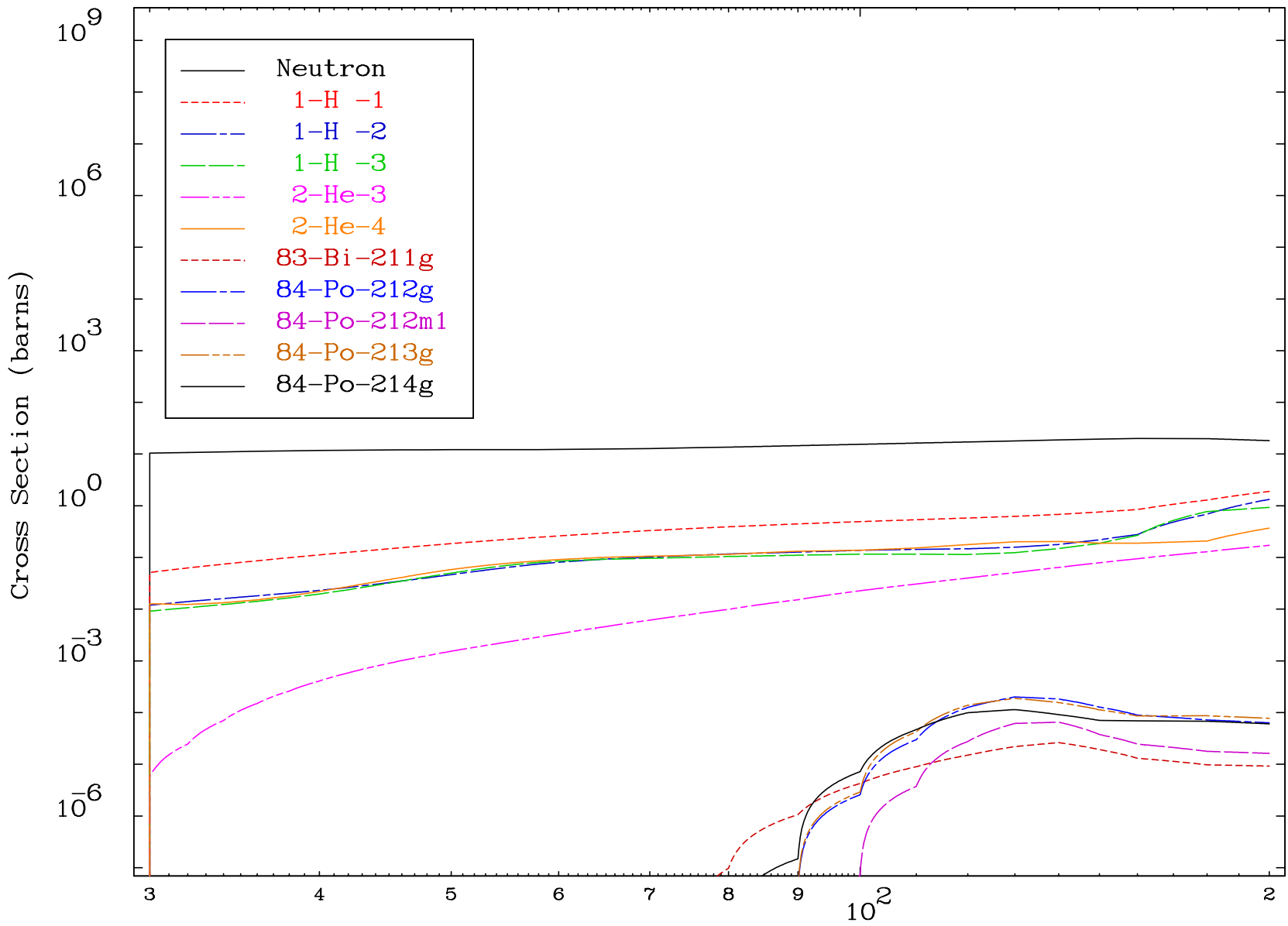




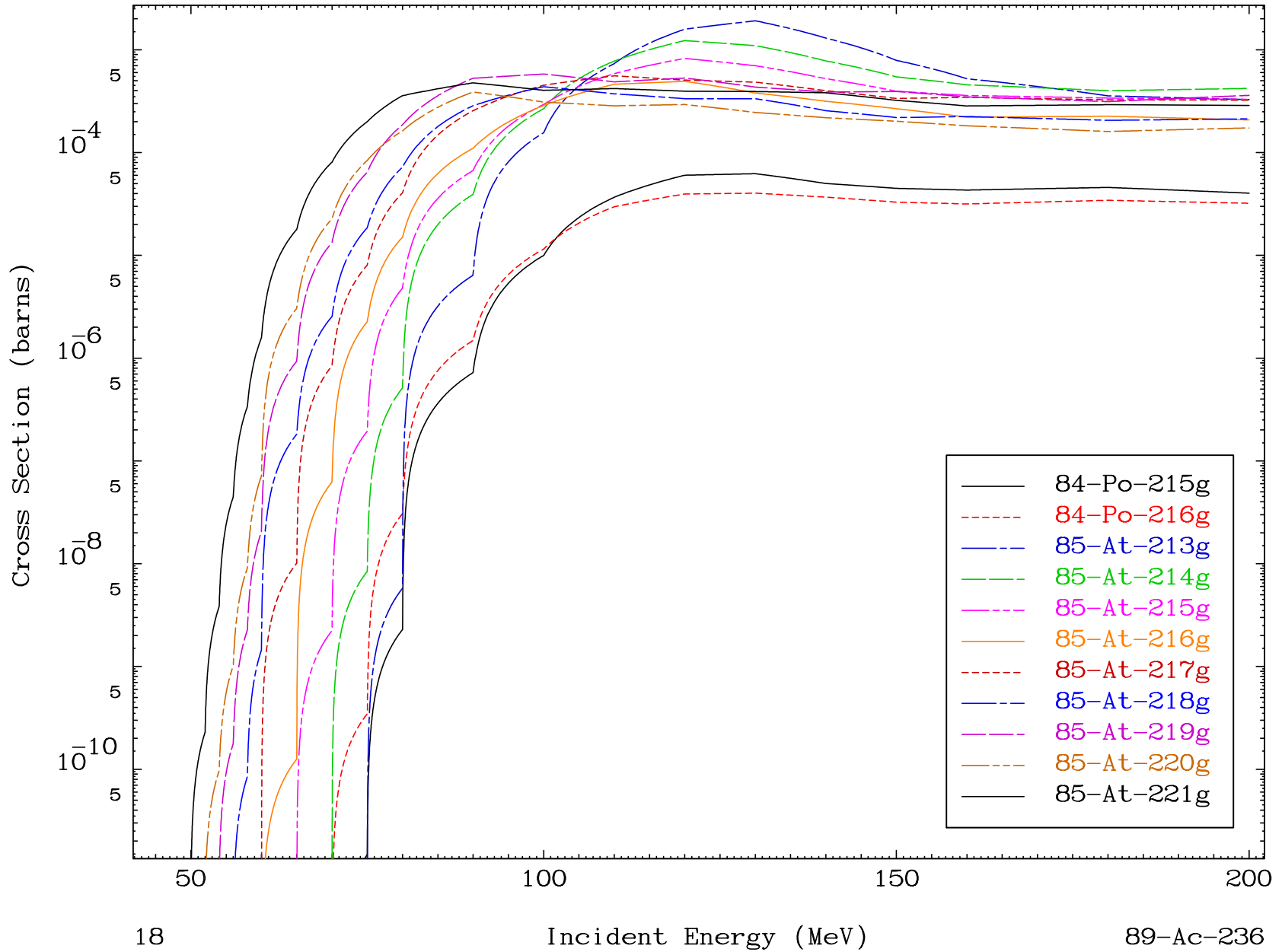


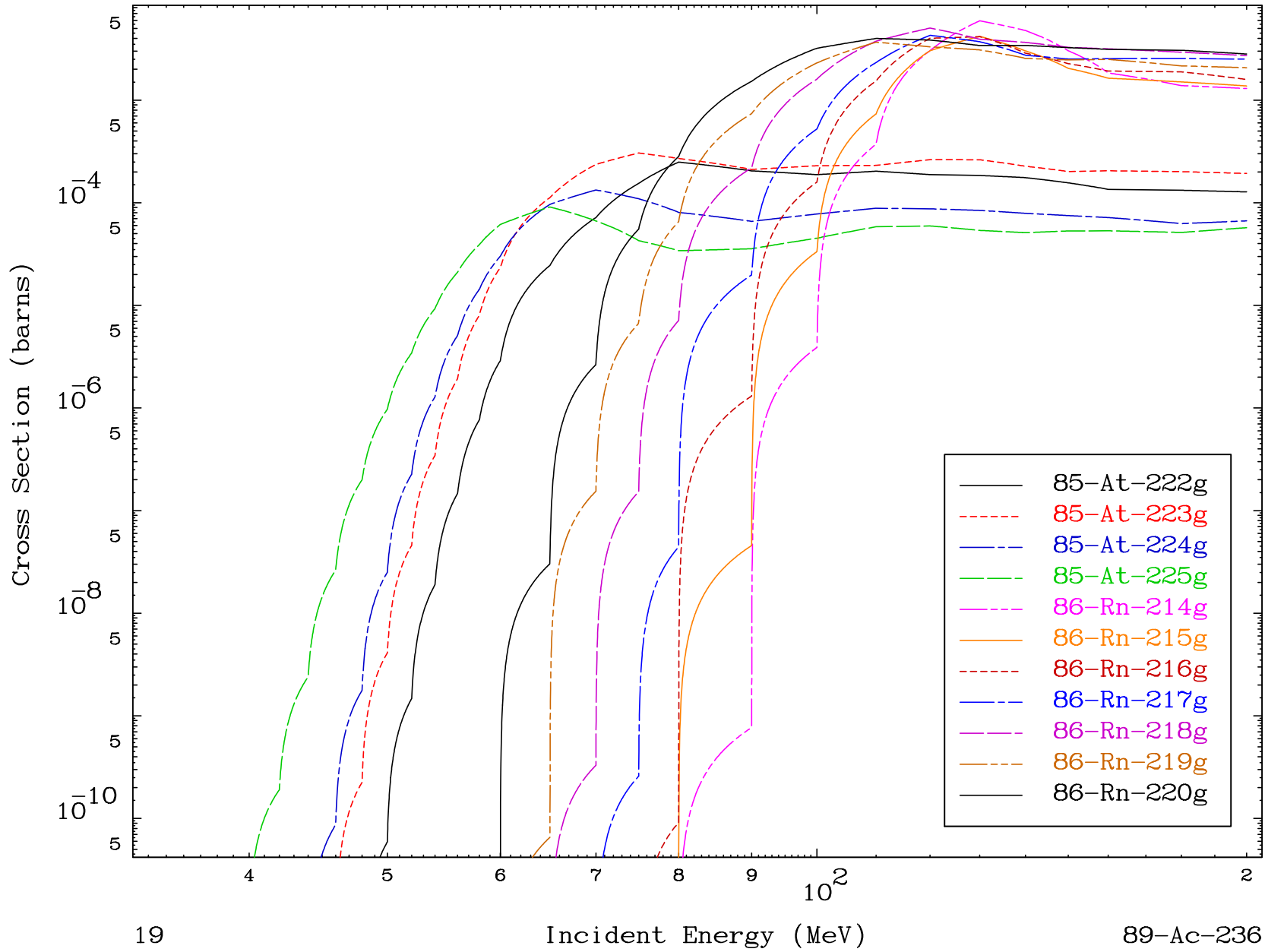


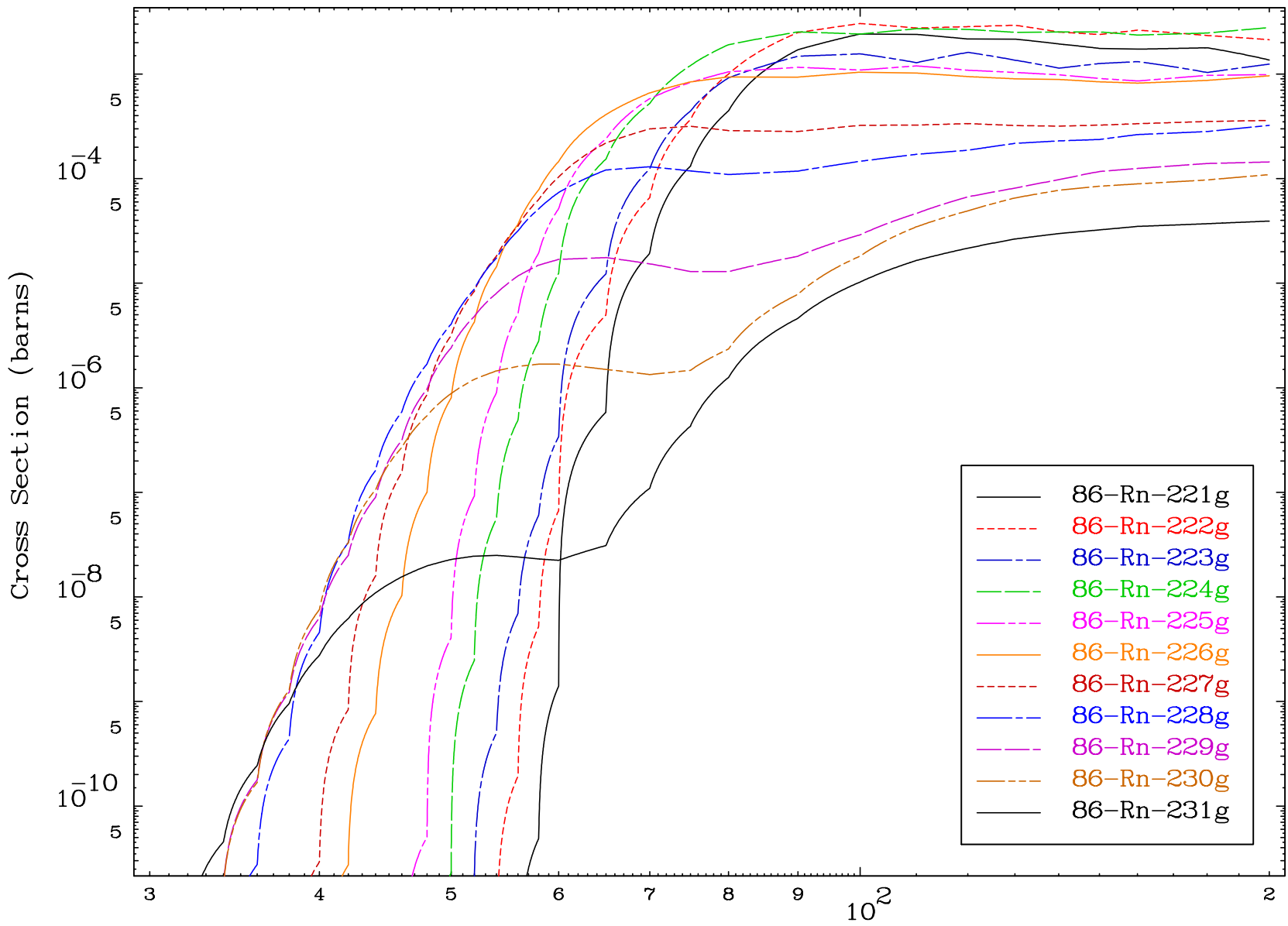
Radionuclide Production Cross Section



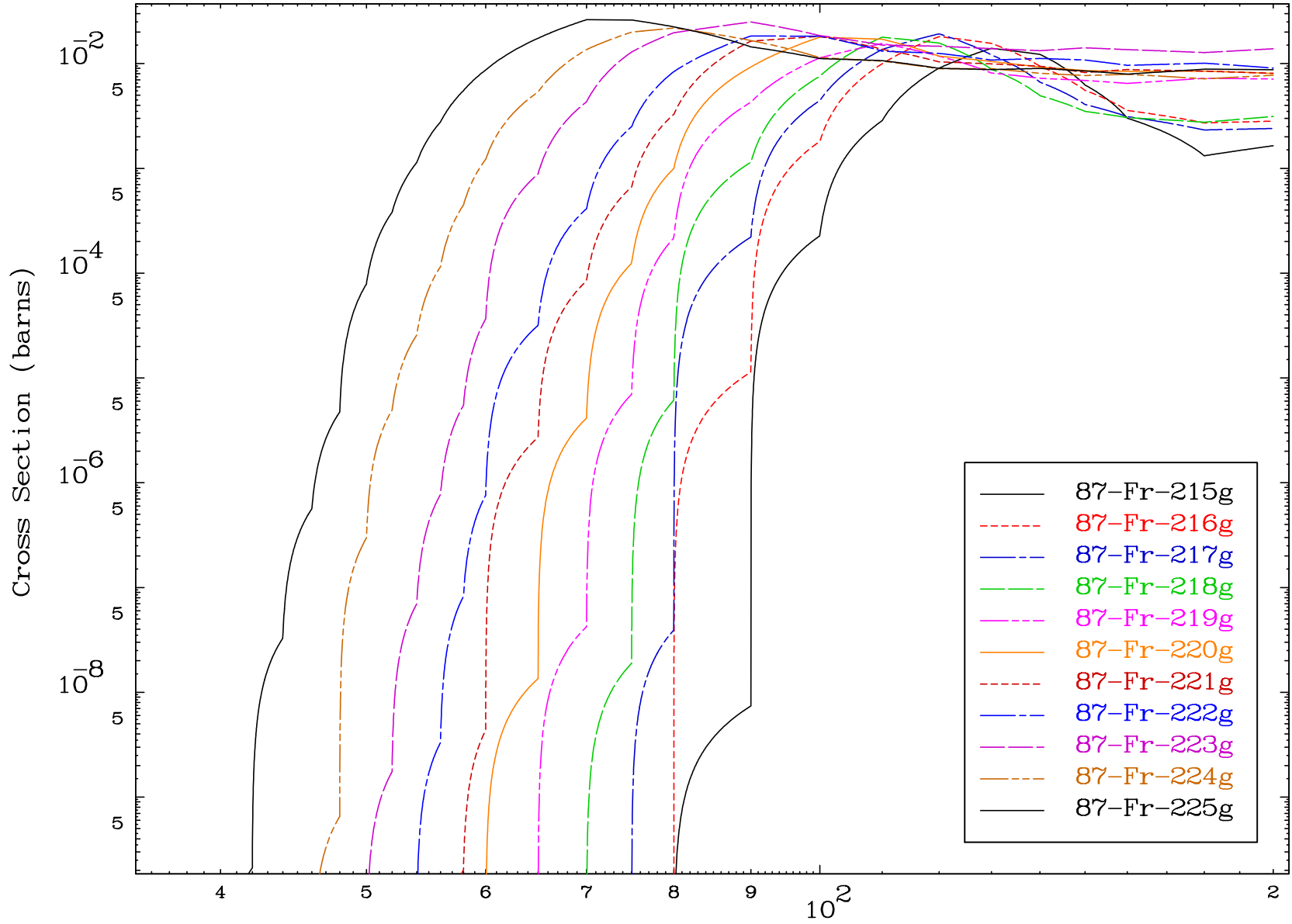
Radionuclide Production Cross Section



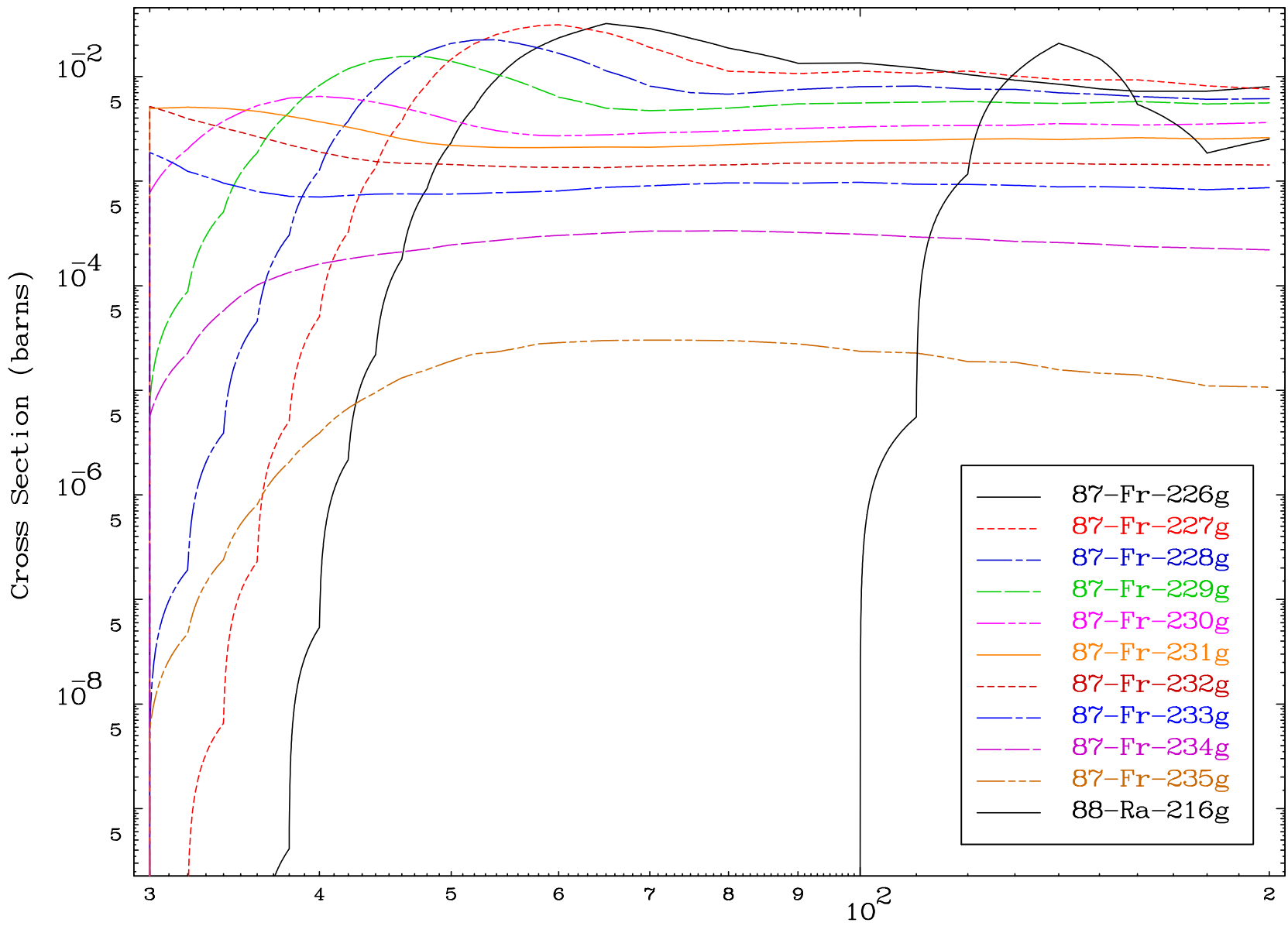


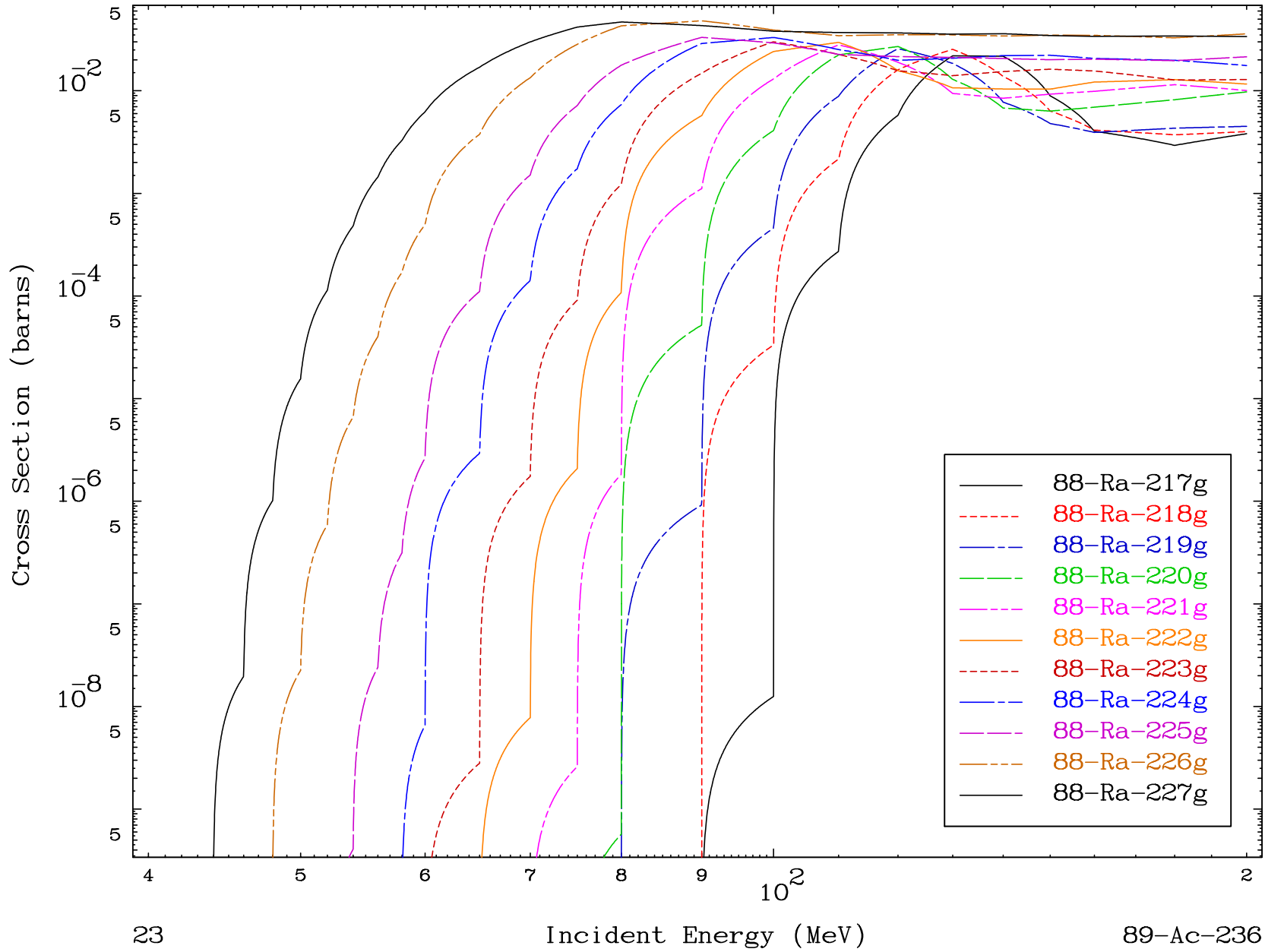


Radionuclide Production Cross Section

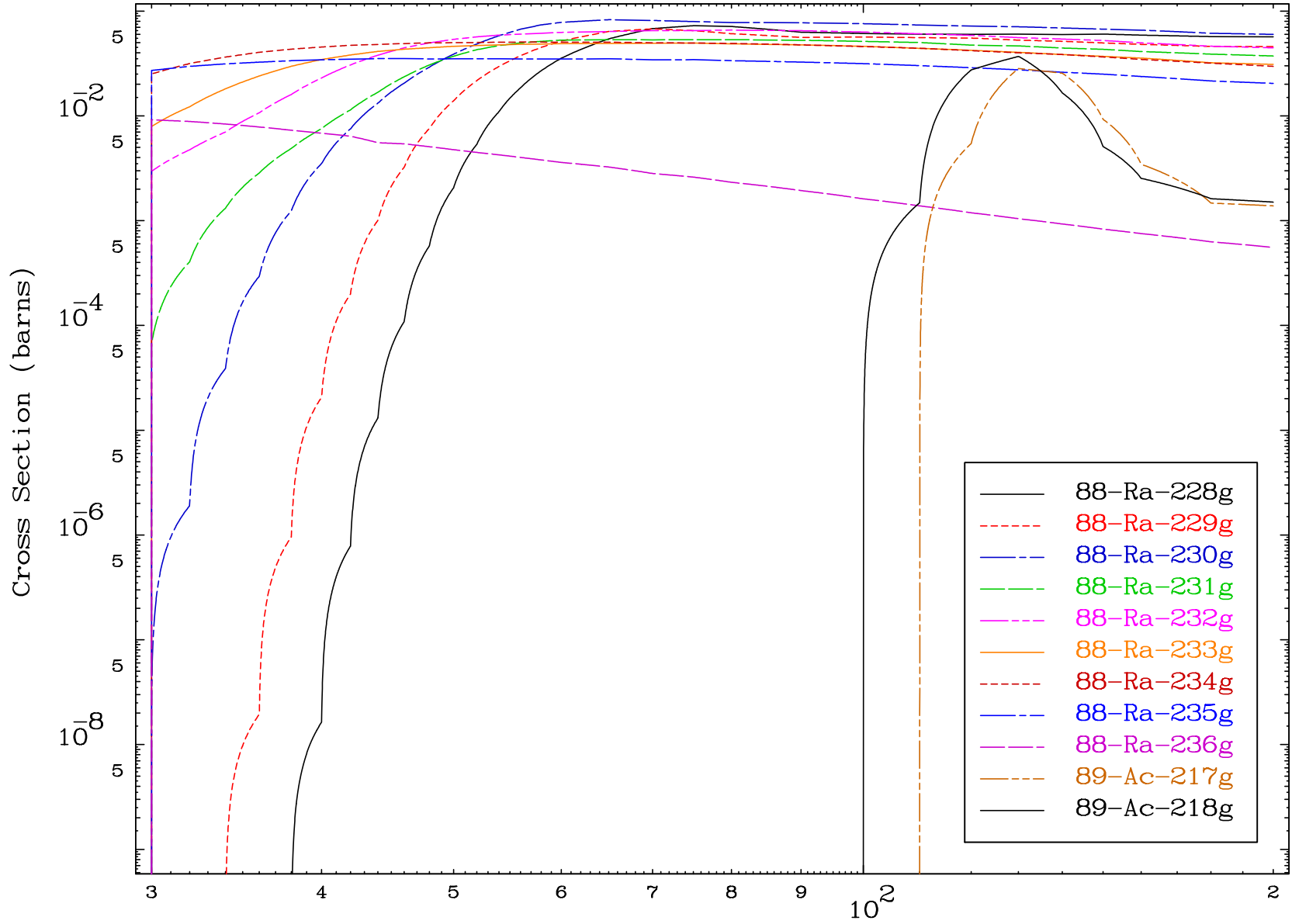


Radionuclide Production Cross Section





Radionuclide Production Cross Section



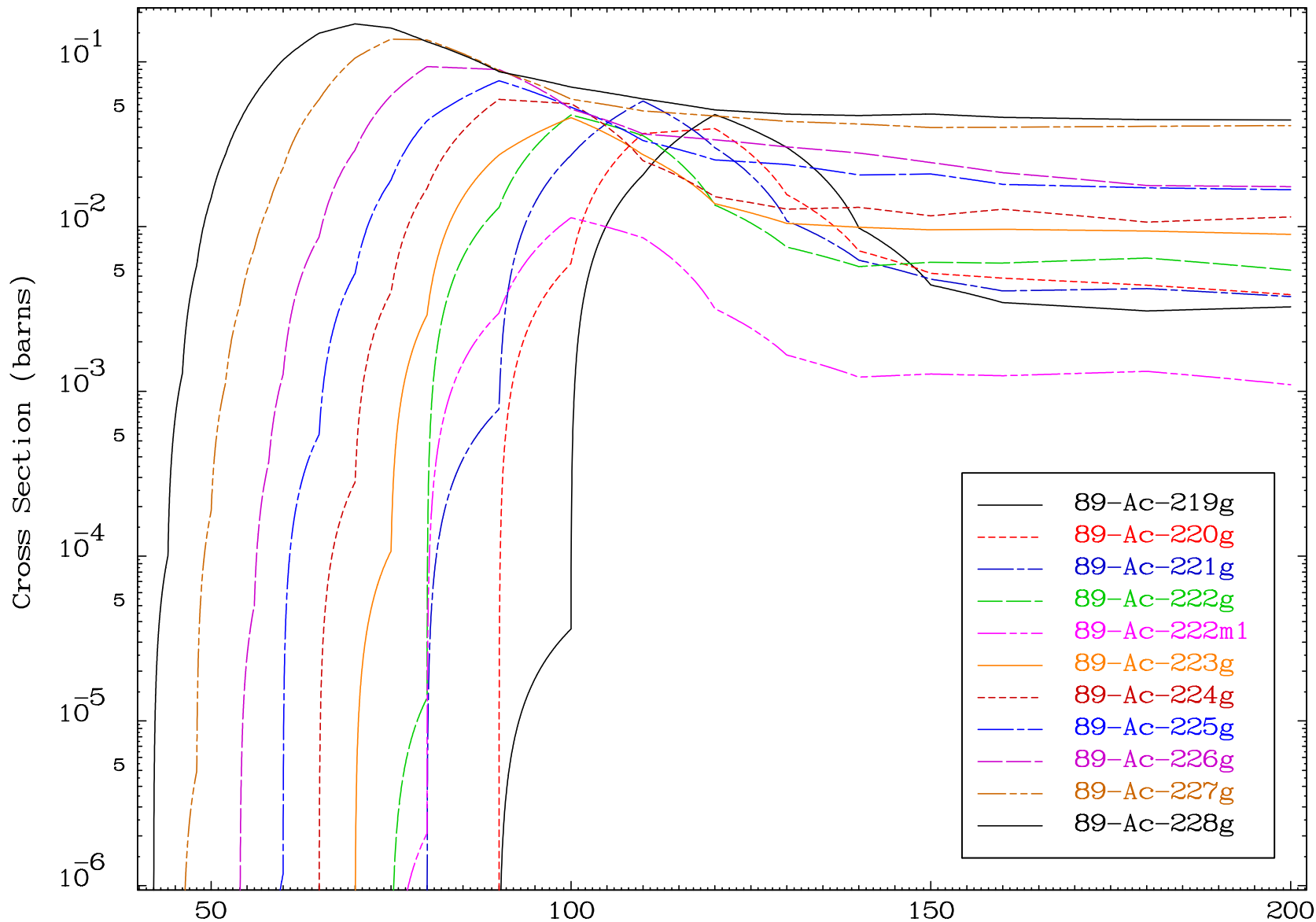


MAT 8958

(n,remainder)

89-Ac-236

### Radionuclide Production Cross Section



25

Incident Energy (MeV)

89-Ac-236

Radionuclide Production Cross Section

