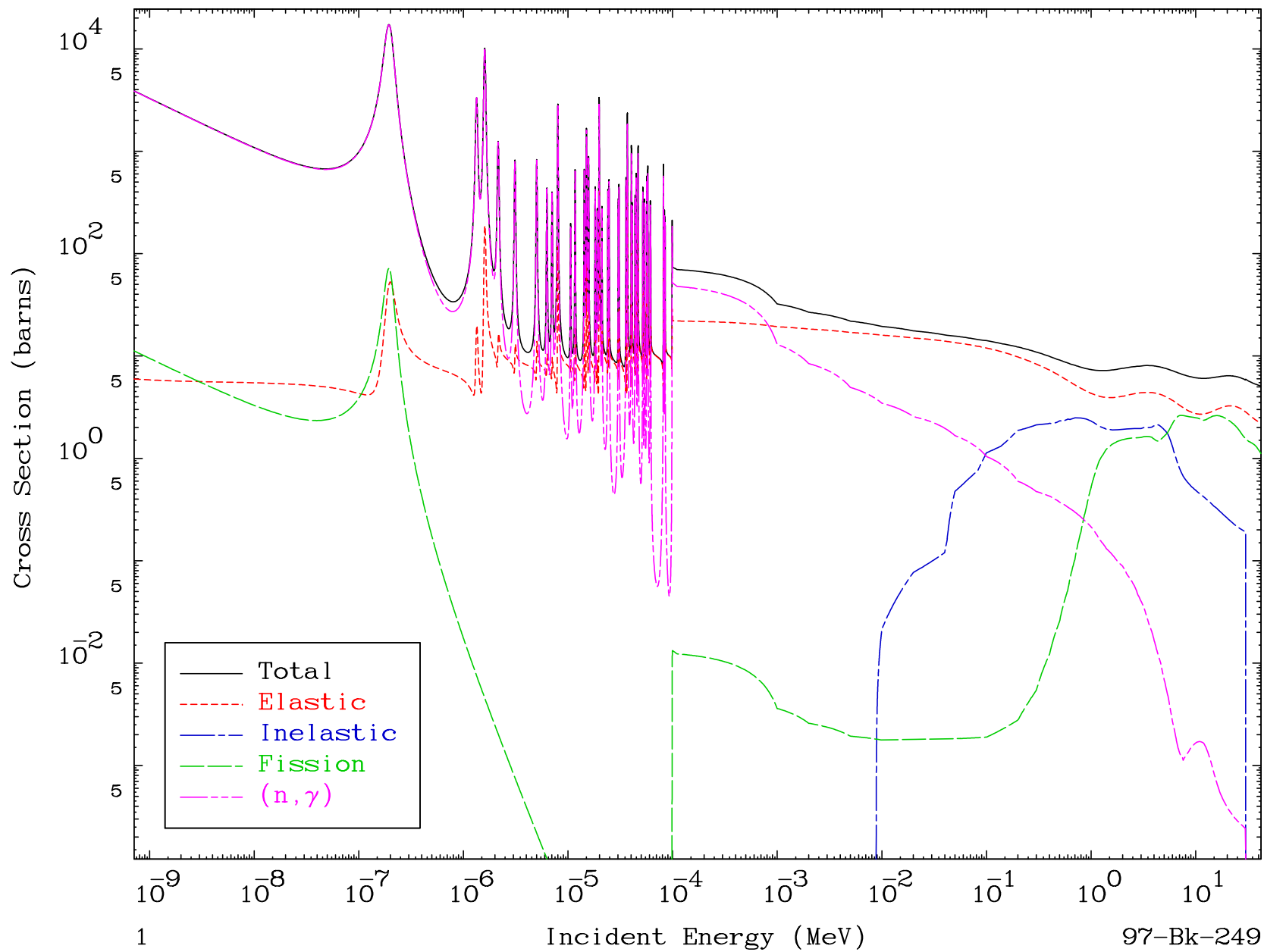
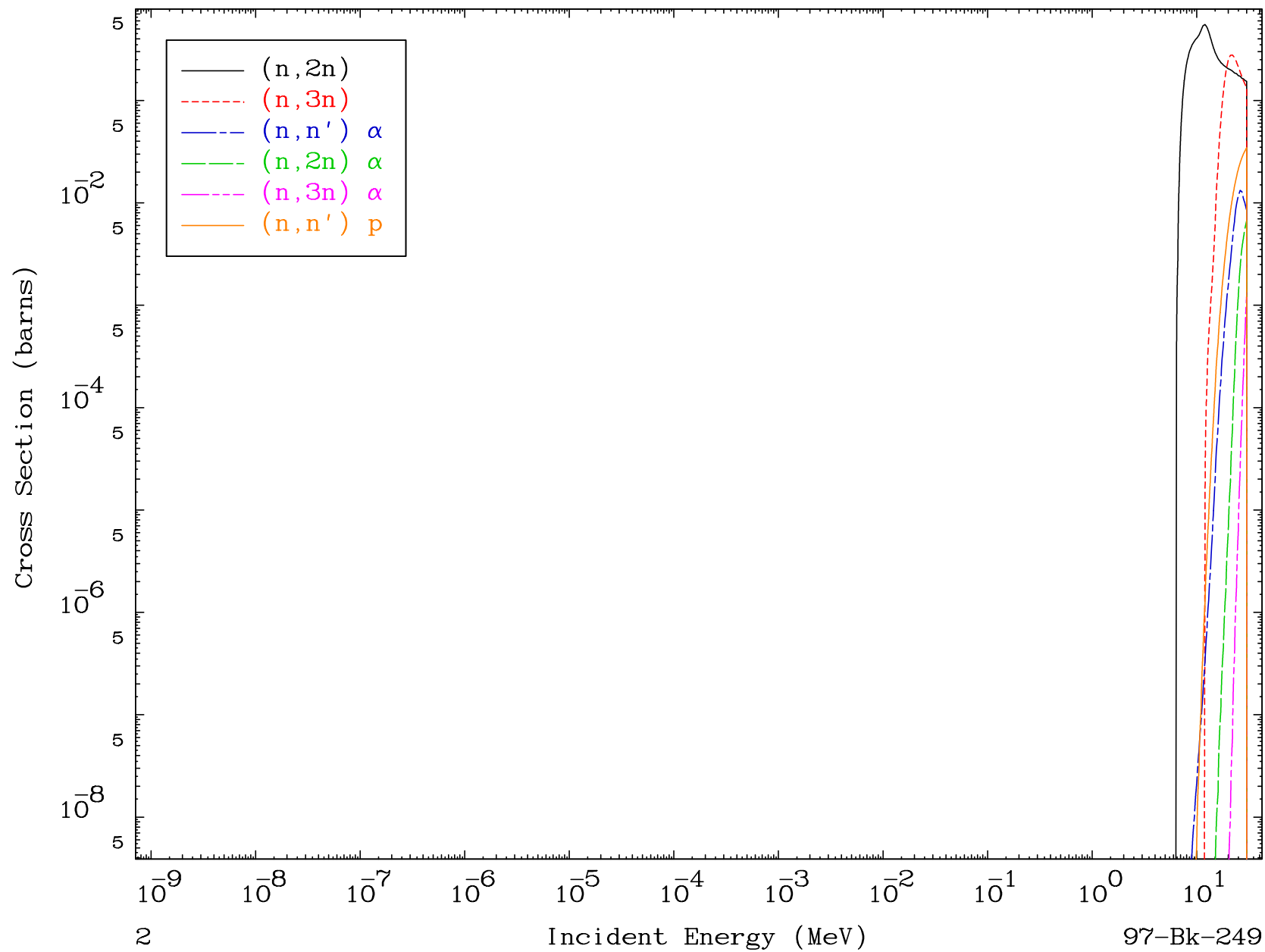


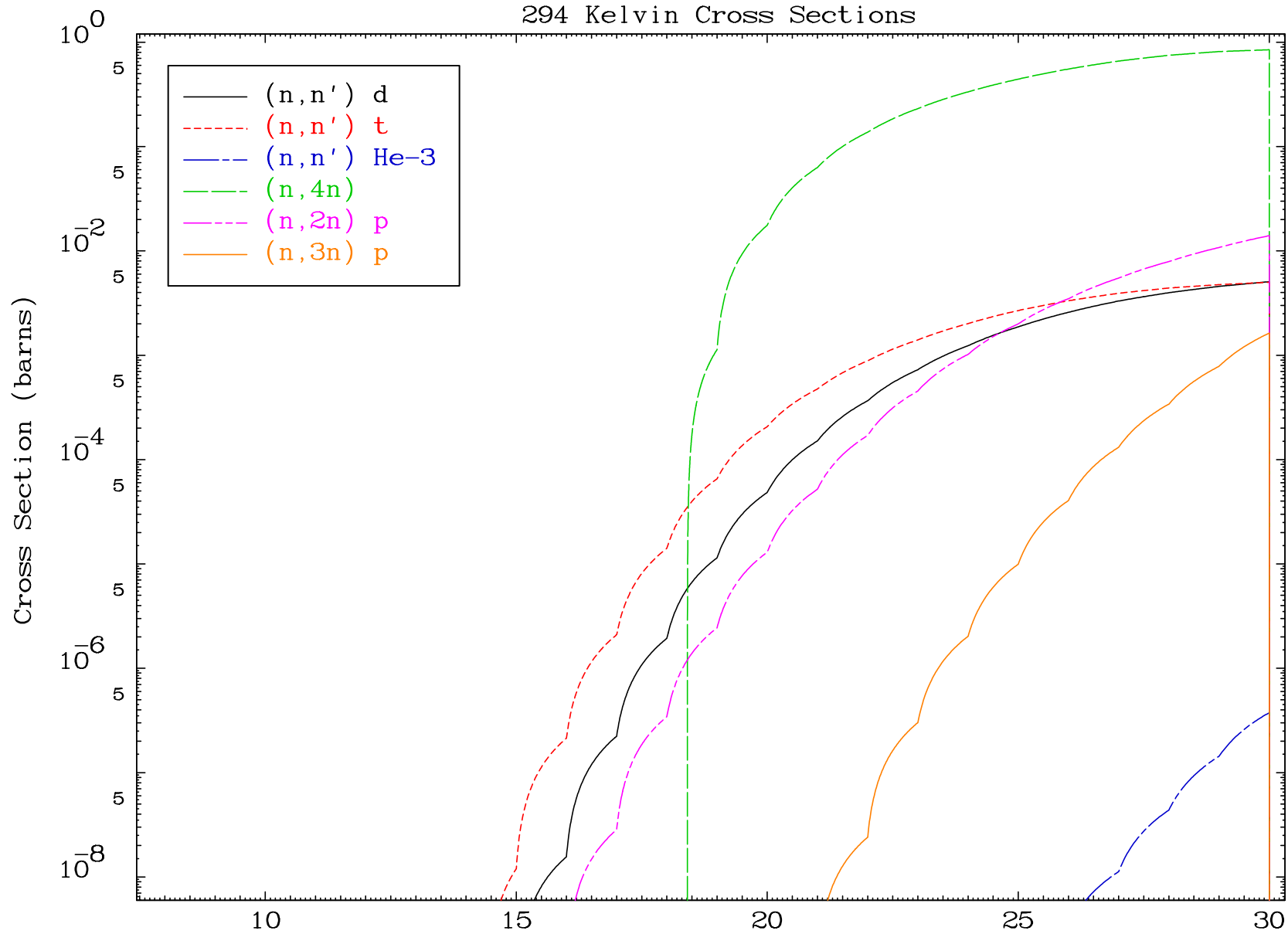




MAT 9752

Neutron Production
294 Kelvin Cross Sections

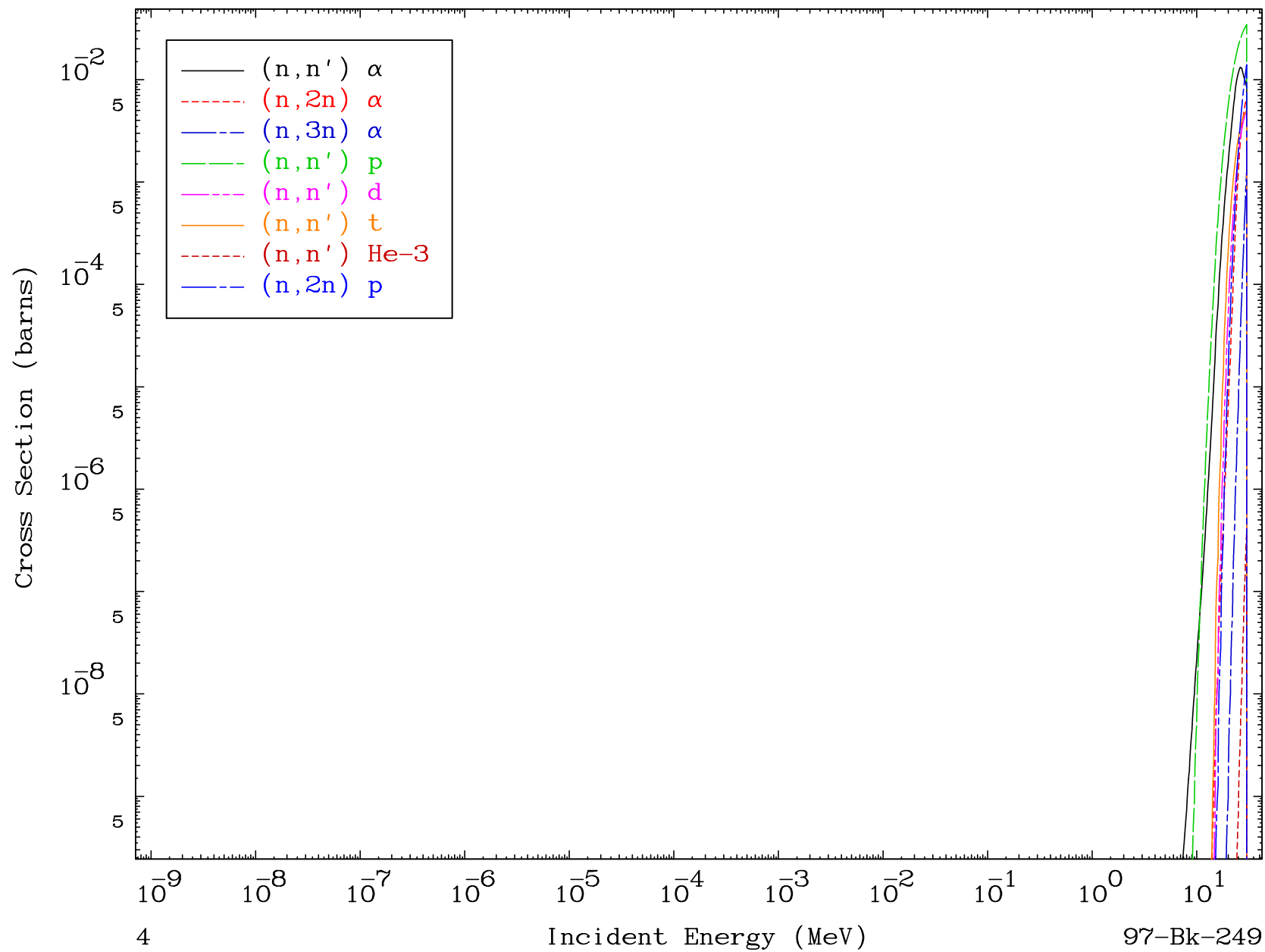
97-Bk-249



3

Incident Energy (MeV)

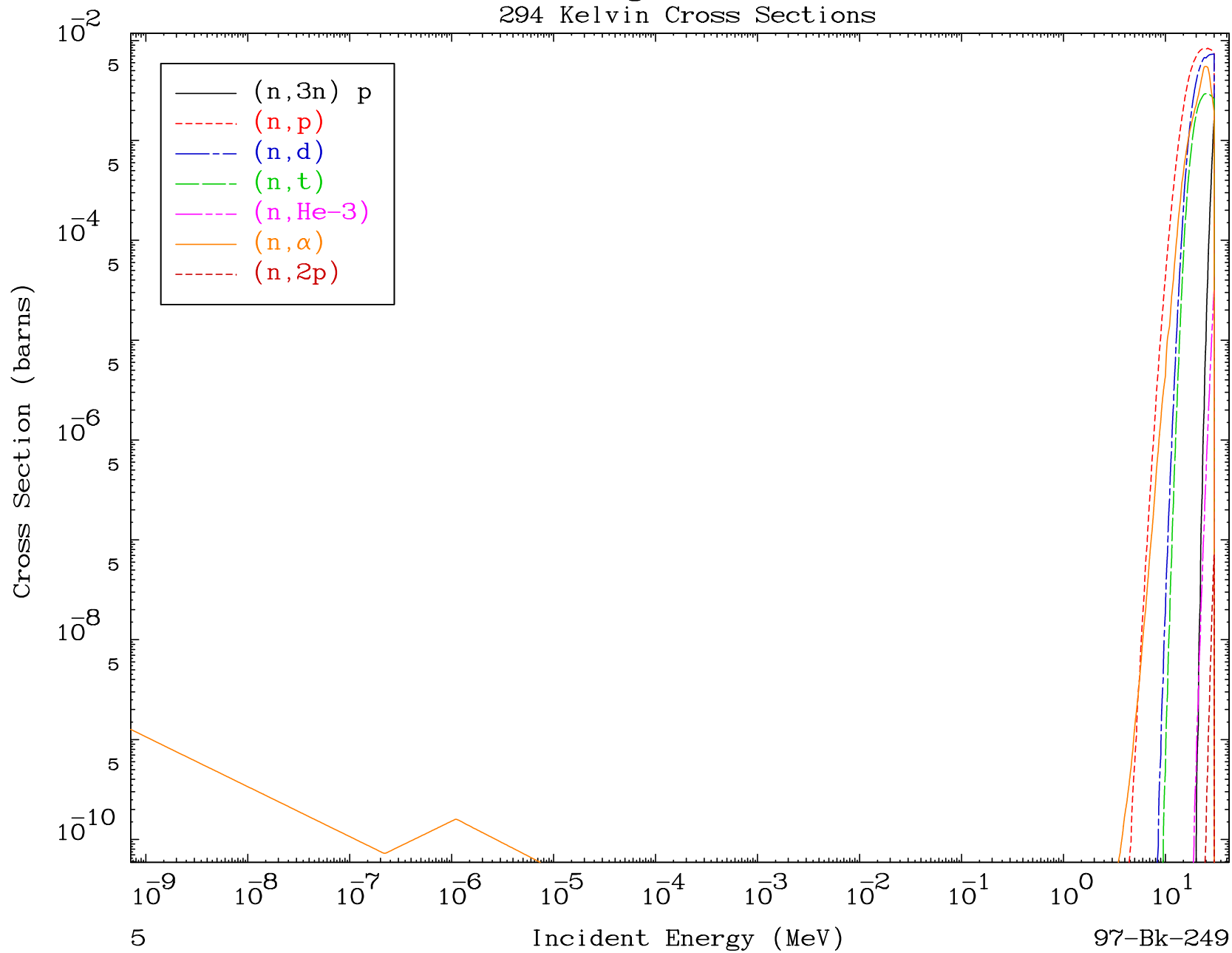
97-Bk-249

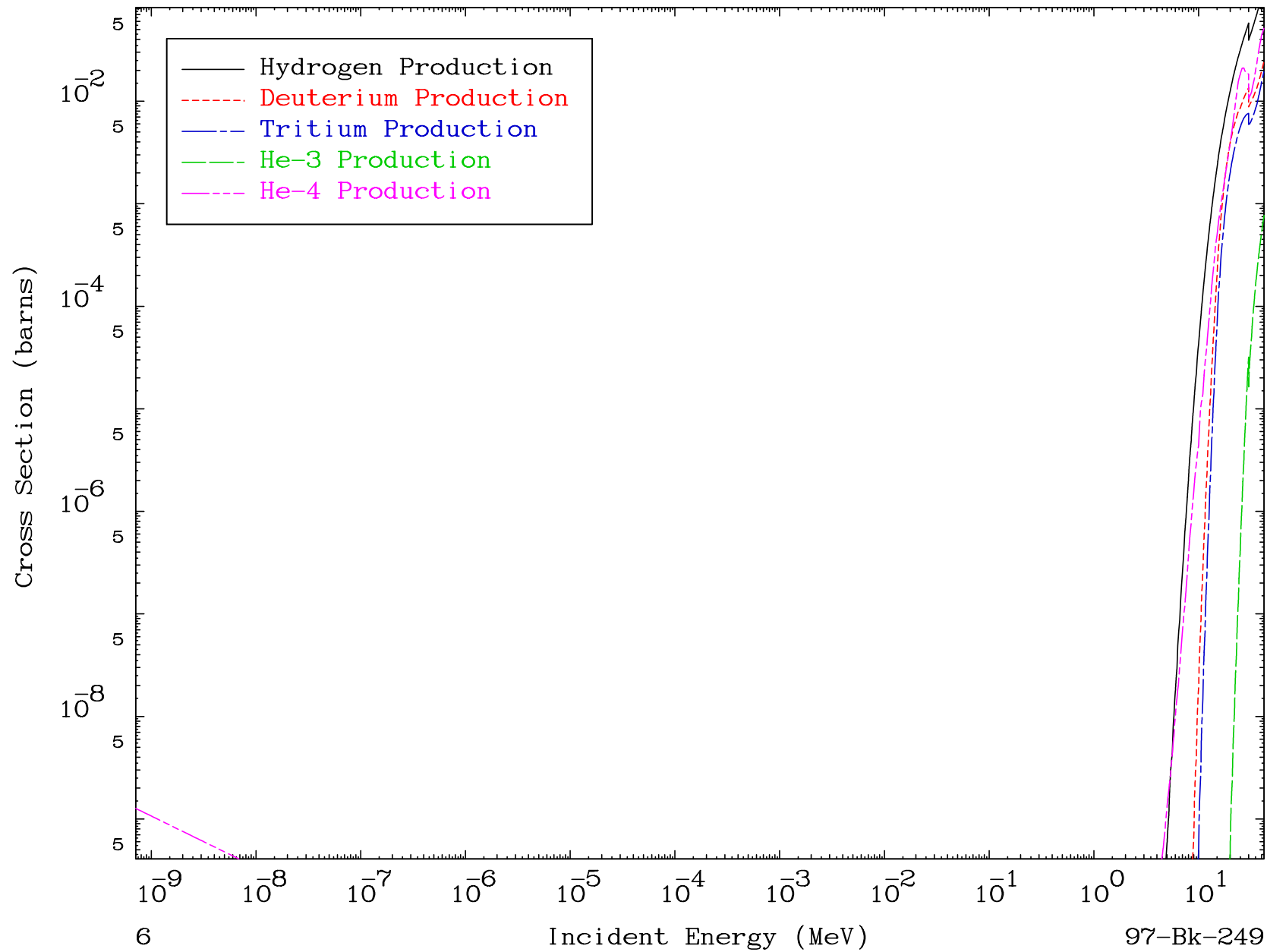


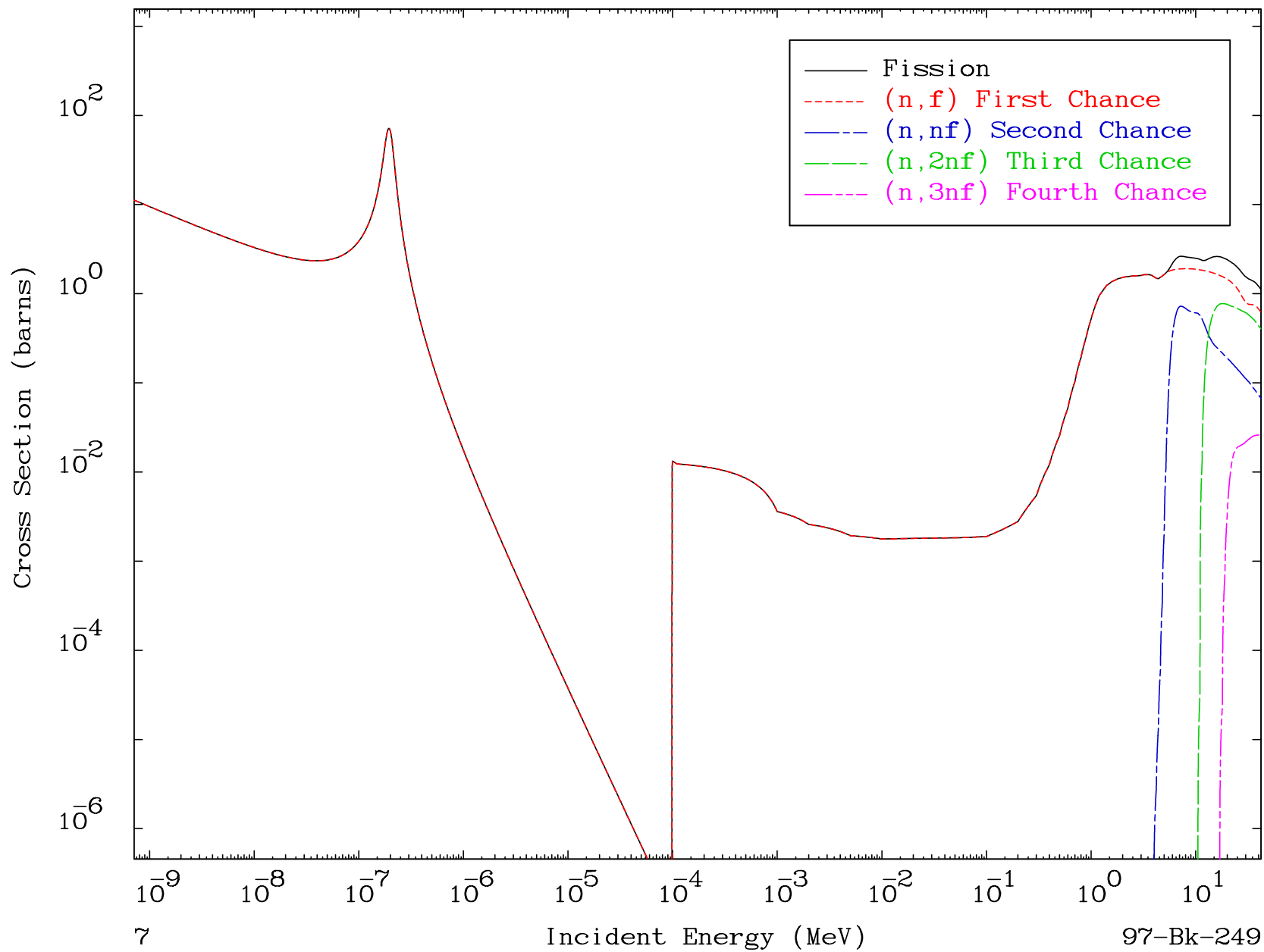
MAT 9752

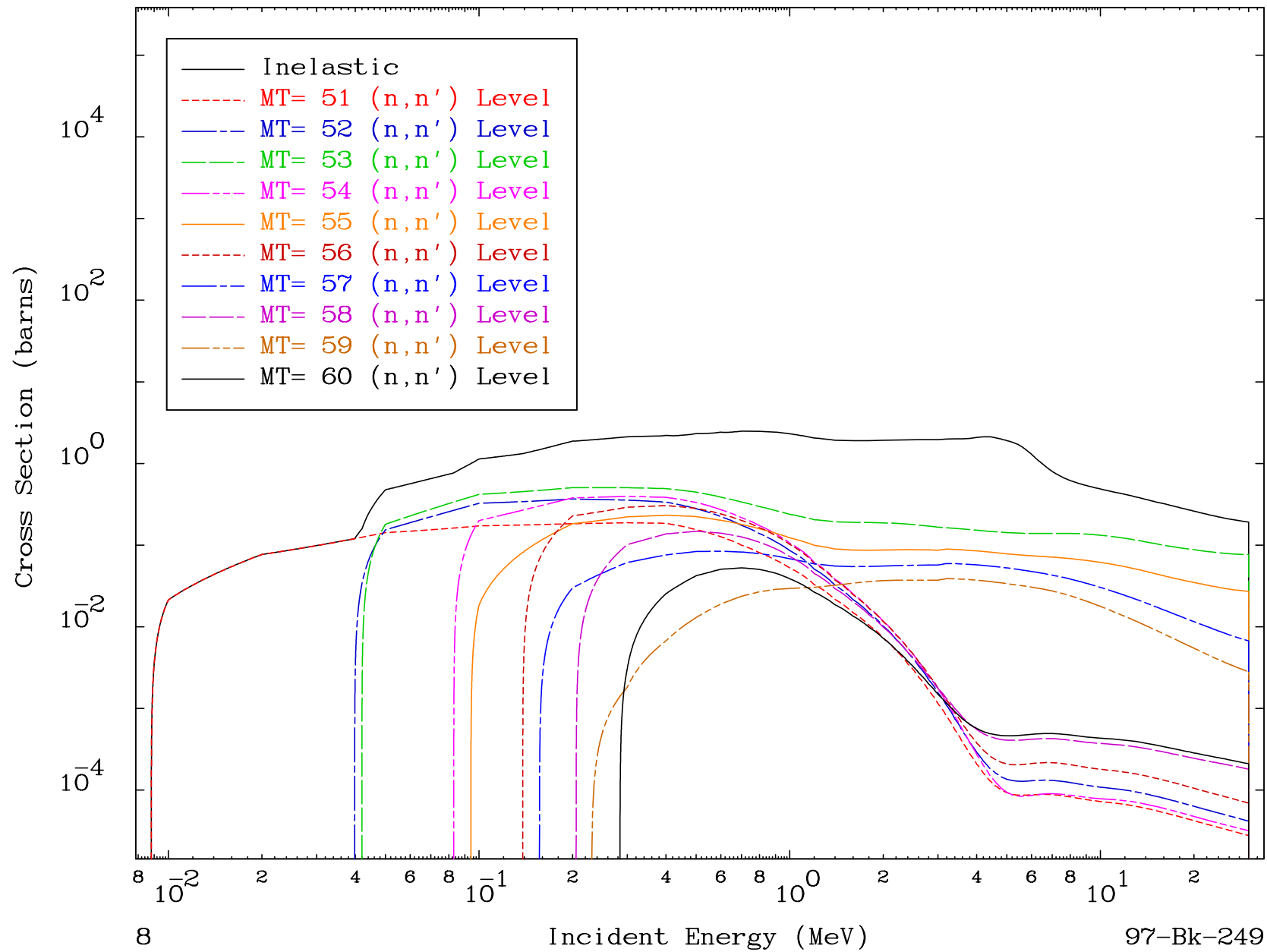
Charged Particle
294 Kelvin Cross Sections

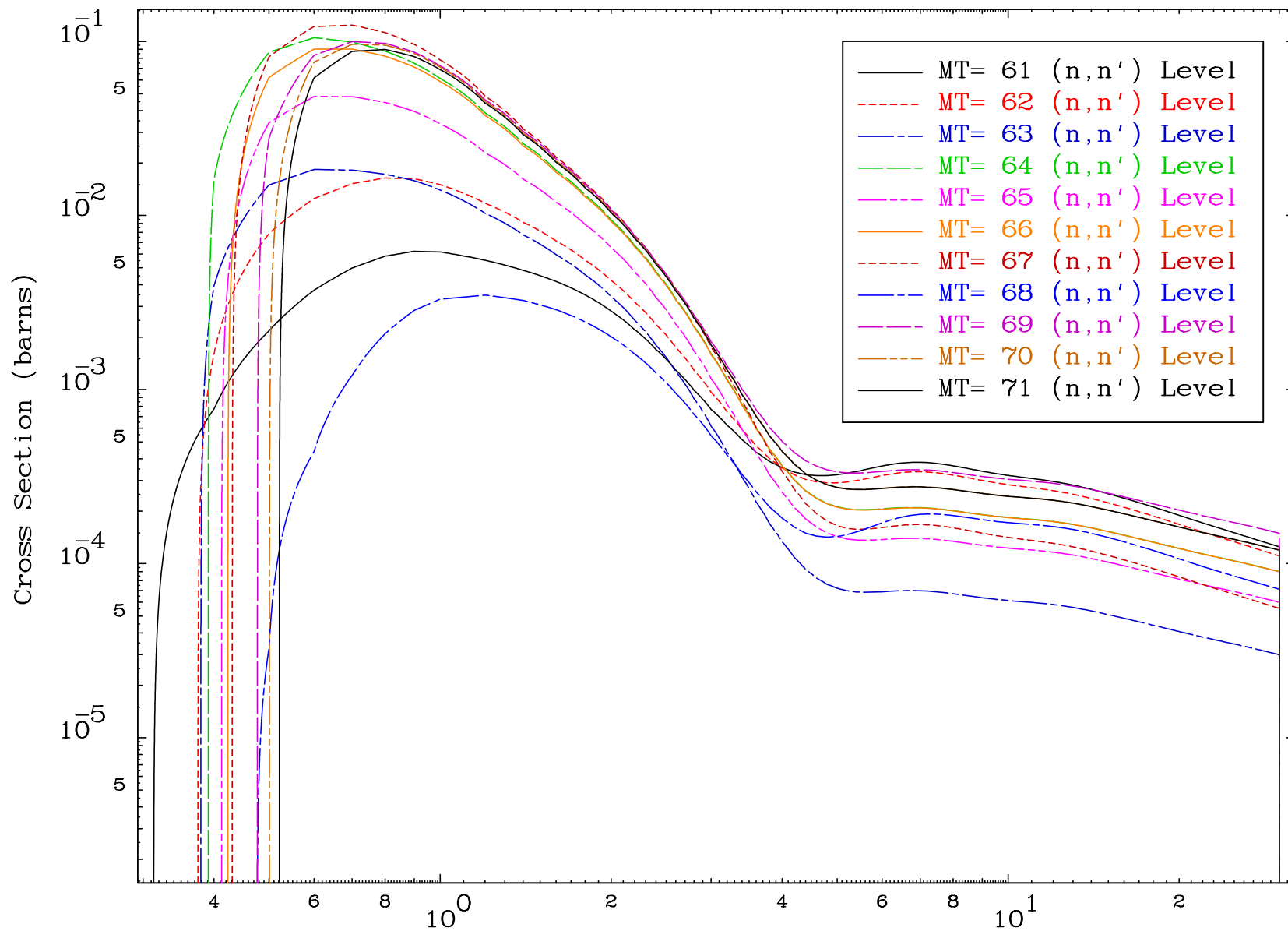
97-Bk-249







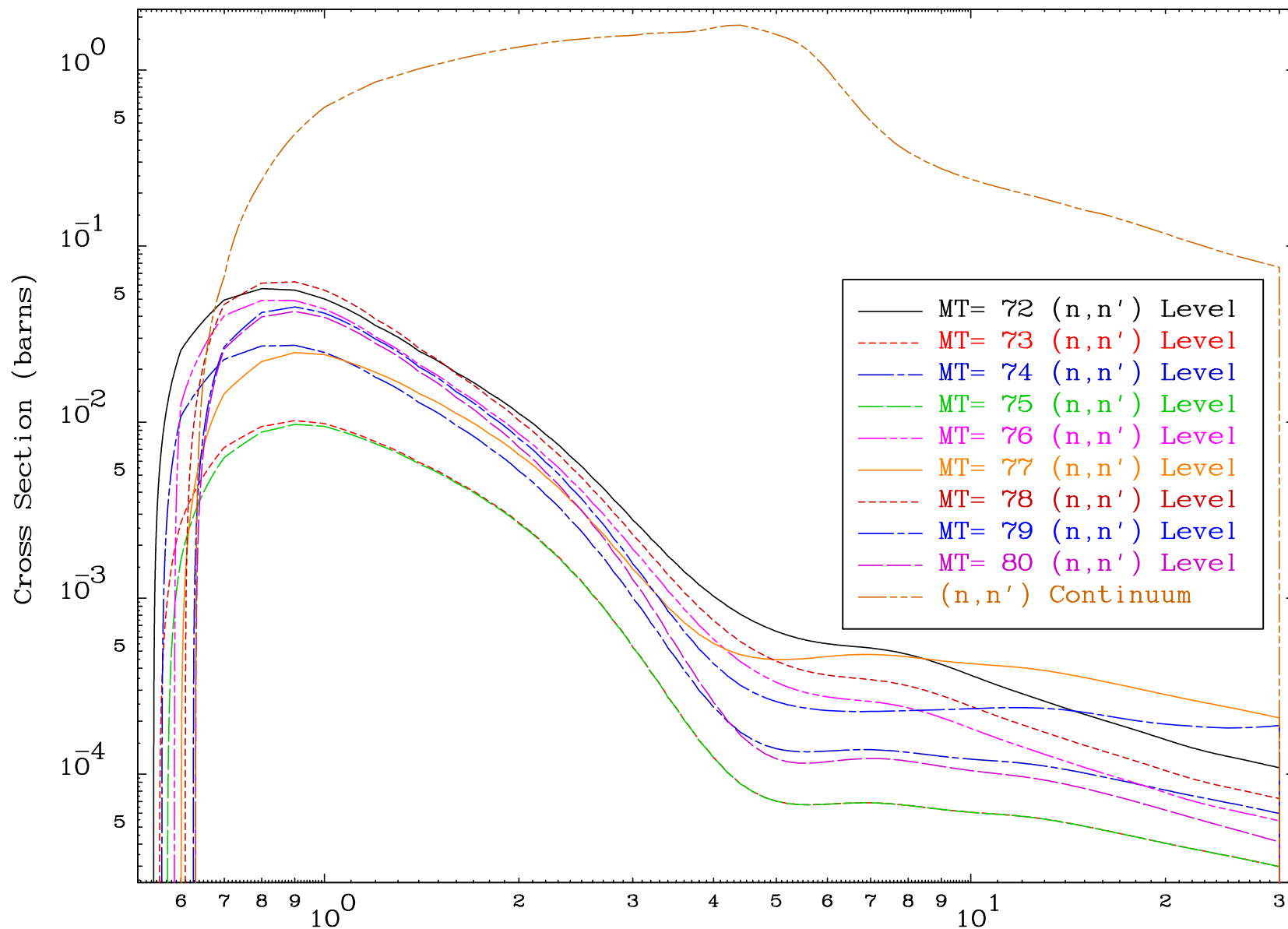




MAT 9752

 (n,n') Level
294 Kelvin Cross Sections

97-Bk-249



10

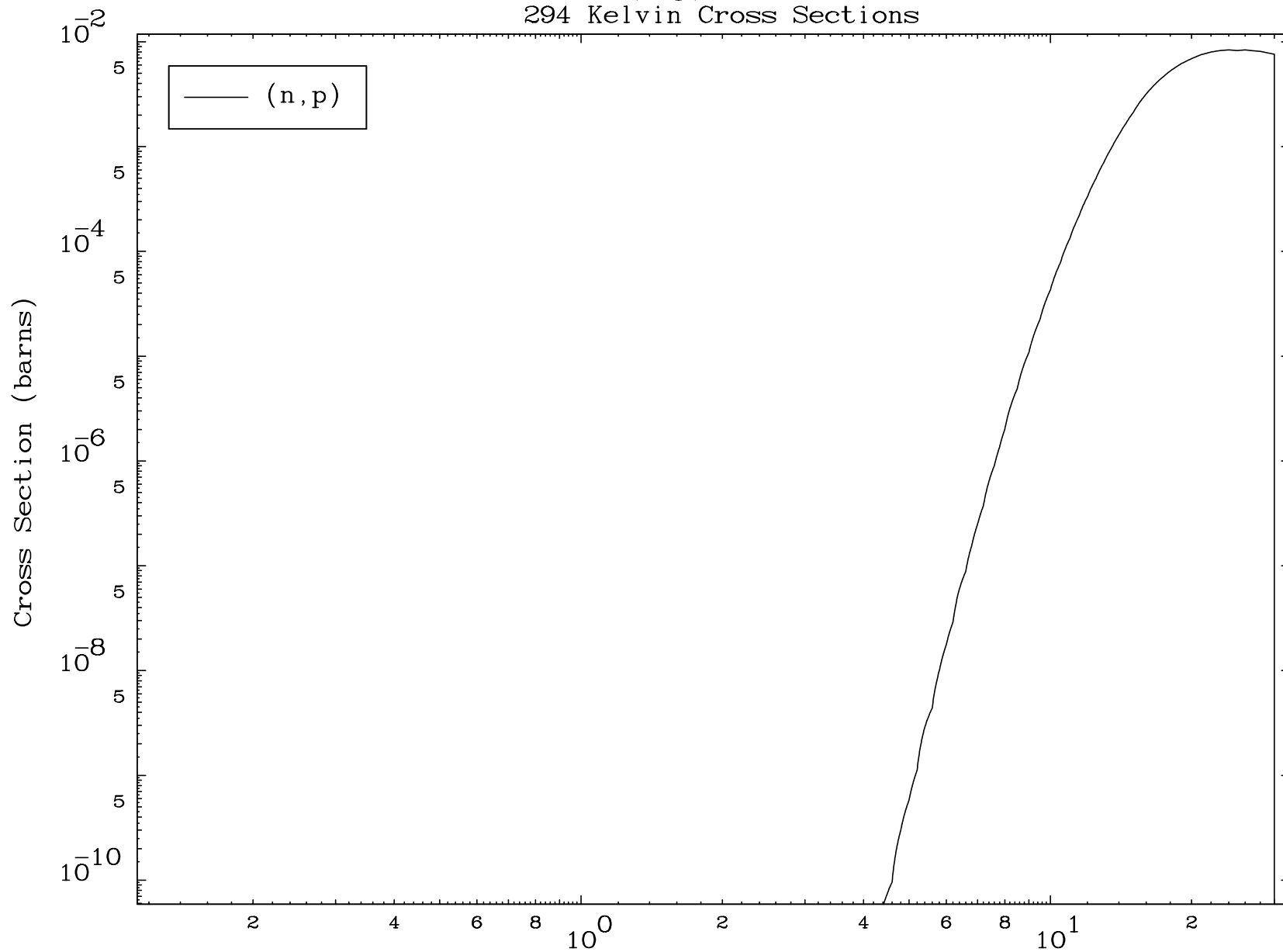
Incident Energy (MeV)

97-Bk-249

MAT 9752

(n,p) Levels
294 Kelvin Cross Sections

97-Bk-249



11

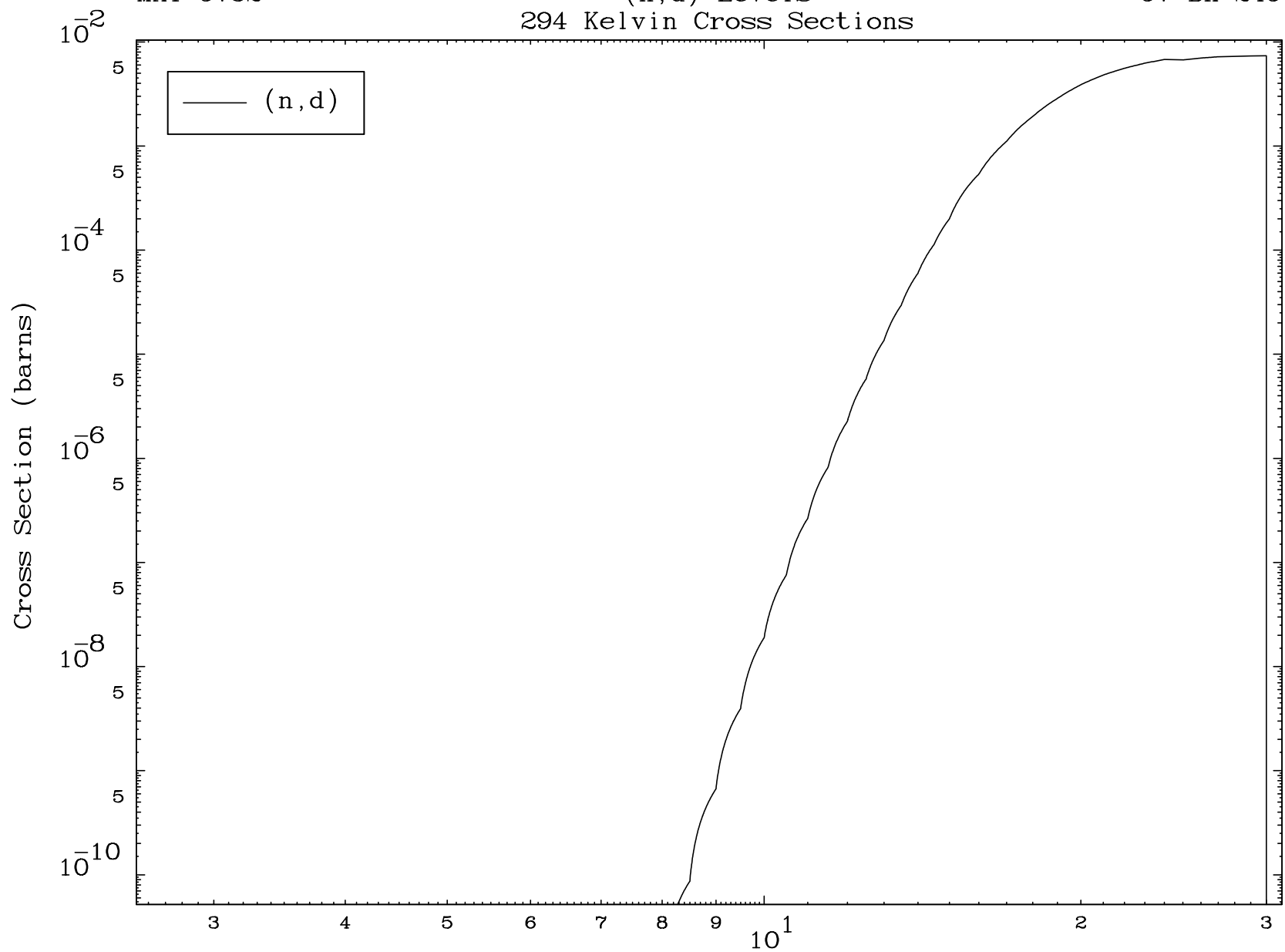
Incident Energy (MeV)

97-Bk-249

MAT 9752

(n,d) Levels
294 Kelvin Cross Sections

97-Bk-249



12

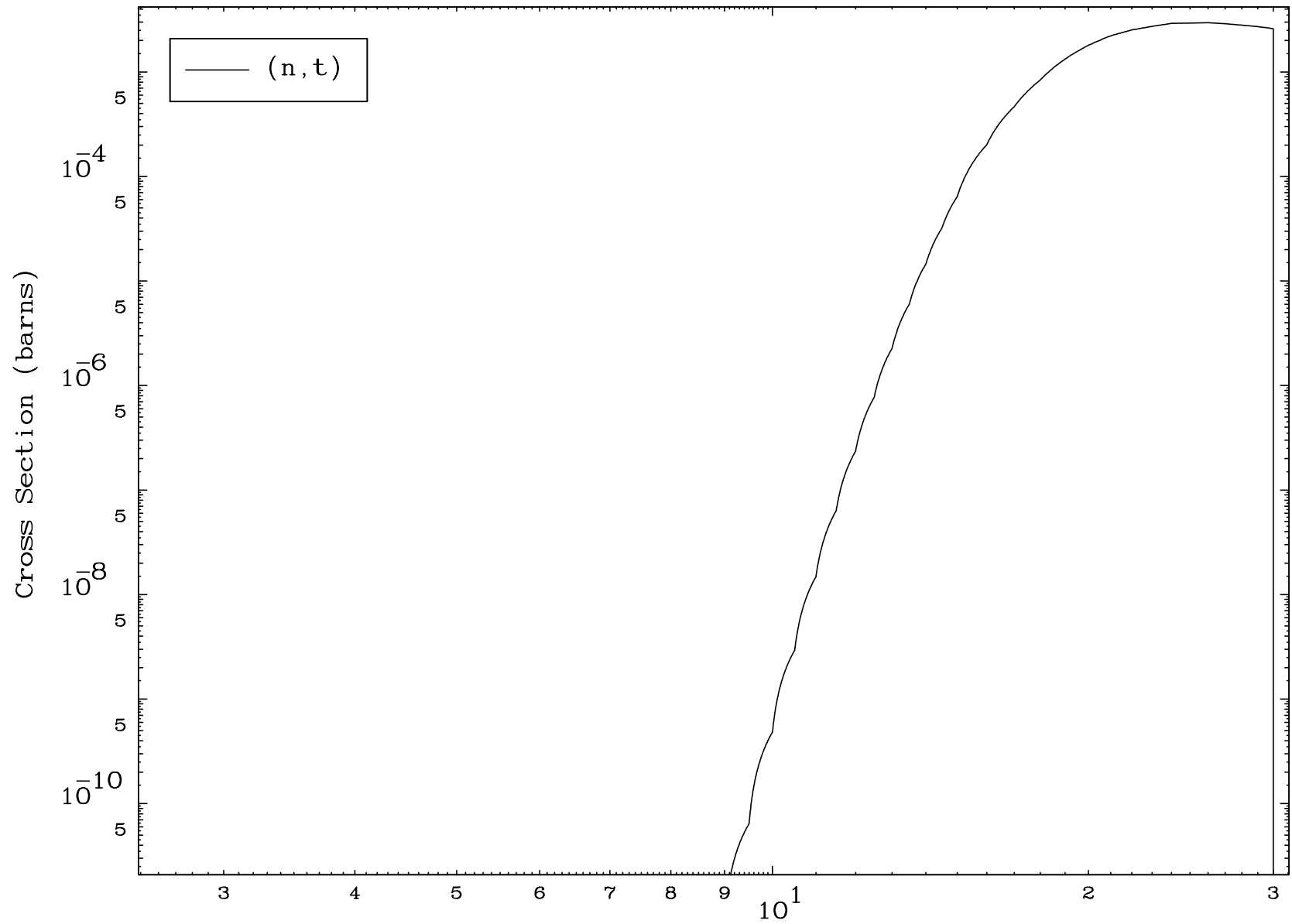
Incident Energy (MeV)

97-Bk-249

MAT 9752

(n,t) Levels
294 Kelvin Cross Sections

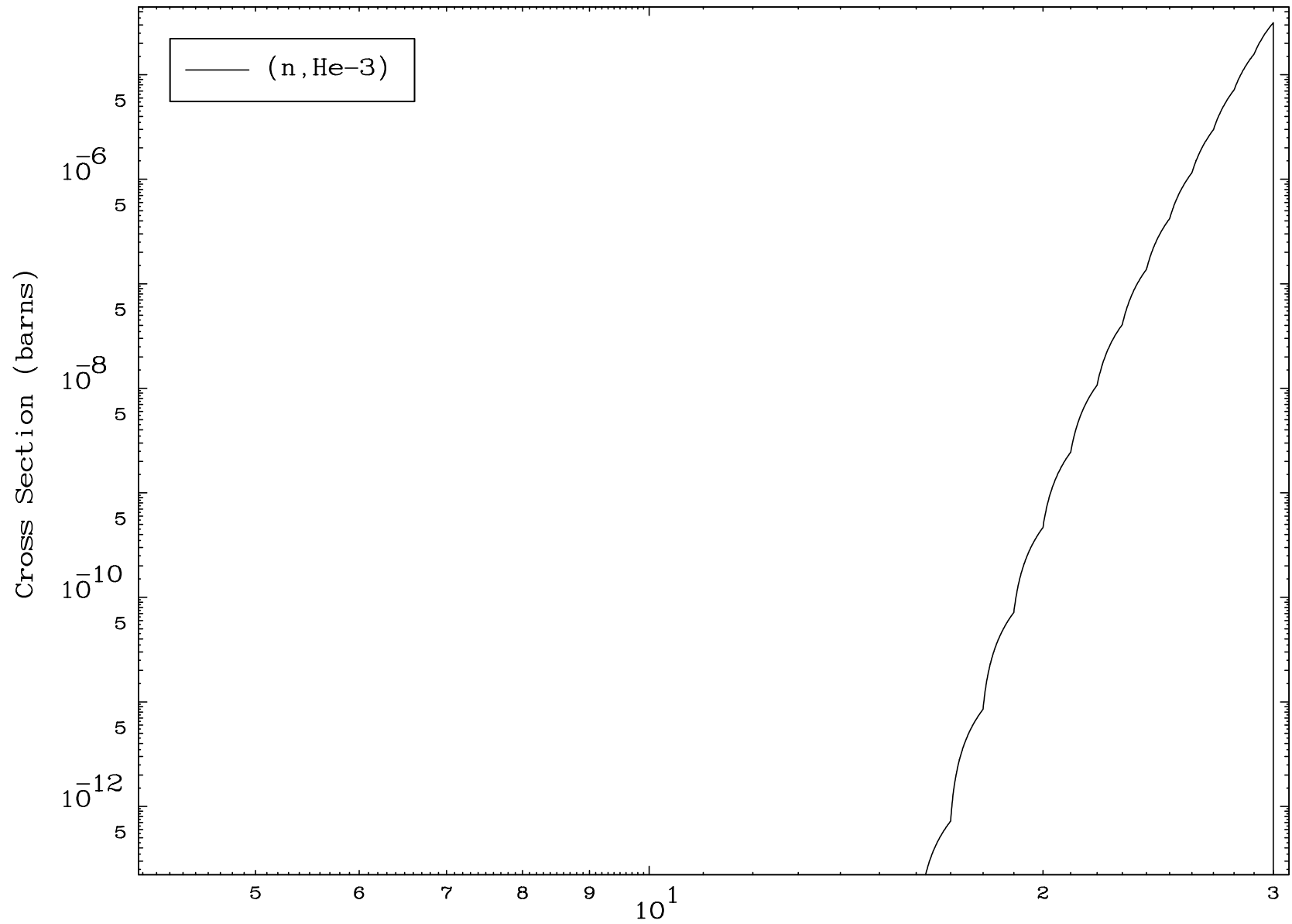
97-Bk-249



13

Incident Energy (MeV)

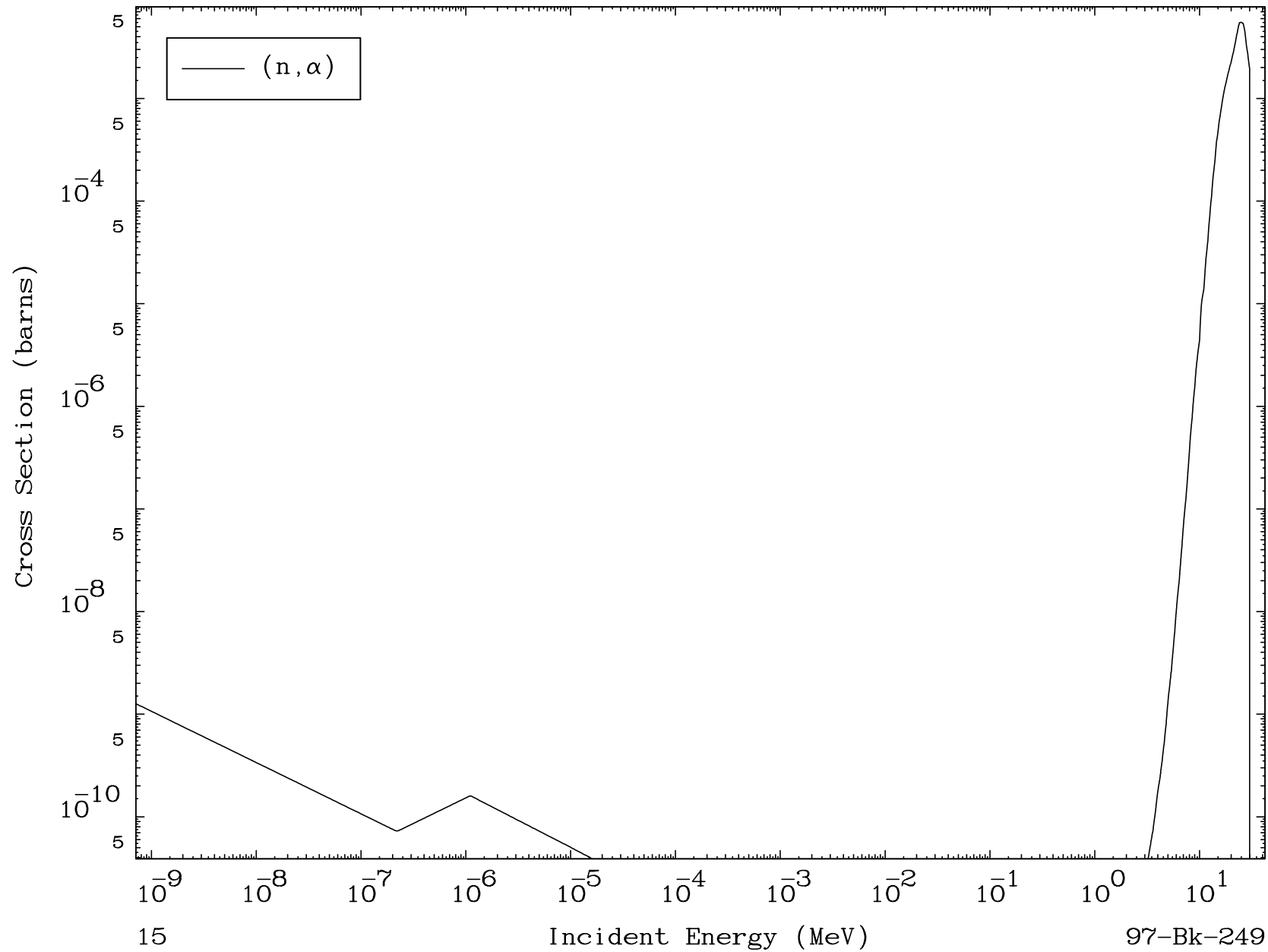
97-Bk-249



MAT 9752

(n, α) Levels
294 Kelvin Cross Sections

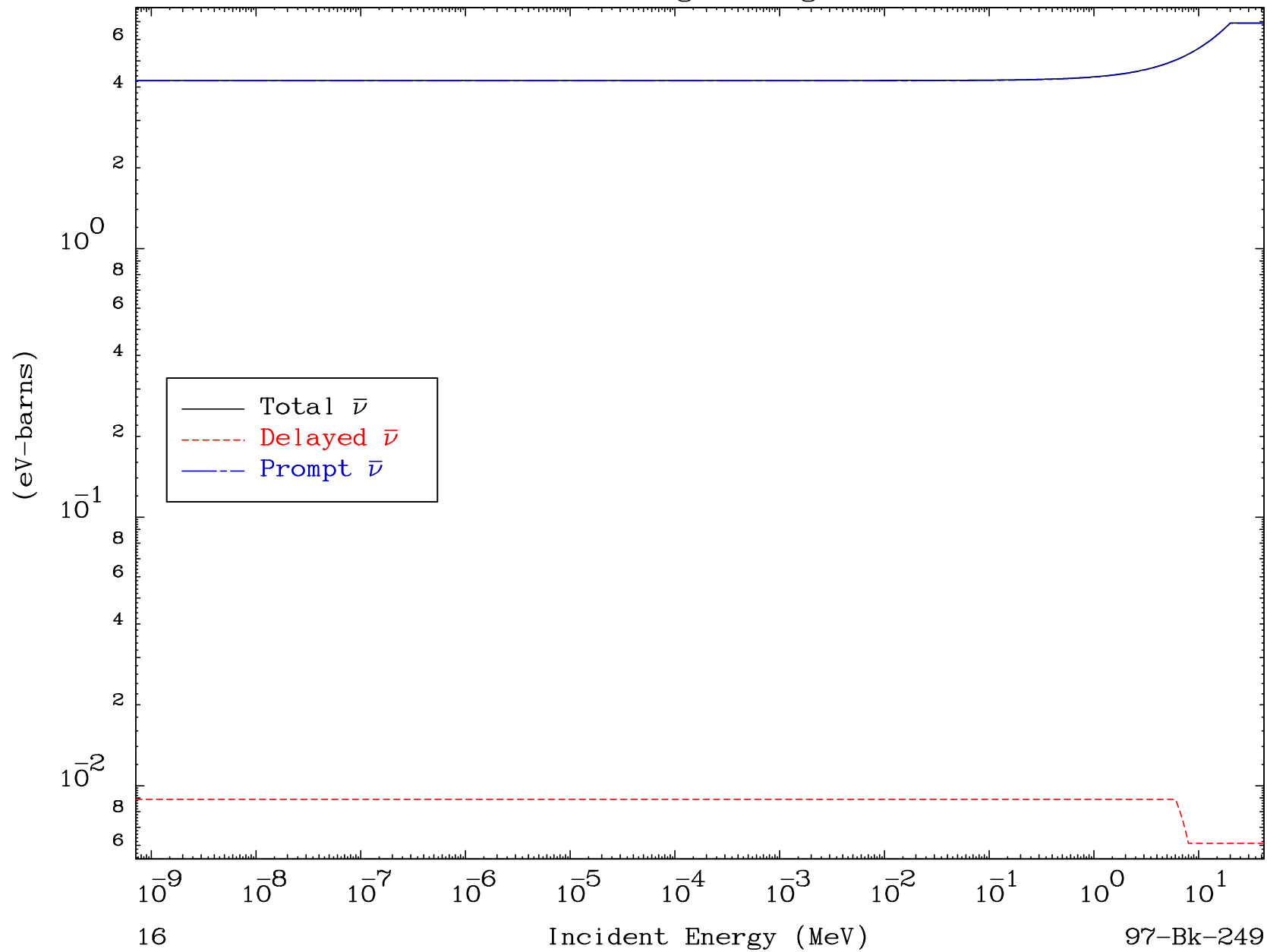
97-Bk-249



MAT 9752

Energy Release
Heating - Damage

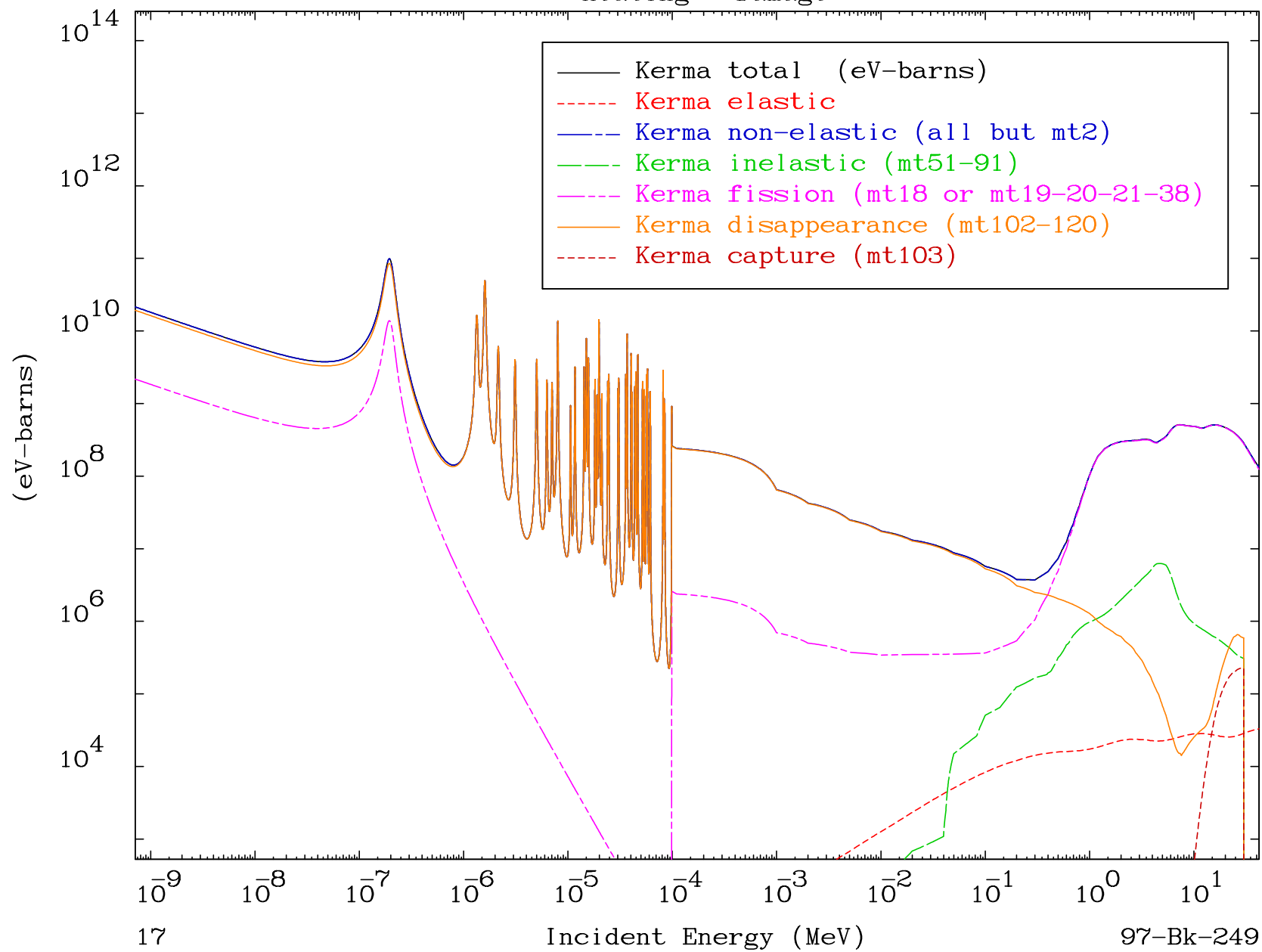
97-Bk-249

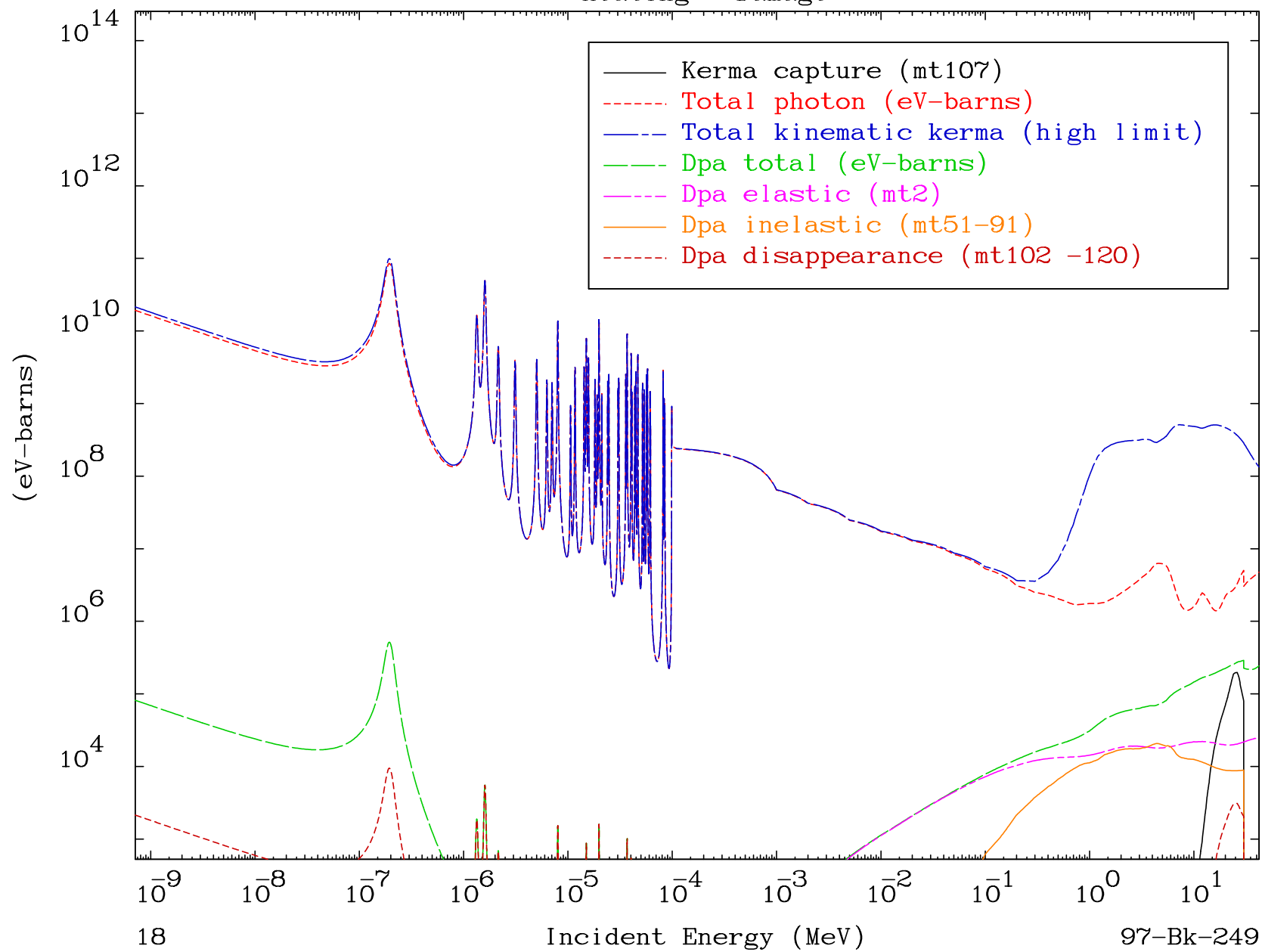


MAT 9752

Energy Release
Heating - Damage

97-Bk-249



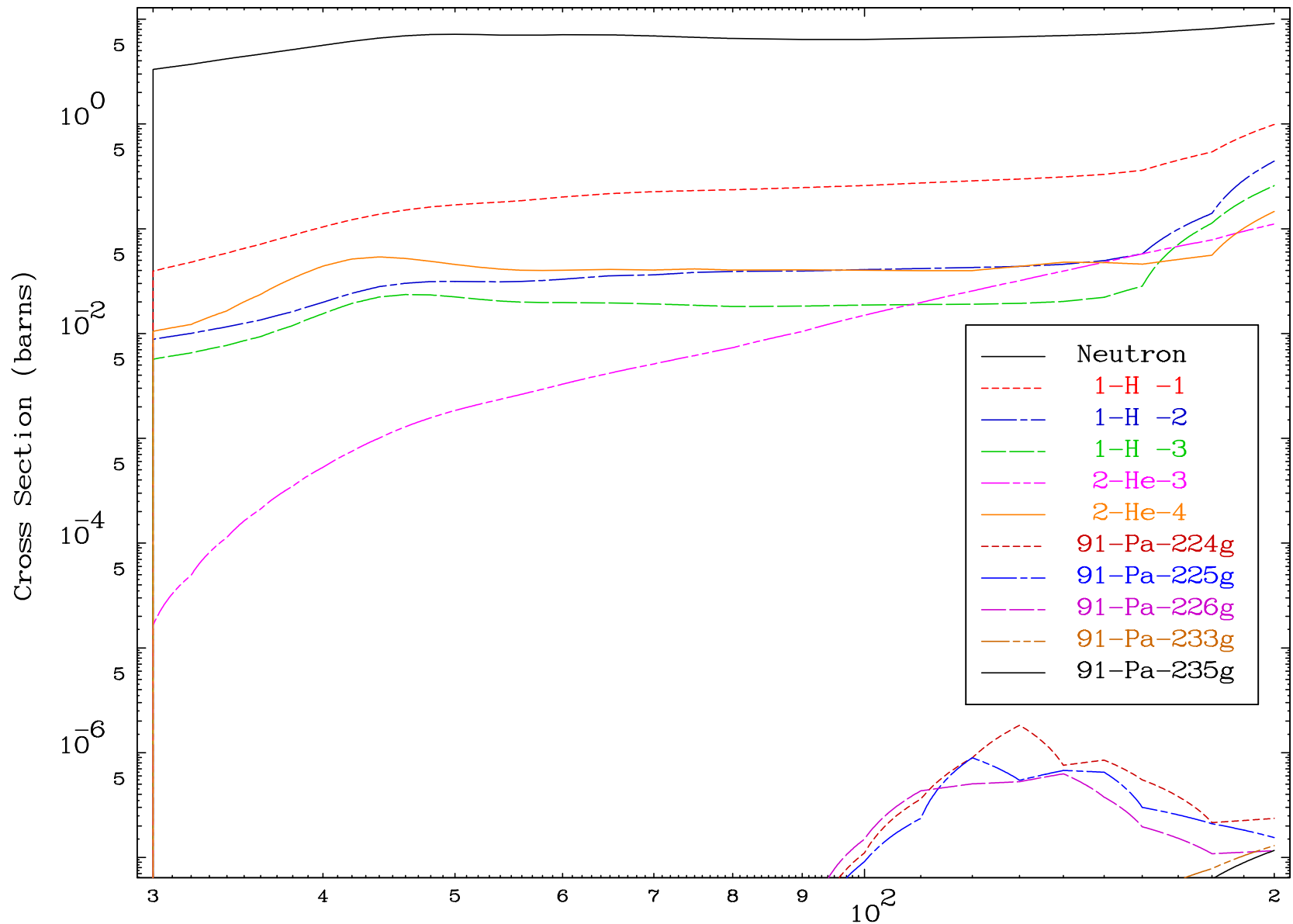


MAT 9752

(n,remainder)

97-Bk-249

Radionuclide Production Cross Section



19

Incident Energy (MeV)

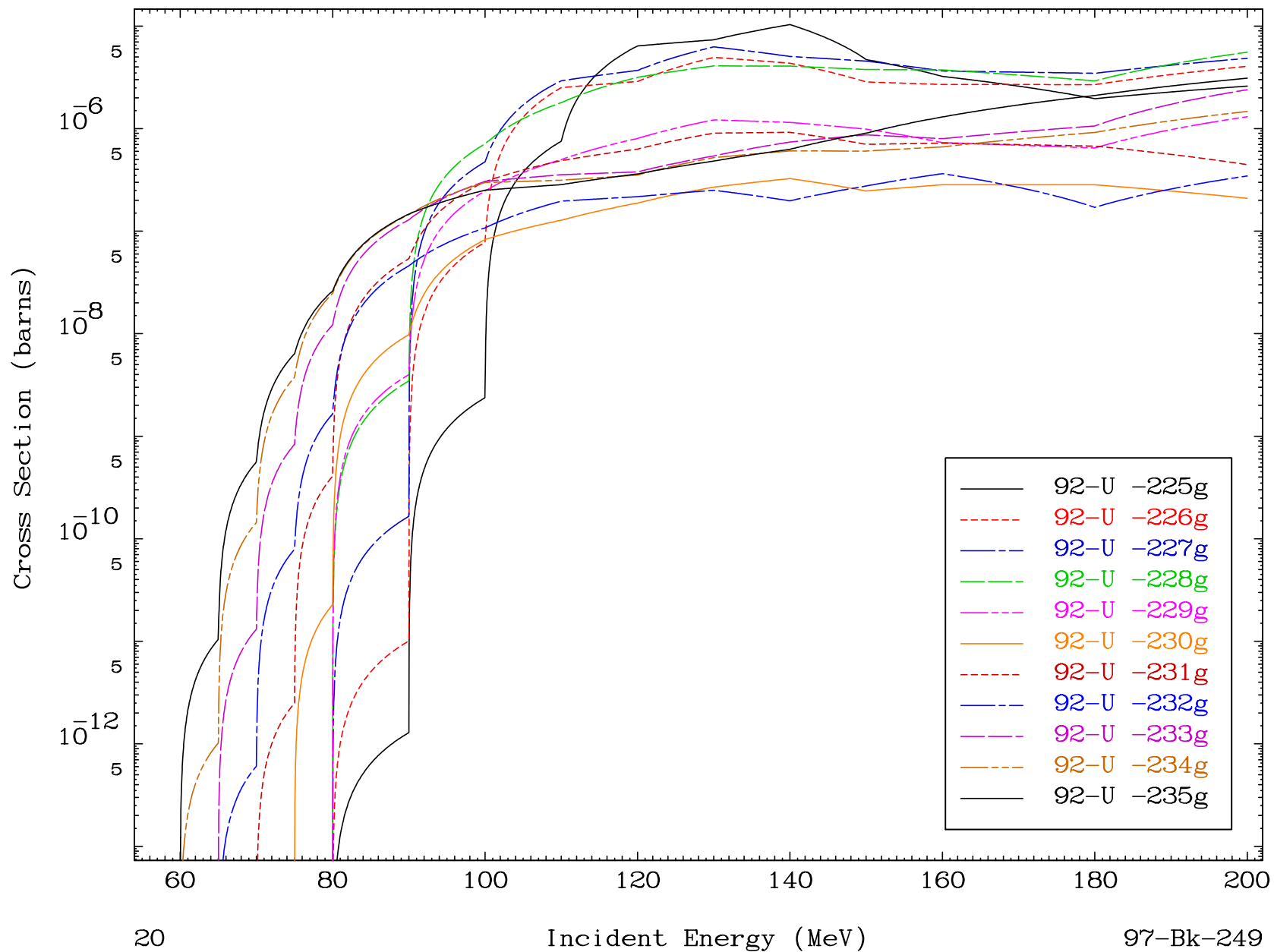
97-Bk-249

MAT 9752

(n,remainder)

97-Bk-249

Radionuclide Production Cross Section

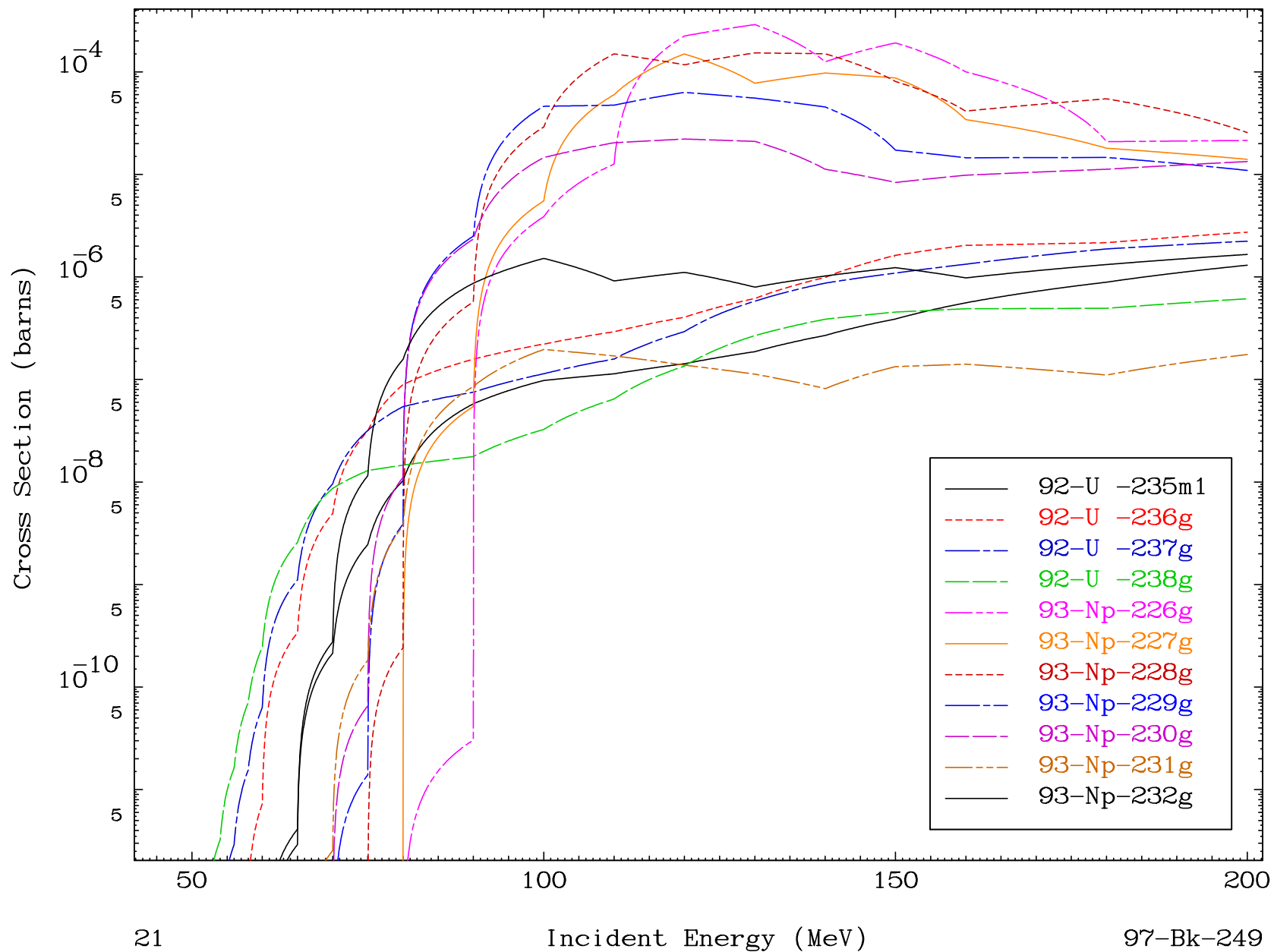


MAT 9752

(n,remainder)

97-Bk-249

Radionuclide Production Cross Section

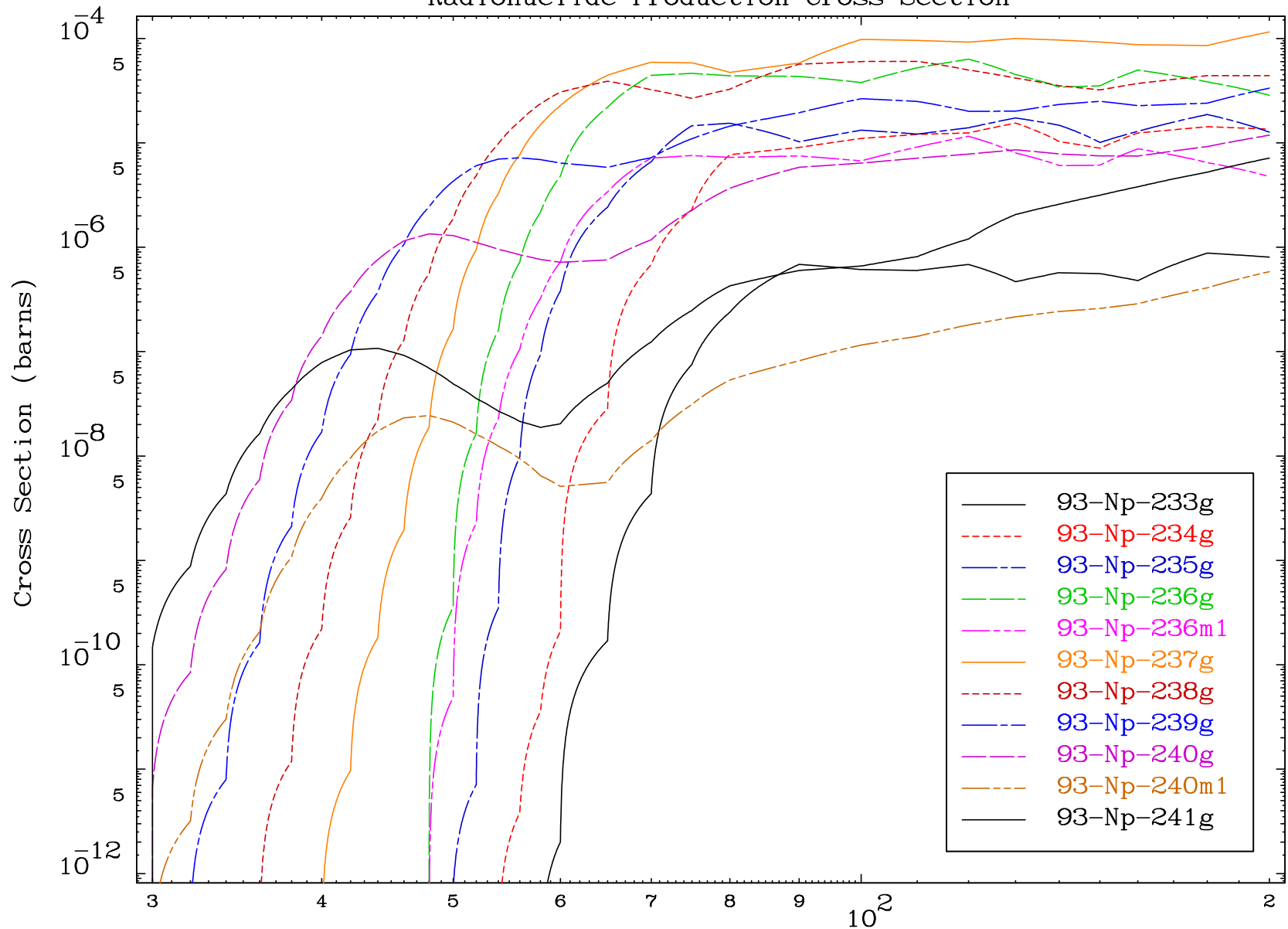


MAT 9752

(n,remainder)

97-Bk-249

Radionuclide Production Cross Section



22

Incident Energy (MeV)

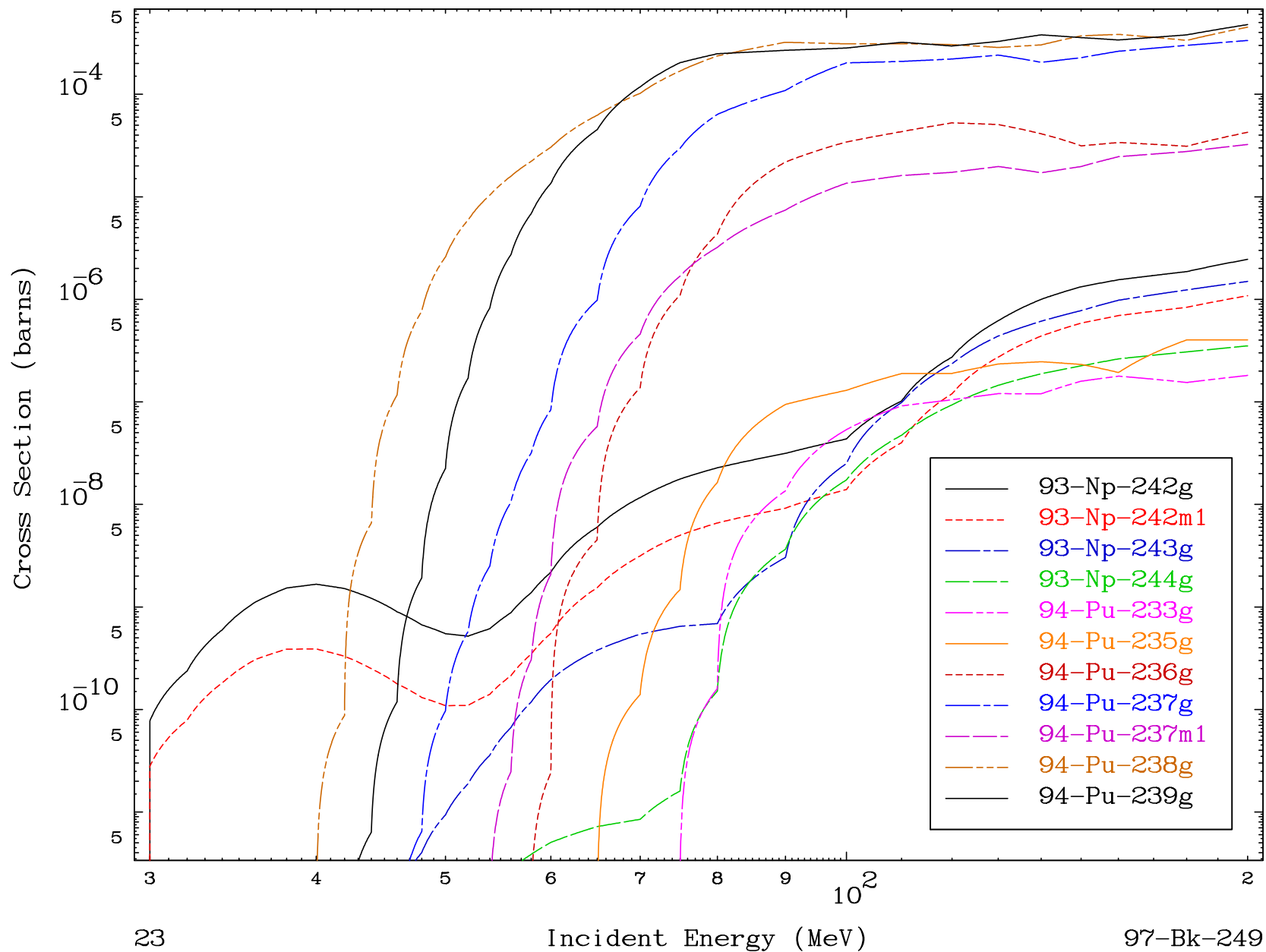
97-Bk-249

MAT 9752

(n,remainder)

97-Bk-249

Radionuclide Production Cross Section

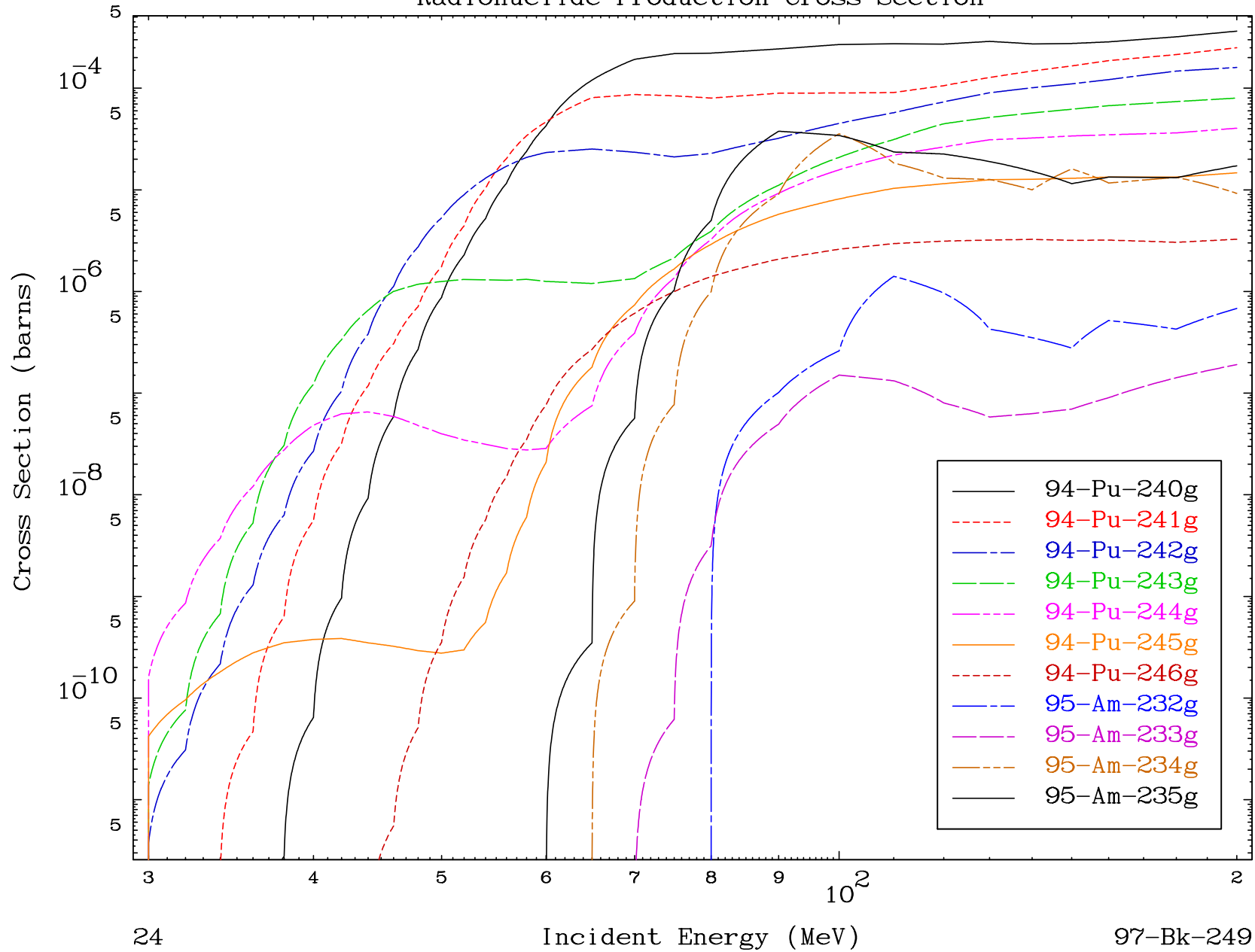


MAT 9752

(n,remainder)

97-Bk-249

Radionuclide Production Cross Section

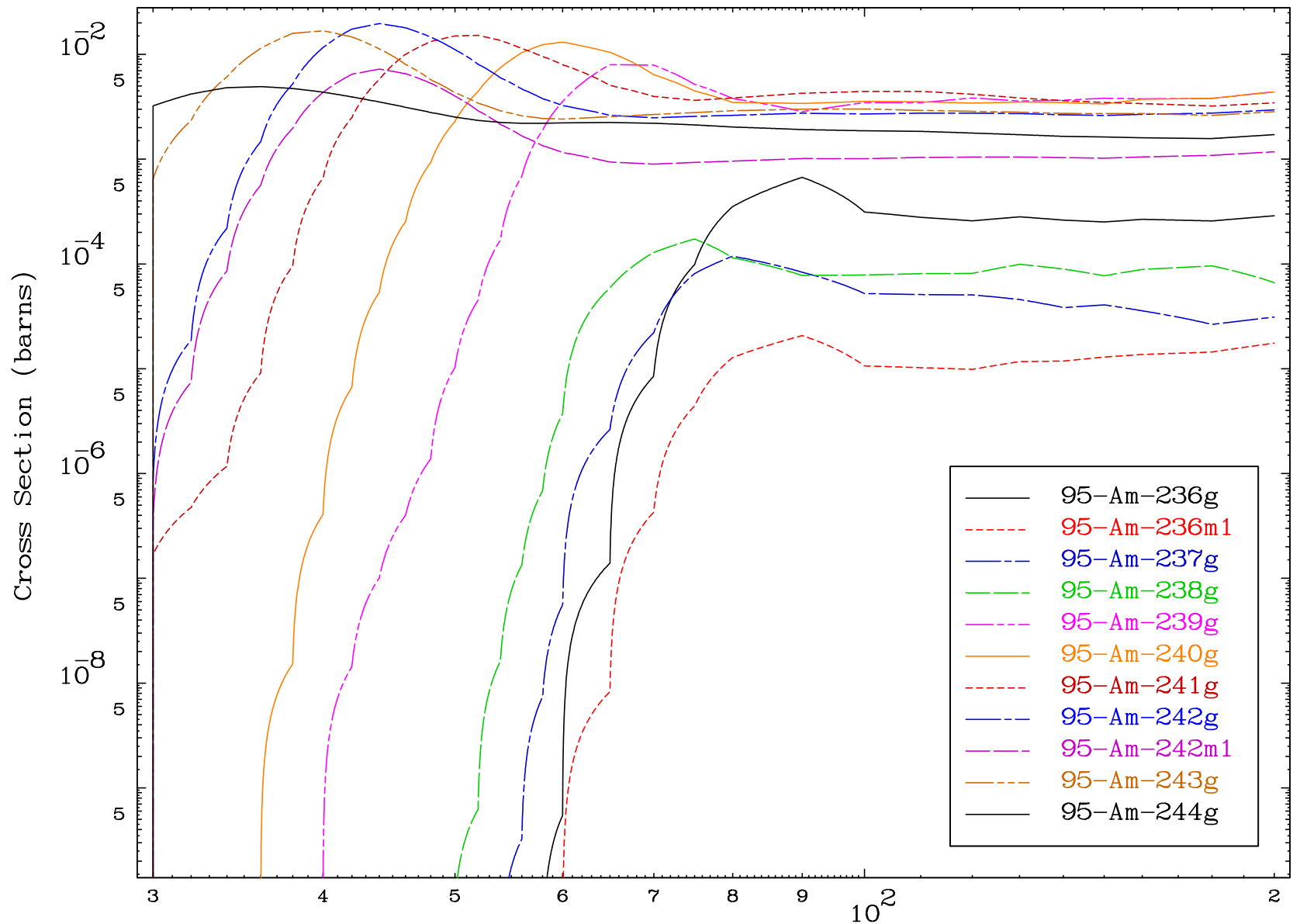


MAT 9752

(n,remainder)

97-Bk-249

Radionuclide Production Cross Section



25

Incident Energy (MeV)

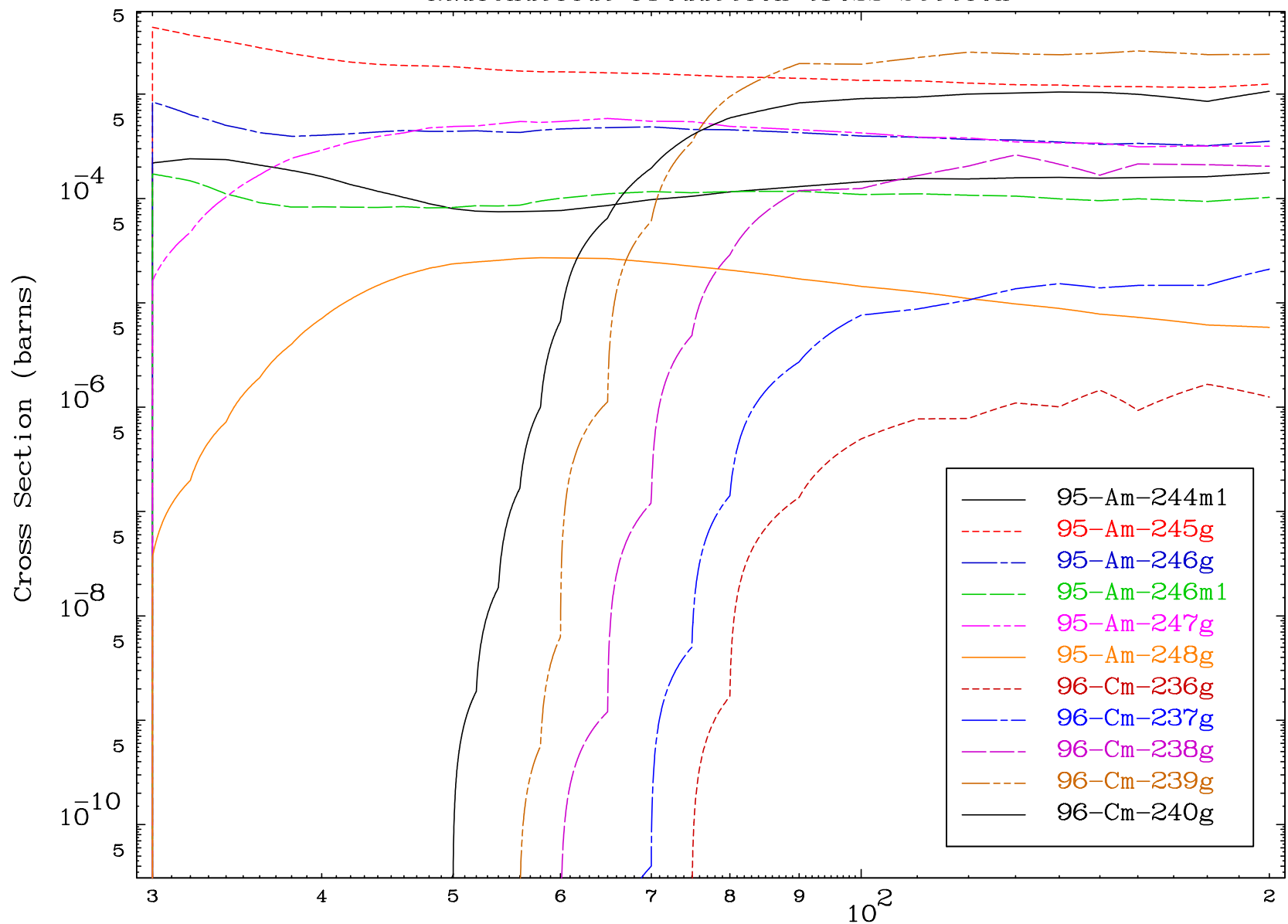
97-Bk-249

MAT 9752

(n,remainder)

97-Bk-249

Radionuclide Production Cross Section



26

Incident Energy (MeV)

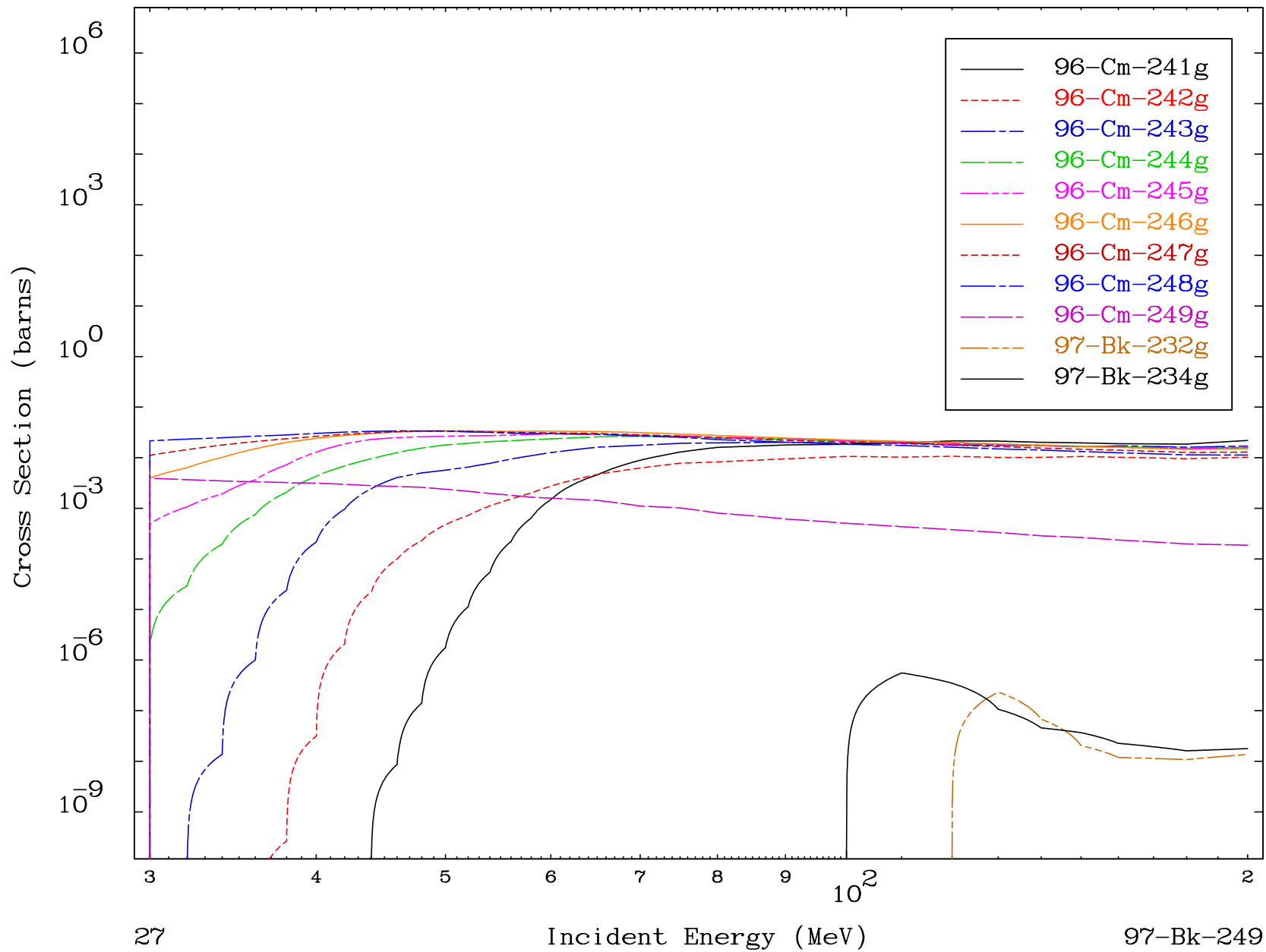
97-Bk-249

MAT 9752

(n,remainder)

97-Bk-249

Radionuclide Production Cross Section

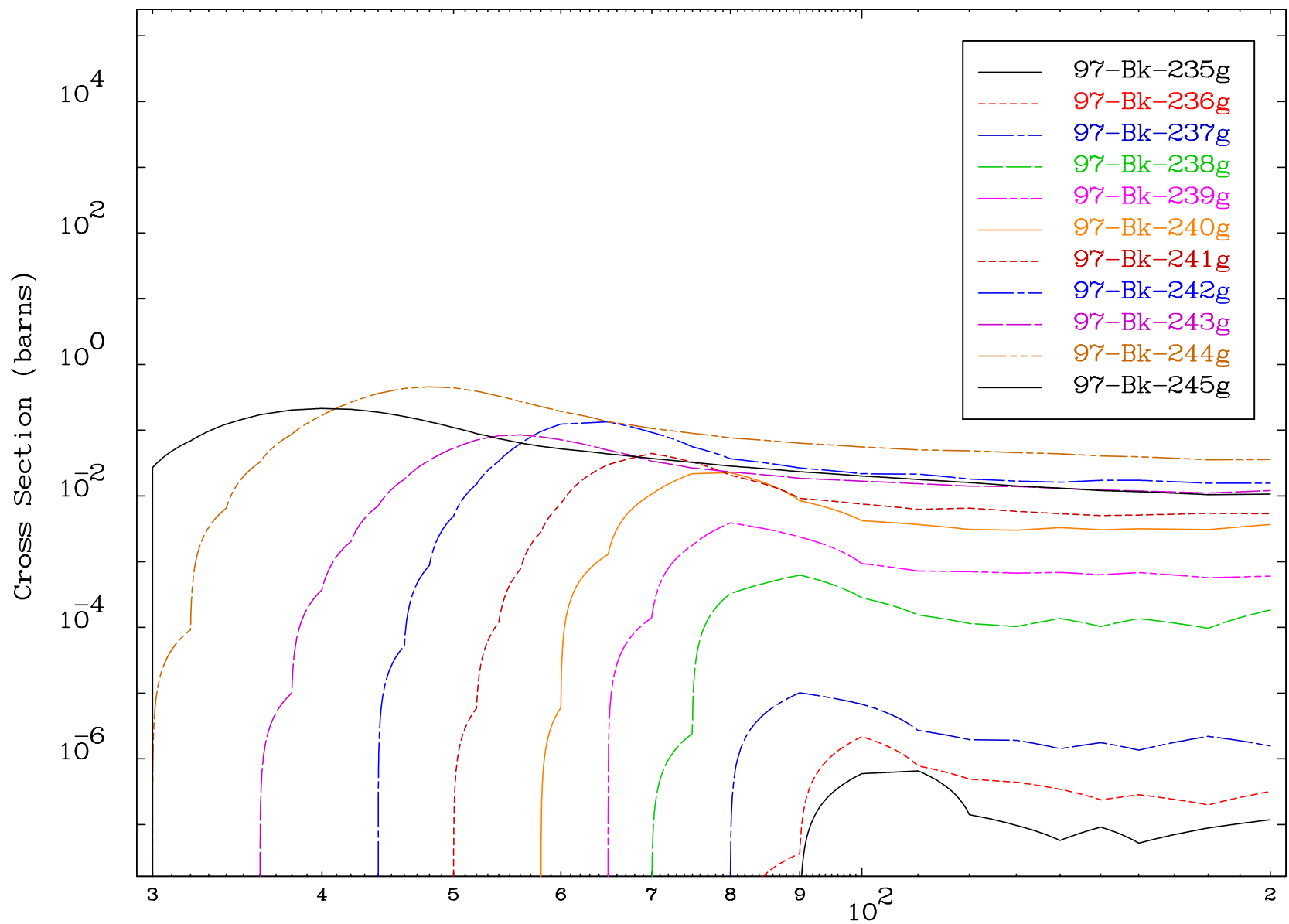


MAT 9752

(n,remainder)

97-Bk-249

Radionuclide Production Cross Section



28

Incident Energy (MeV)

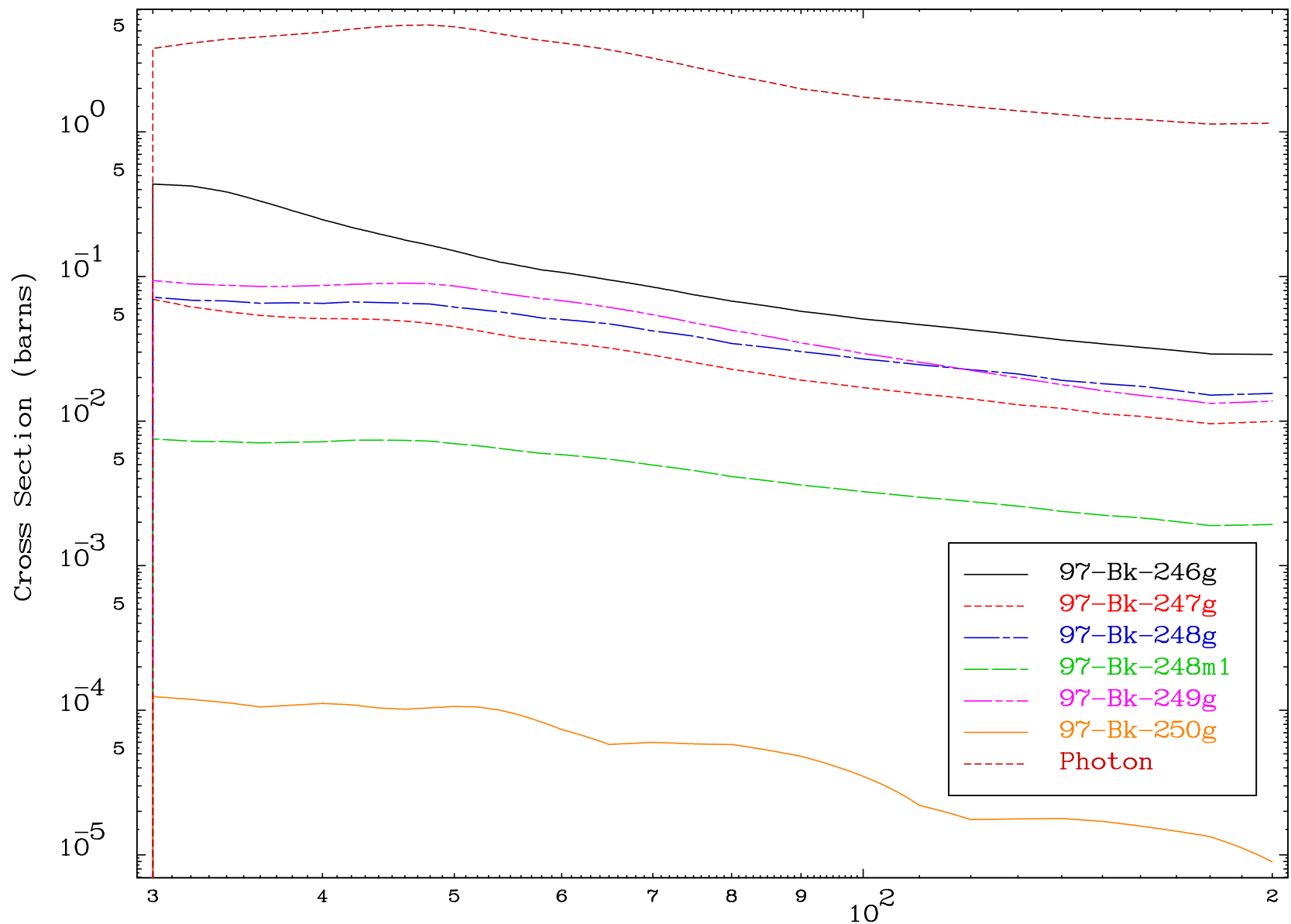
97-Bk-249

MAT 9752

(n,remainder)

97-Bk-249

Radionuclide Production Cross Section



29

Incident Energy (MeV)

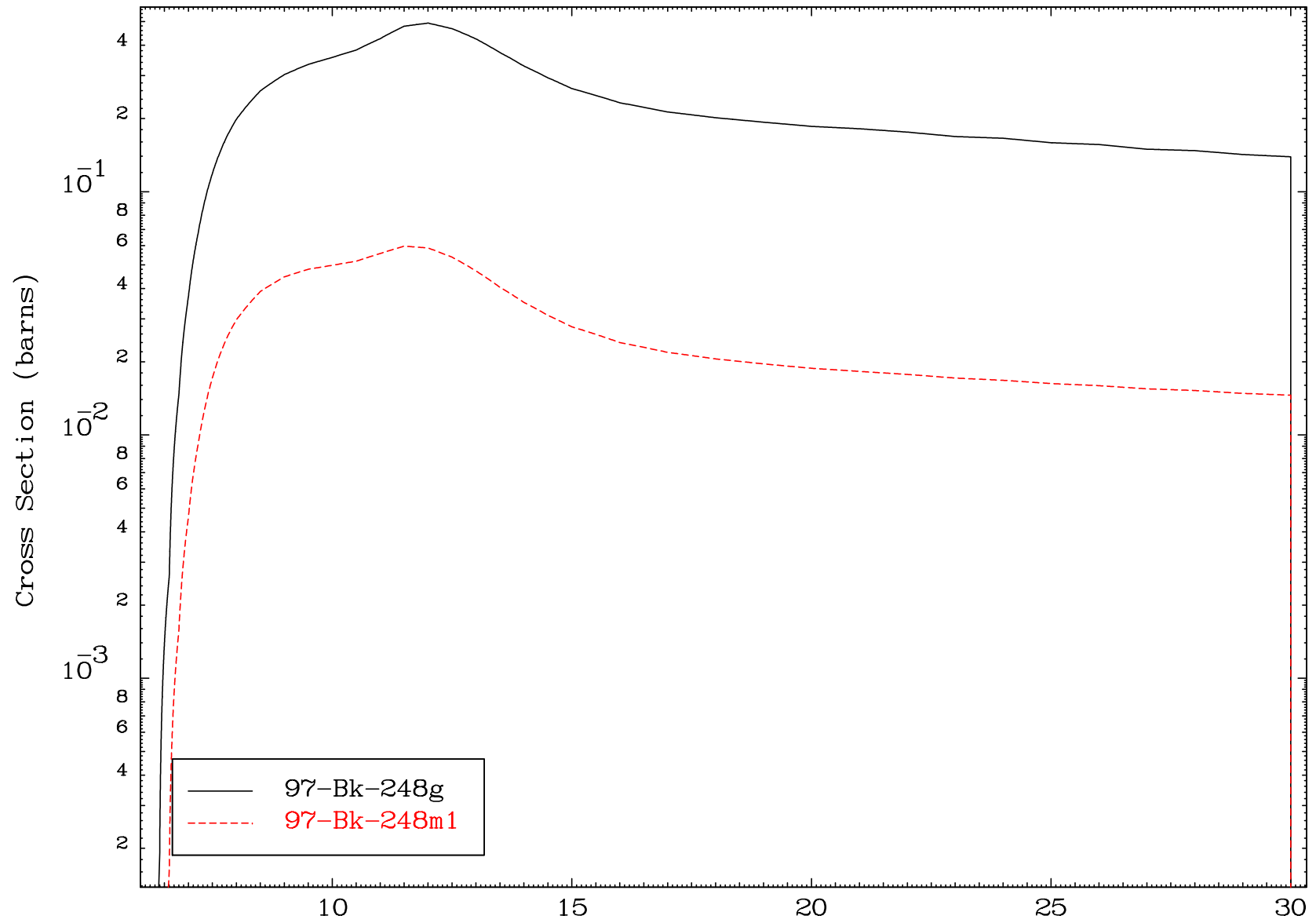
97-Bk-249

MAT 9752

(n,2n)

97-Bk-249

Radionuclide Production Cross Section



30

Incident Energy (MeV)

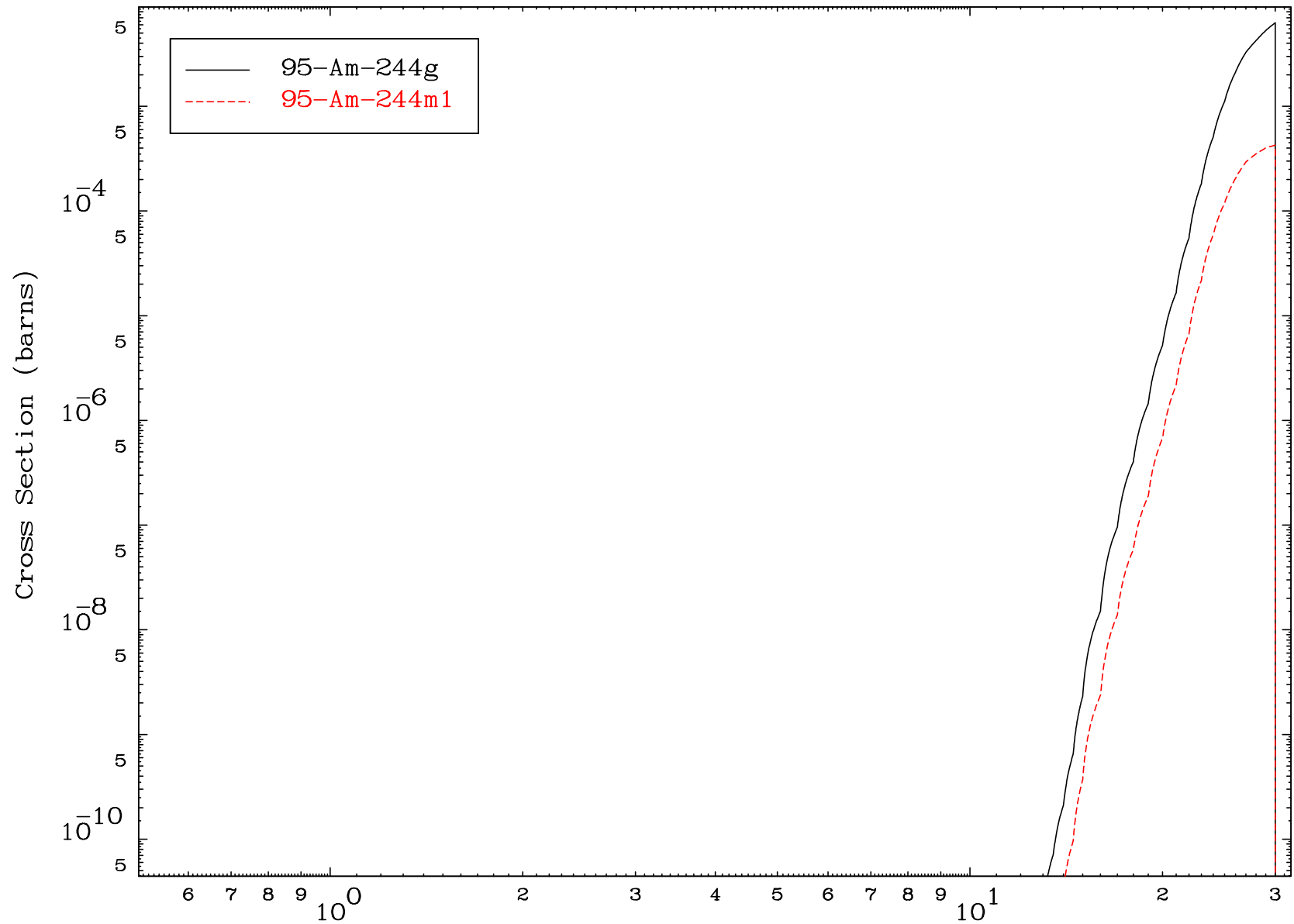
97-Bk-249

MAT 9752

 $(n,2n) \alpha$

97-Bk-249

Radionuclide Production Cross Section



31

Incident Energy (MeV)

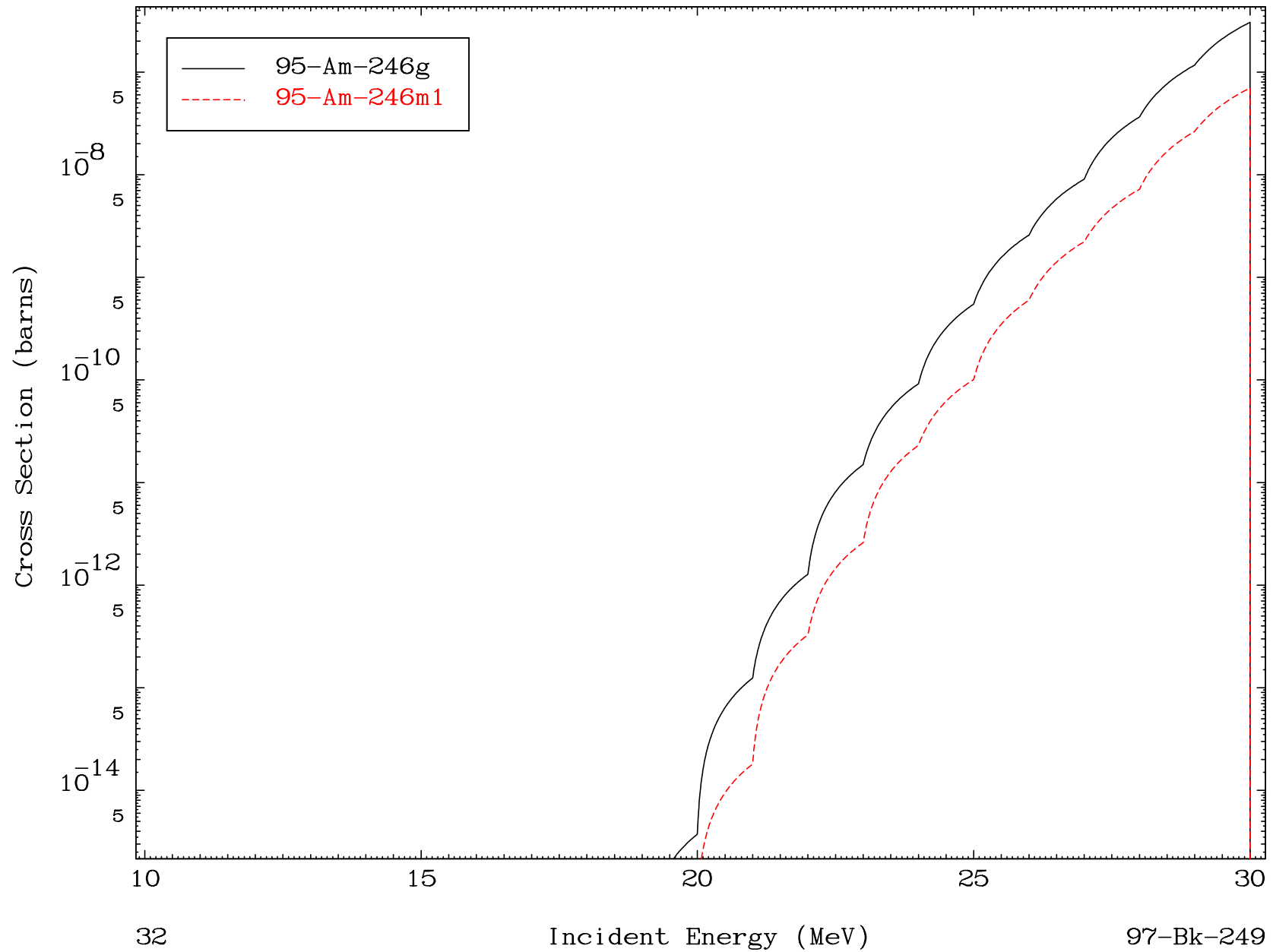
97-Bk-249

MAT 9752

(n,n') He-3

97-Bk-249

Radionuclide Production Cross Section



MAT 9752

(n, α)

97-Bk-249

Radionuclide Production Cross Section

