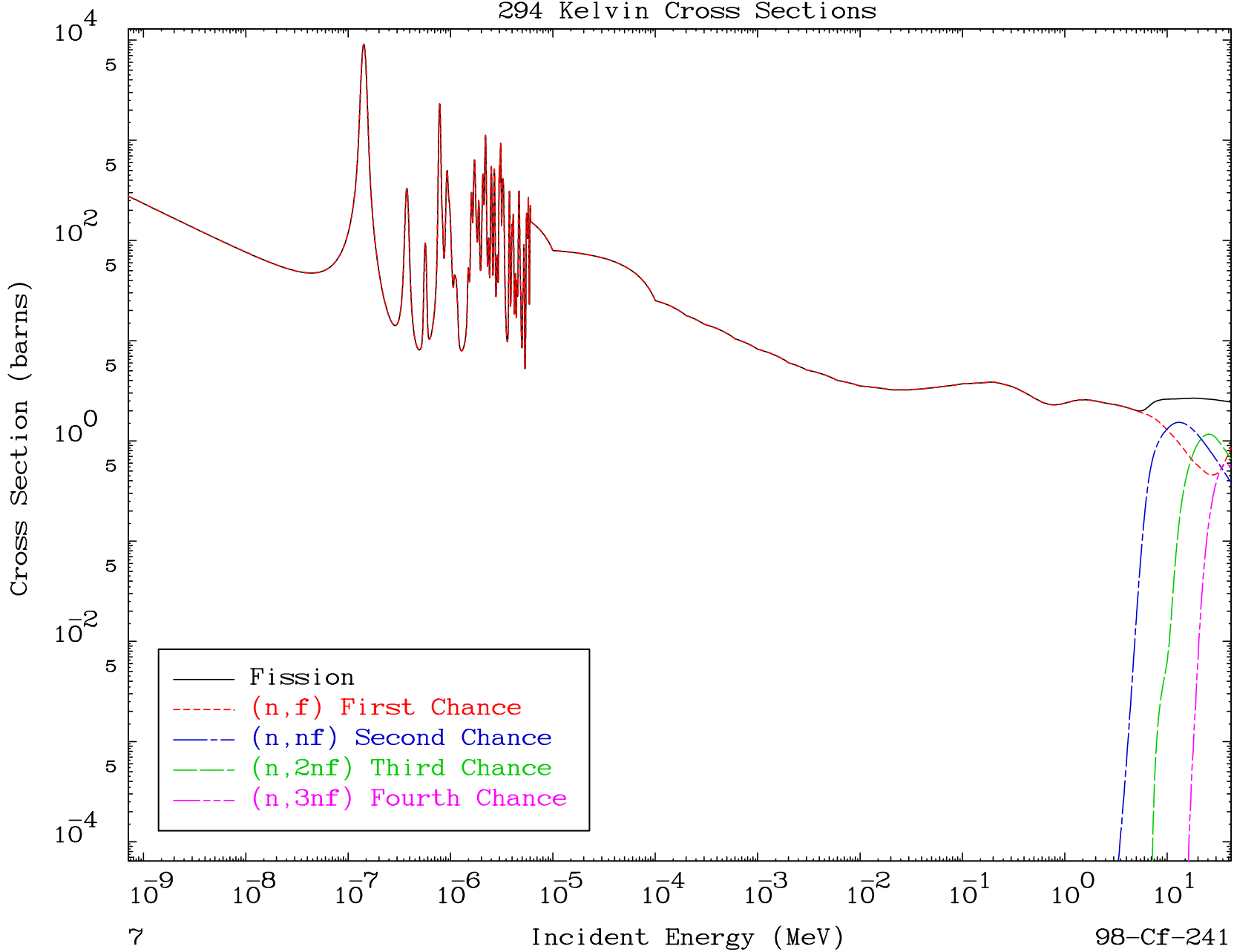
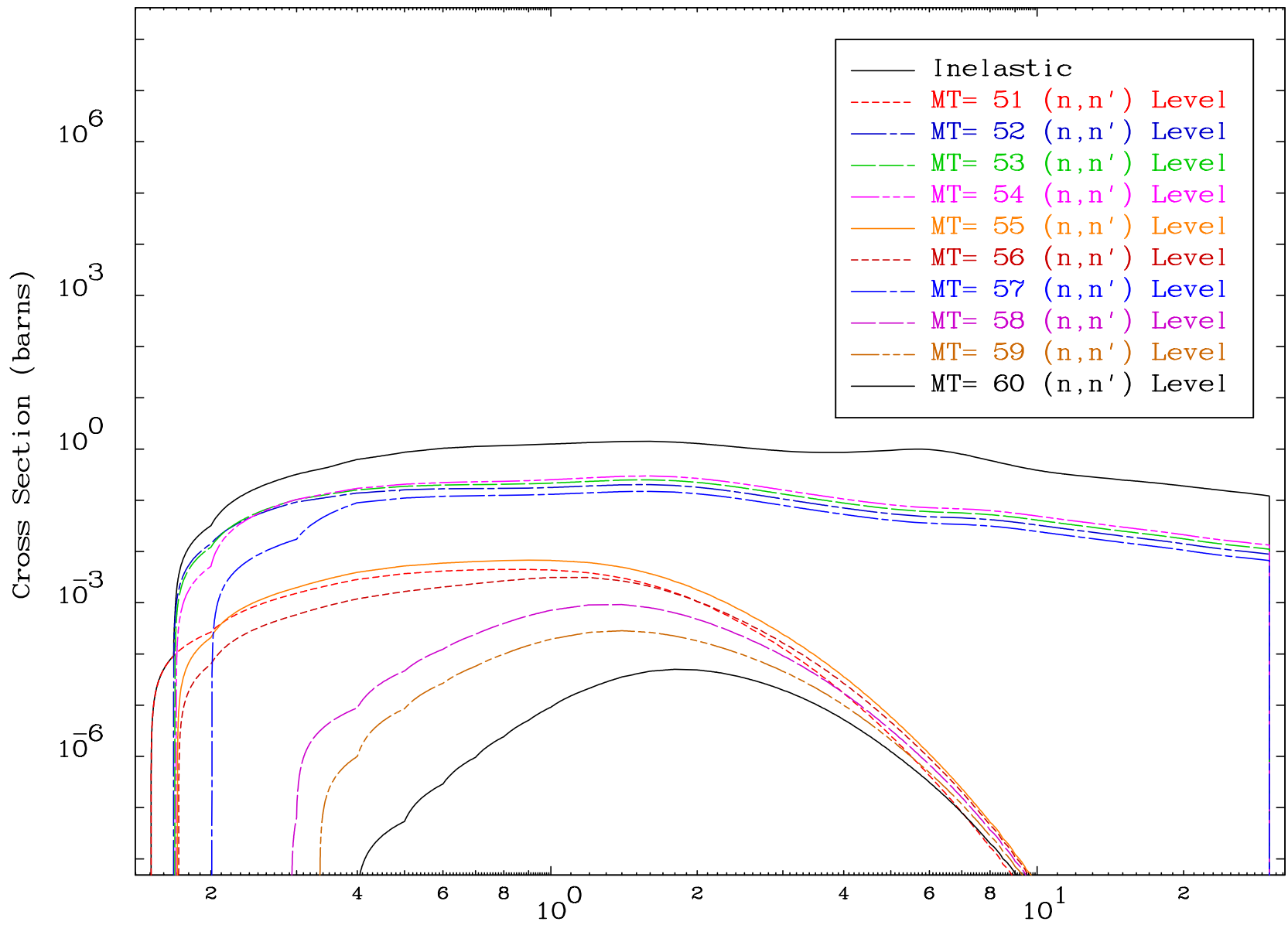


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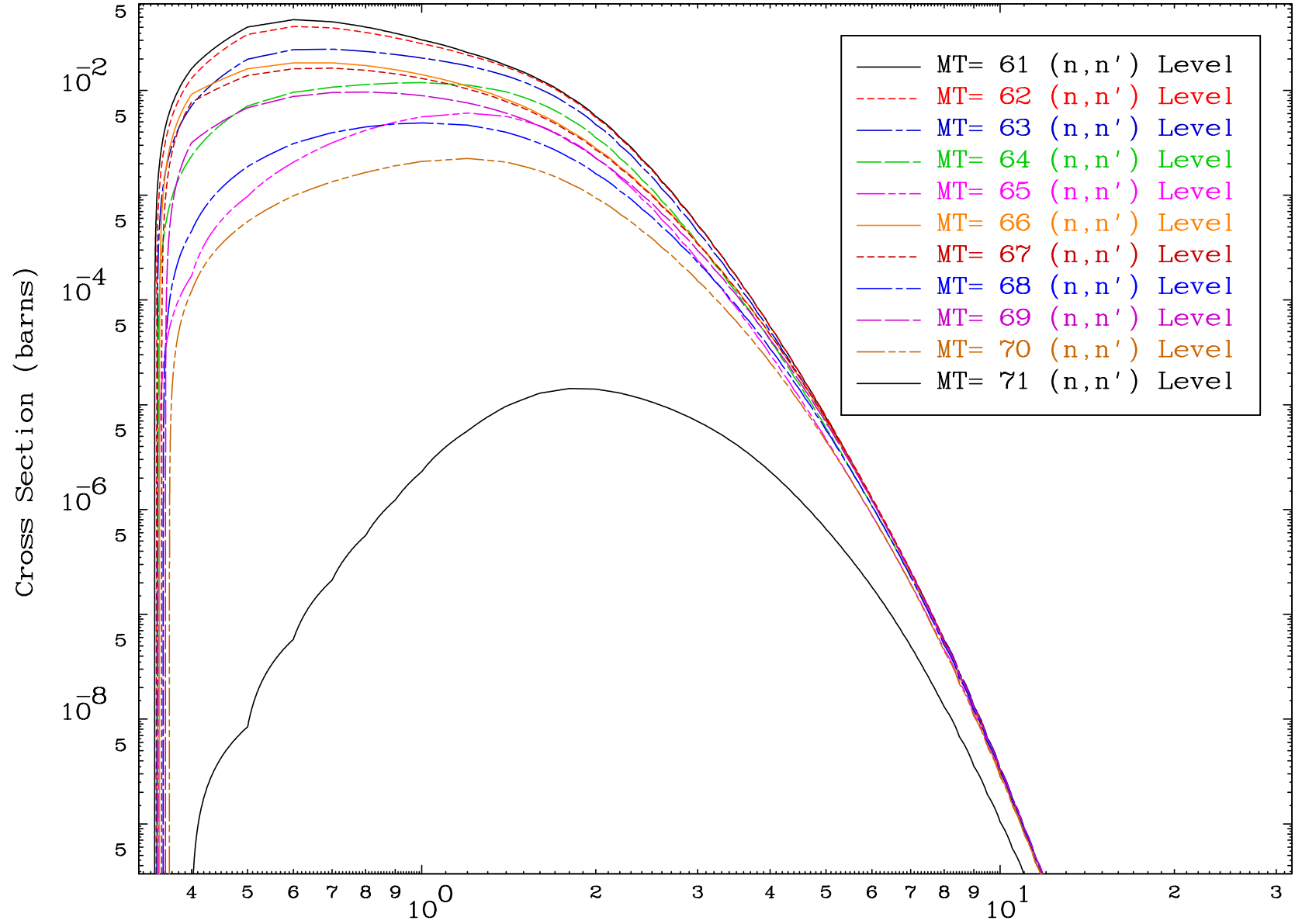
Fission  
294 Kelvin Cross Sections

98-Cf-241





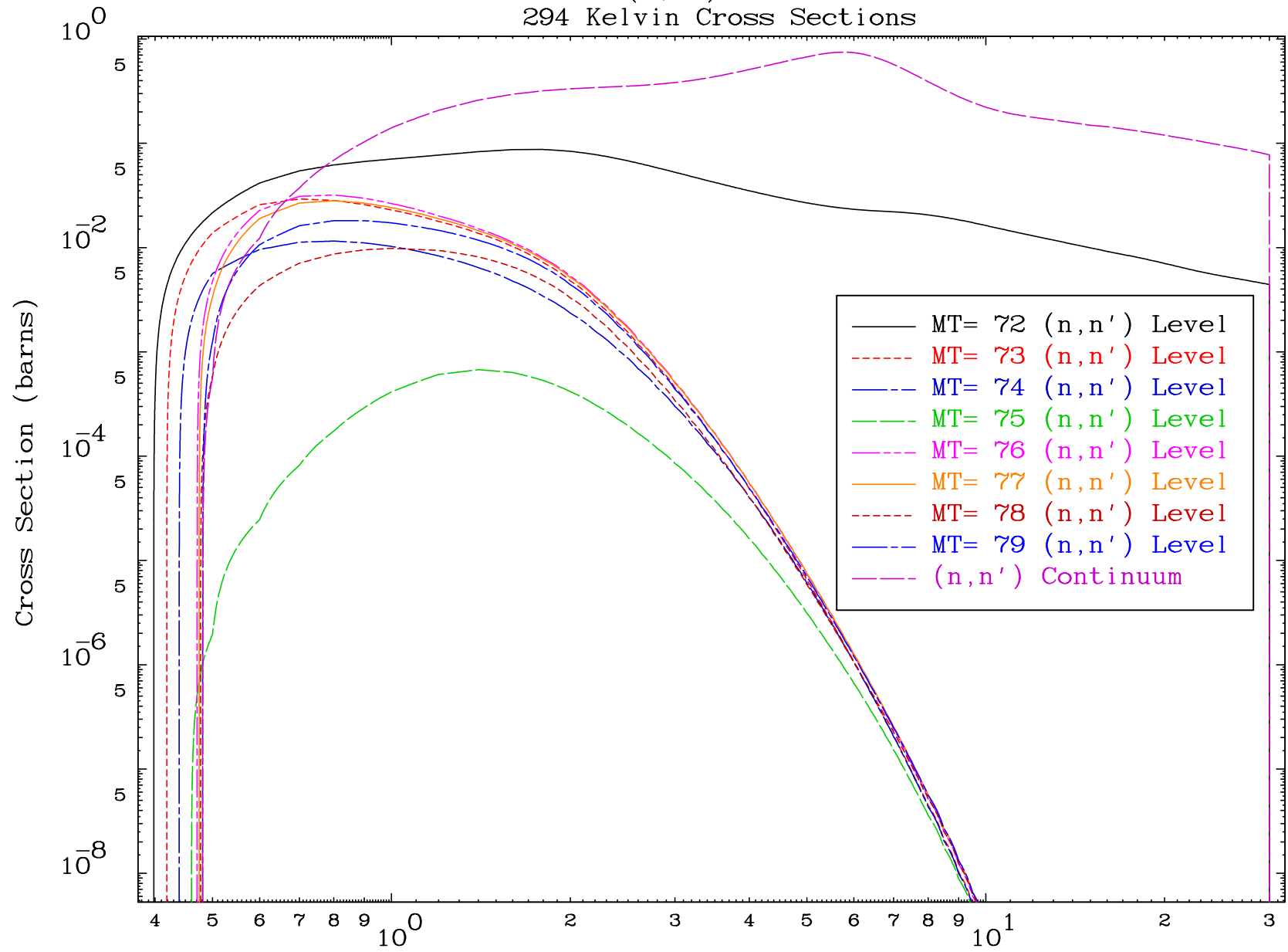




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(n,n') Level  
294 Kelvin Cross Sections

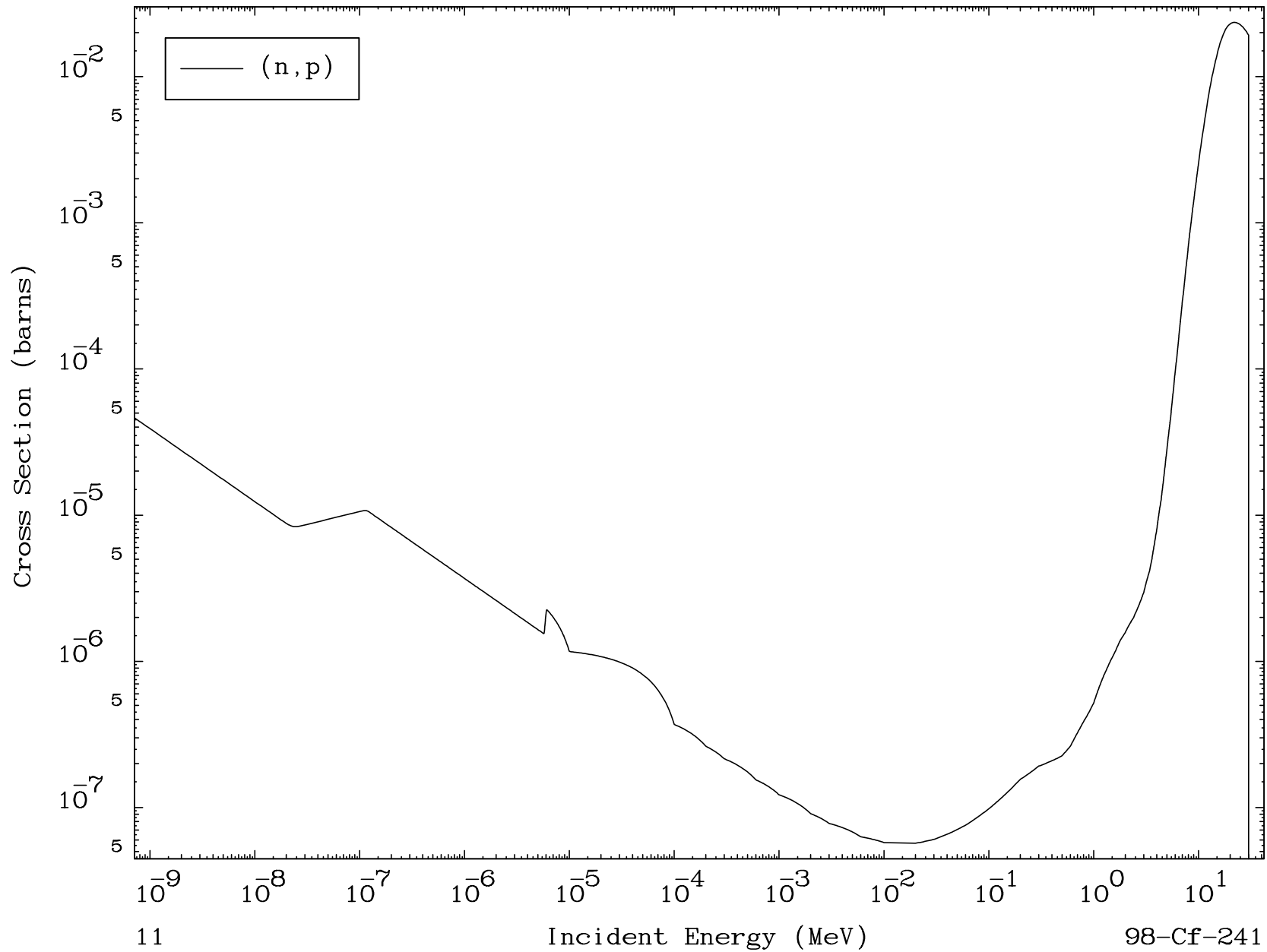
98-Cf-241

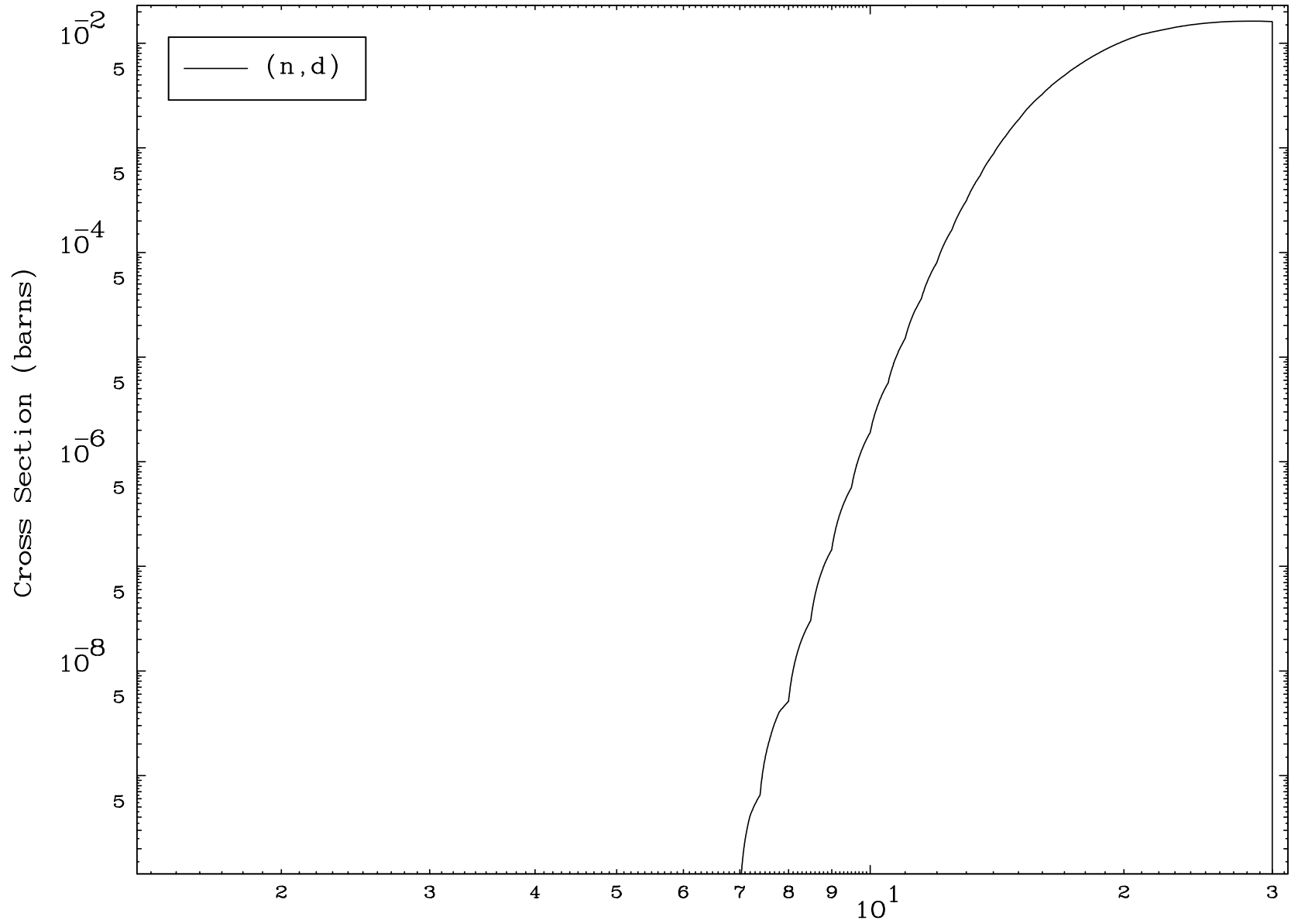


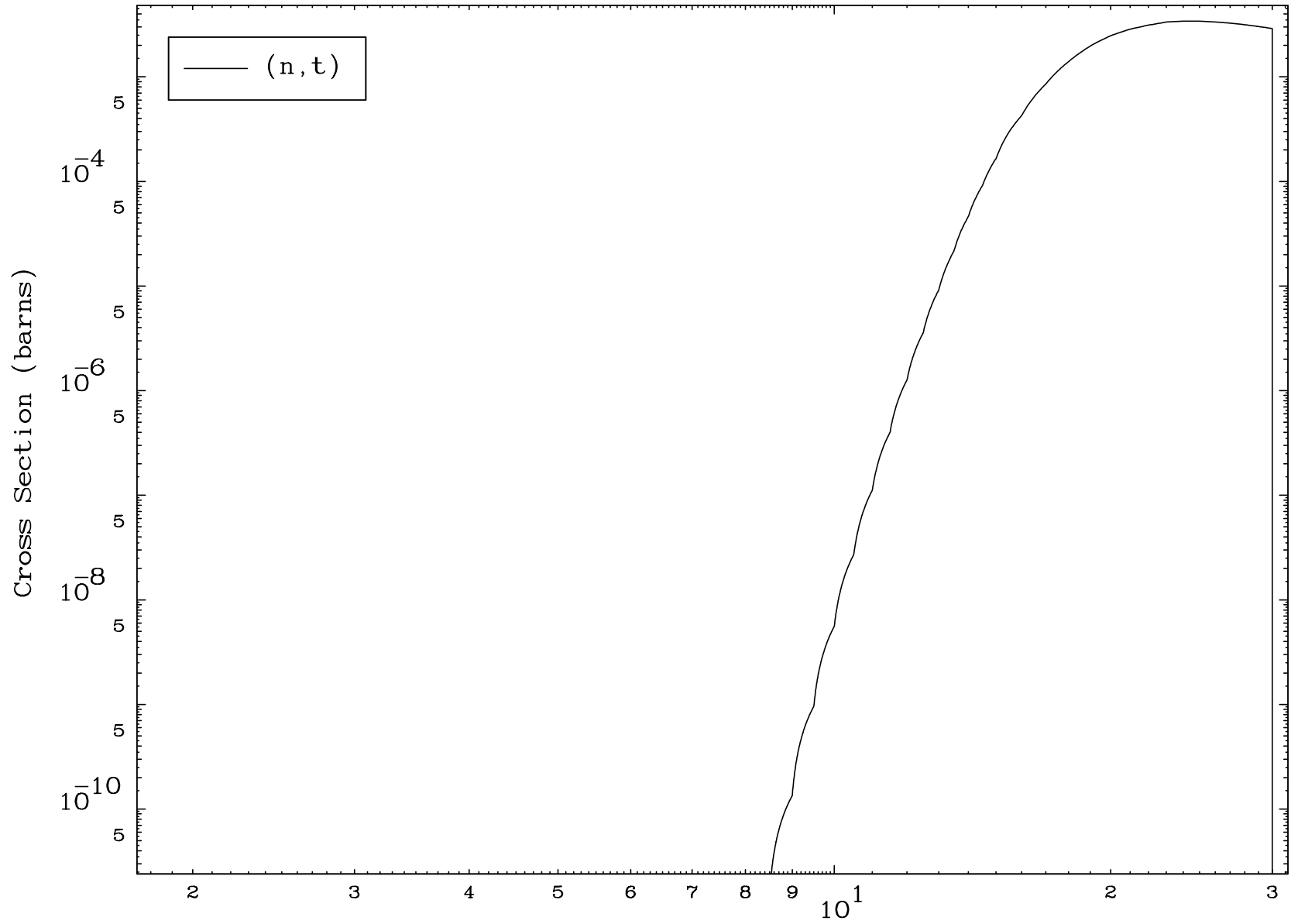
10

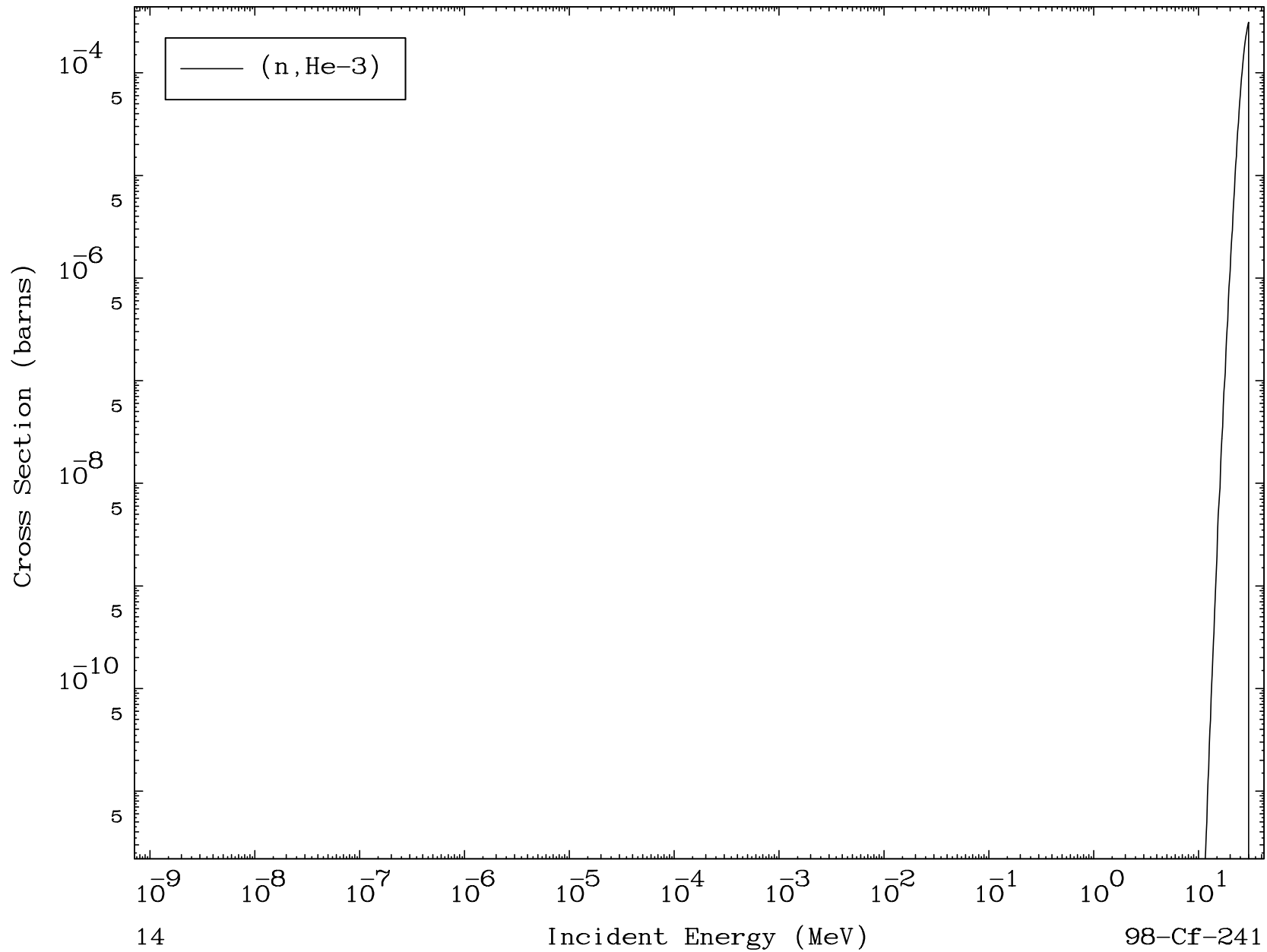
Incident Energy (MeV)

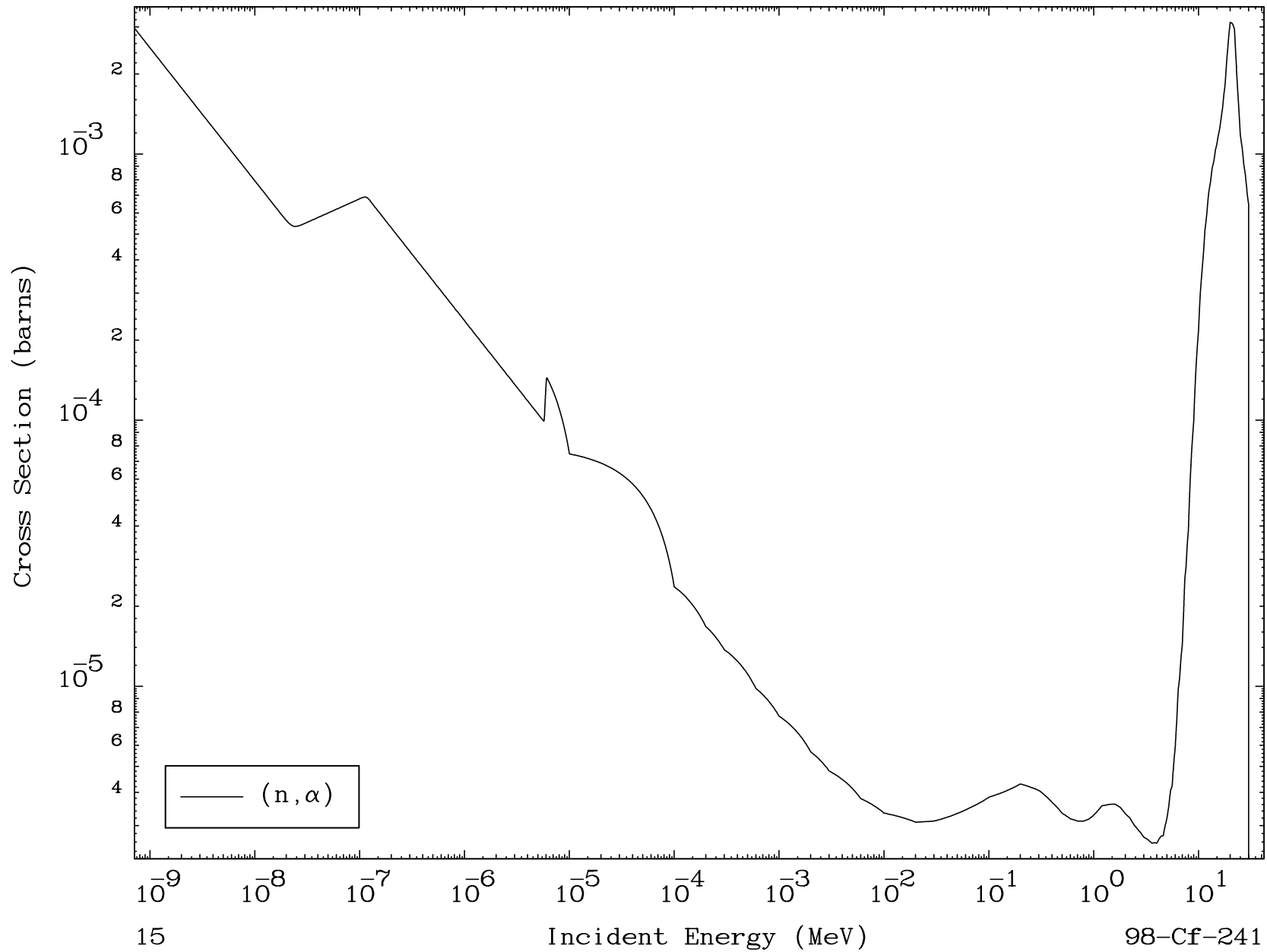
98-Cf-241







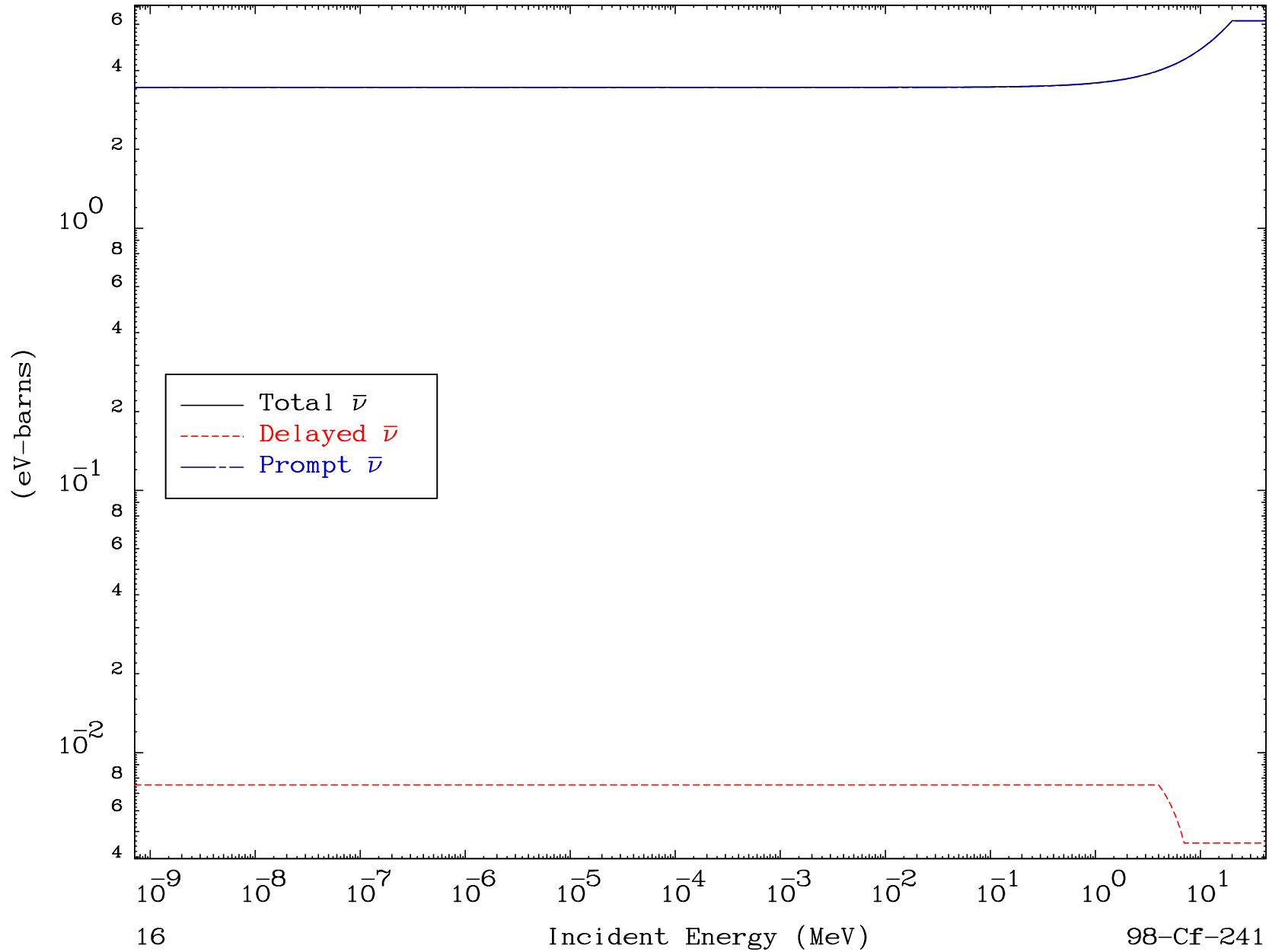




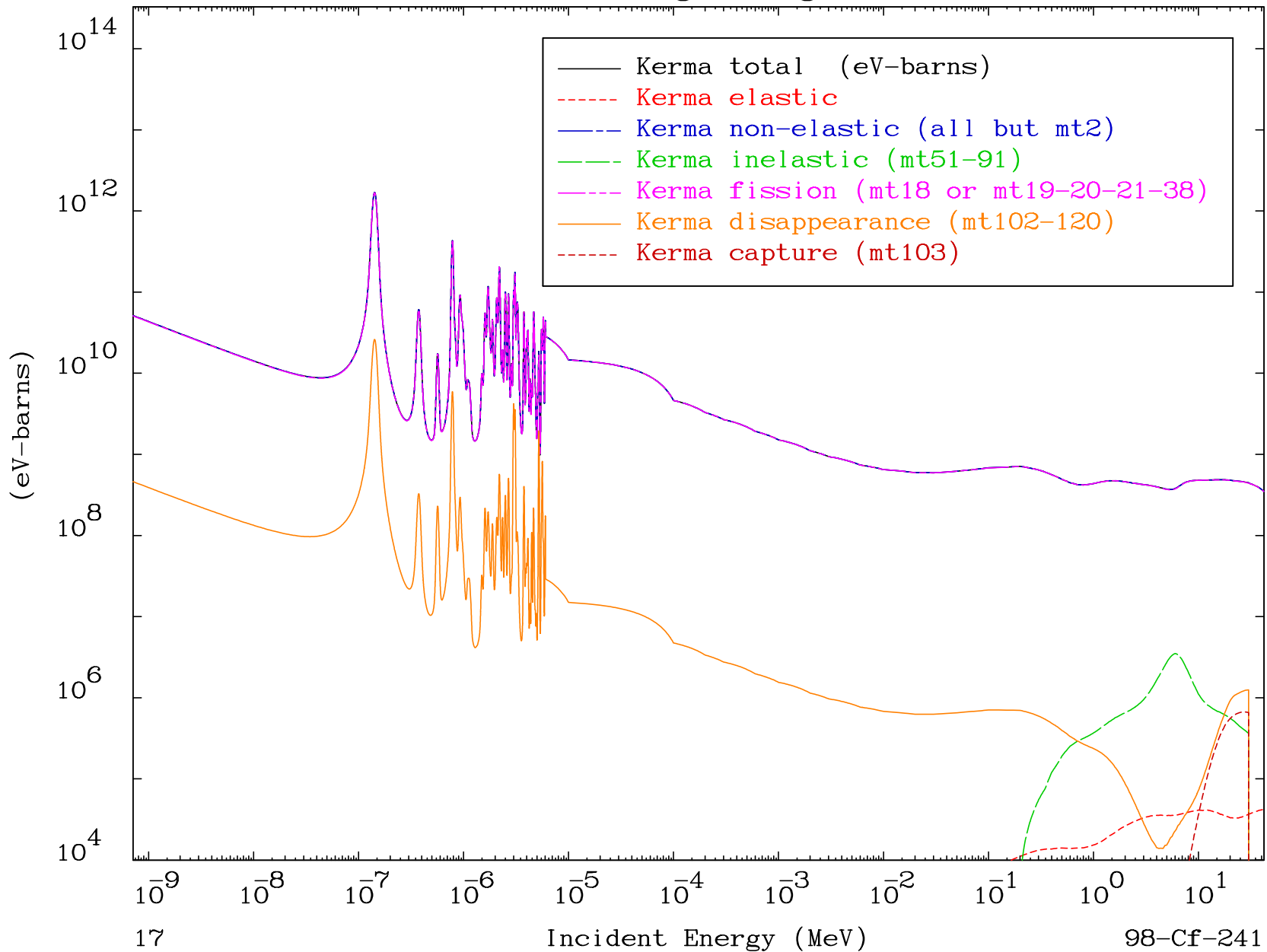
MAT 9828

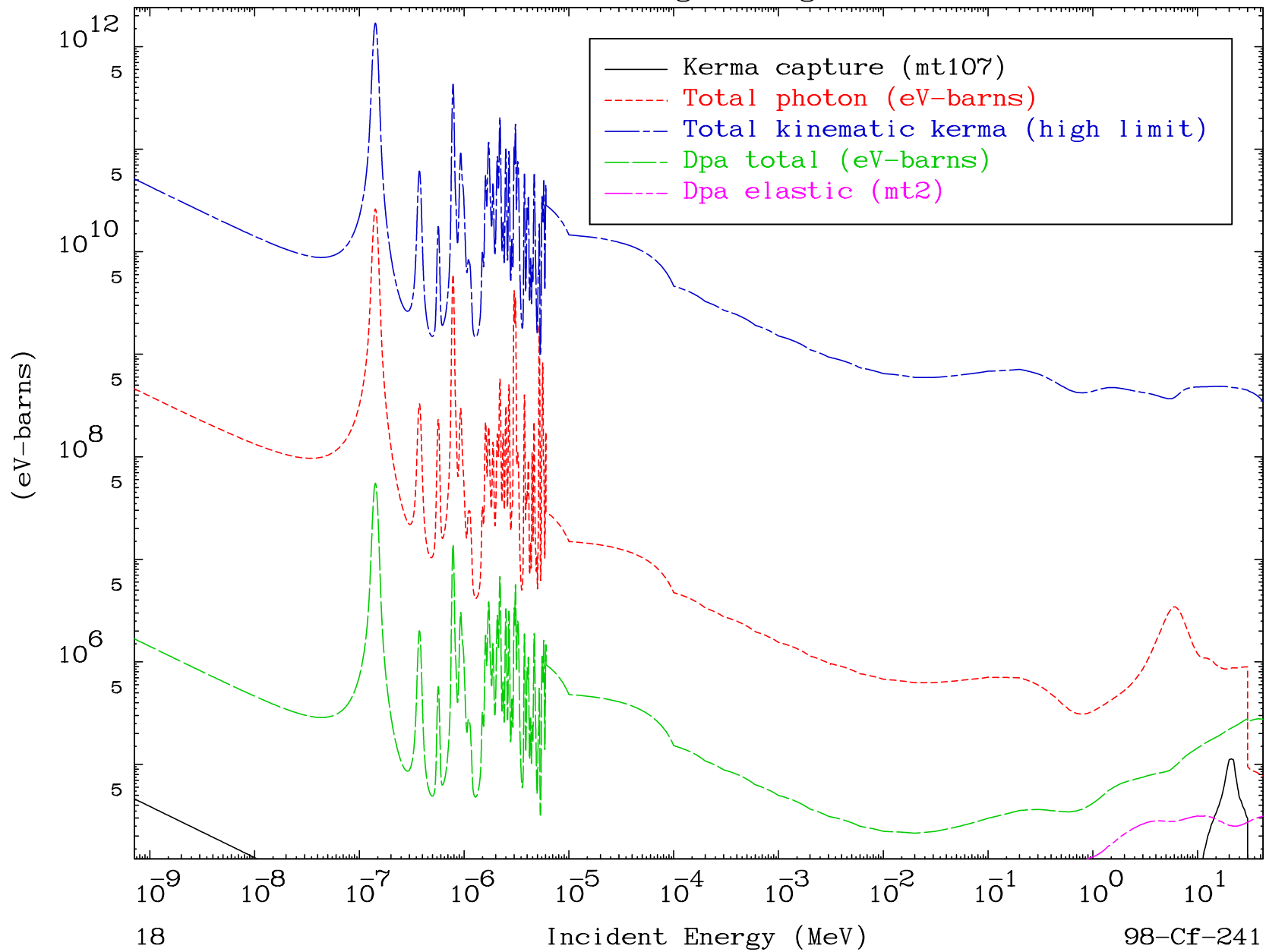
Energy Release  
Heating - Damage

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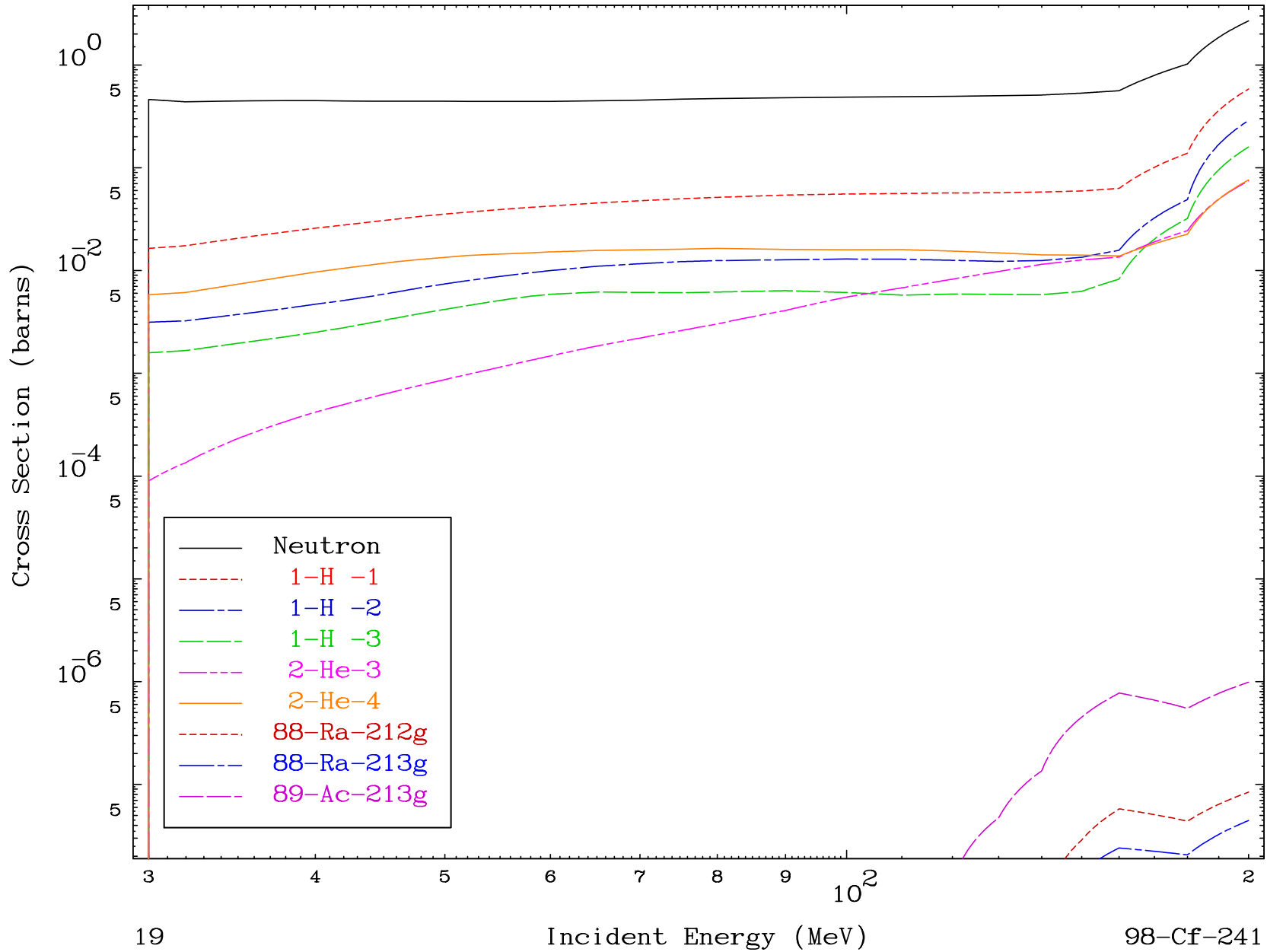


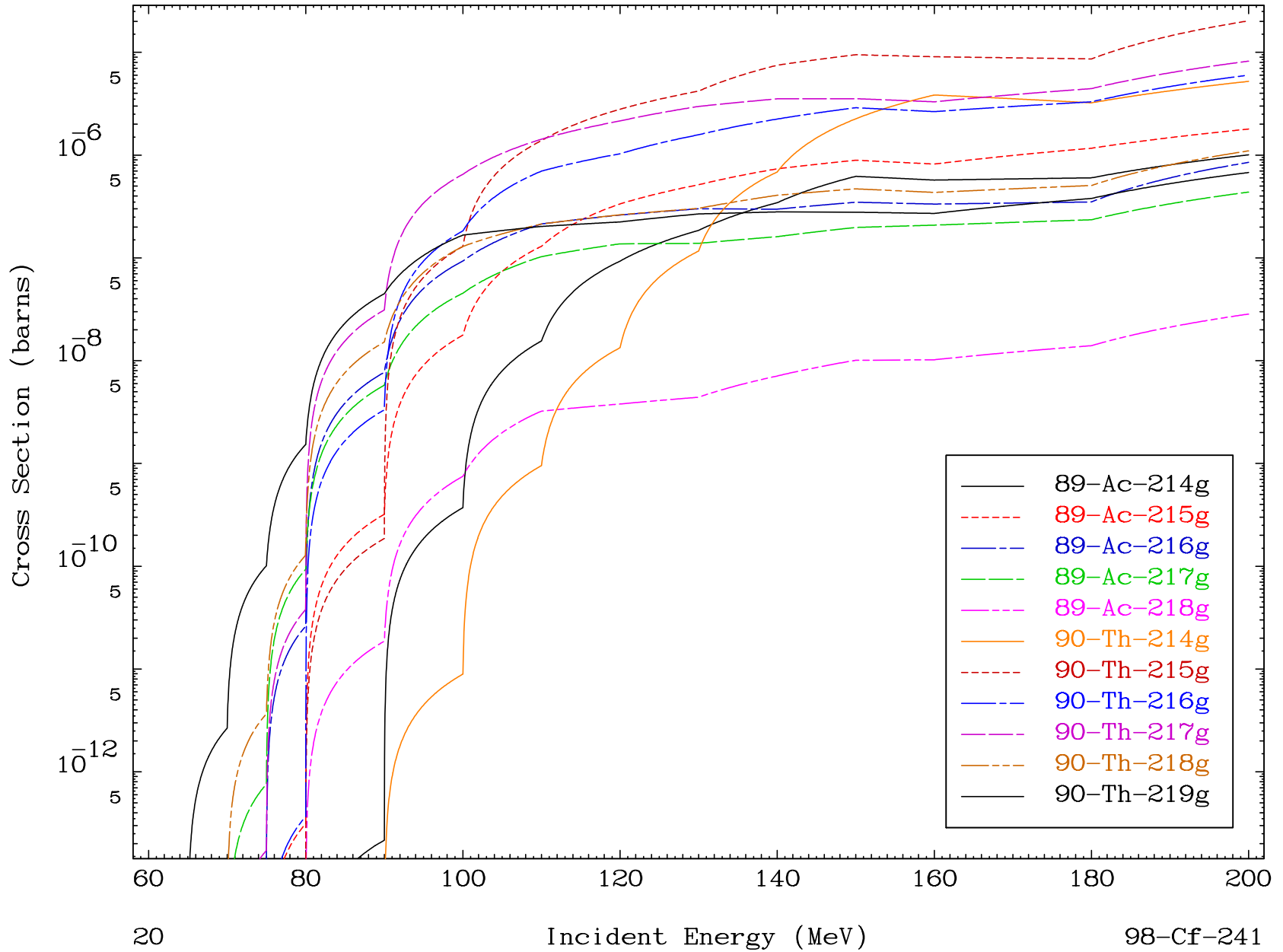




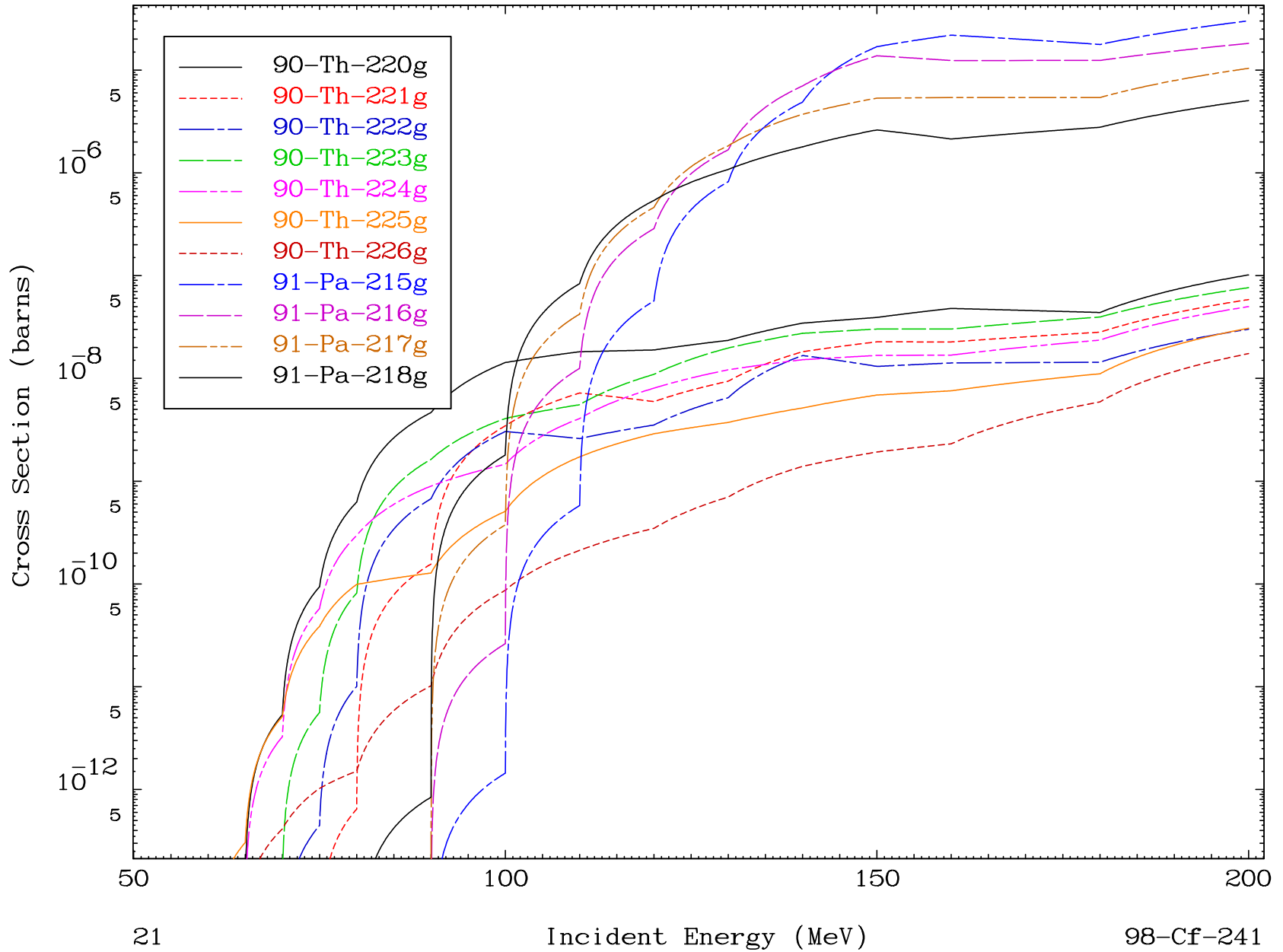


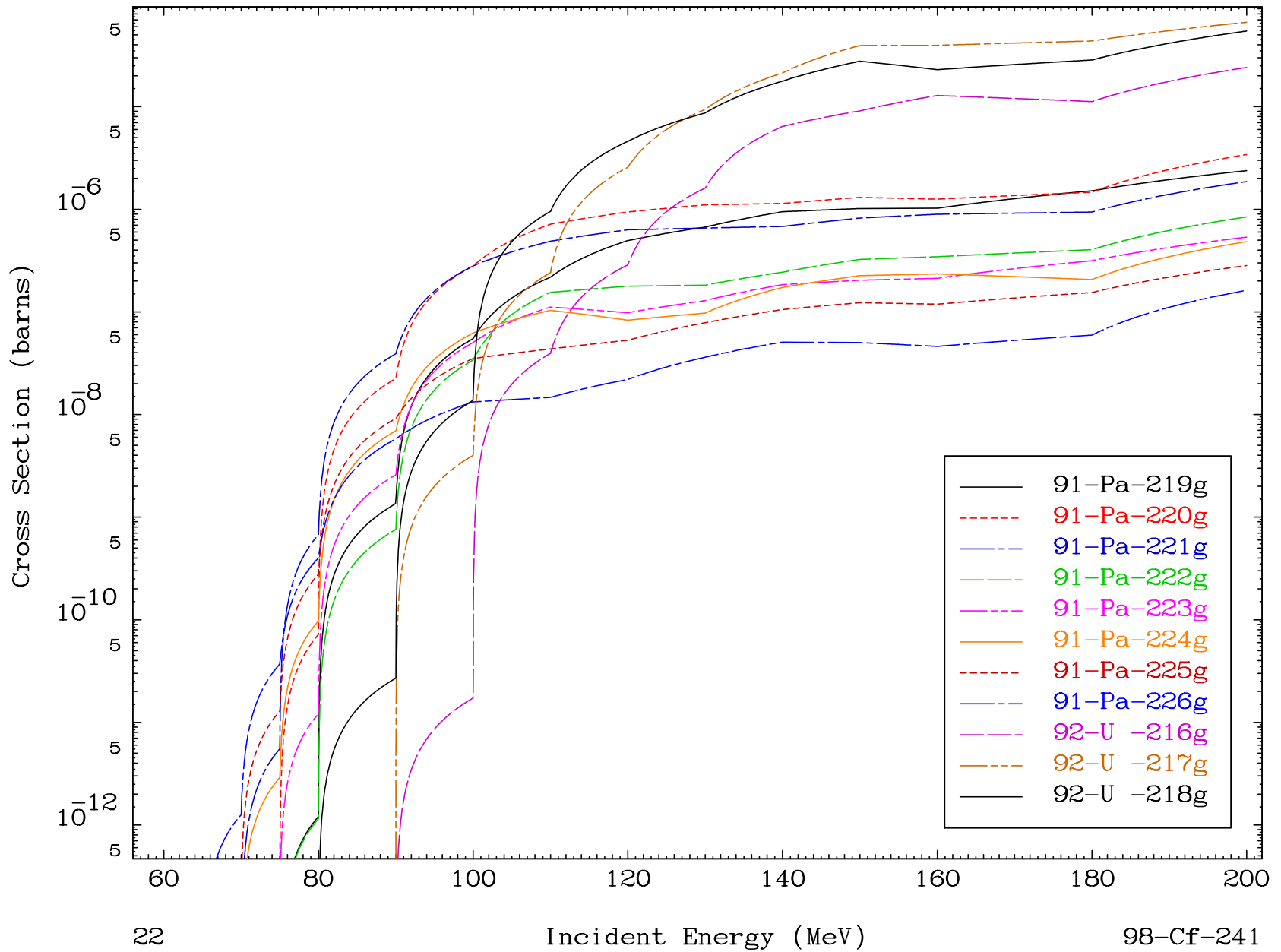
Radionuclide Production Cross Section

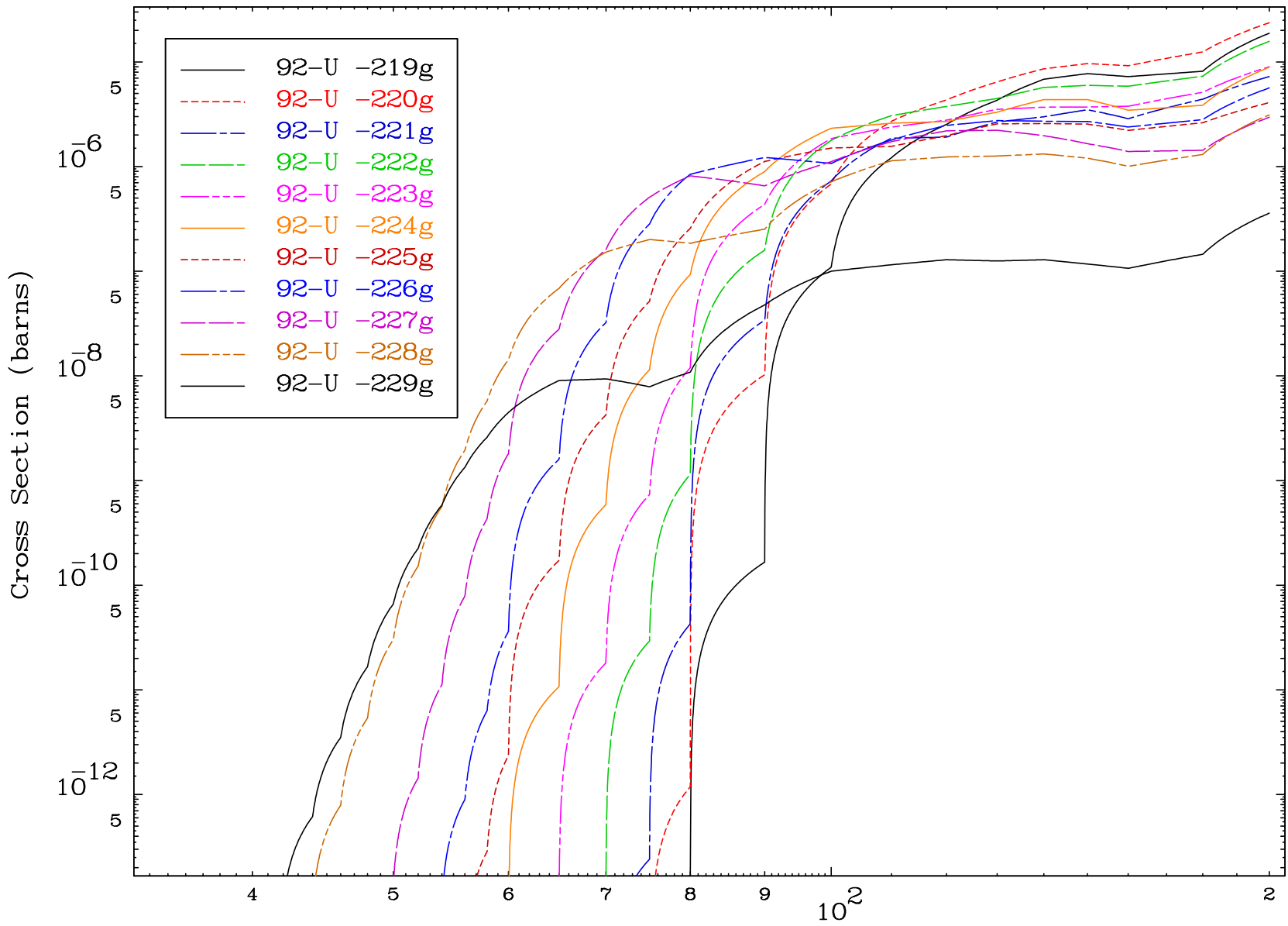


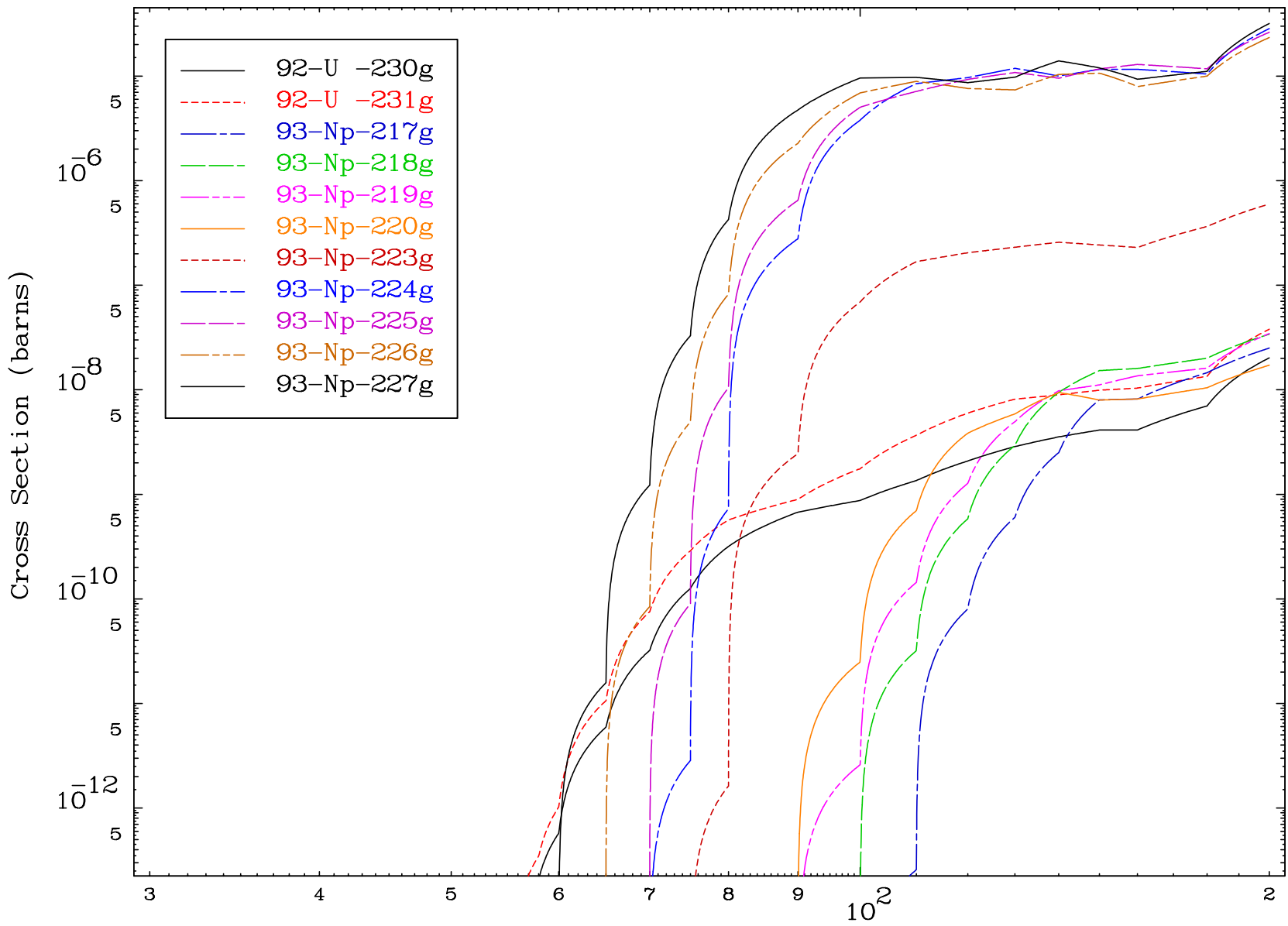


Radionuclide Production Cross Section









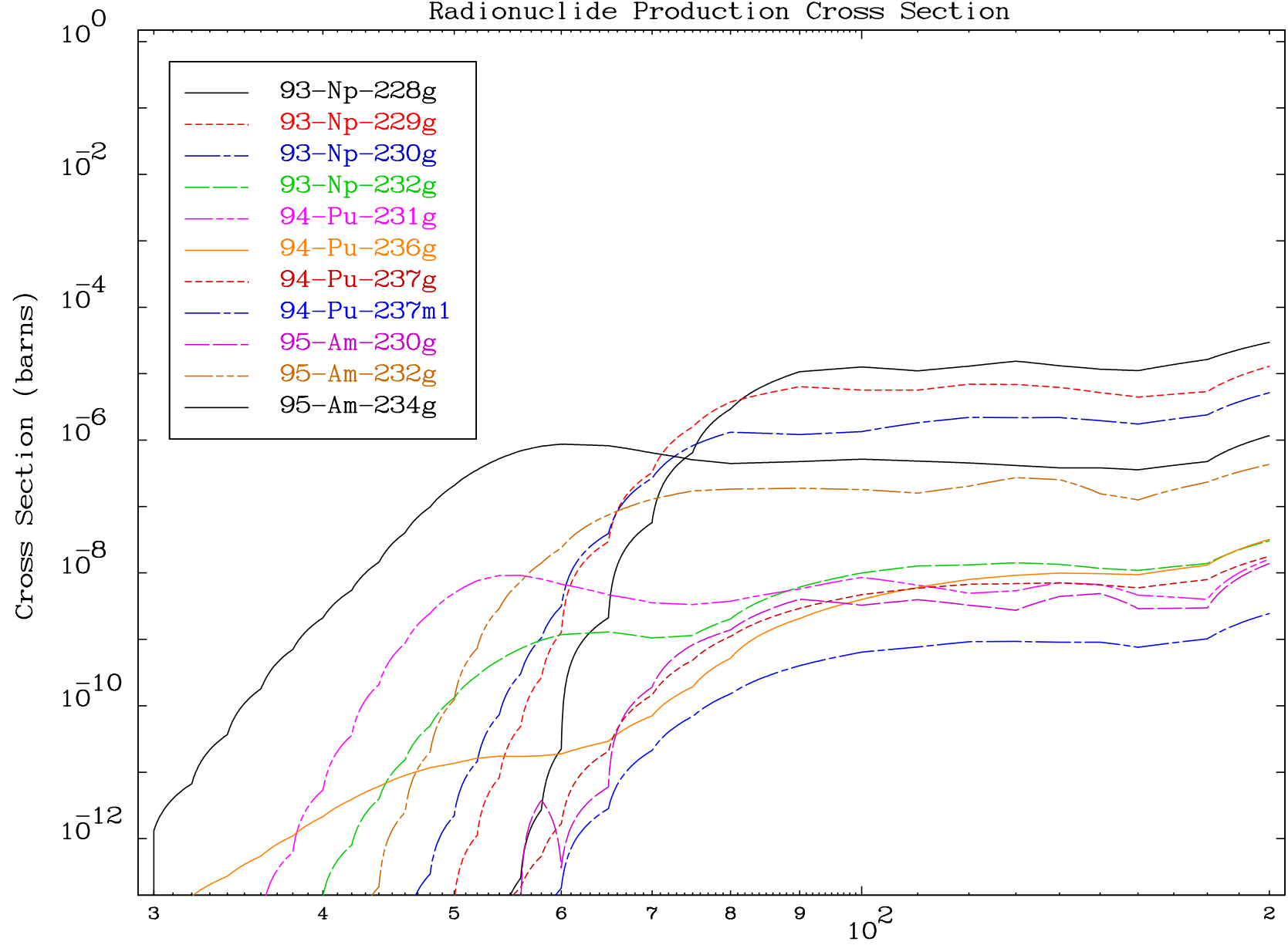


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(n,remainder)

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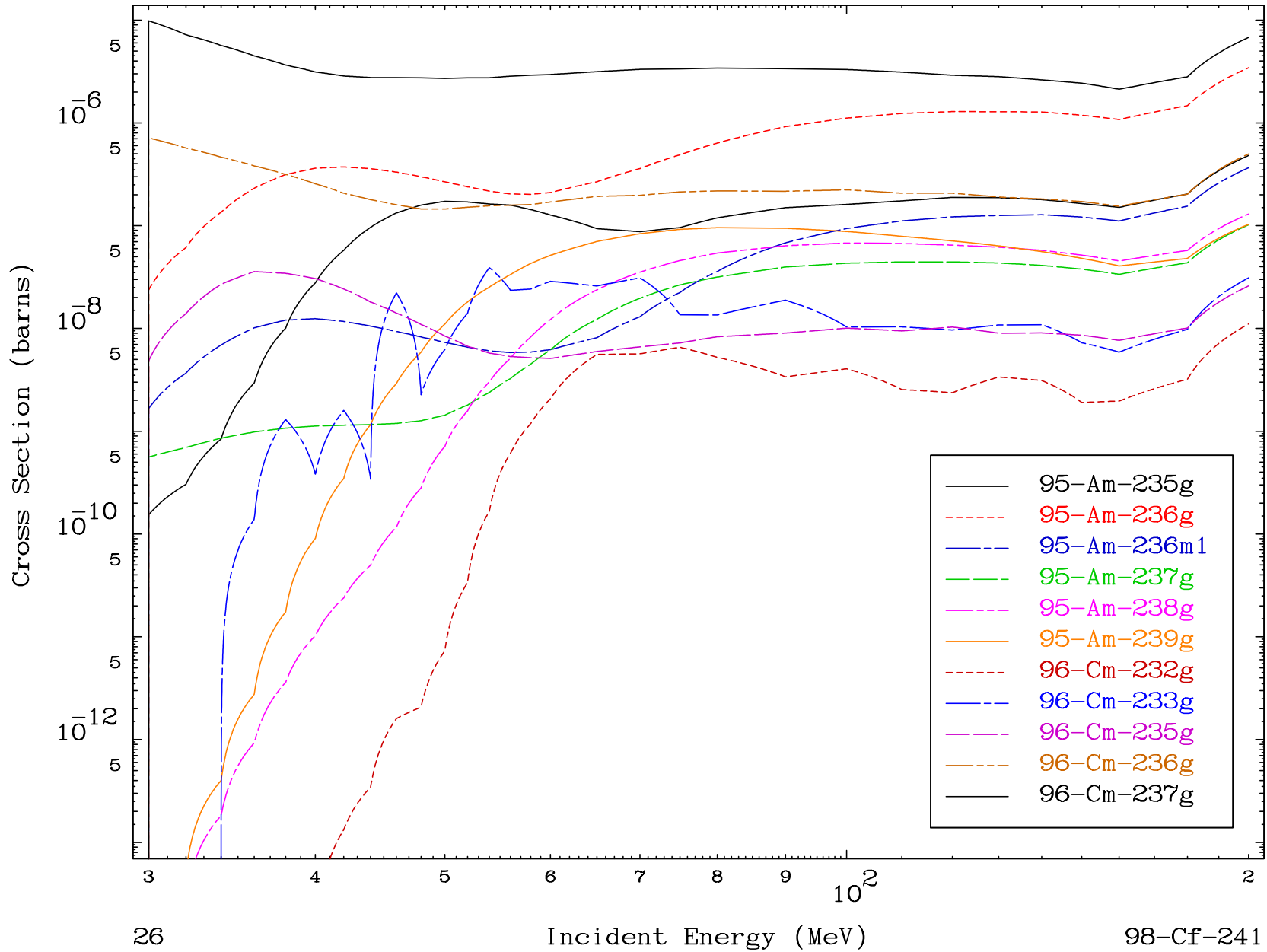
### Radionuclide Production Cross Section



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Incident Energy (MeV)

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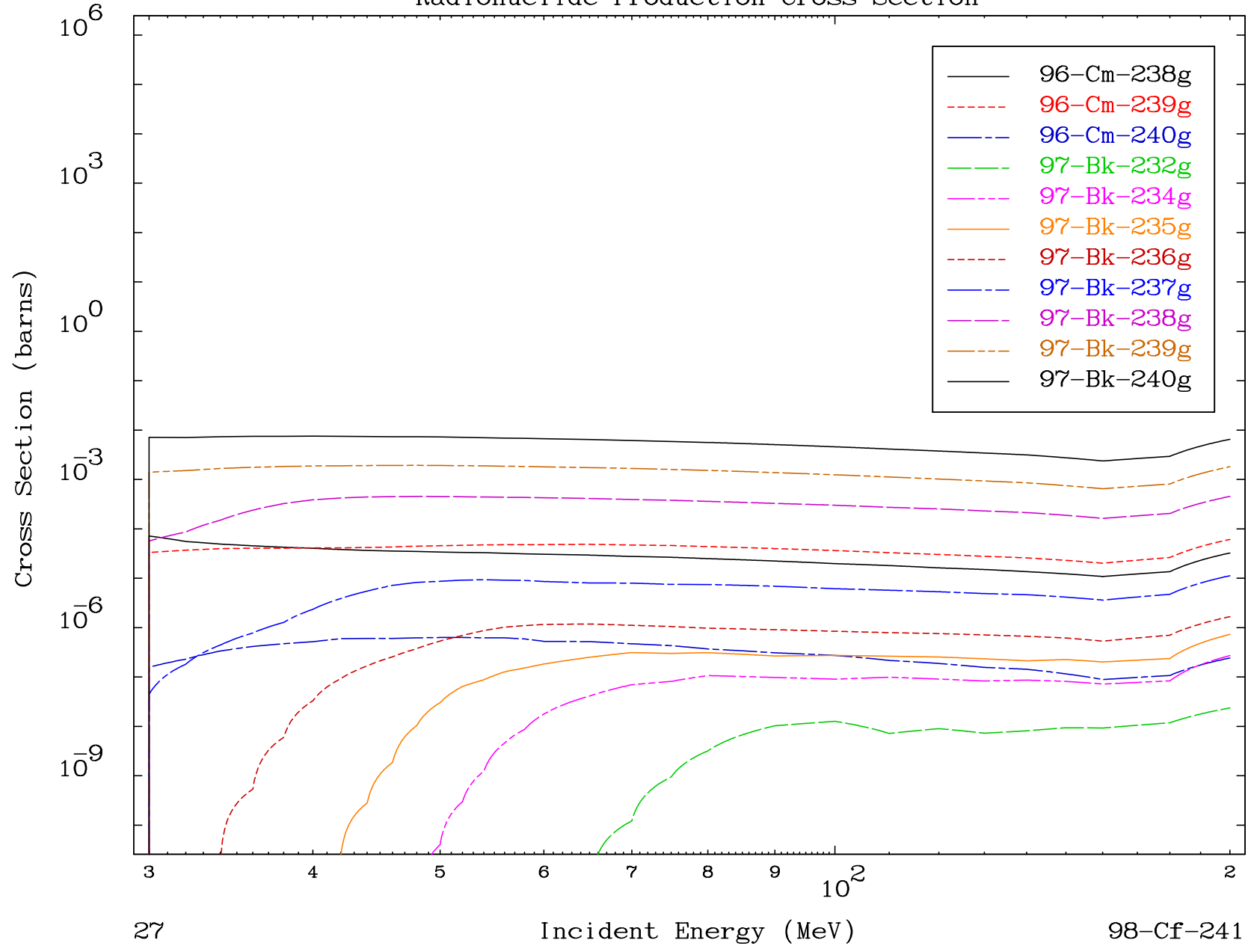


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(n,remainder)

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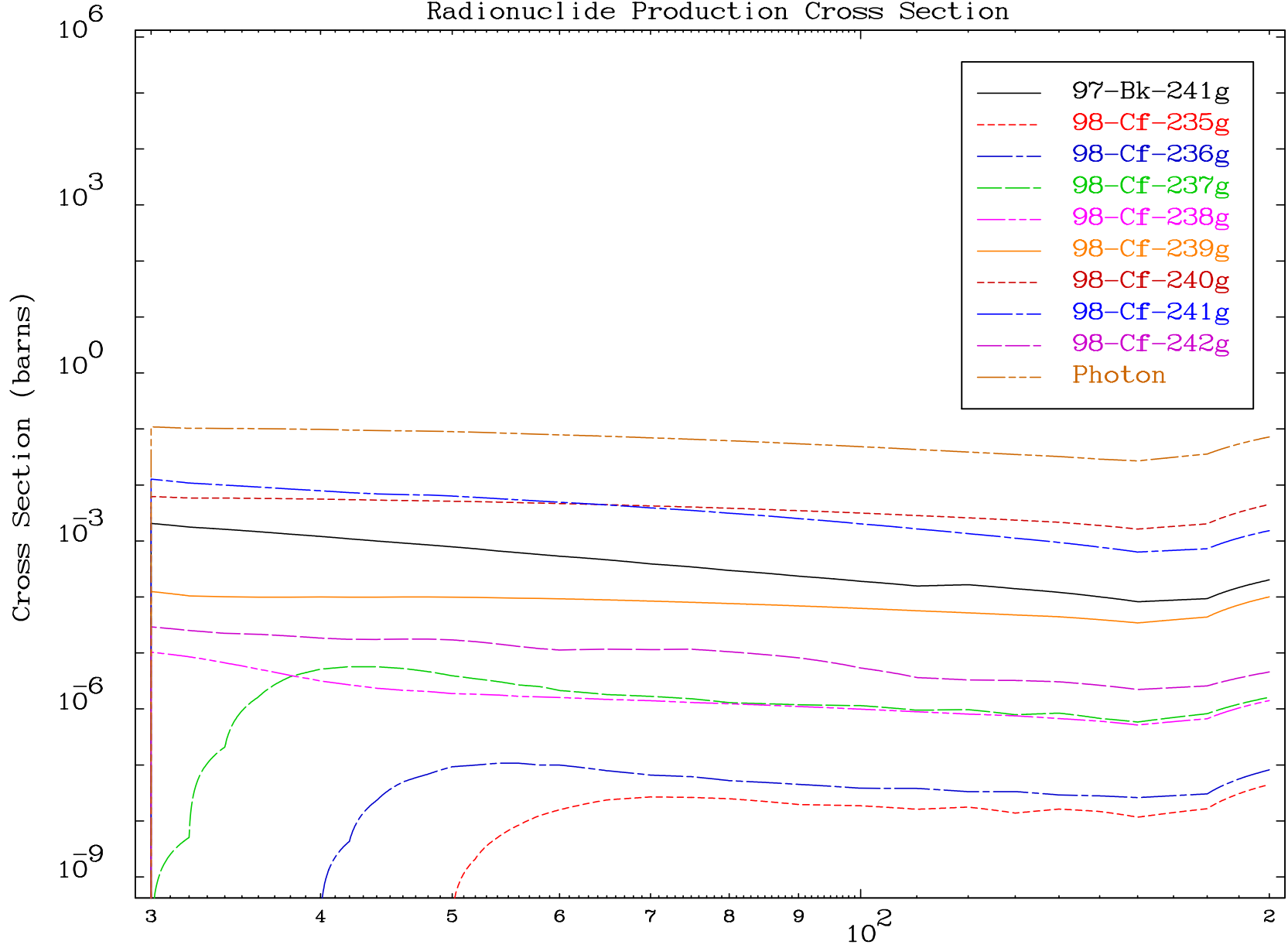
Radionuclide Production Cross Section



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(n,remainder)  
Radionuclide Production Cross Section

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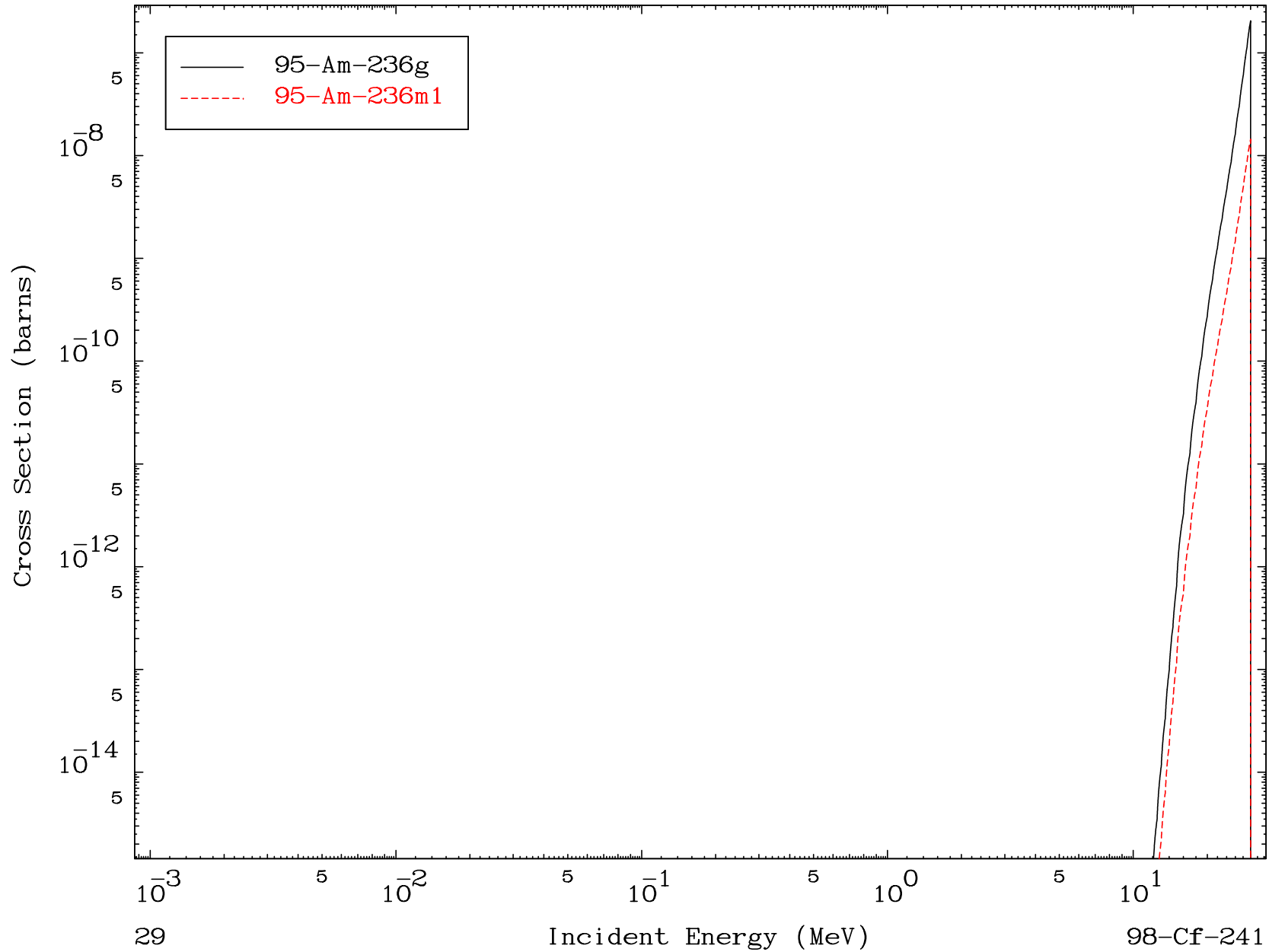
Incident Energy (MeV)

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(n,n') p α  
Radionuclide Production Cross Section

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MAT 9828

(n,d)  $\alpha$

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Radionuclide Production Cross Section

