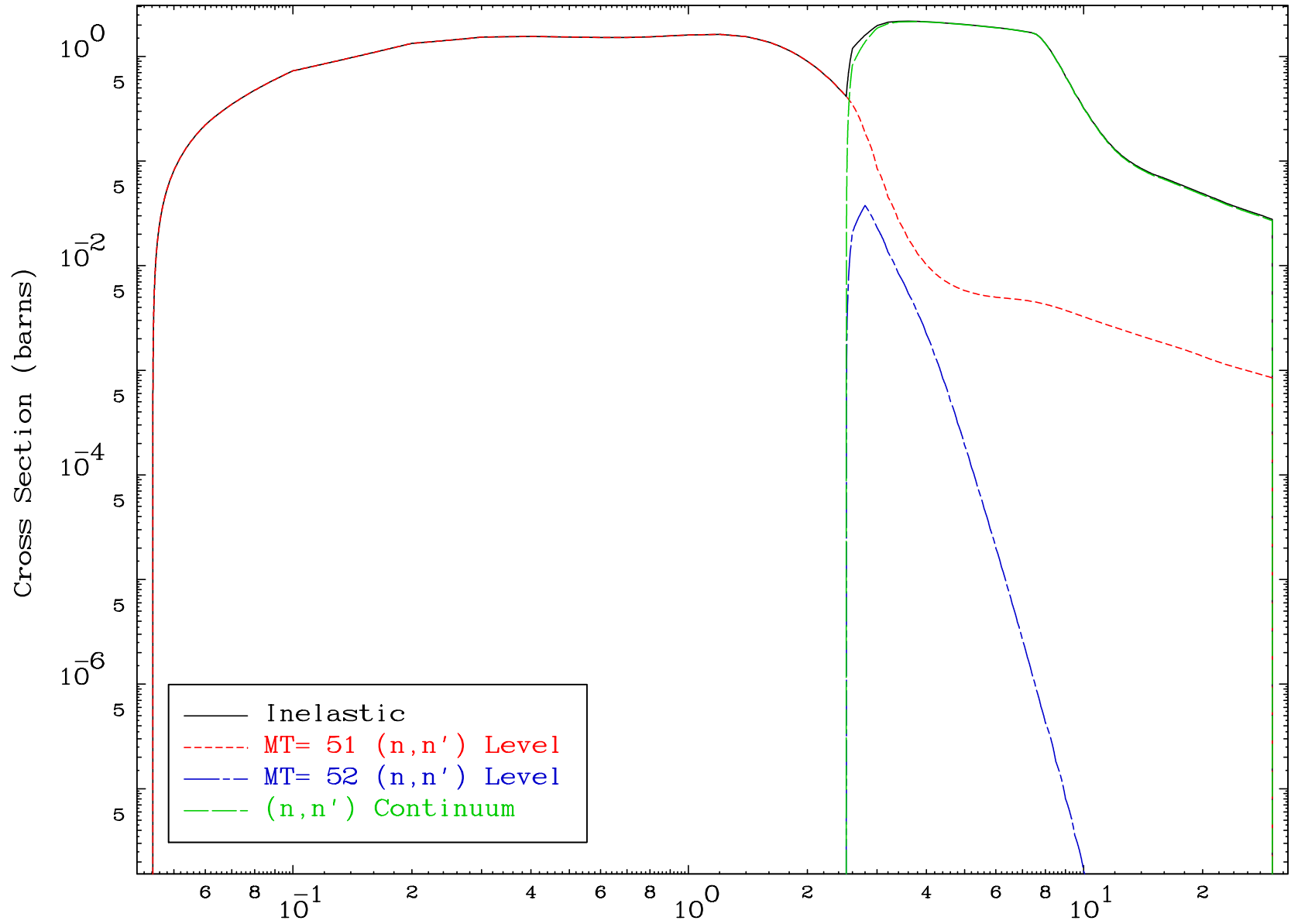


MAT 9843

(n,n') Level
294 Kelvin Cross Sections

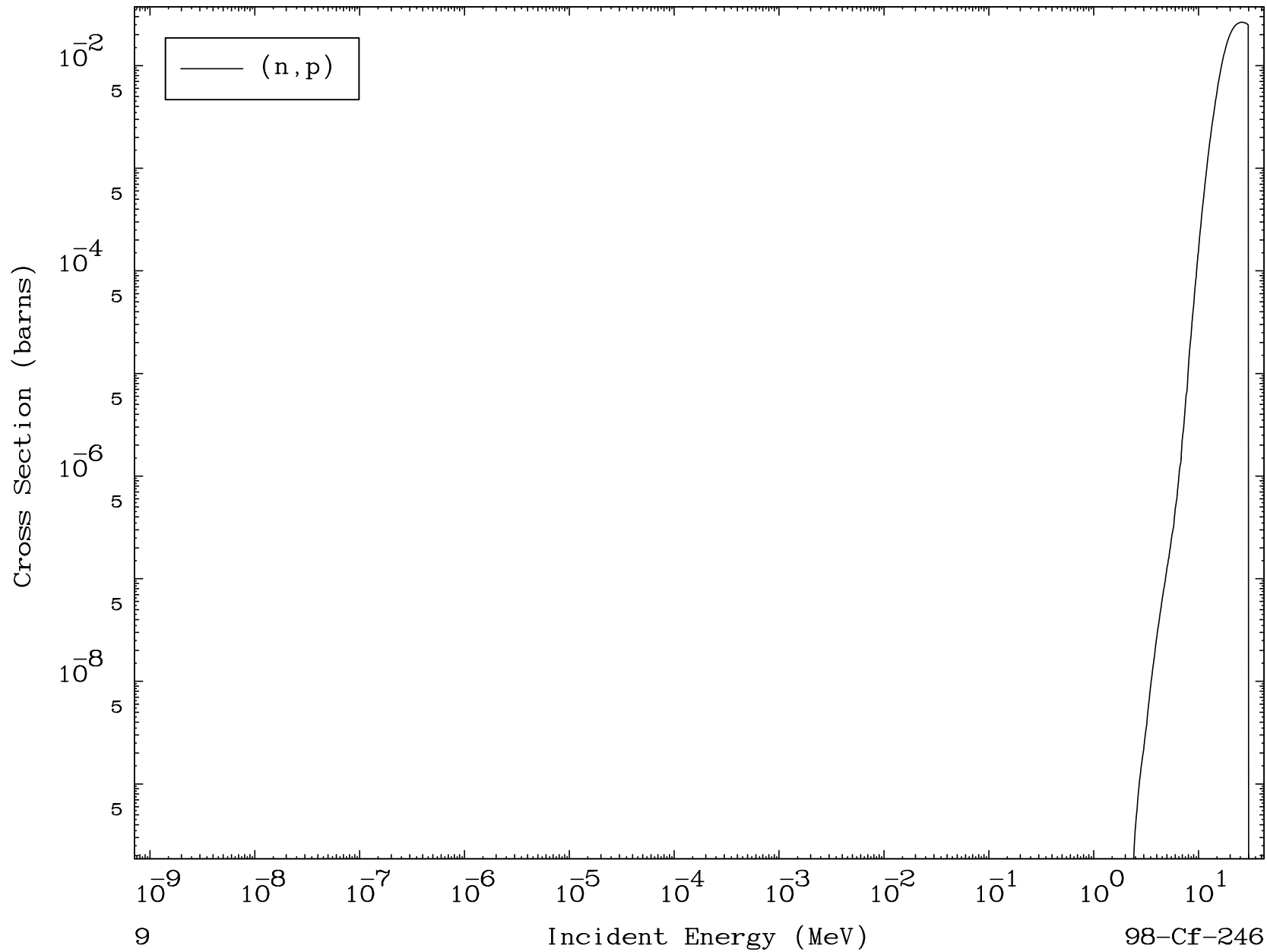
98-Cf-246

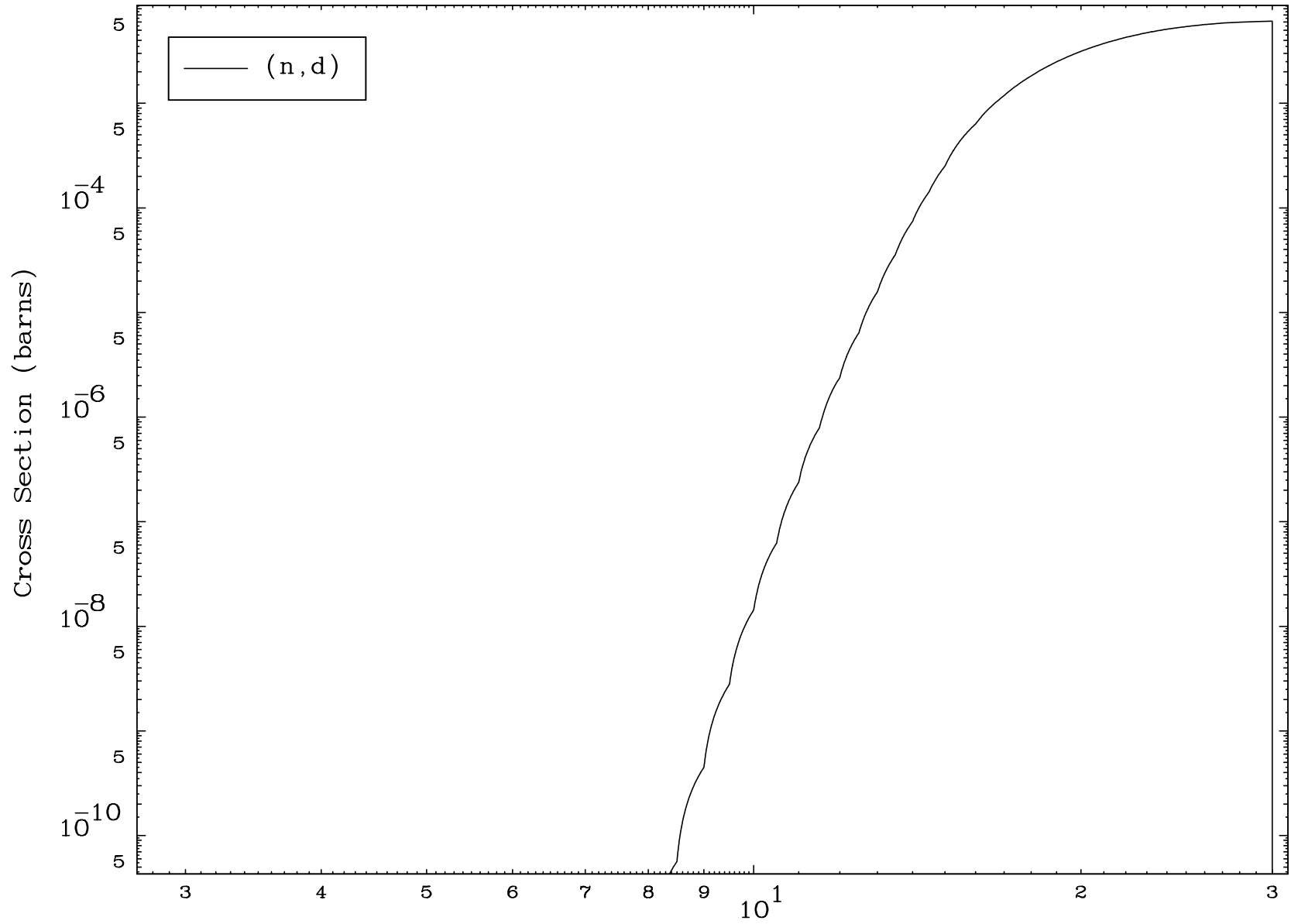


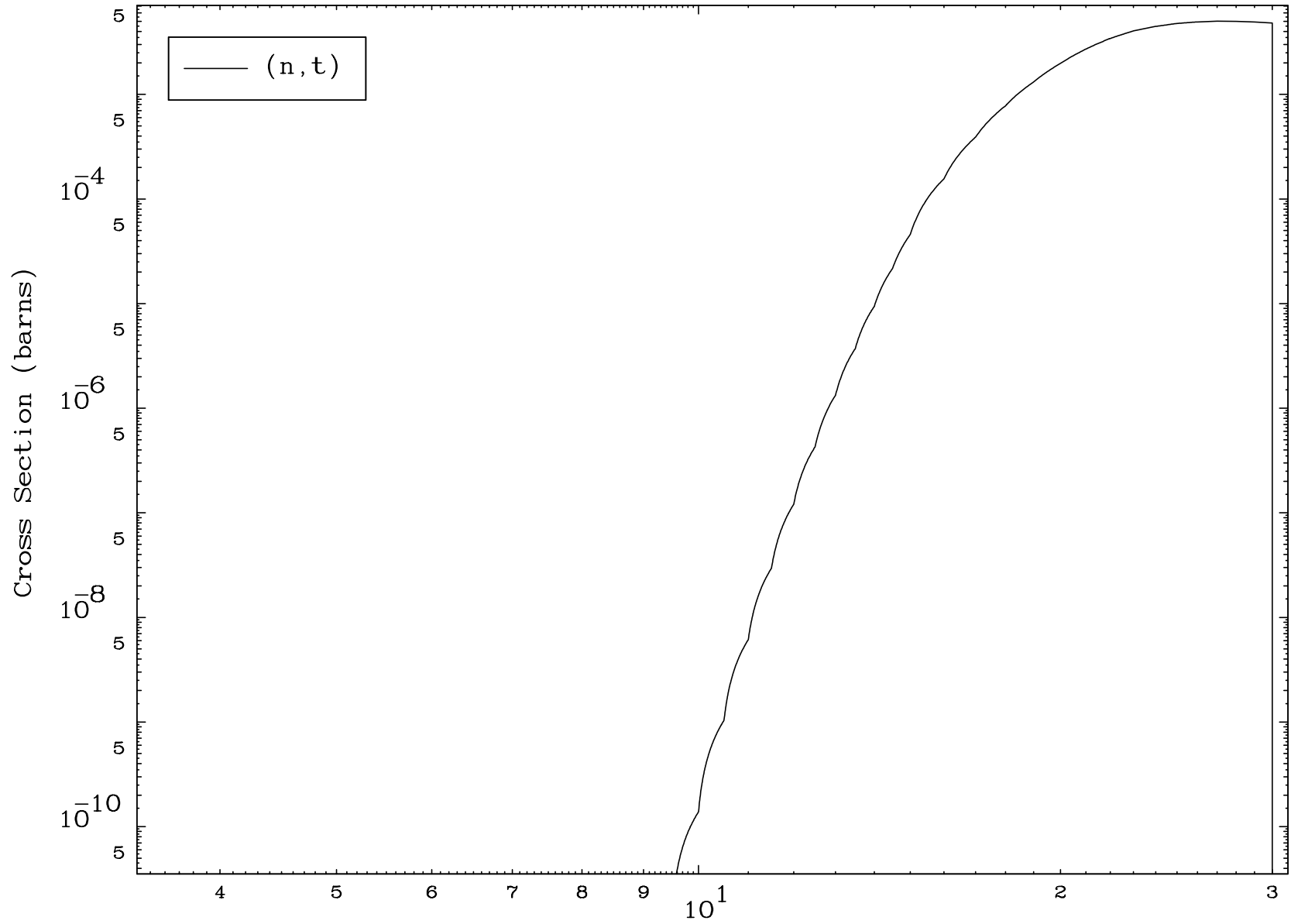
8

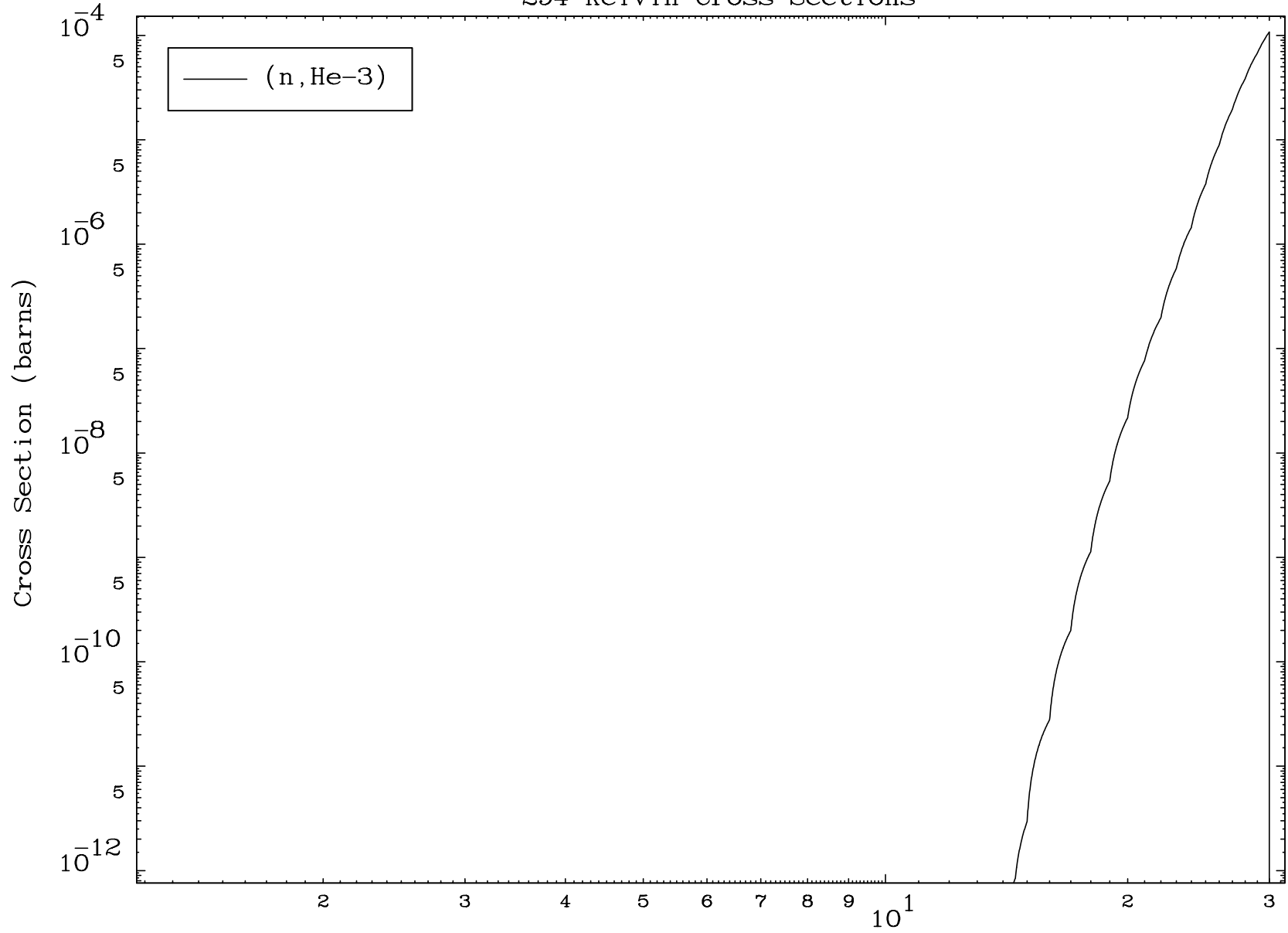
Incident Energy (MeV)

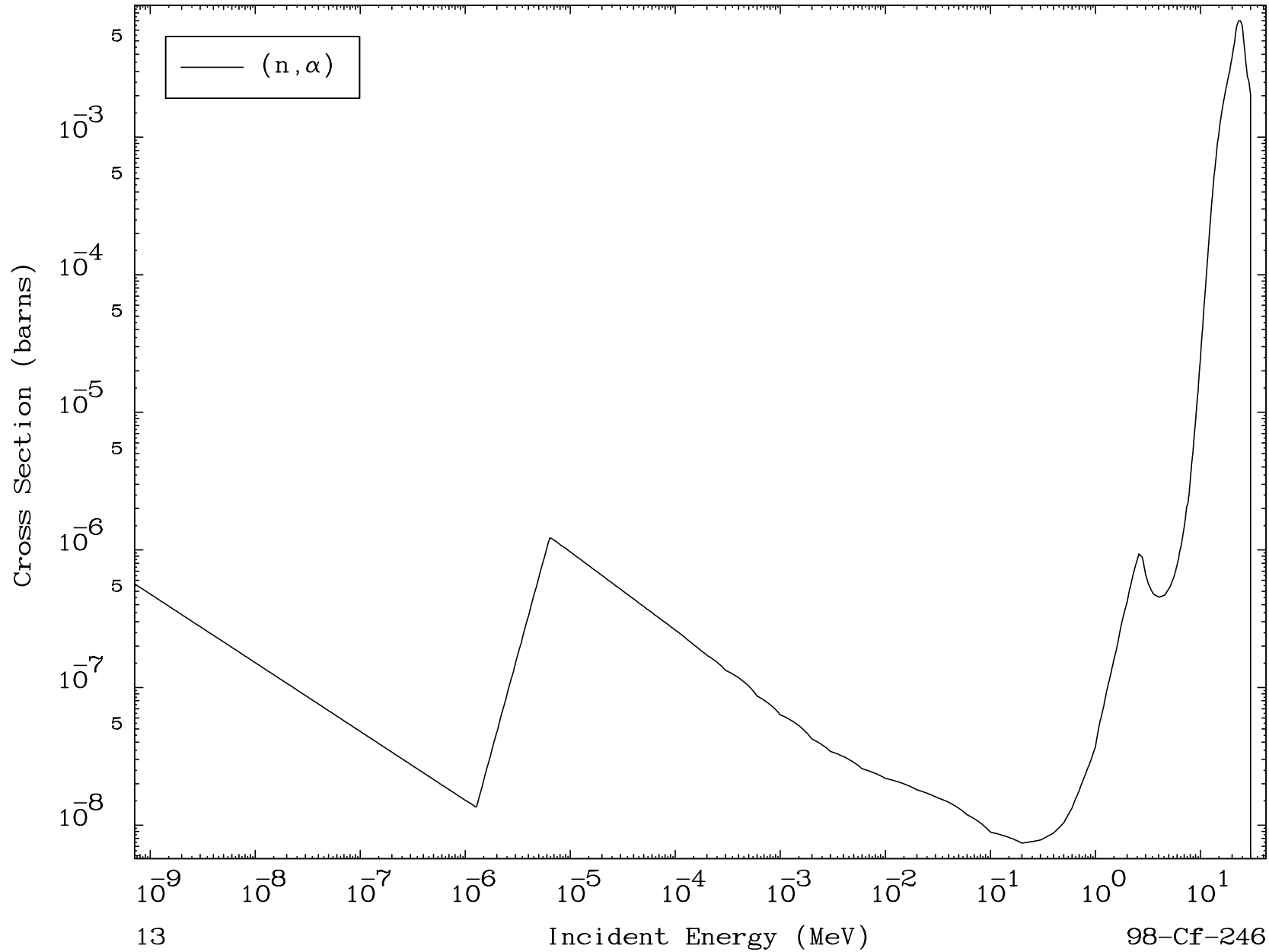
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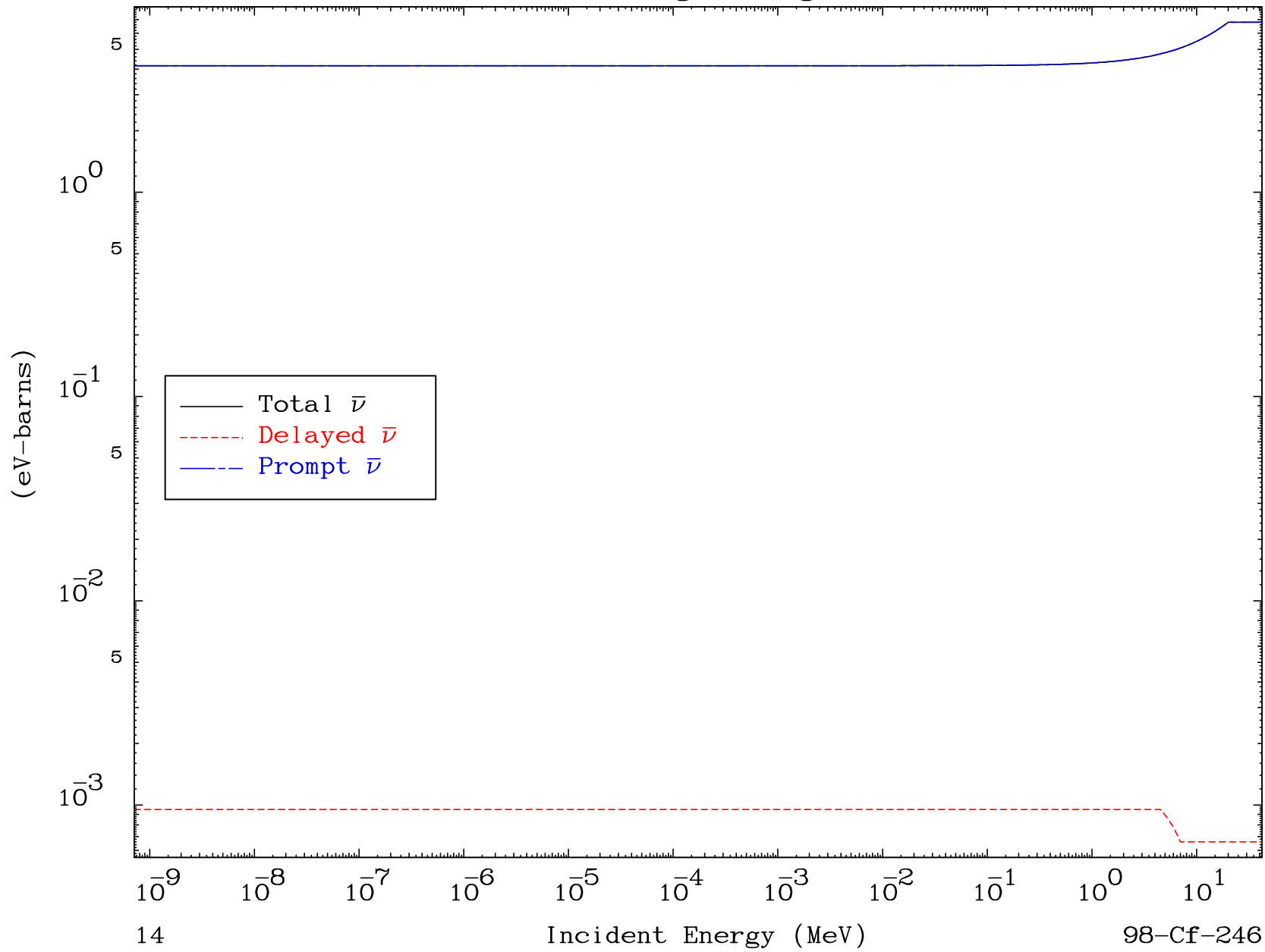


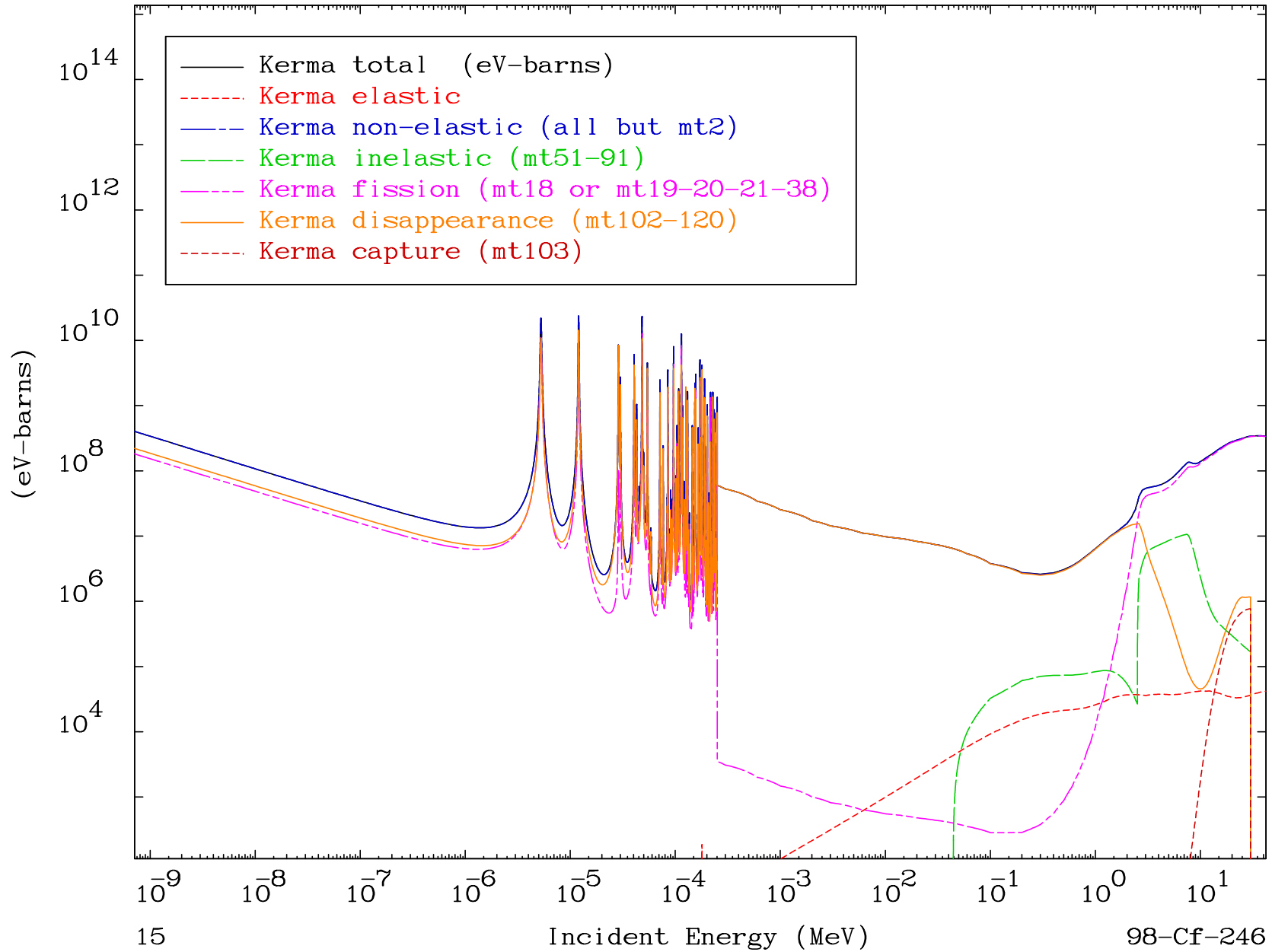


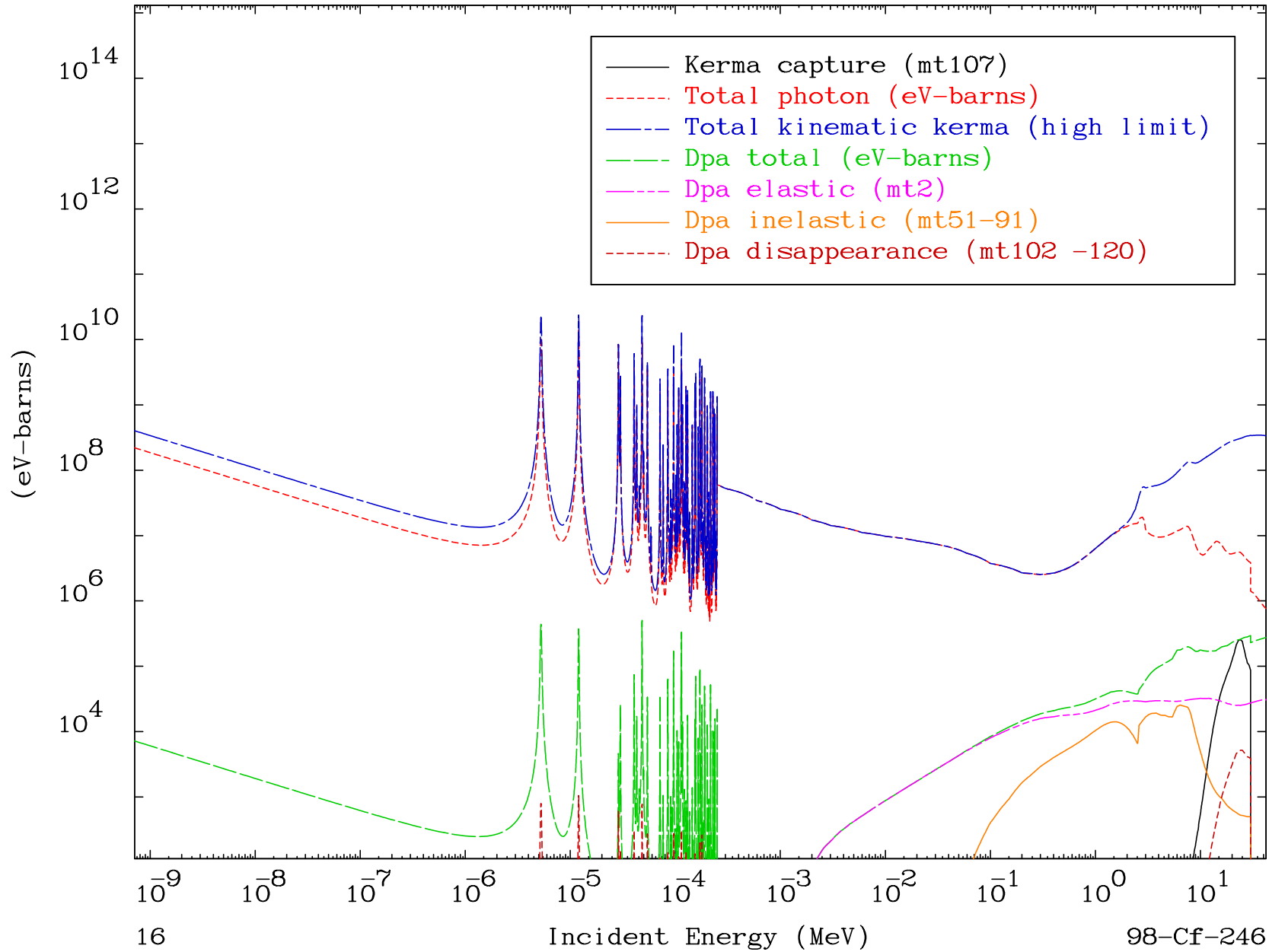


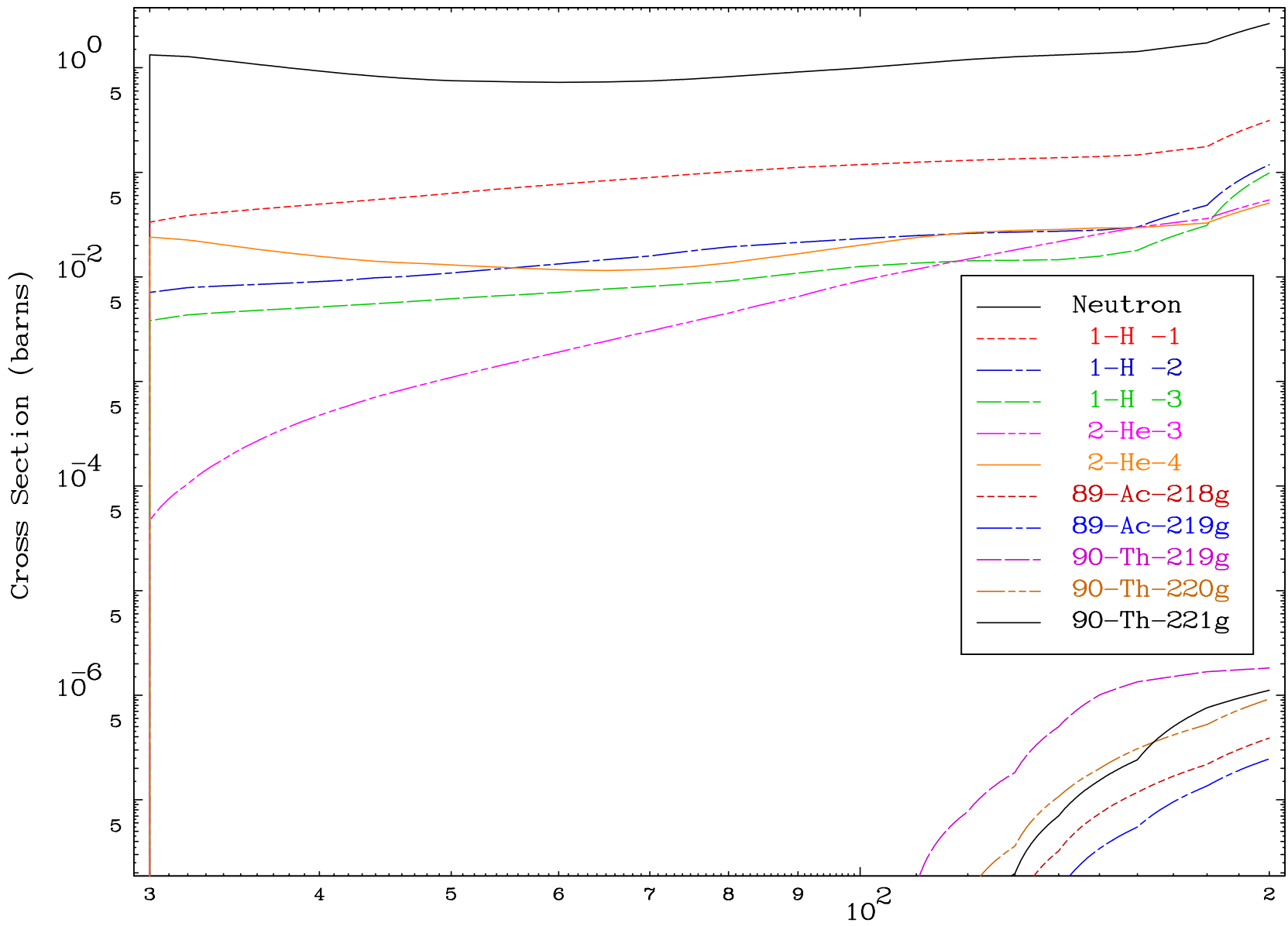


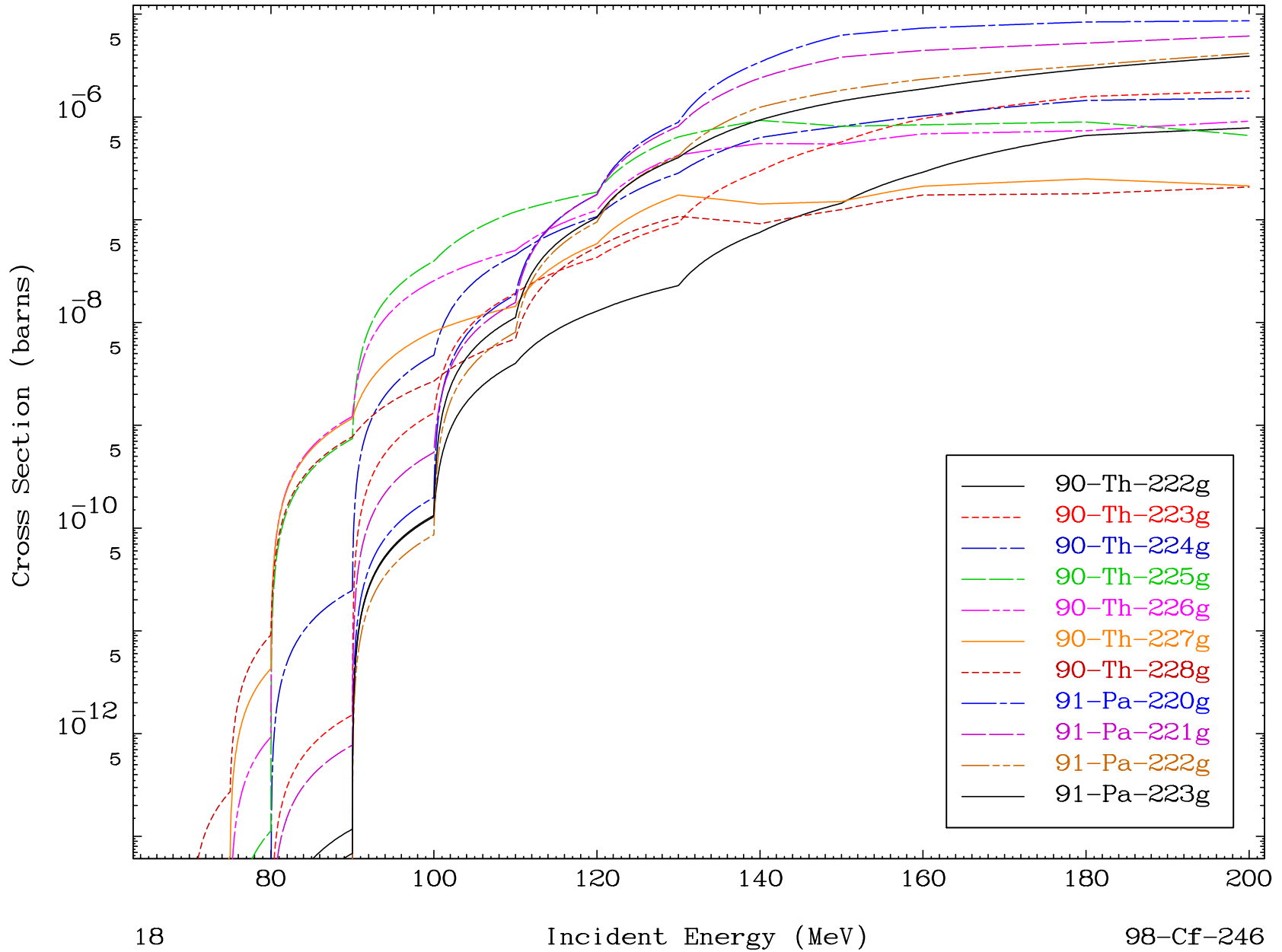


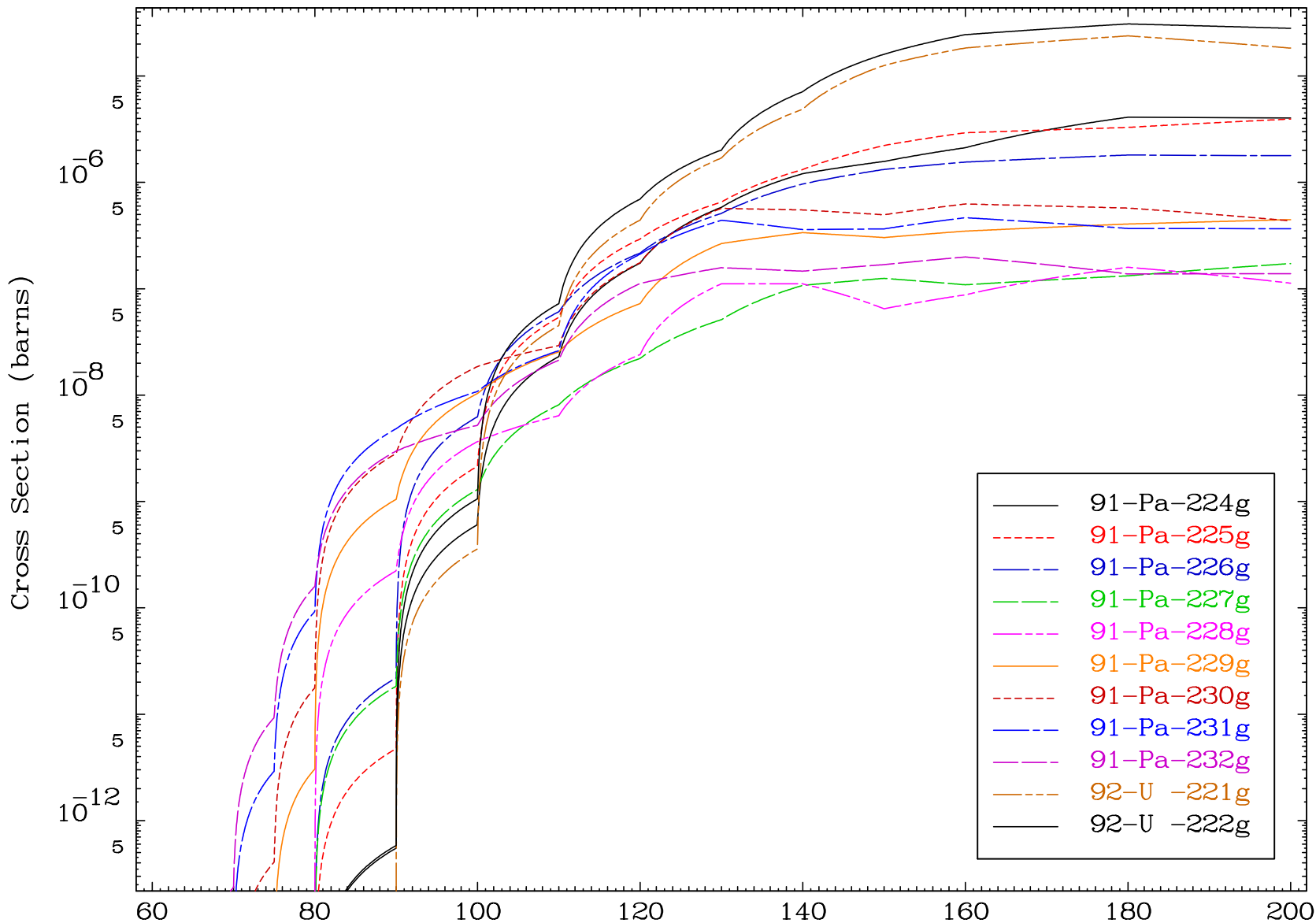




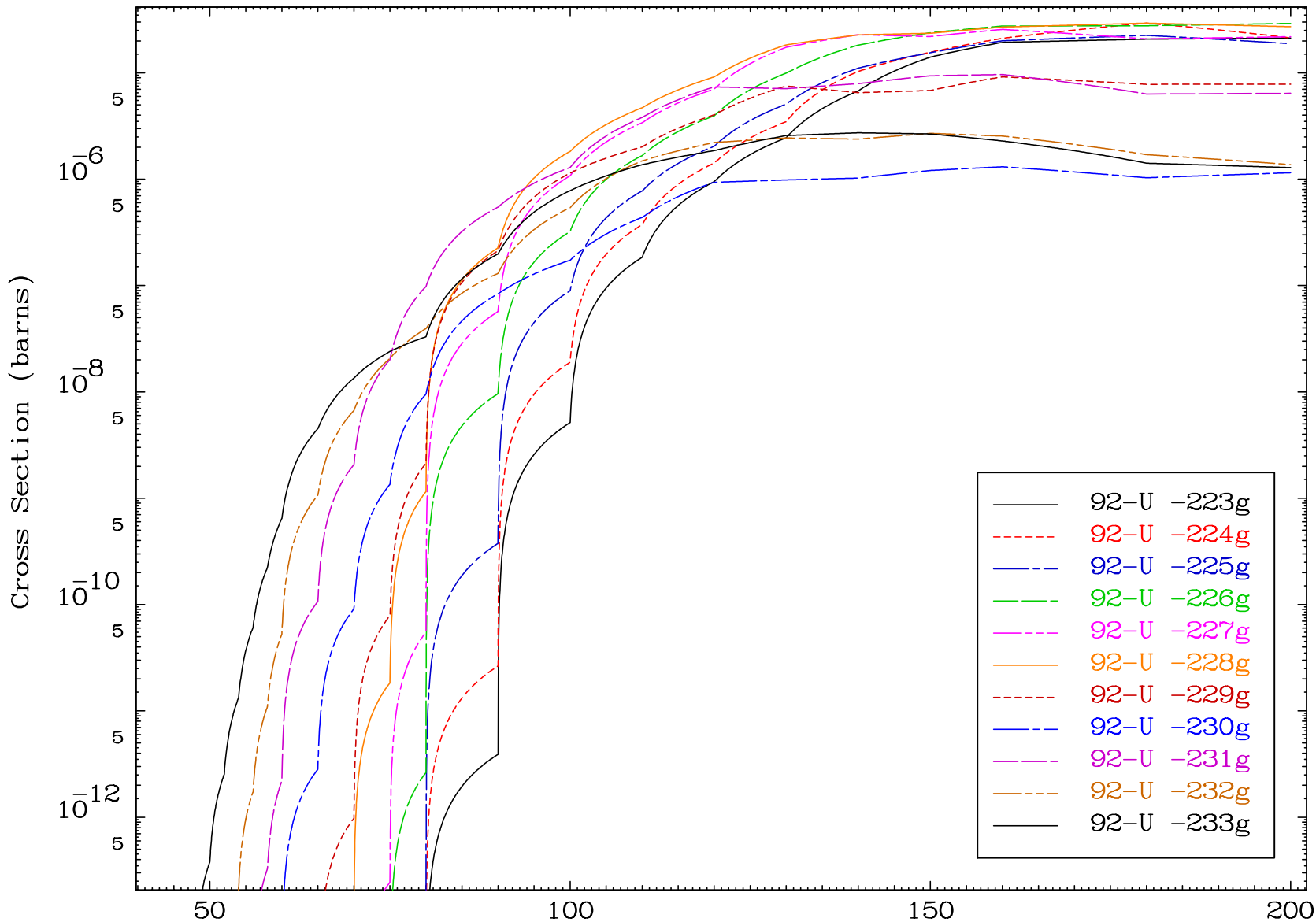




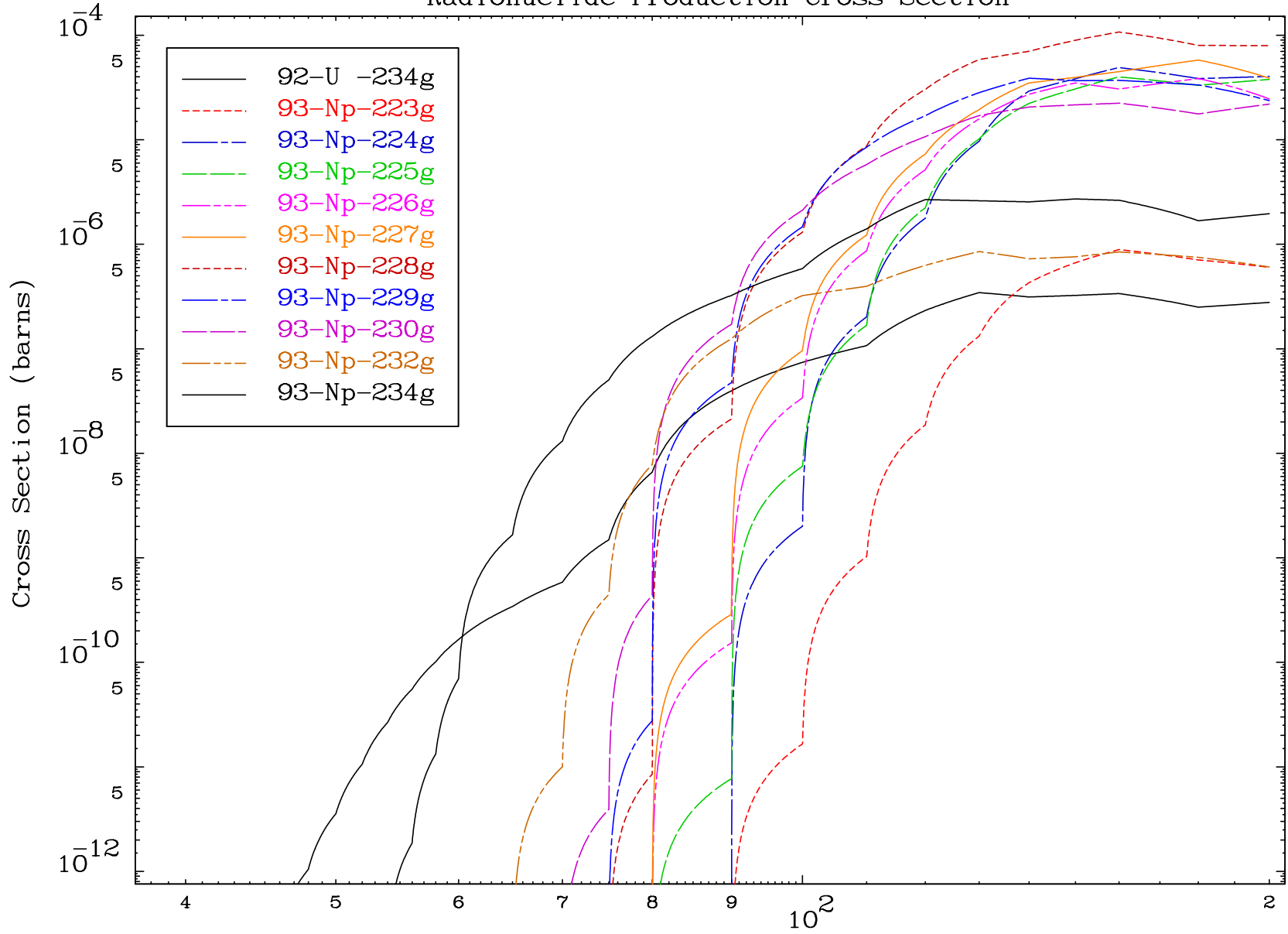


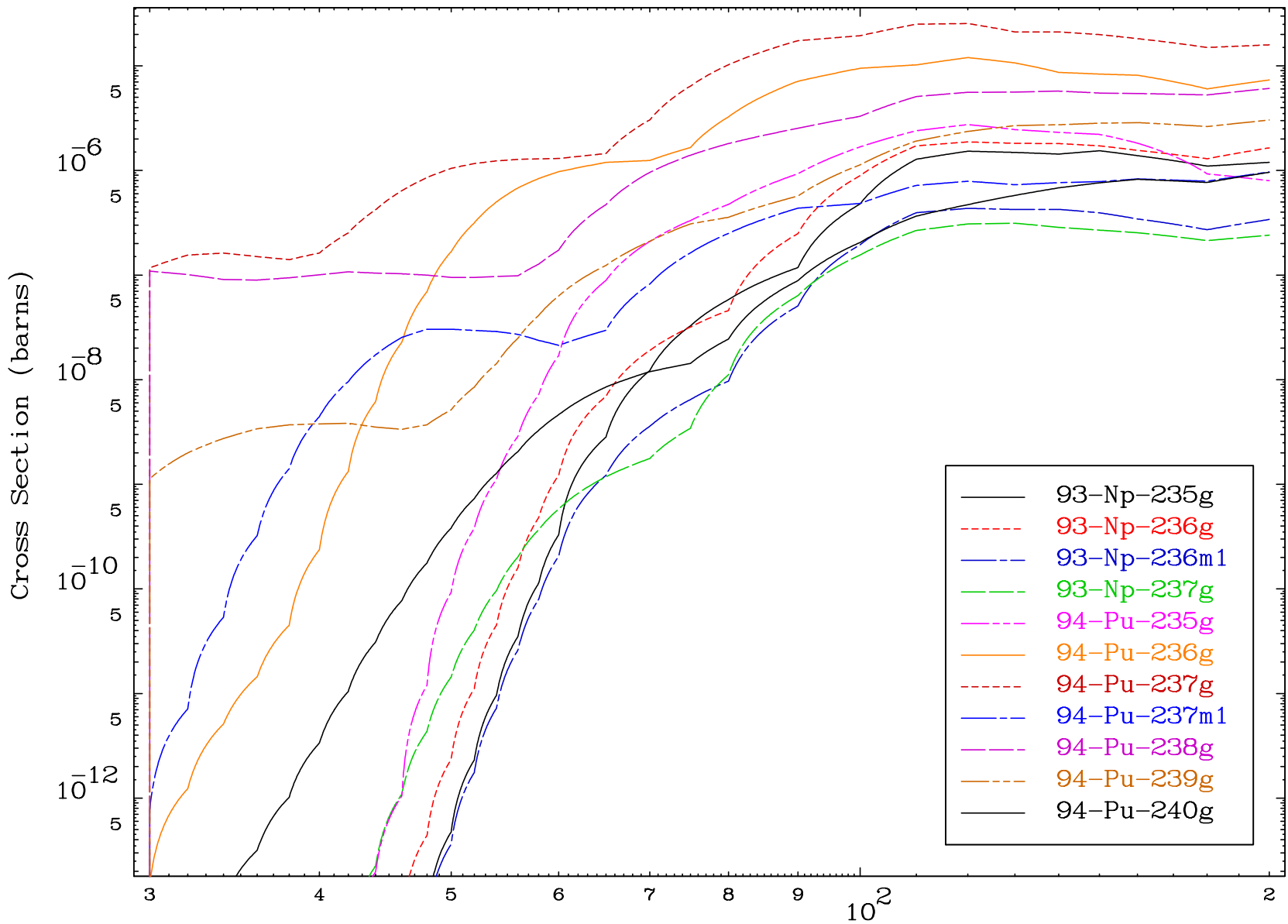


Radionuclide Production Cross Section



Radionuclide Production Cross Section



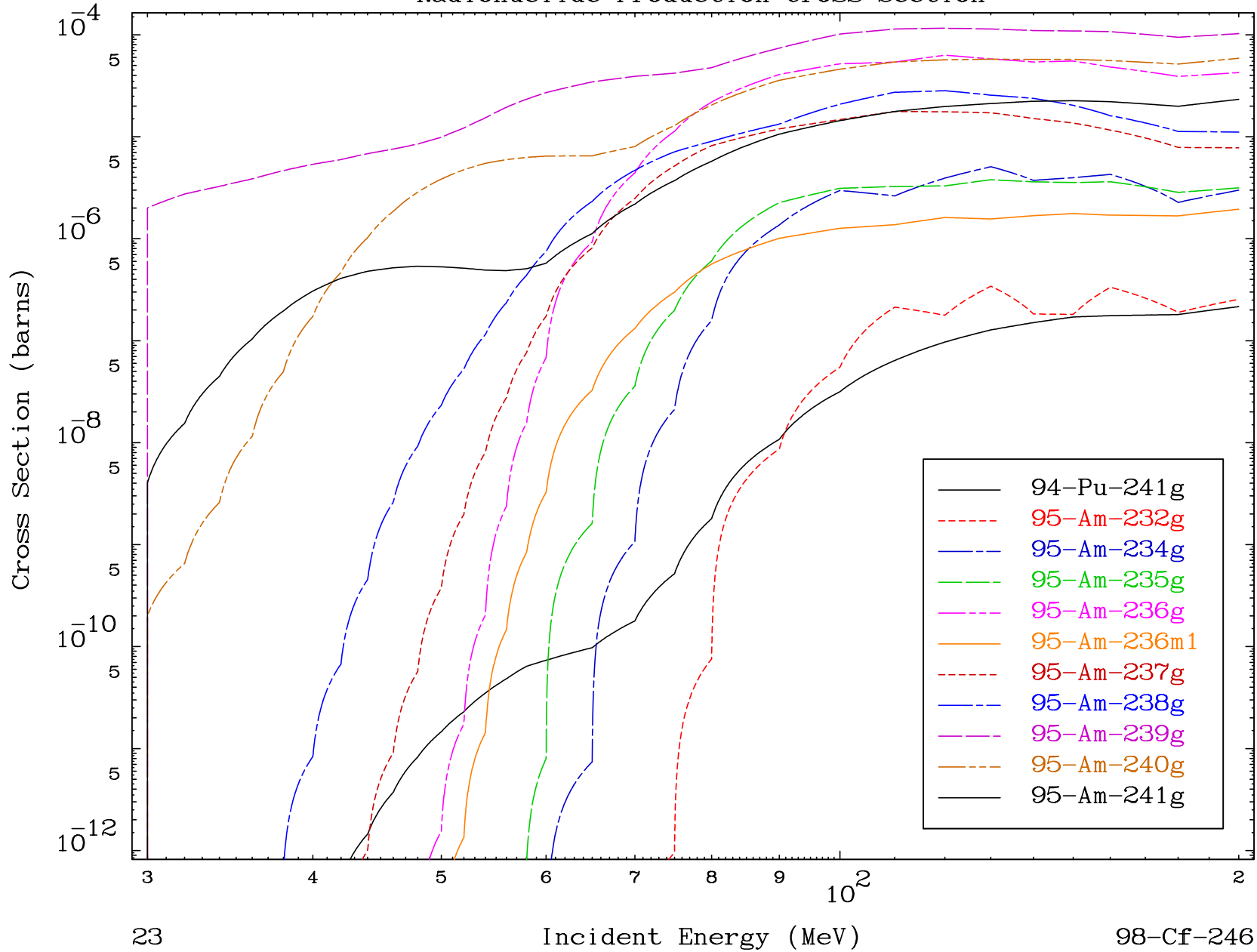


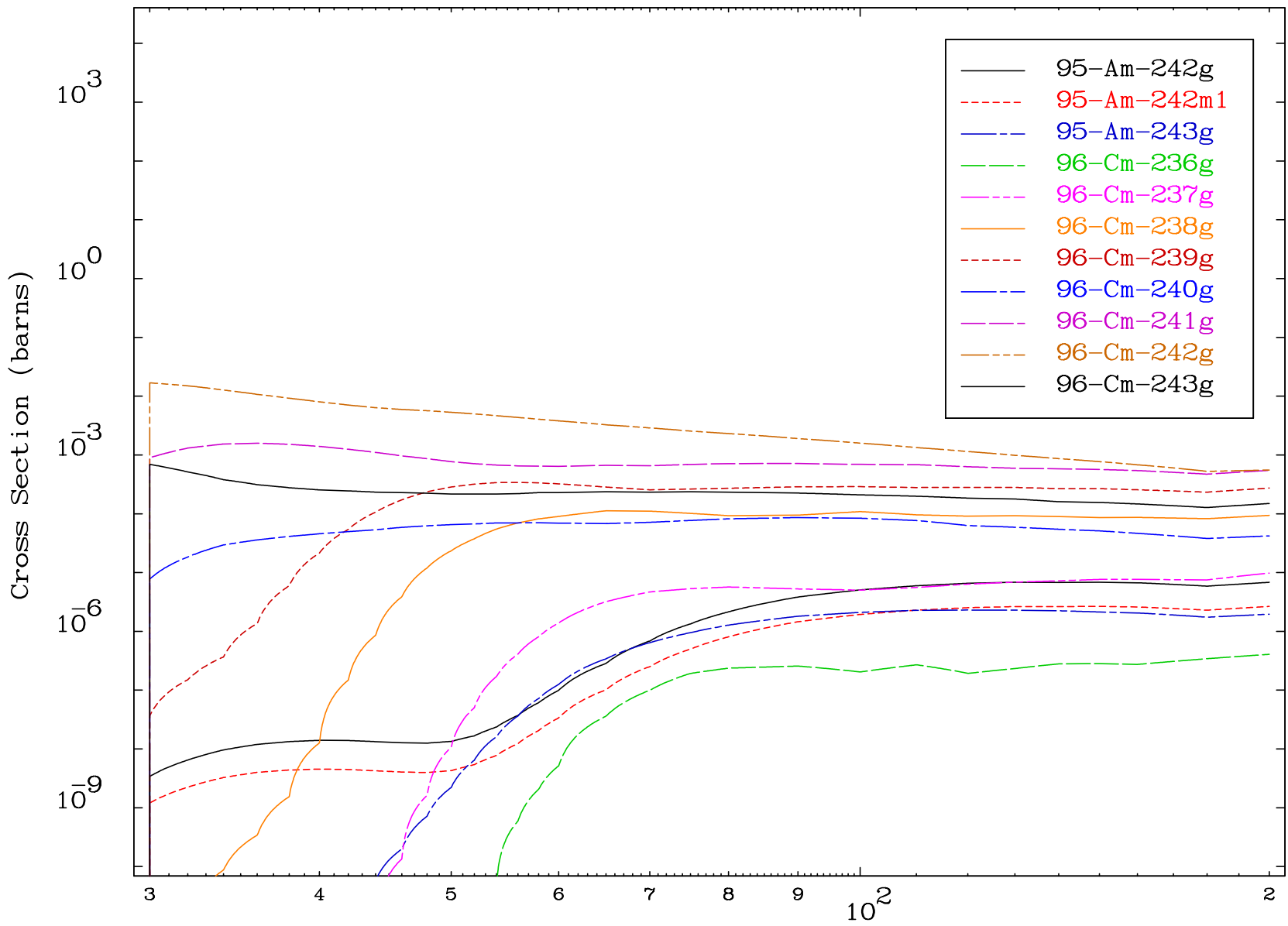
MAT 9843

(n,remainder)

98-Cf-246

Radionuclide Production Cross Section

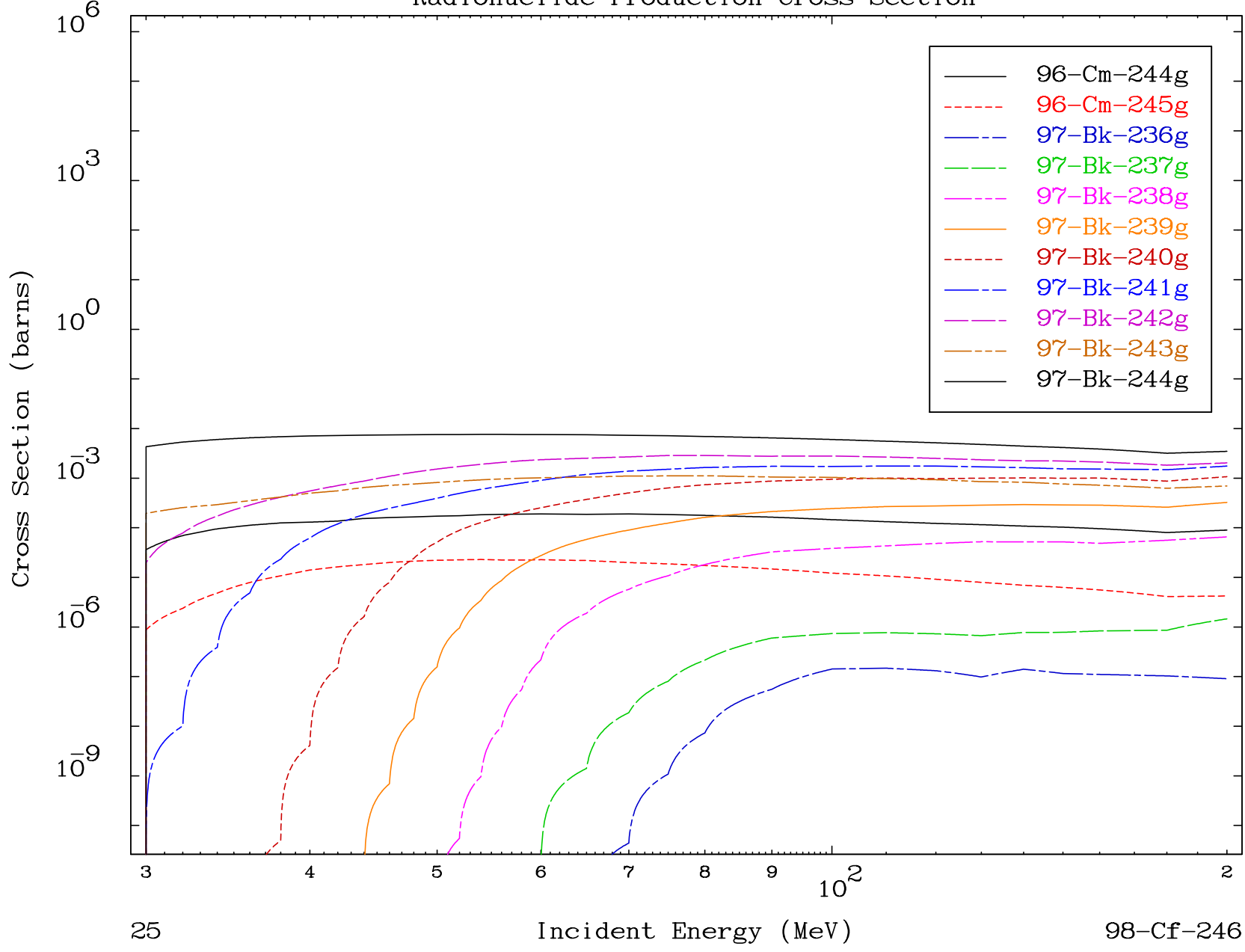




MAT 9843

(n,remainder)
Radionuclide Production Cross Section

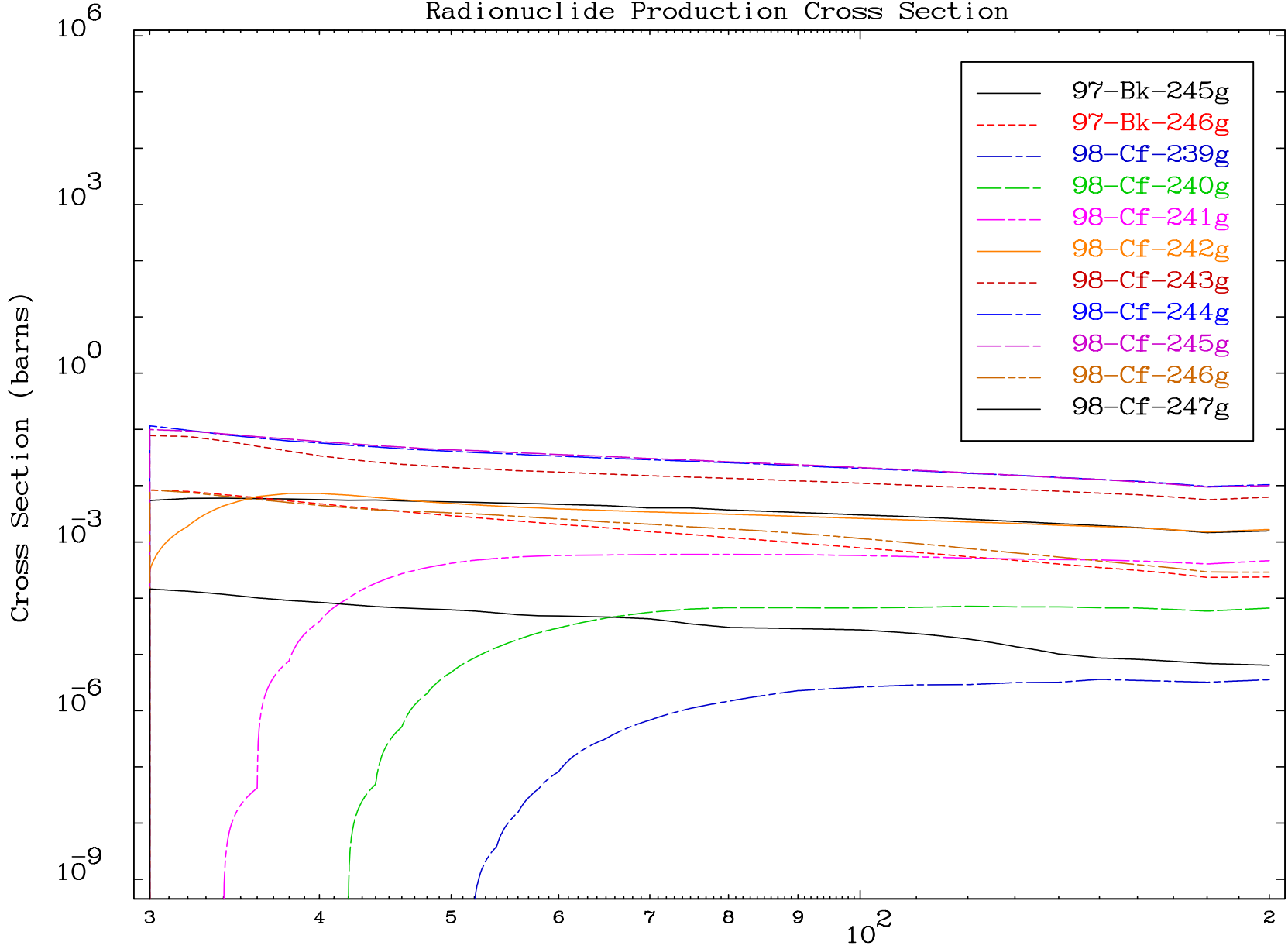
98-Cf-246



MAT 9843

(n,remainder)
Radionuclide Production Cross Section

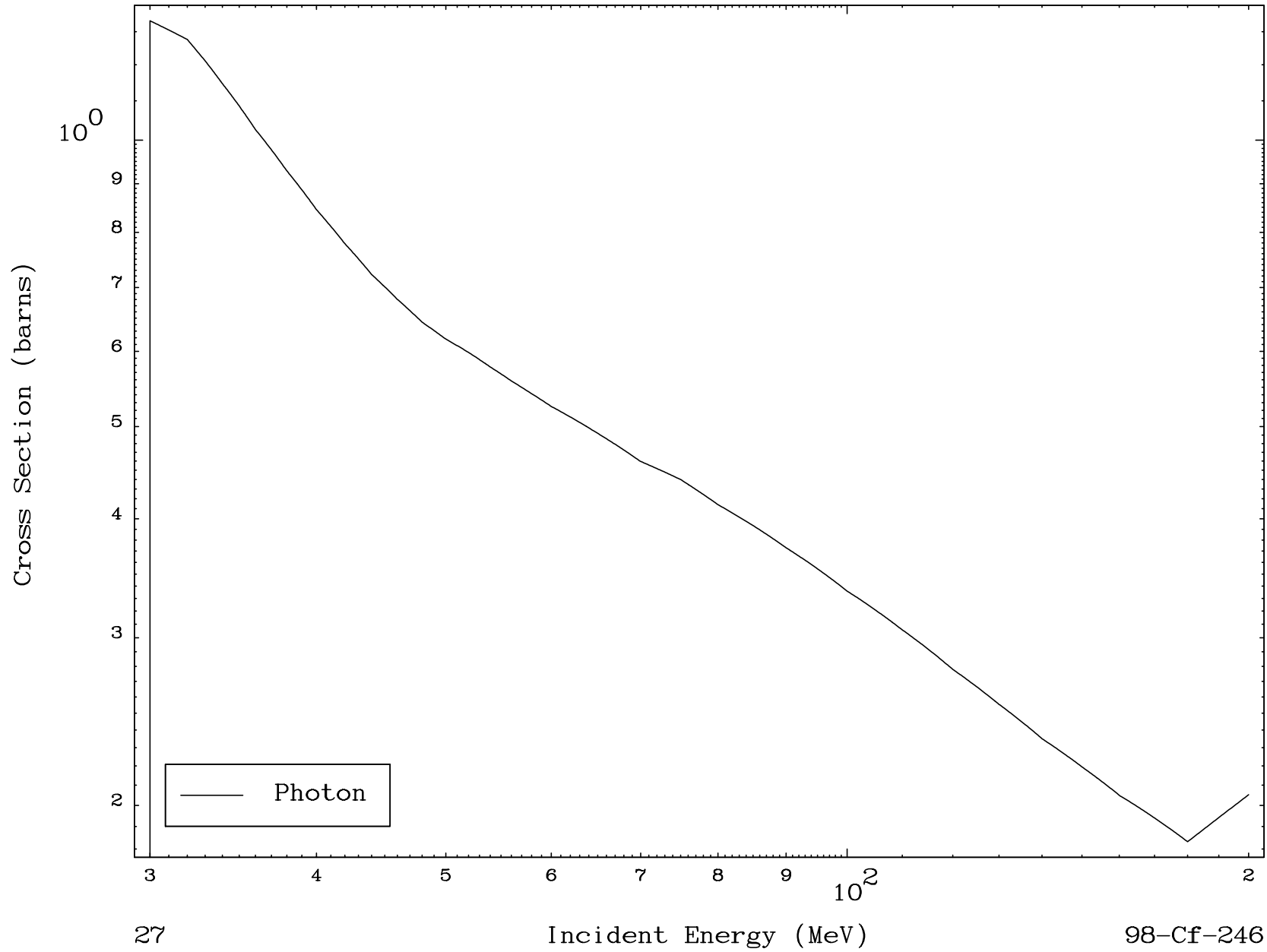
98-Cf-246



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Incident Energy (MeV)

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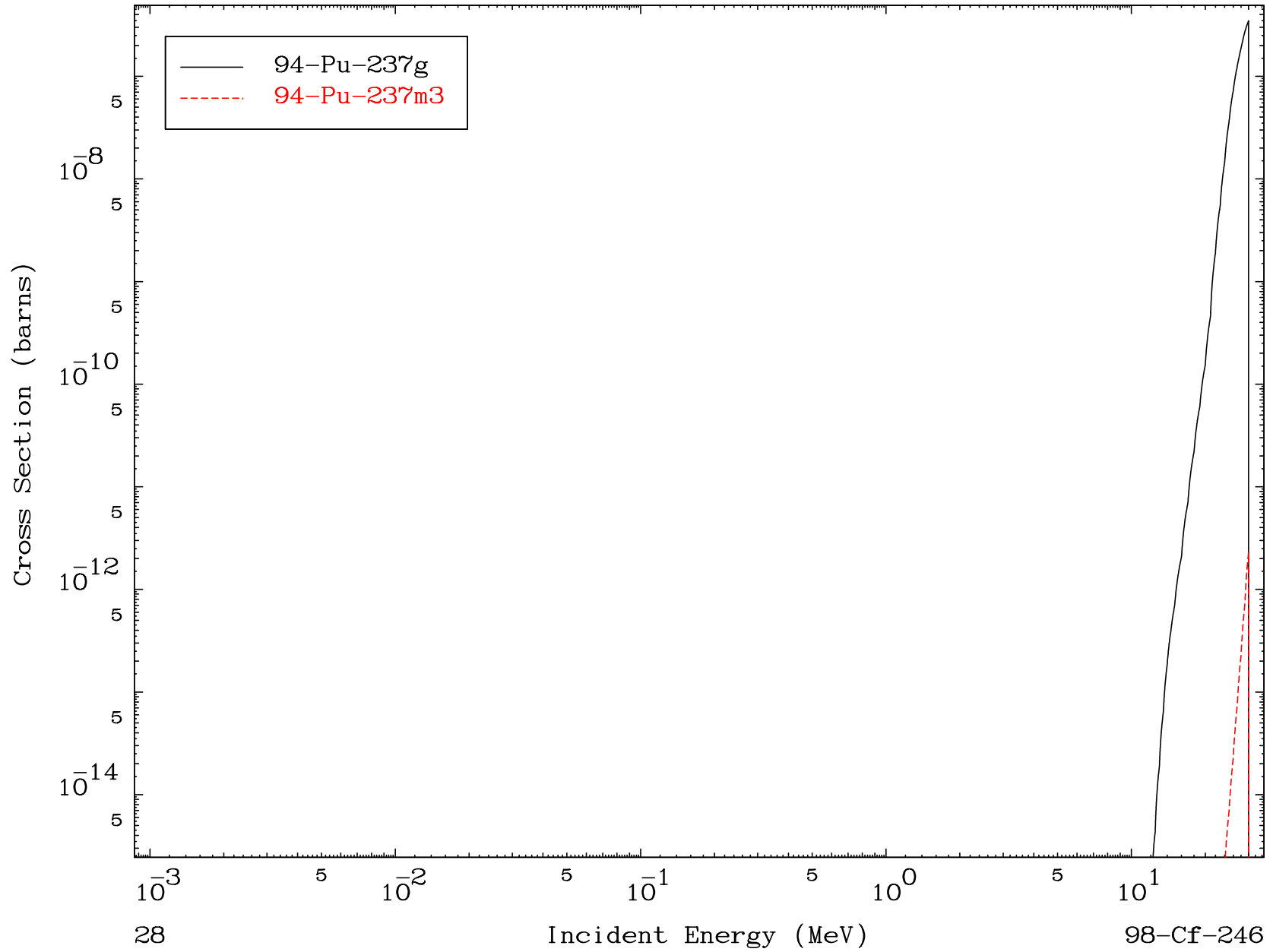


MAT 9843

(n,2n) 2 α

98-Cf-246

Radionuclide Production Cross Section



MAT 9843

(n,p) α

98-Cf-246

Radionuclide Production Cross Section

