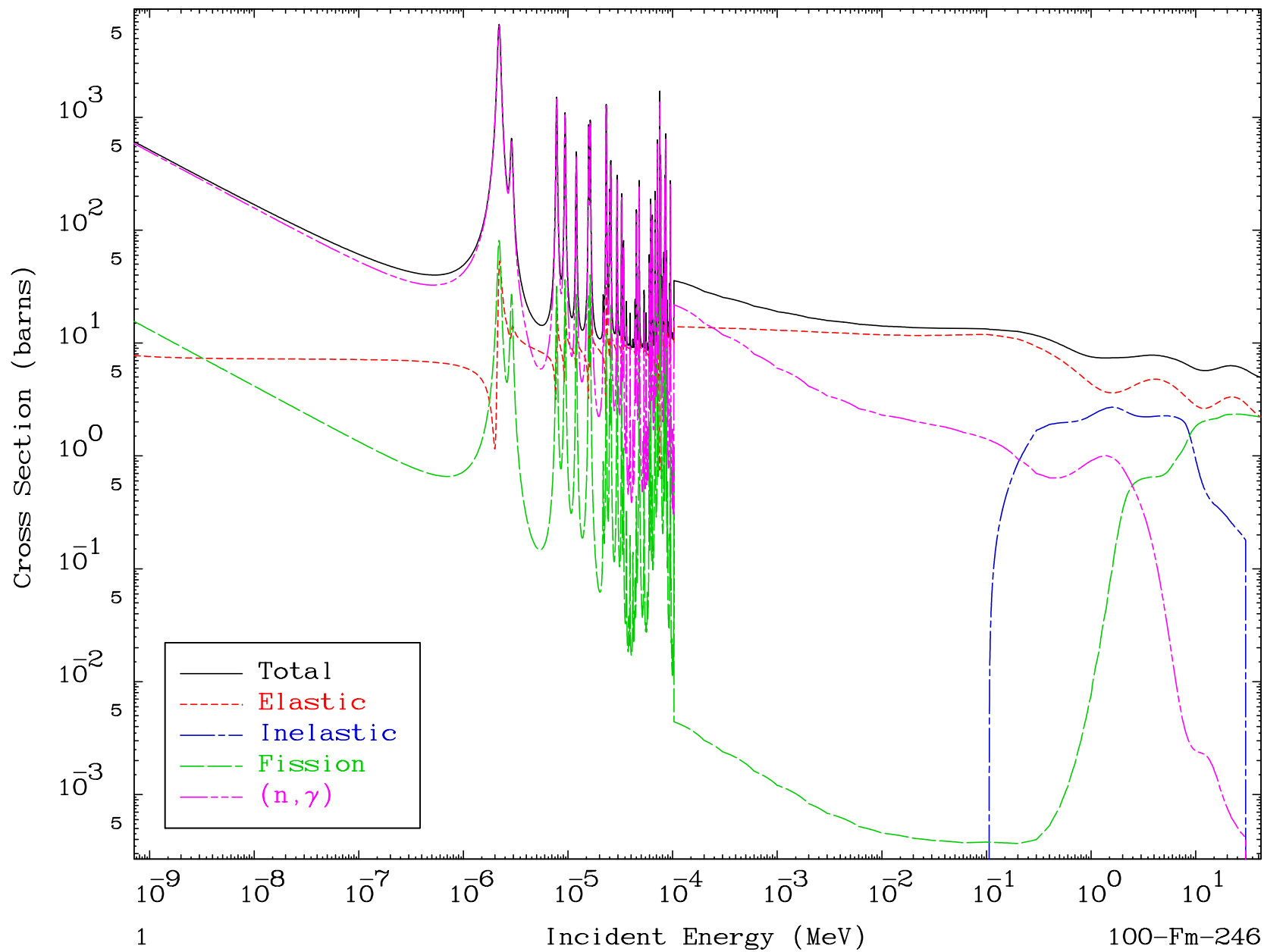
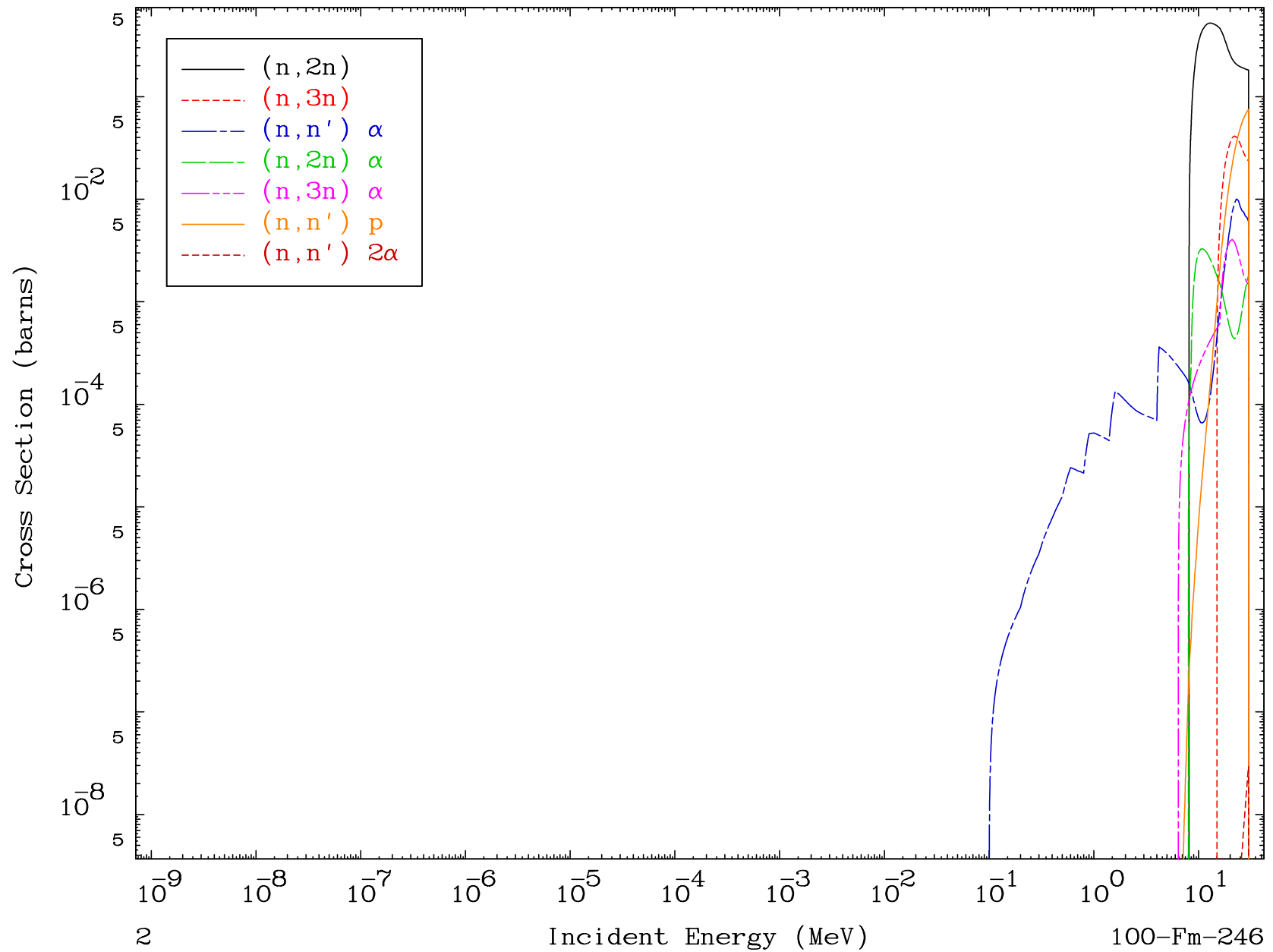
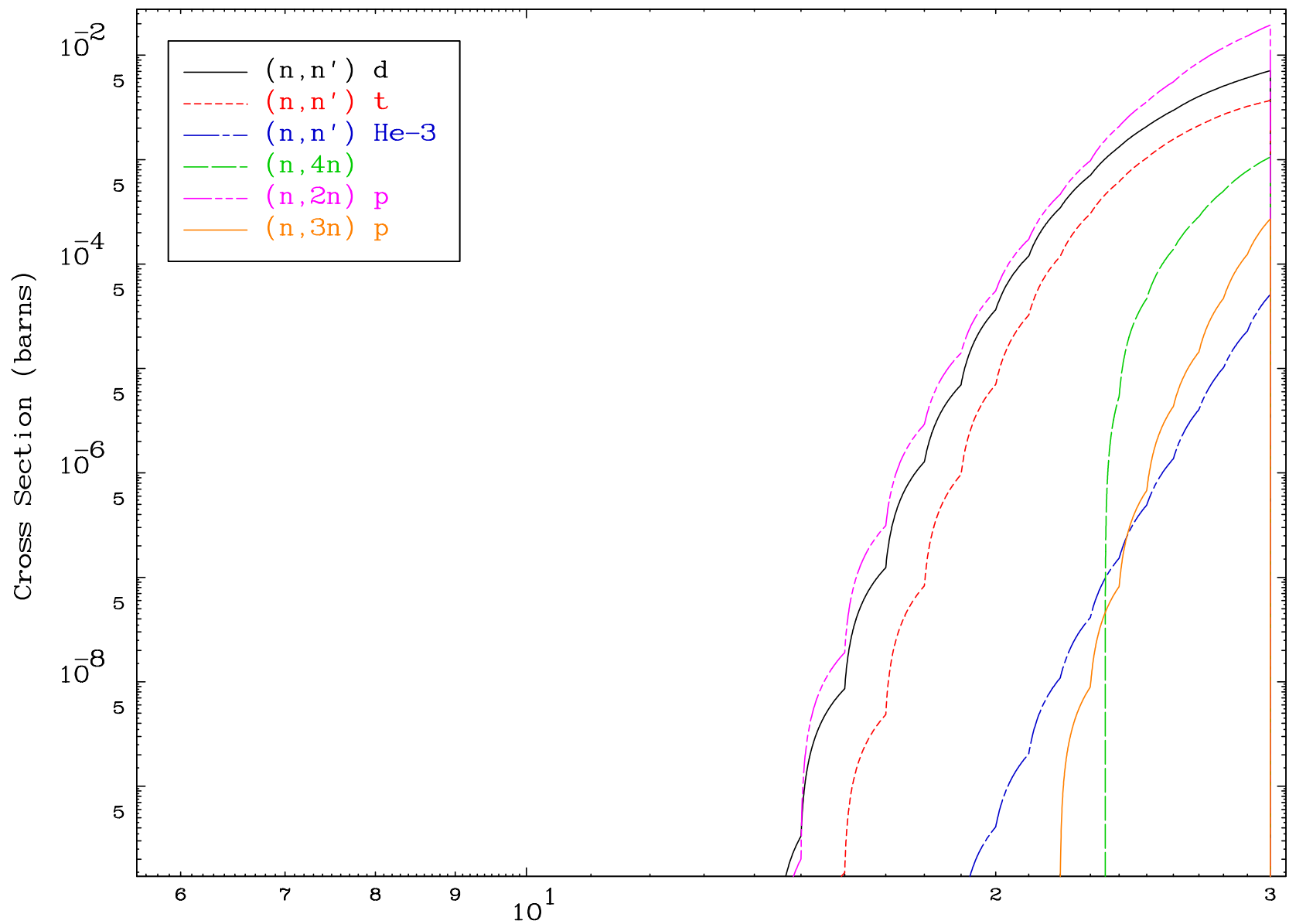
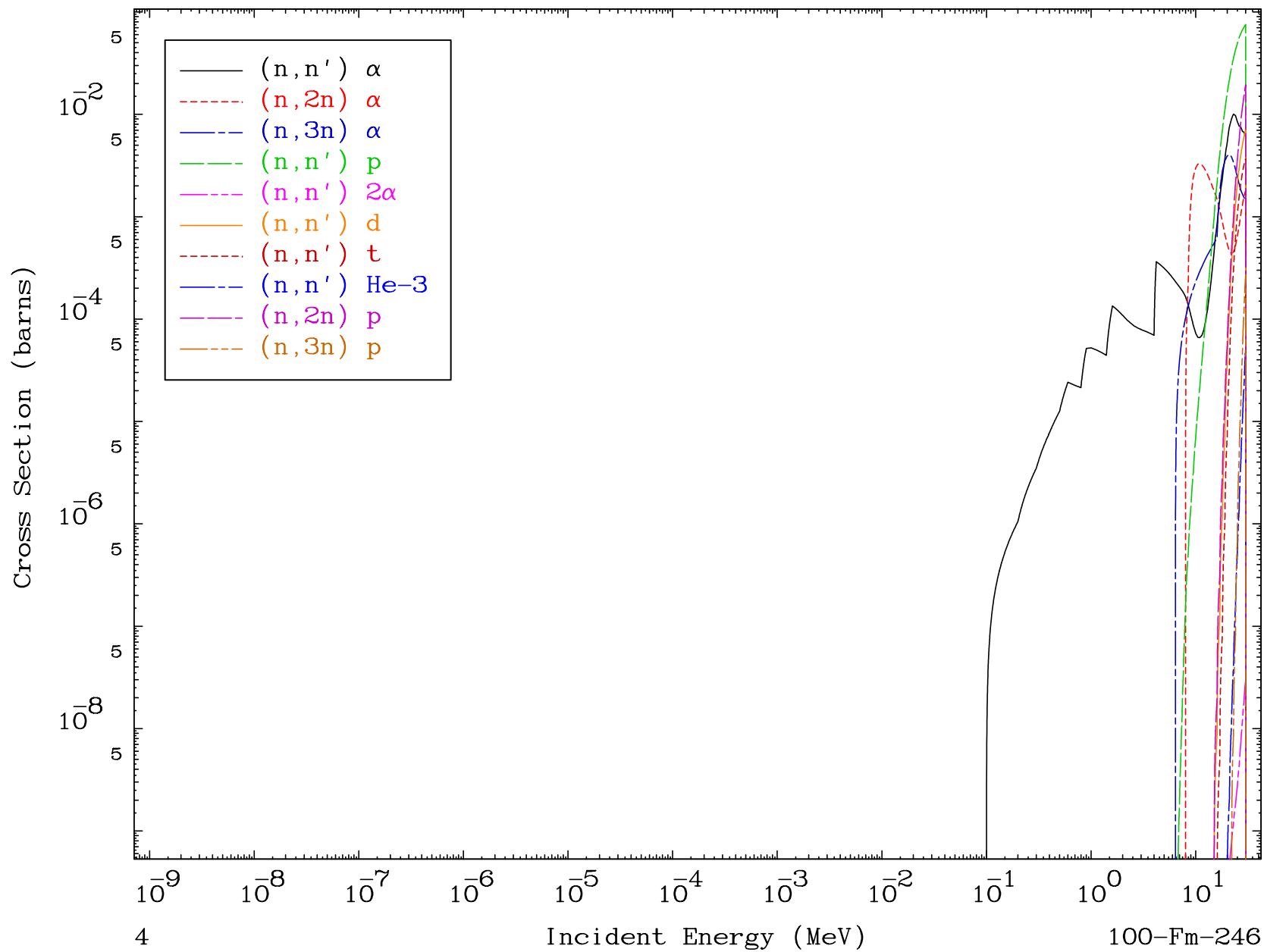
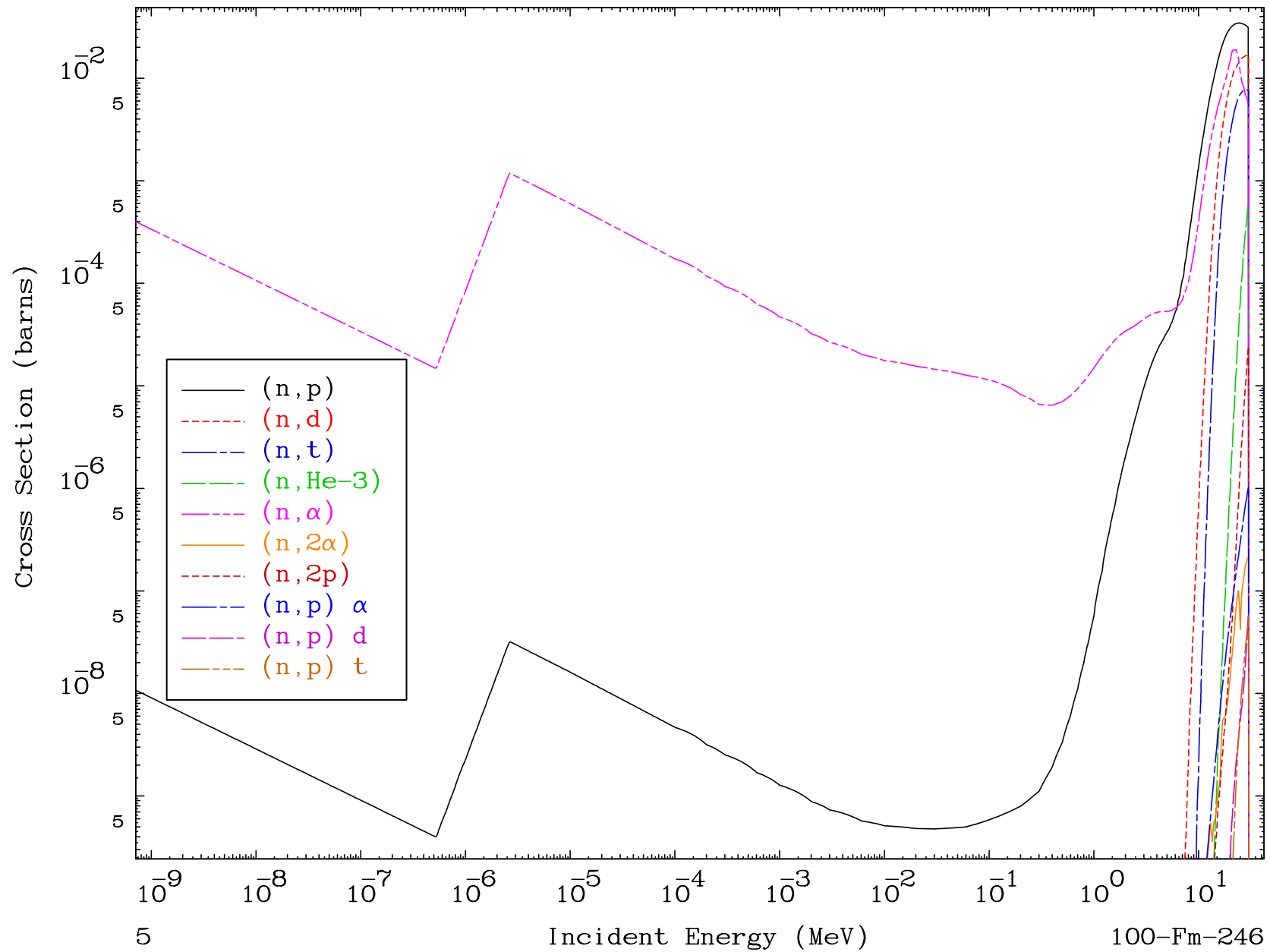


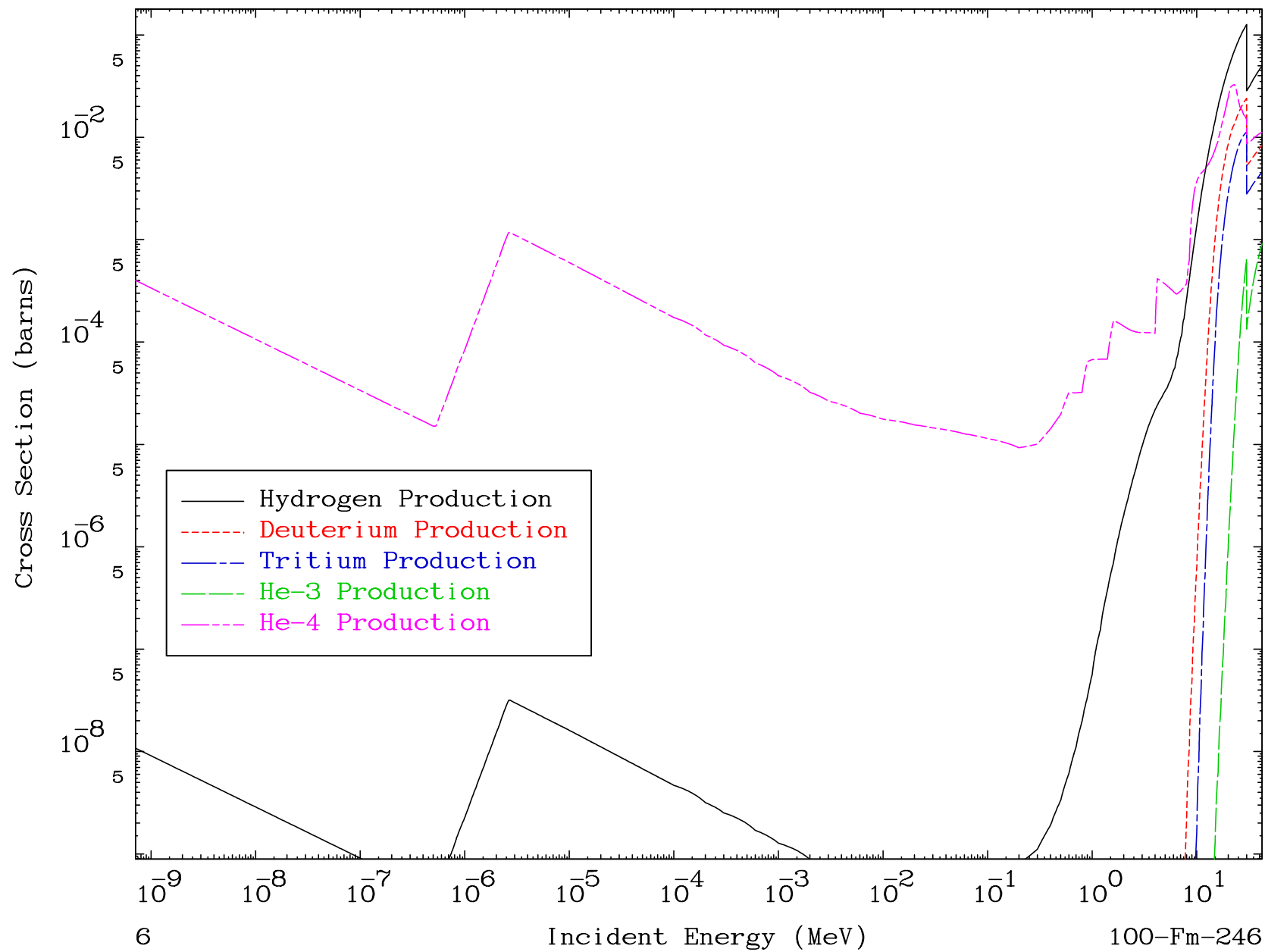


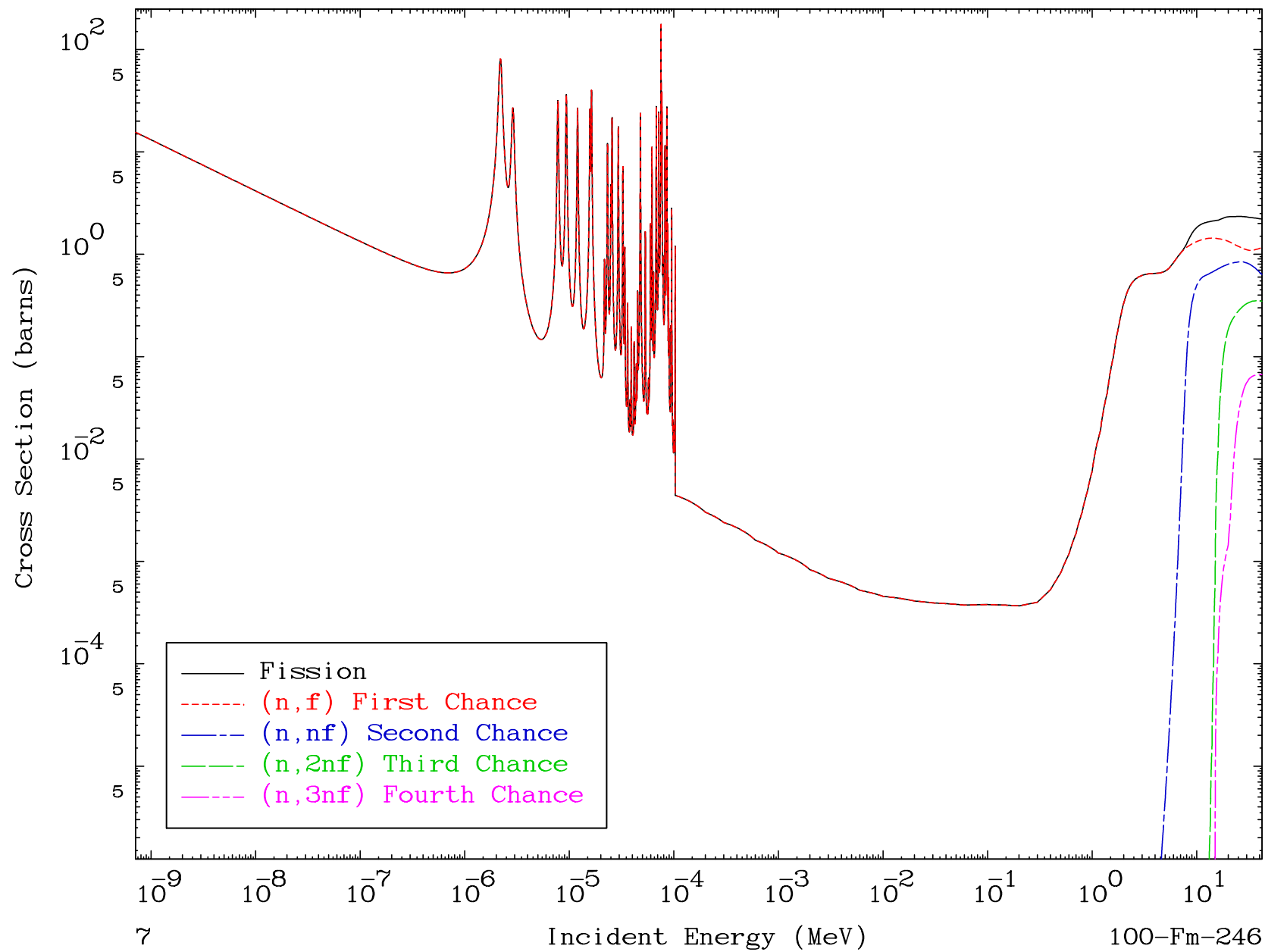








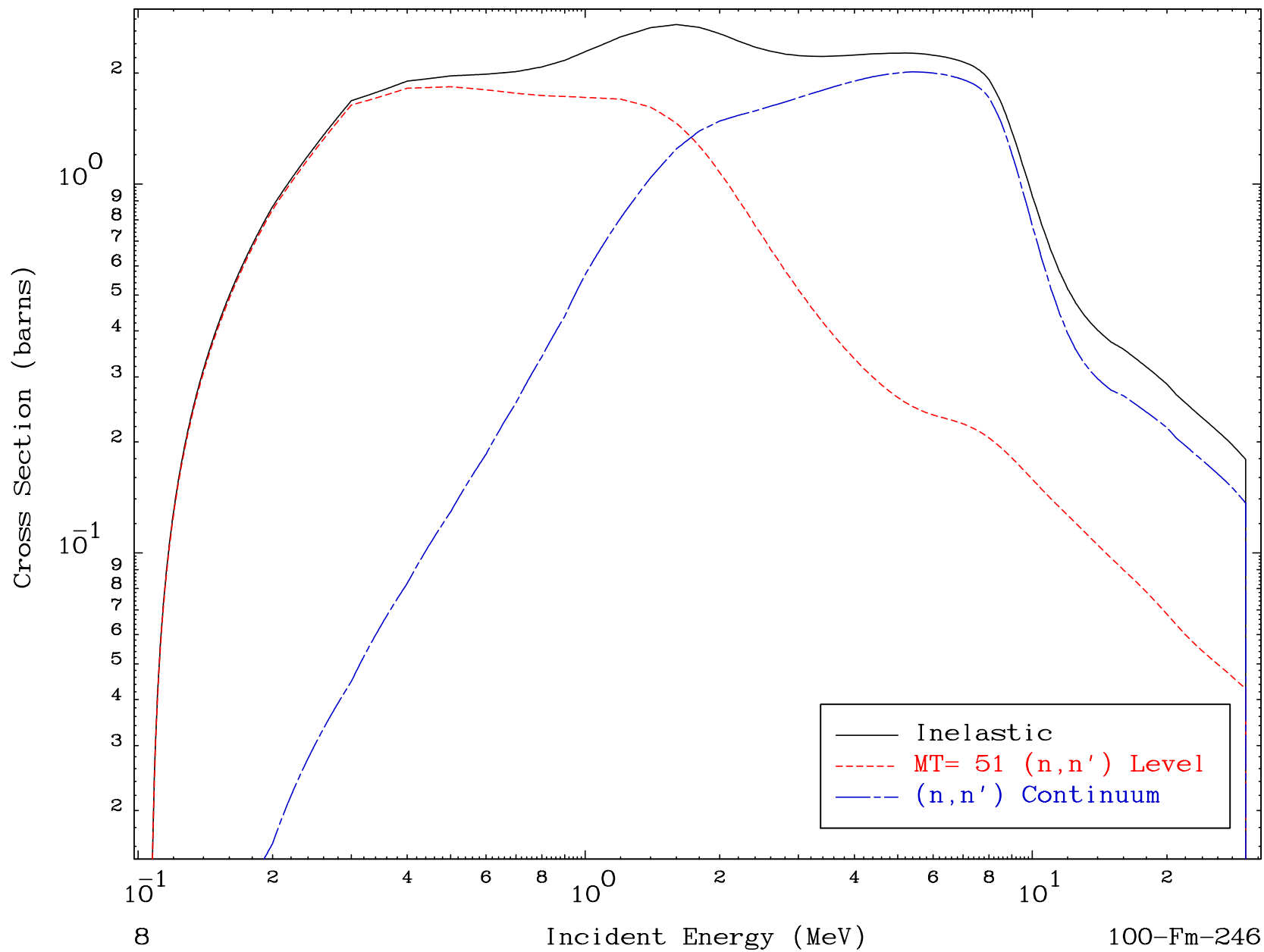


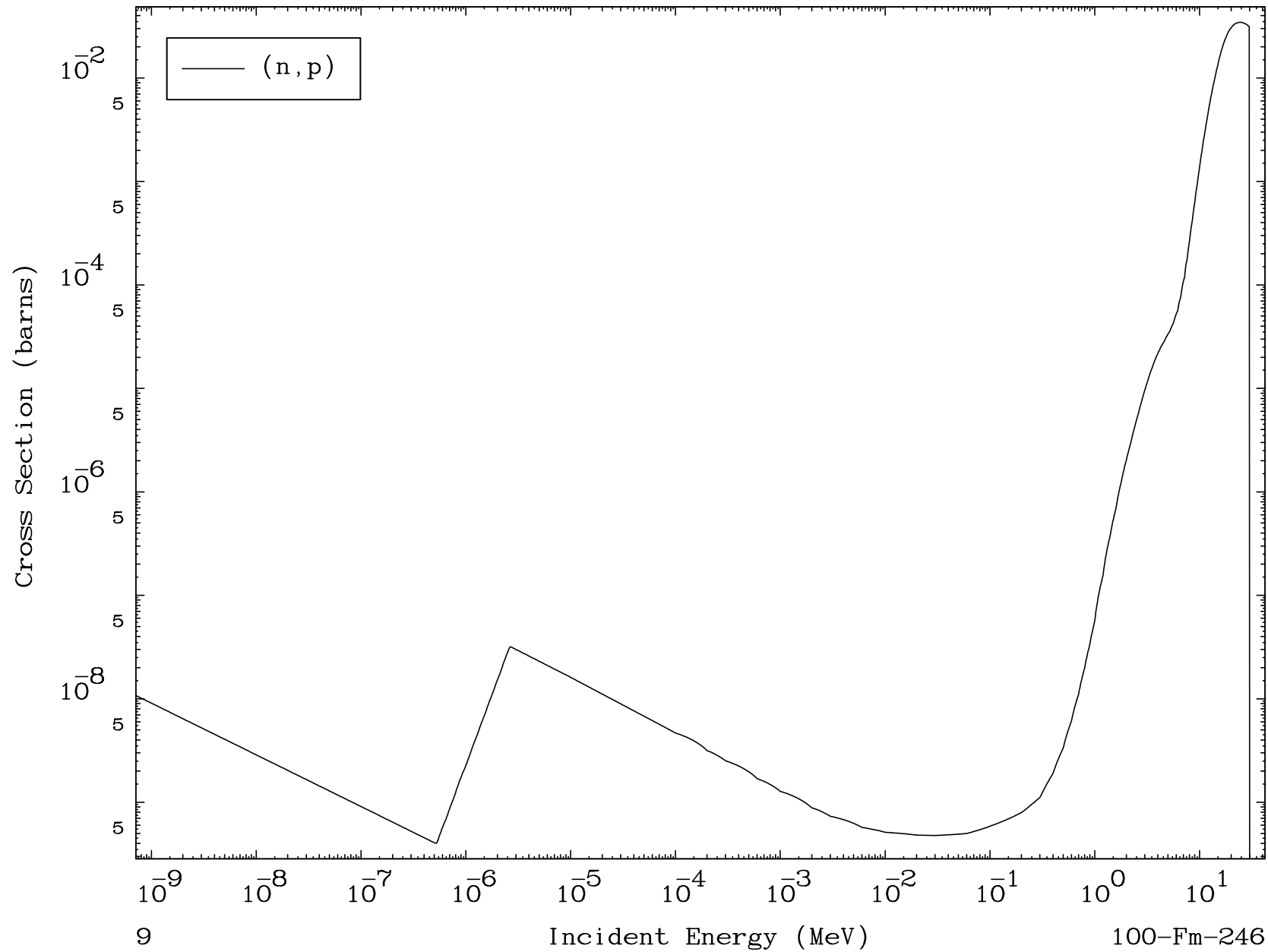


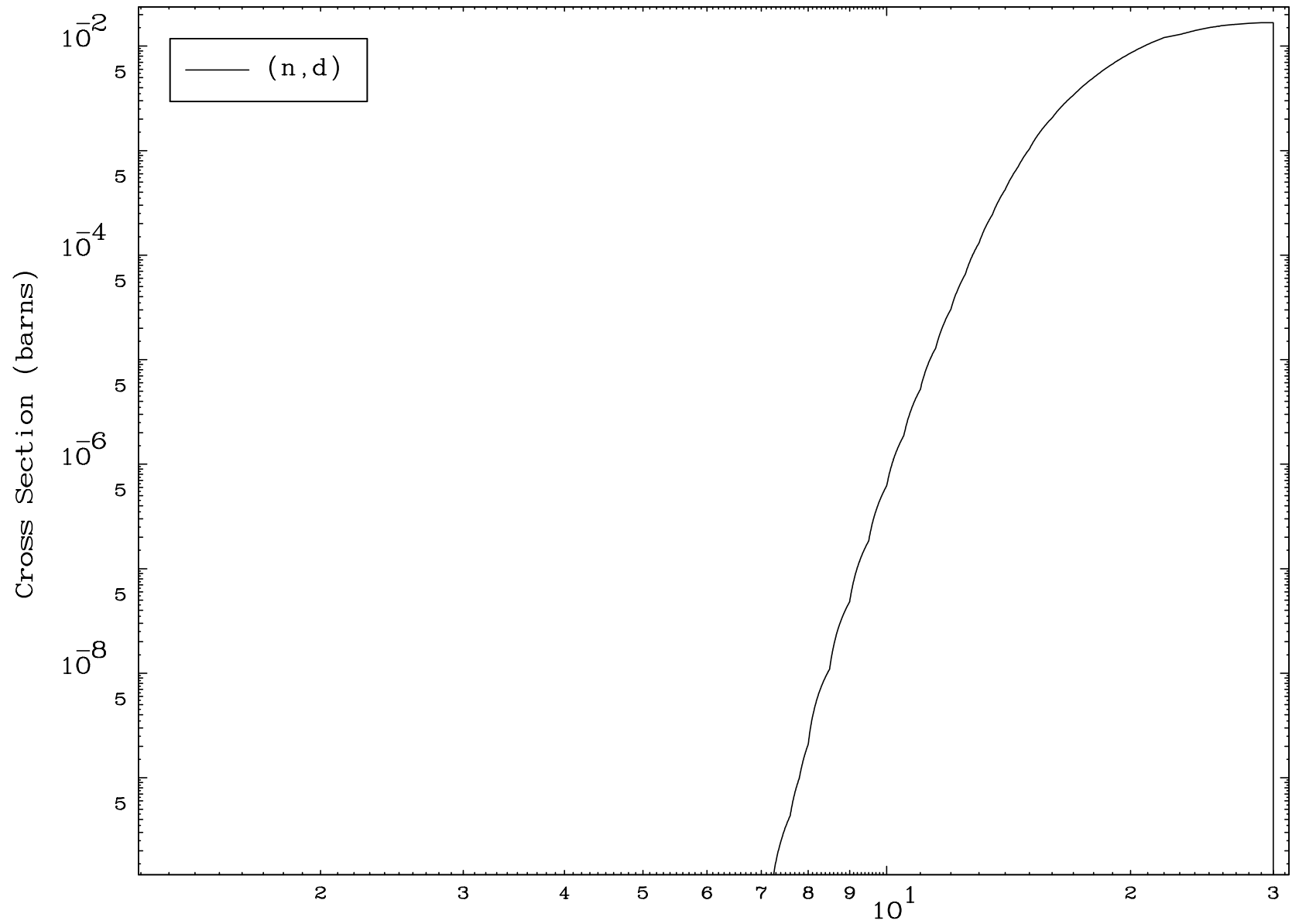
MAT 9927

 (n,n') Level
294 Kelvin Cross Sections

100-Fm-246



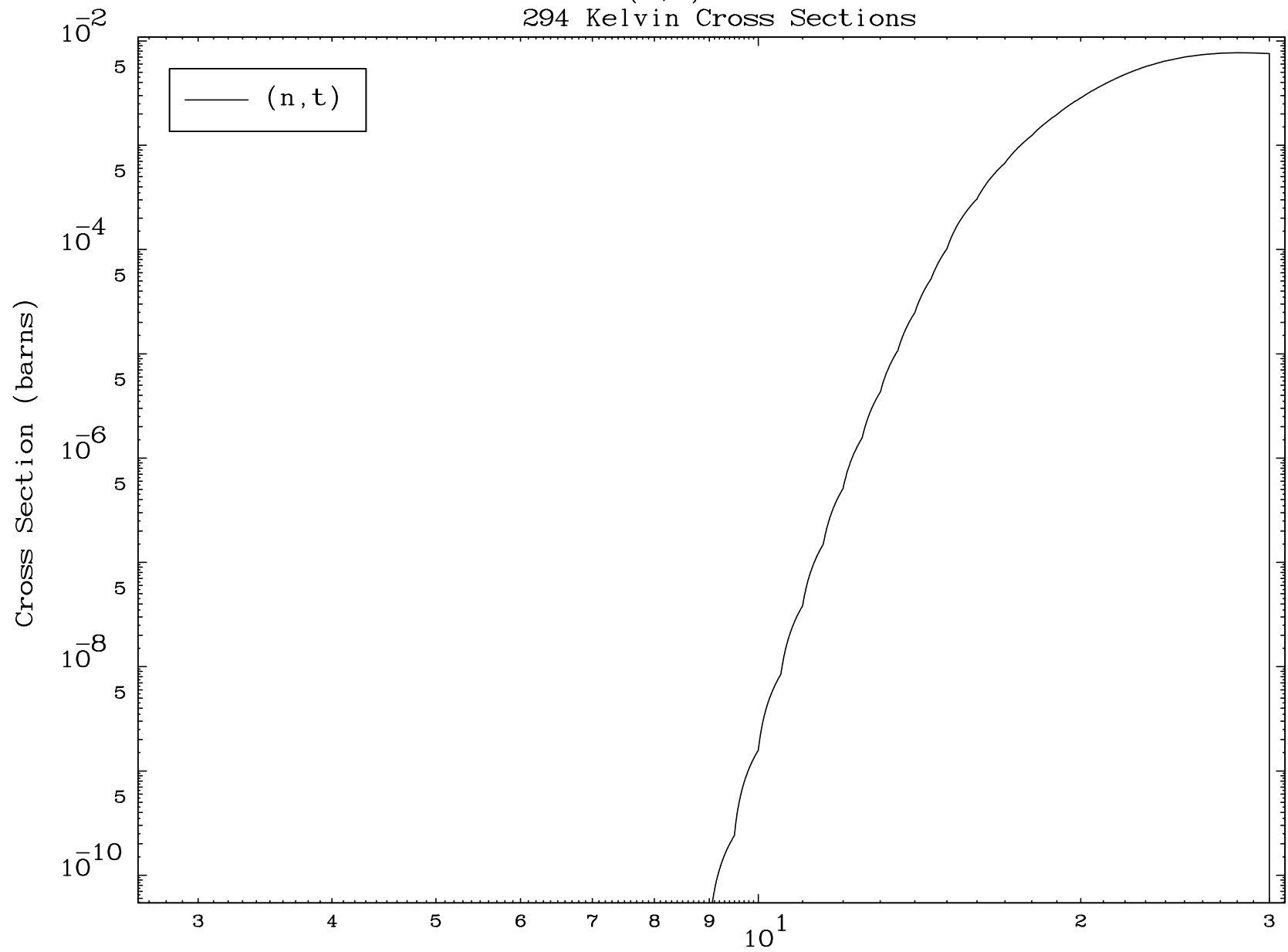




MAT 9927

(n,t) Levels
294 Kelvin Cross Sections

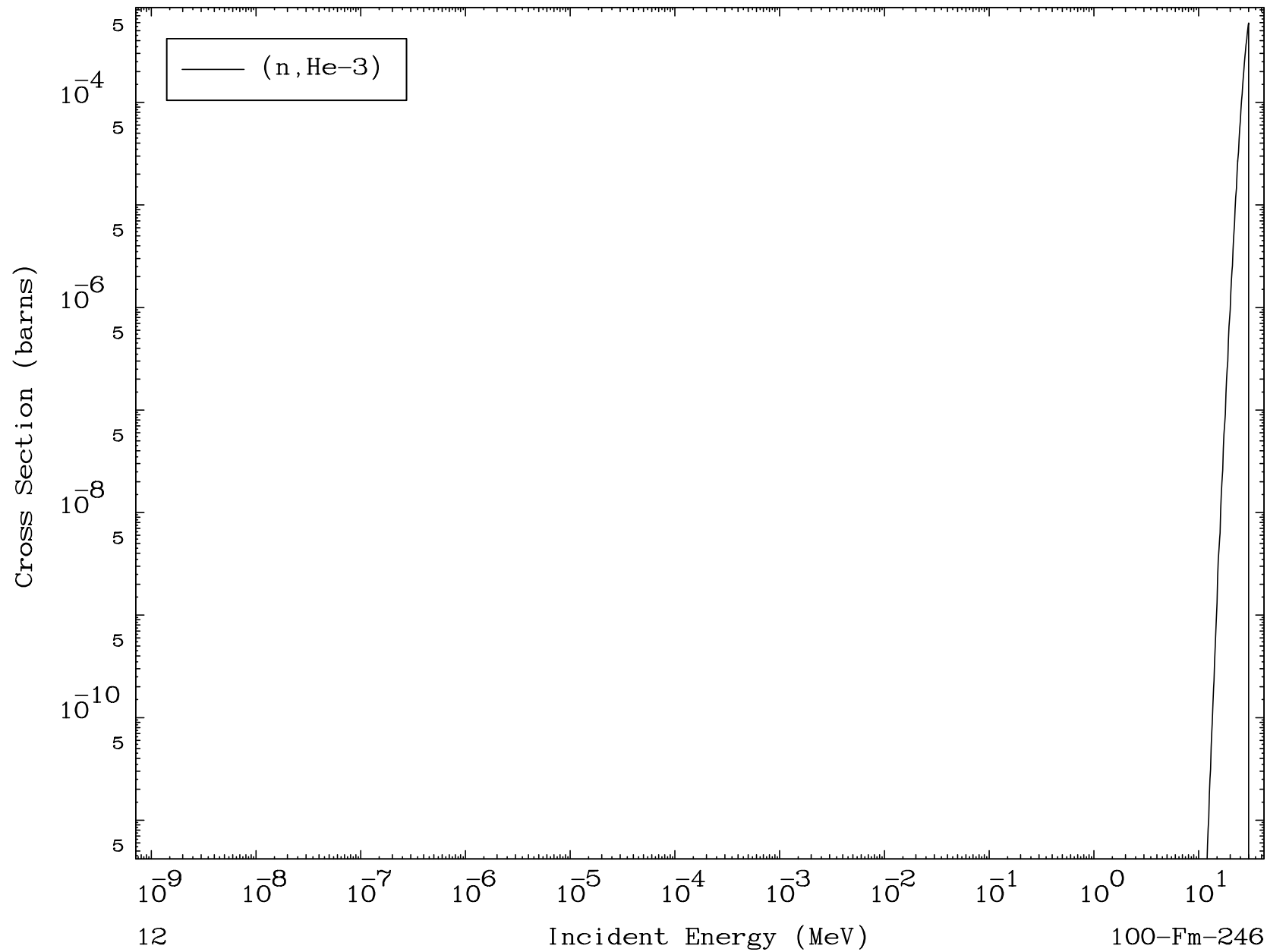
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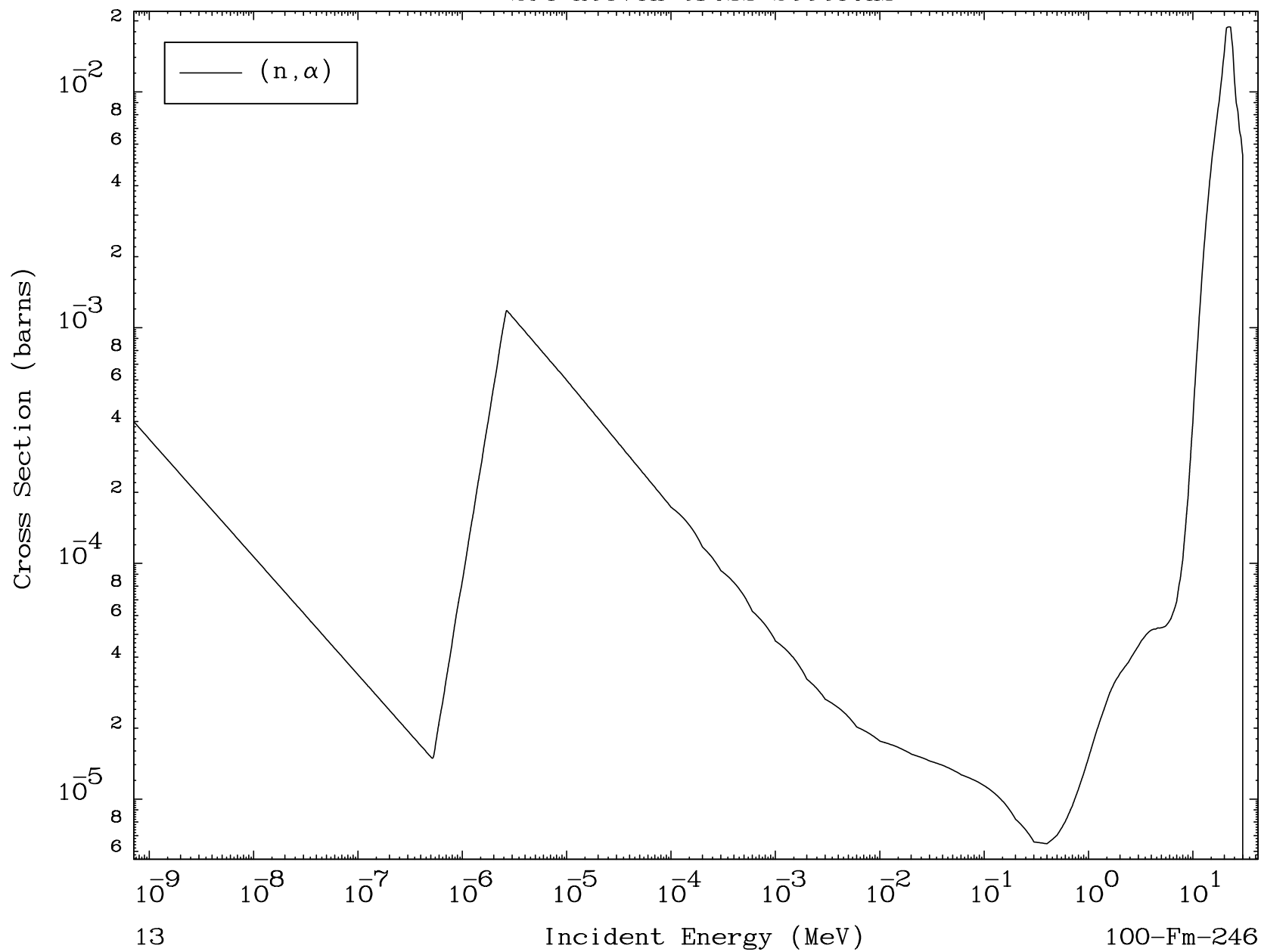


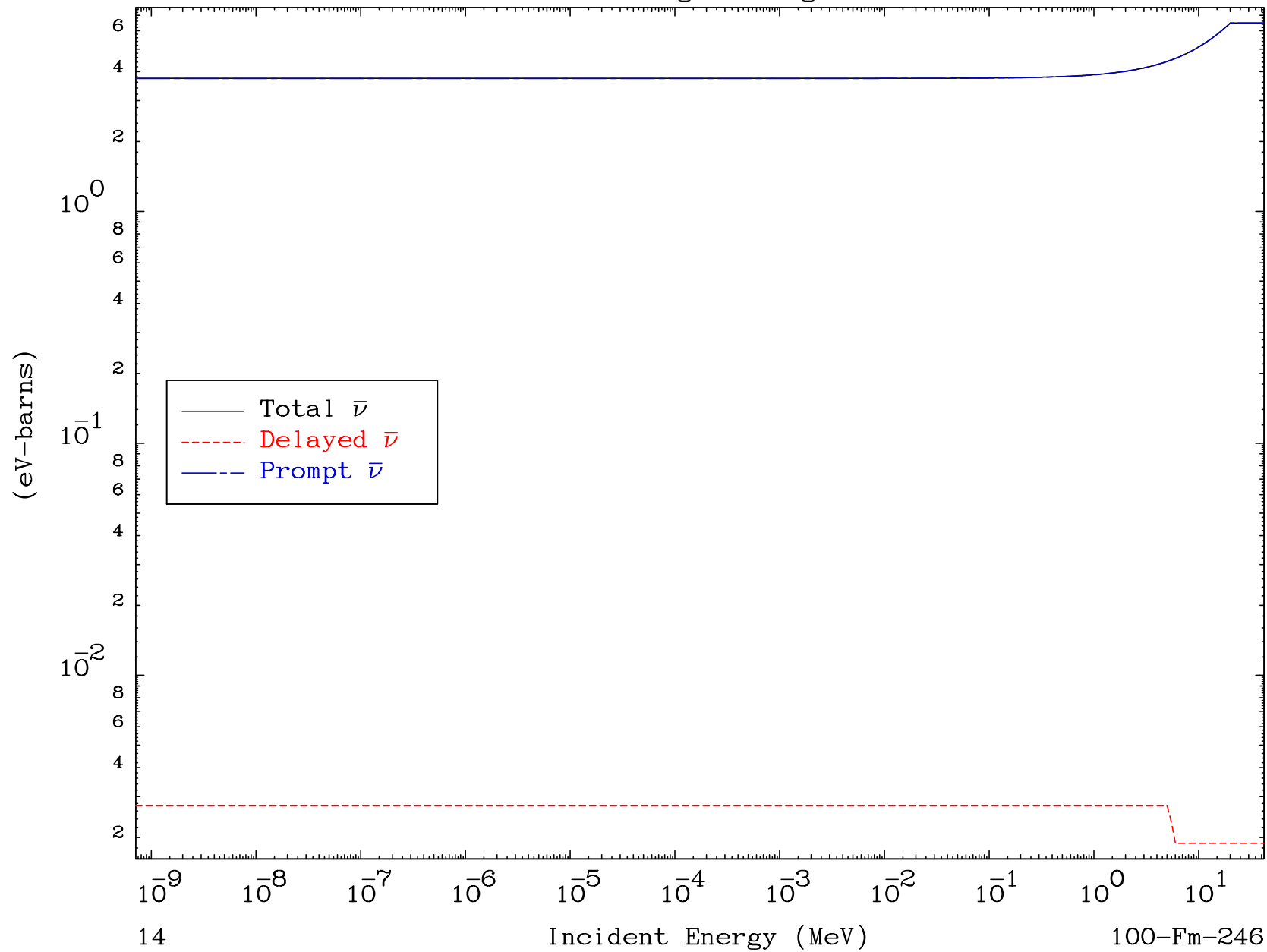
11

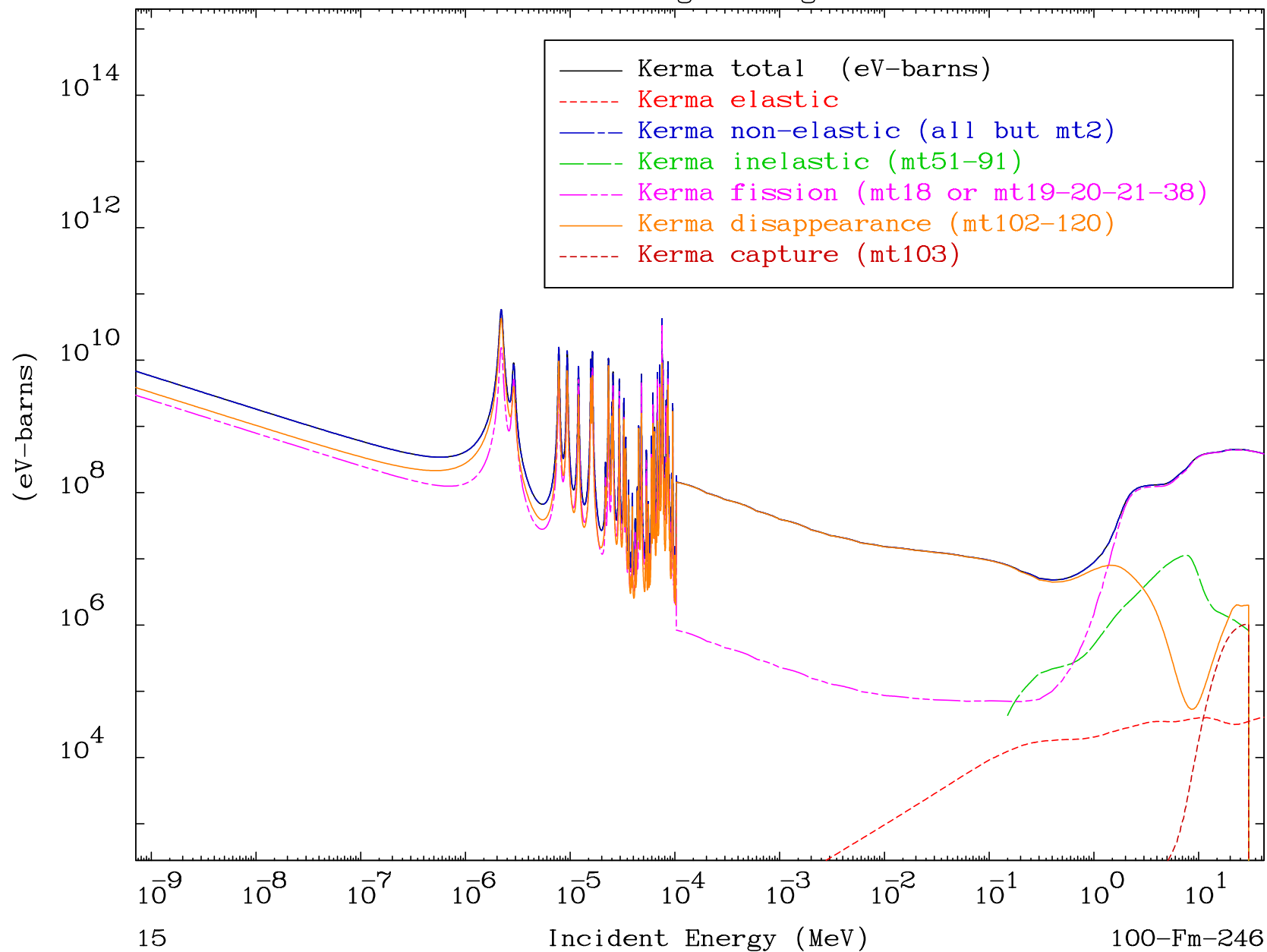
Incident Energy (MeV)

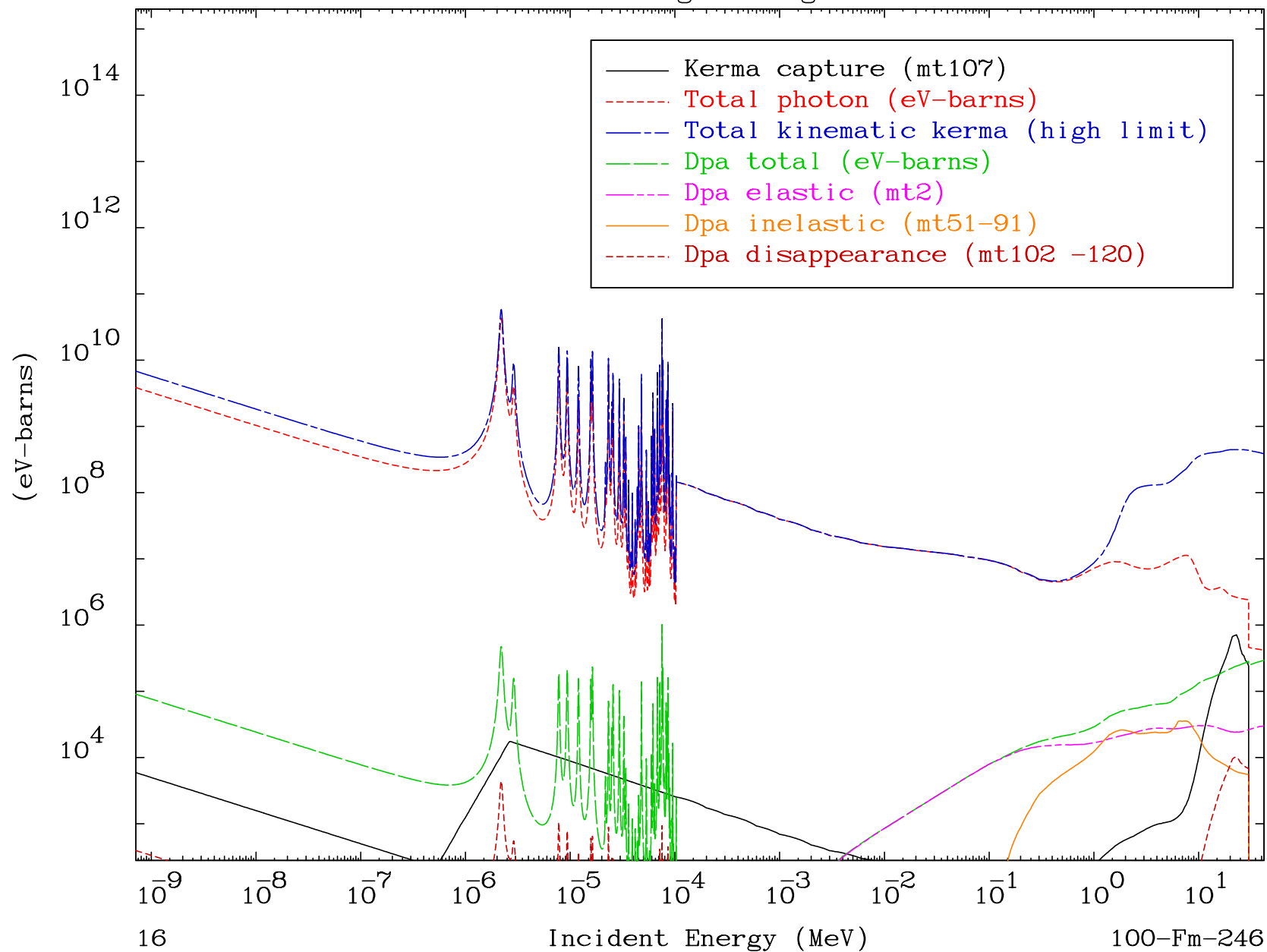
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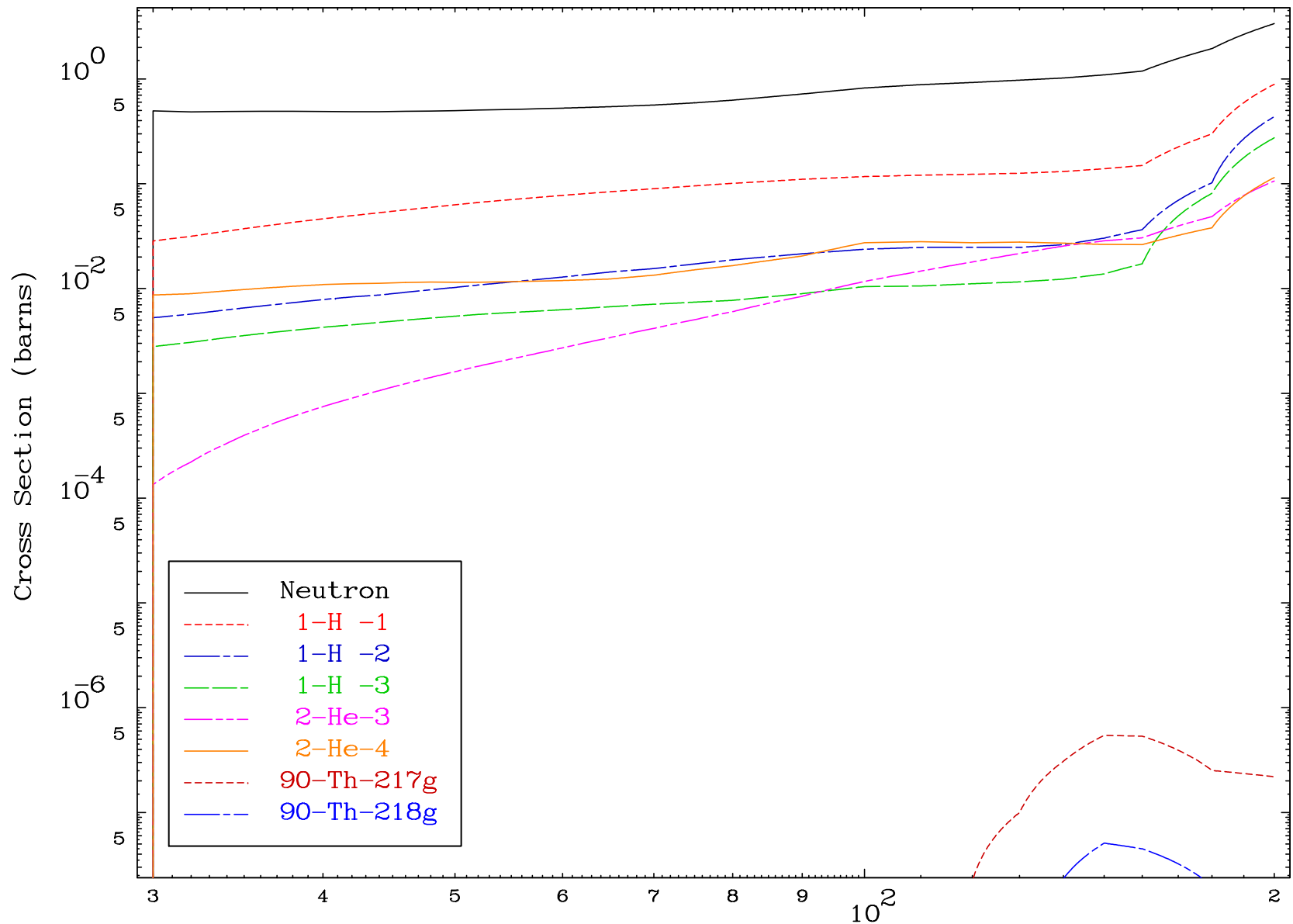


MAT 9927

(n,remainder)

100-Fm-246

Radionuclide Production Cross Section



17

Incident Energy (MeV)

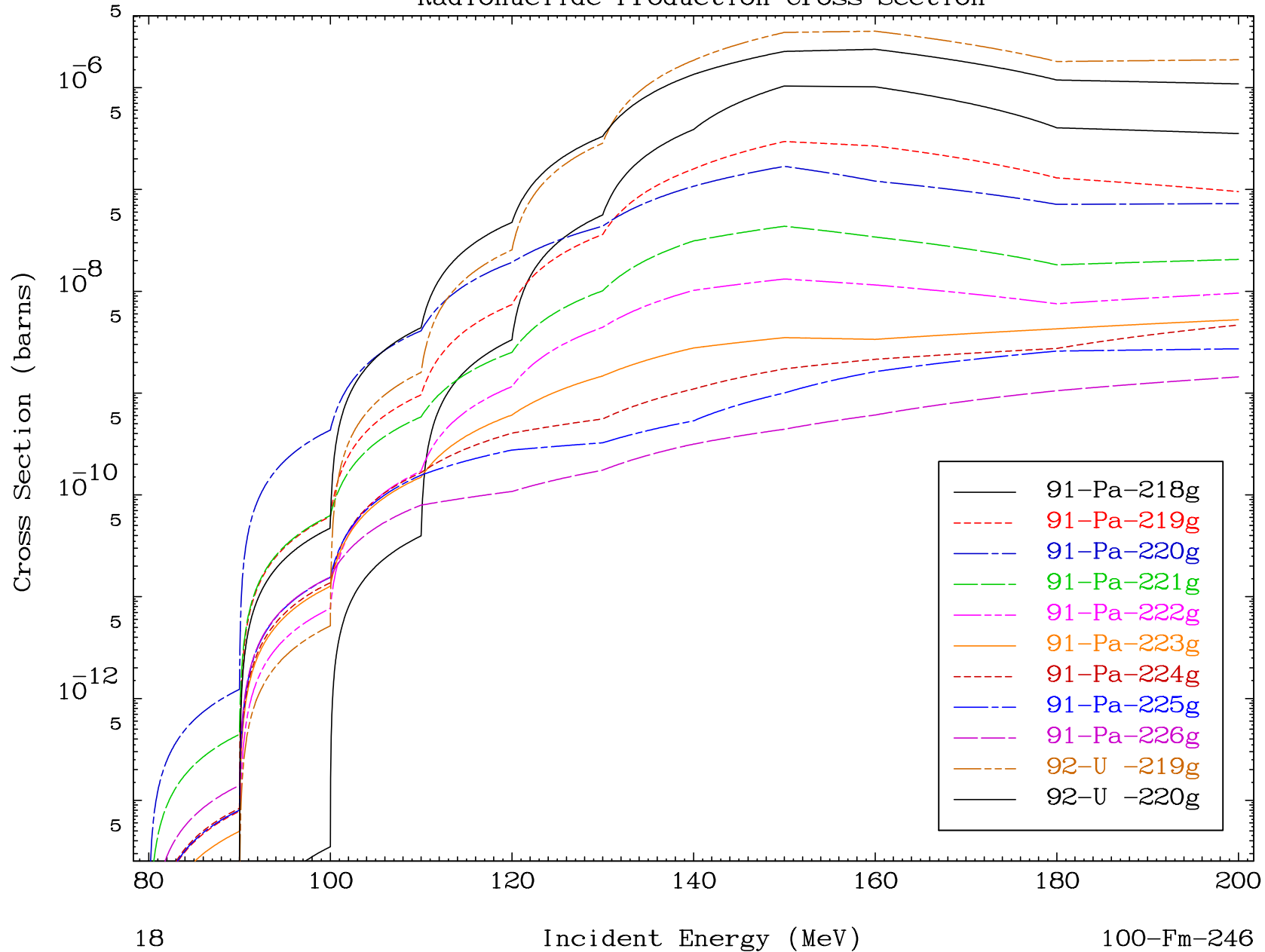
100-Fm-246

MAT 9927

(n,remainder)

100-Fm-246

Radionuclide Production Cross Section

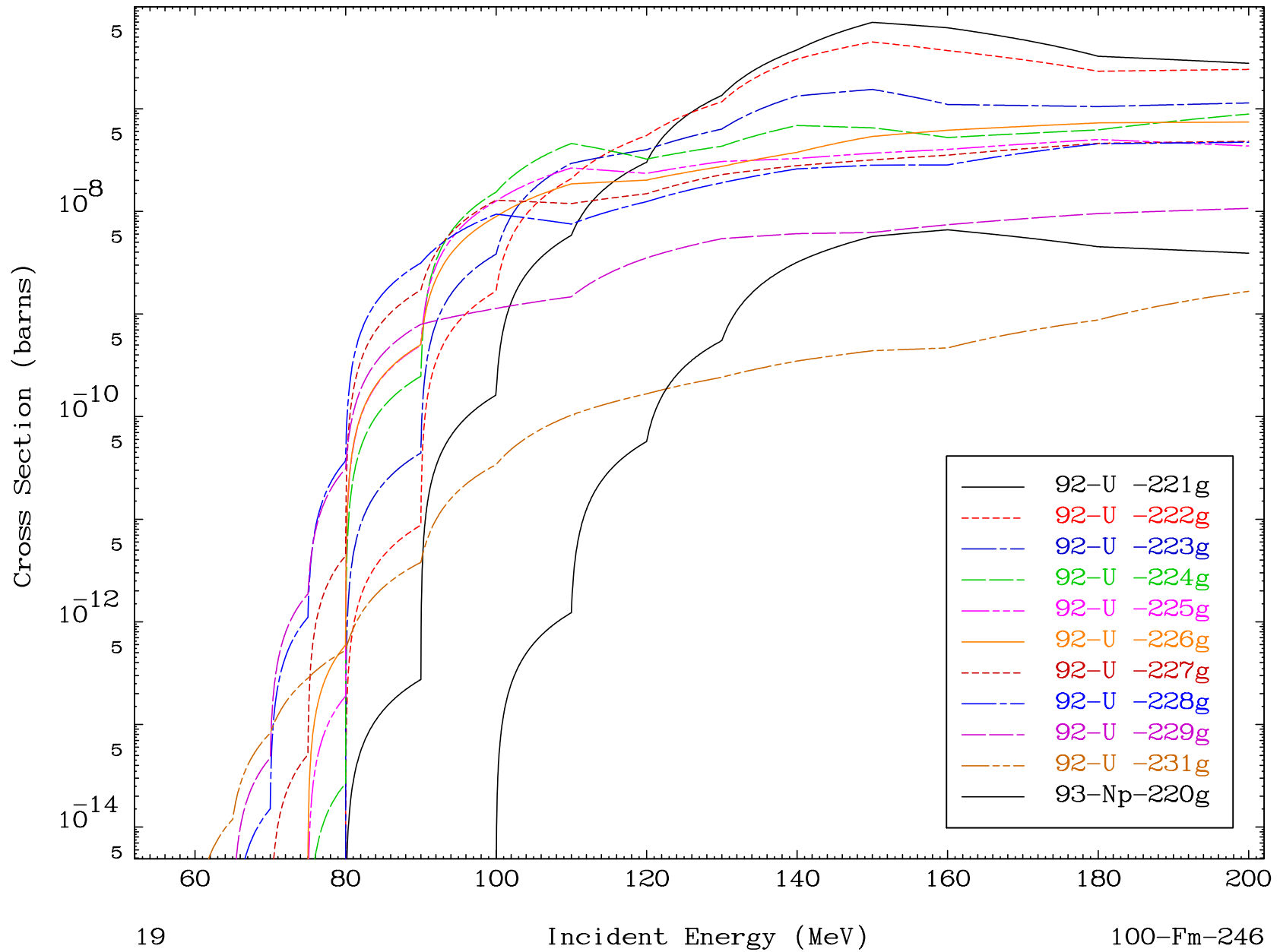


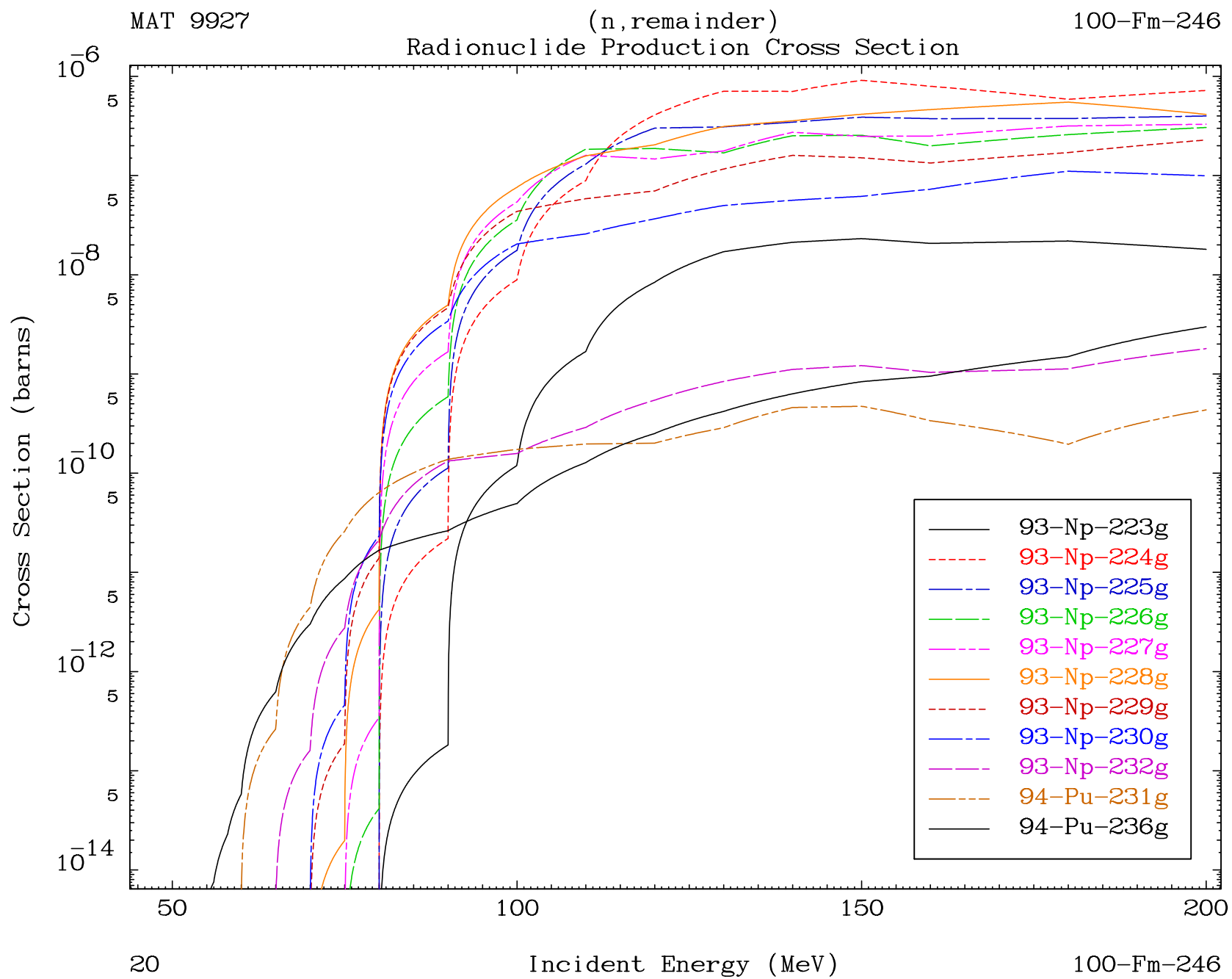
MAT 9927

(n,remainder)

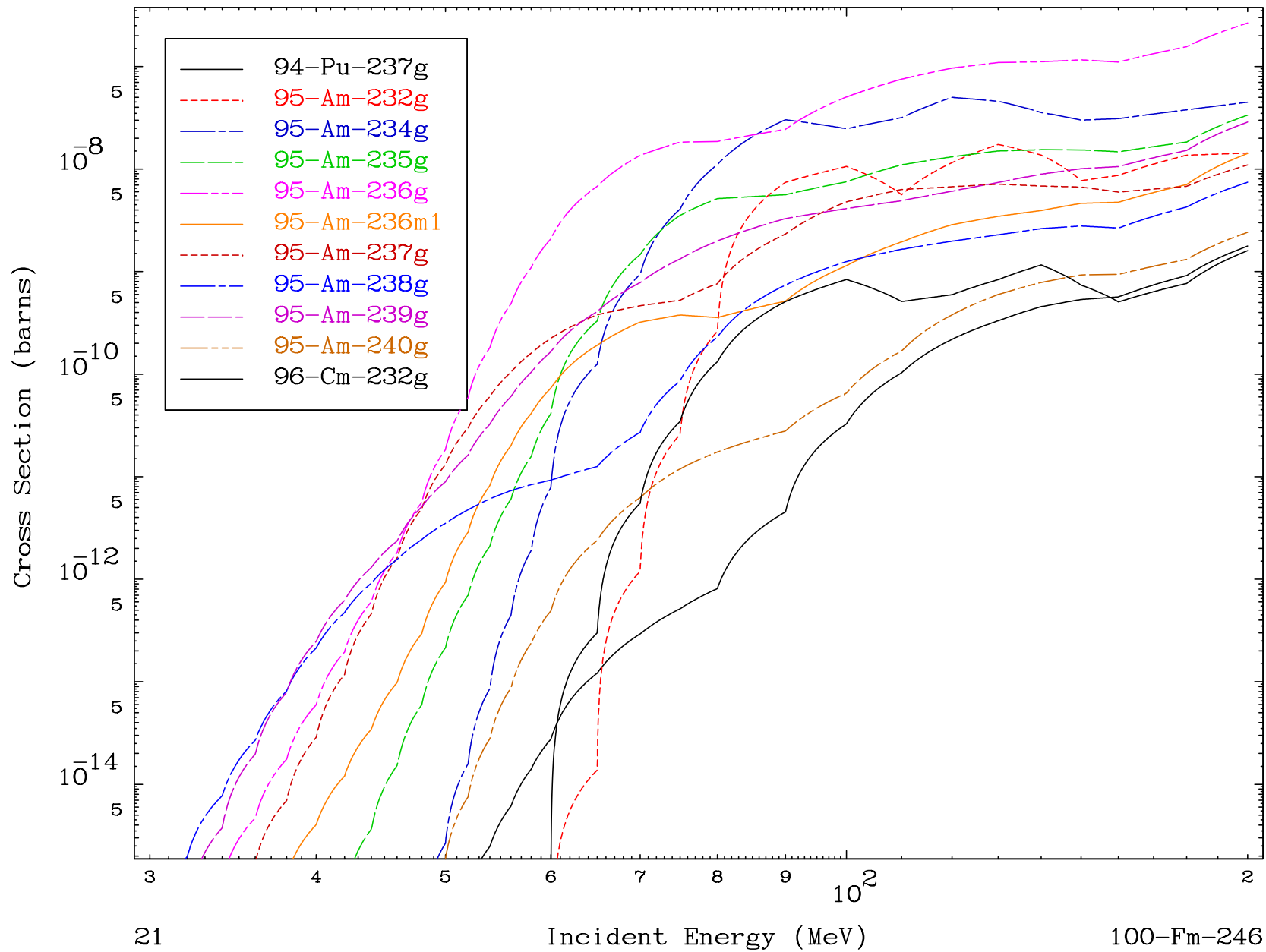
100-Fm-246

Radionuclide Production Cross Section





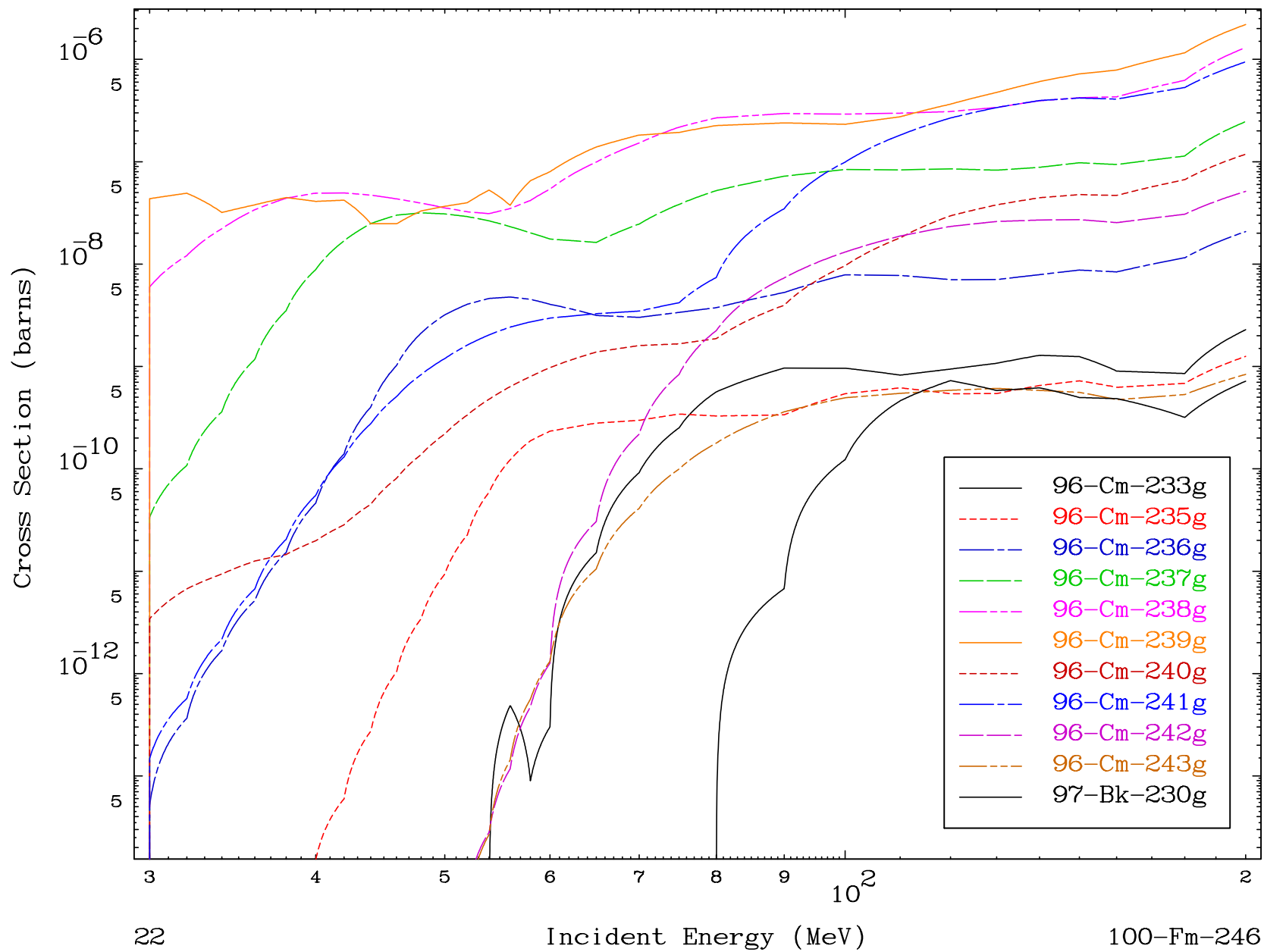
Radionuclide Production Cross Section



MAT 9927

(n,remainder)
Radionuclide Production Cross Section

100-Fm-246

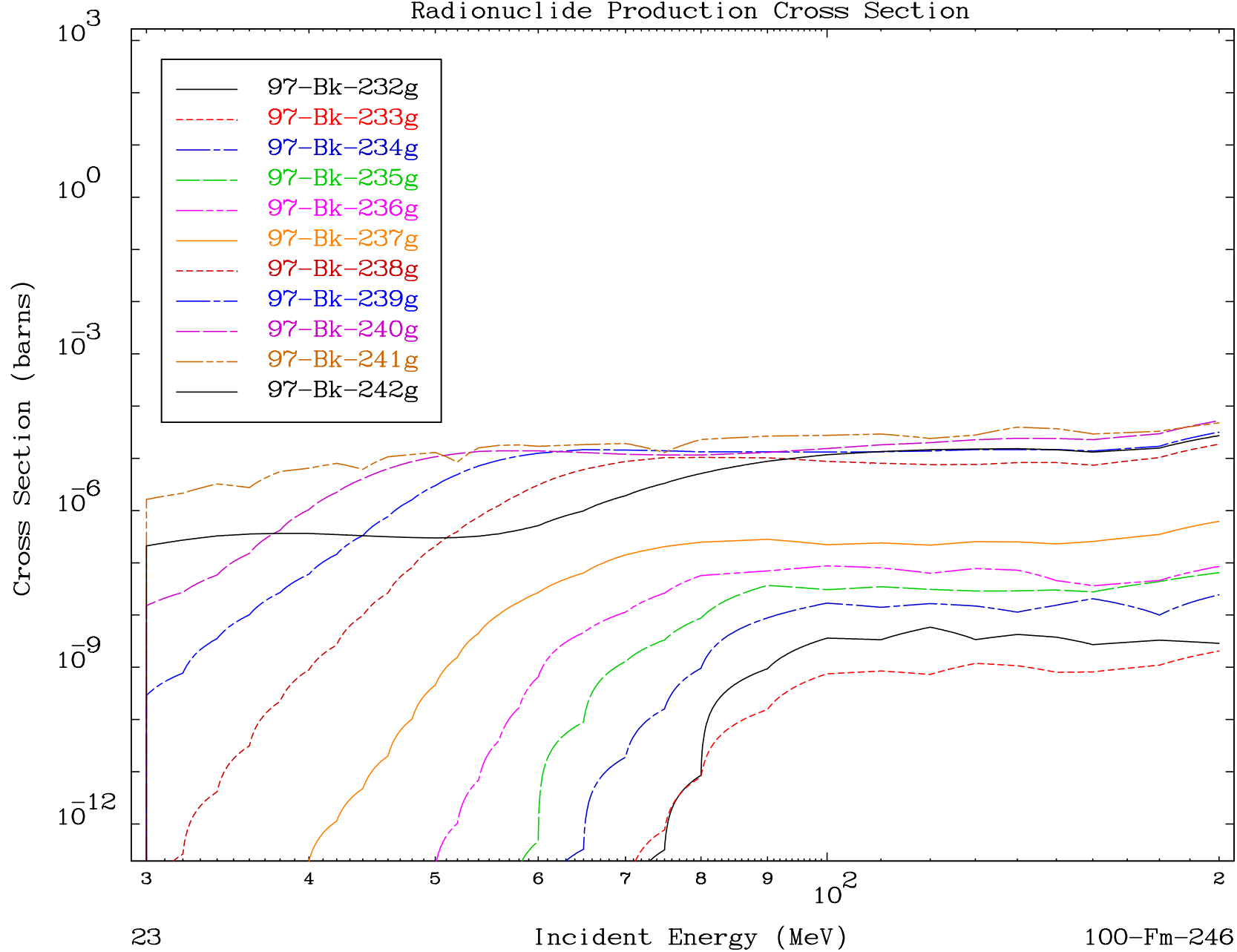


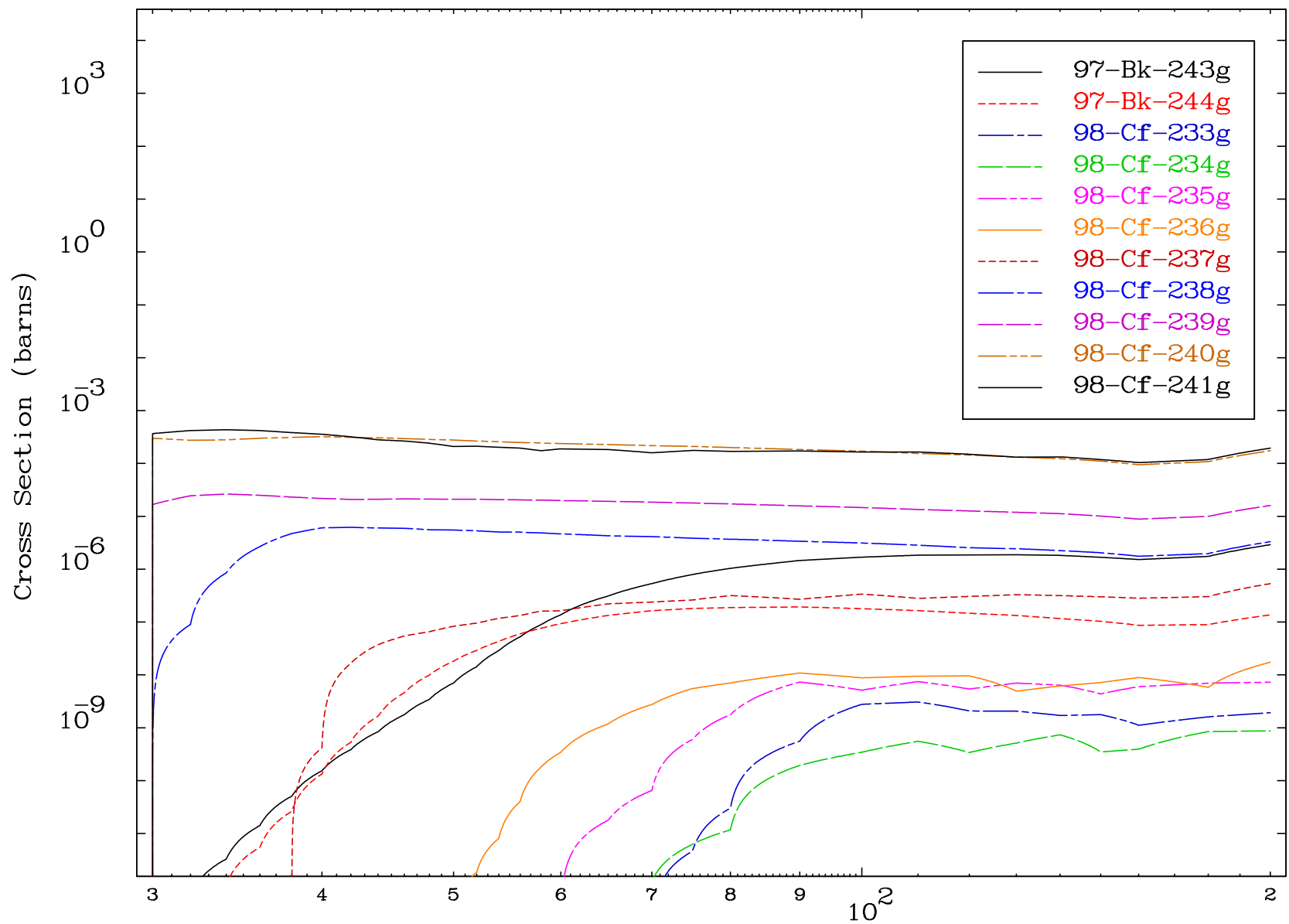
MAT 9927

(n,remainder)

100-Fm-246

Radionuclide Production Cross Section



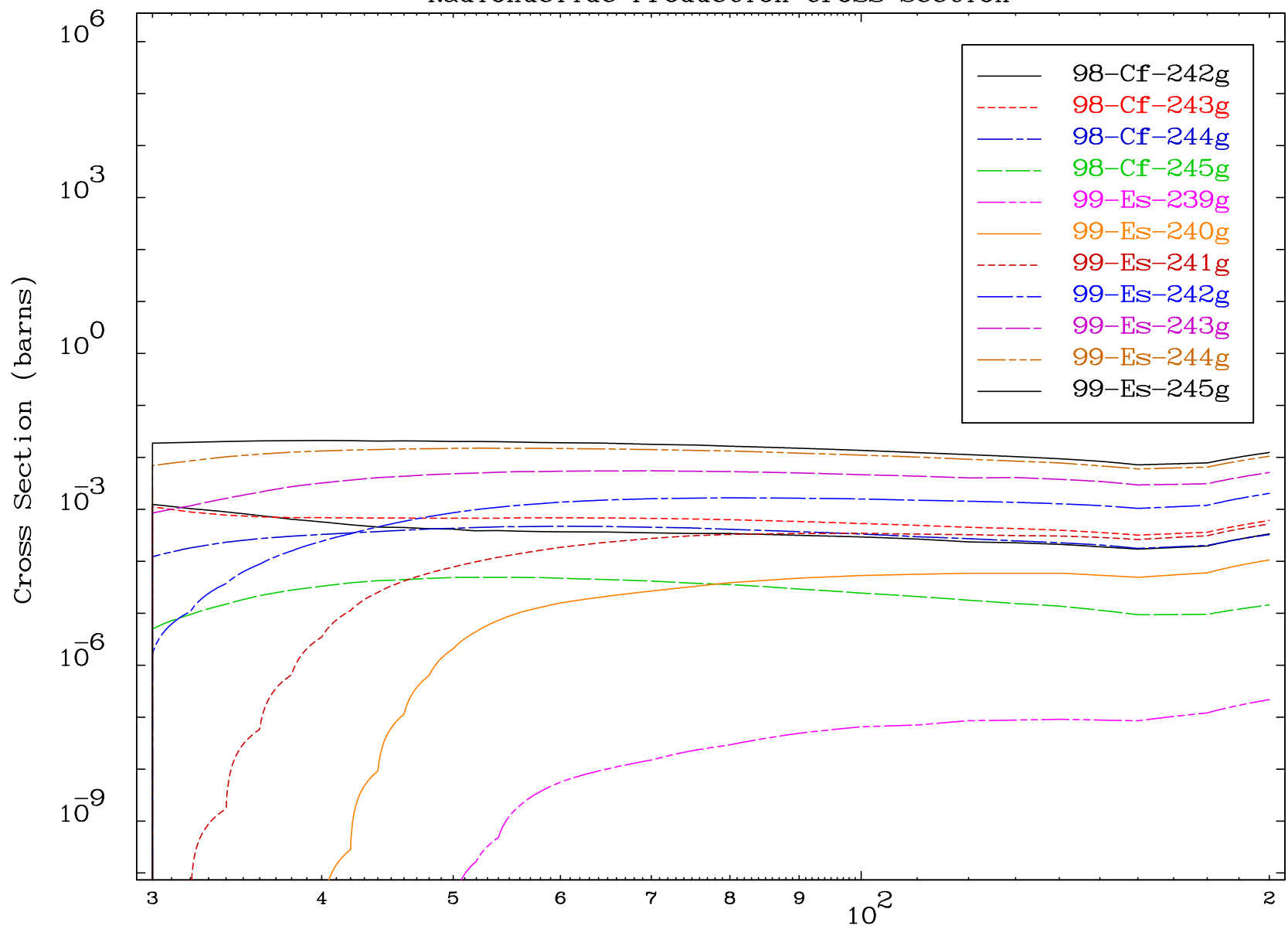


MAT 9927

(n,remainder)

100-Fm-246

Radionuclide Production Cross Section



25

Incident Energy (MeV)

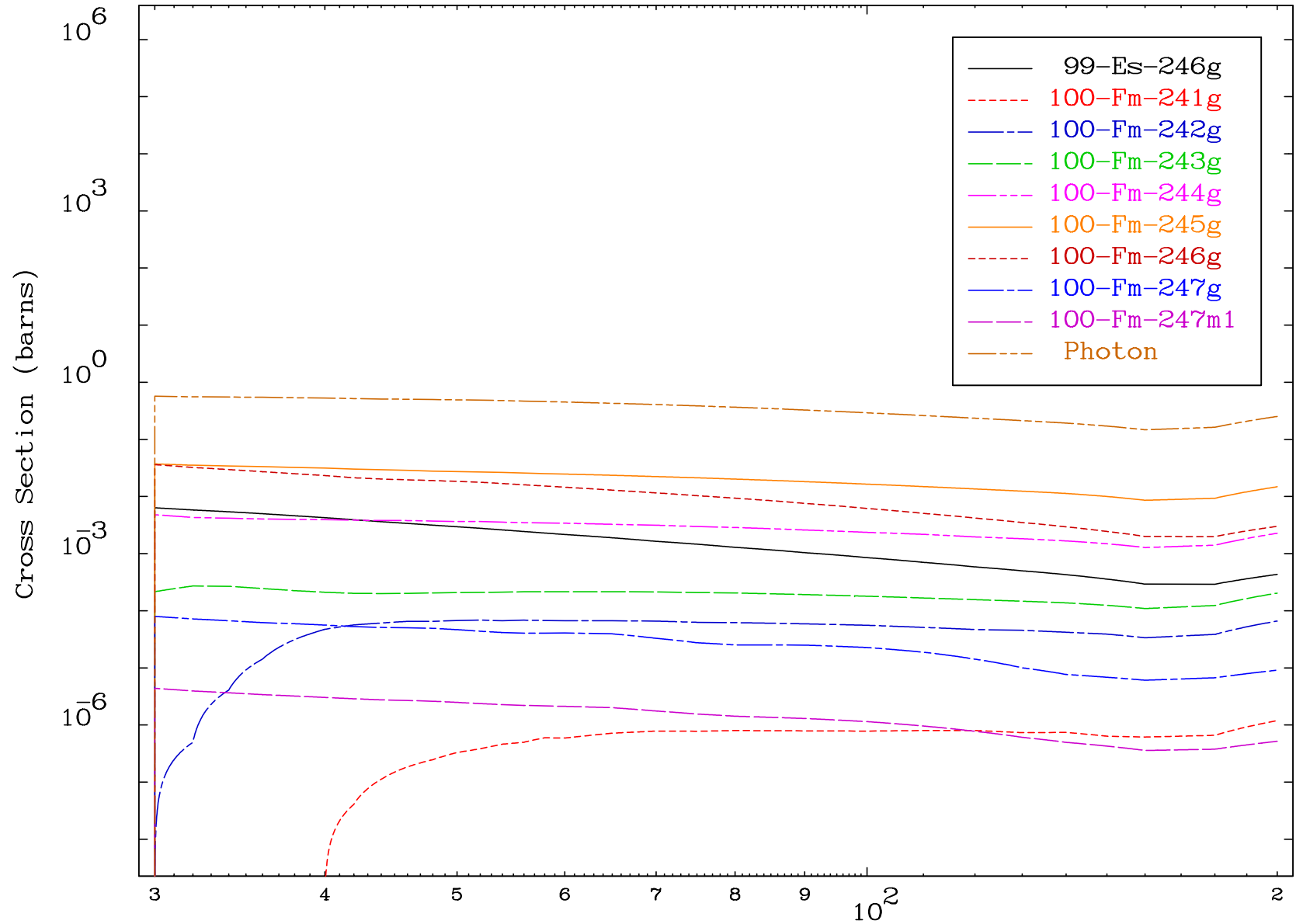
100-Fm-246

MAT 9927

(n,remainder)

100-Fm-246

Radionuclide Production Cross Section



26

Incident Energy (MeV)

100-Fm-246

MAT 9927

 (n, γ)

100-Fm-246

Radionuclide Production Cross Section

