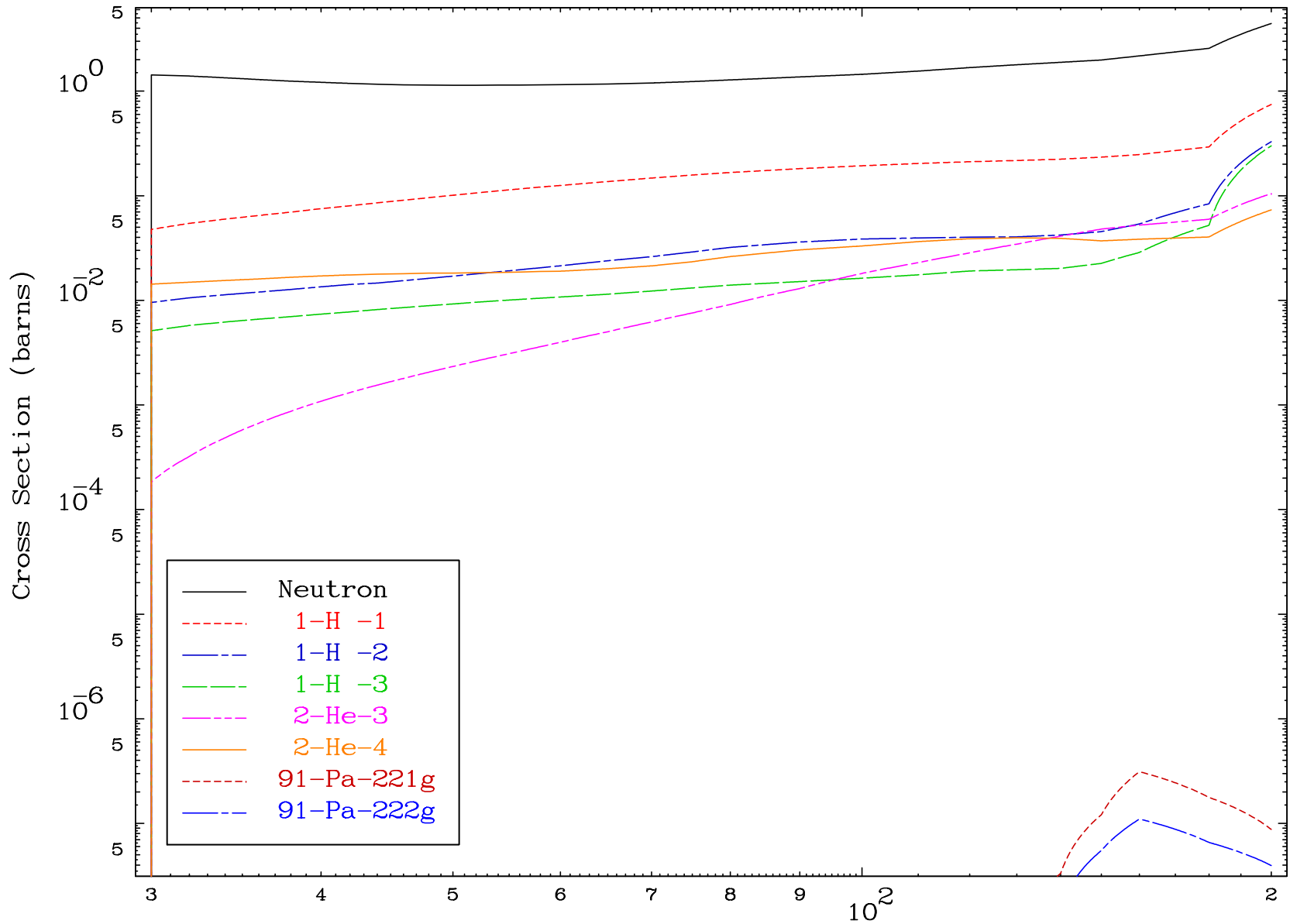
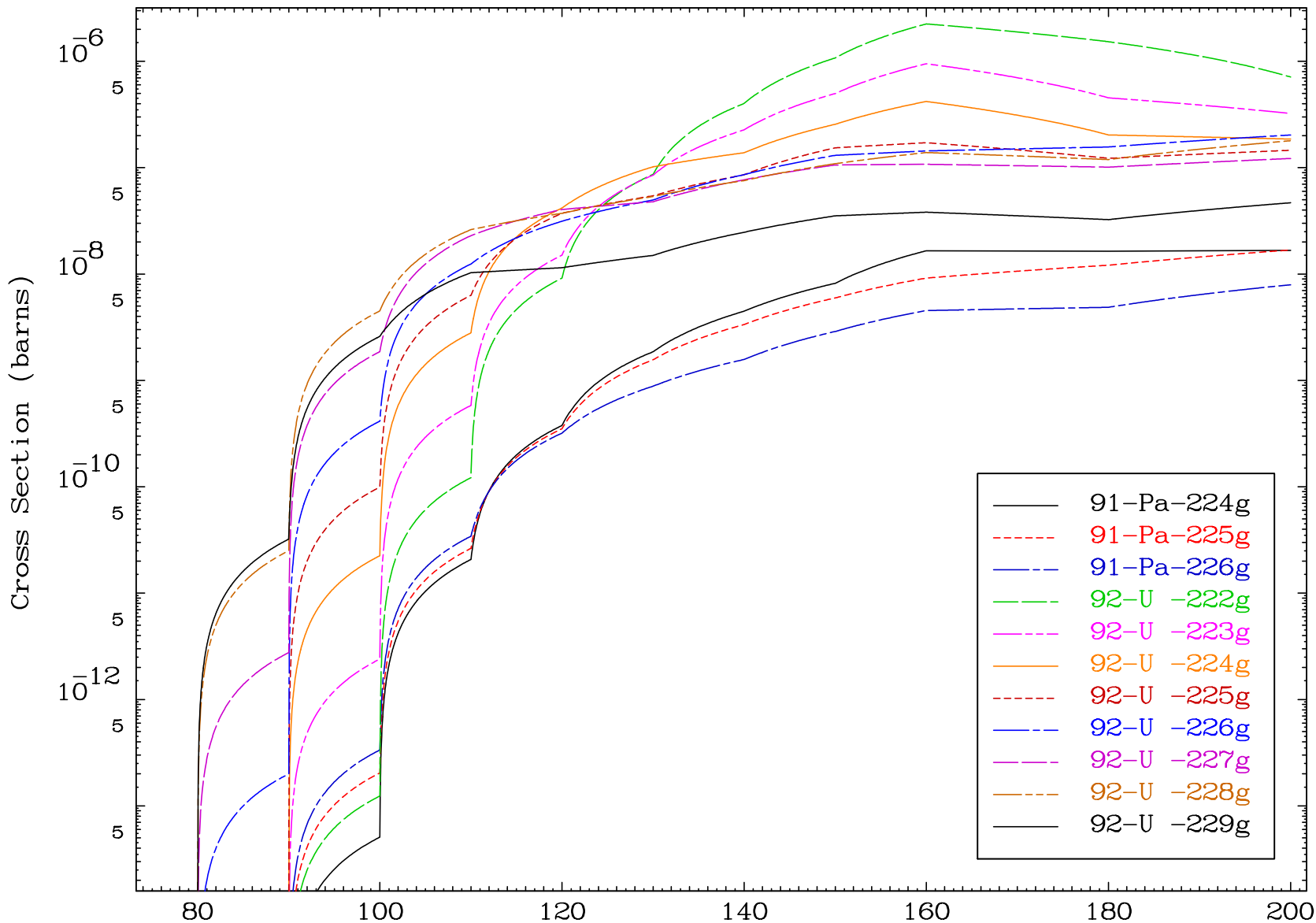


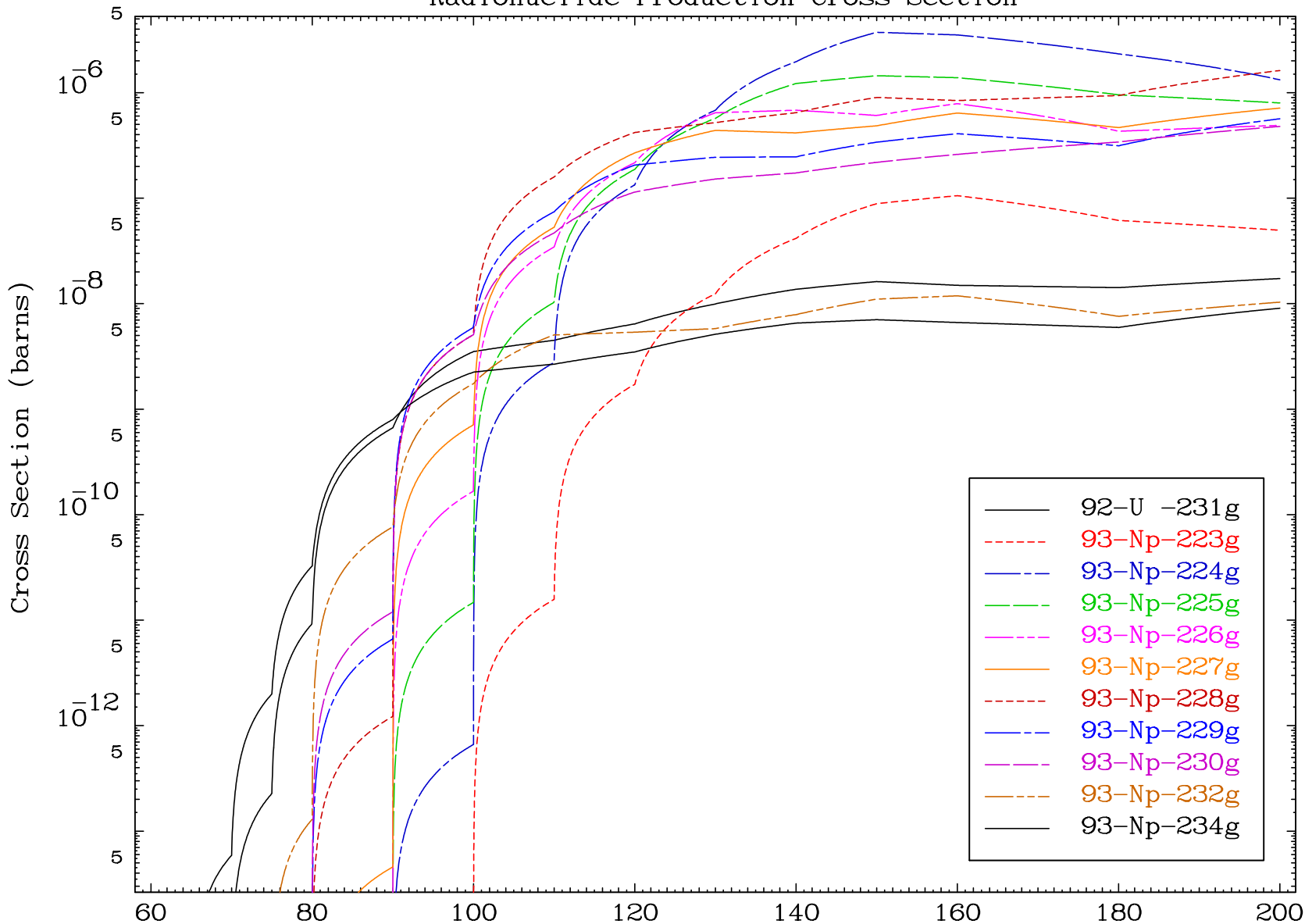
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

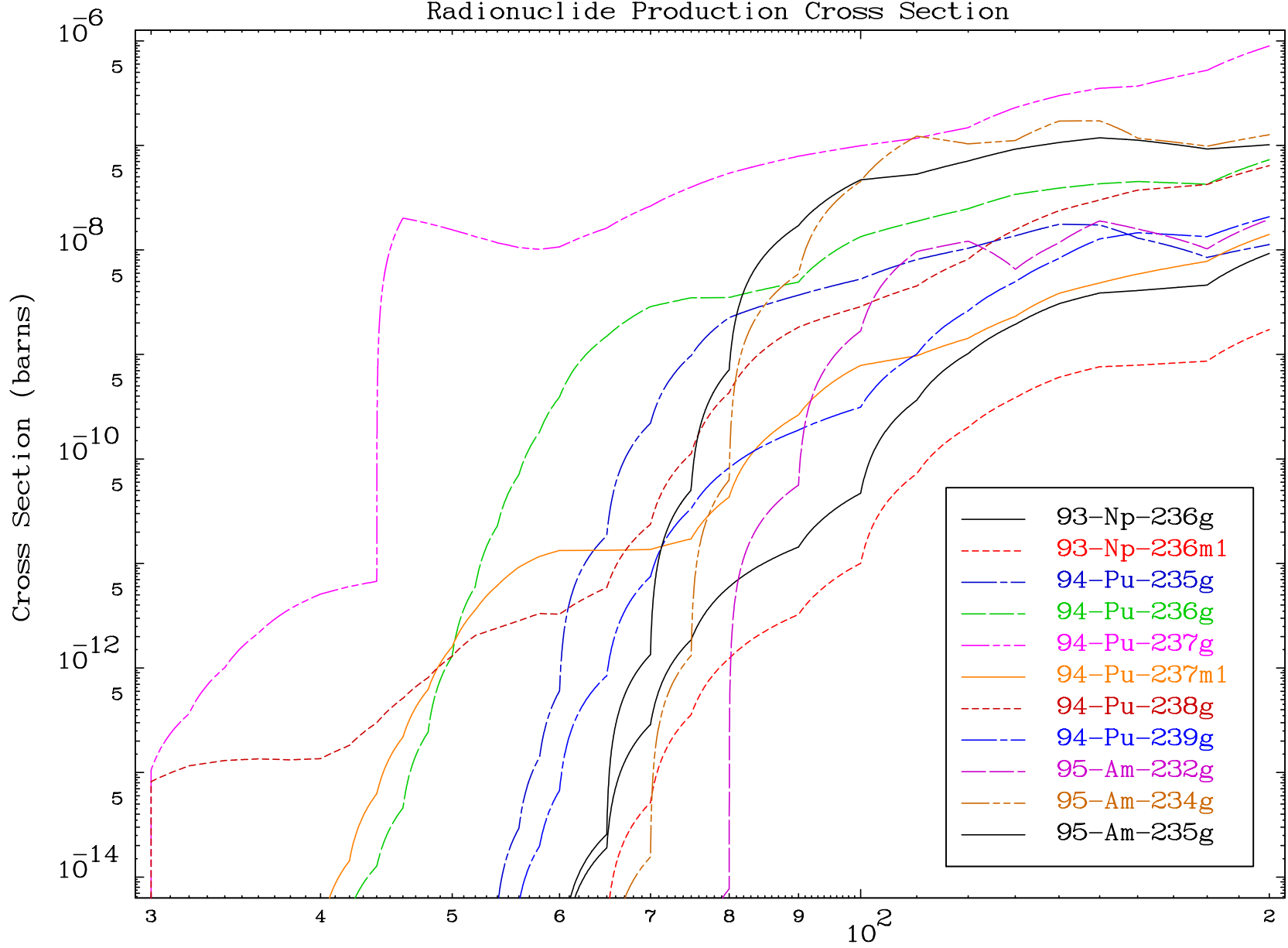


MAT 9930

(n,remainder)

100-Fm-249

Radionuclide Production Cross Section



22

Incident Energy (MeV)

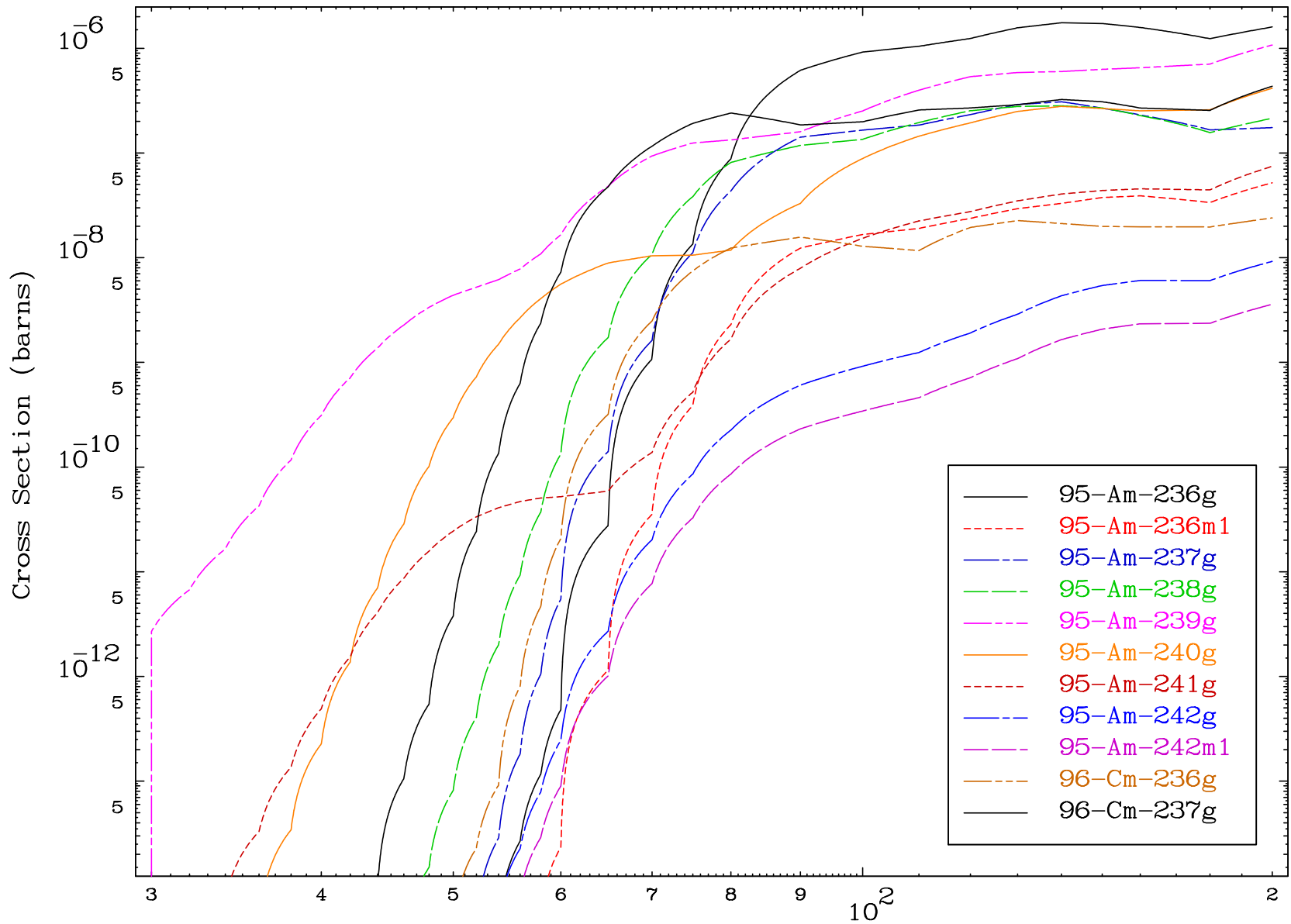
100-Fm-249

MAT 9930

(n,remainder)

100-Fm-249

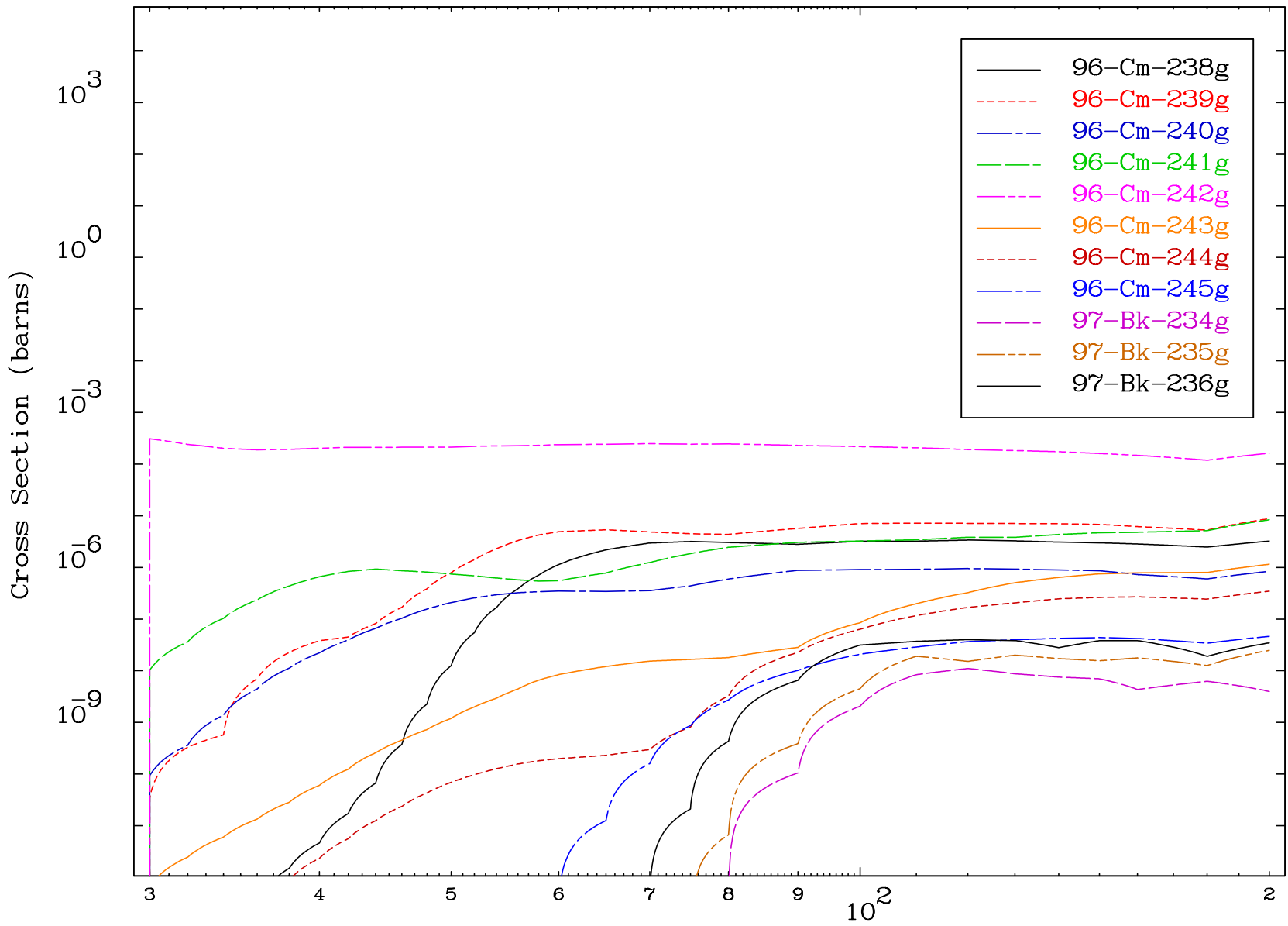
Radionuclide Production Cross Section



23

Incident Energy (MeV)

100-Fm-249

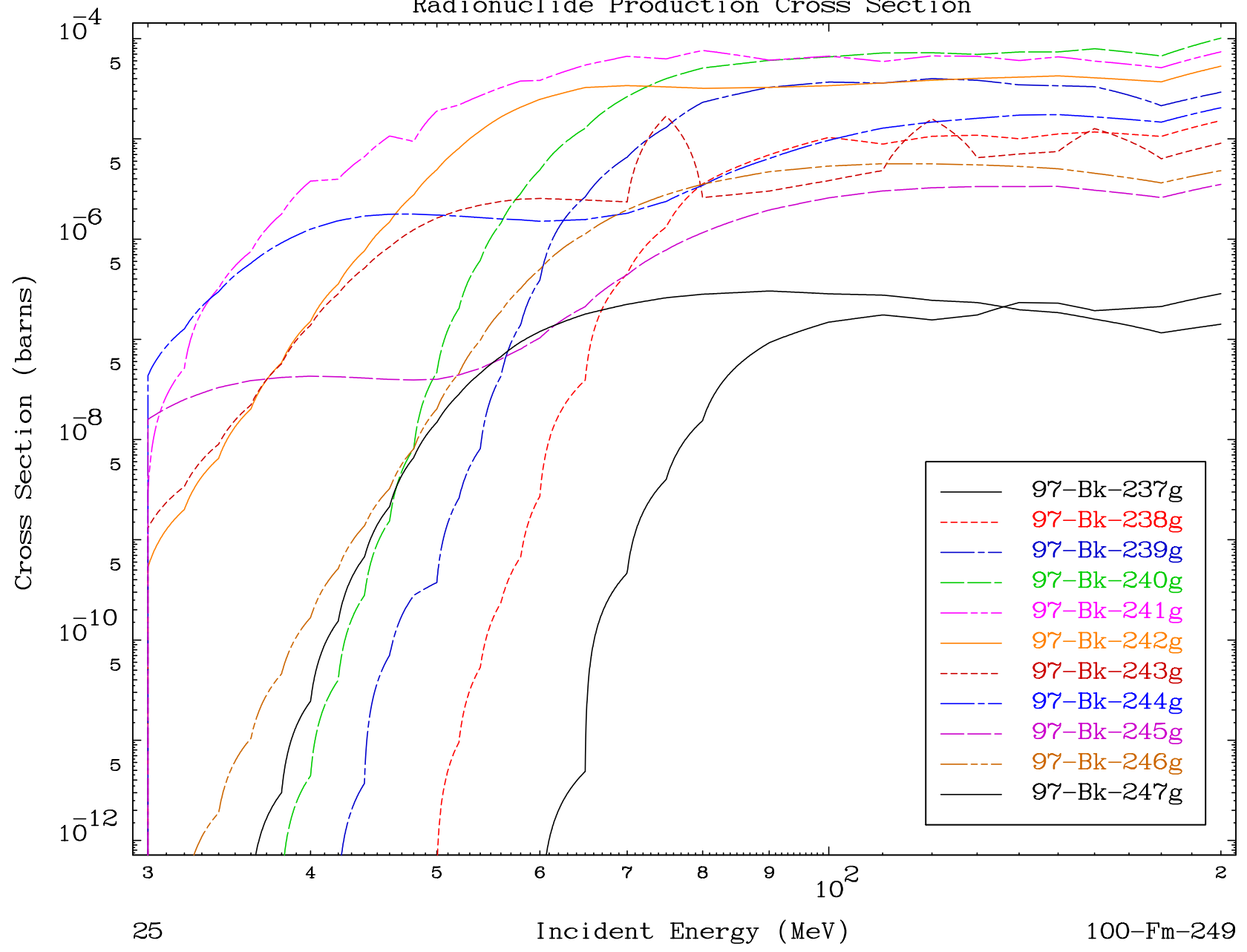


MAT 9930

(n,remainder)

100-Fm-249

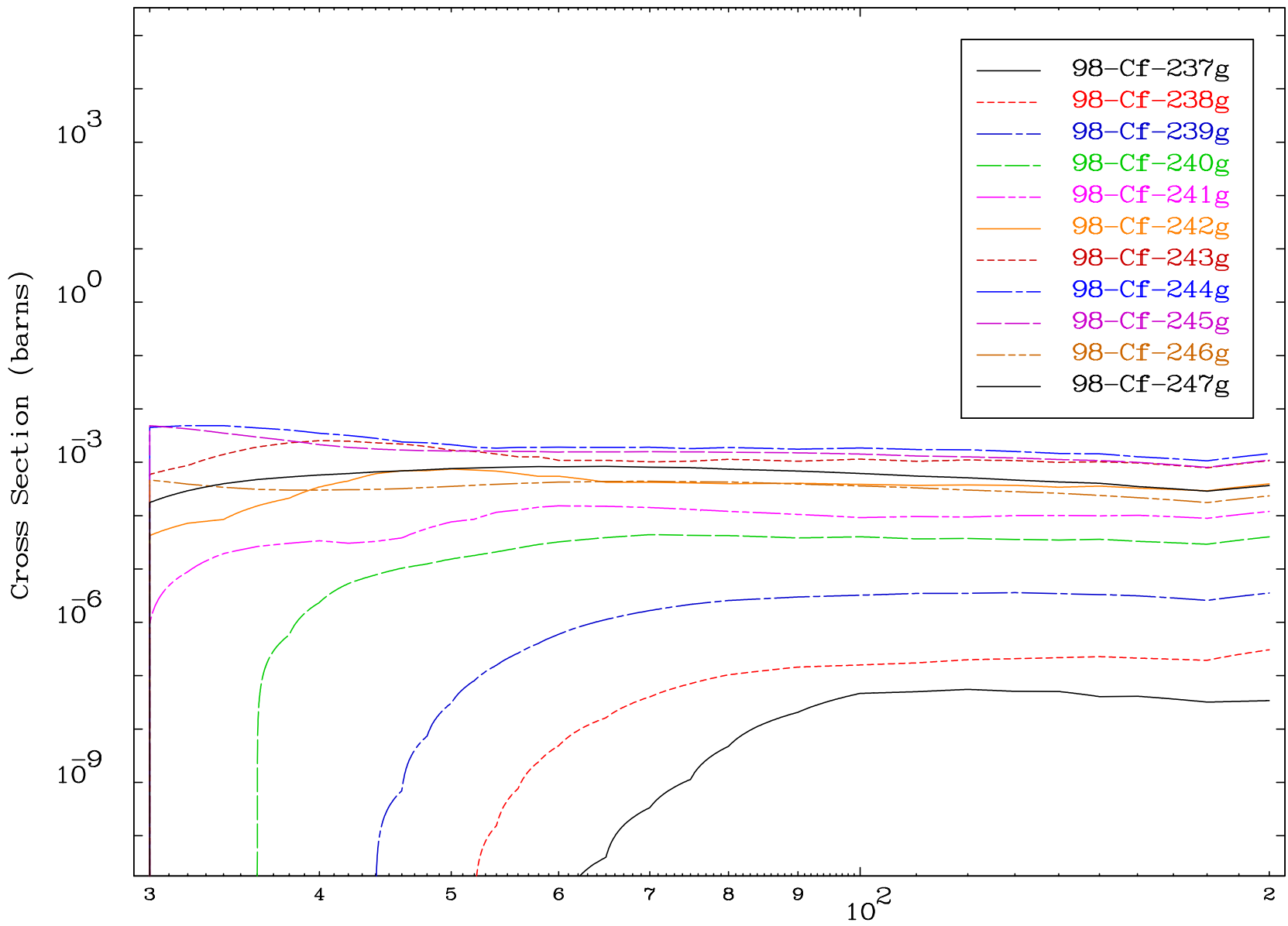
Radionuclide Production Cross Section

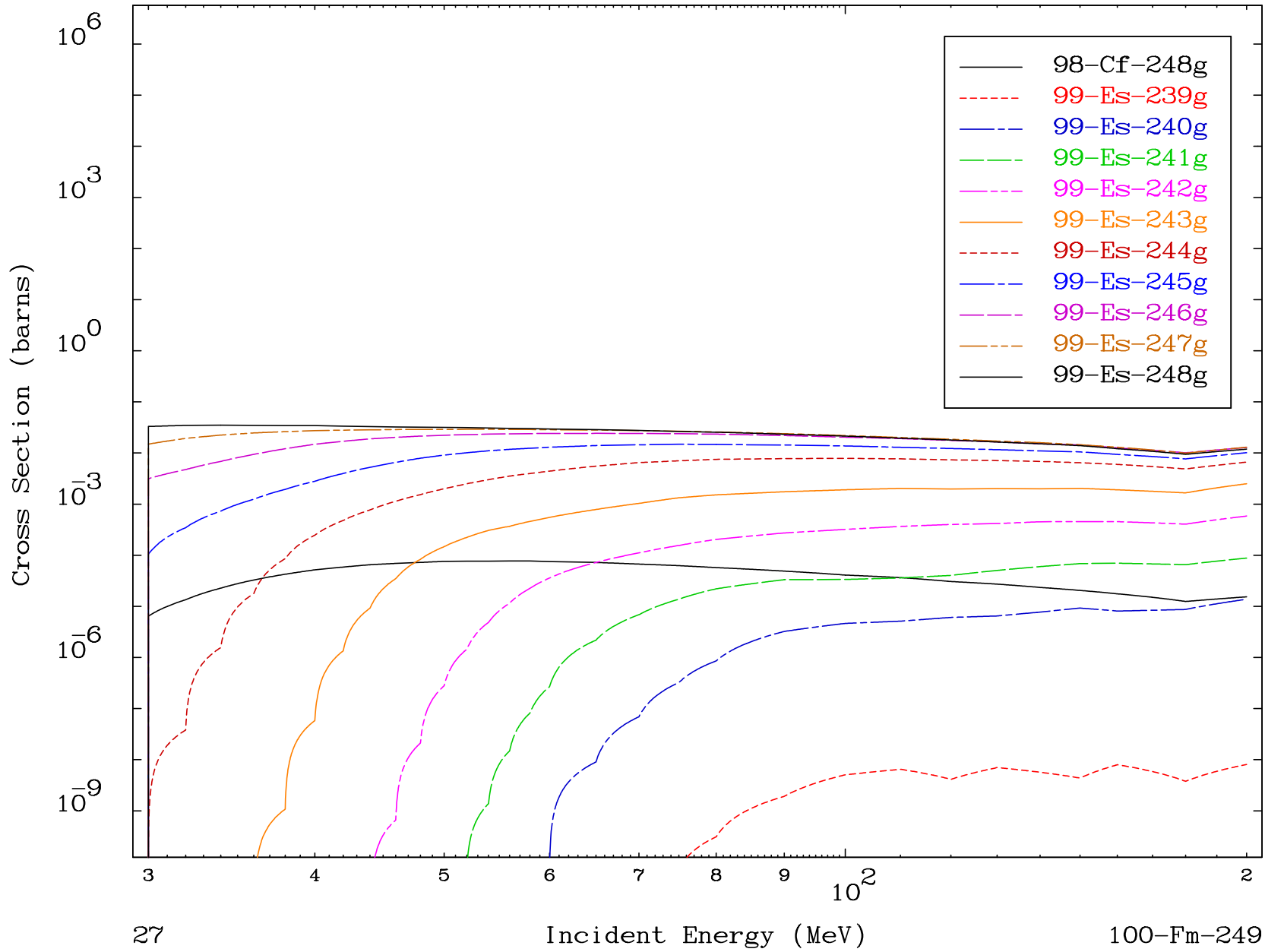


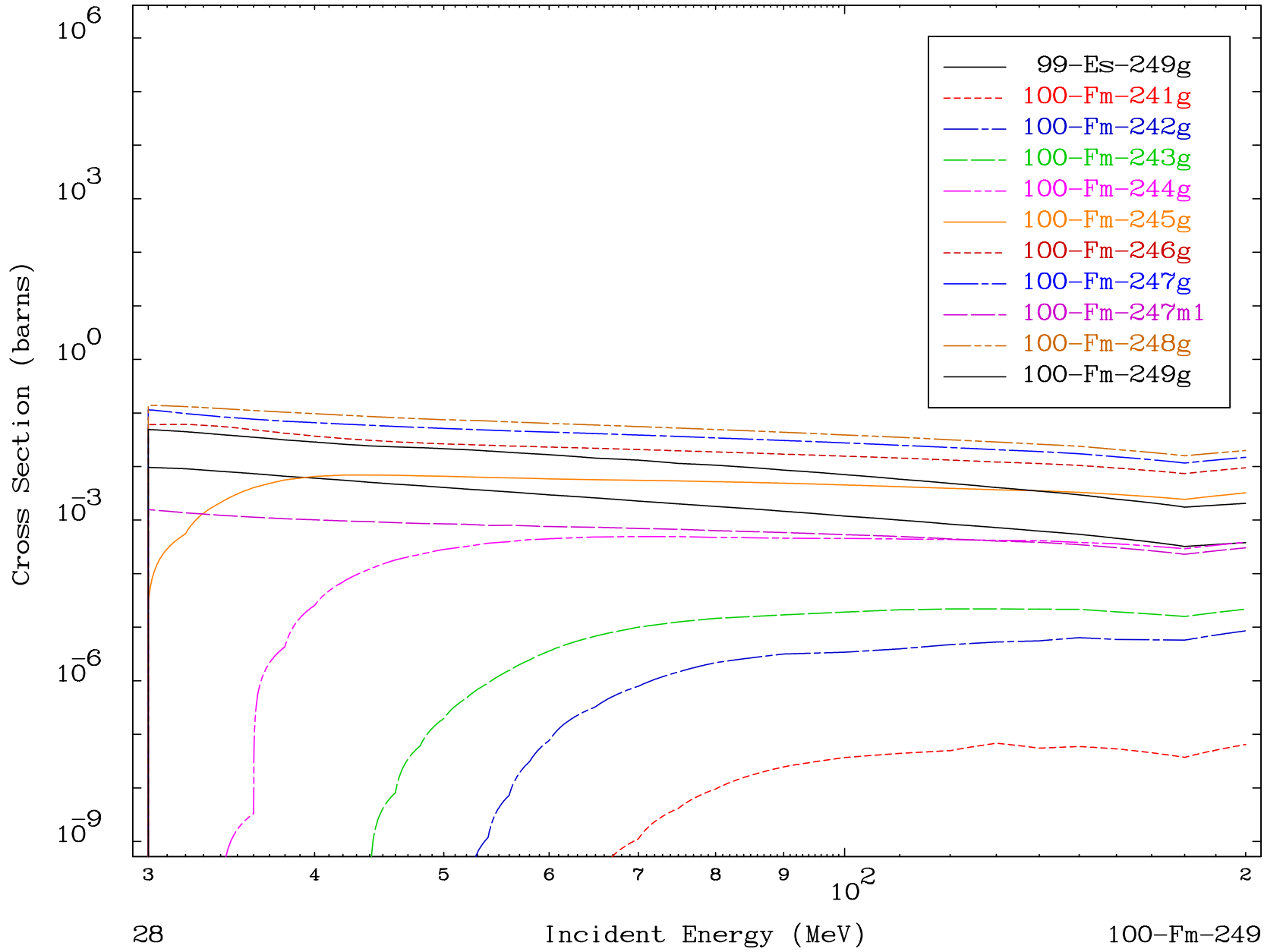
25

Incident Energy (MeV)

100-Fm-249





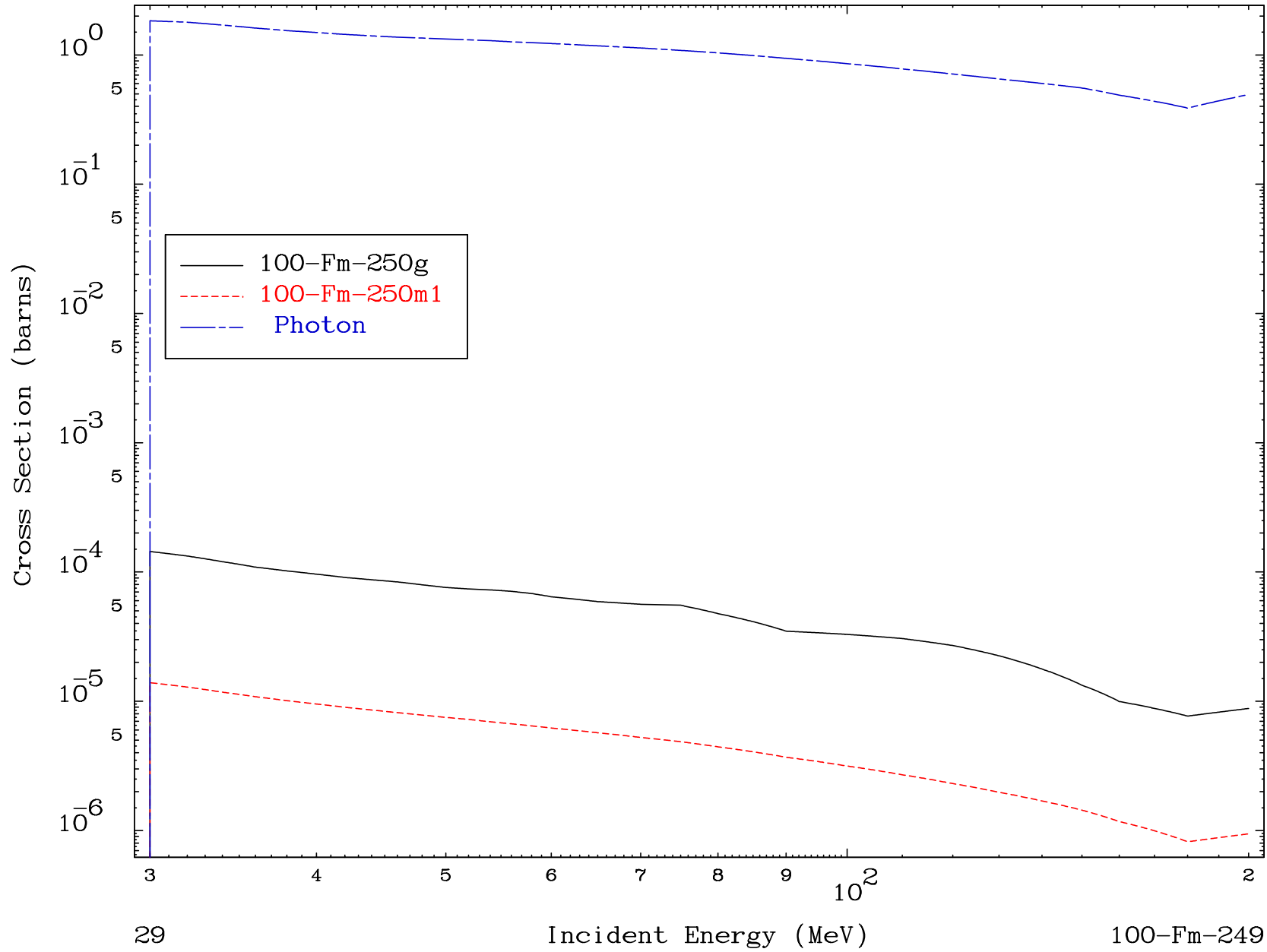


MAT 9930

(n,remainder)

100-Fm-249

Radionuclide Production Cross Section



29

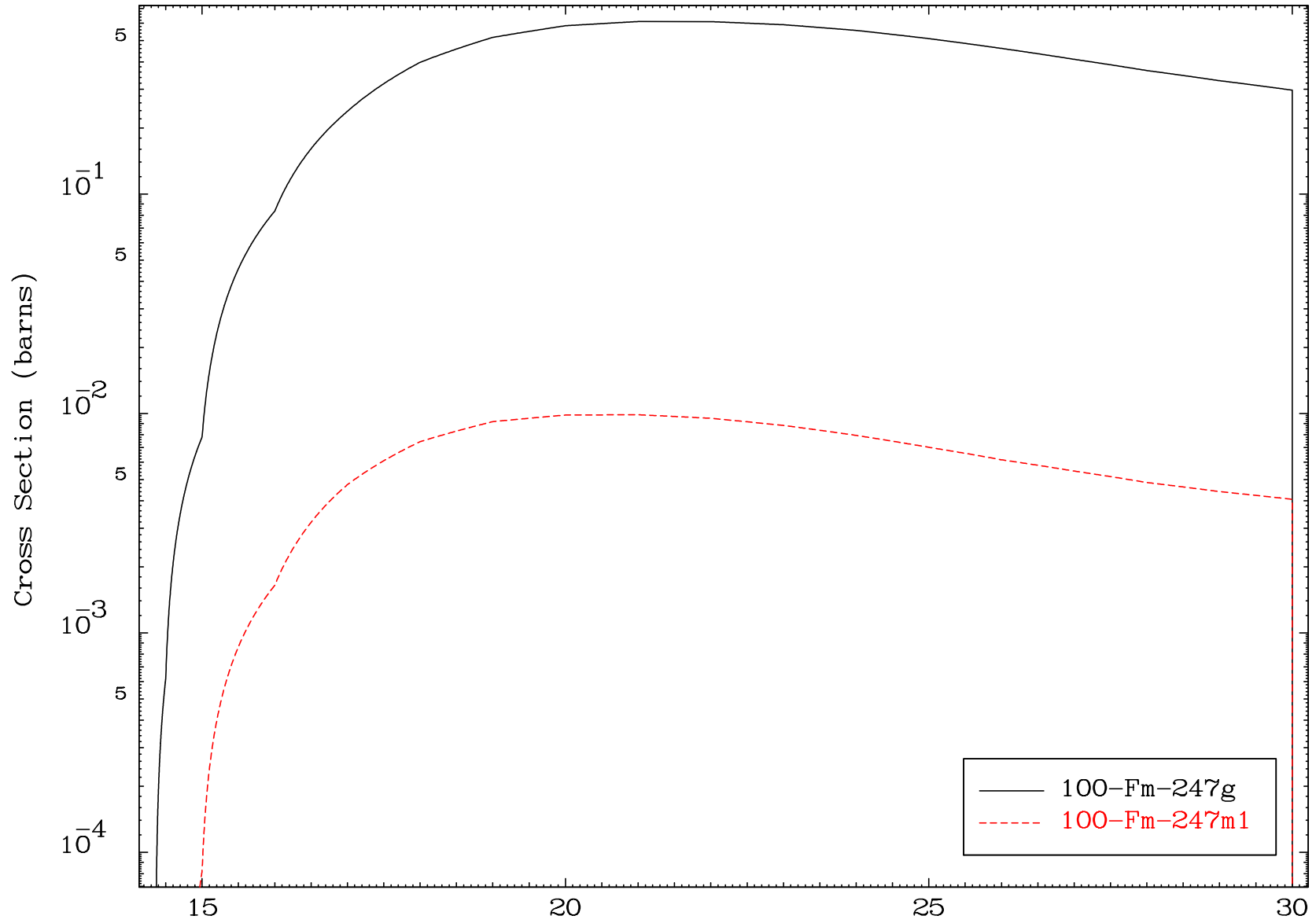
100-Fm-249

MAT 9930

(n,3n)

100-Fm-249

Radionuclide Production Cross Section



30

Incident Energy (MeV)

100-Fm-249

MAT 9930

(n, γ)

100-Fm-249

Radionuclide Production Cross Section

