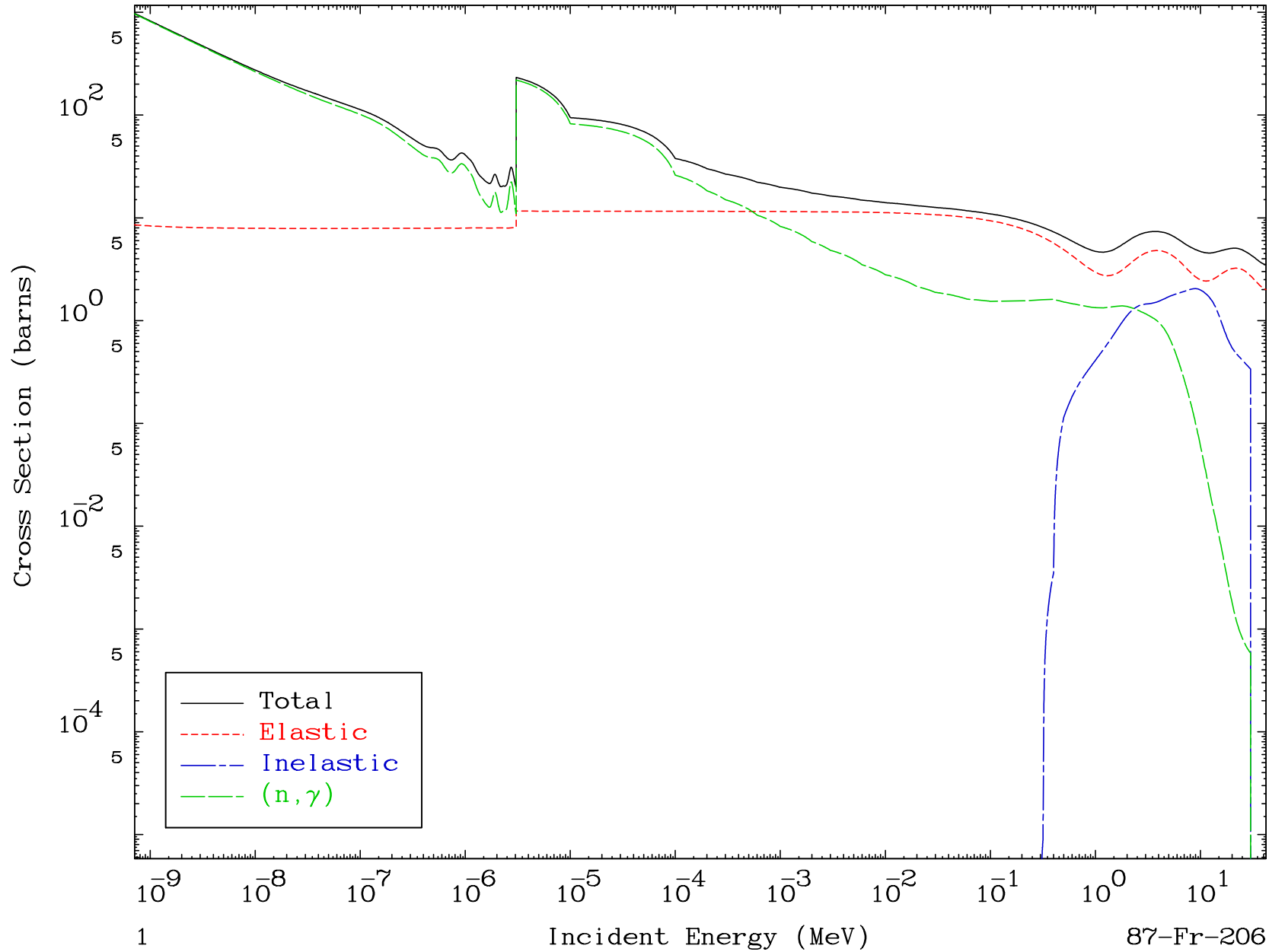


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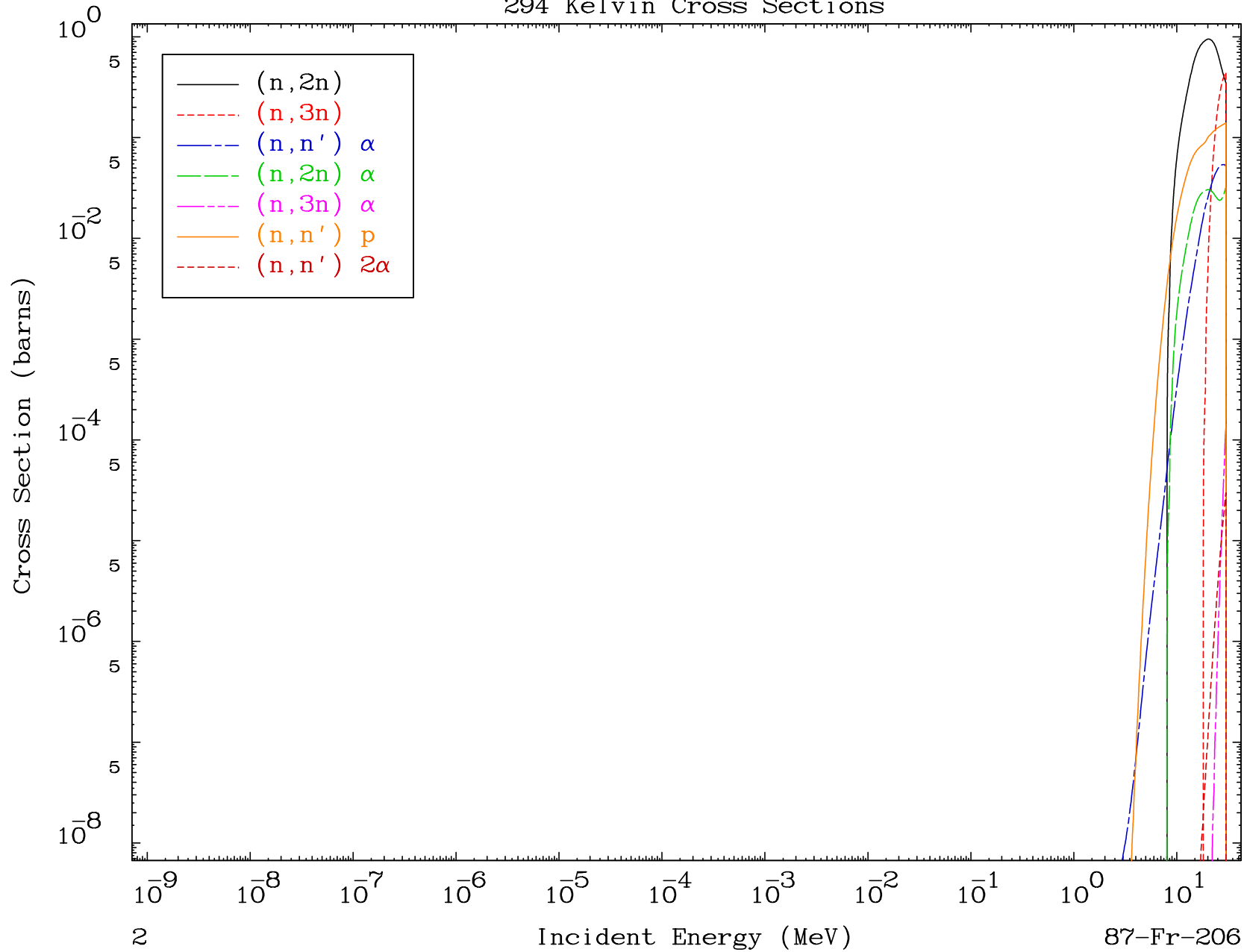
Major  
294 Kelvin Cross Sections

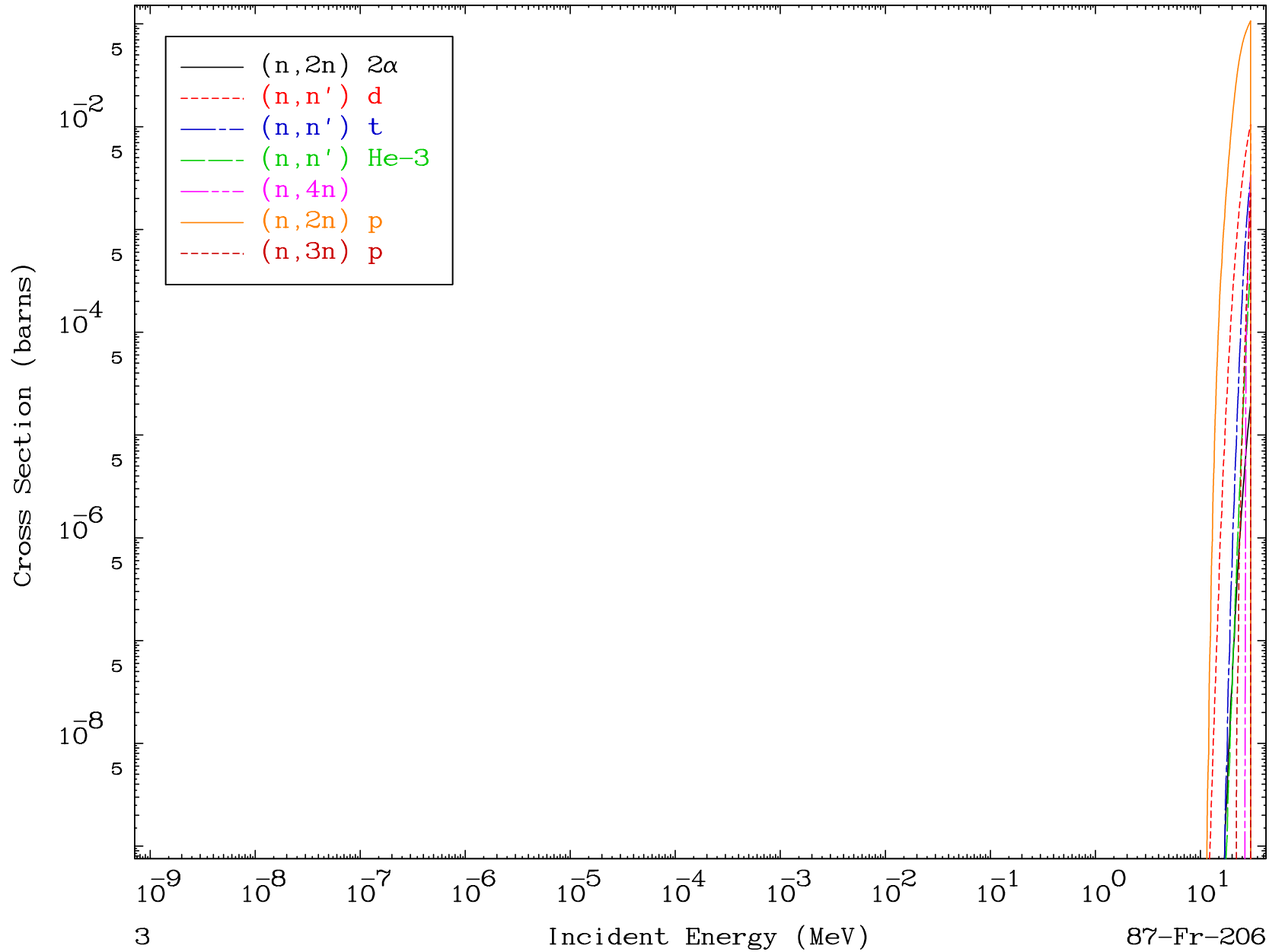
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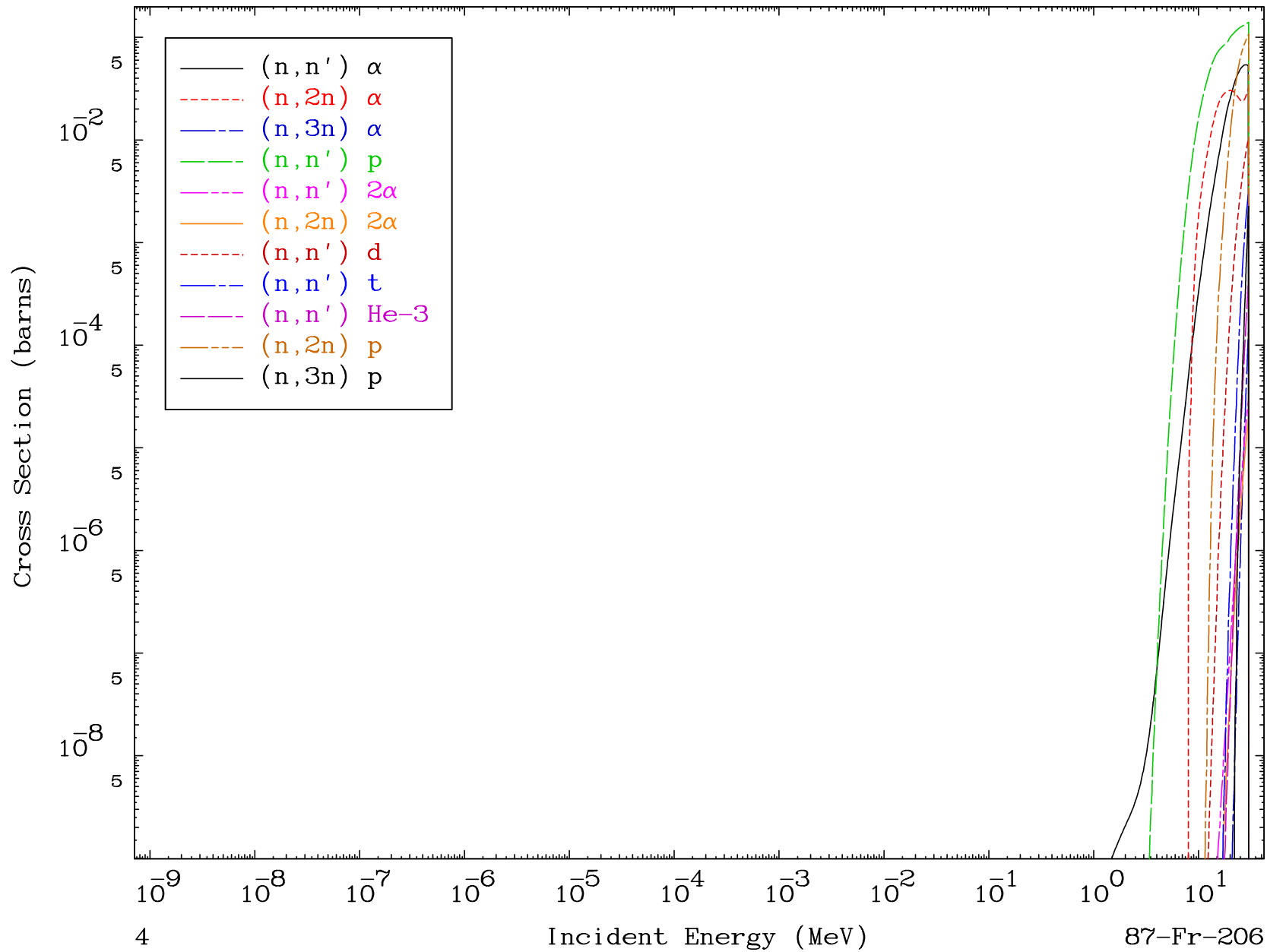


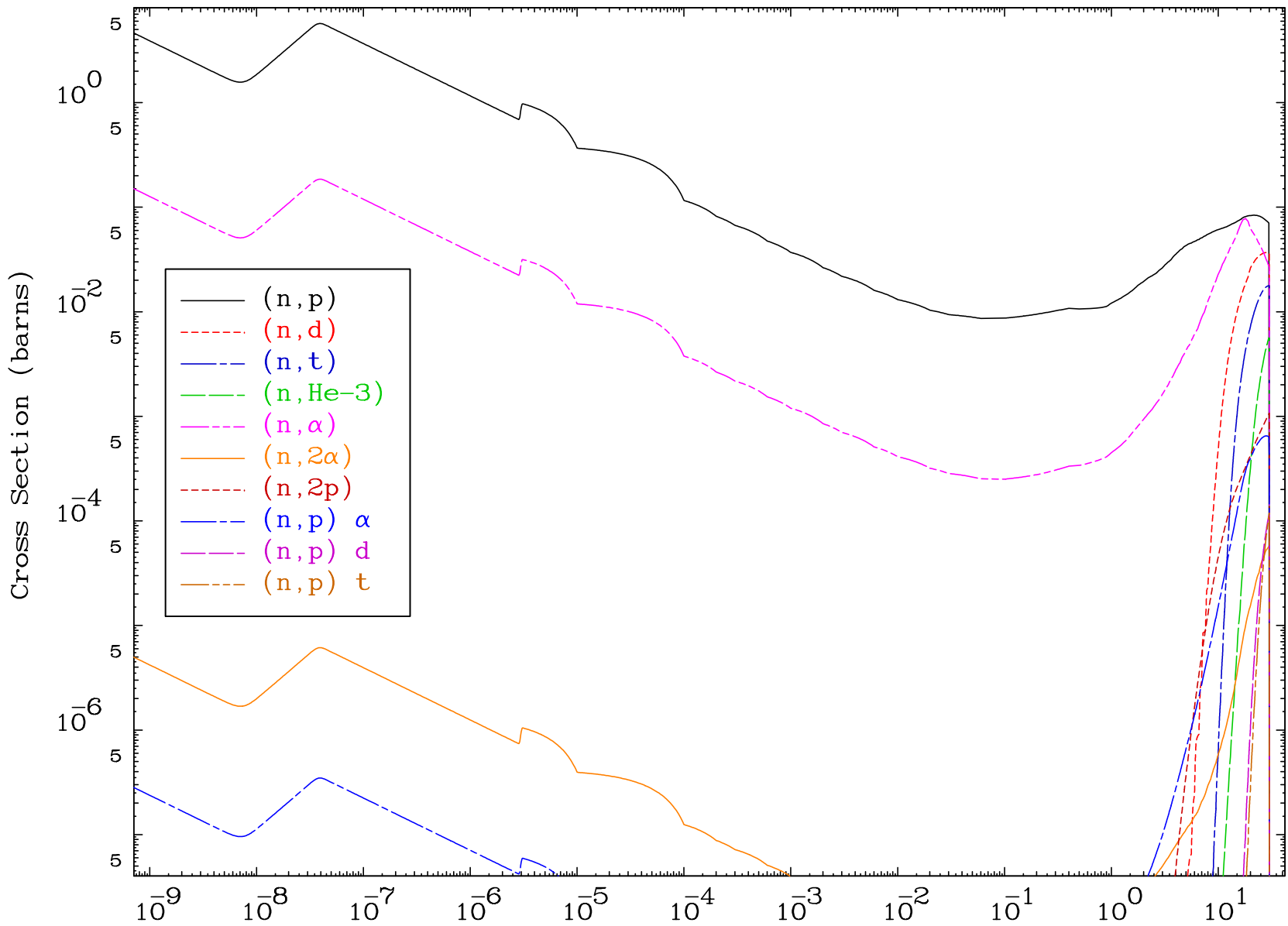
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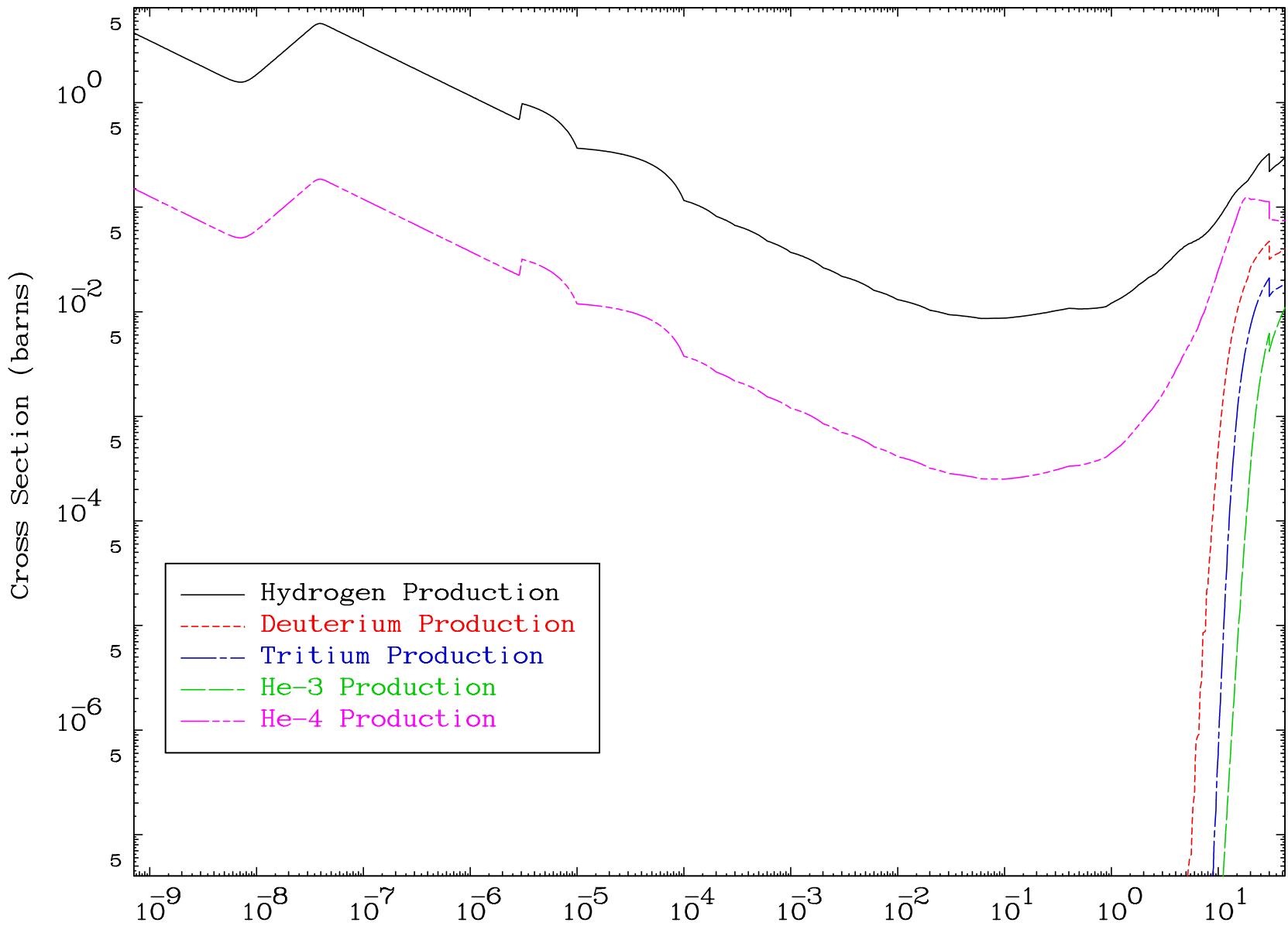
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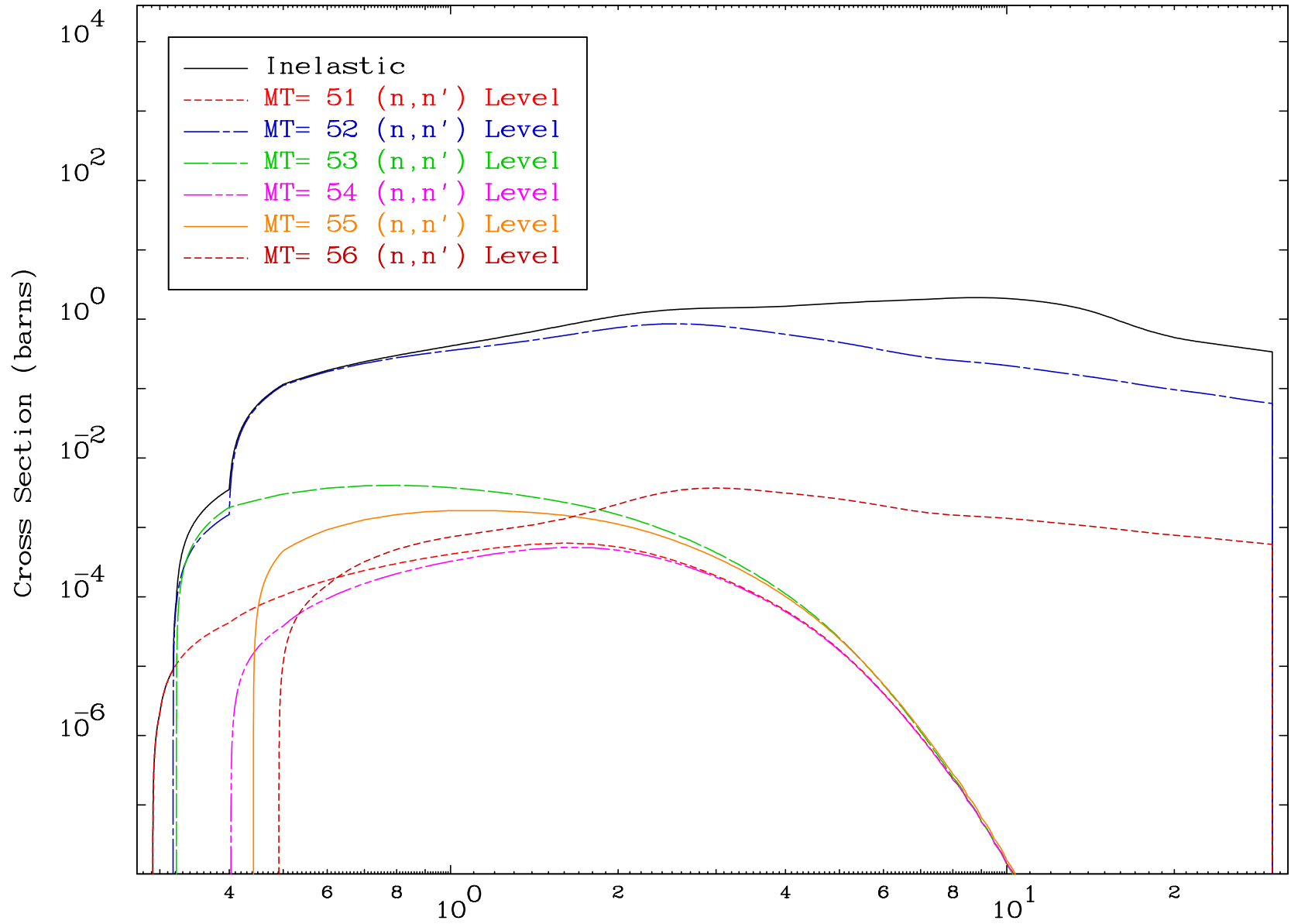


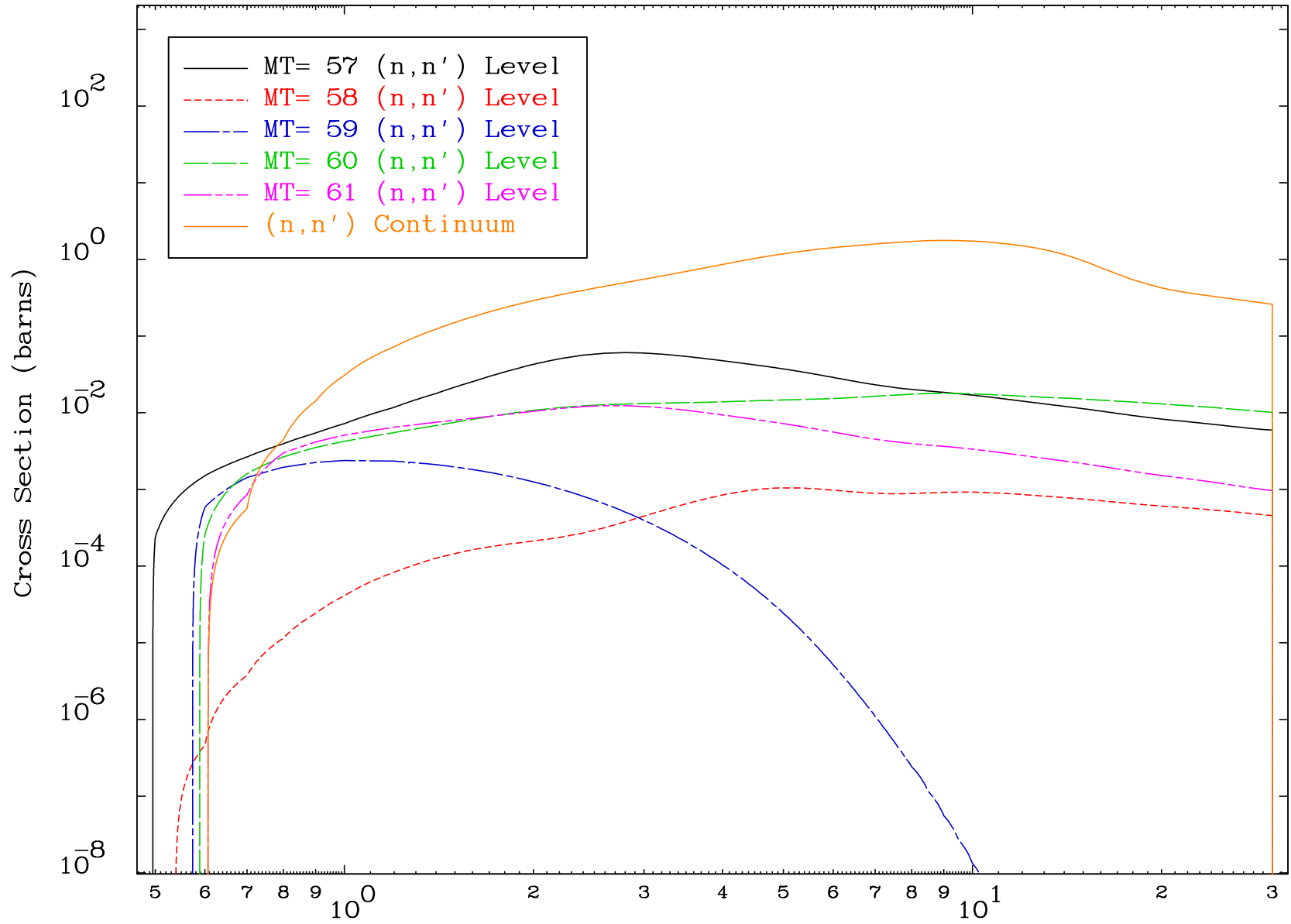










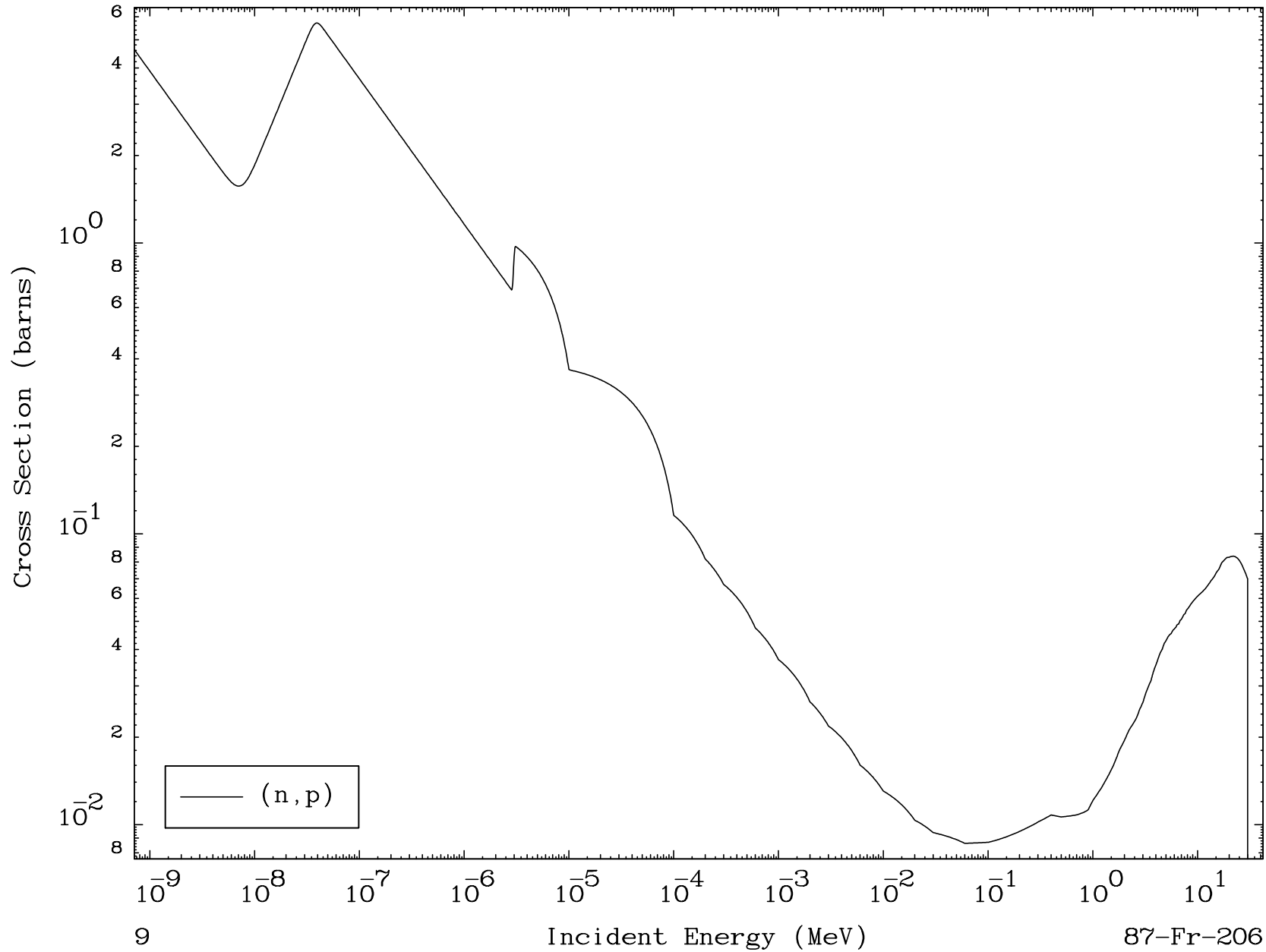




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(n,p) Levels  
294 Kelvin Cross Sections

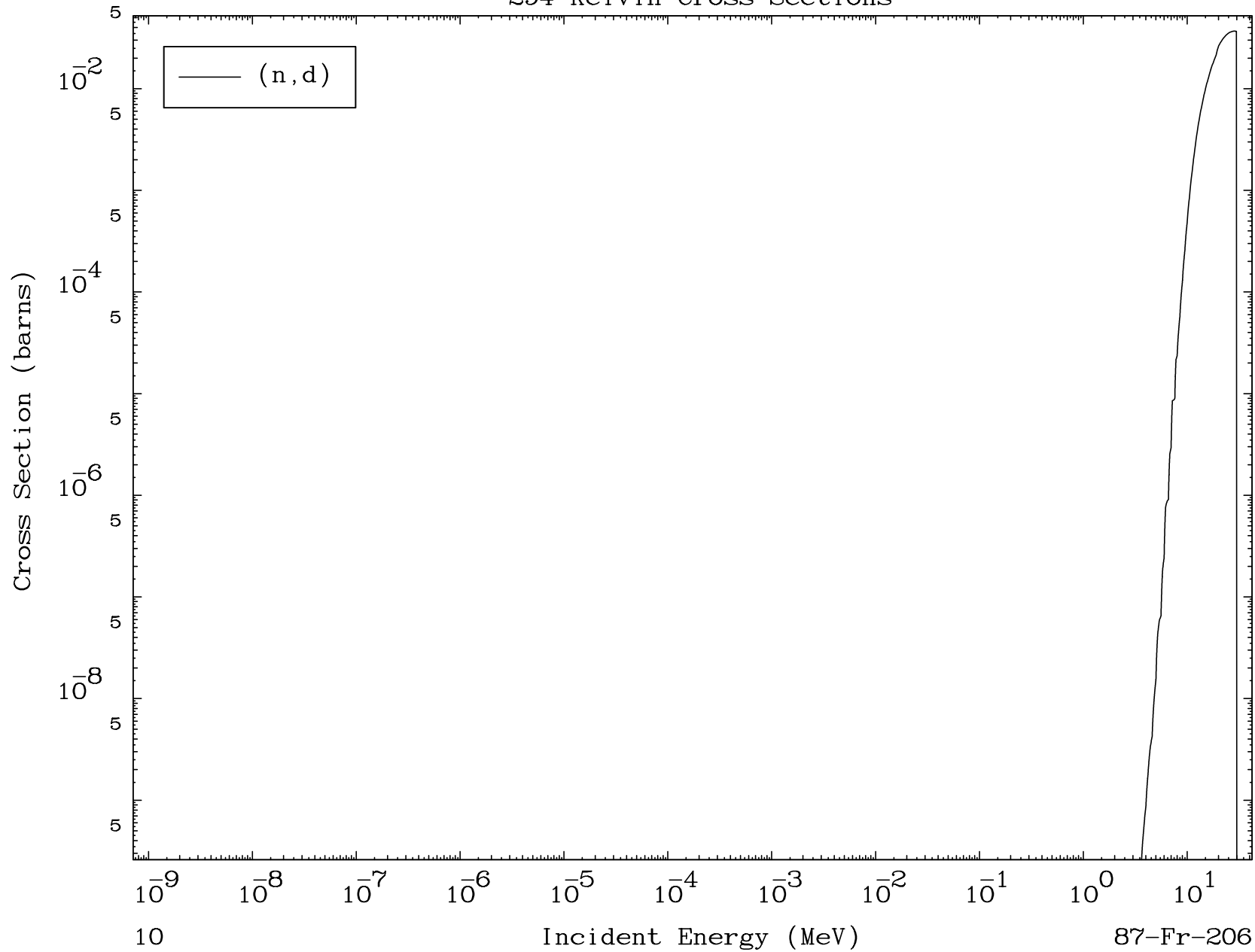
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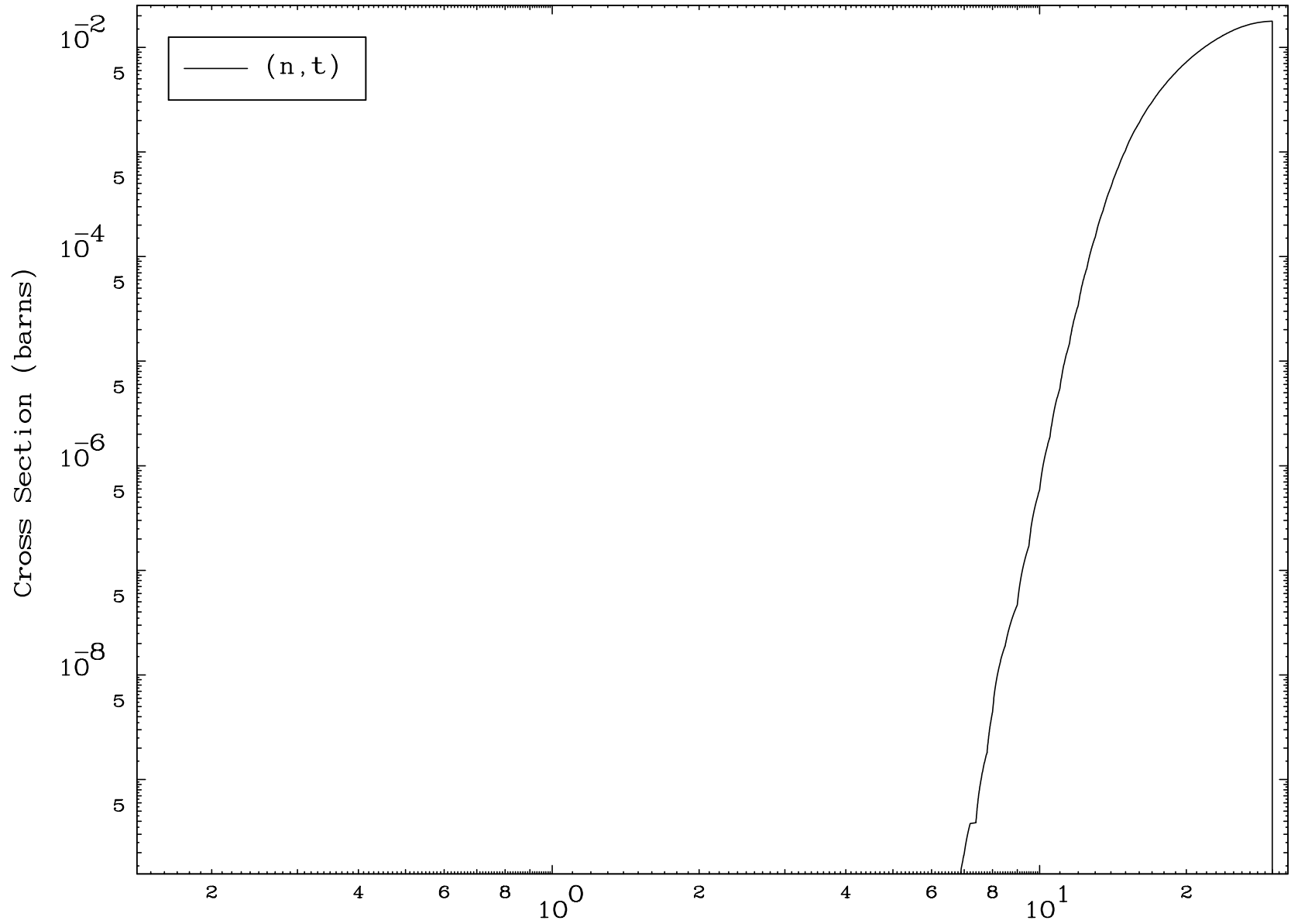


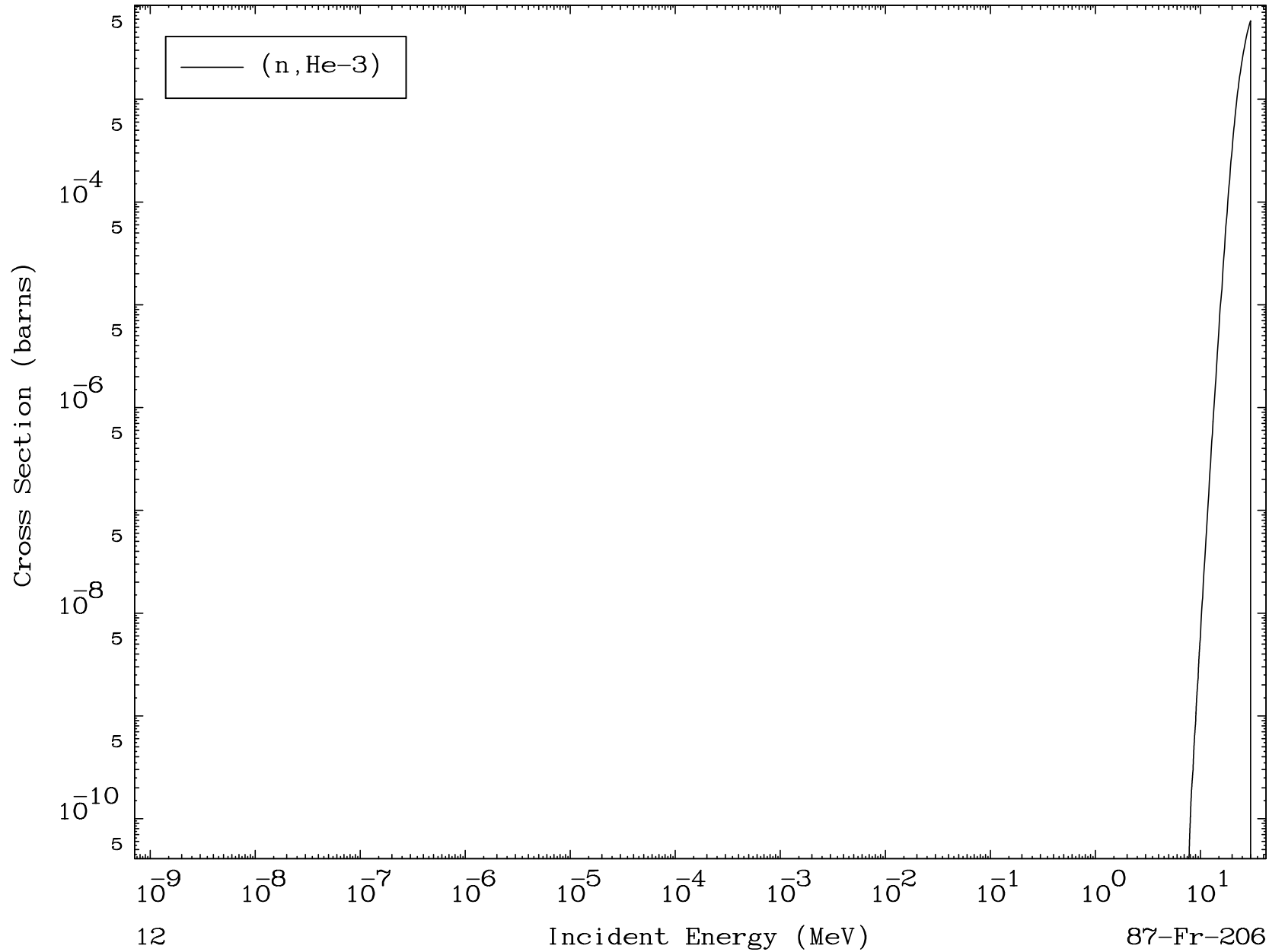
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(n,d) Levels  
294 Kelvin Cross Sections

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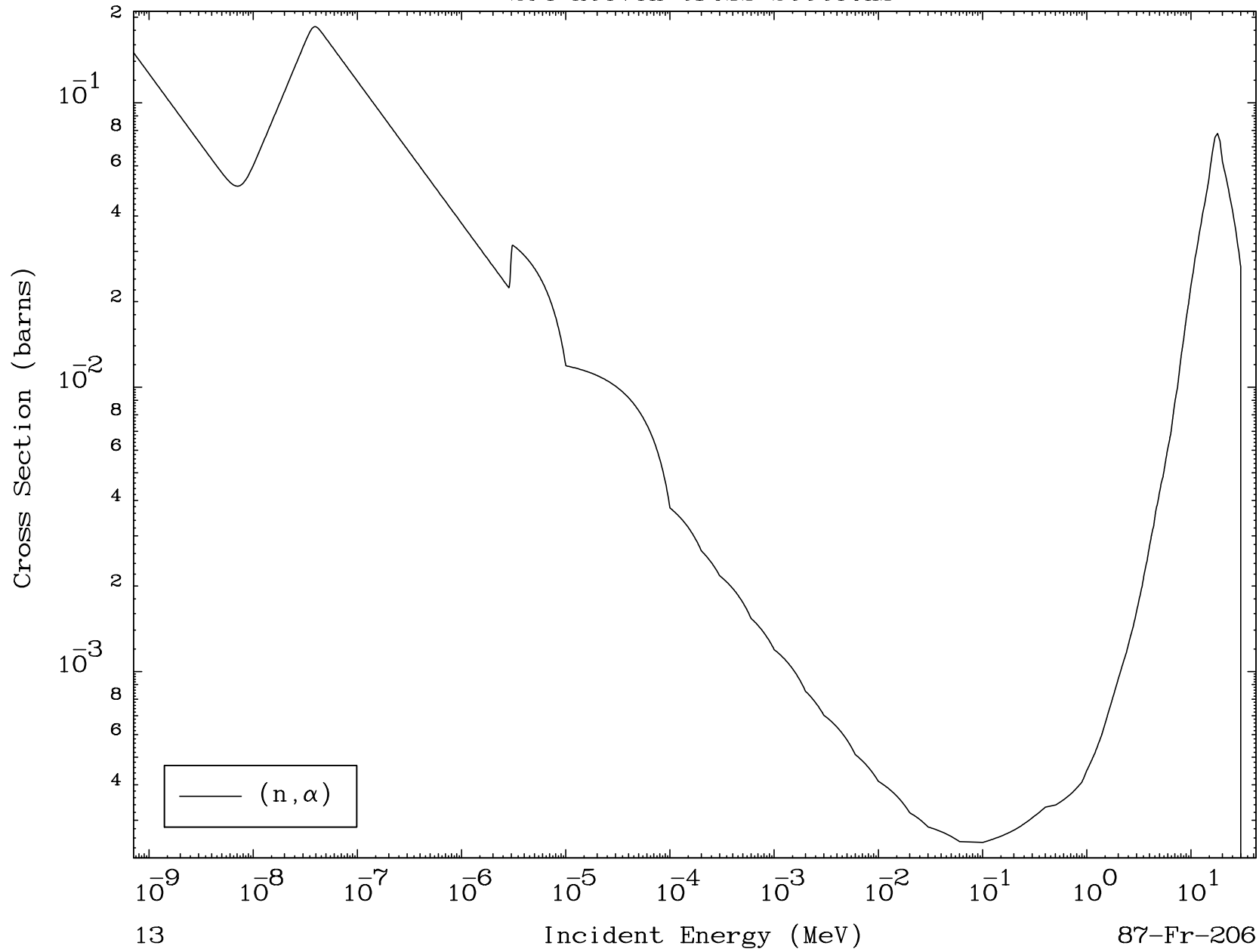




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(n,  $\alpha$ ) Levels  
294 Kelvin Cross Sections

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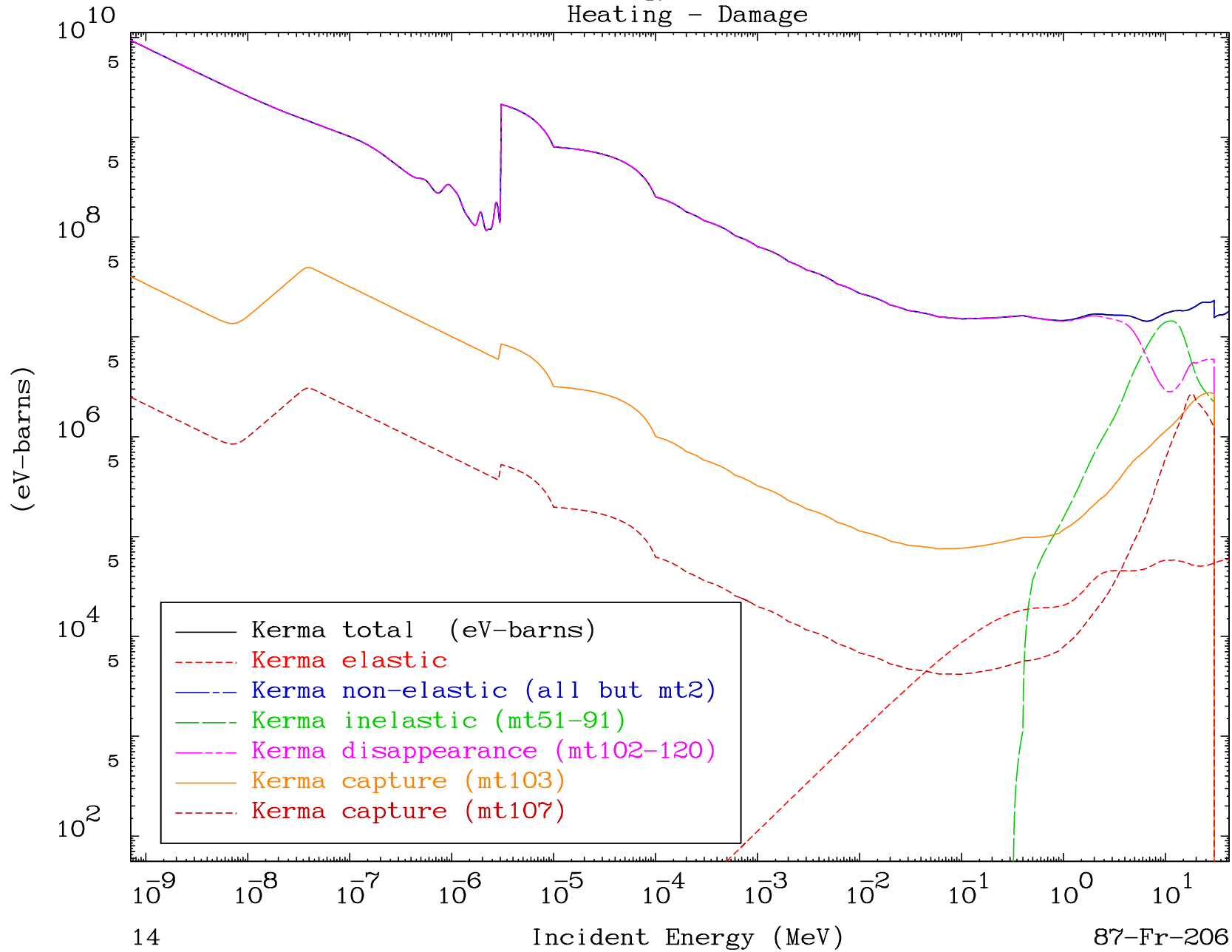
Incident Energy (MeV)

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Energy Release  
Heating - Damage

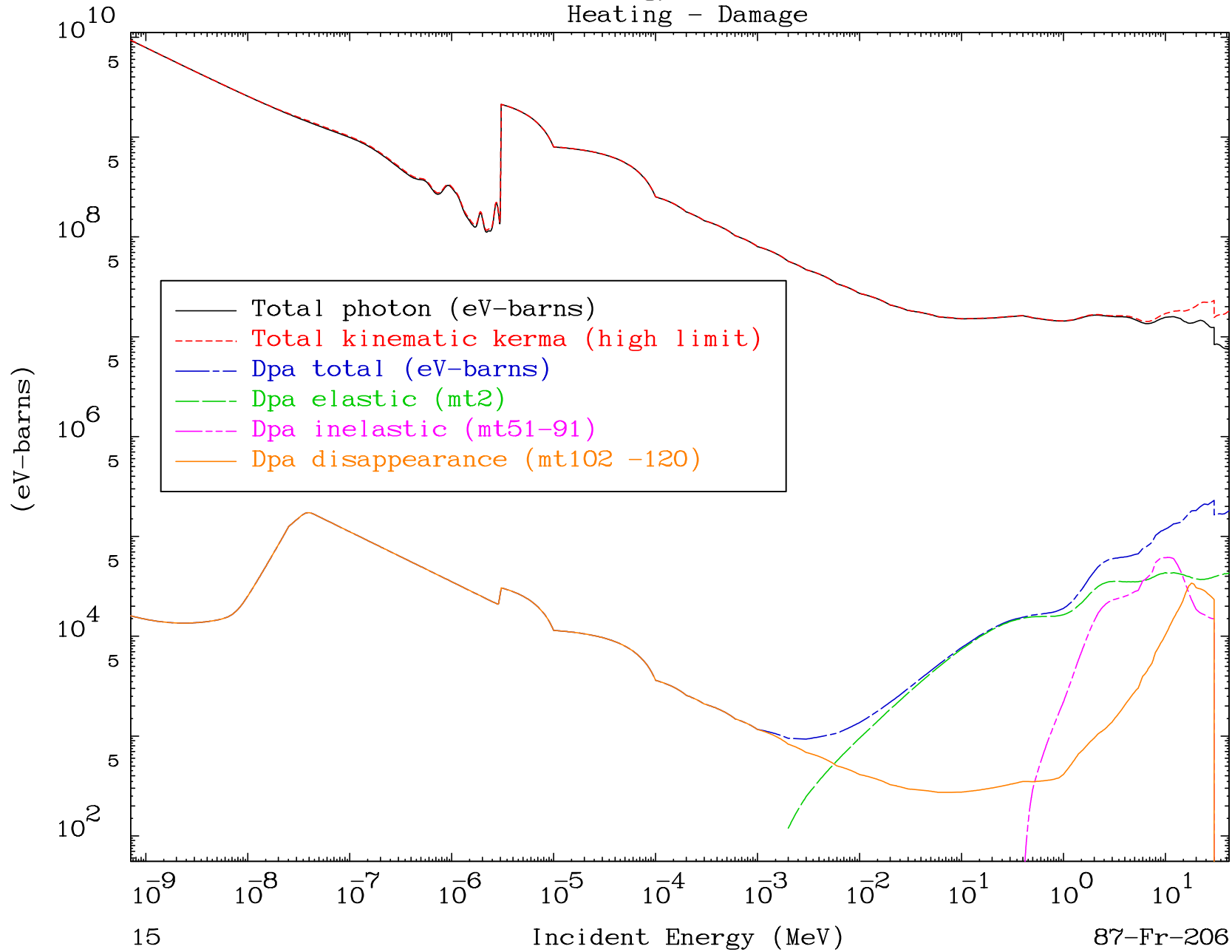
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Energy Release  
Heating - Damage

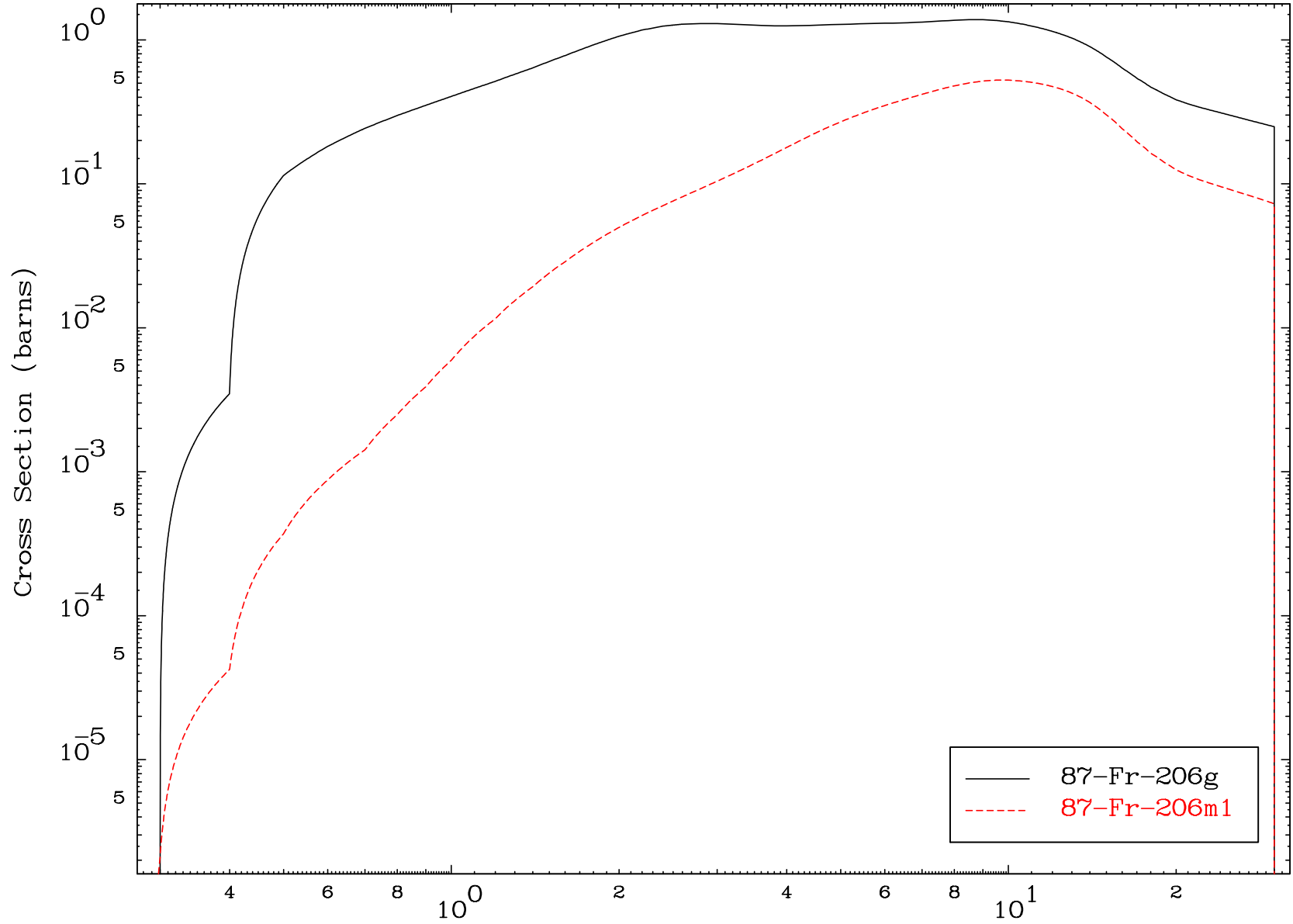
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Inelastic  
Radionuclide Production Cross Section

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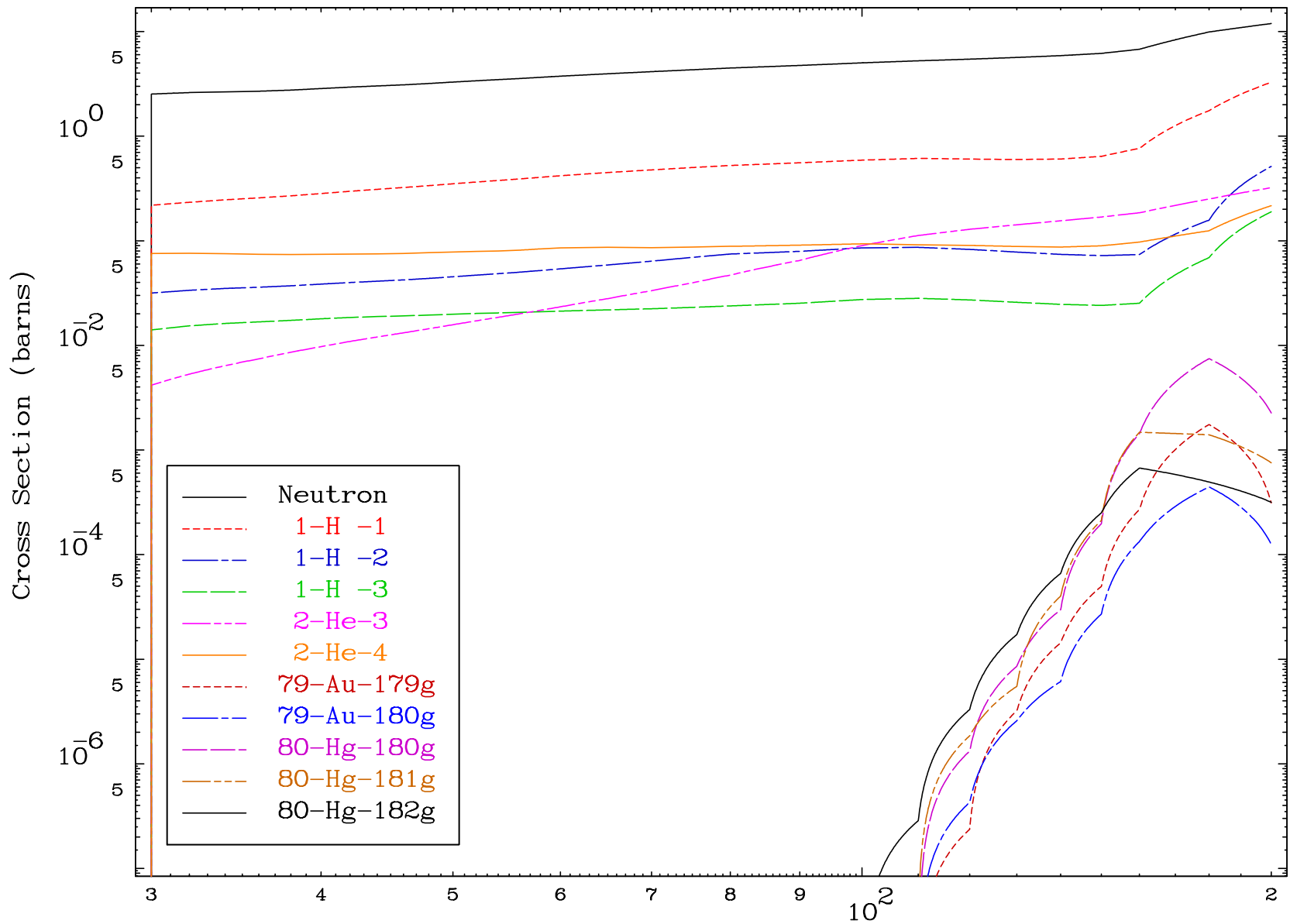


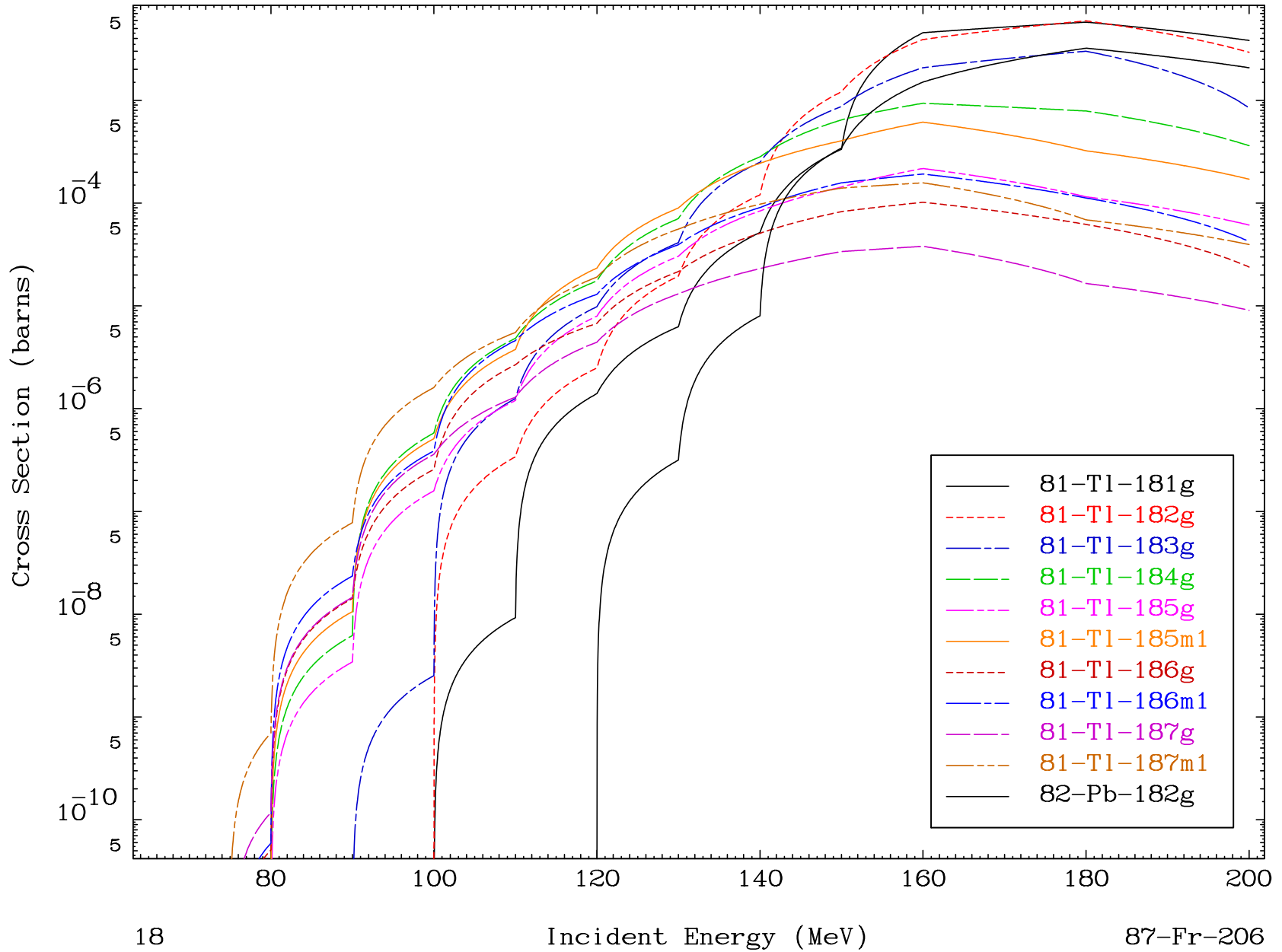
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Incident Energy (MeV)

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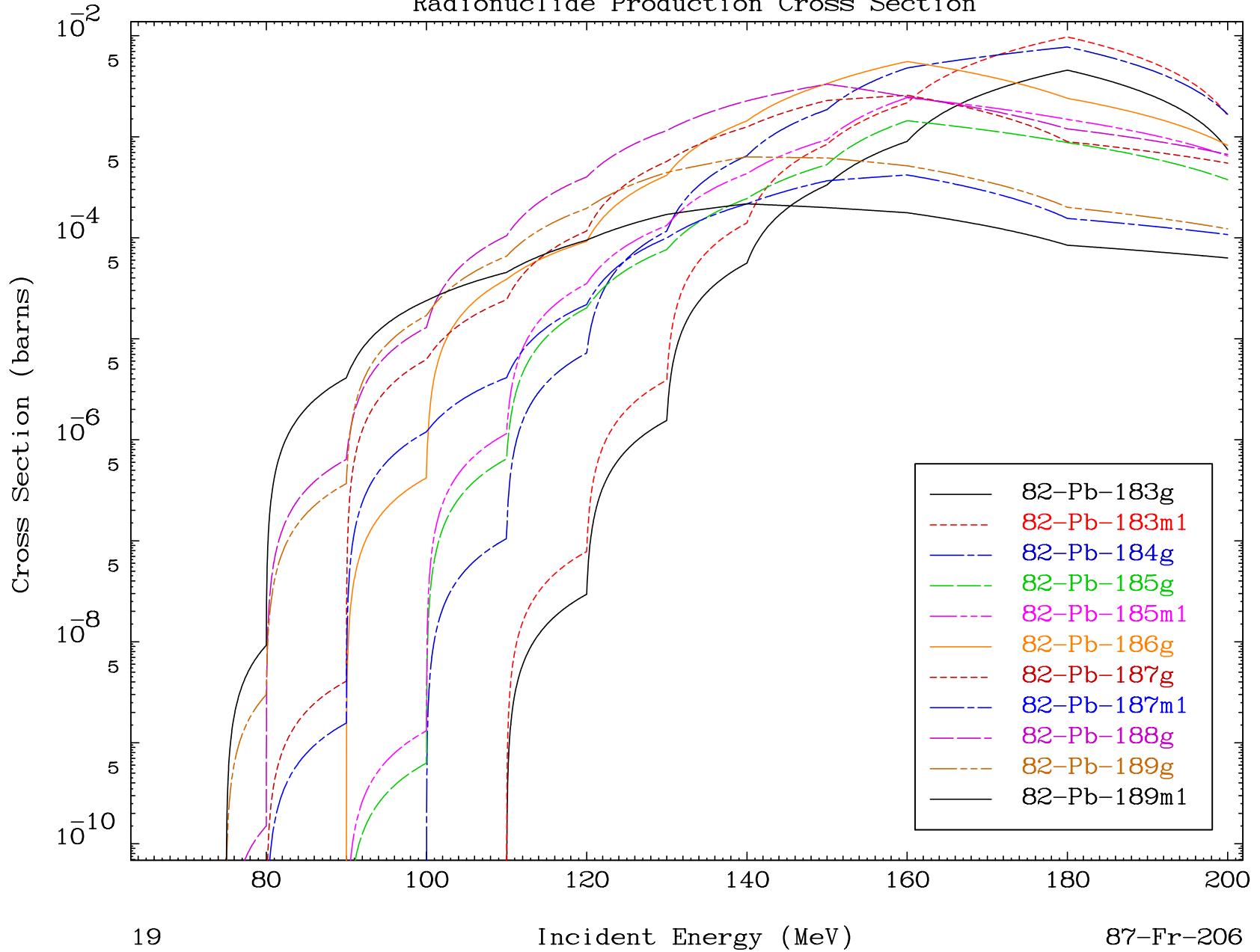


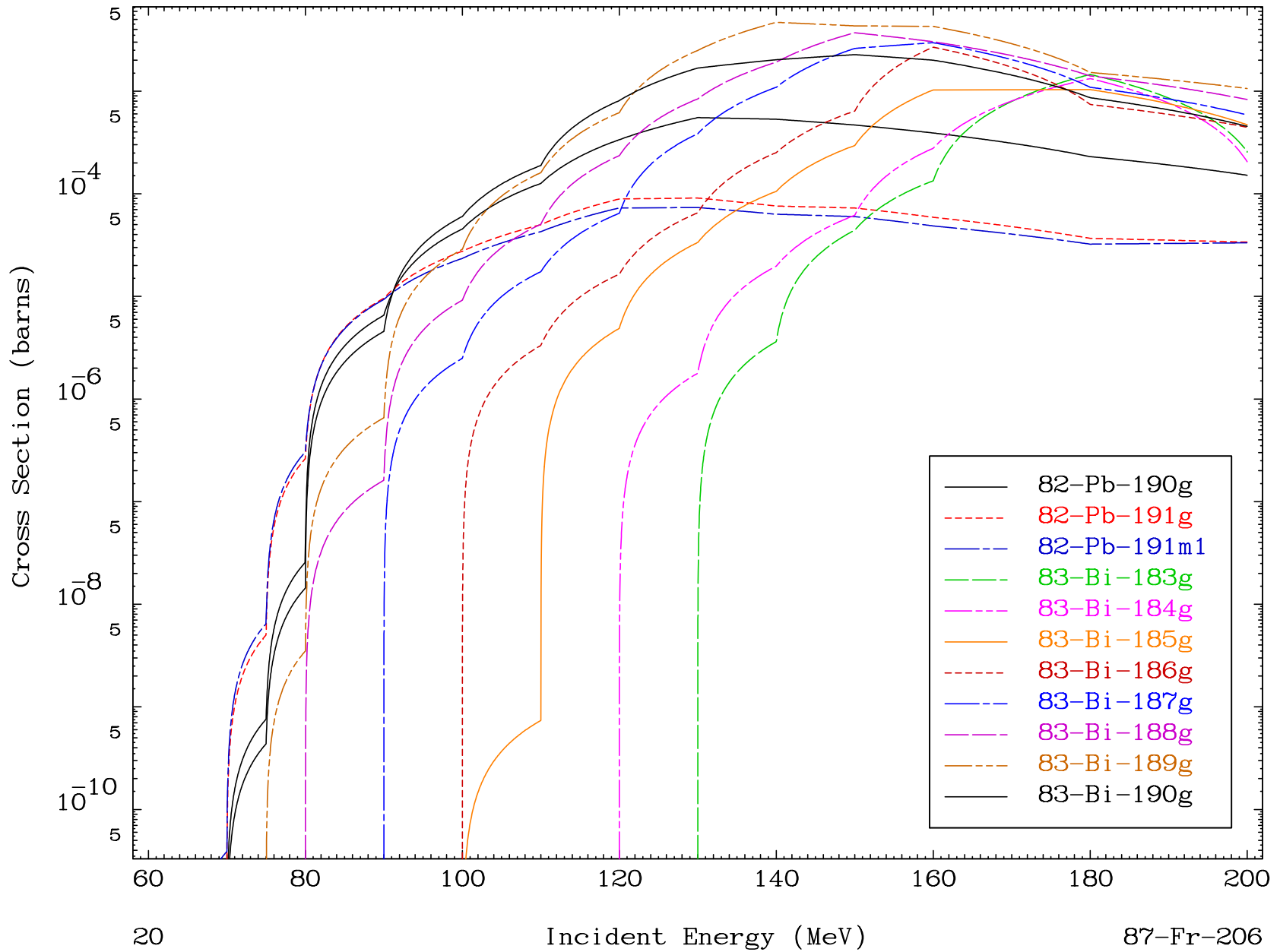
MAT 8707

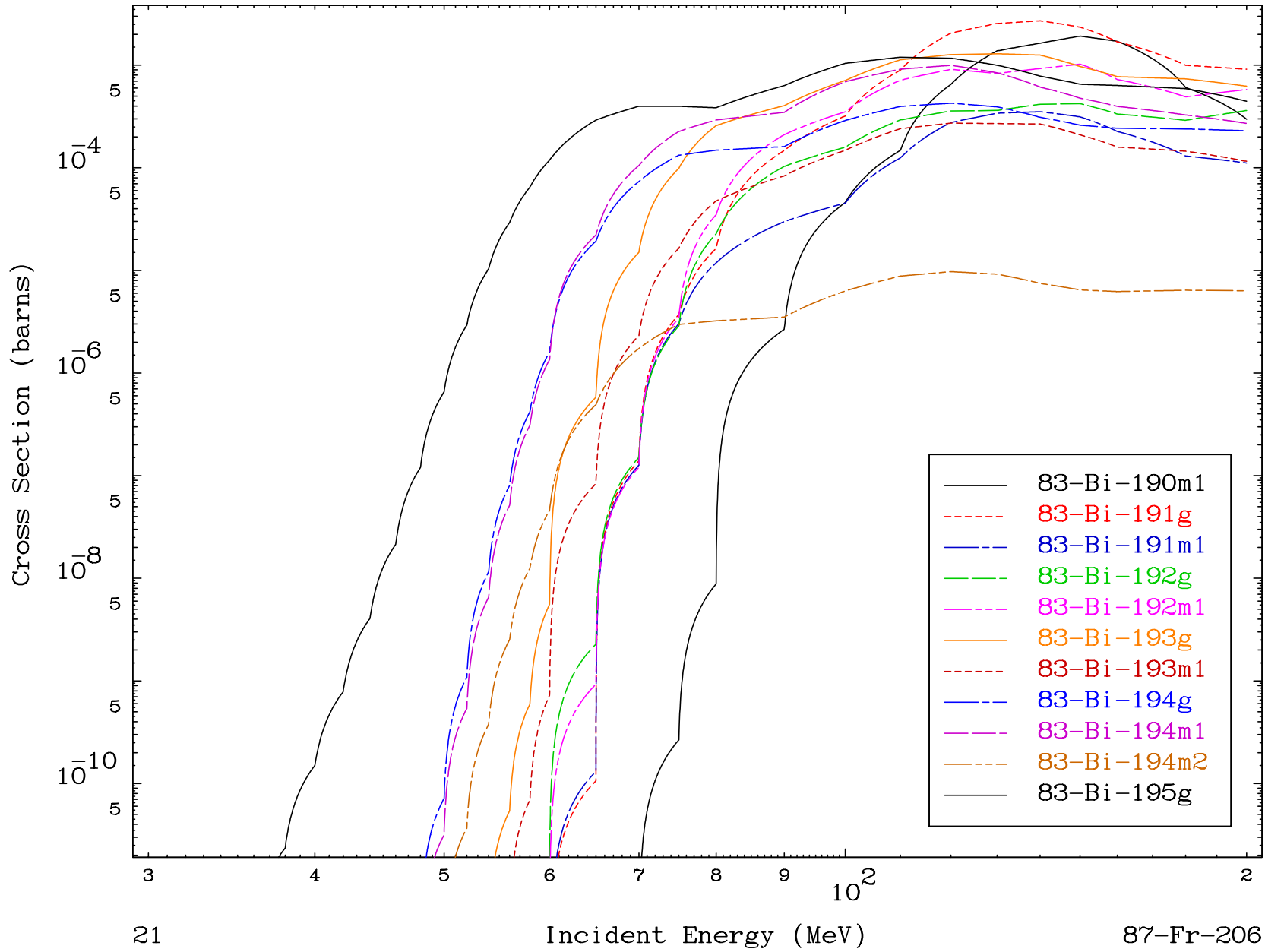
(n,remainder)

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### Radionuclide Production Cross Section



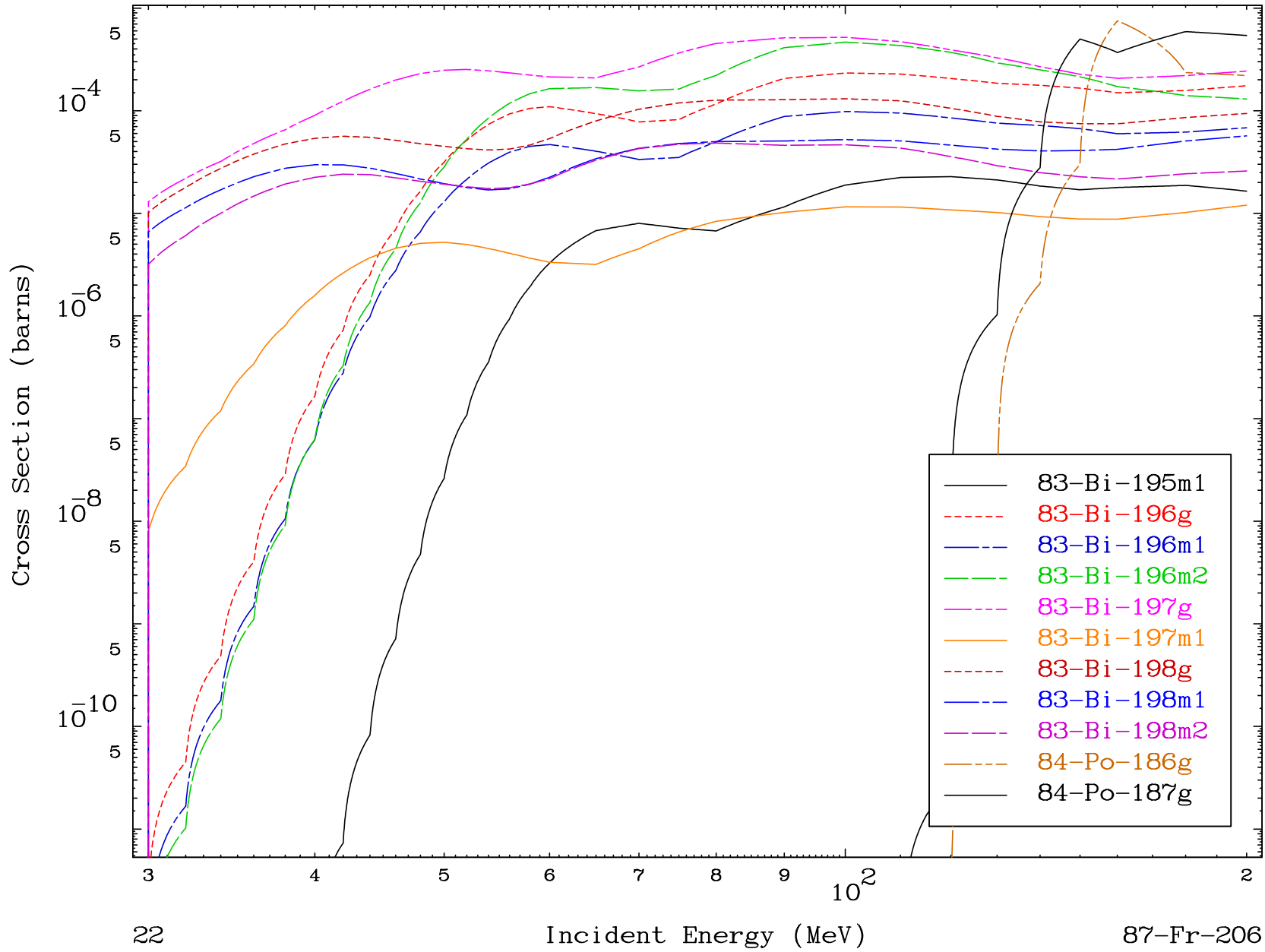


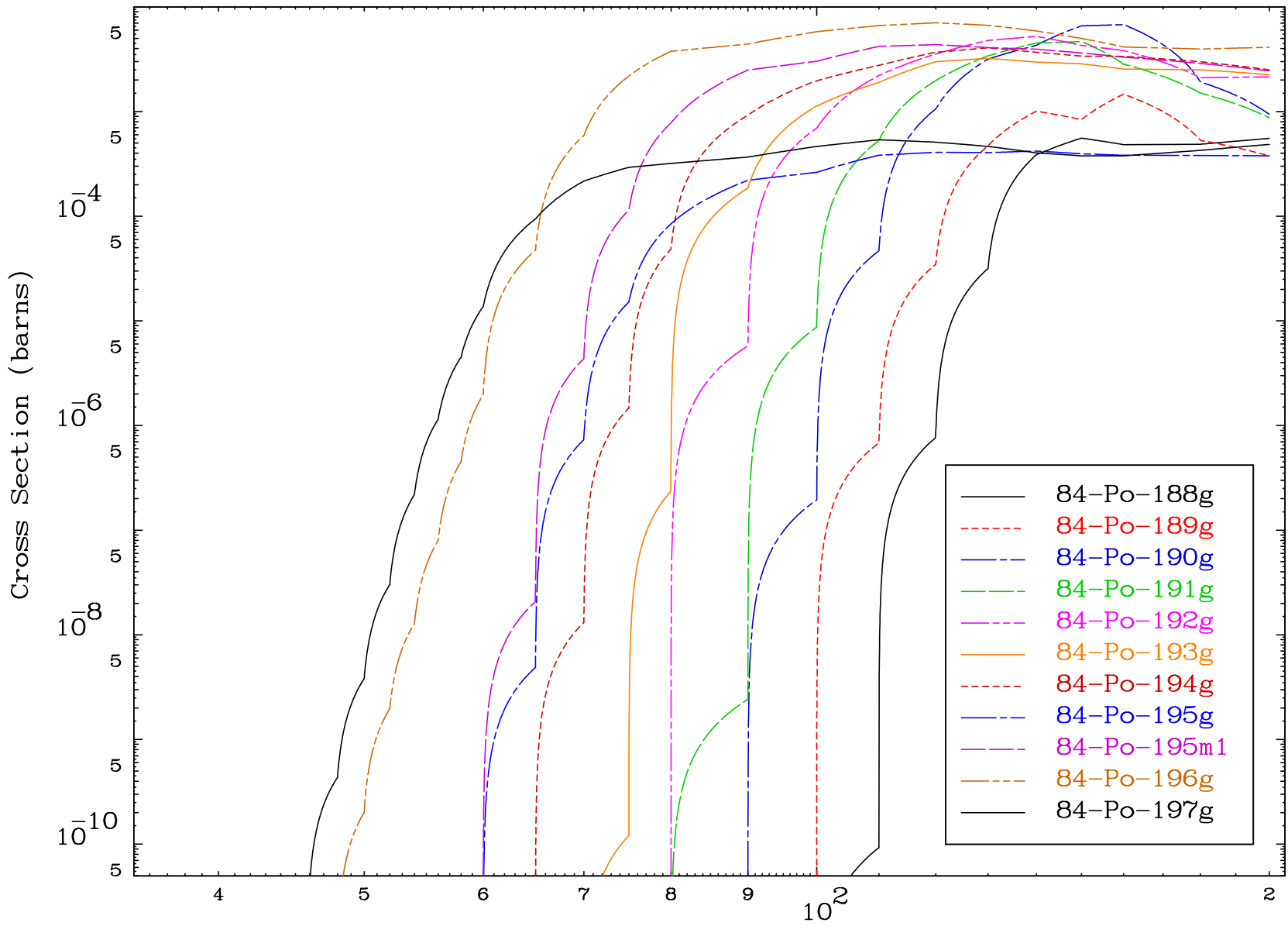


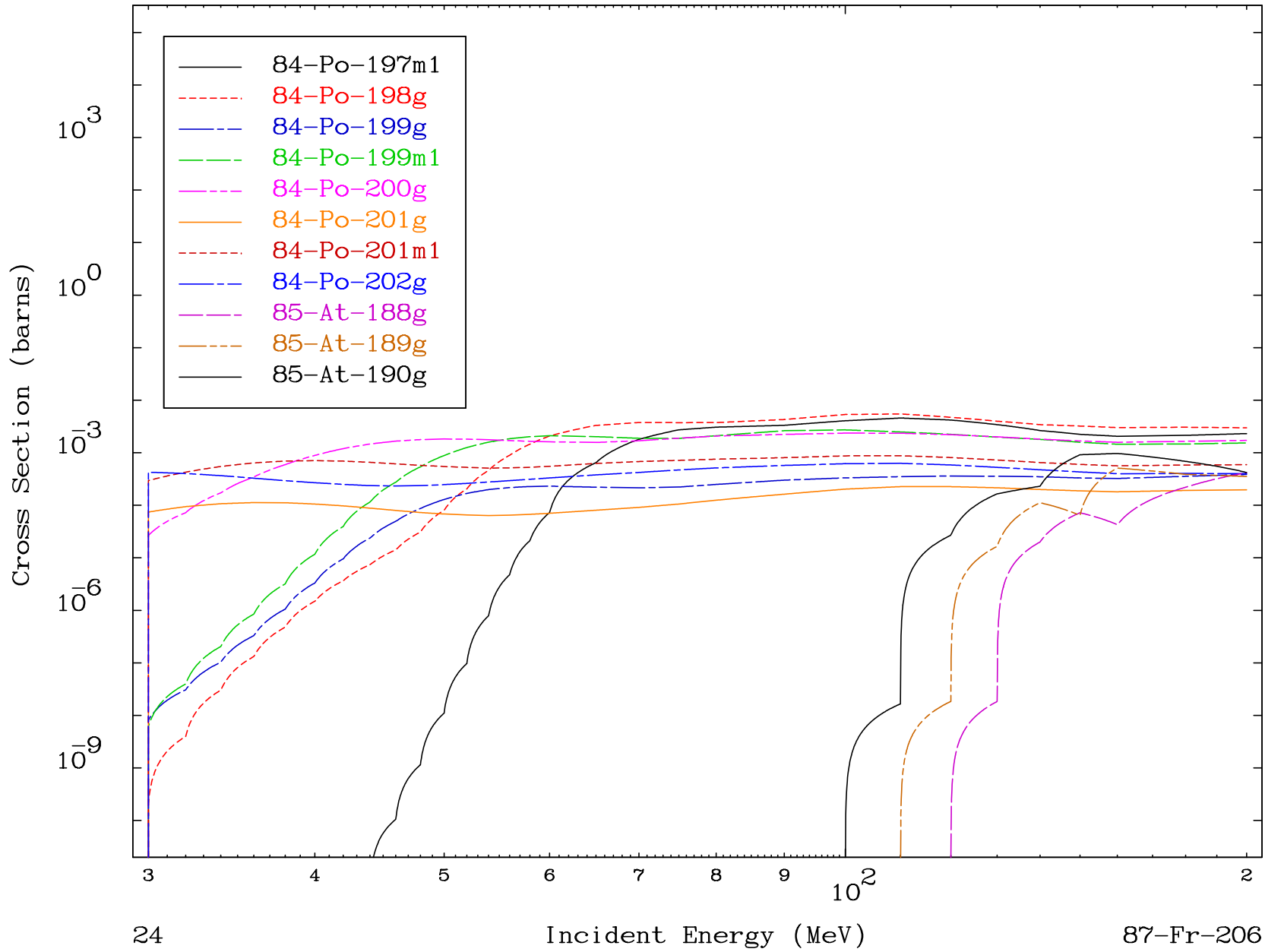
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(n,remainder)  
Radionuclide Production Cross Section

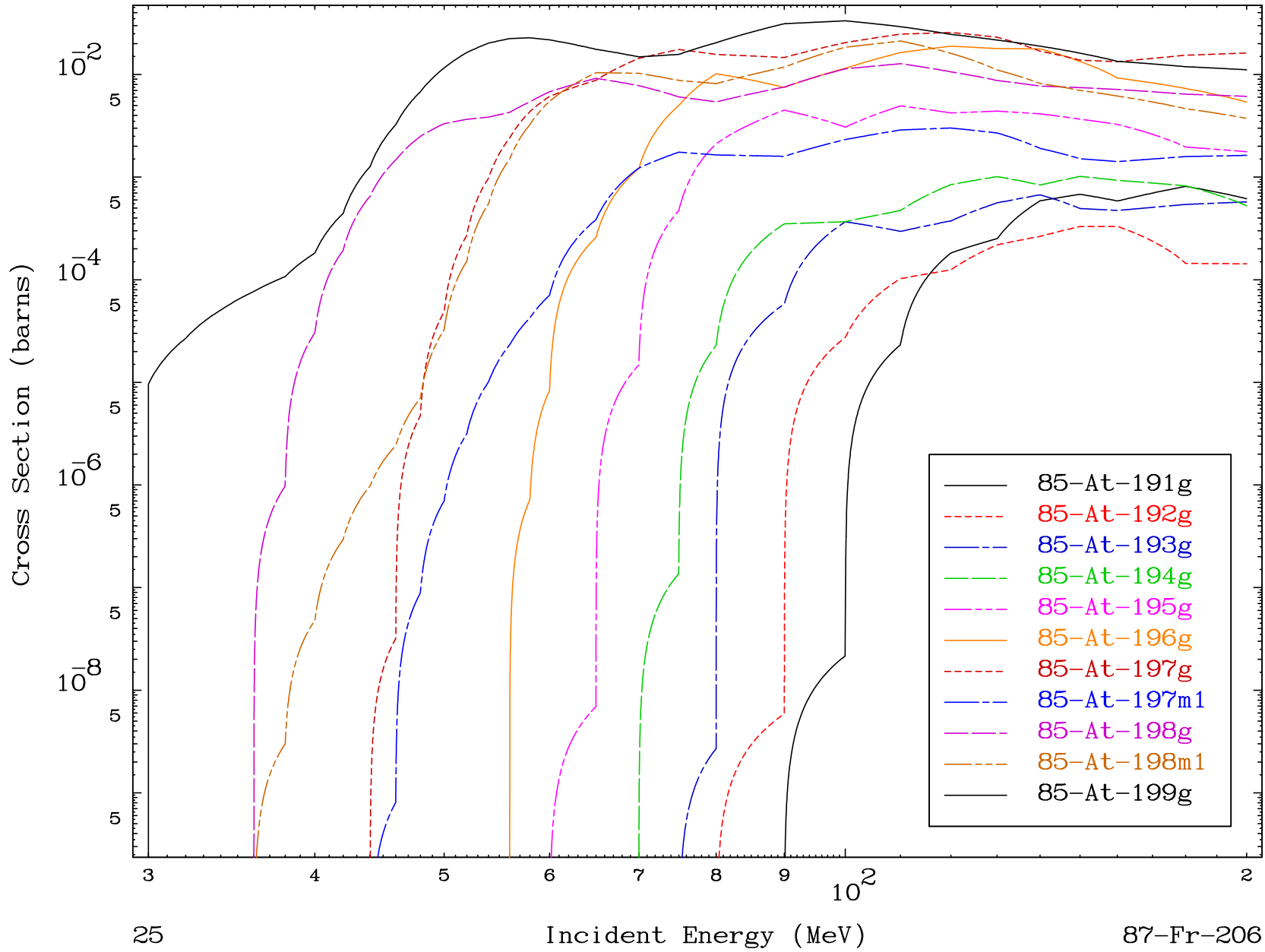
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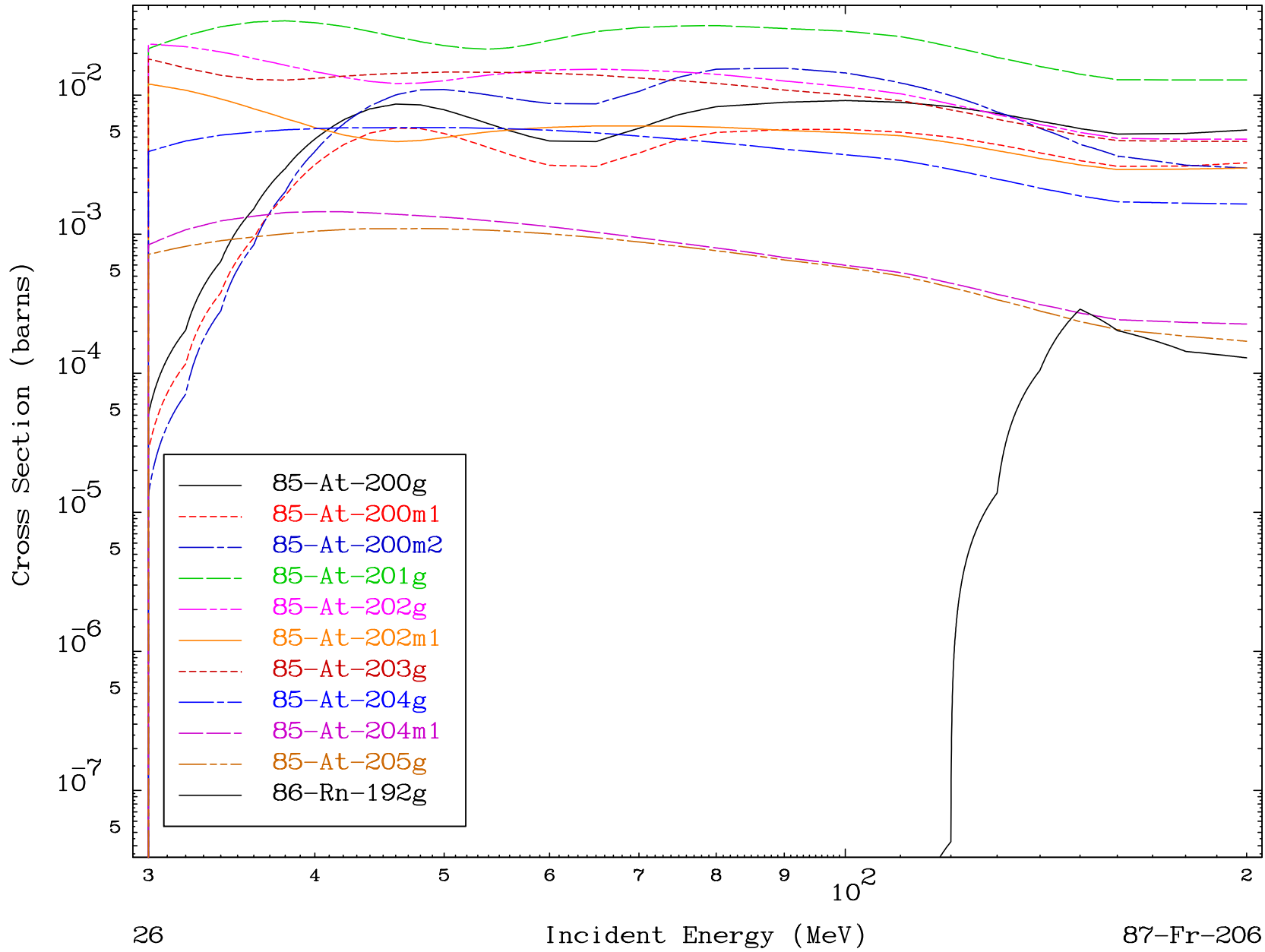




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(n,remainder)  
Radionuclide Production Cross Section

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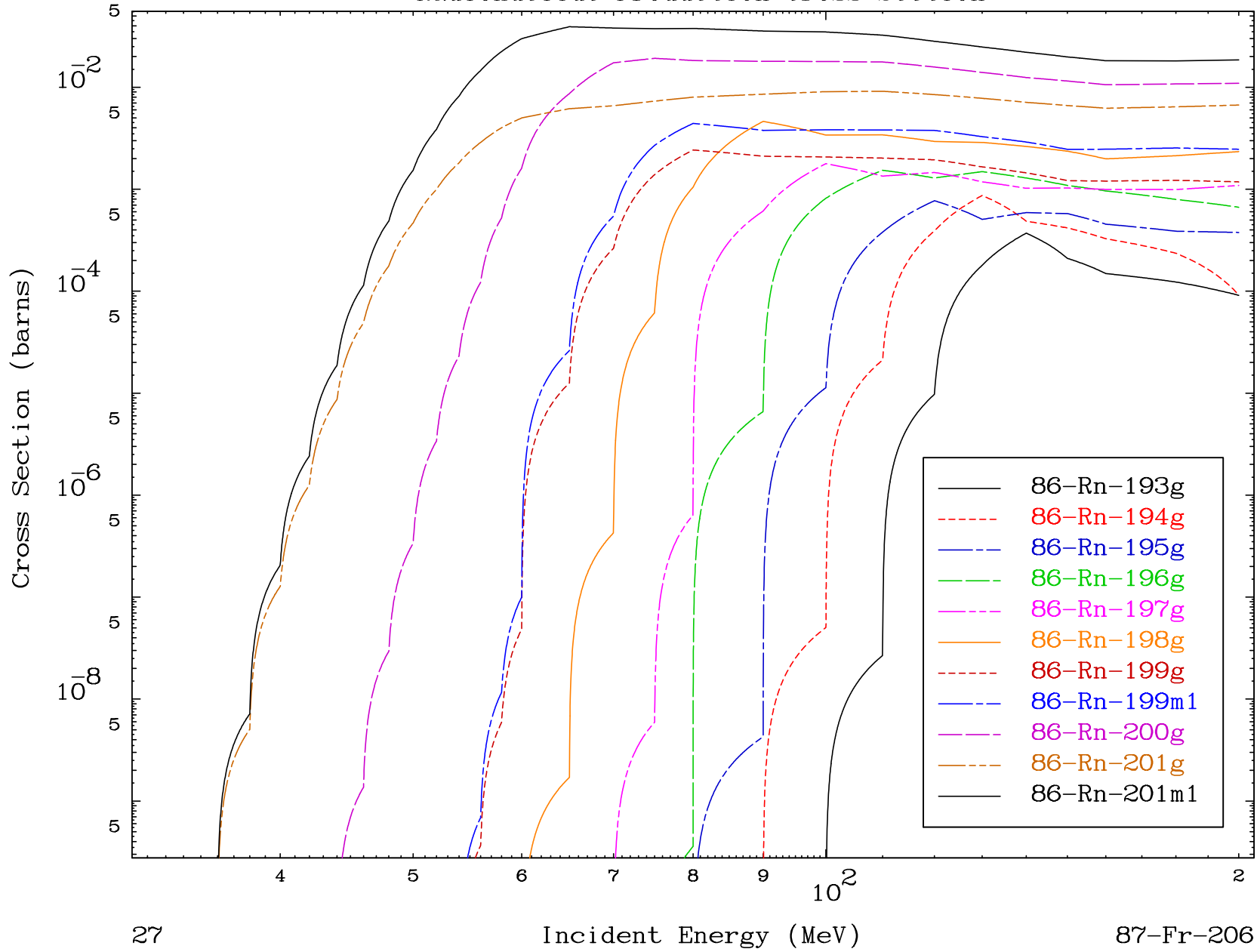


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(n,remainder)

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### Radionuclide Production Cross Section

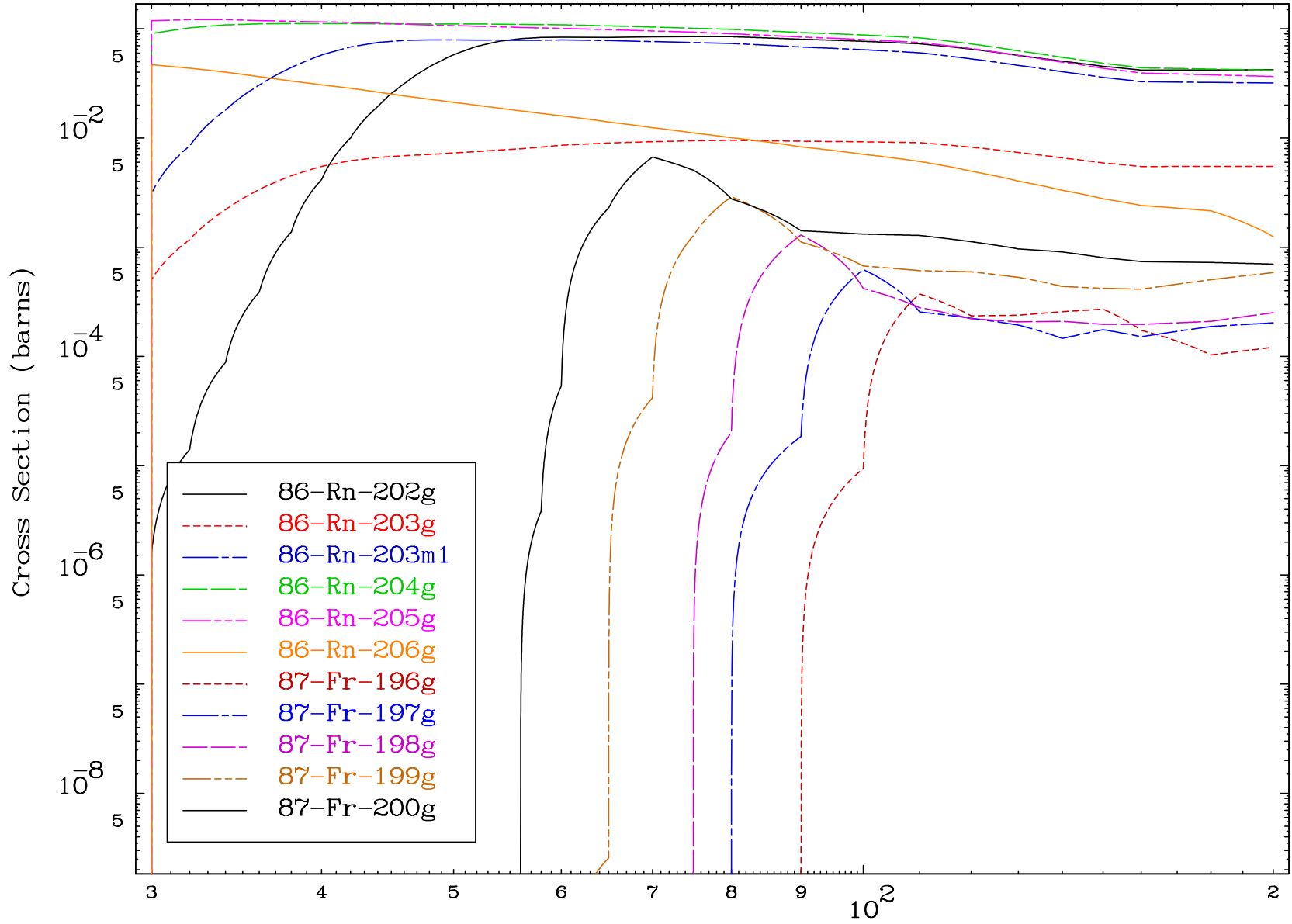


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(n,remainder)

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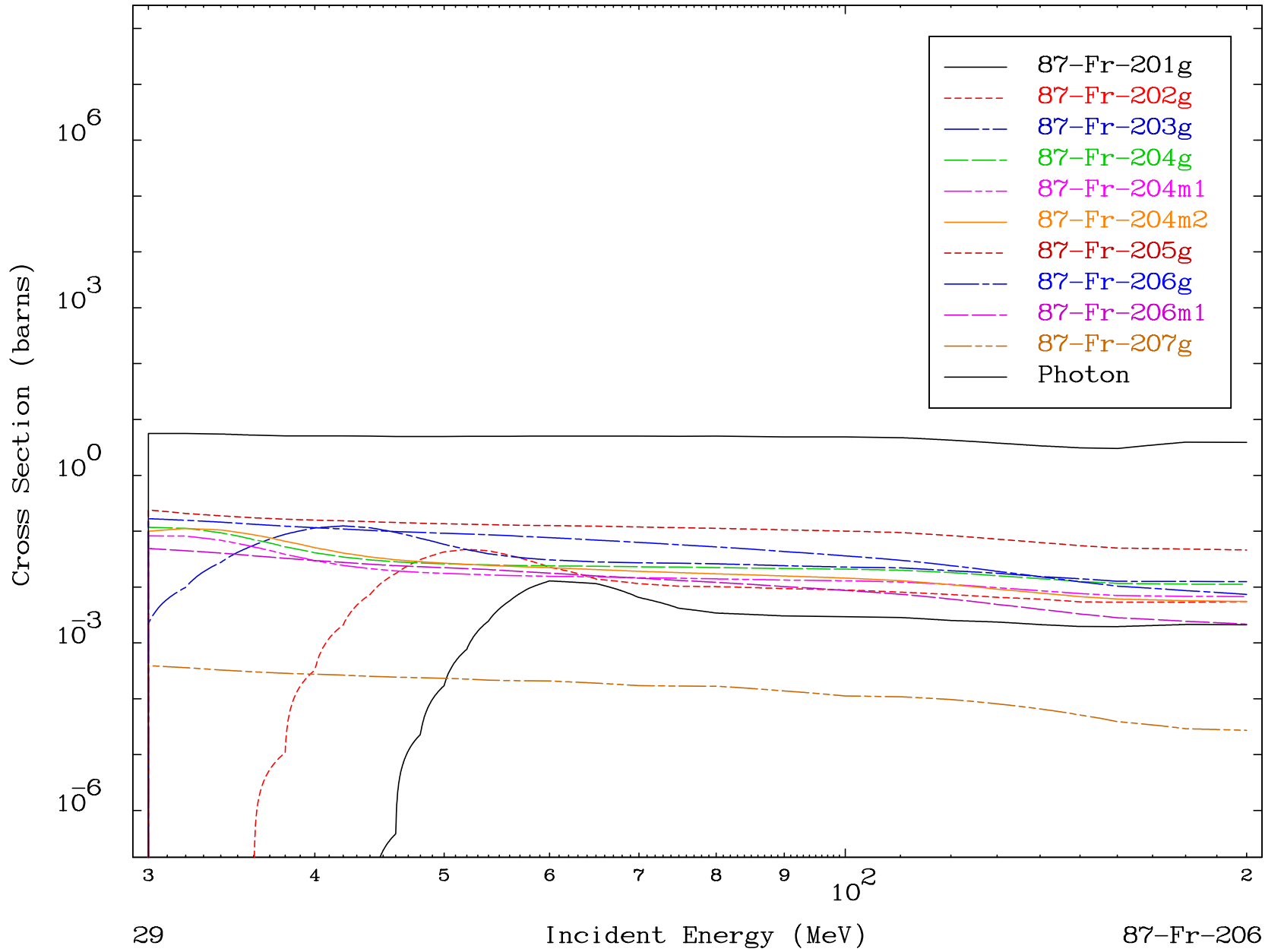
### Radionuclide Production Cross Section



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Incident Energy (MeV)

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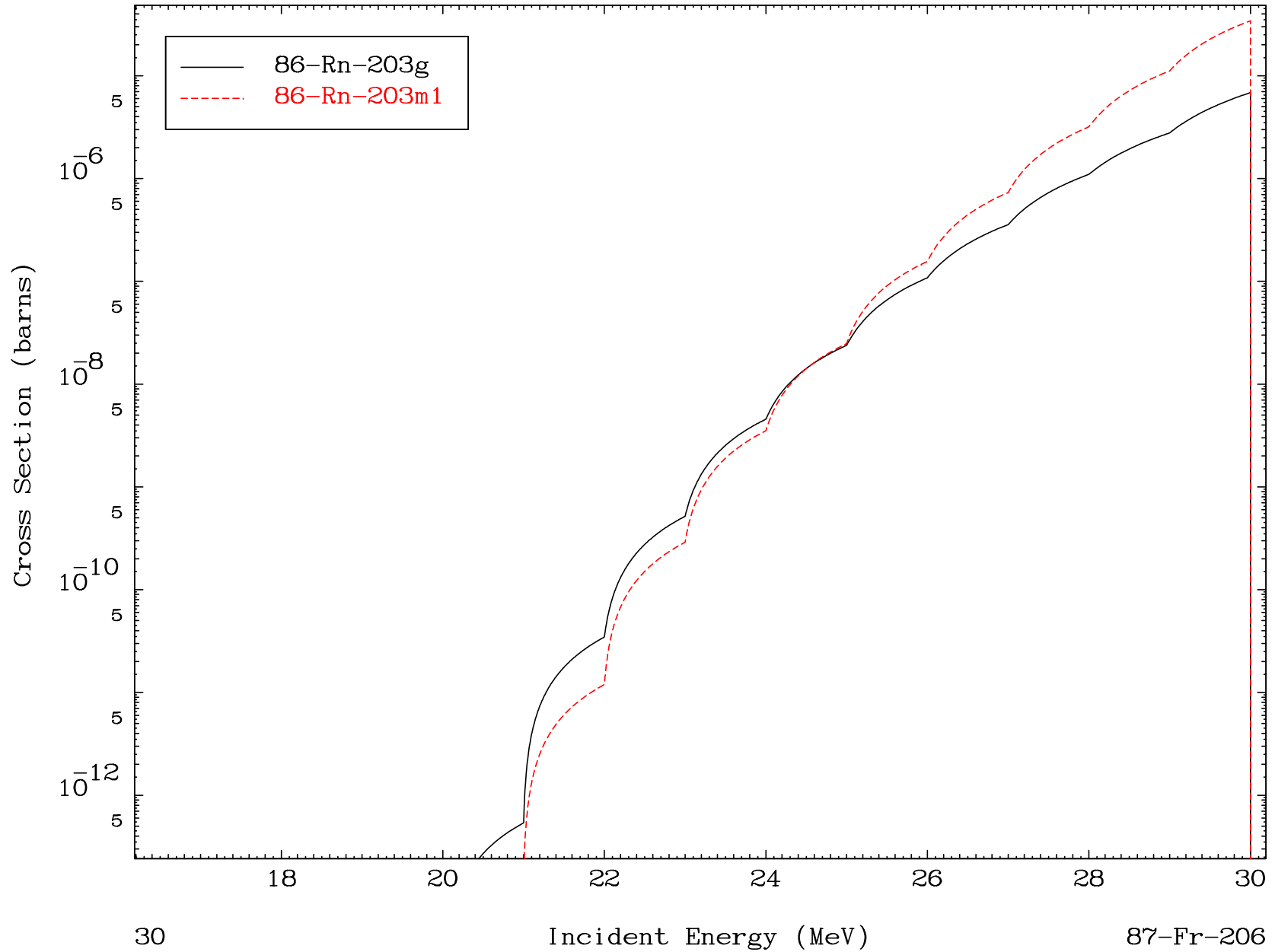


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(n,2n) d

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Radionuclide Production Cross Section

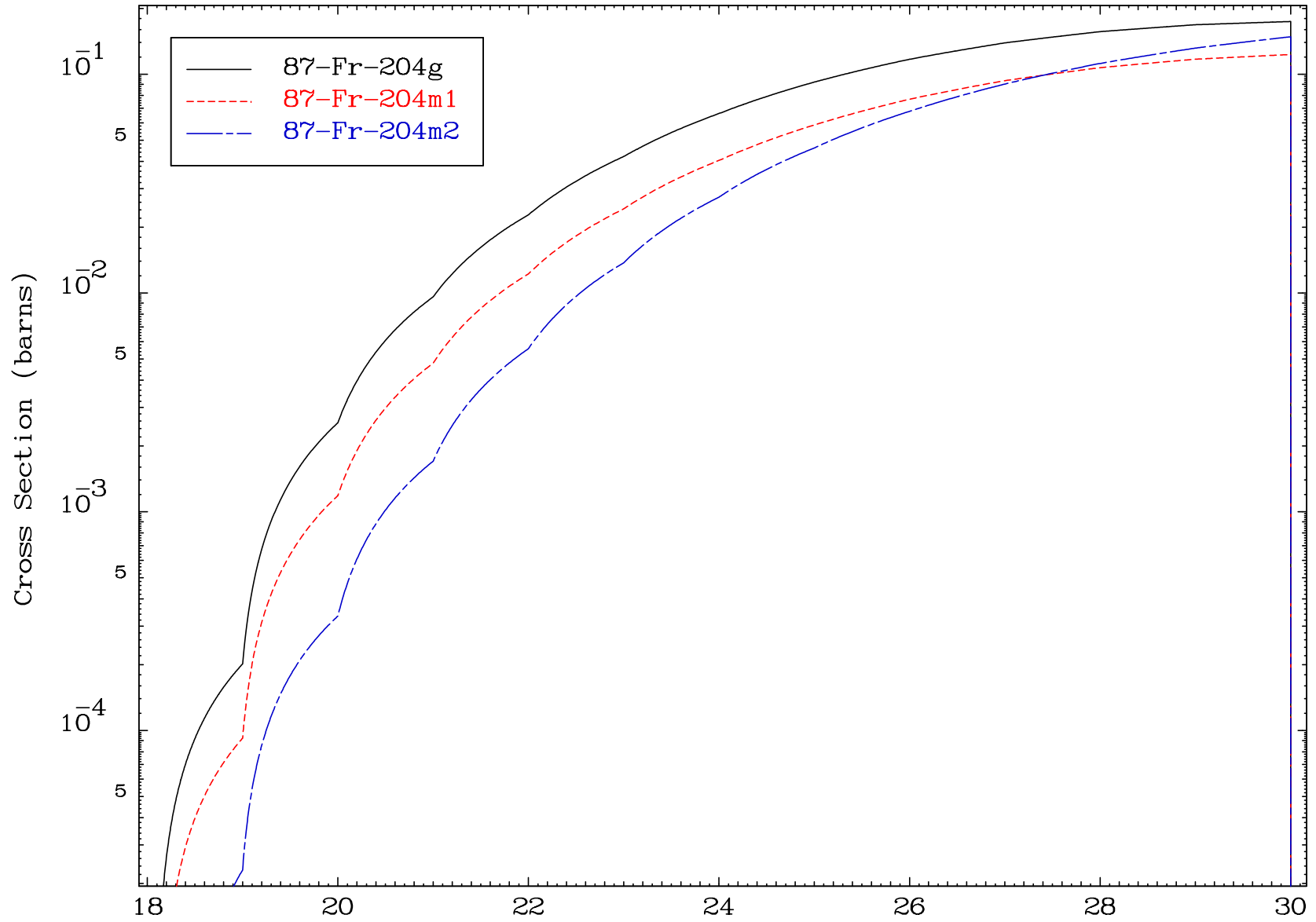


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(n,3n)

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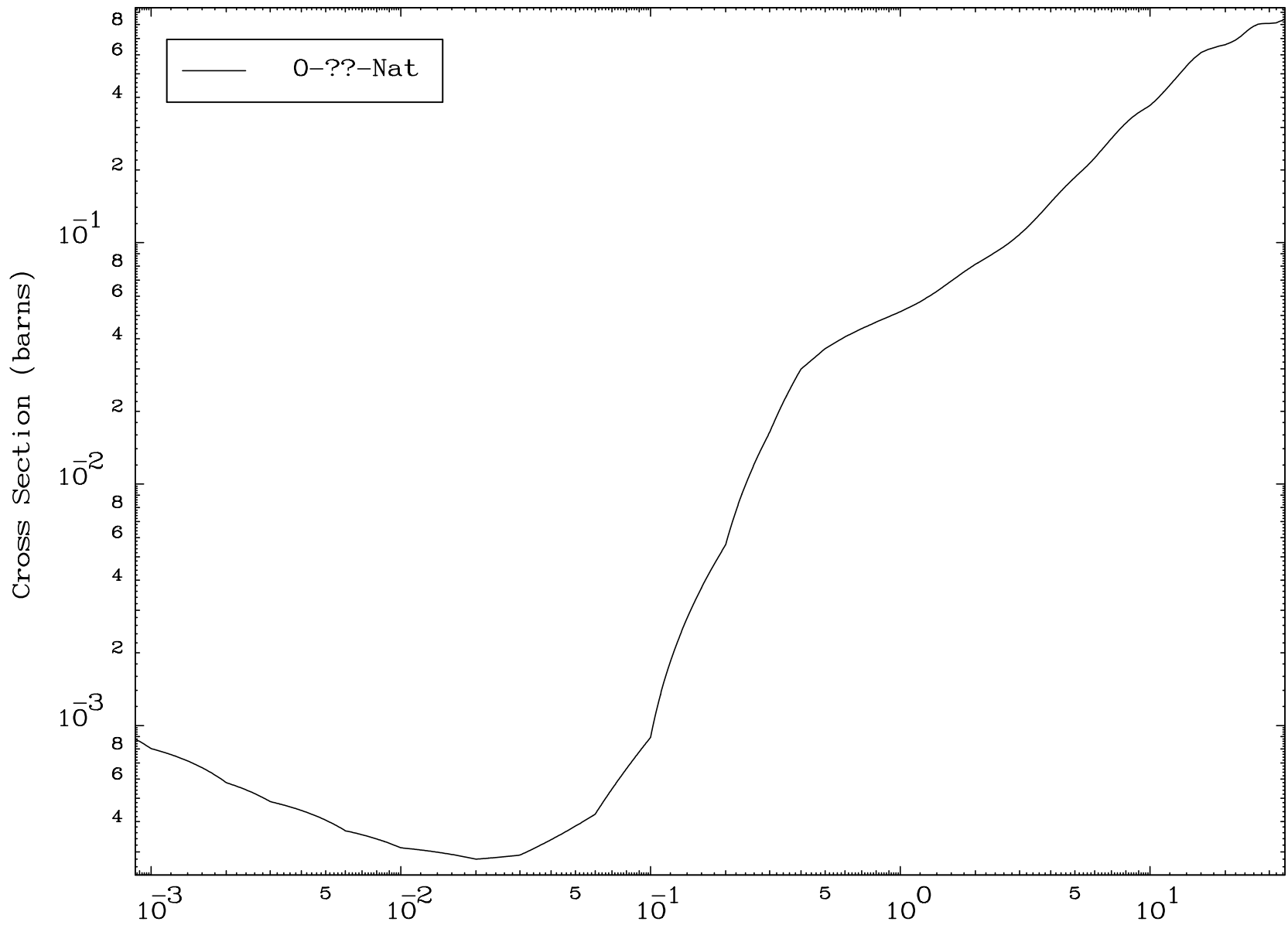
Radionuclide Production Cross Section



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Incident Energy (MeV)

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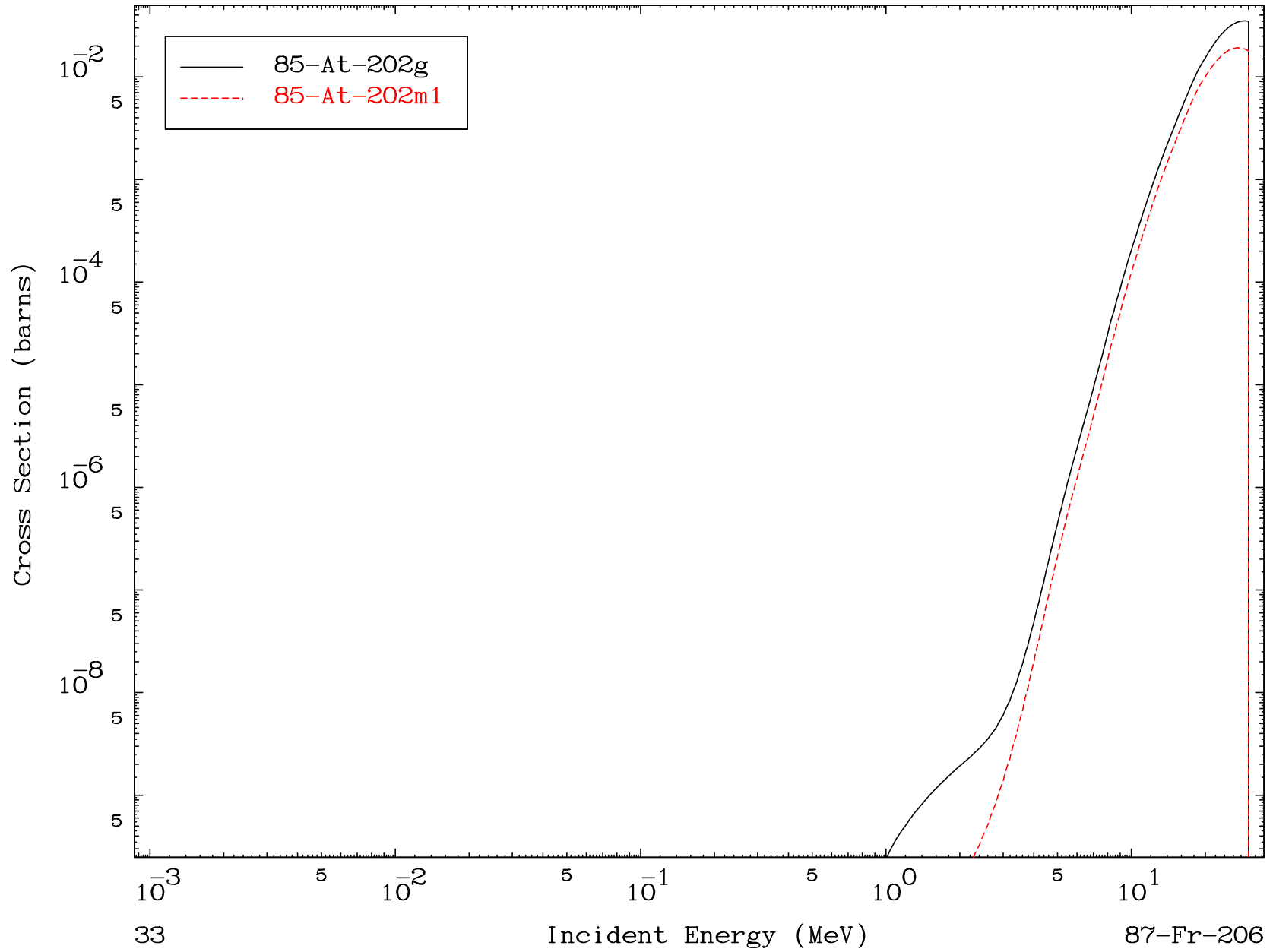




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(n,n')  $\alpha$   
Radionuclide Production Cross Section

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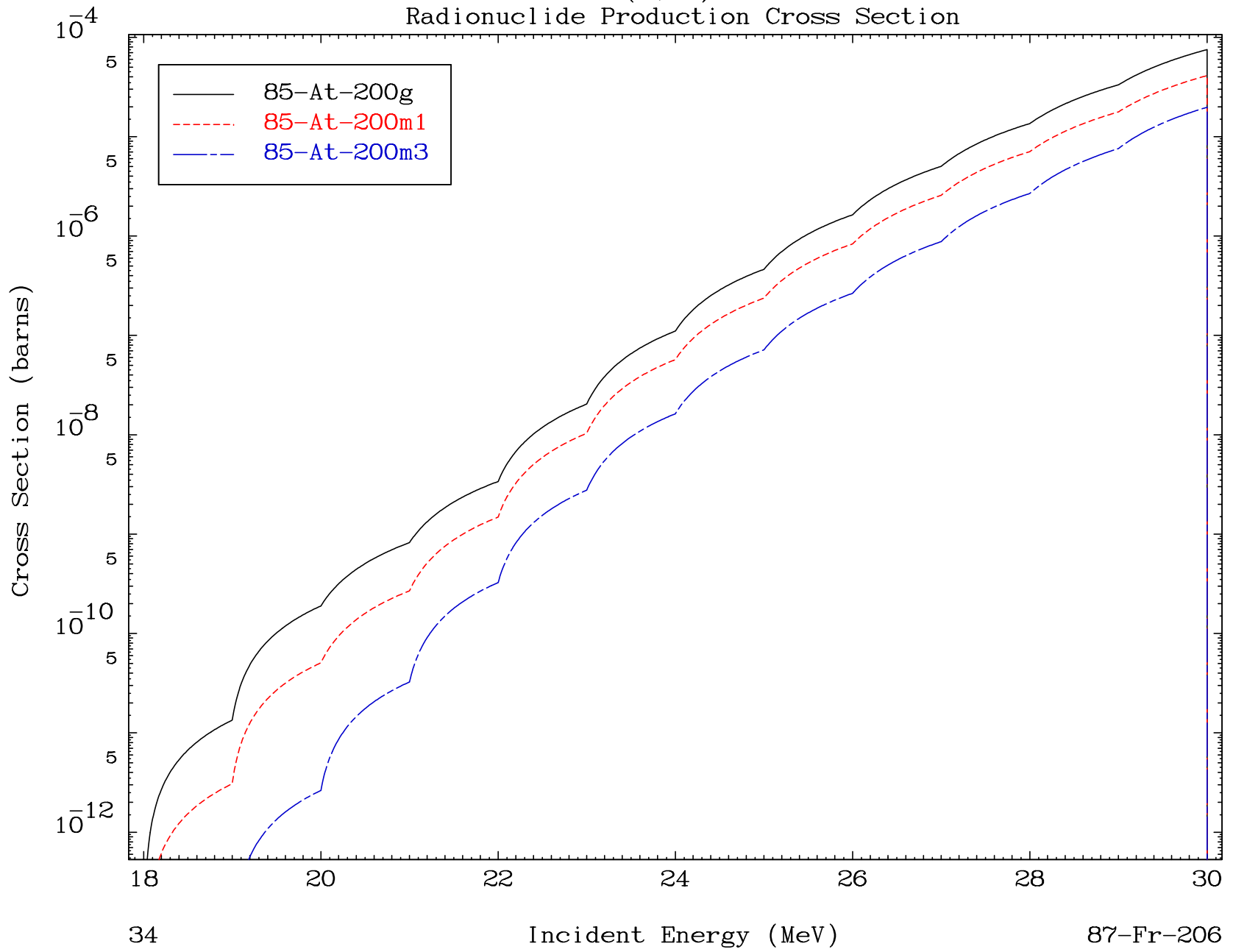


MAT 8707

(n,3n)  $\alpha$

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Radionuclide Production Cross Section

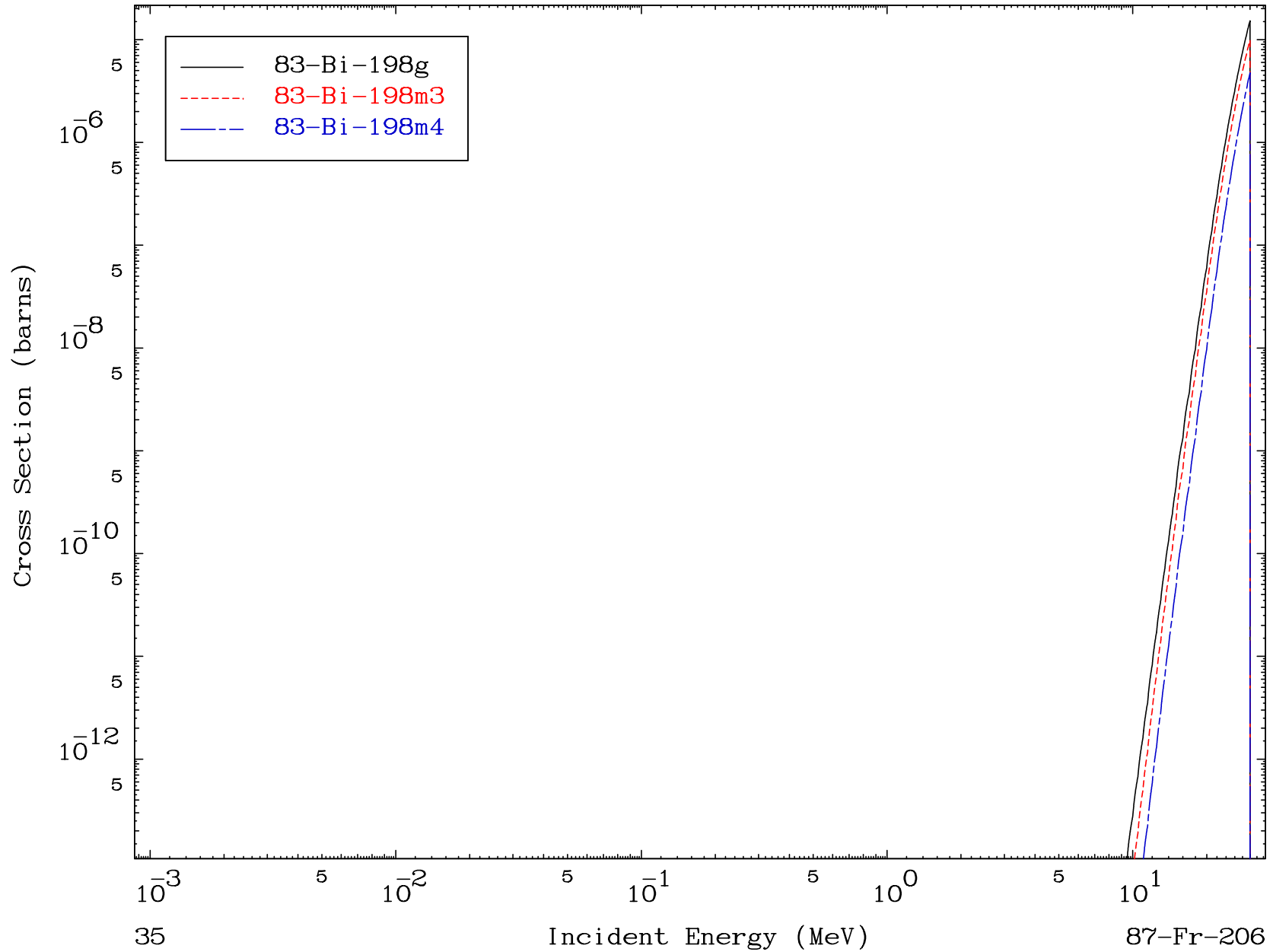


MAT 8707

(n,n') 2 $\alpha$

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Radionuclide Production Cross Section

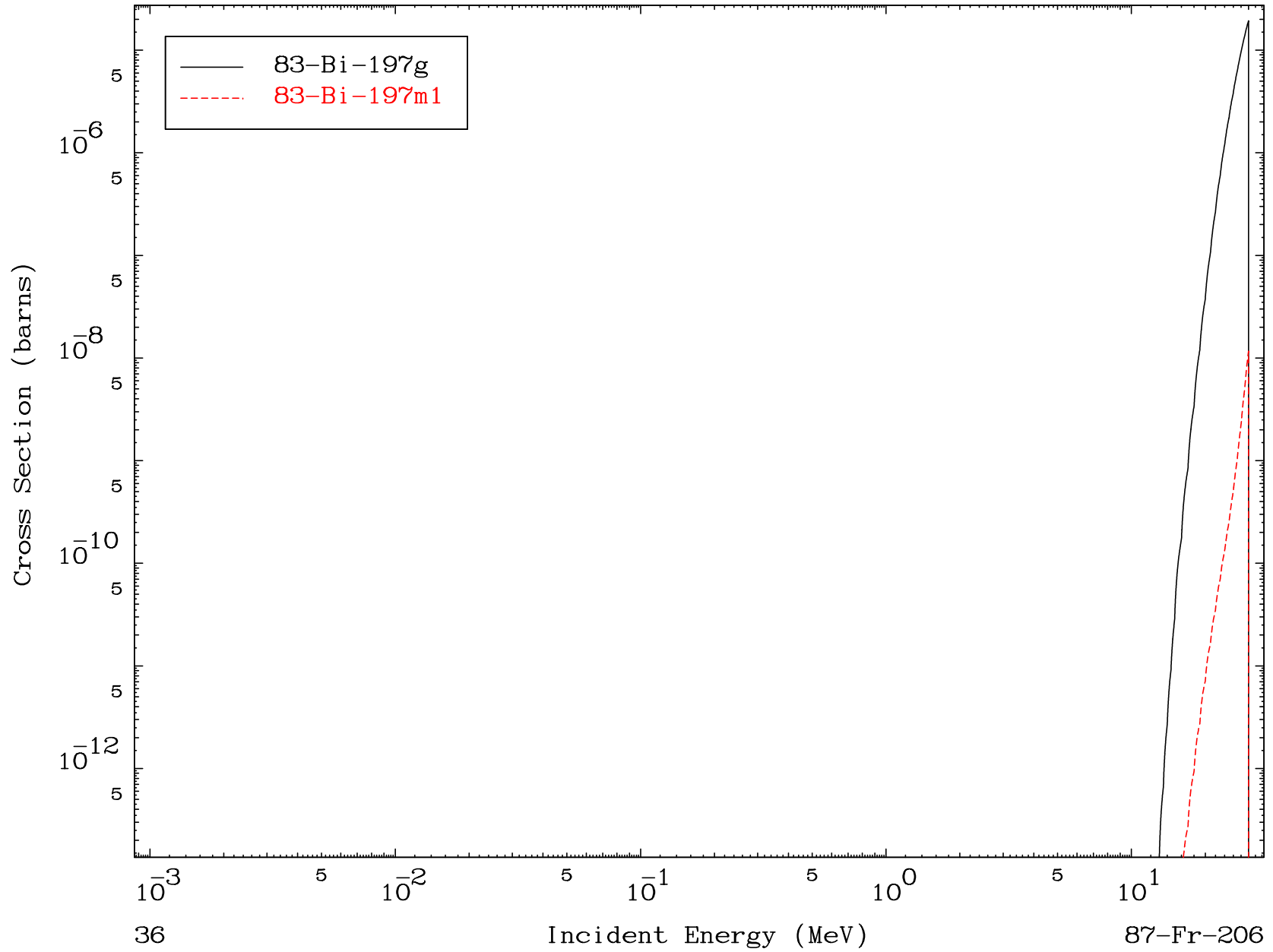


MAT 8707

(n,2n) 2 $\alpha$

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Radionuclide Production Cross Section

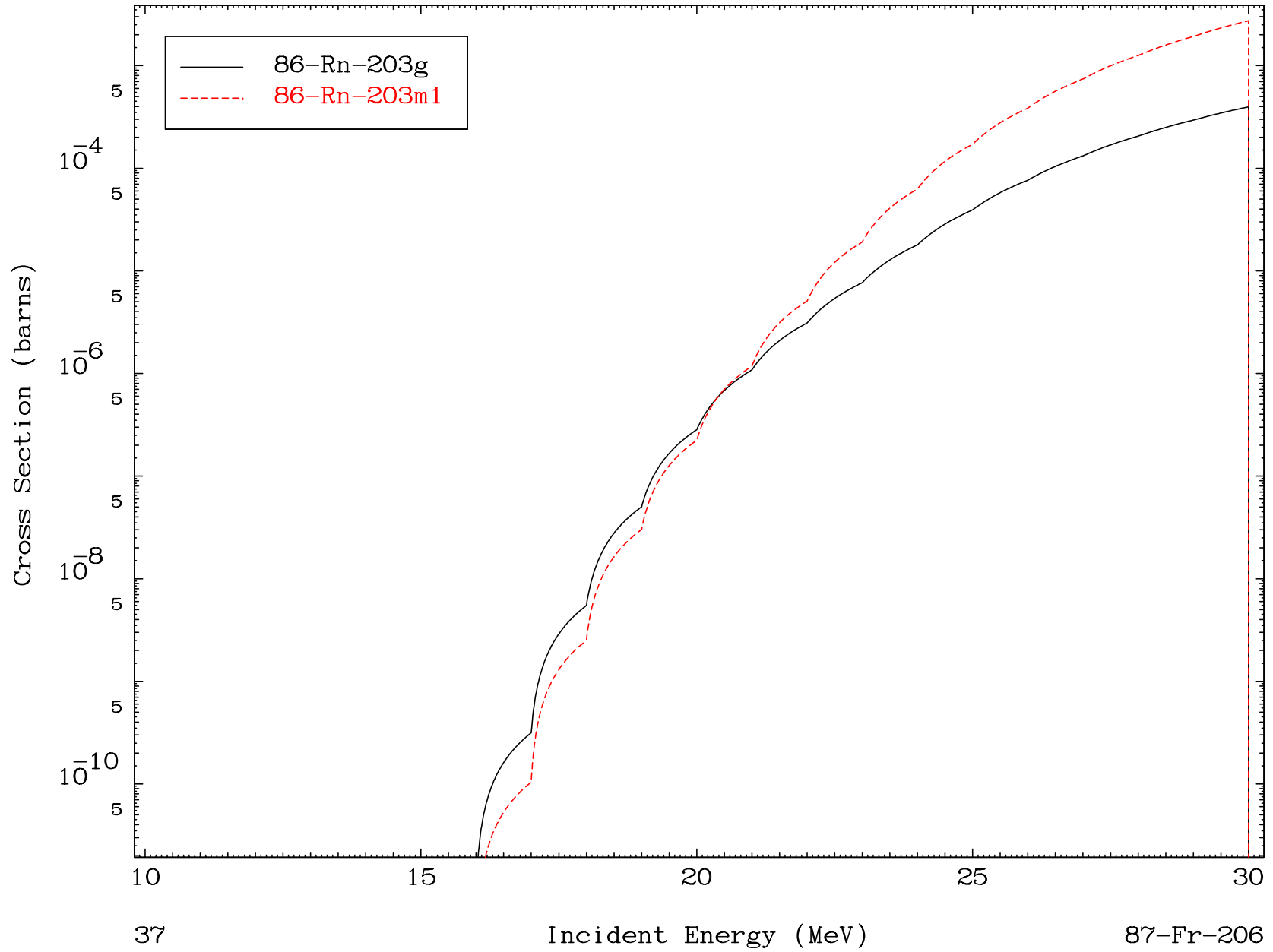


MAT 8707

(n,n') t

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Radionuclide Production Cross Section



37

Incident Energy (MeV)

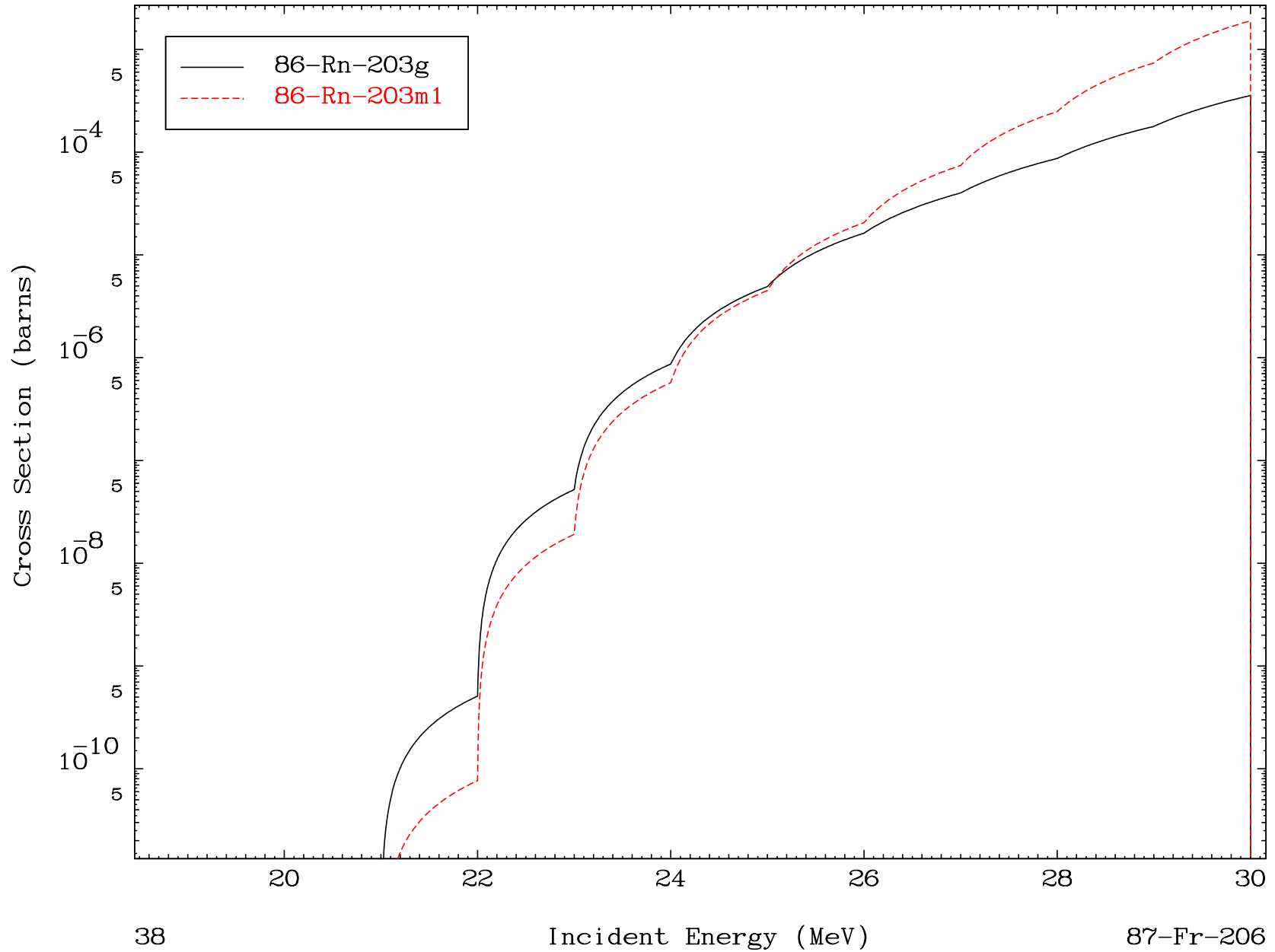
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(n,3n) p

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Radionuclide Production Cross Section

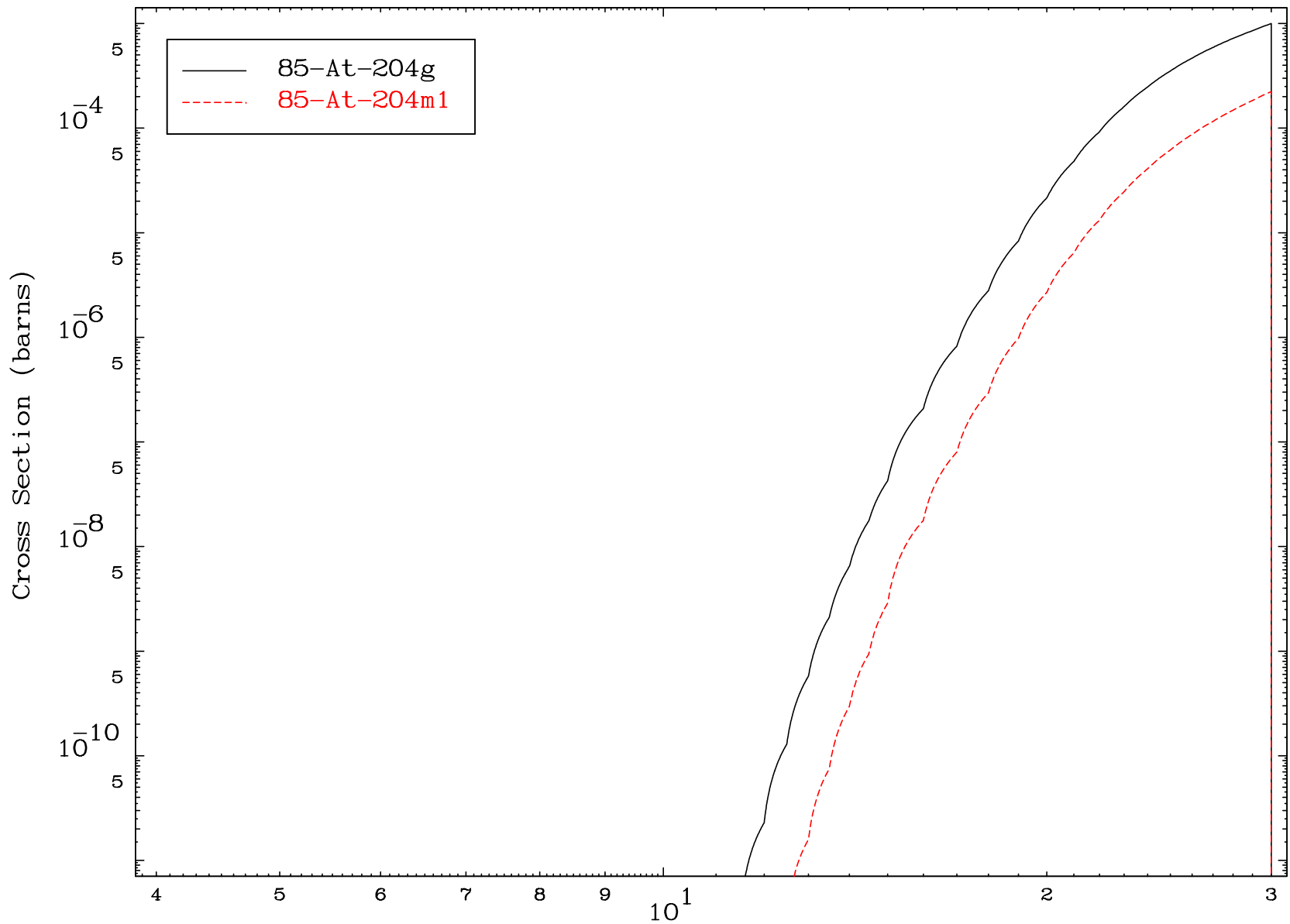


MAT 8707

(n,2n) p

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Radionuclide Production Cross Section



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Incident Energy (MeV)

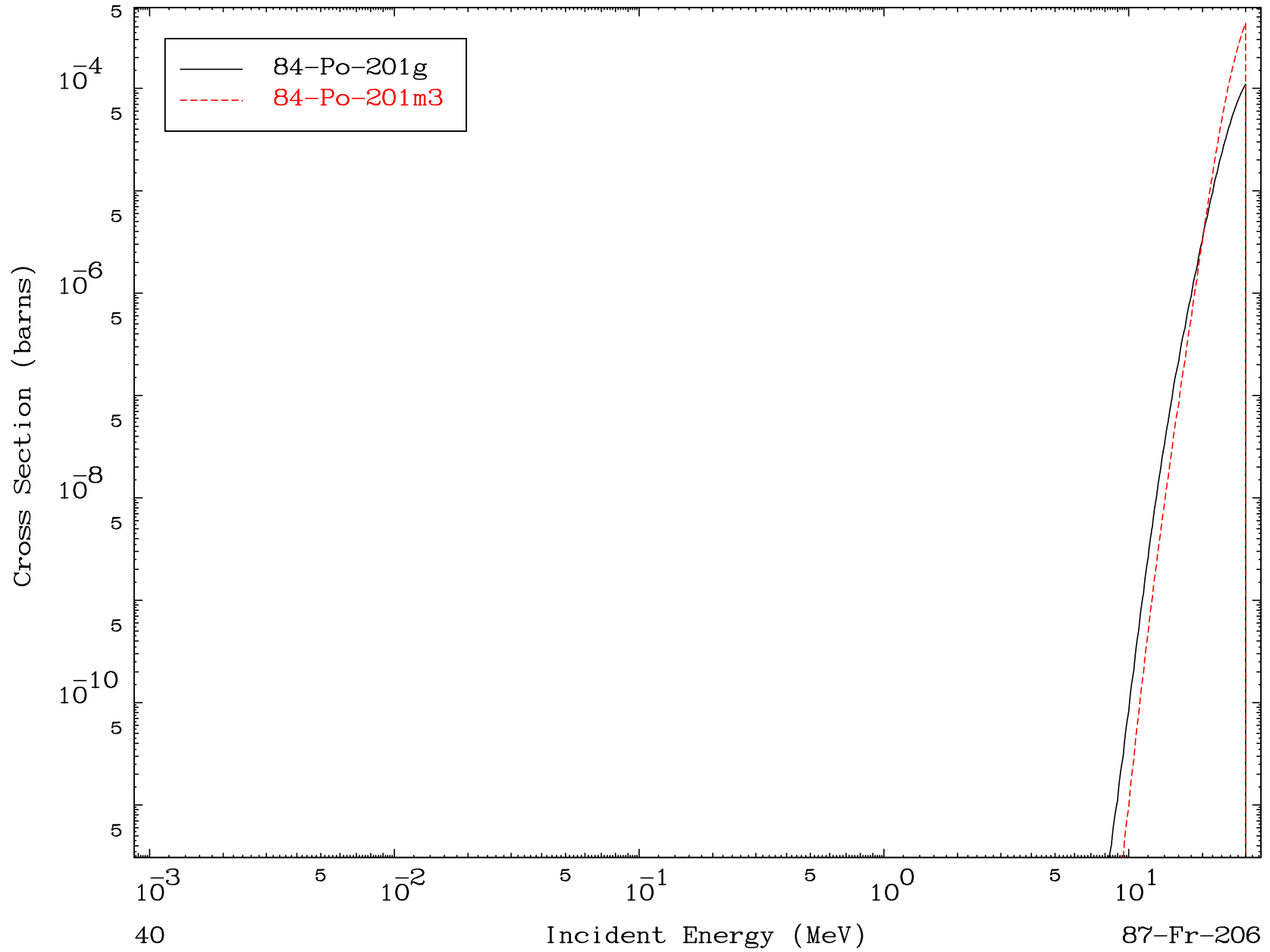
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MAT 8707

(n,n') p  $\alpha$

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Radionuclide Production Cross Section



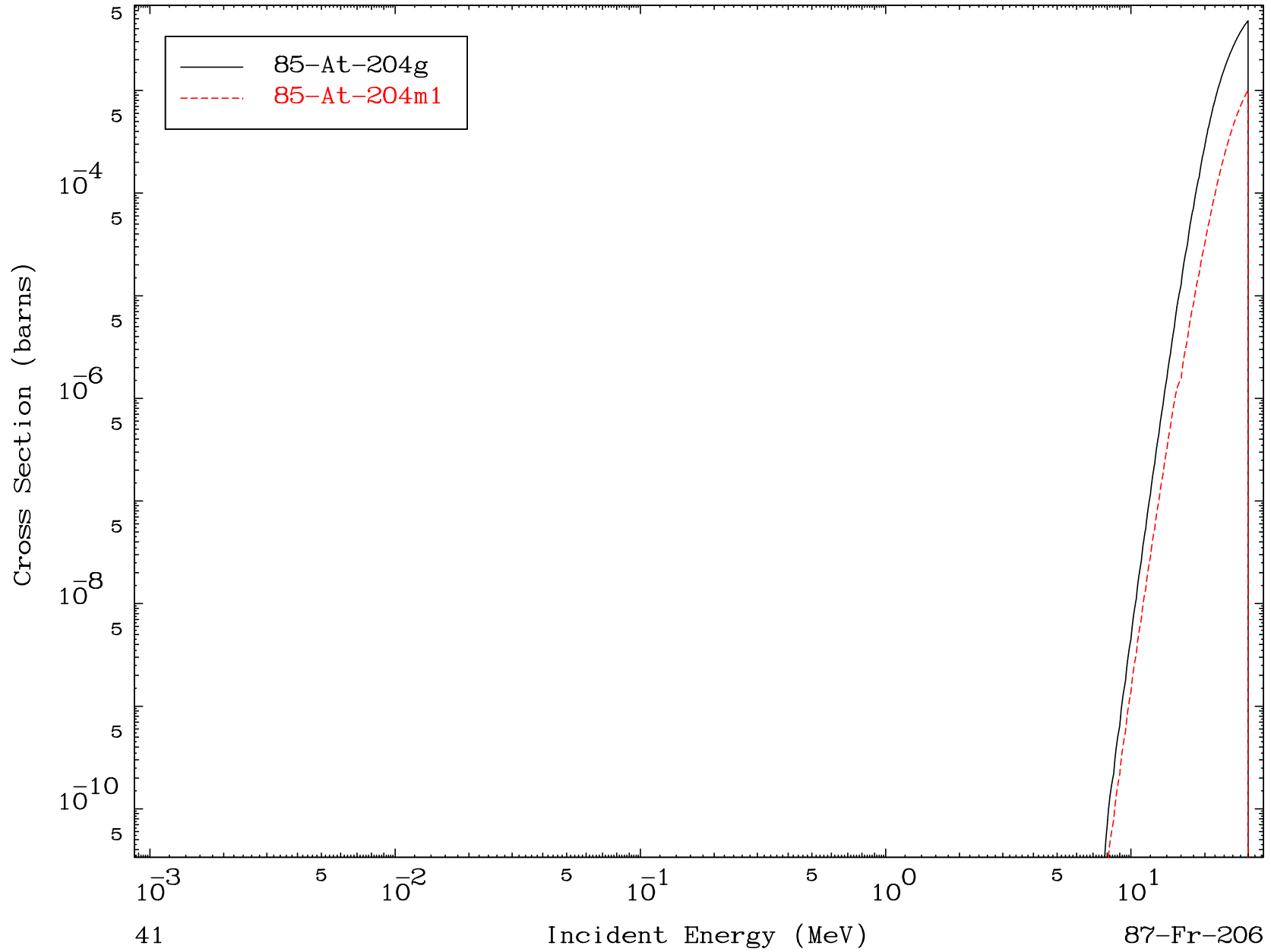


MAT 8707

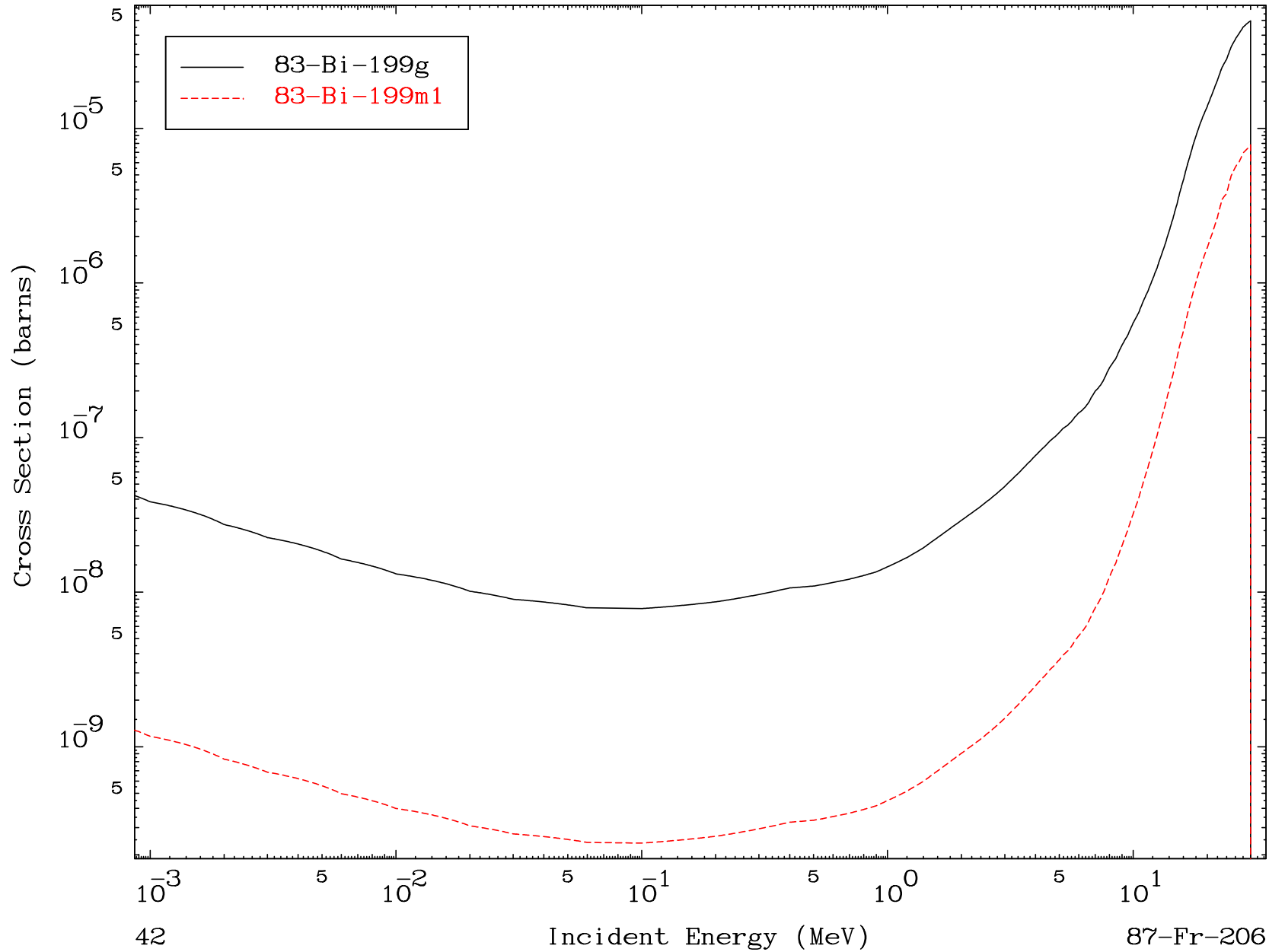
(n,He-3)

87-Fr-206

Radionuclide Production Cross Section



Radionuclide Production Cross Section

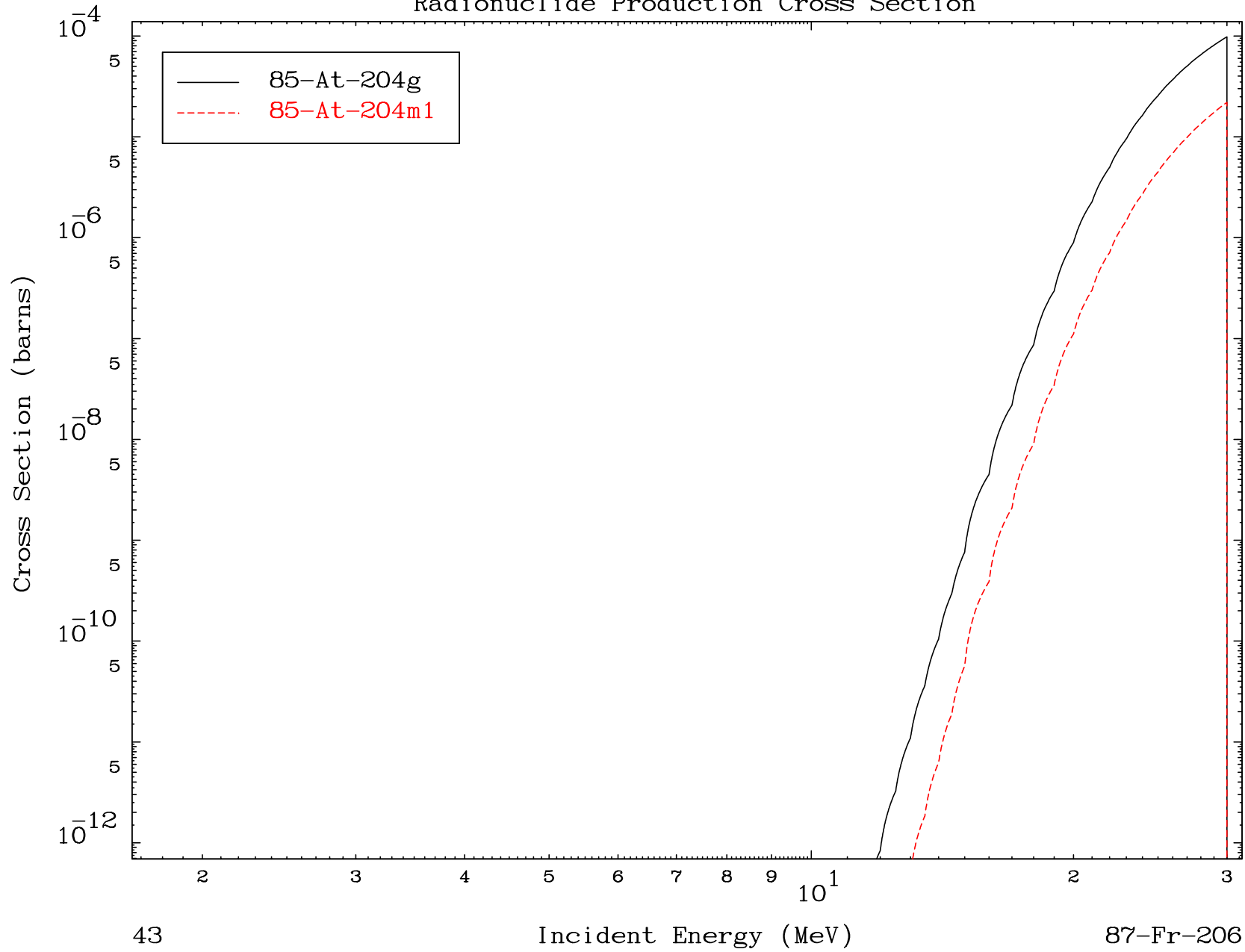


MAT 8707

(n,p) d

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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(n,d)  $\alpha$

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Radionuclide Production Cross Section

