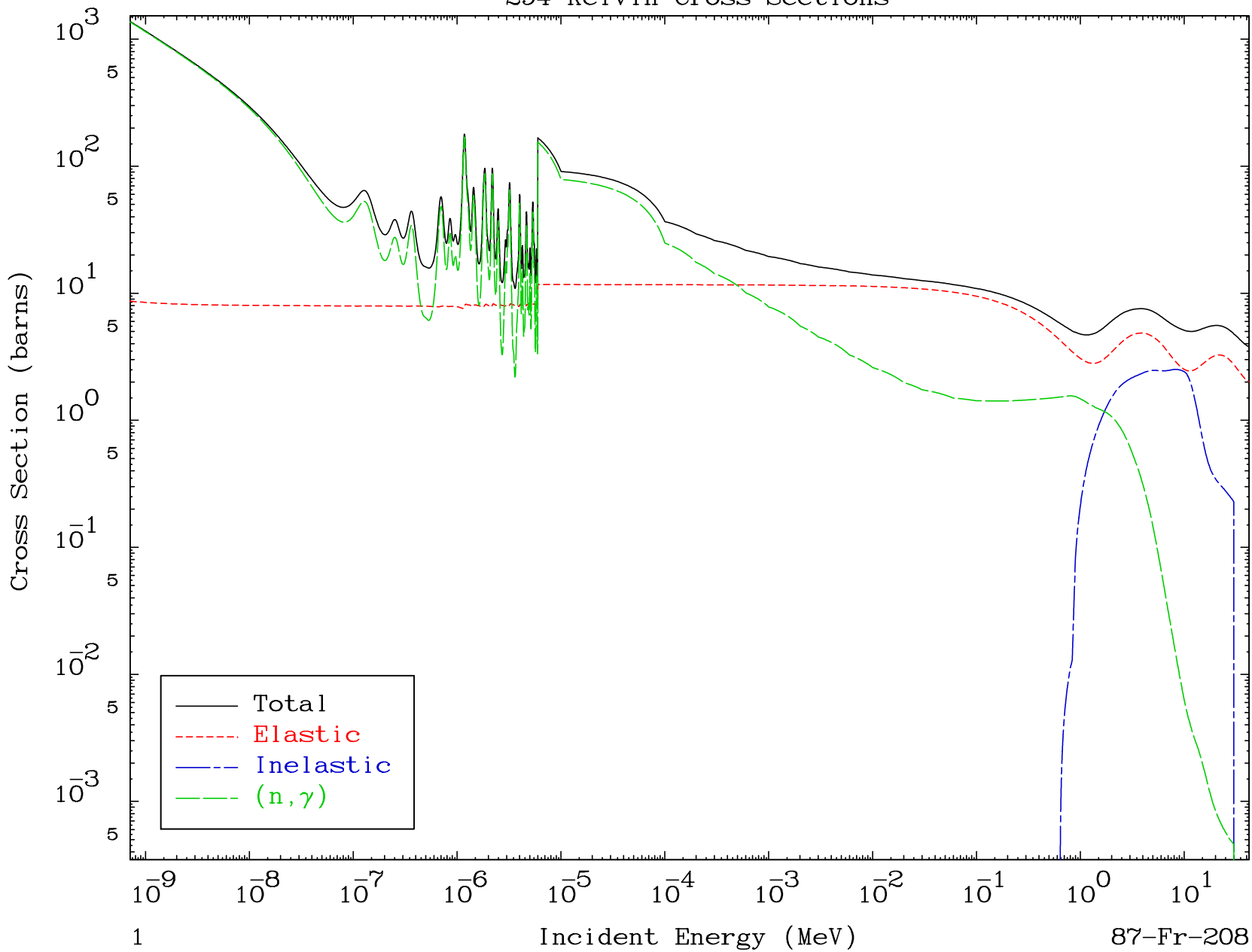
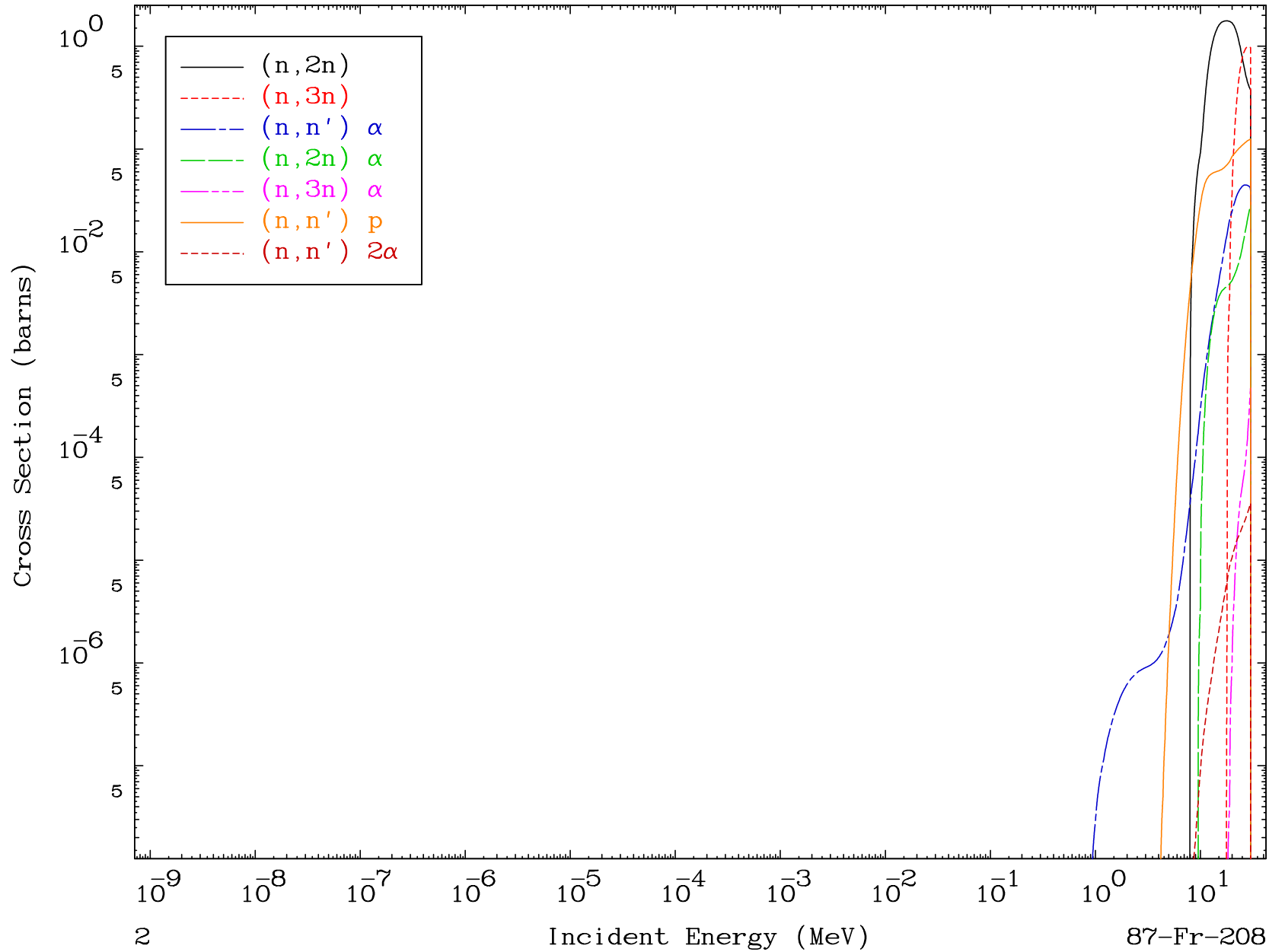


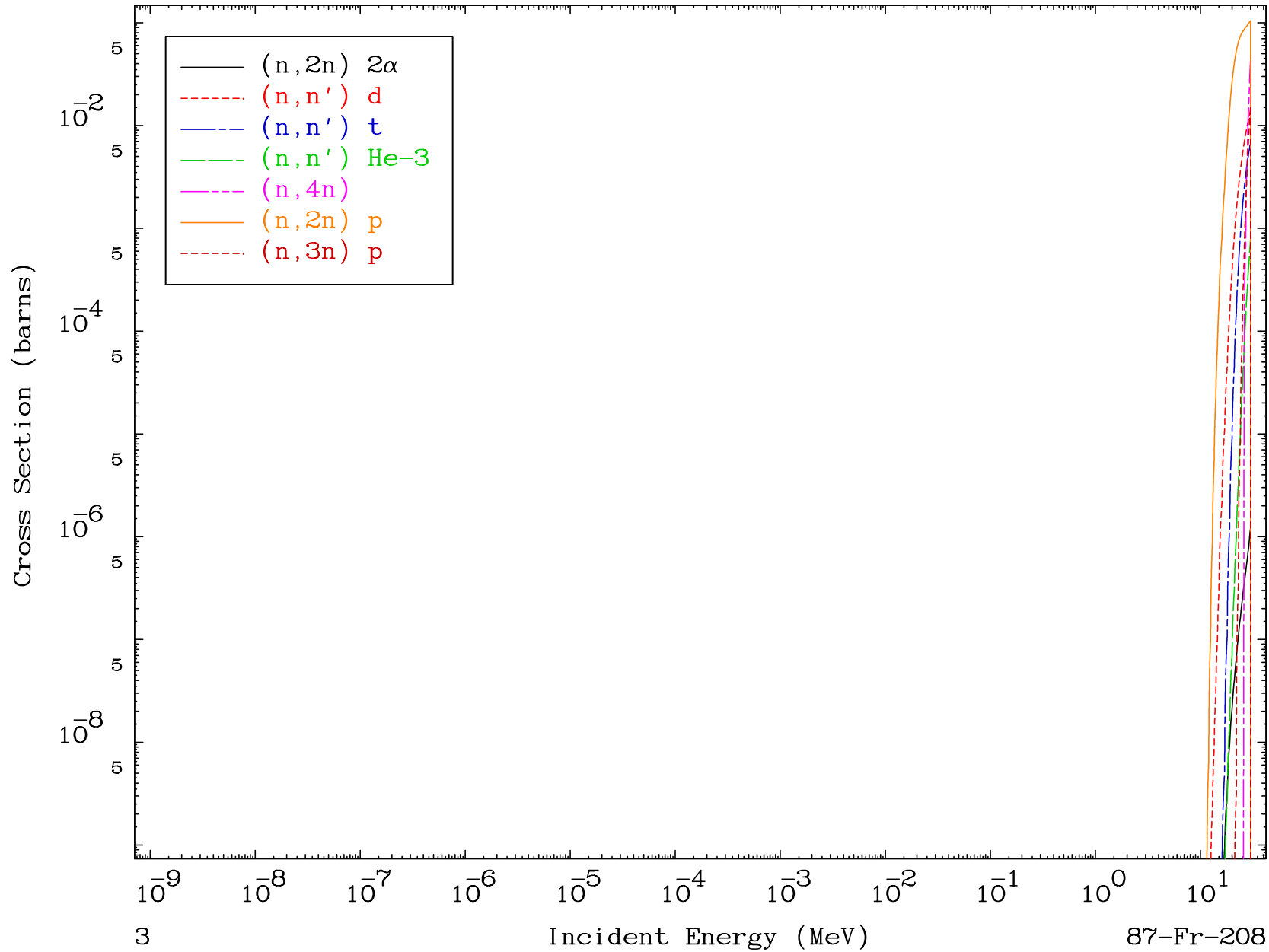
MAT 8713

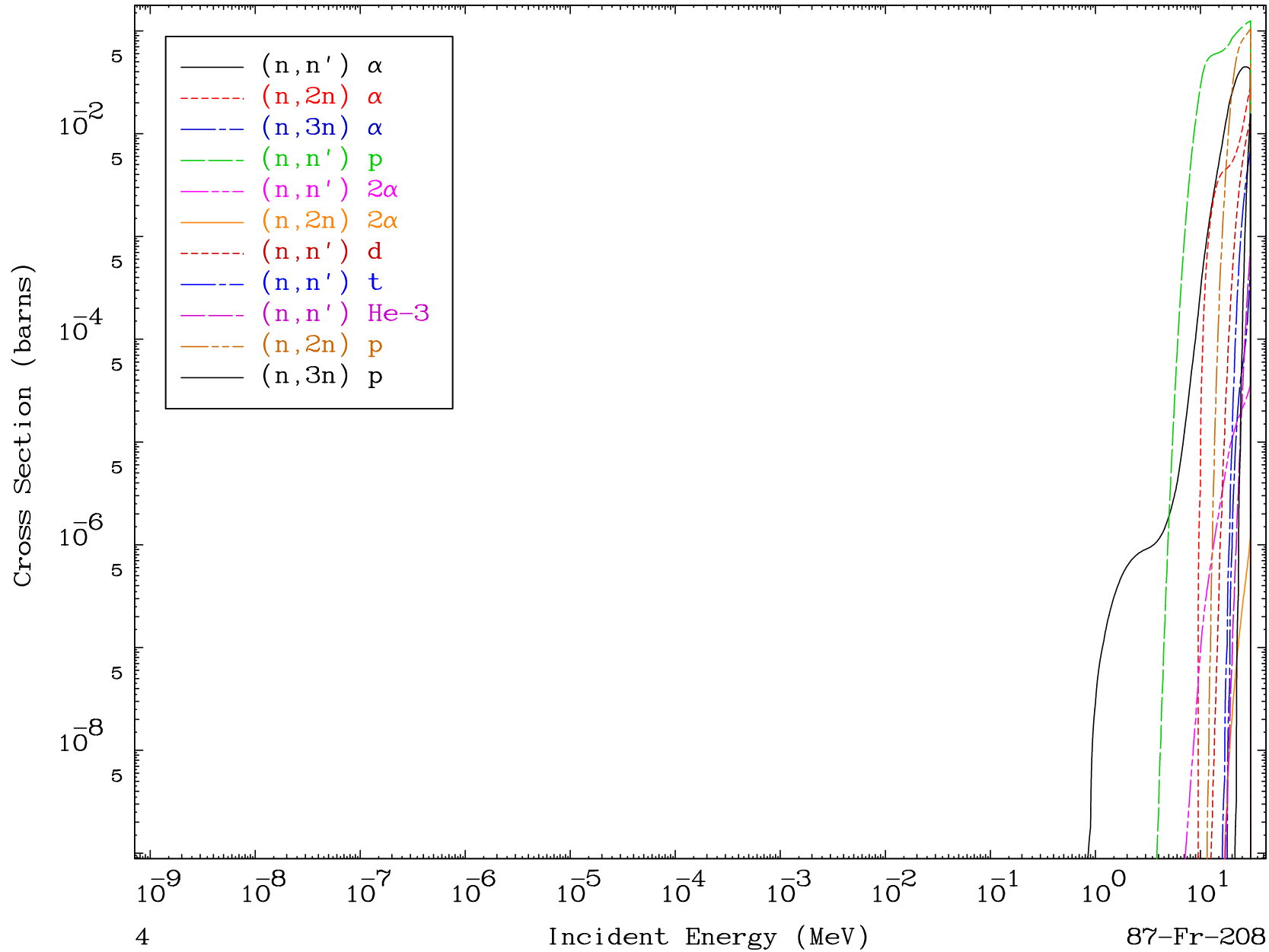
Major  
294 Kelvin Cross Sections

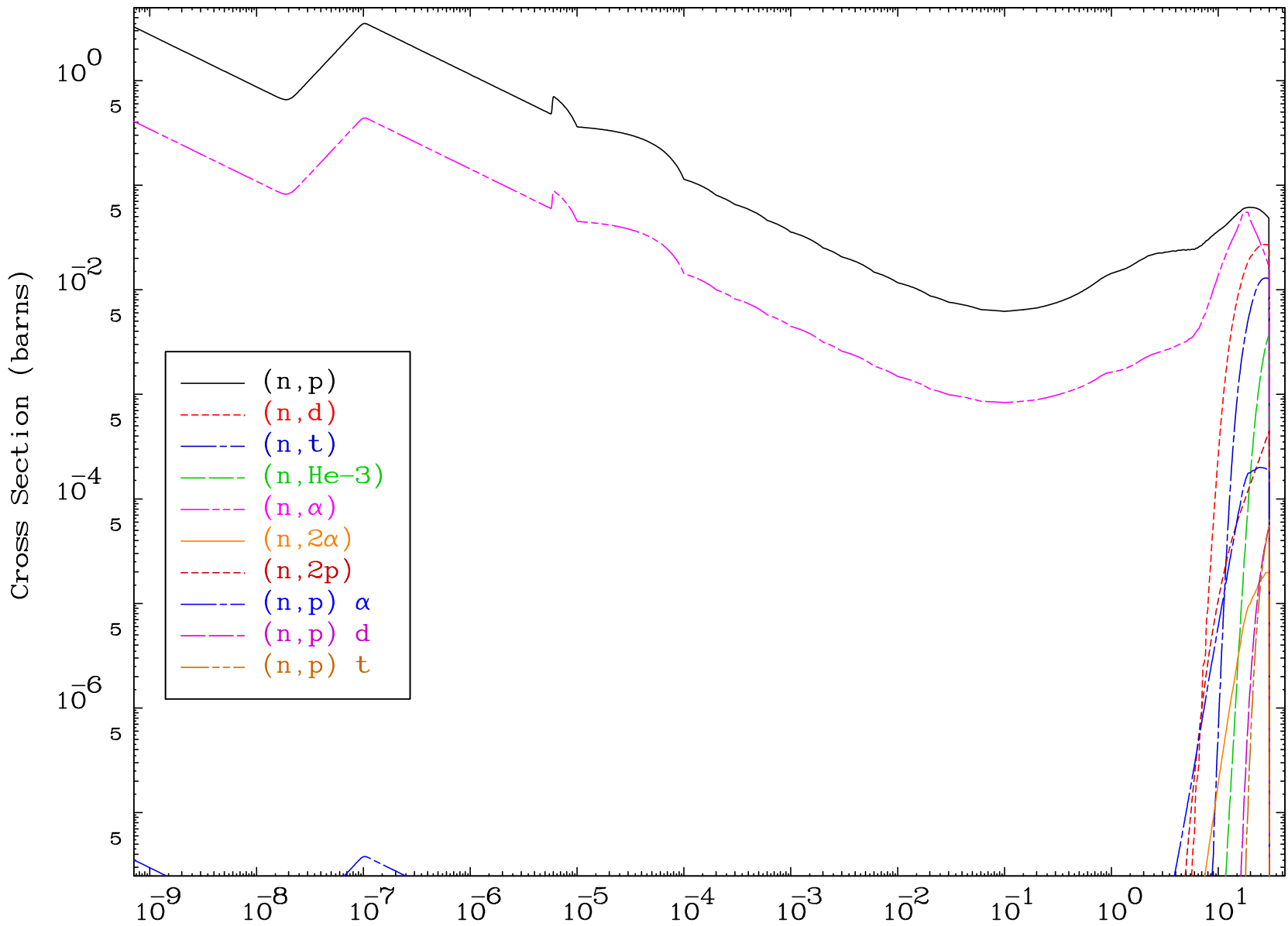
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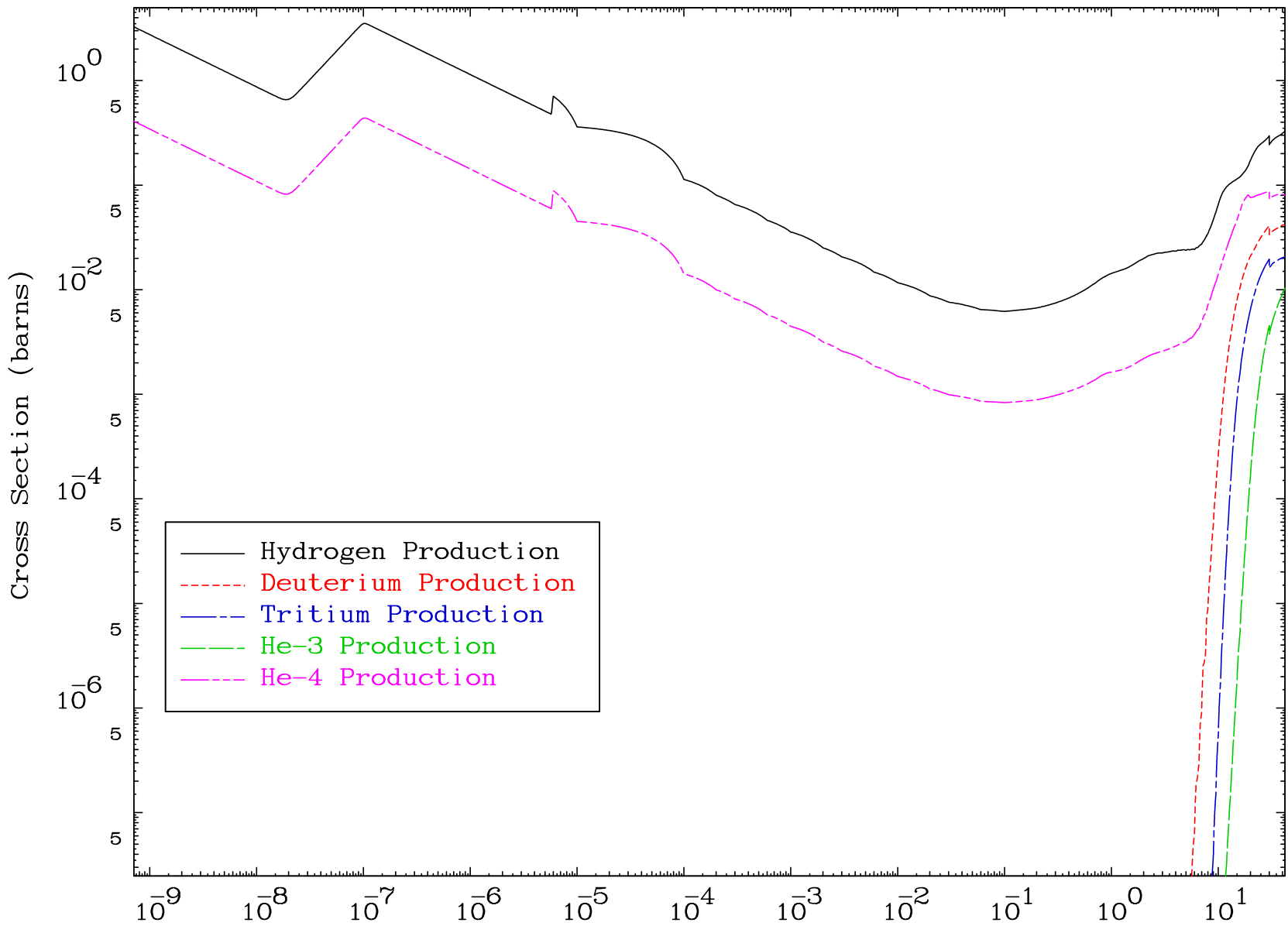


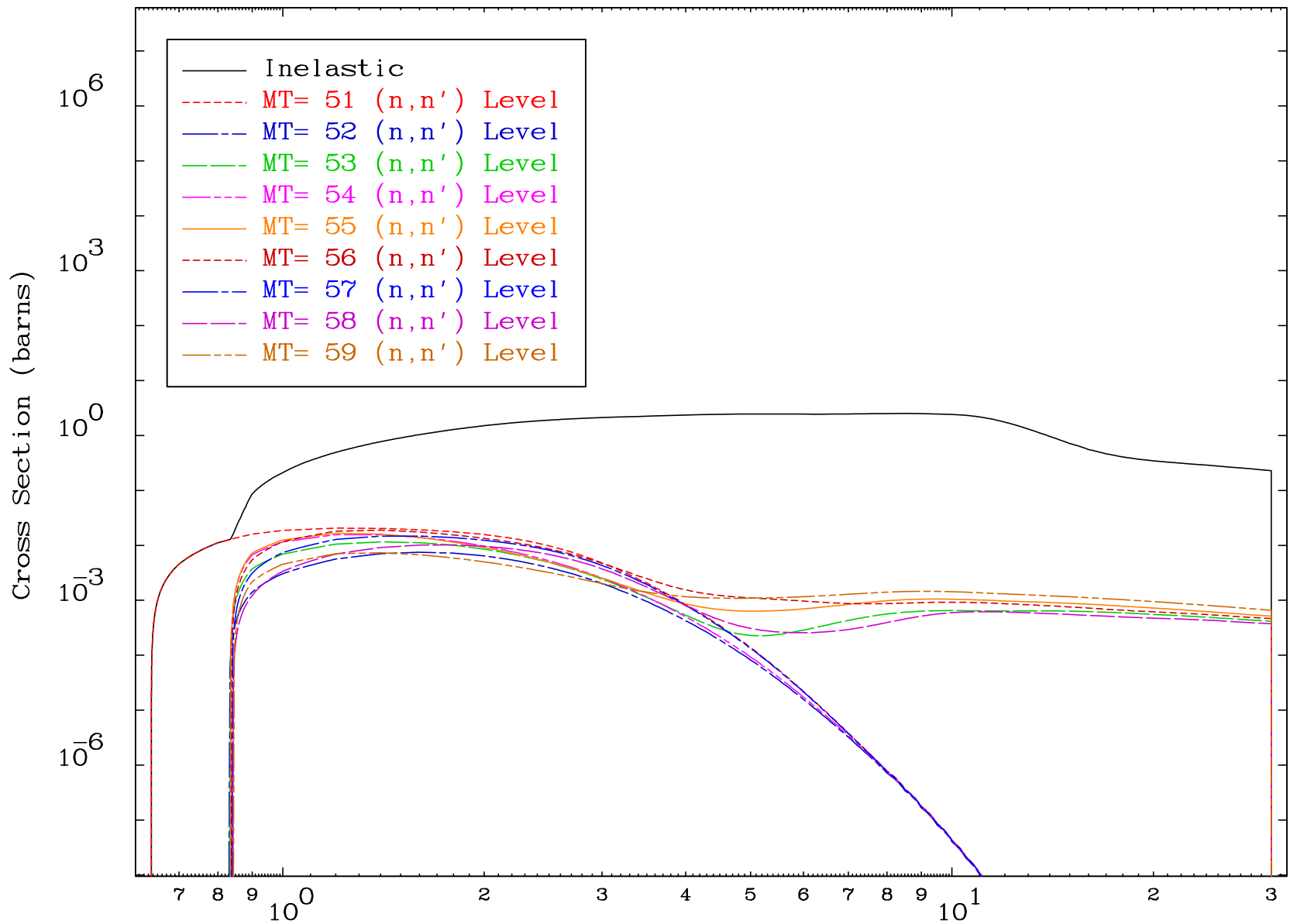


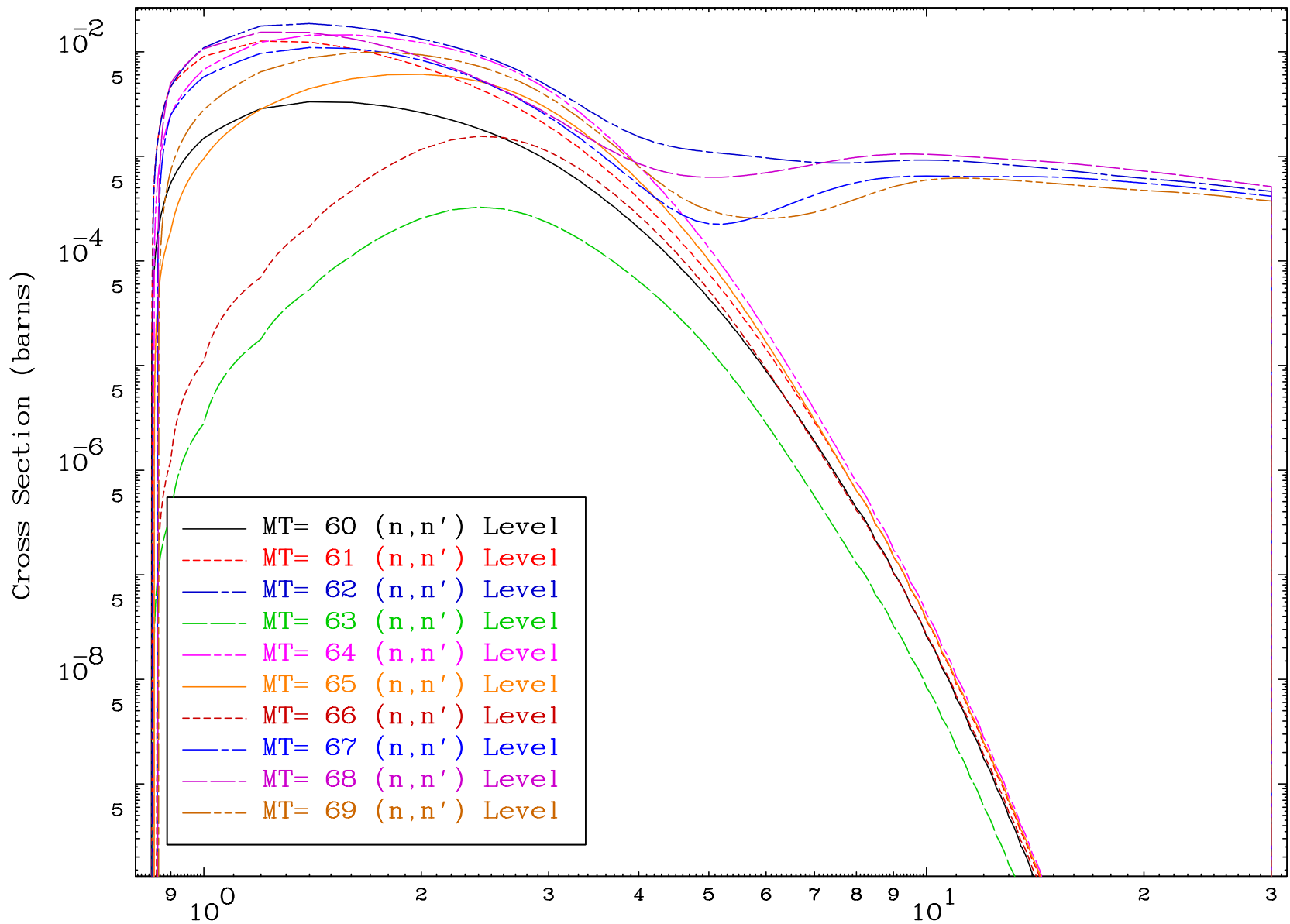




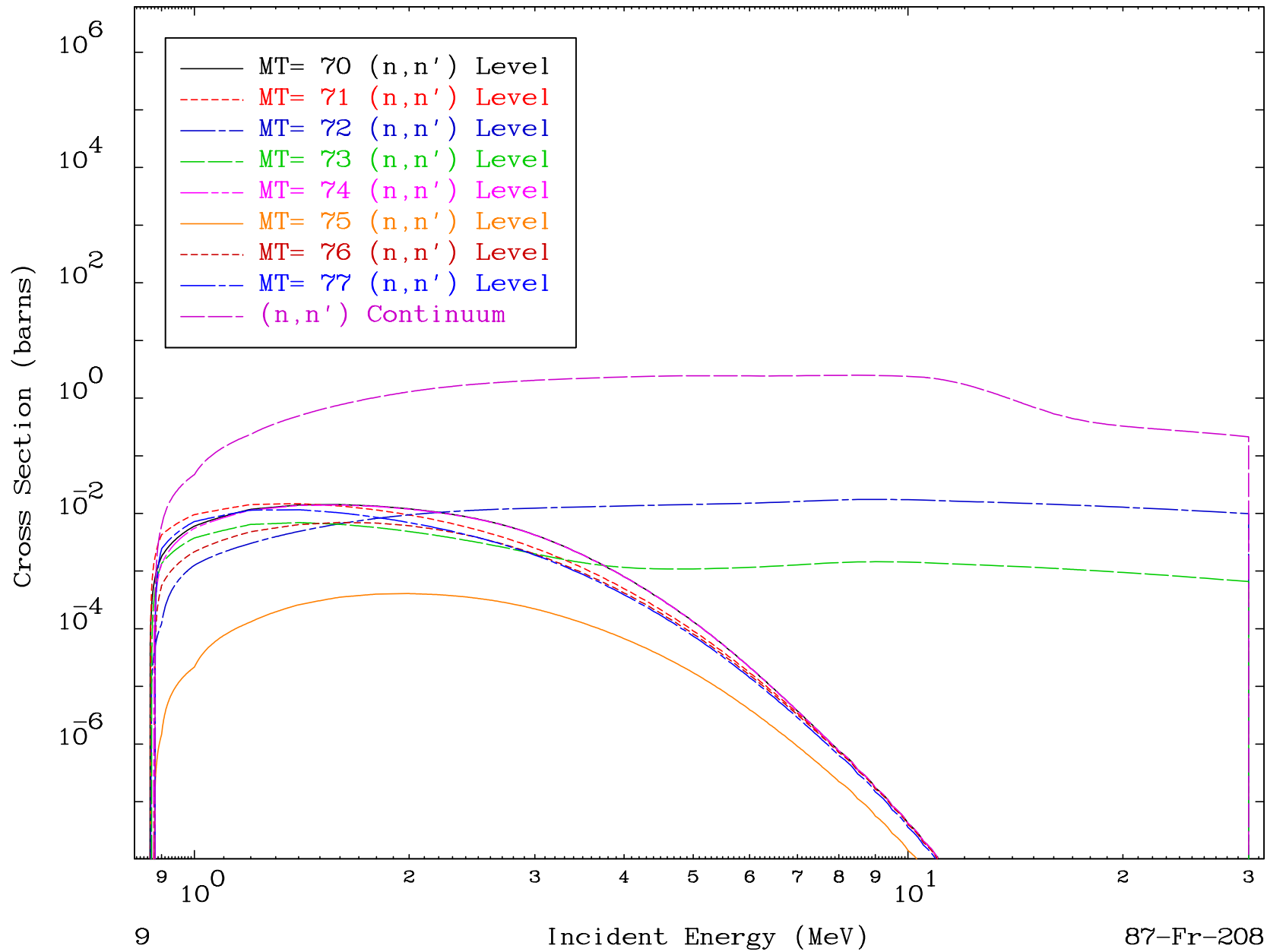








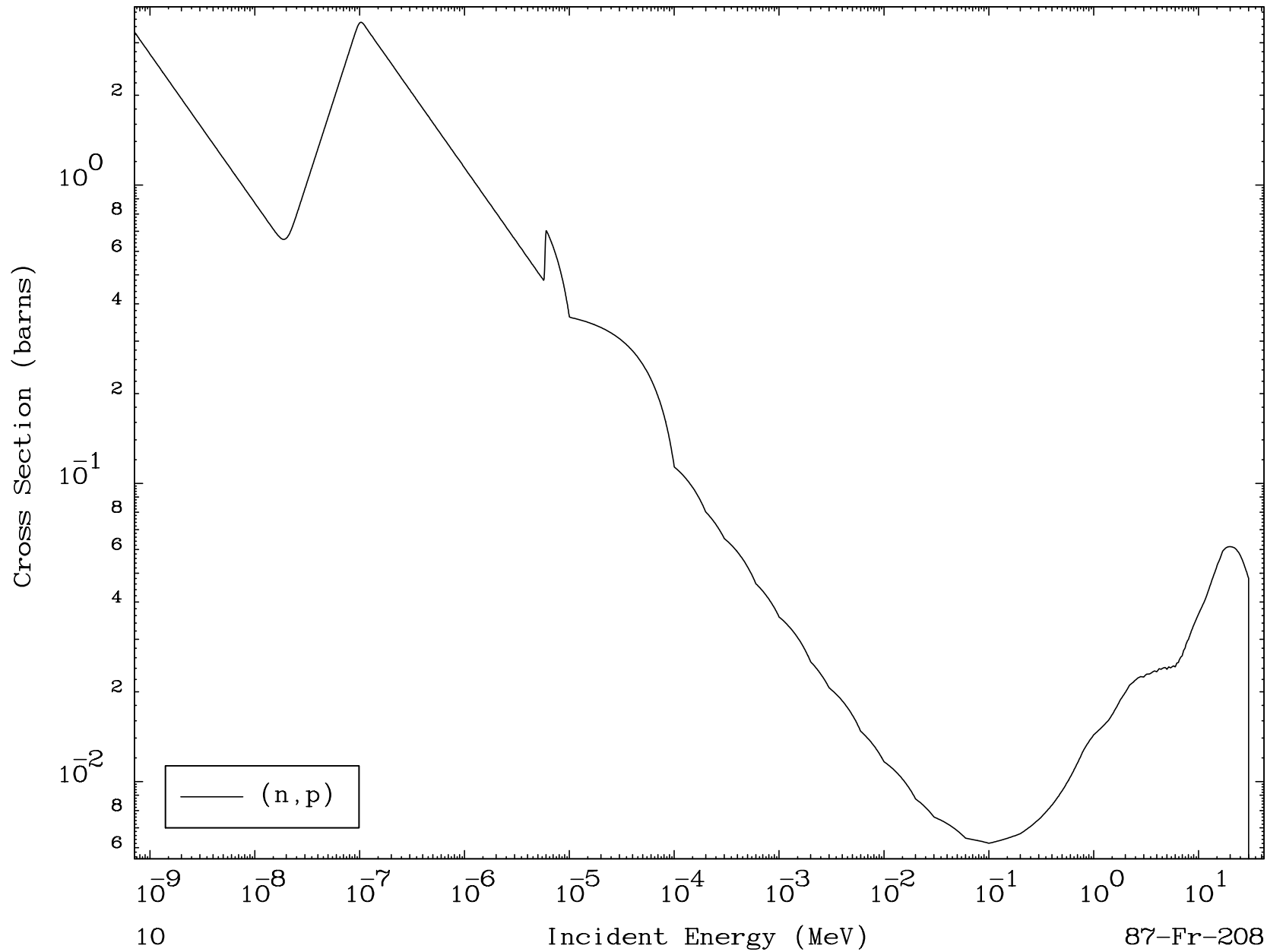


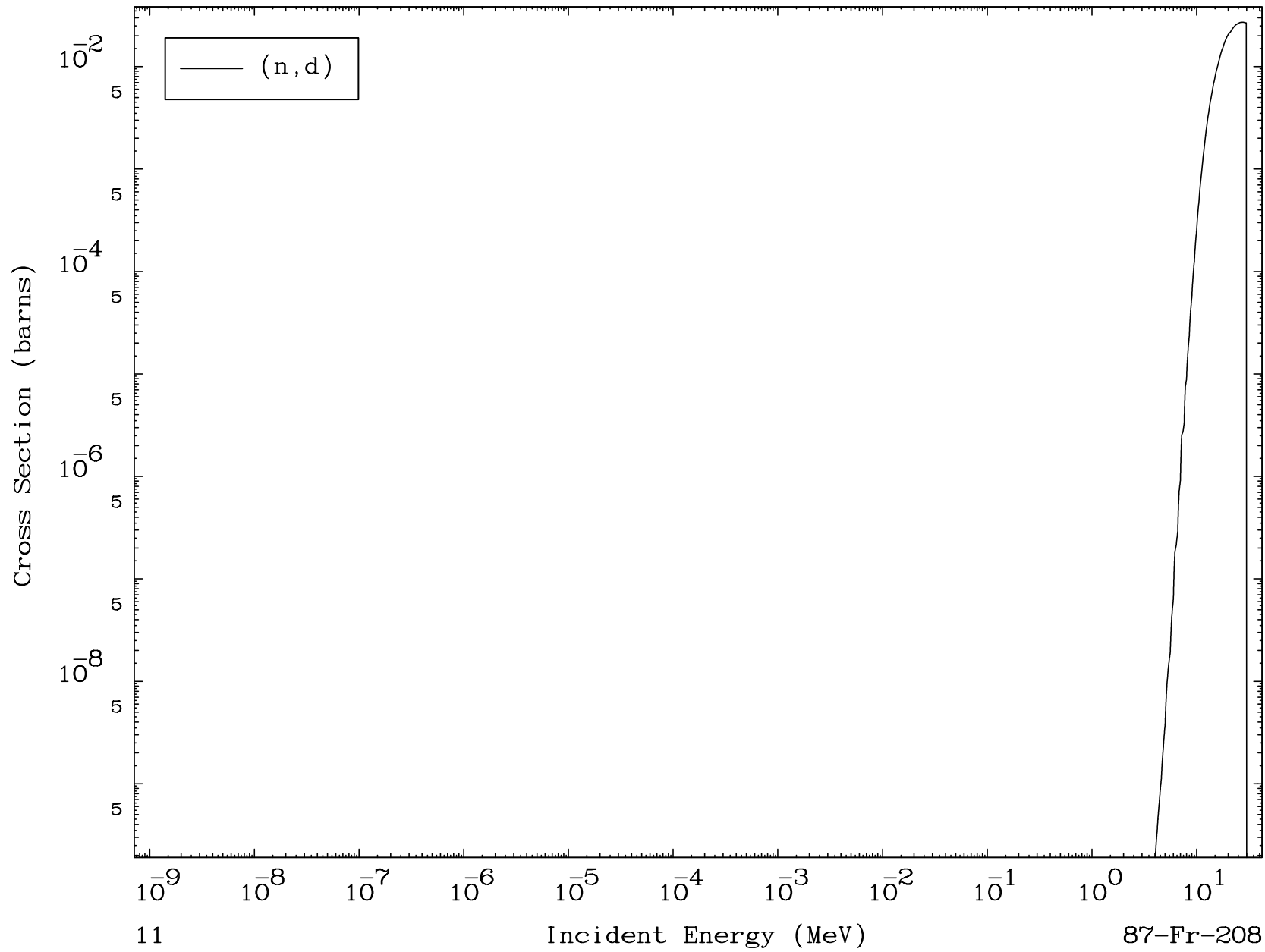


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(n,p) Levels  
294 Kelvin Cross Sections

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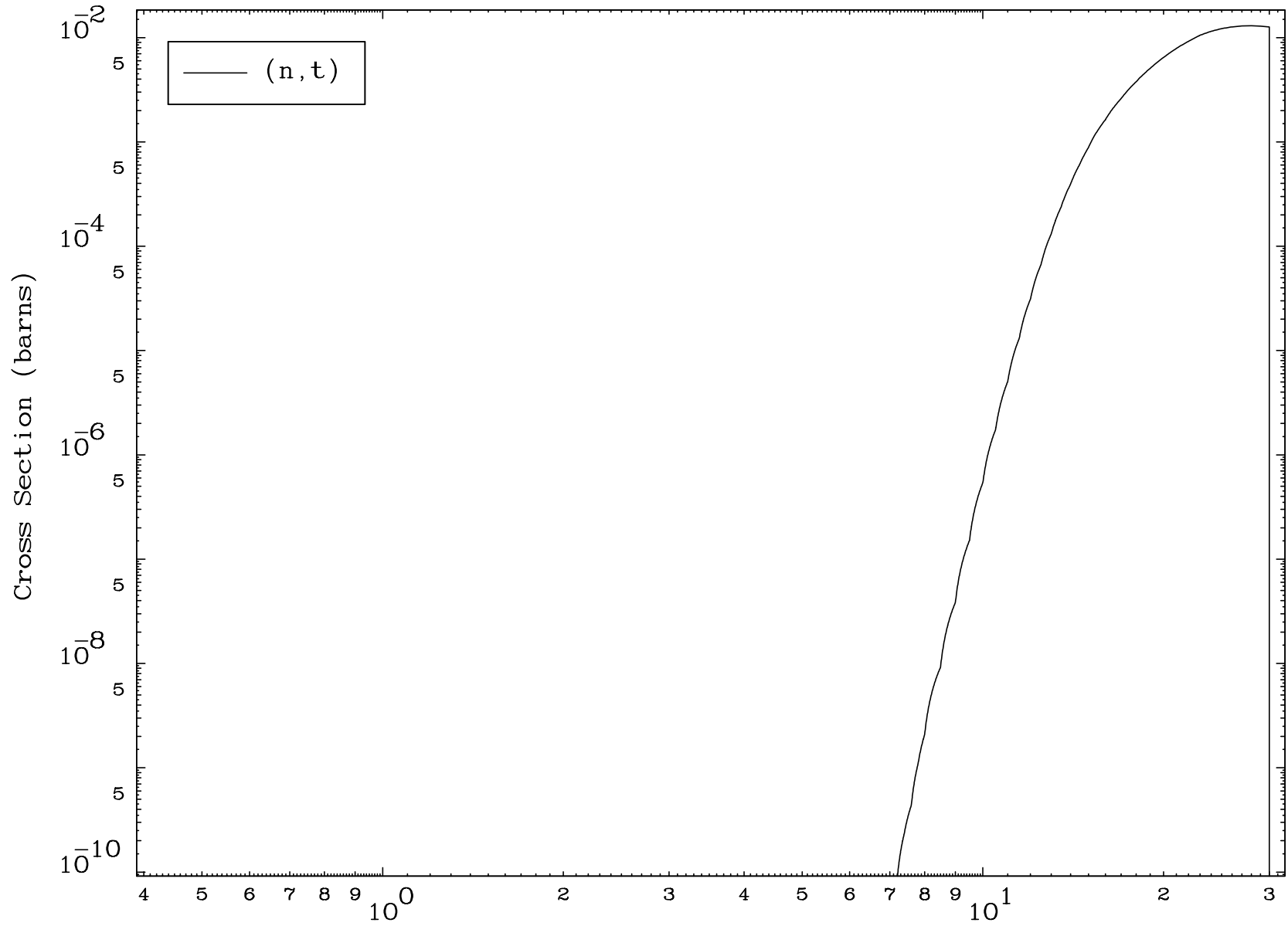




MAT 8713

(n,t) Levels  
294 Kelvin Cross Sections

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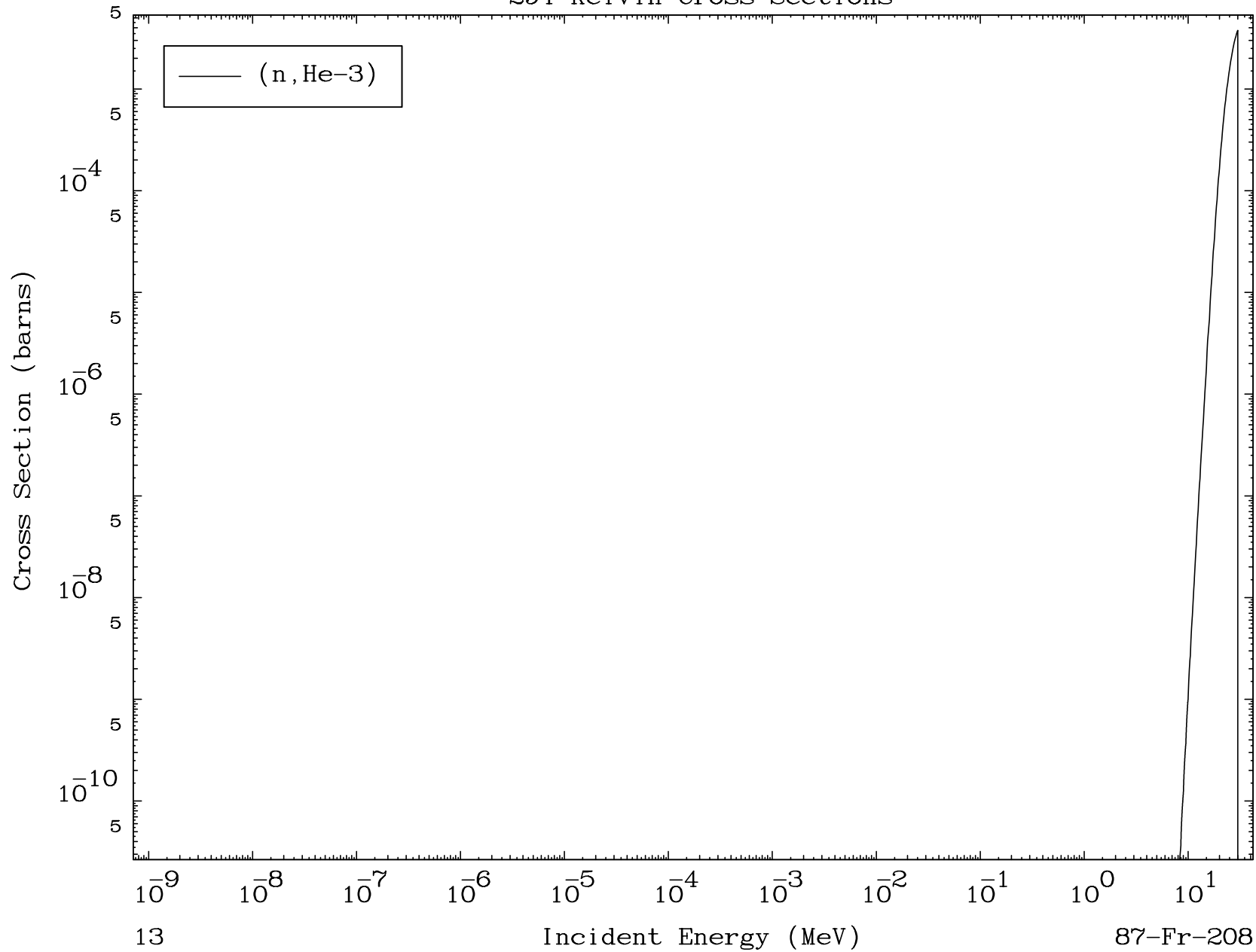
Incident Energy (MeV)

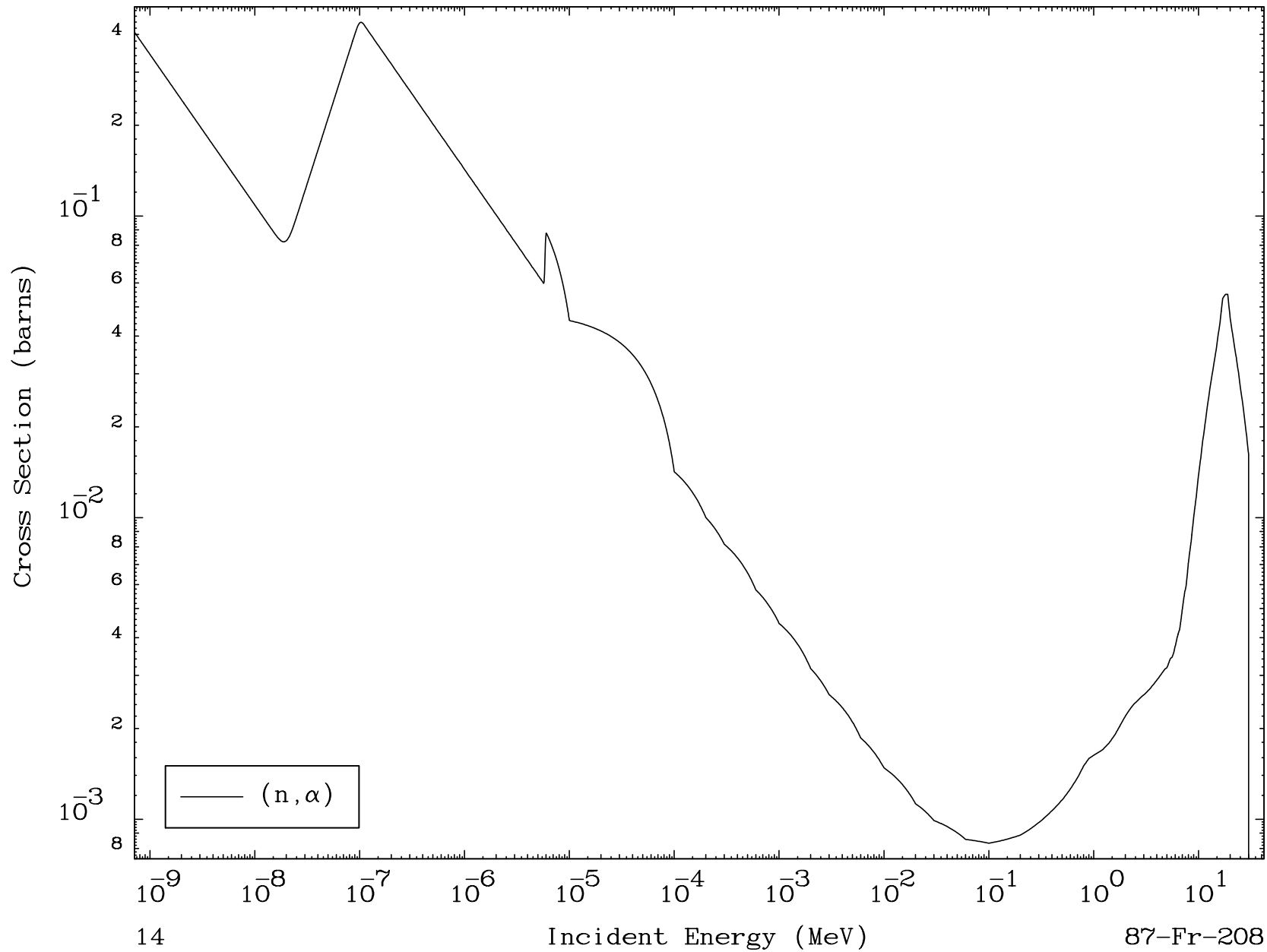
87-Fr-208

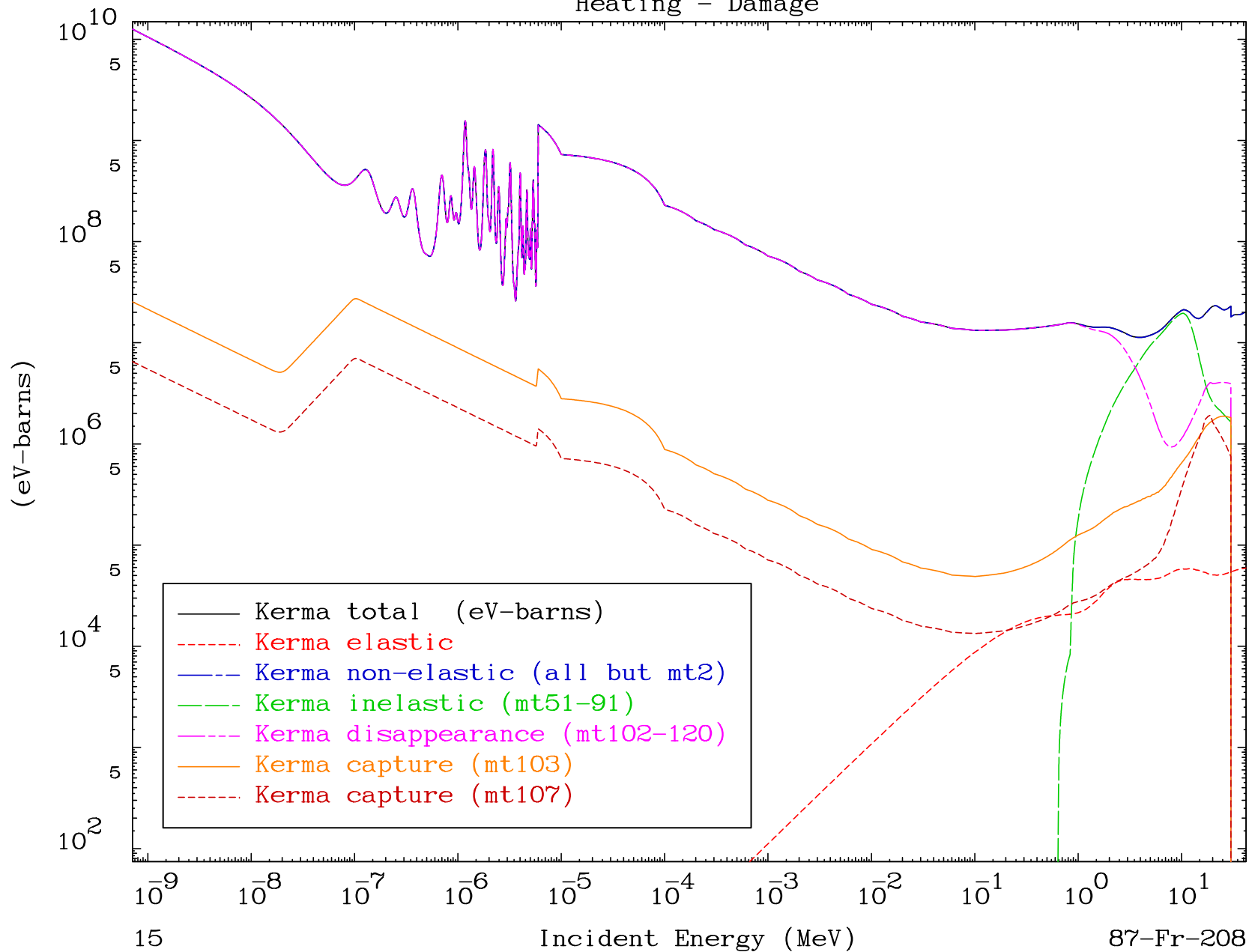
MAT 8713

(n,He3) Levels  
294 Kelvin Cross Sections

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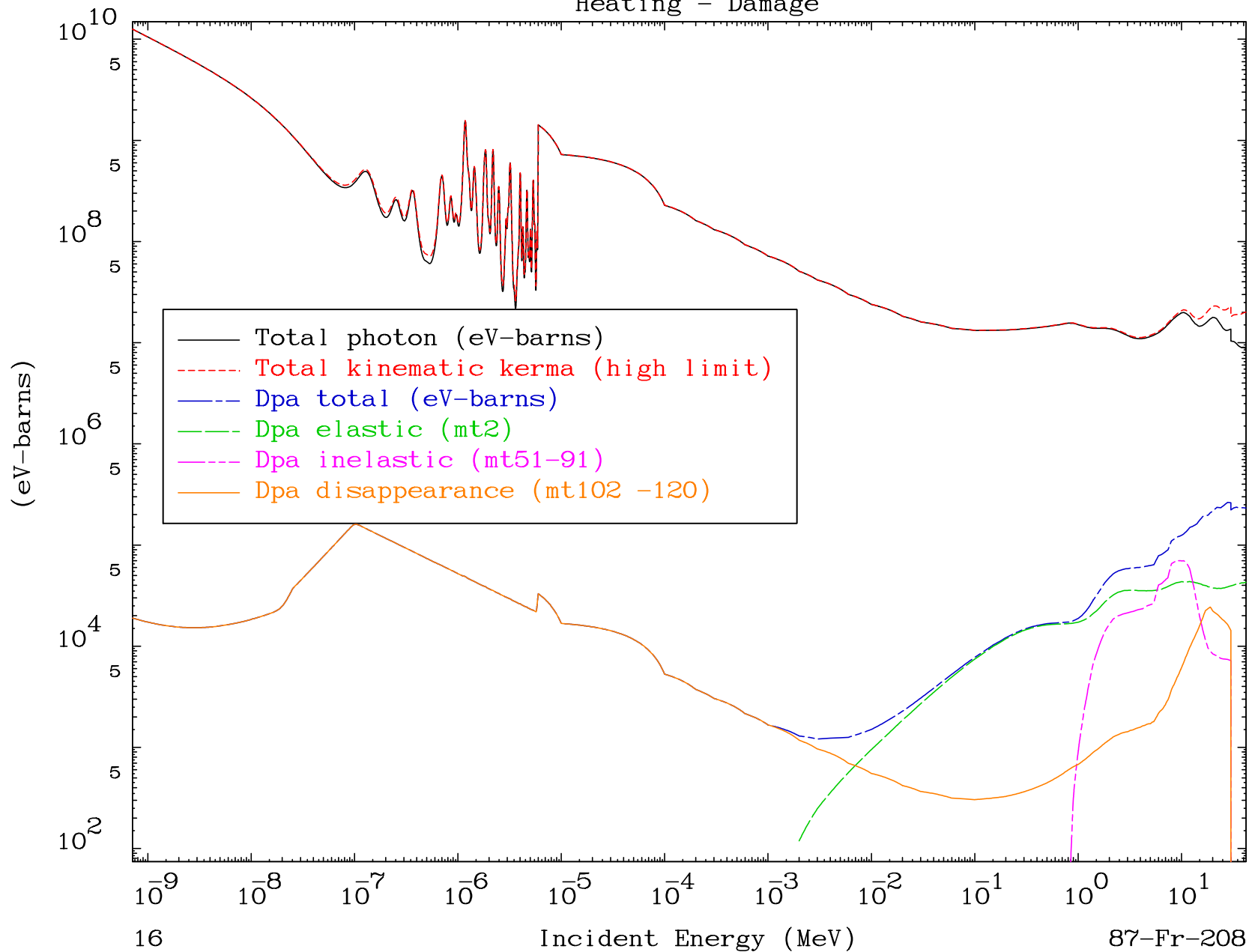




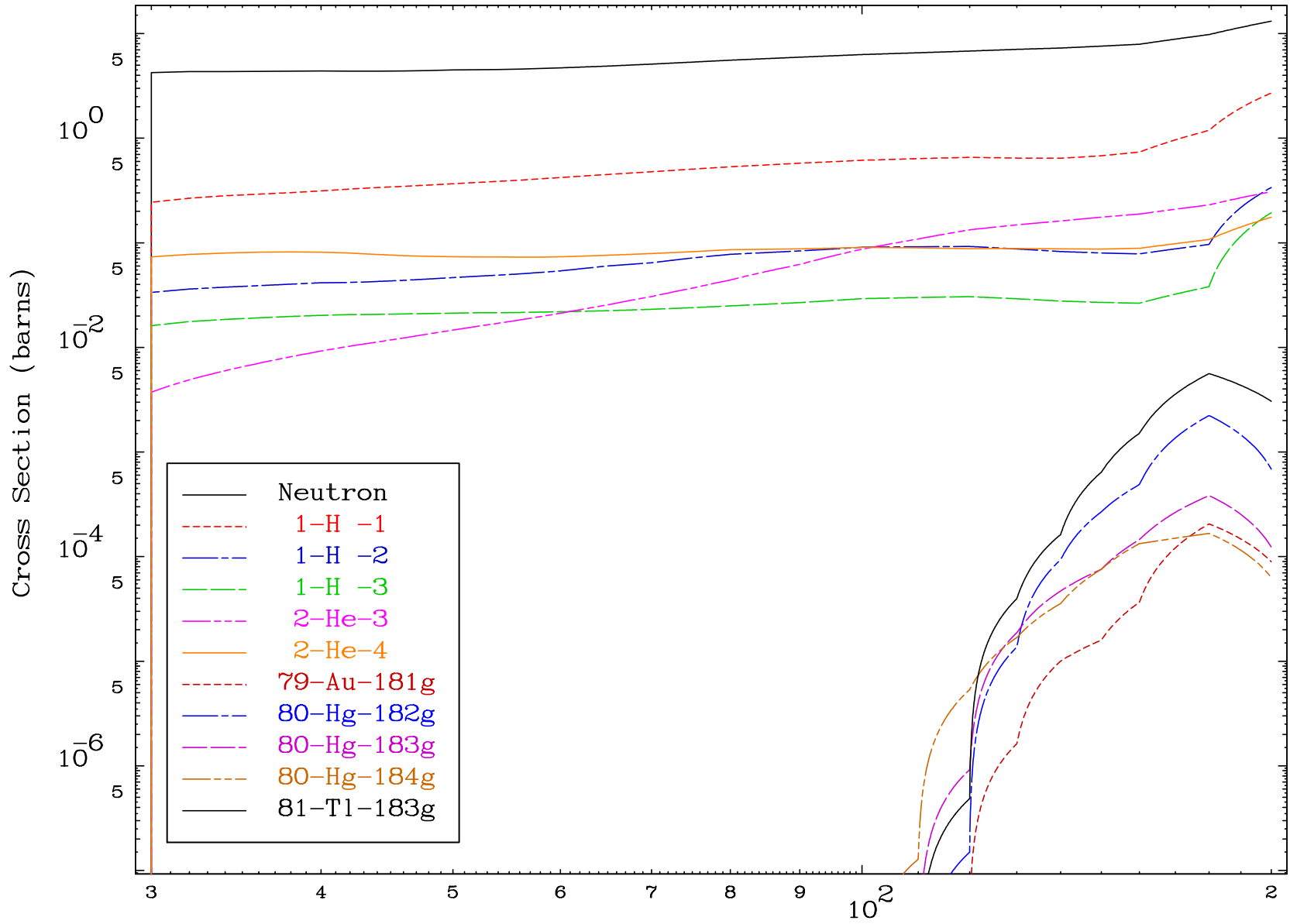
MAT 8713

Energy Release  
Heating - Damage

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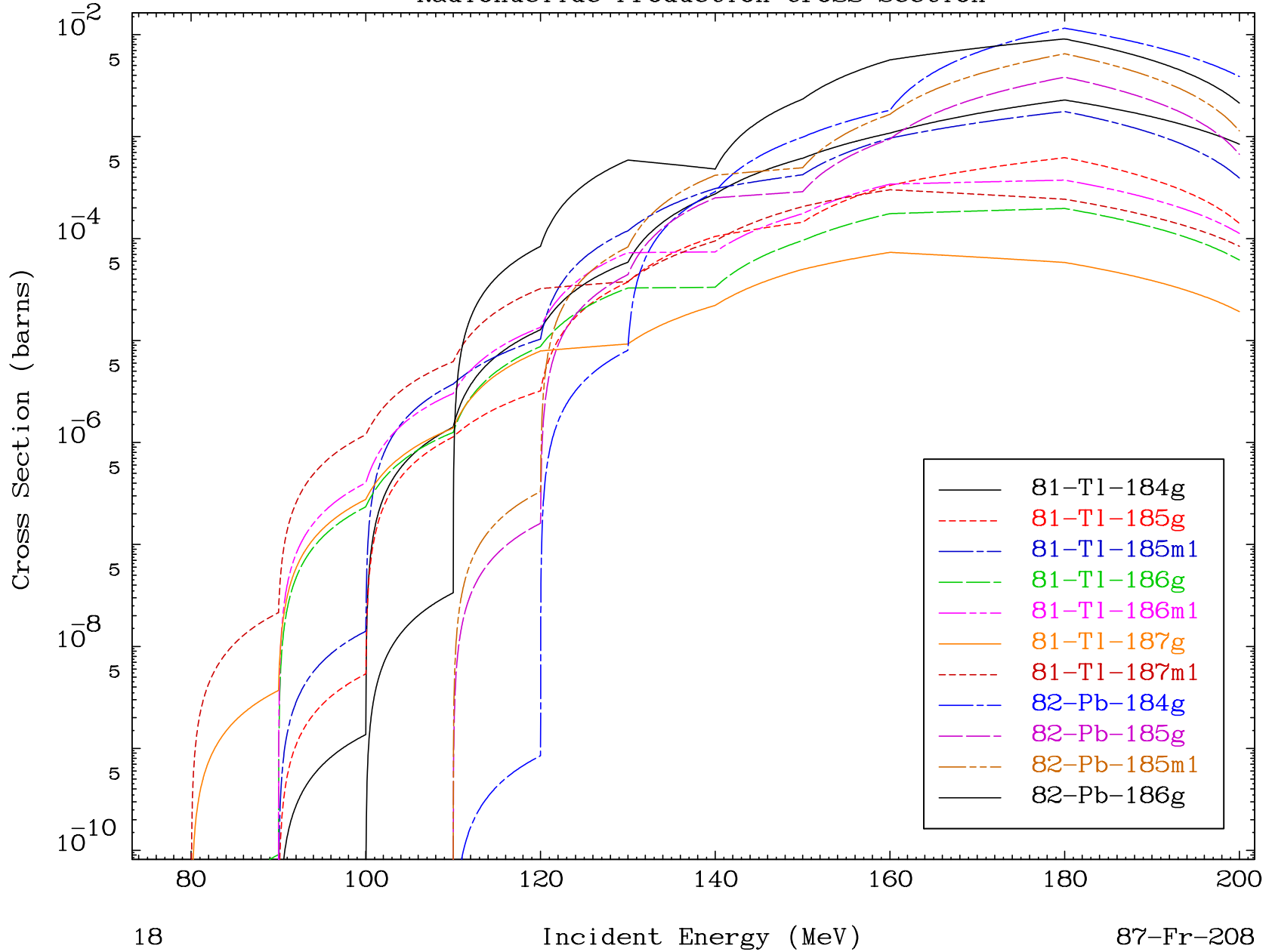


MAT 8713

(n,remainder)

87-Fr-208

### Radionuclide Production Cross Section

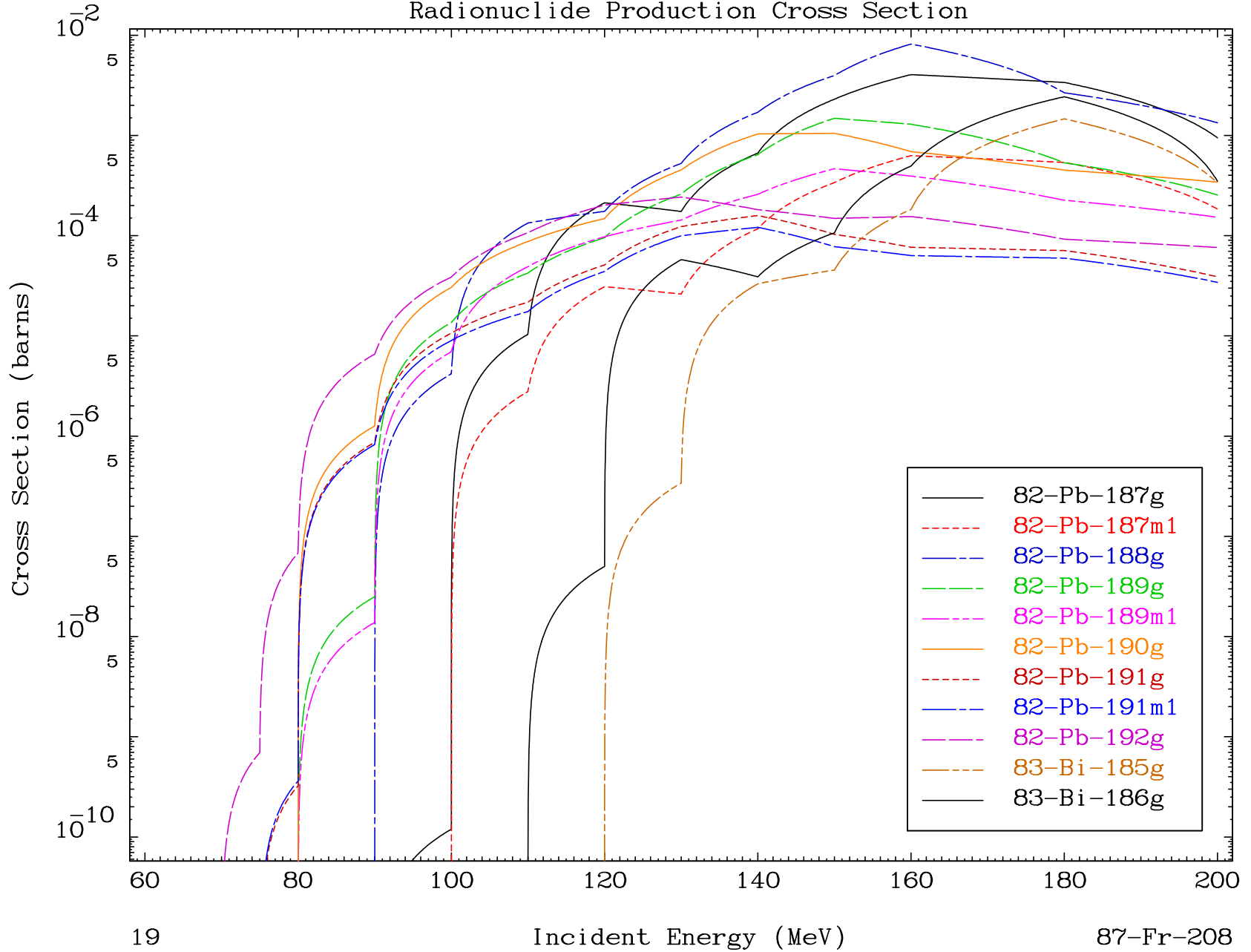


MAT 8713

(n,remainder)

87-Fr-208

### Radionuclide Production Cross Section

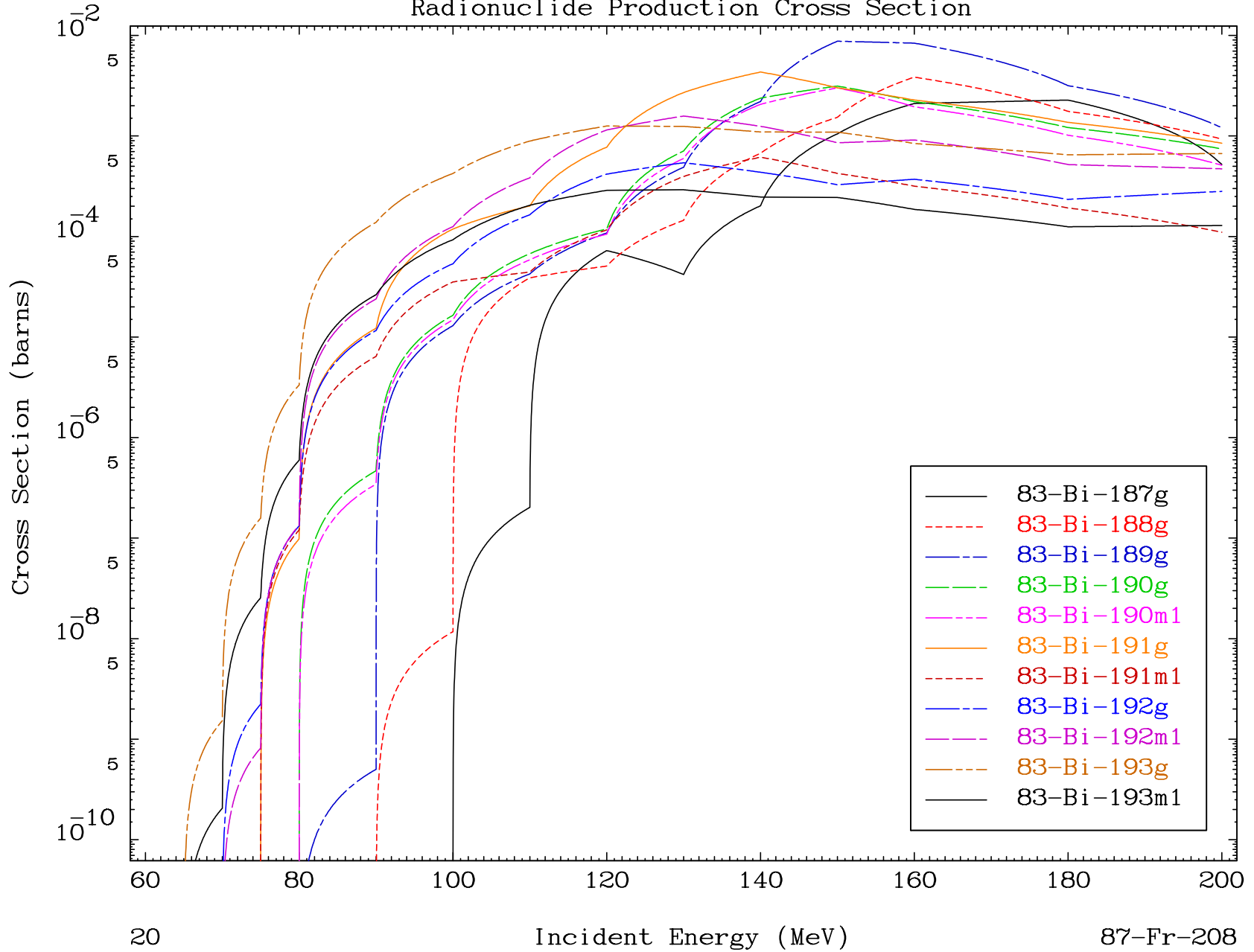


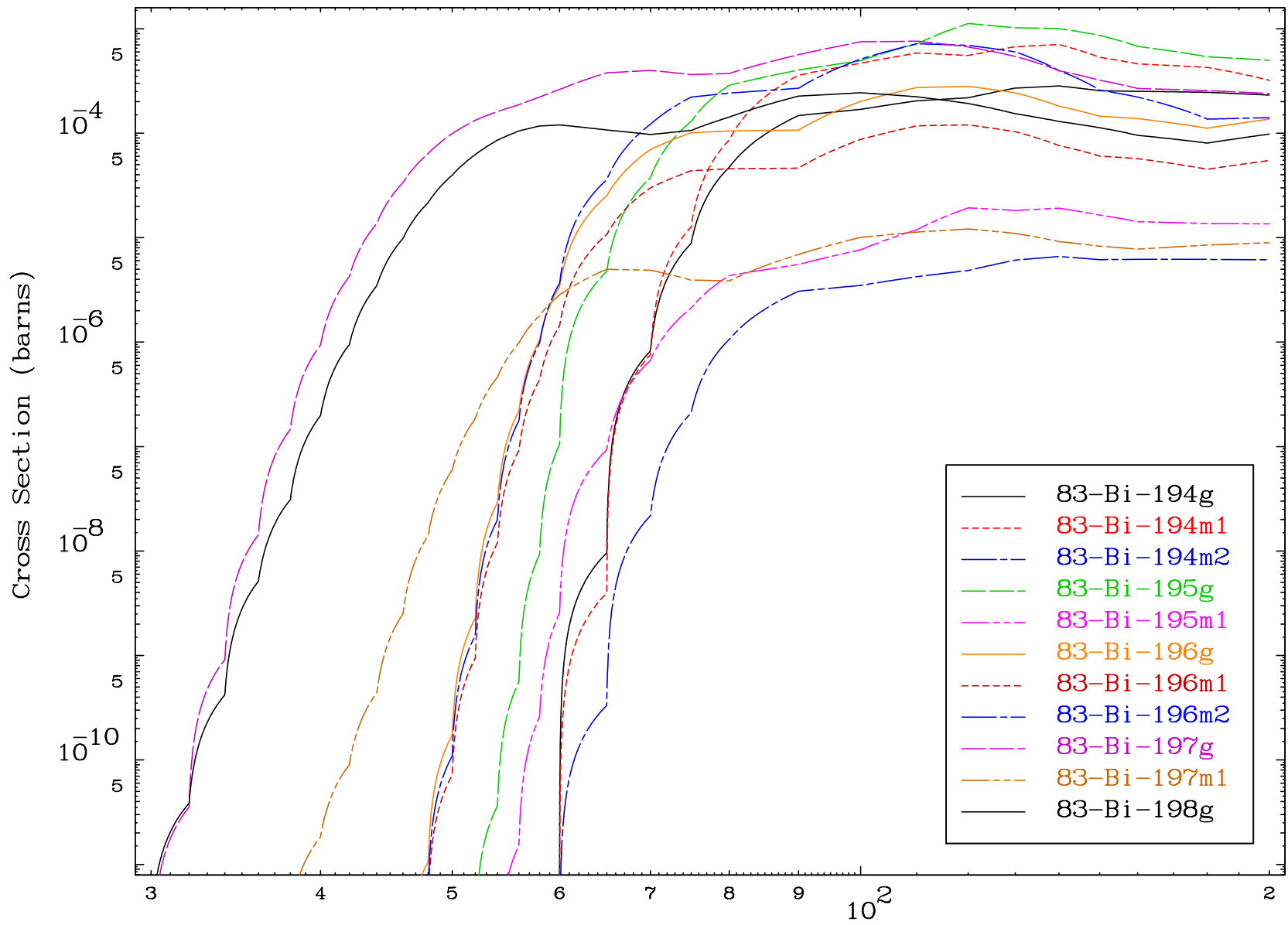
MAT 8713

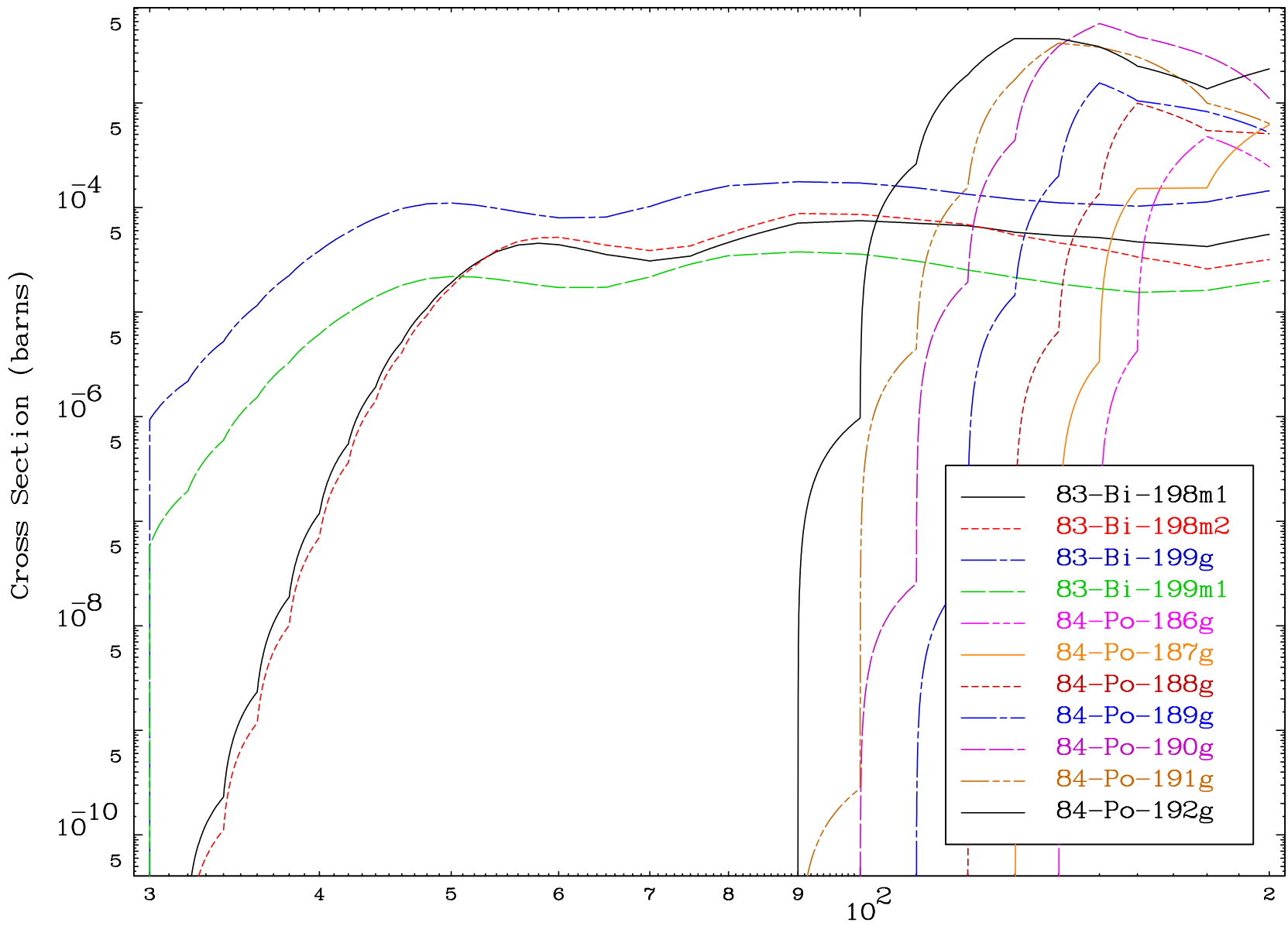
(n,remainder)

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### Radionuclide Production Cross Section





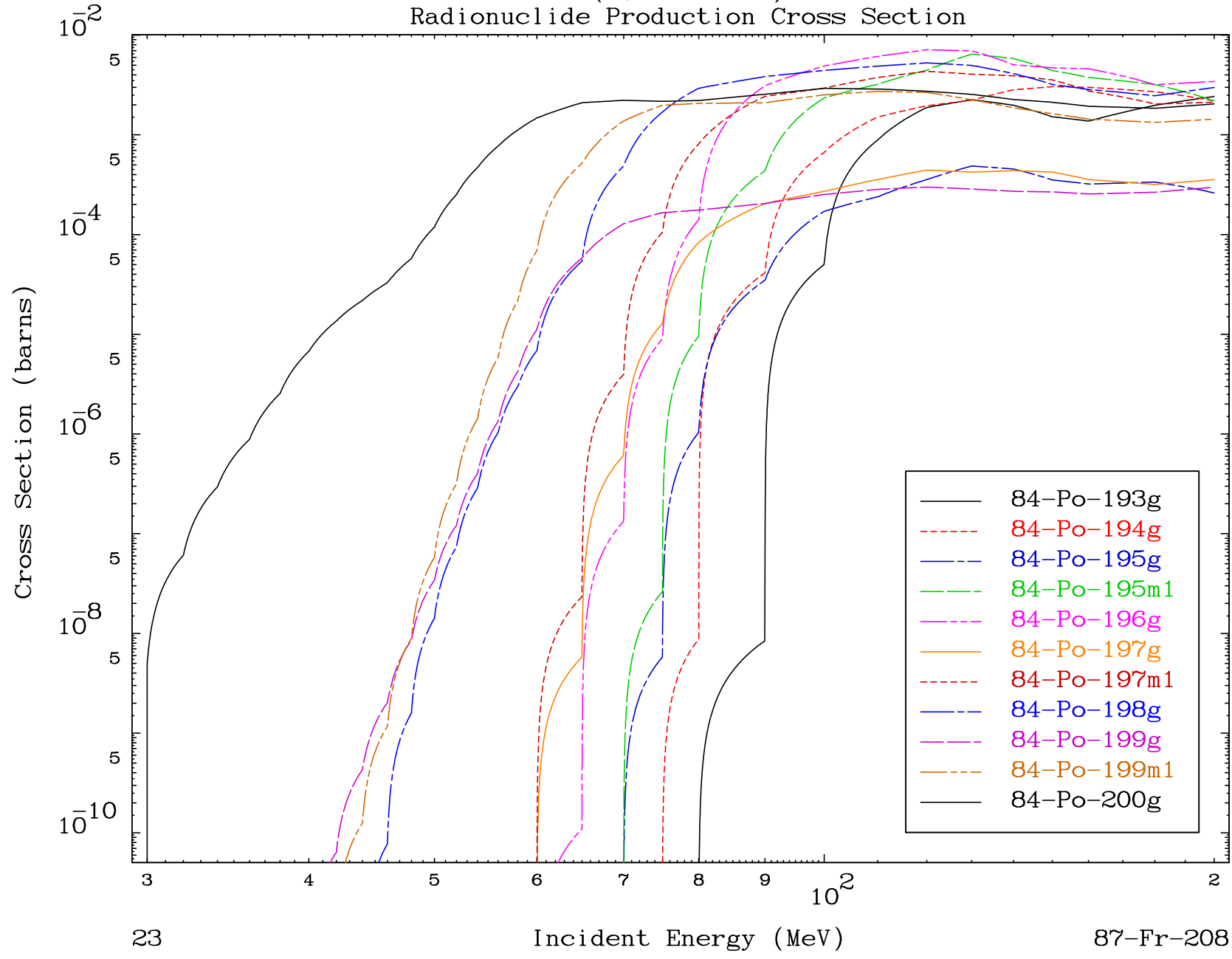


MAT 8713

(n,remainder)

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### Radionuclide Production Cross Section



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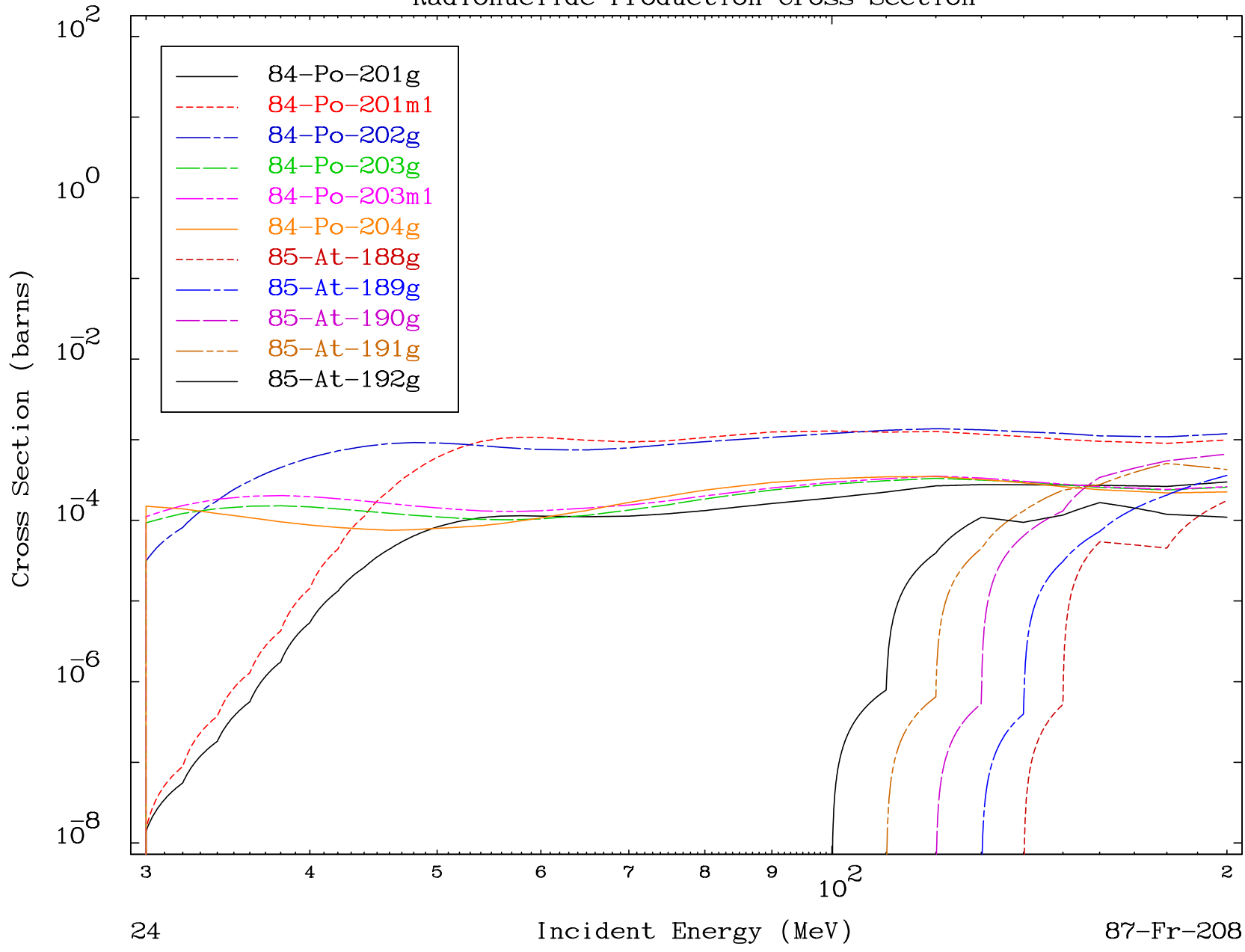
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MAT 8713

(n,remainder)

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### Radionuclide Production Cross Section



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Incident Energy (MeV)

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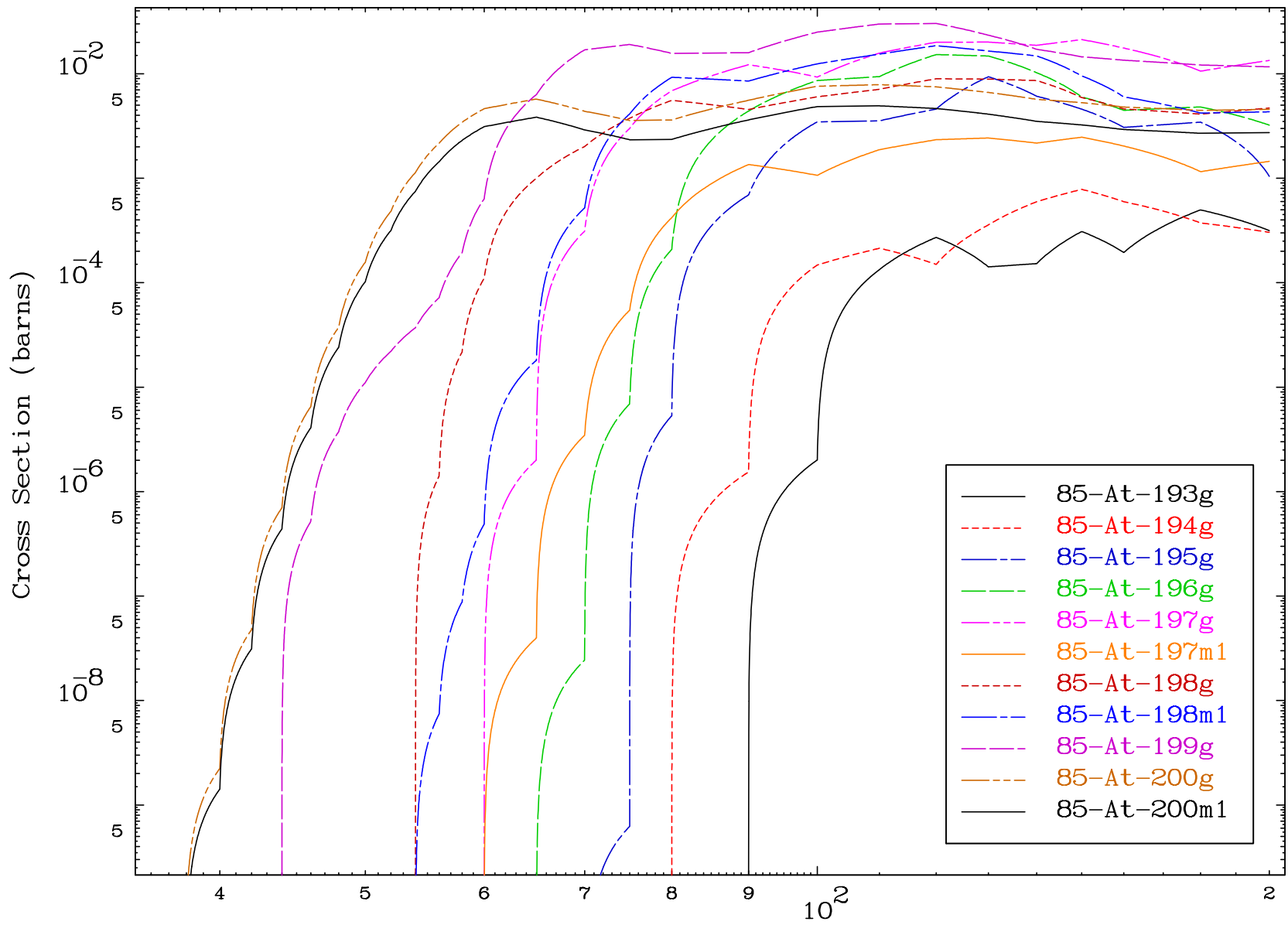


MAT 8713

(n,remainder)

87-Fr-208

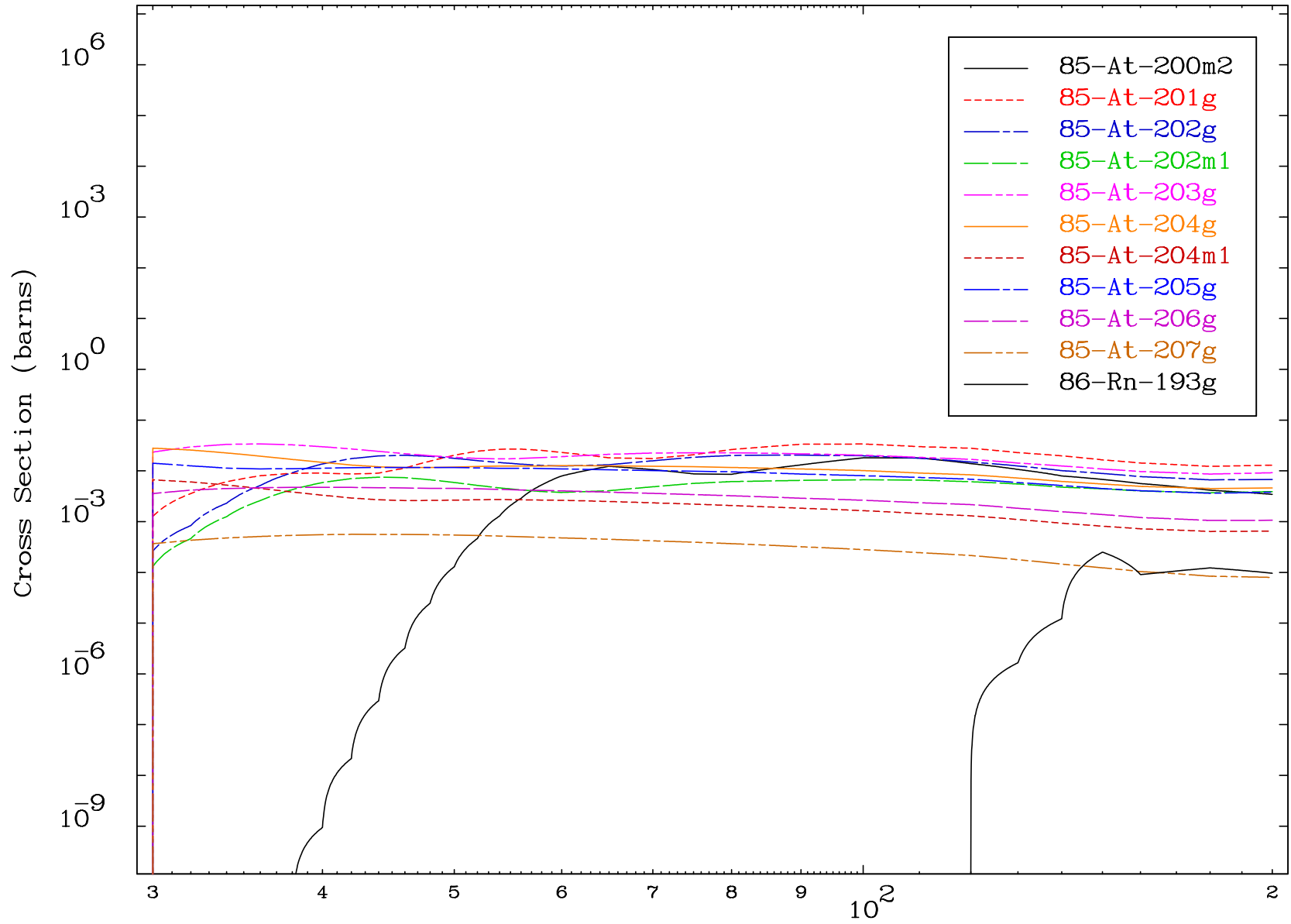
### Radionuclide Production Cross Section



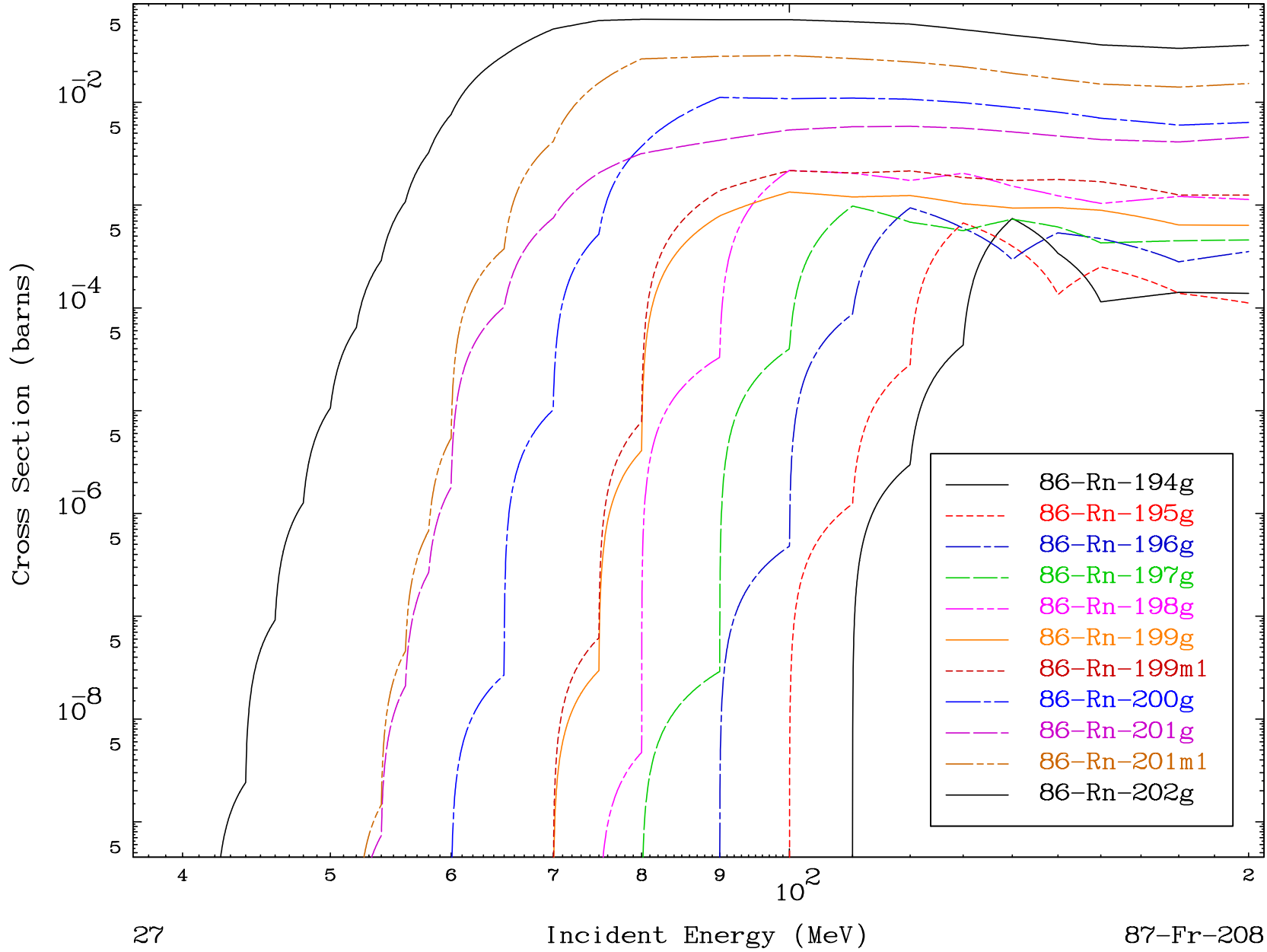
25

Incident Energy (MeV)

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Radionuclide Production Cross Section

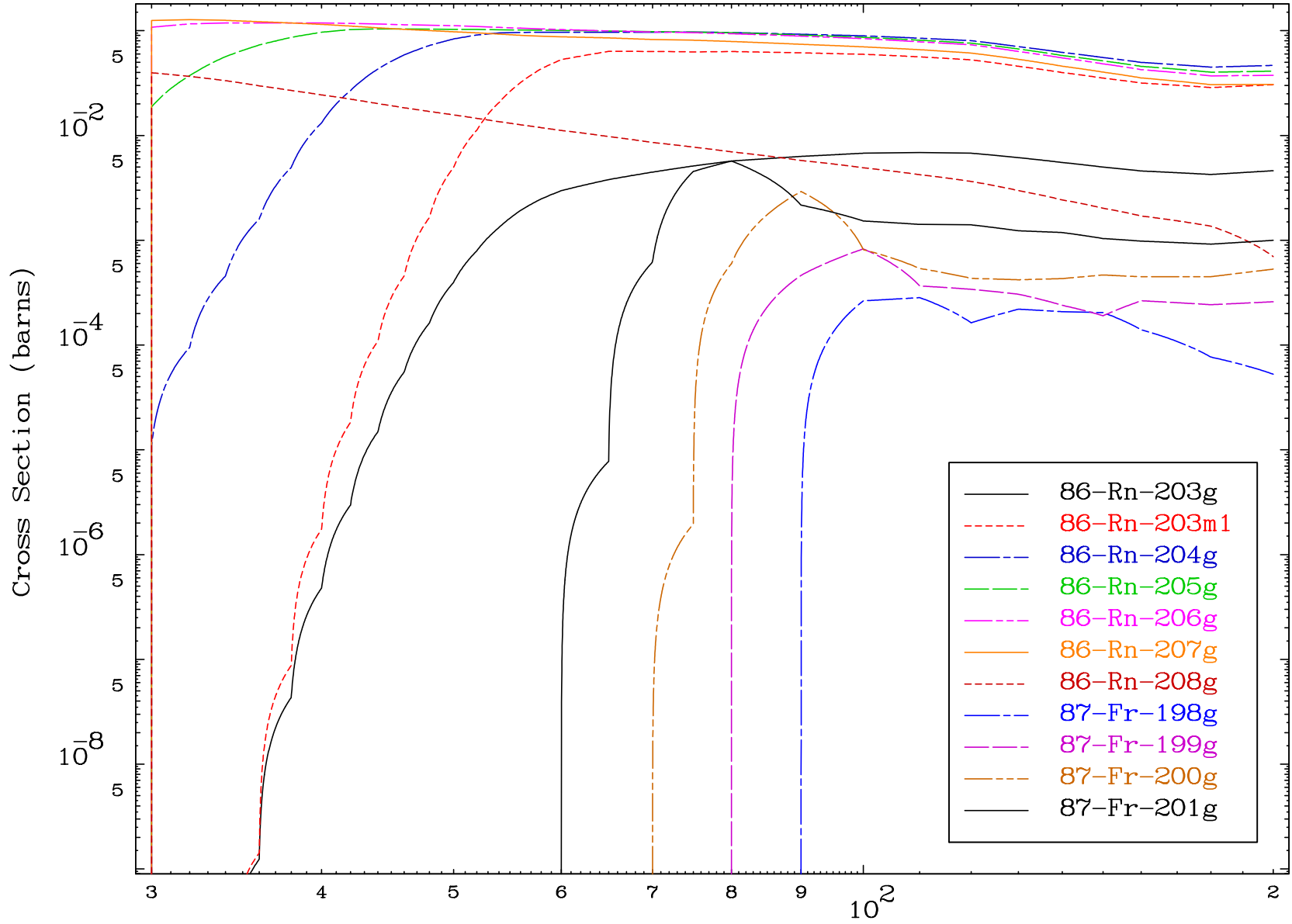


MAT 8713

(n,remainder)

87-Fr-208

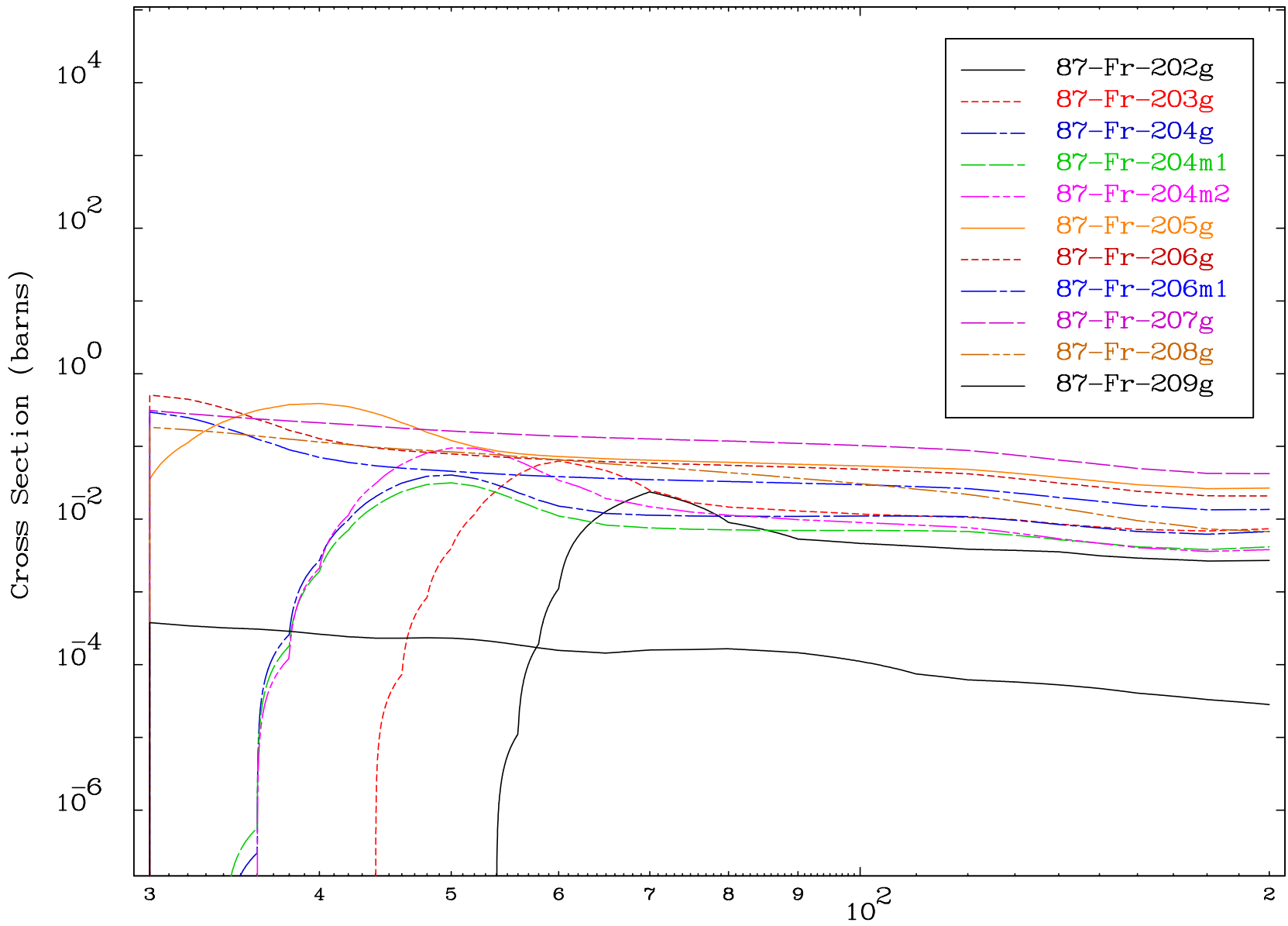
### Radionuclide Production Cross Section



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Incident Energy (MeV)

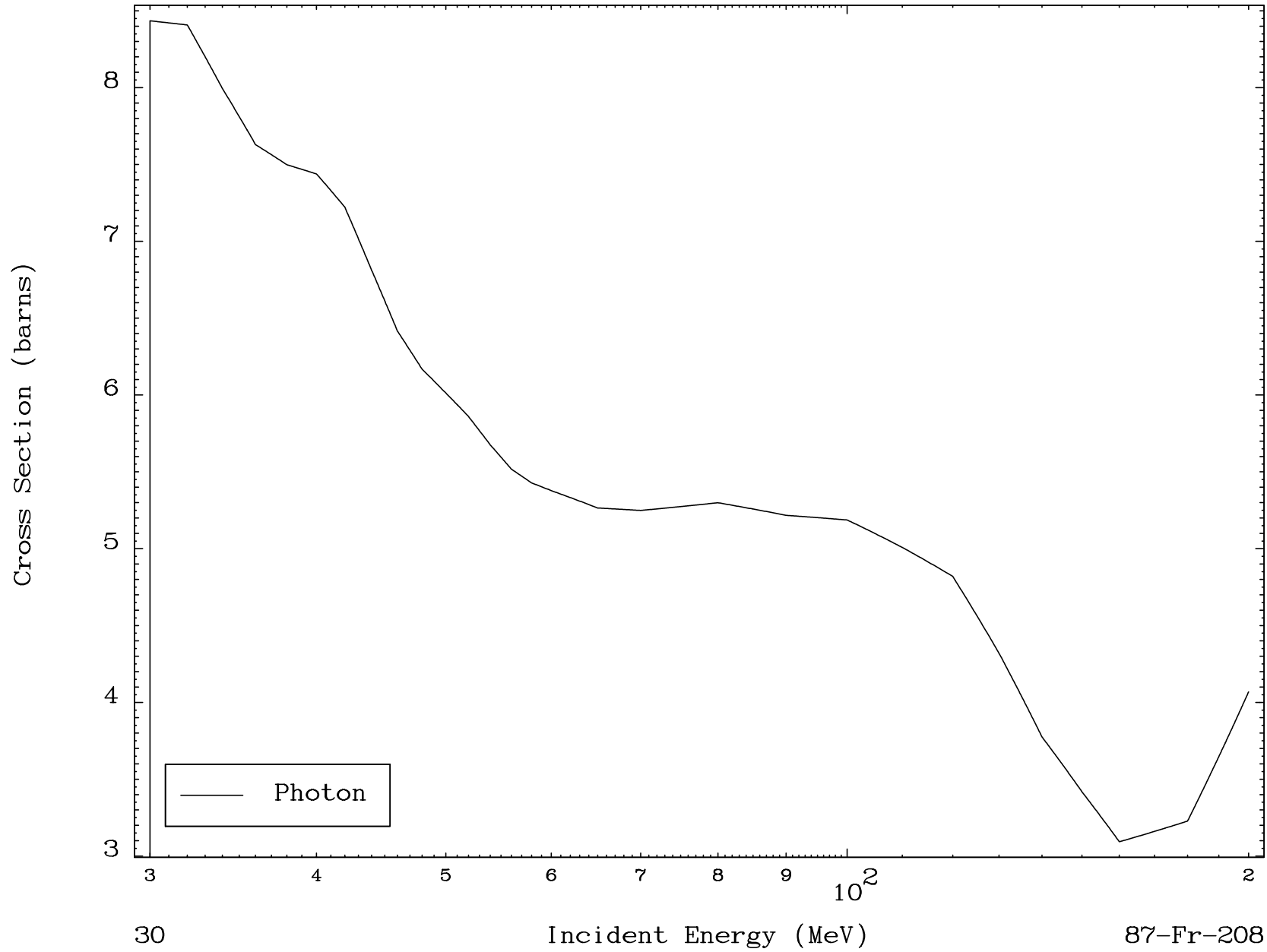
87-Fr-208



MAT 8713

(n,remainder)  
Radionuclide Production Cross Section

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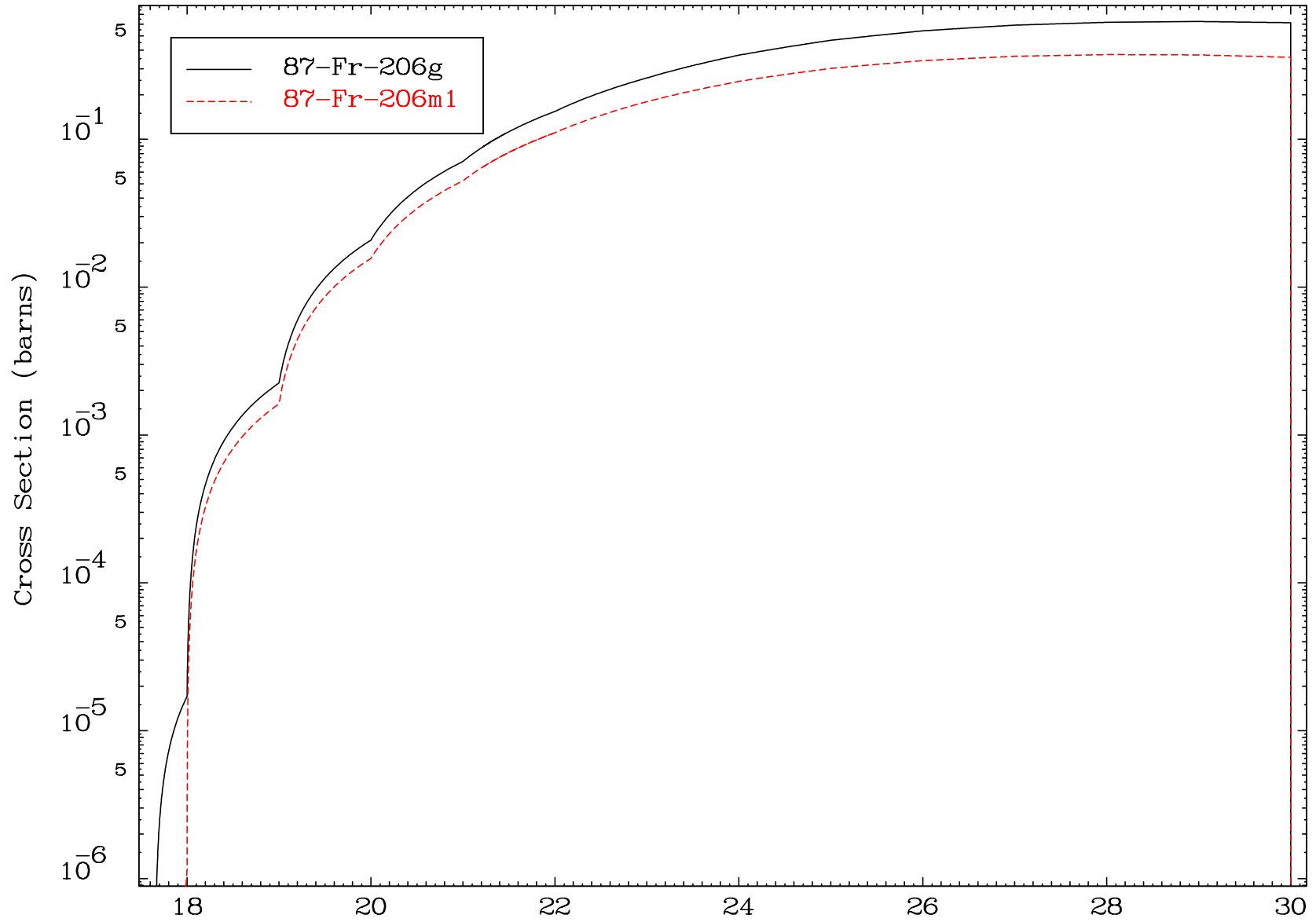


MAT 8713

(n,3n)

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Radionuclide Production Cross Section



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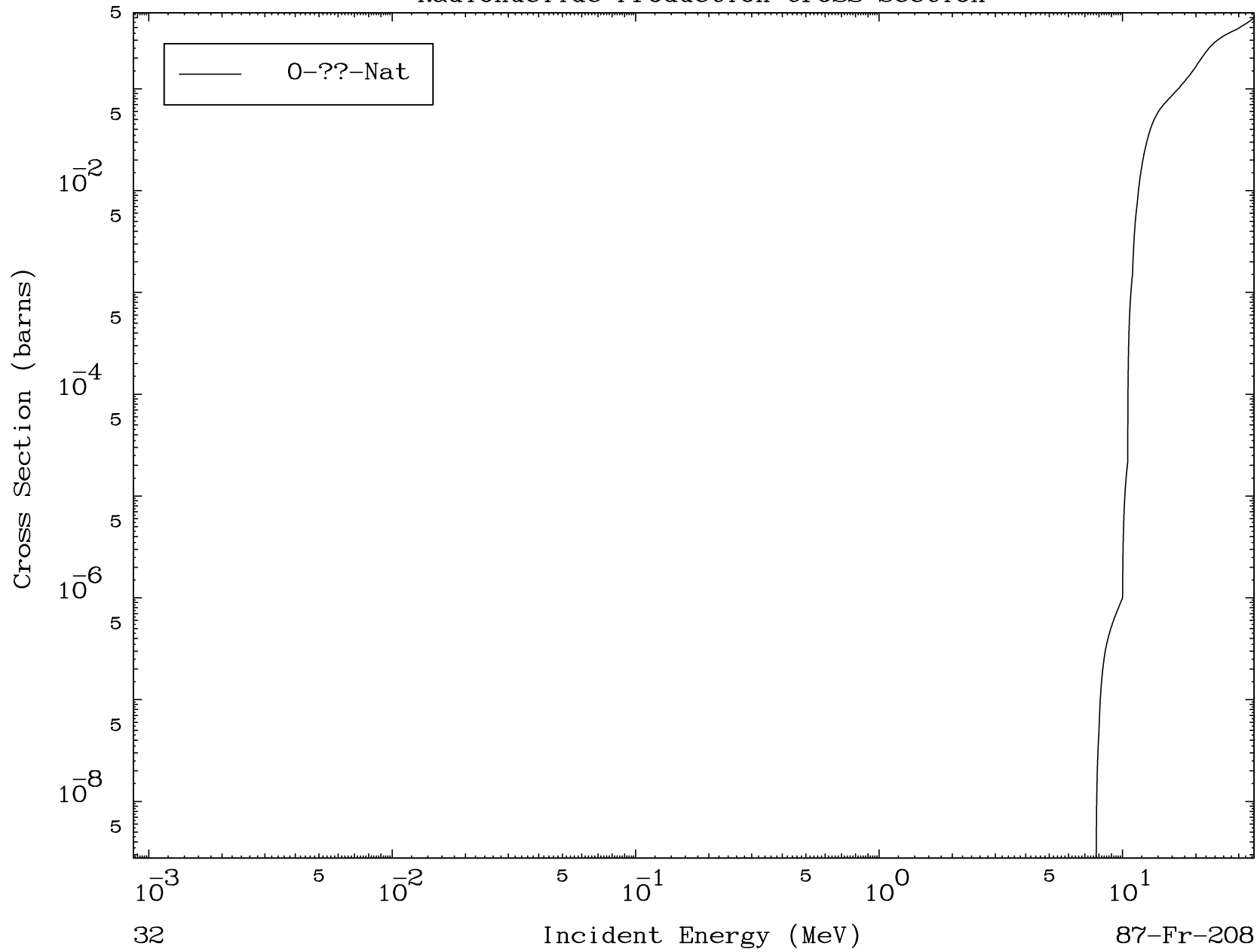
Incident Energy (MeV)

87-Fr-208

MAT 8713

Fission  
Radionuclide Production Cross Section

87-Fr-208



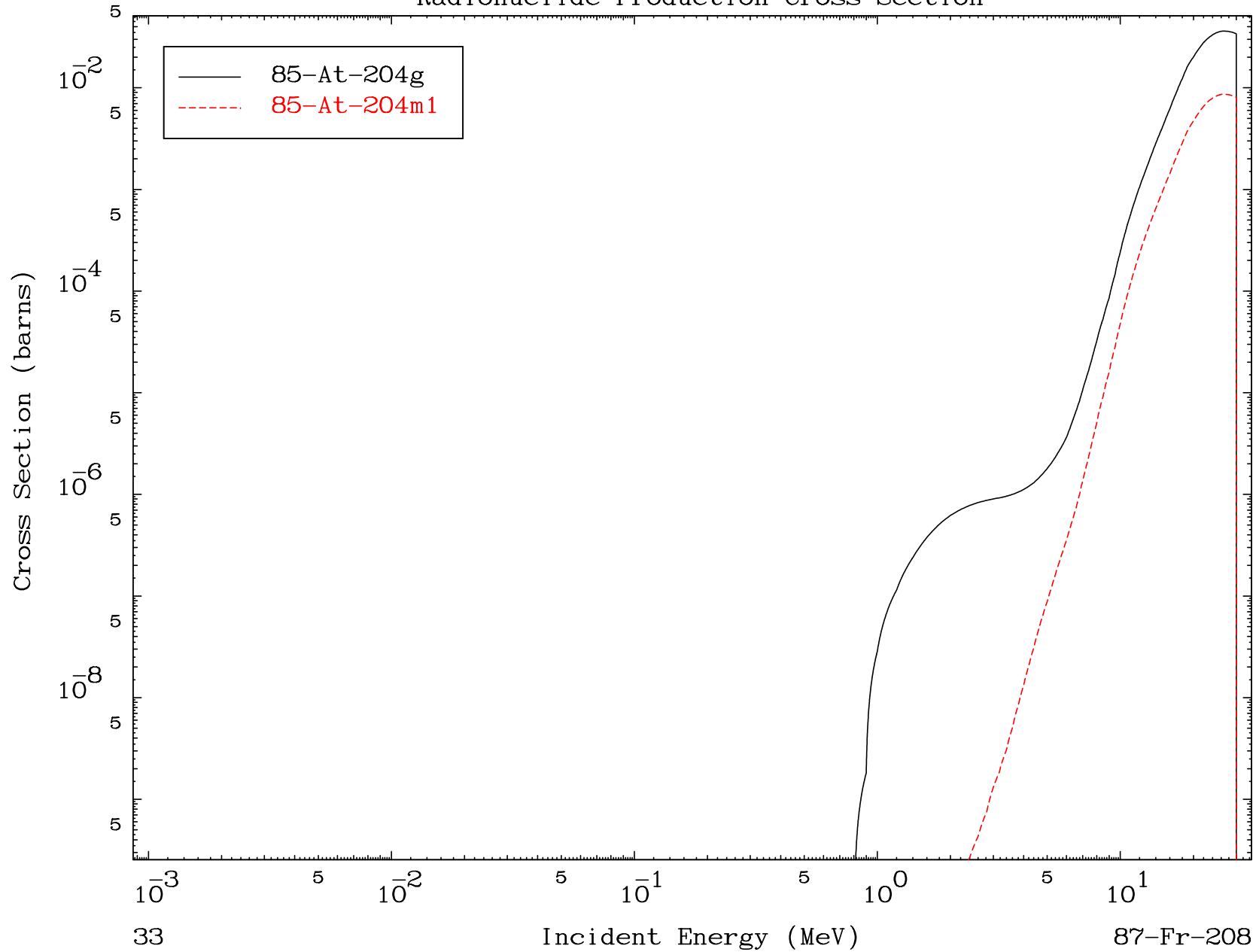


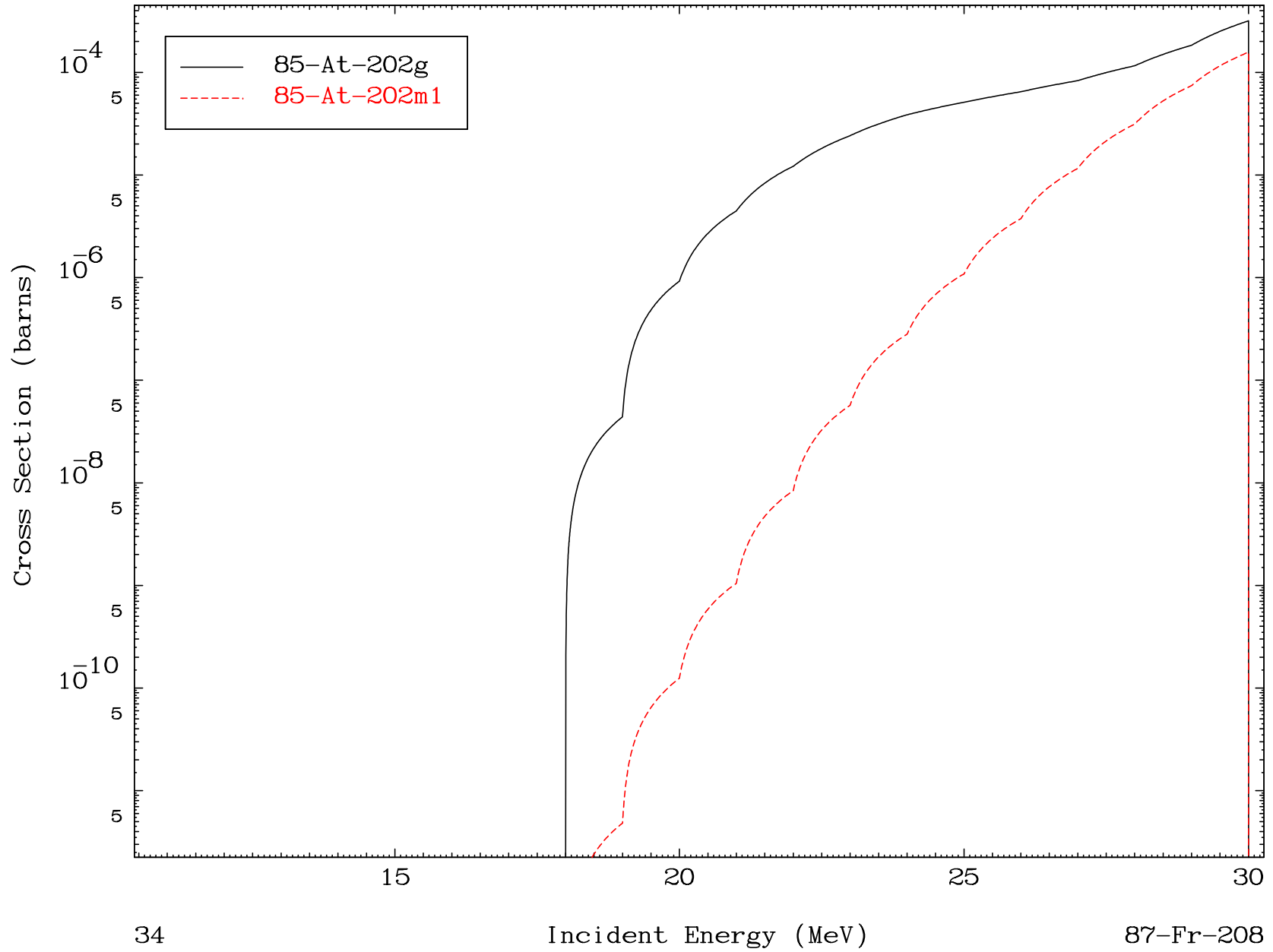
MAT 8713

(n,n')  $\alpha$

87-Fr-208

Radionuclide Production Cross Section



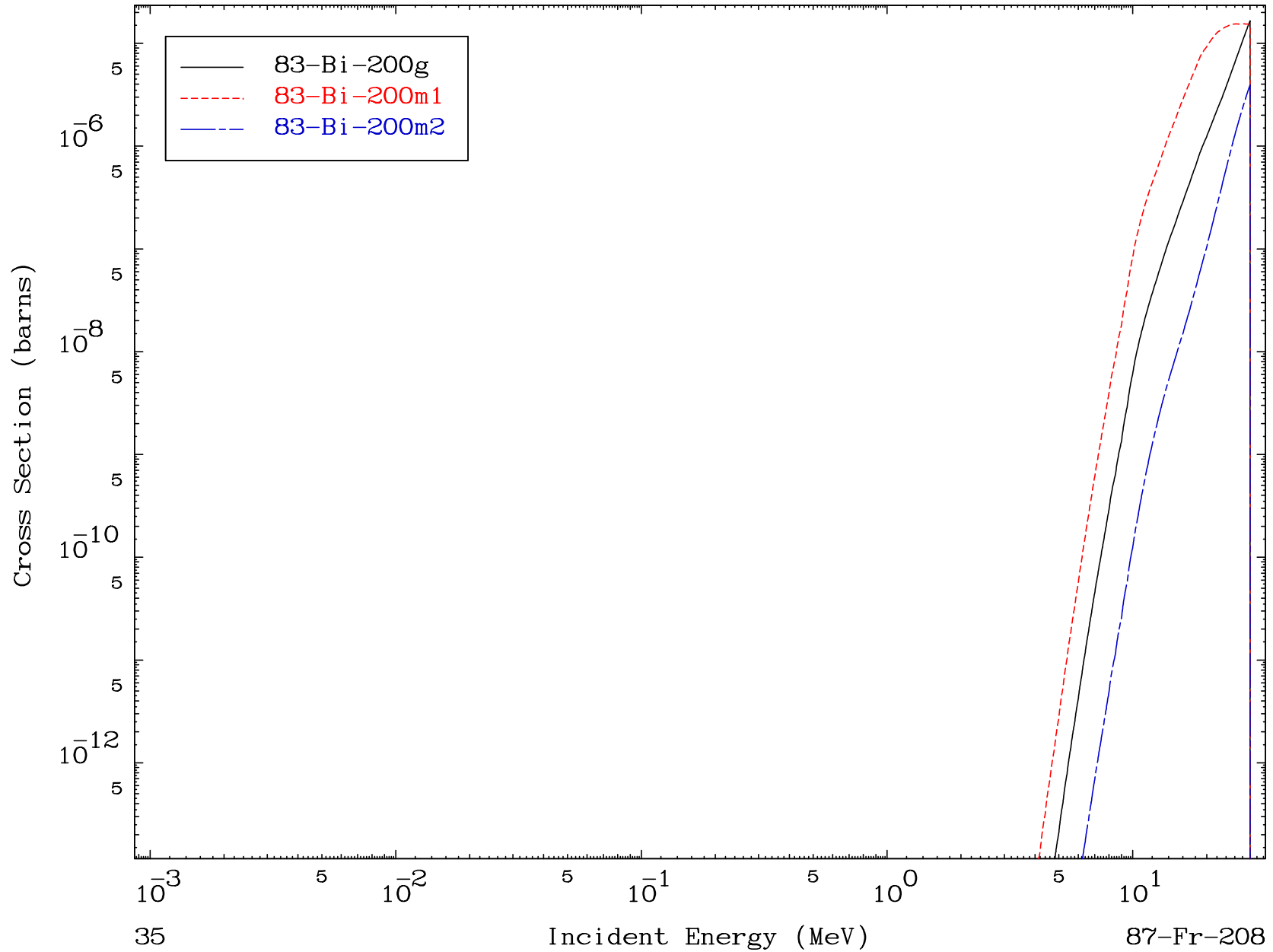


MAT 8713

(n,n') 2α

87-Fr-208

Radionuclide Production Cross Section

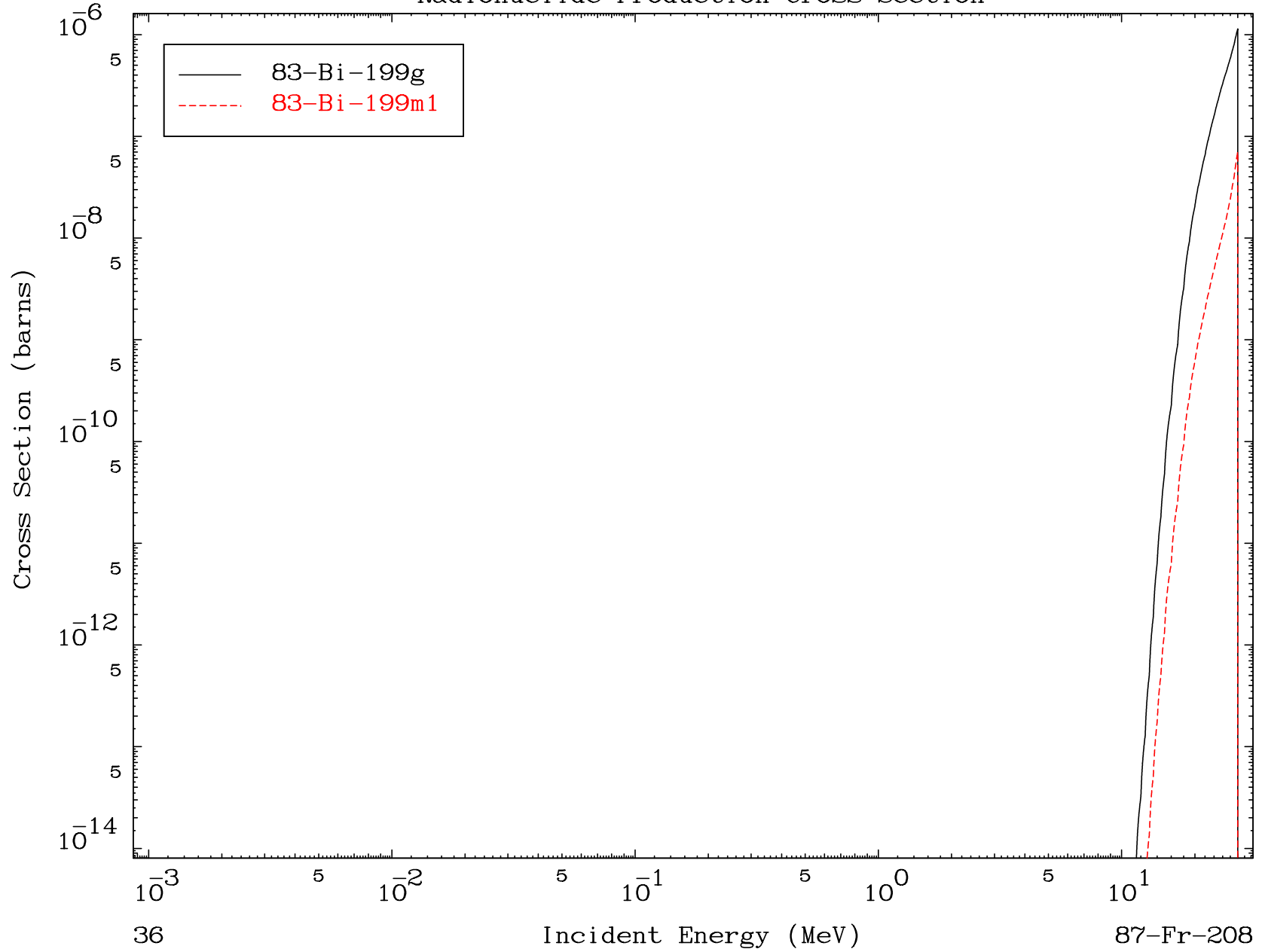


MAT 8713

(n,2n) 2 $\alpha$

87-Fr-208

Radionuclide Production Cross Section

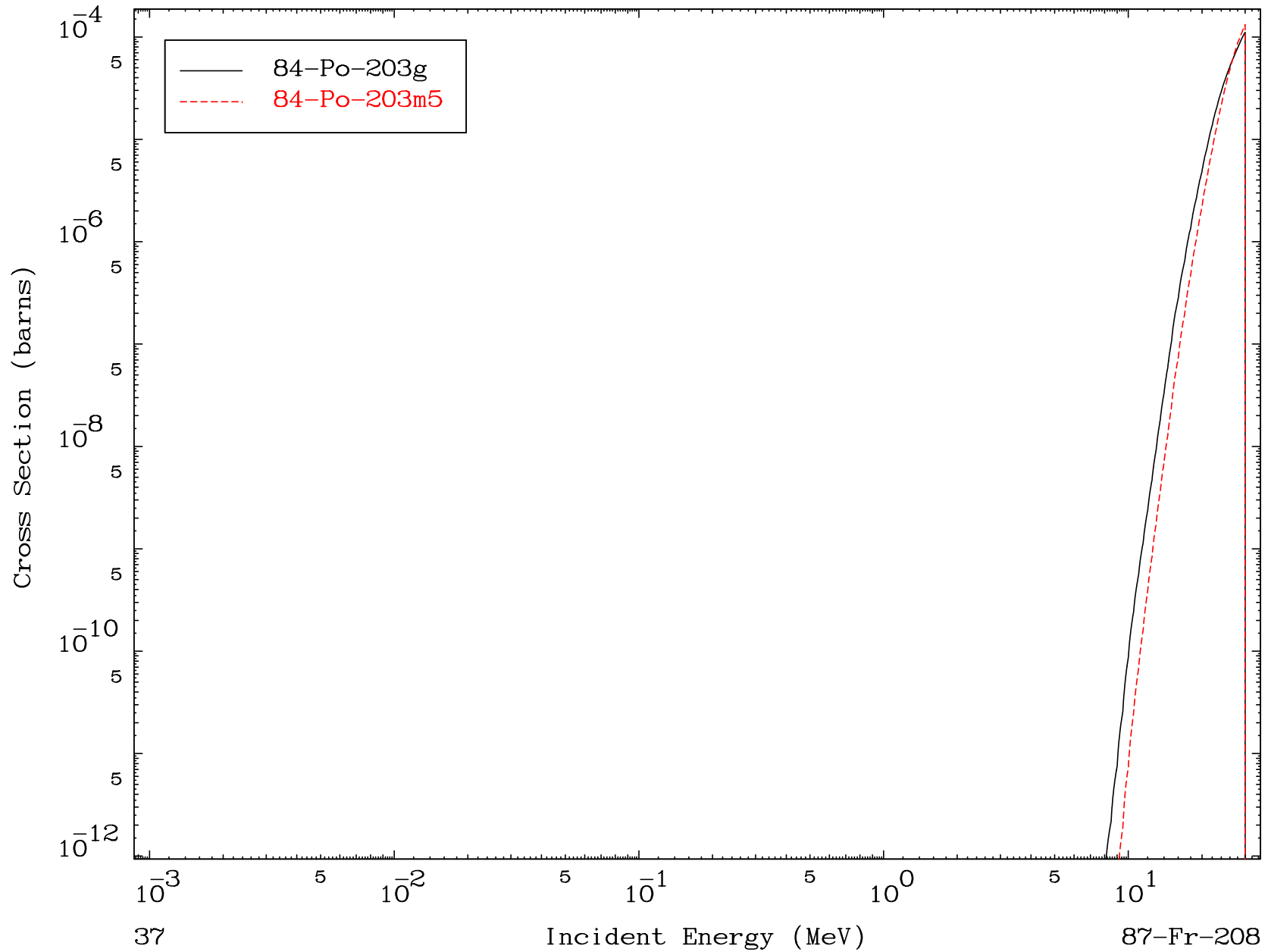


MAT 8713

(n,n') p  $\alpha$

87-Fr-208

Radionuclide Production Cross Section

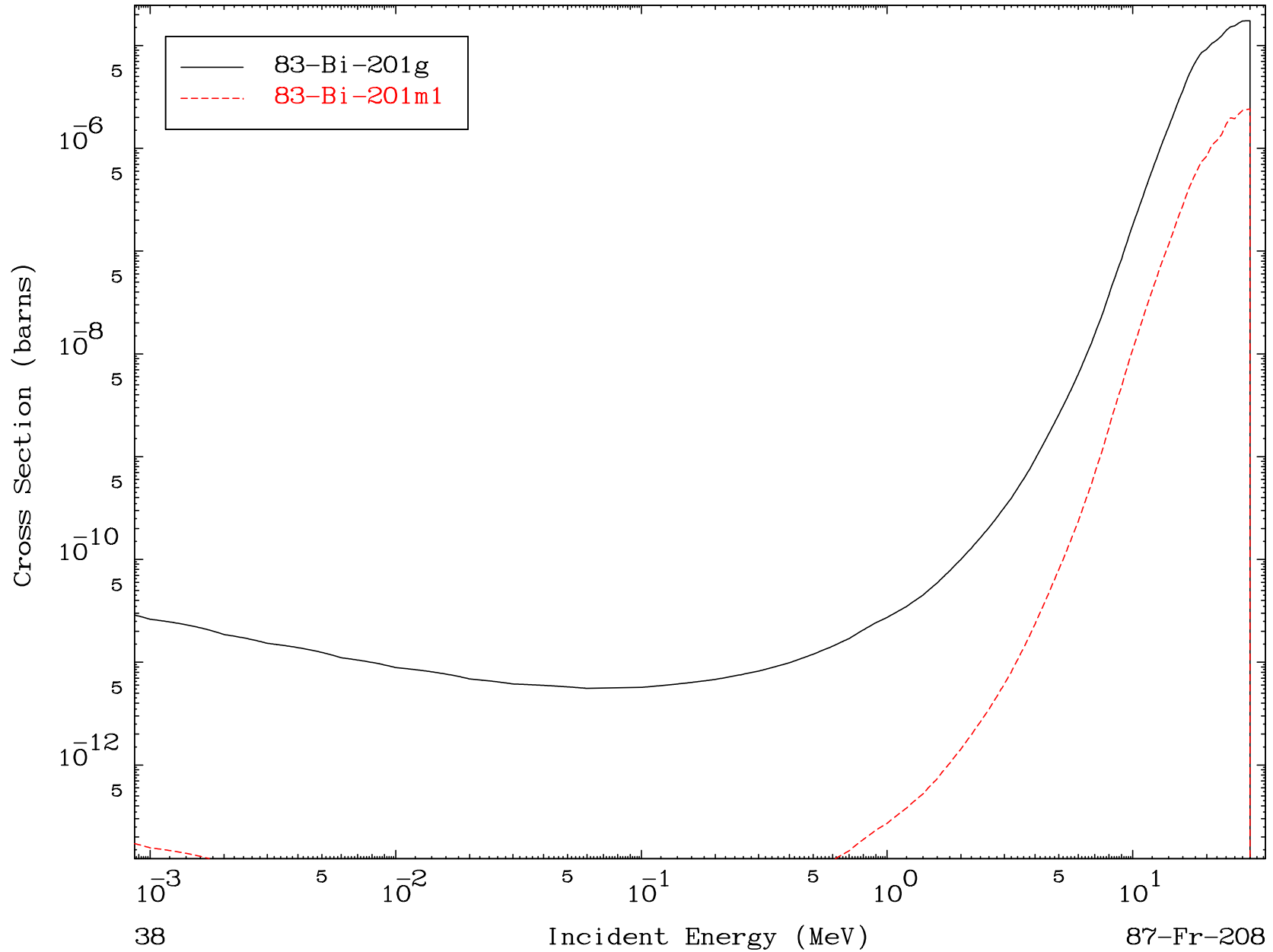


MAT 8713

(n,2 $\alpha$ )

87-Fr-208

Radionuclide Production Cross Section



MAT 8713

(n,d)  $\alpha$

87-Fr-208

Radionuclide Production Cross Section

