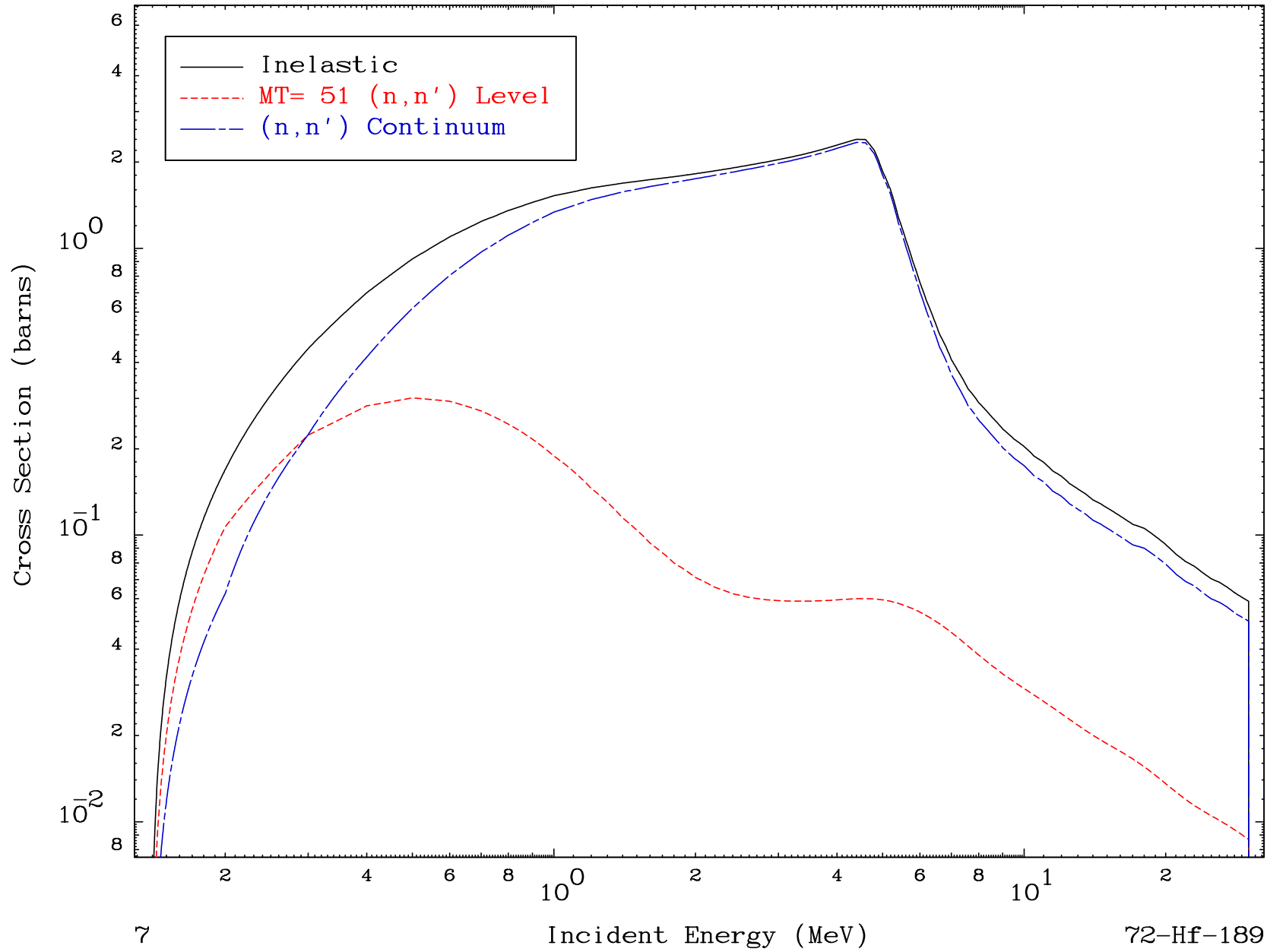


MAT 7270

(n,n') Level
294 Kelvin Cross Sections

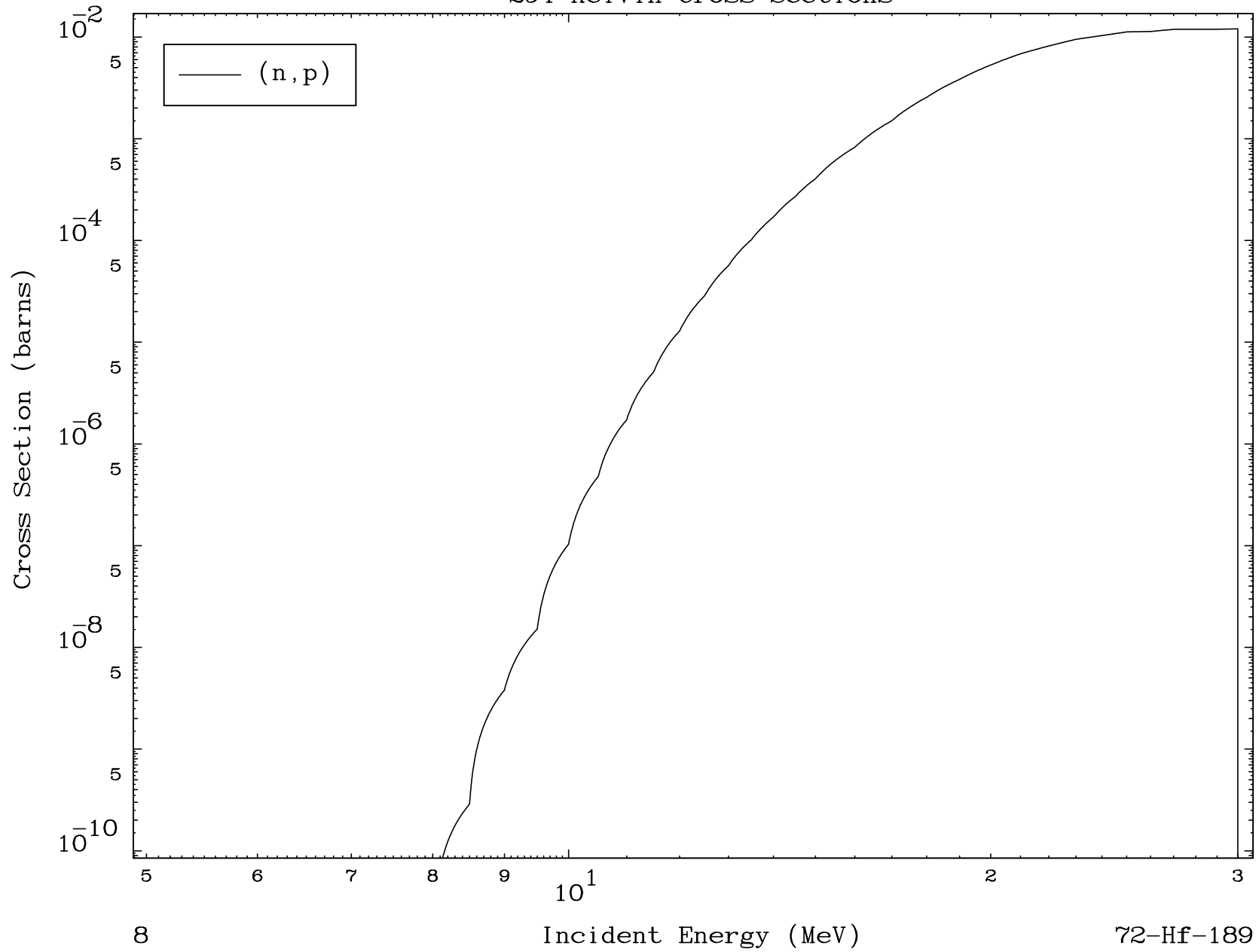
72-Hf-189



MAT 7270

(n,p) Levels
294 Kelvin Cross Sections

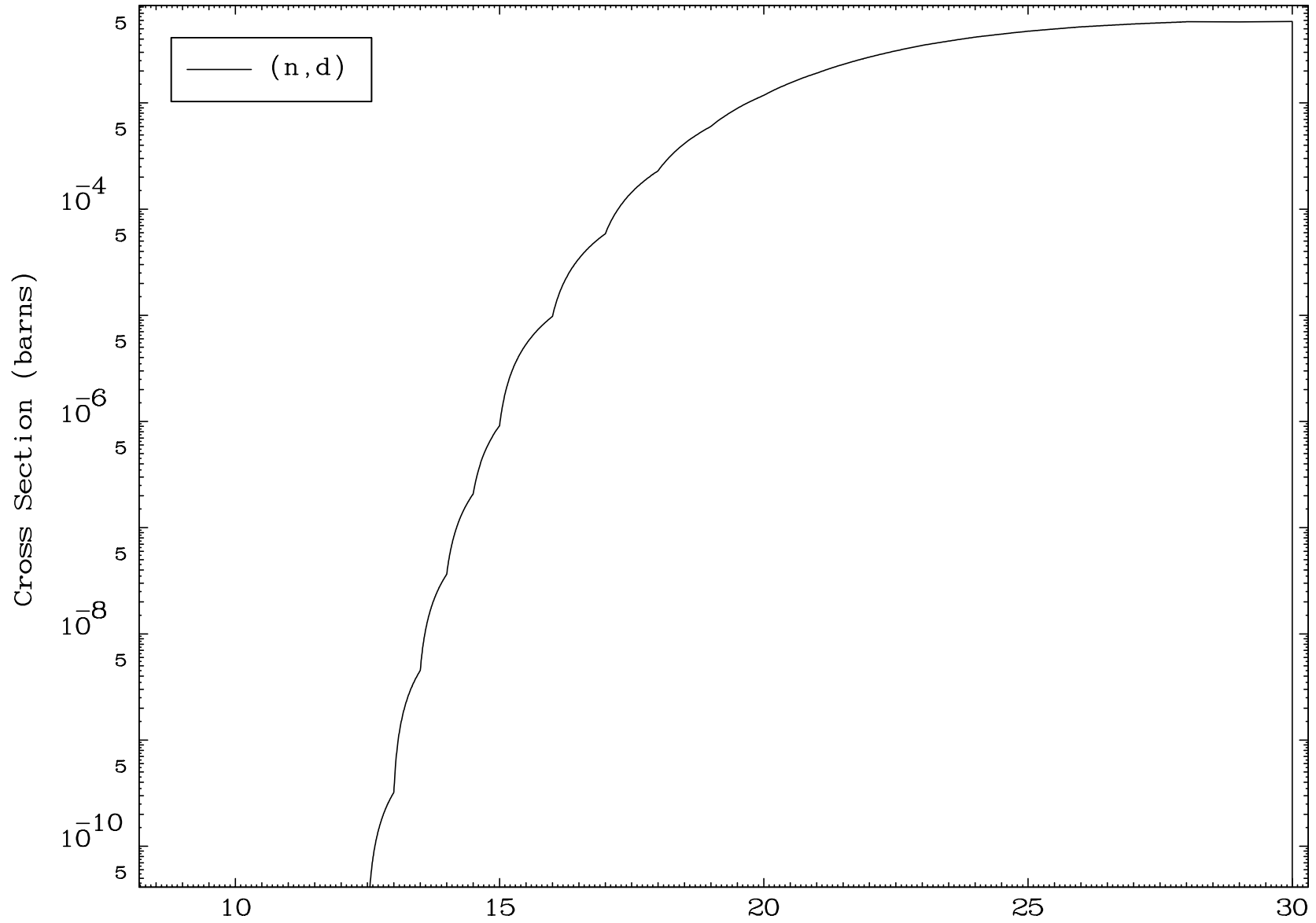
72-Hf-189

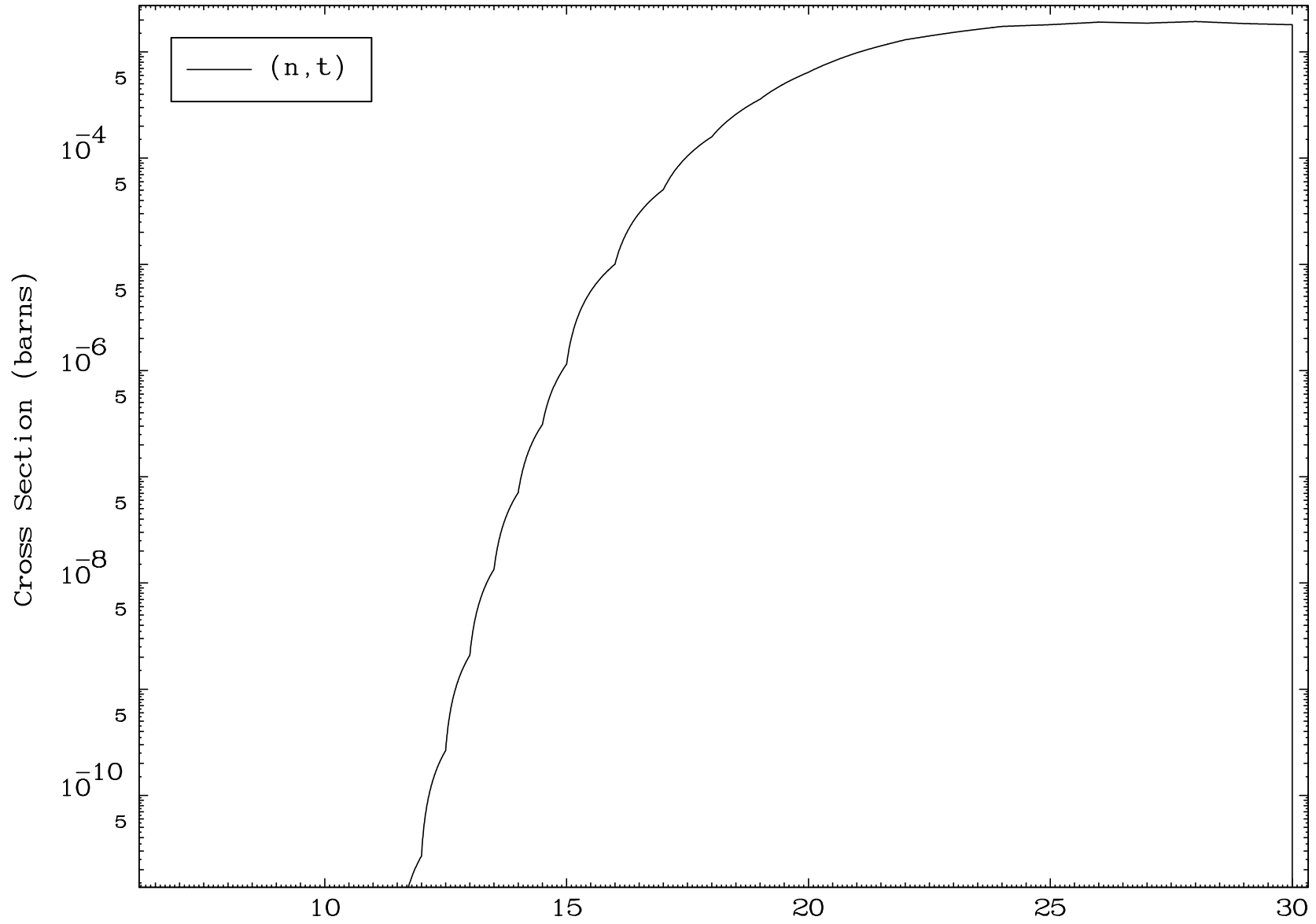


8

Incident Energy (MeV)

72-Hf-189

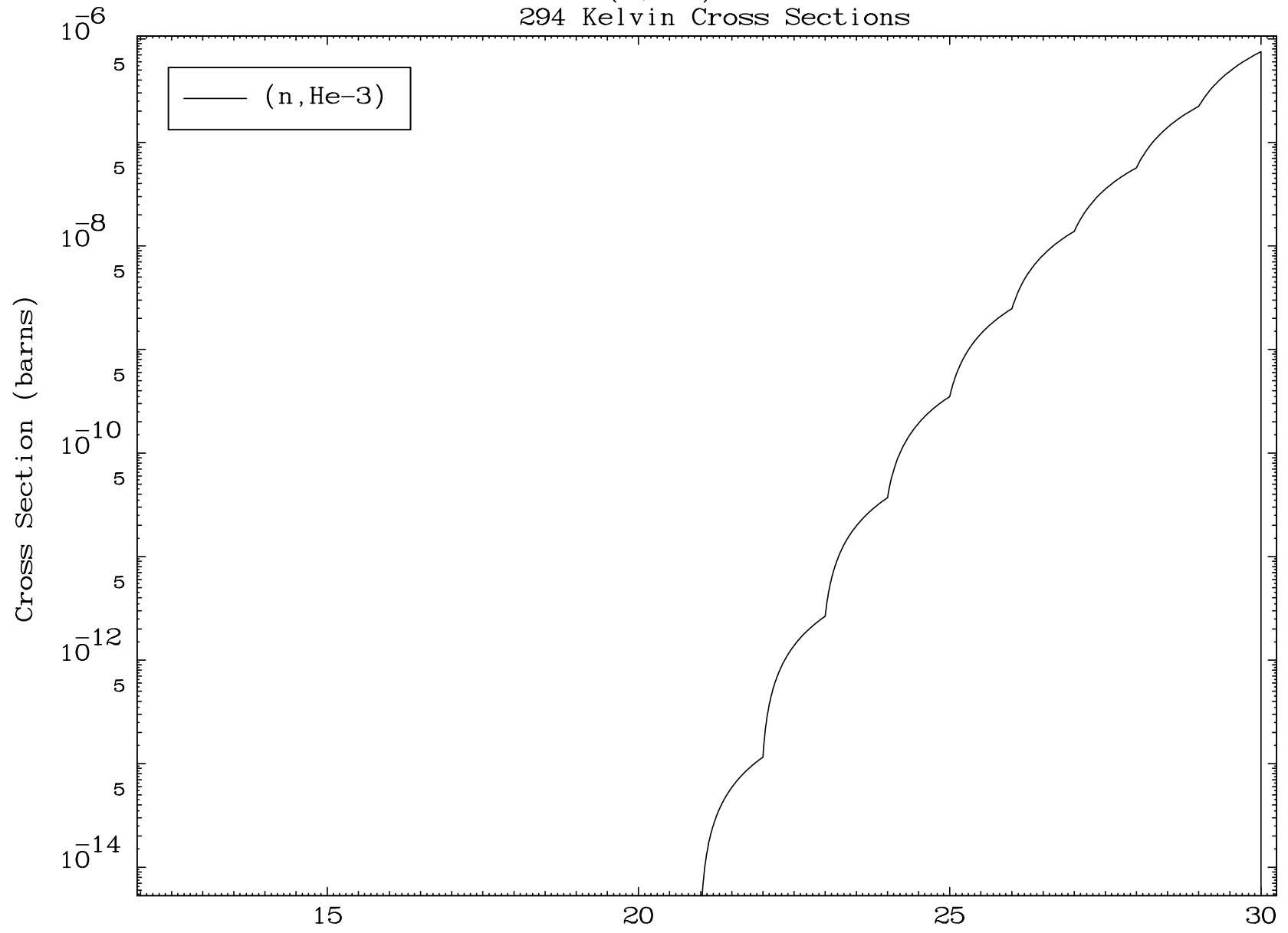




MAT 7270

(n,He3) Levels
294 Kelvin Cross Sections

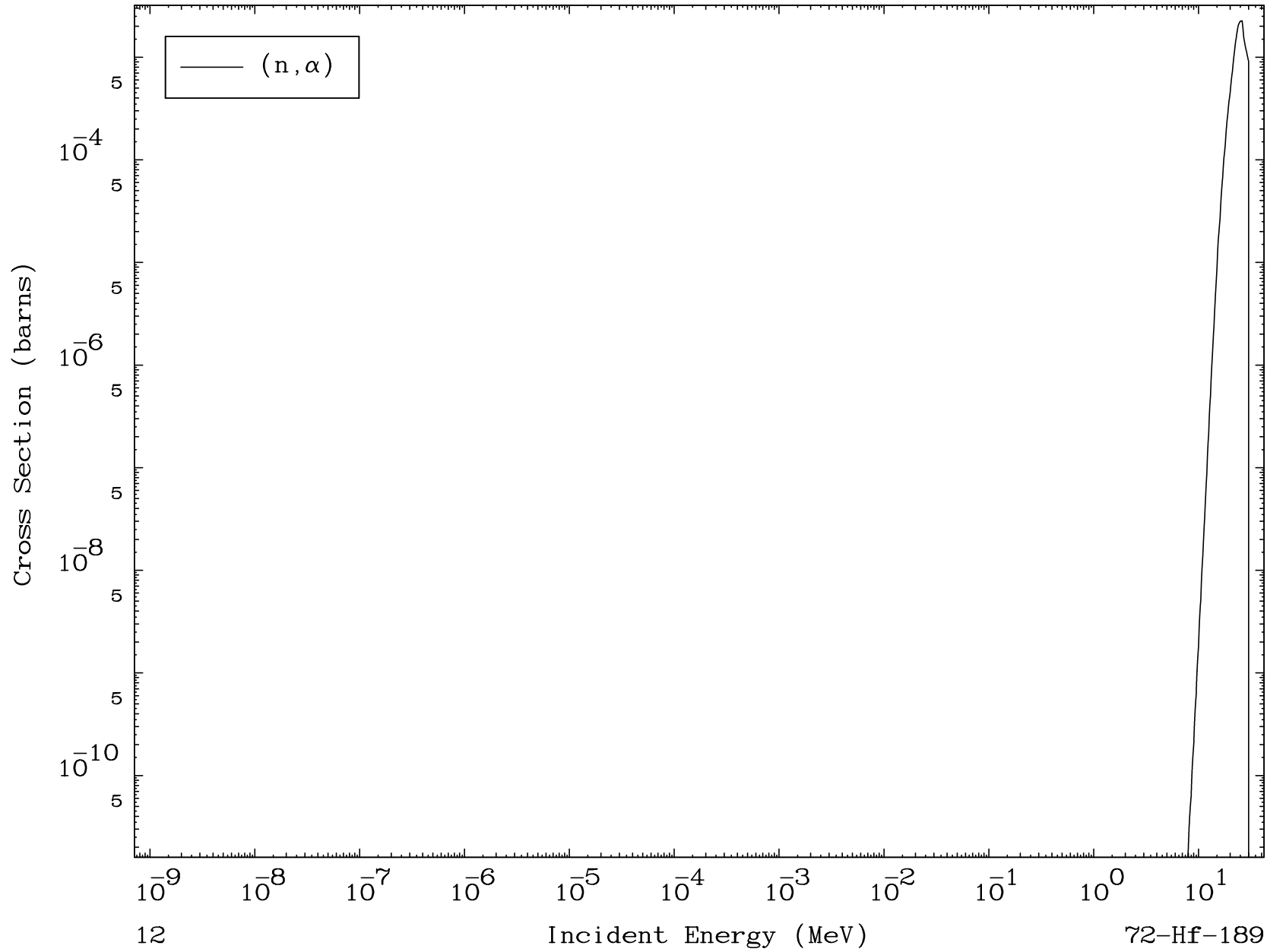
72-Hf-189

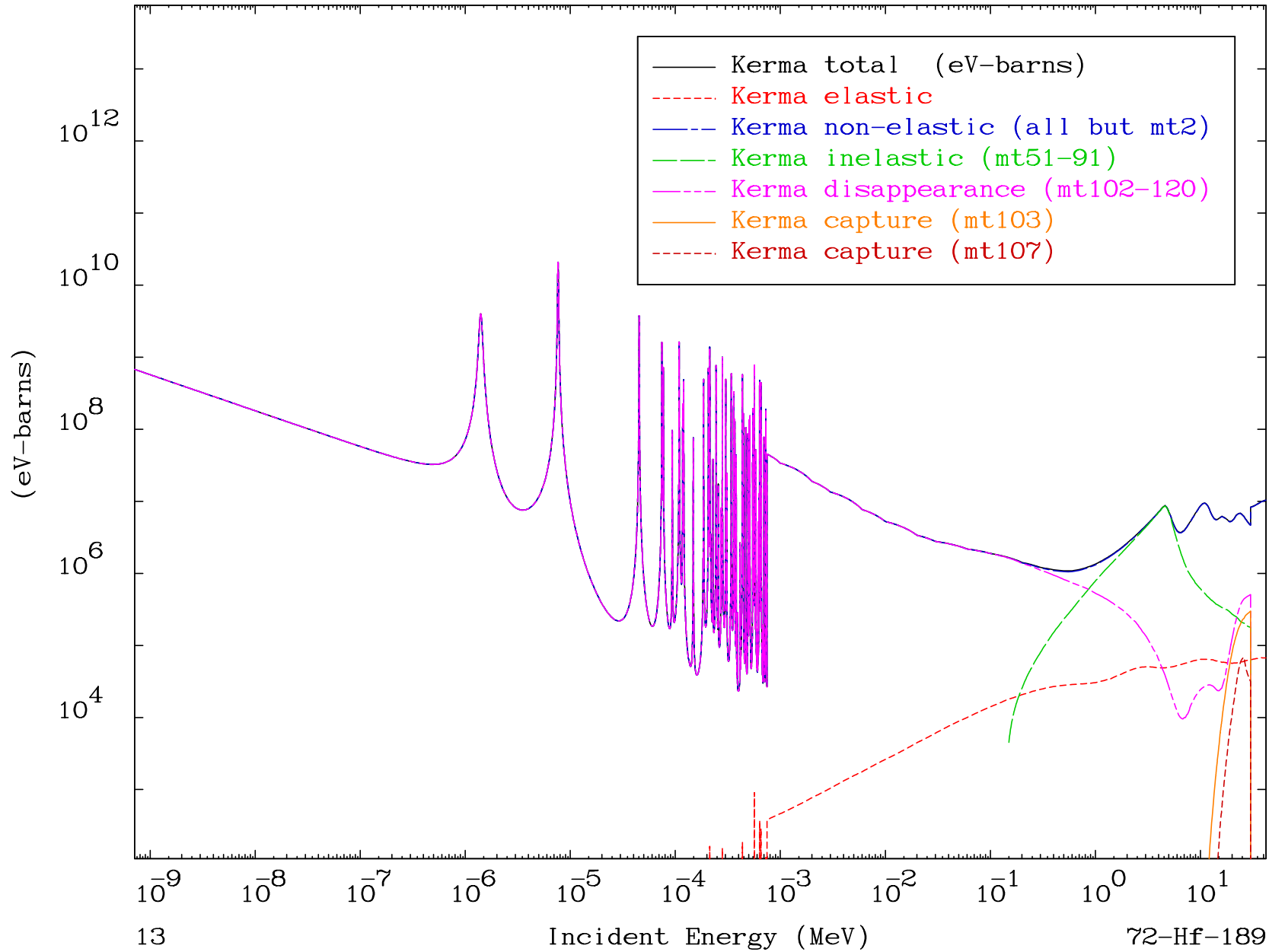


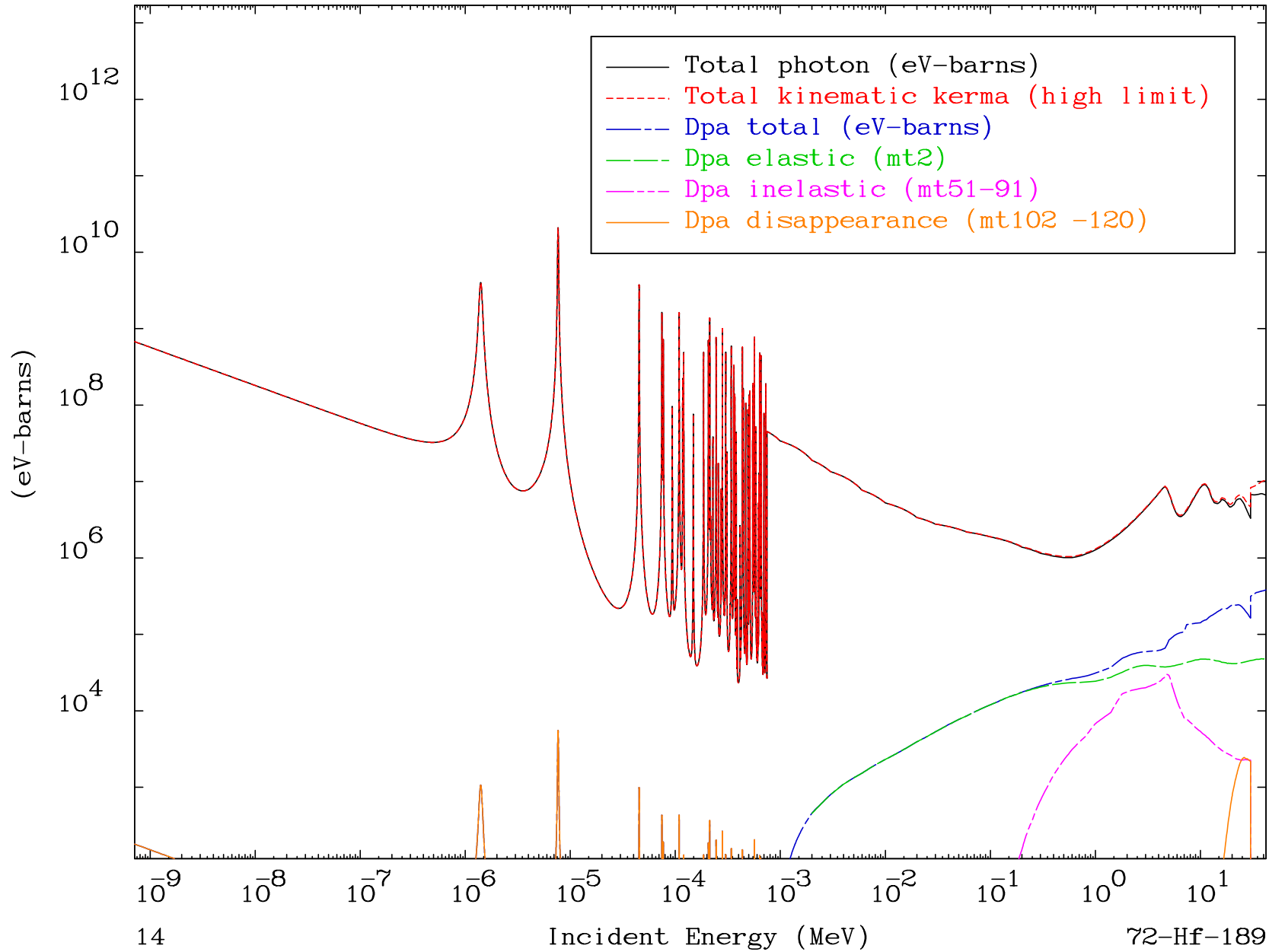
11

Incident Energy (MeV)

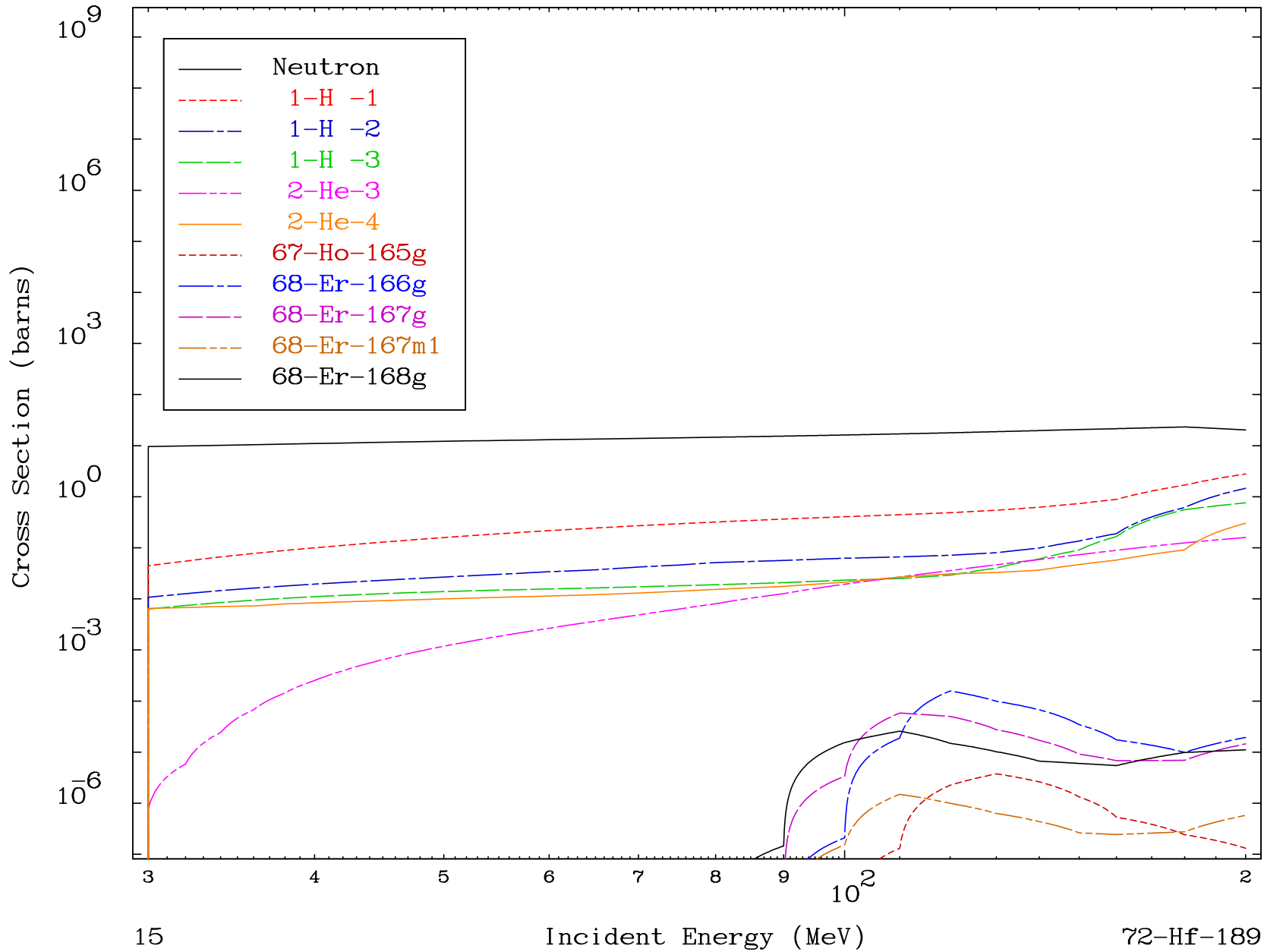
72-Hf-189

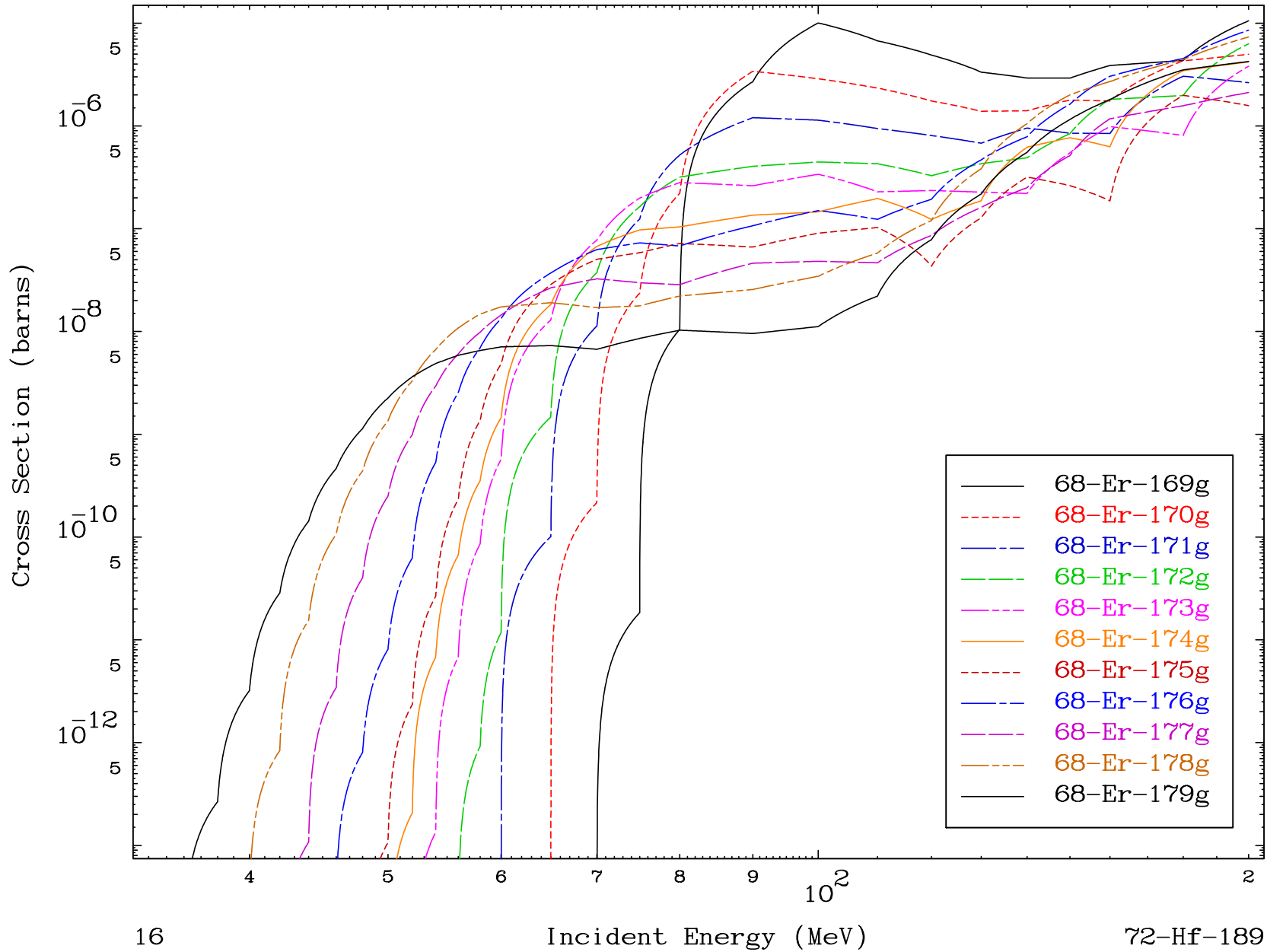




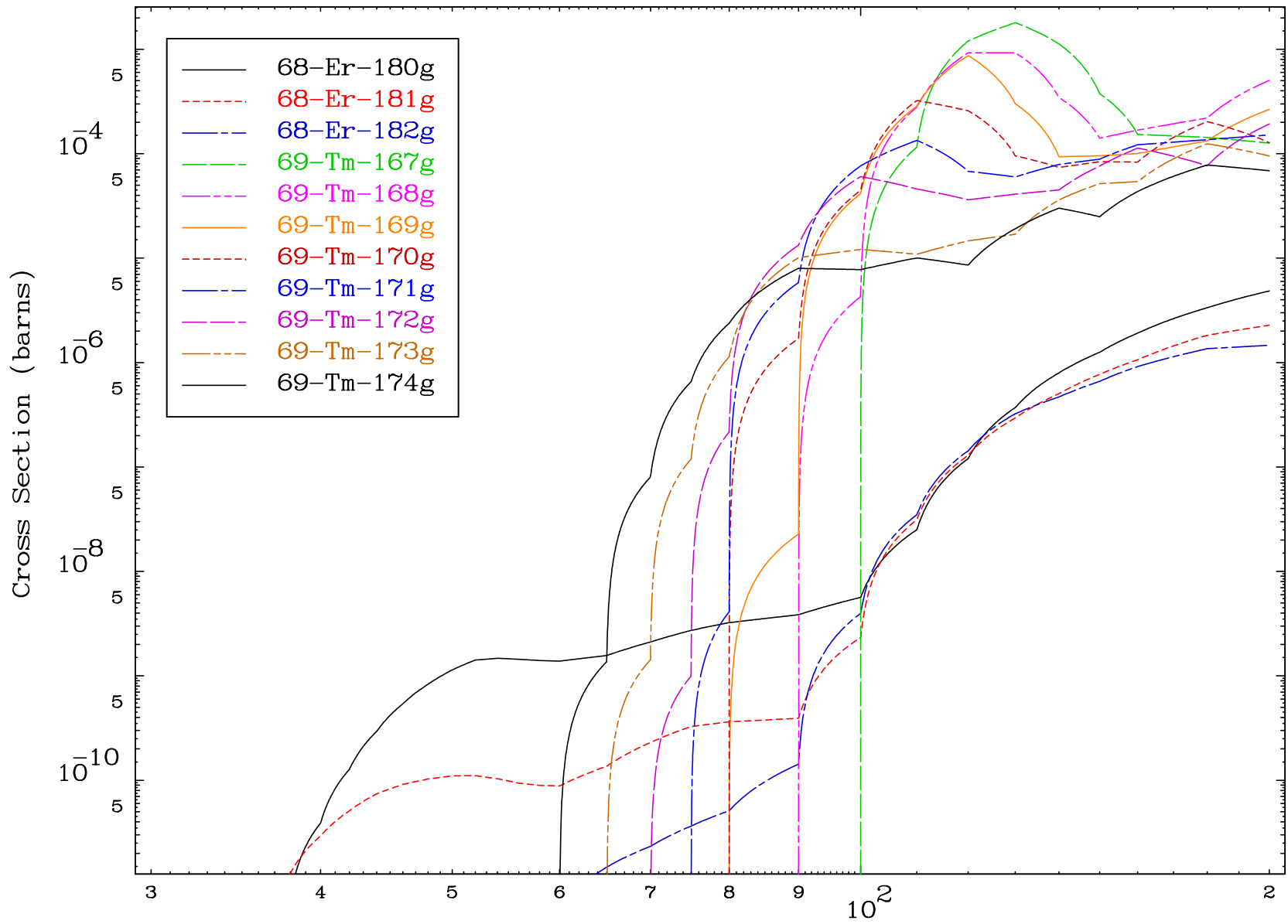


Radionuclide Production Cross Section

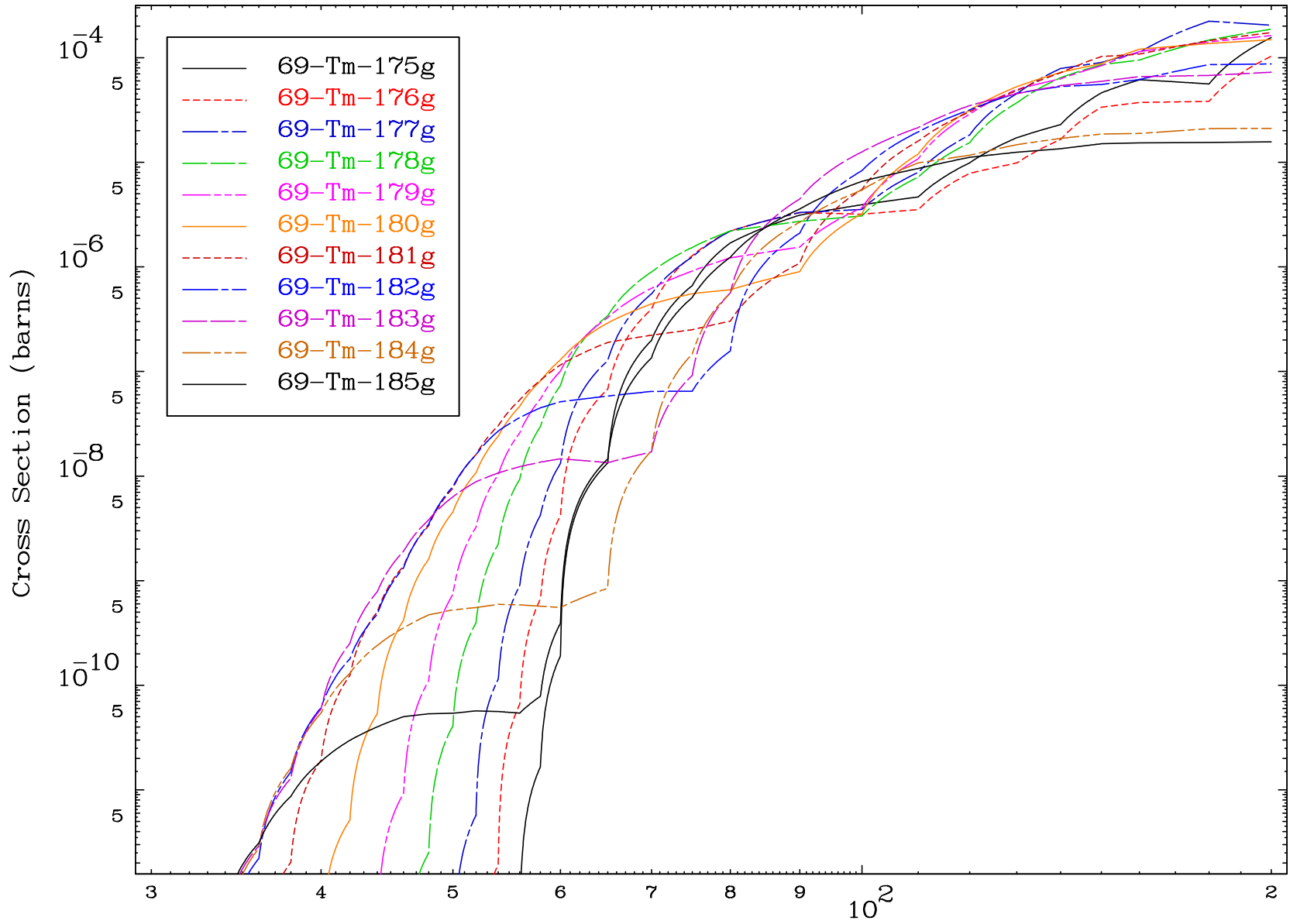




Radionuclide Production Cross Section



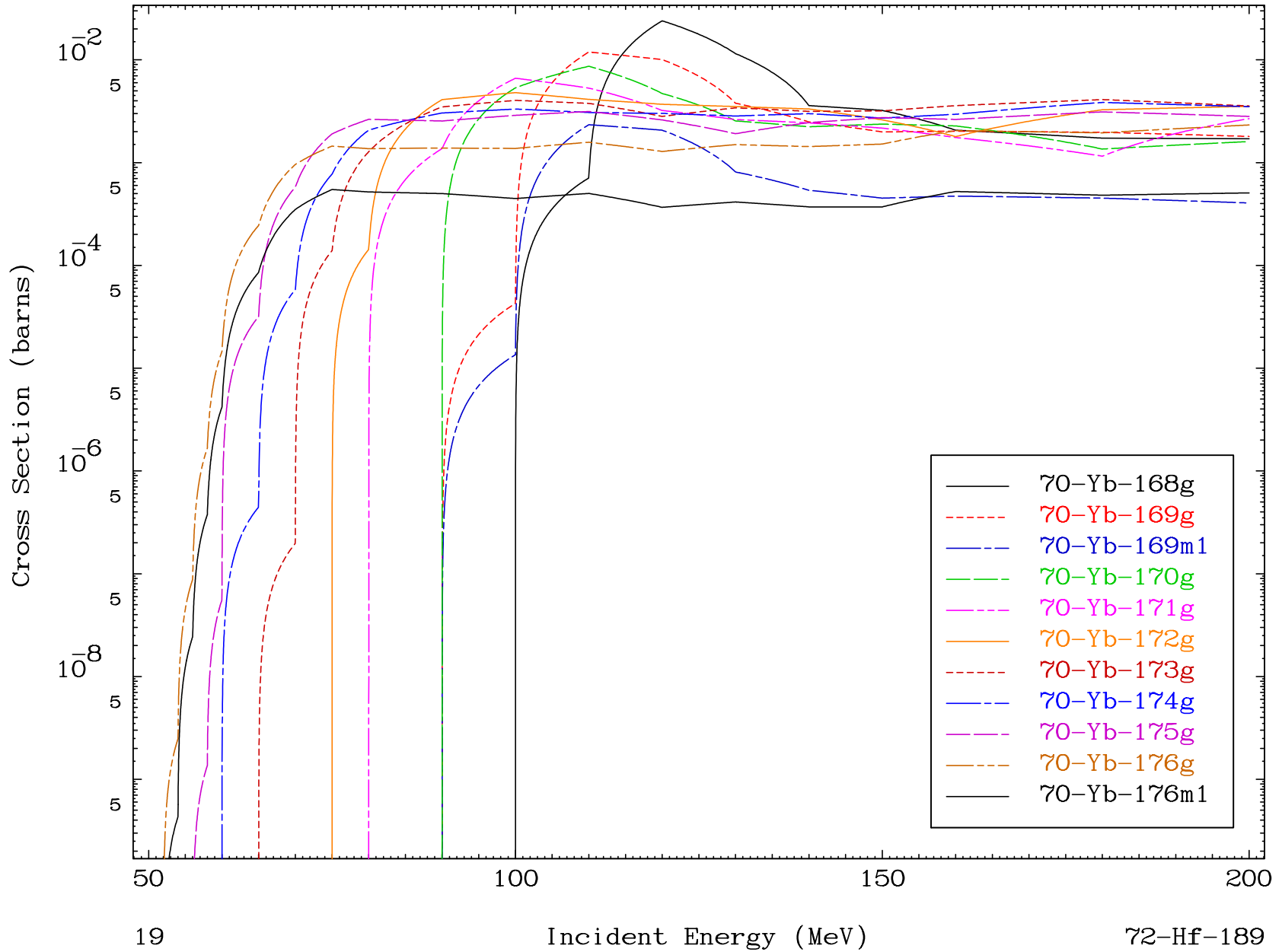
Radionuclide Production Cross Section

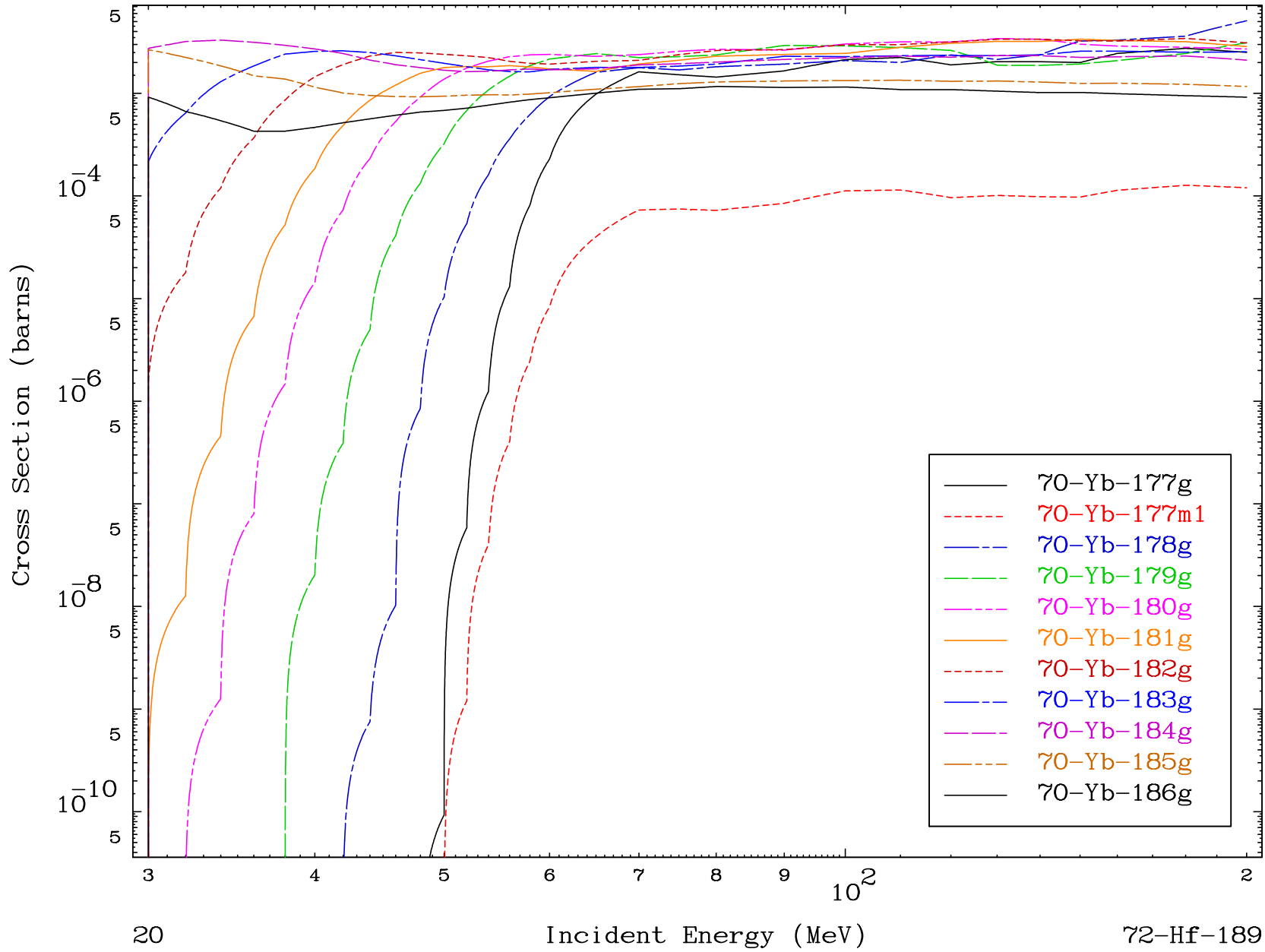


MAT 7270

(n,remainder)
Radionuclide Production Cross Section

72-Hf-189

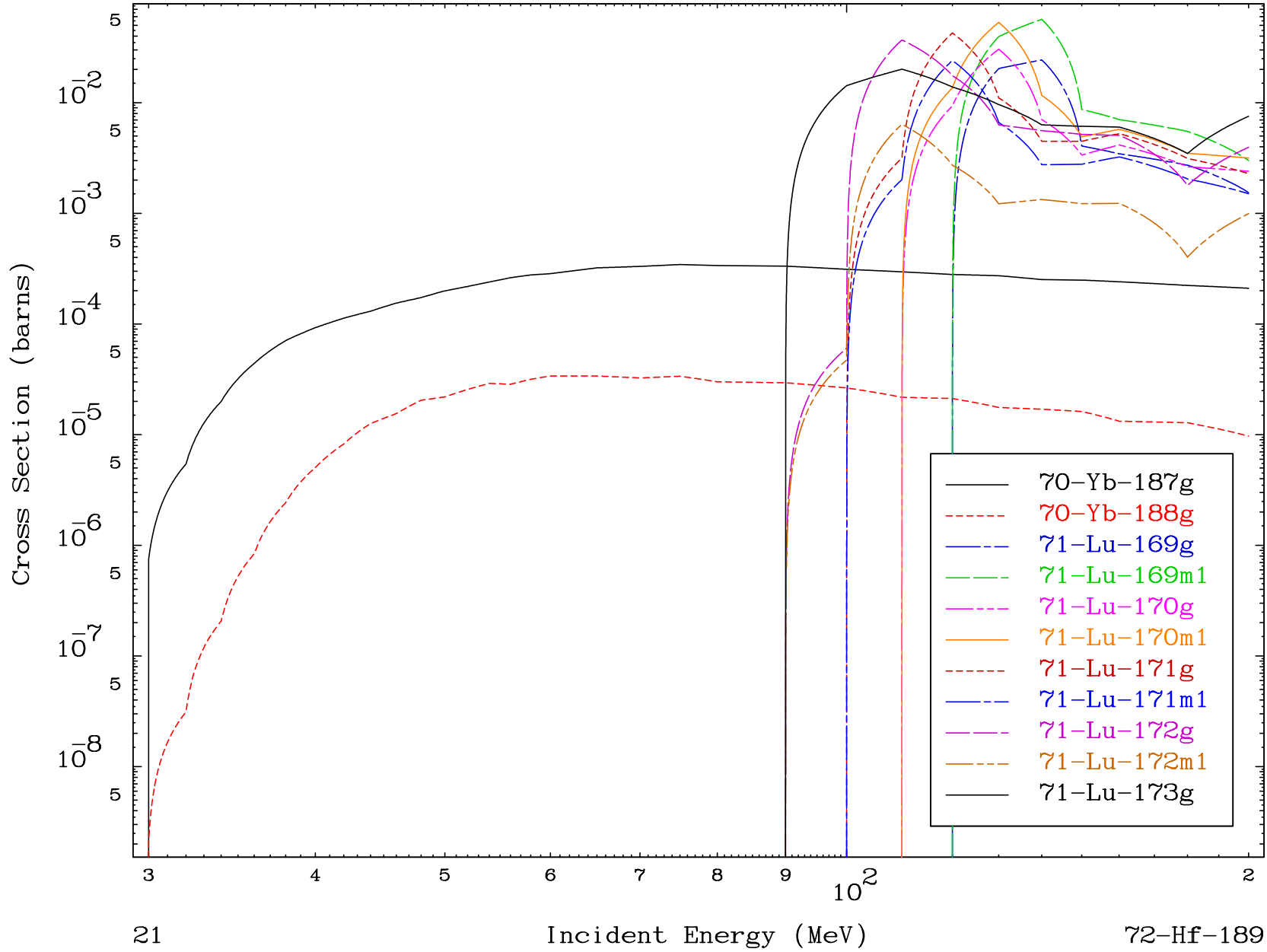




MAT 7270

(n,remainder)
Radionuclide Production Cross Section

72-Hf-189

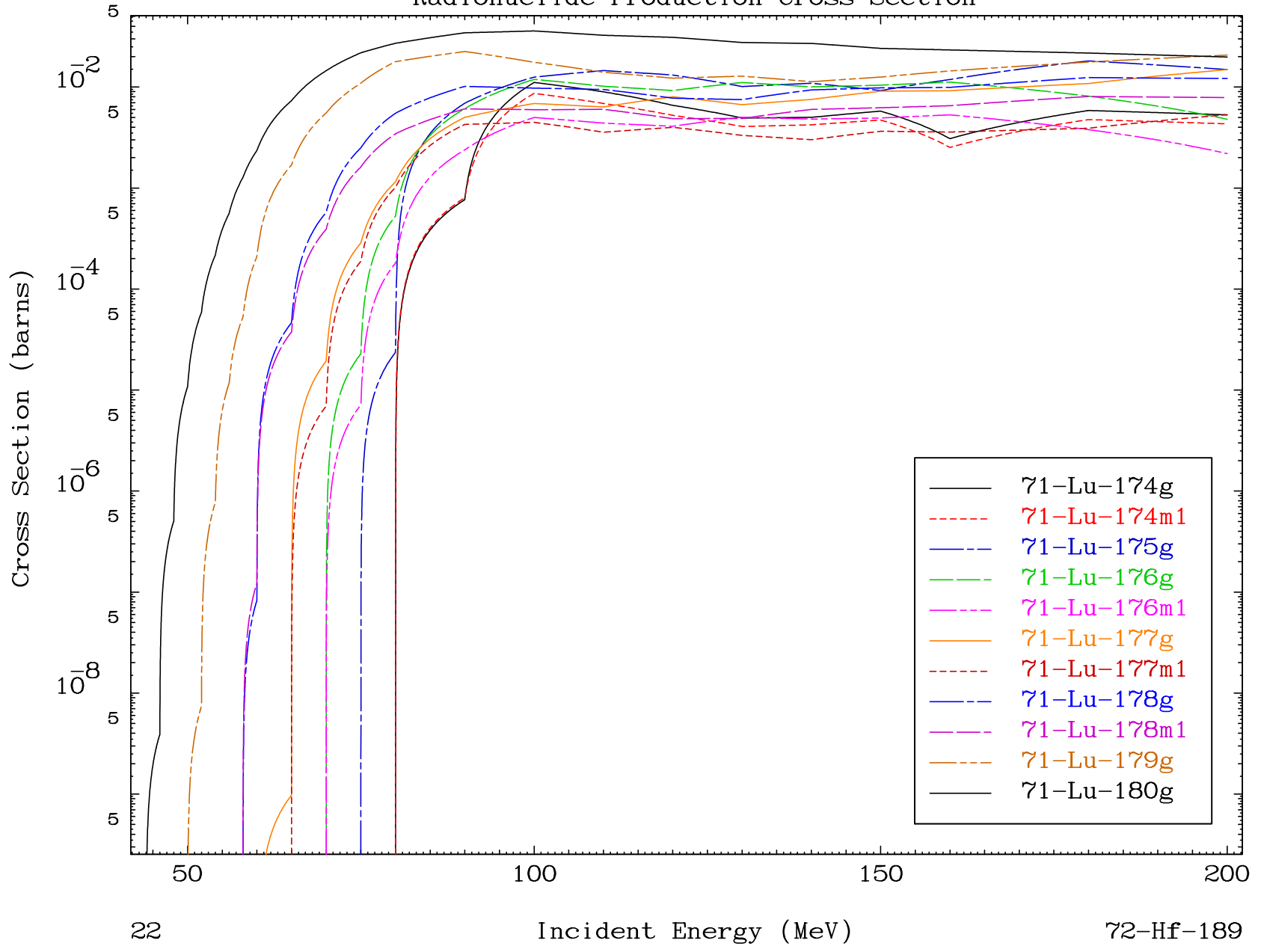


MAT 7270

(n,remainder)

72-Hf-189

Radionuclide Production Cross Section

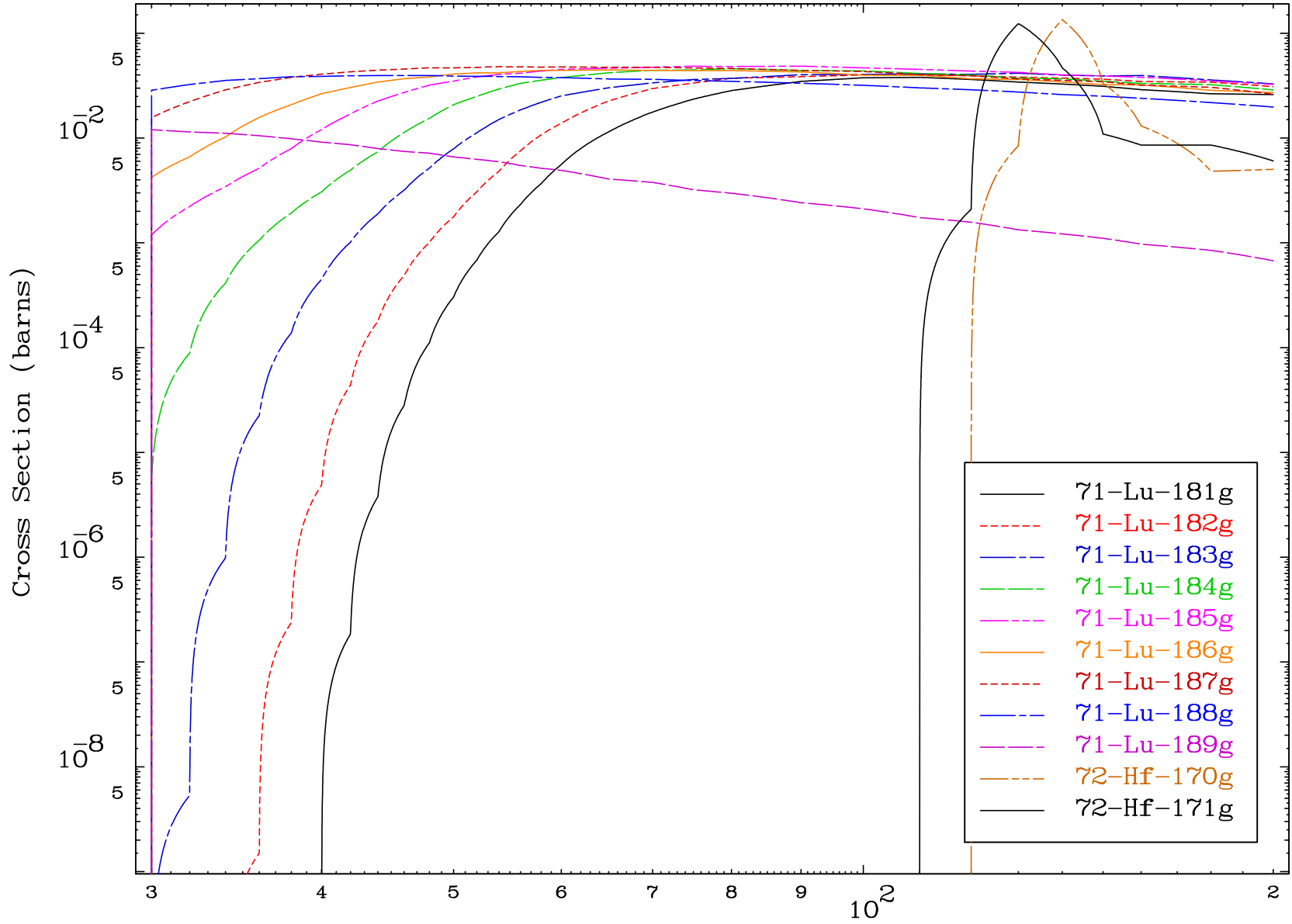


MAT 7270

(n,remainder)

72-Hf-189

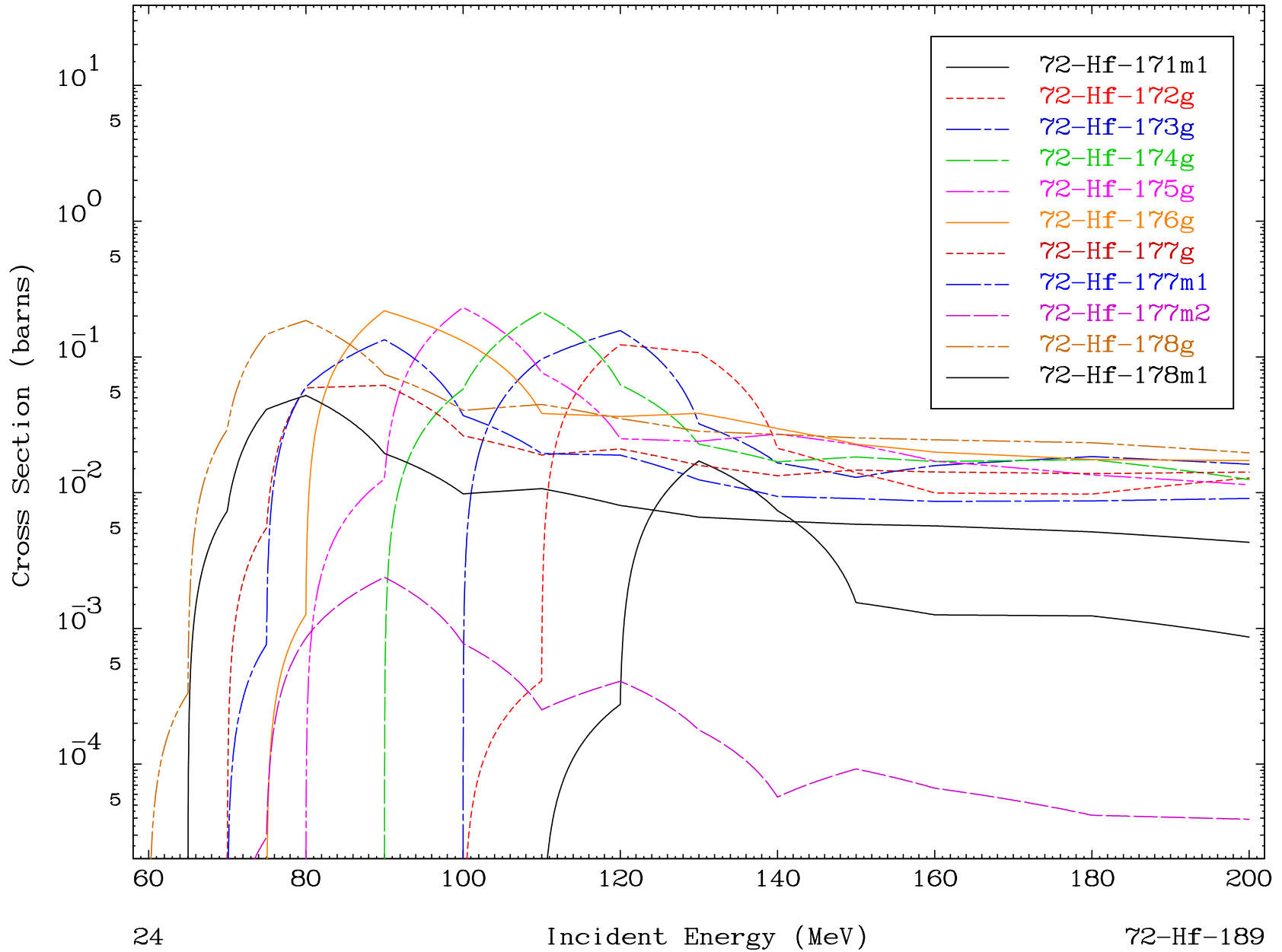
Radionuclide Production Cross Section



23

Incident Energy (MeV)

72-Hf-189

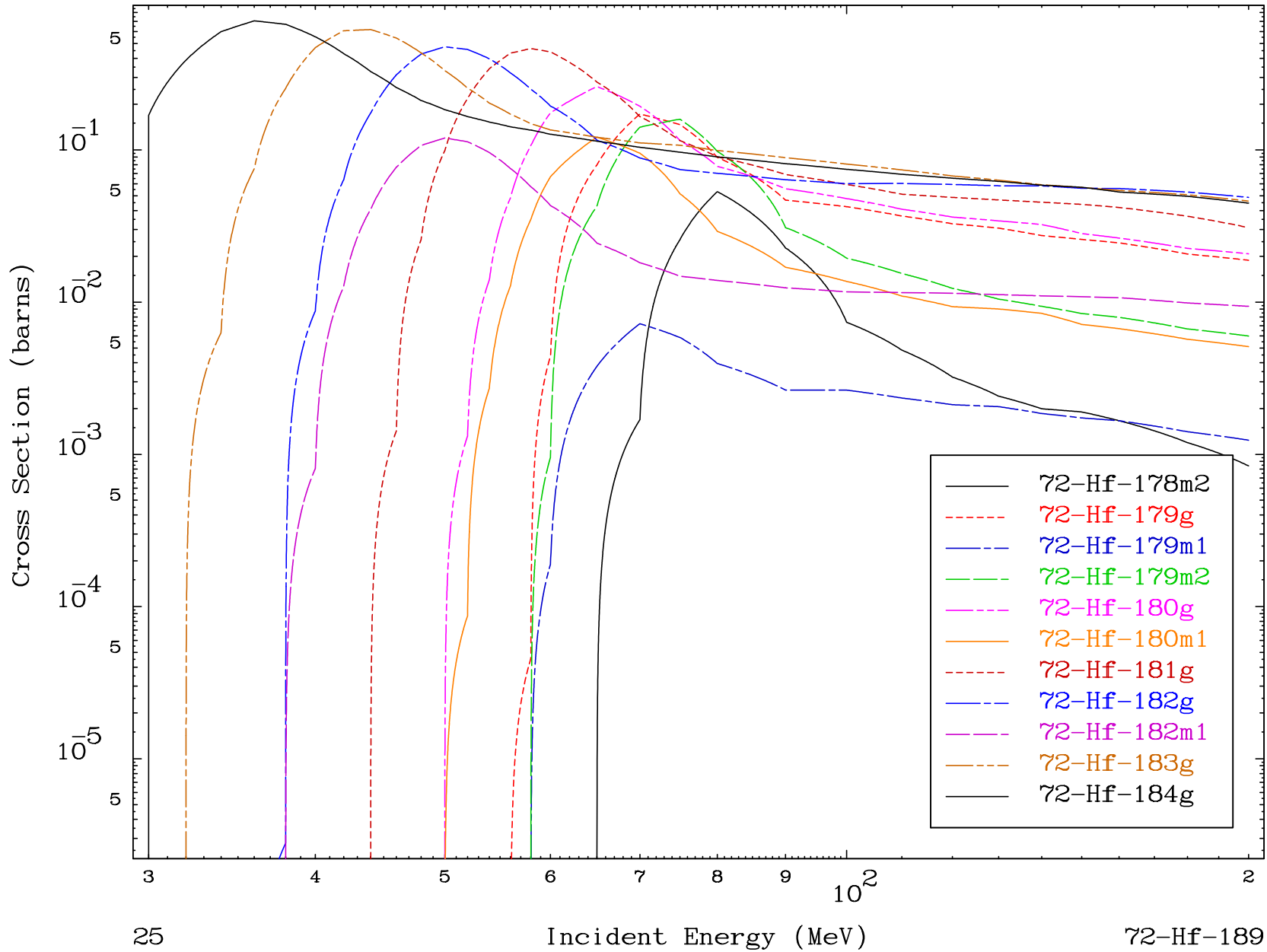


MAT 7270

(n,remainder)

72-Hf-189

Radionuclide Production Cross Section



25

Incident Energy (MeV)

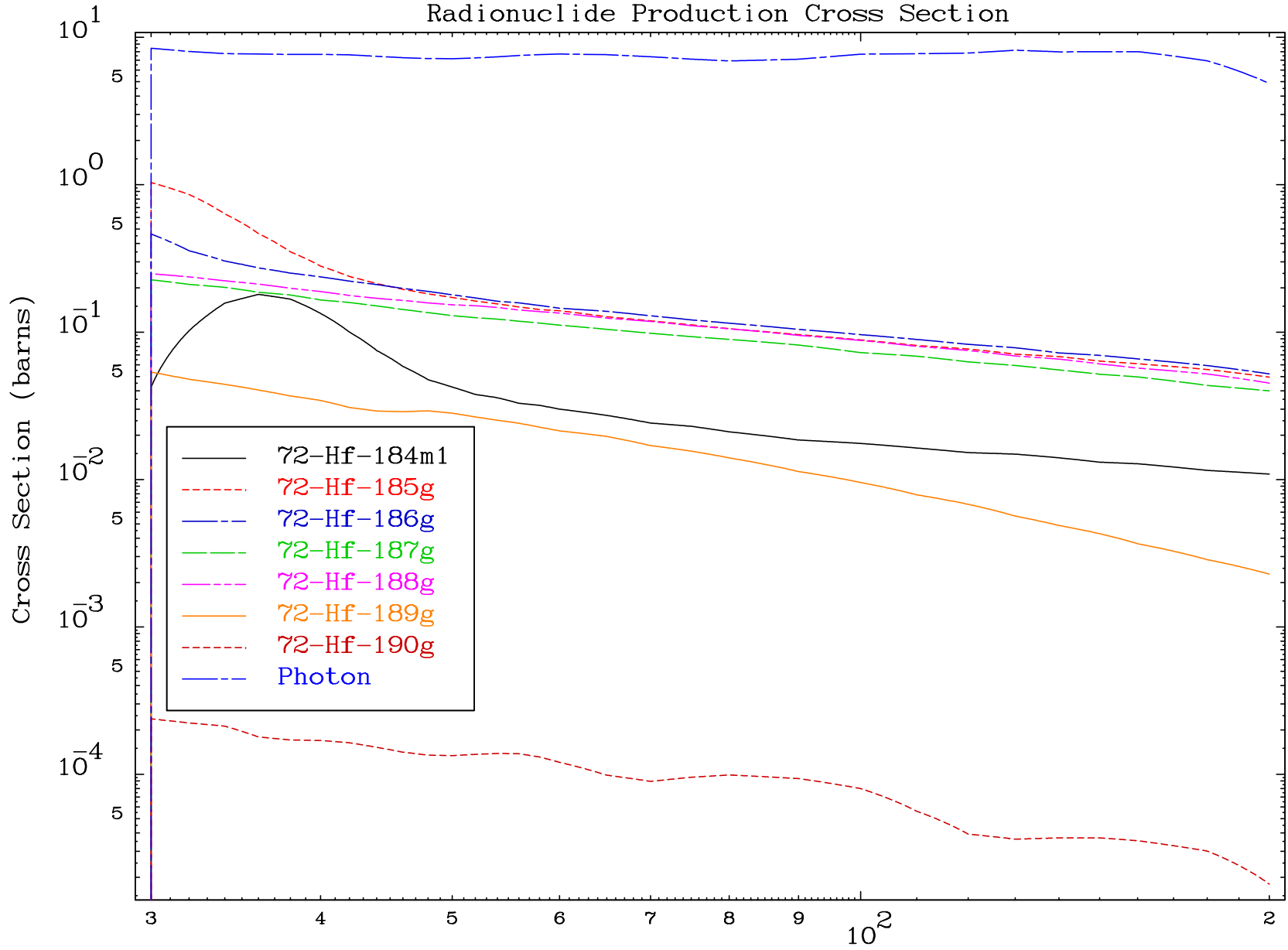
72-Hf-189

MAT 7270

(n,remainder)

72-Hf-189

Radionuclide Production Cross Section



26

Incident Energy (MeV)

72-Hf-189