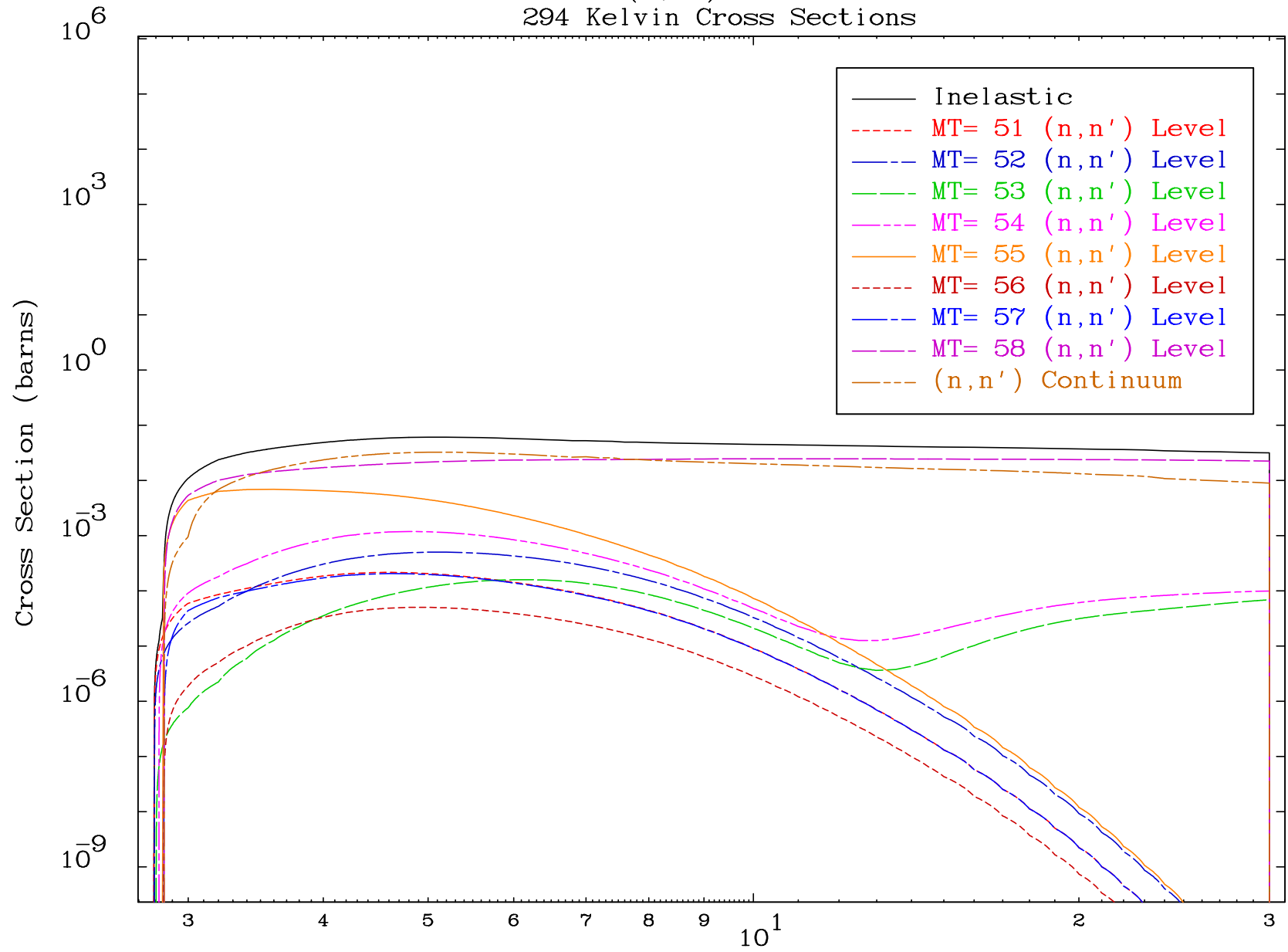


MAT 4886

(n,n') Level  
294 Kelvin Cross Sections

49-In-100



6

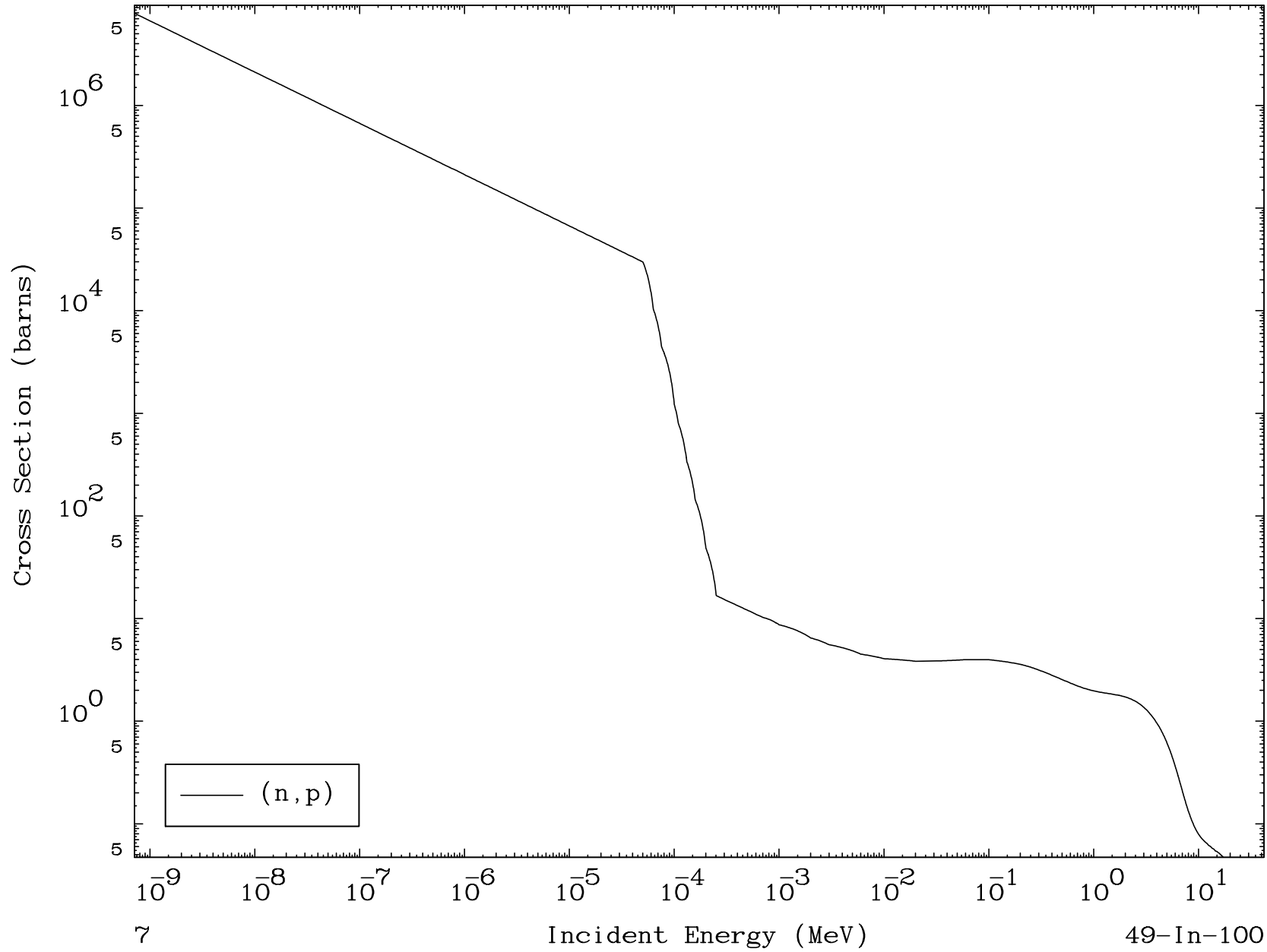
Incident Energy (MeV)

49-In-100

MAT 4886

(n,p) Levels  
294 Kelvin Cross Sections

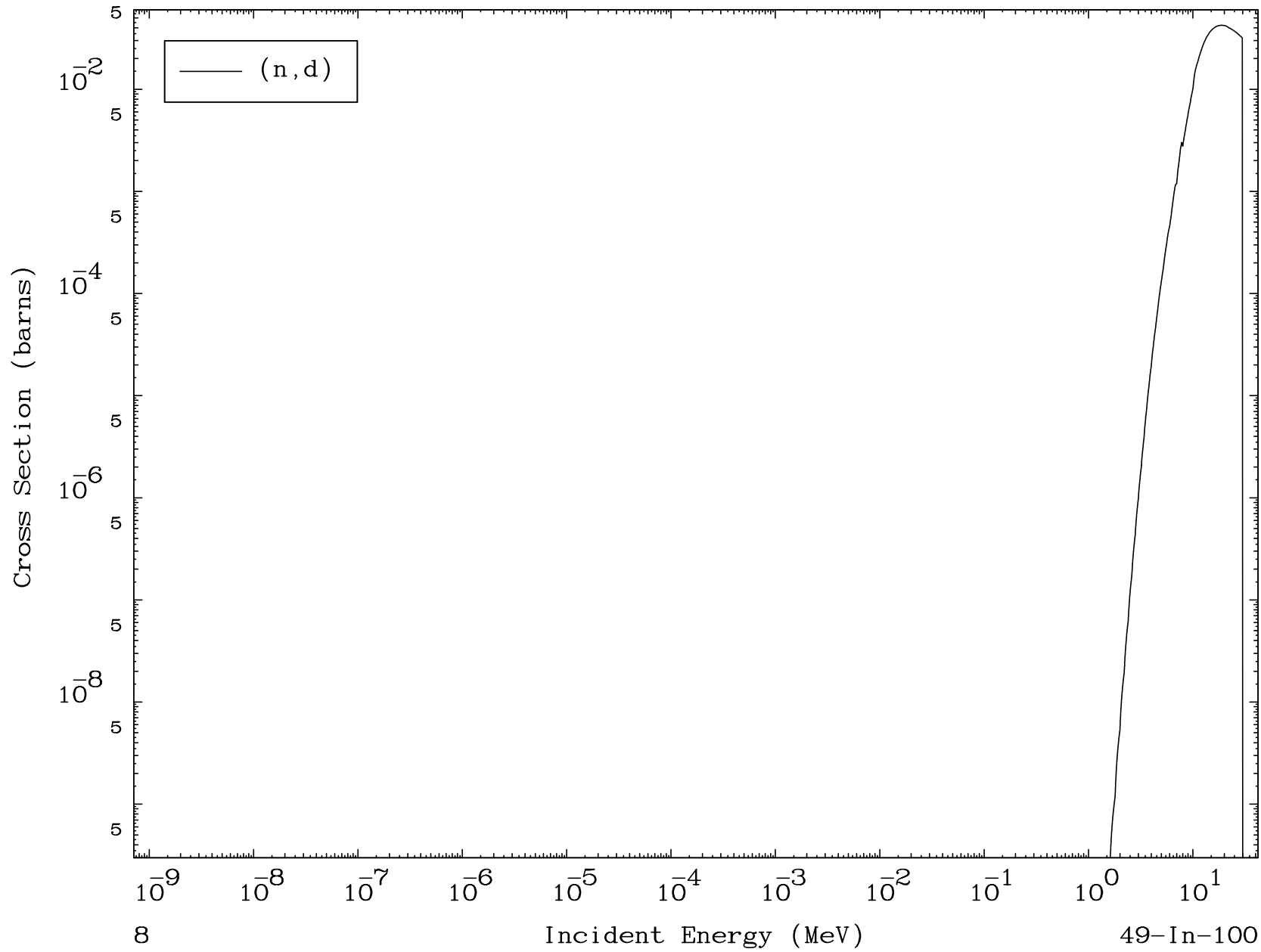
49-In-100



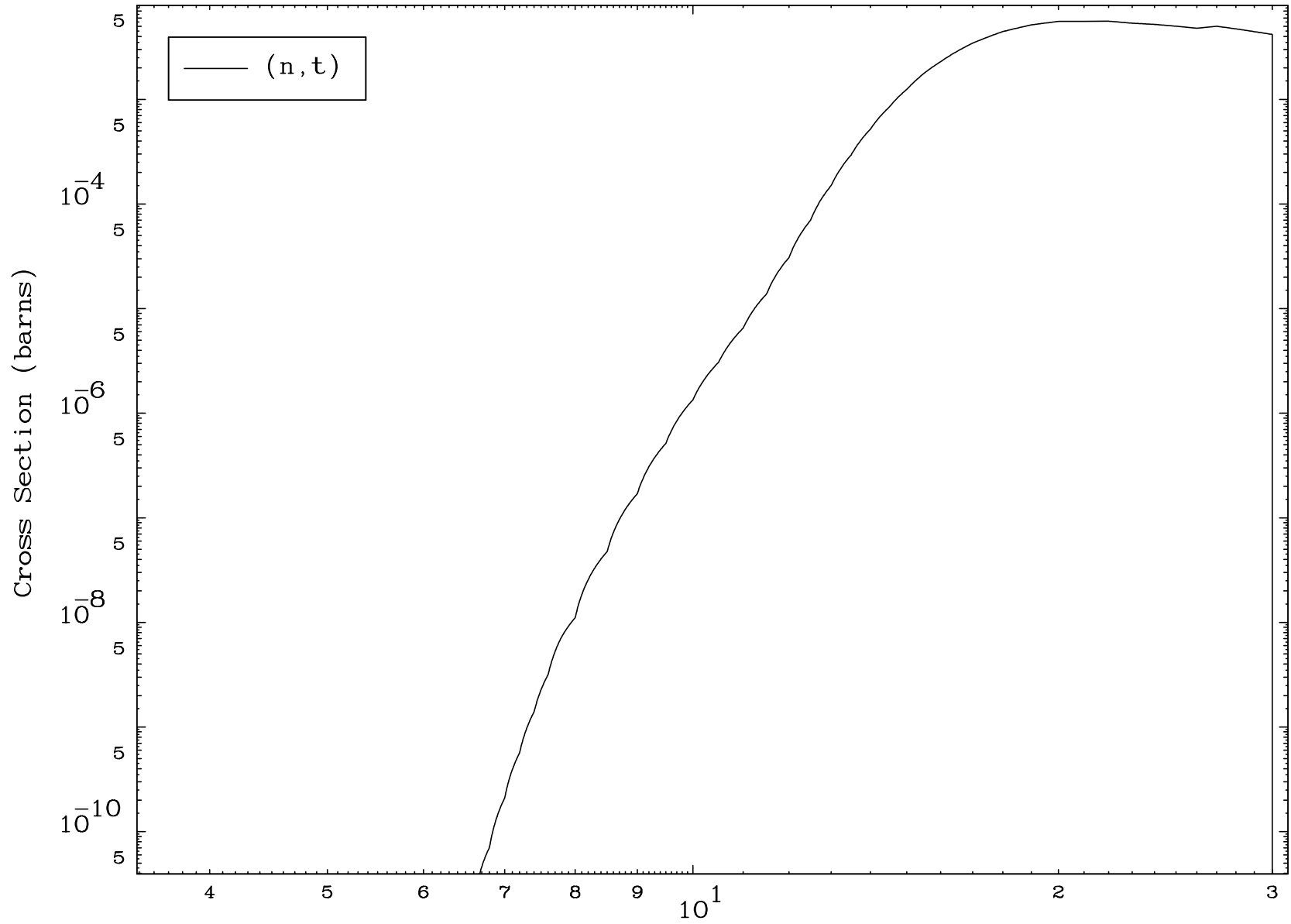
MAT 4886

(n,d) Levels  
294 Kelvin Cross Sections

49-In-100



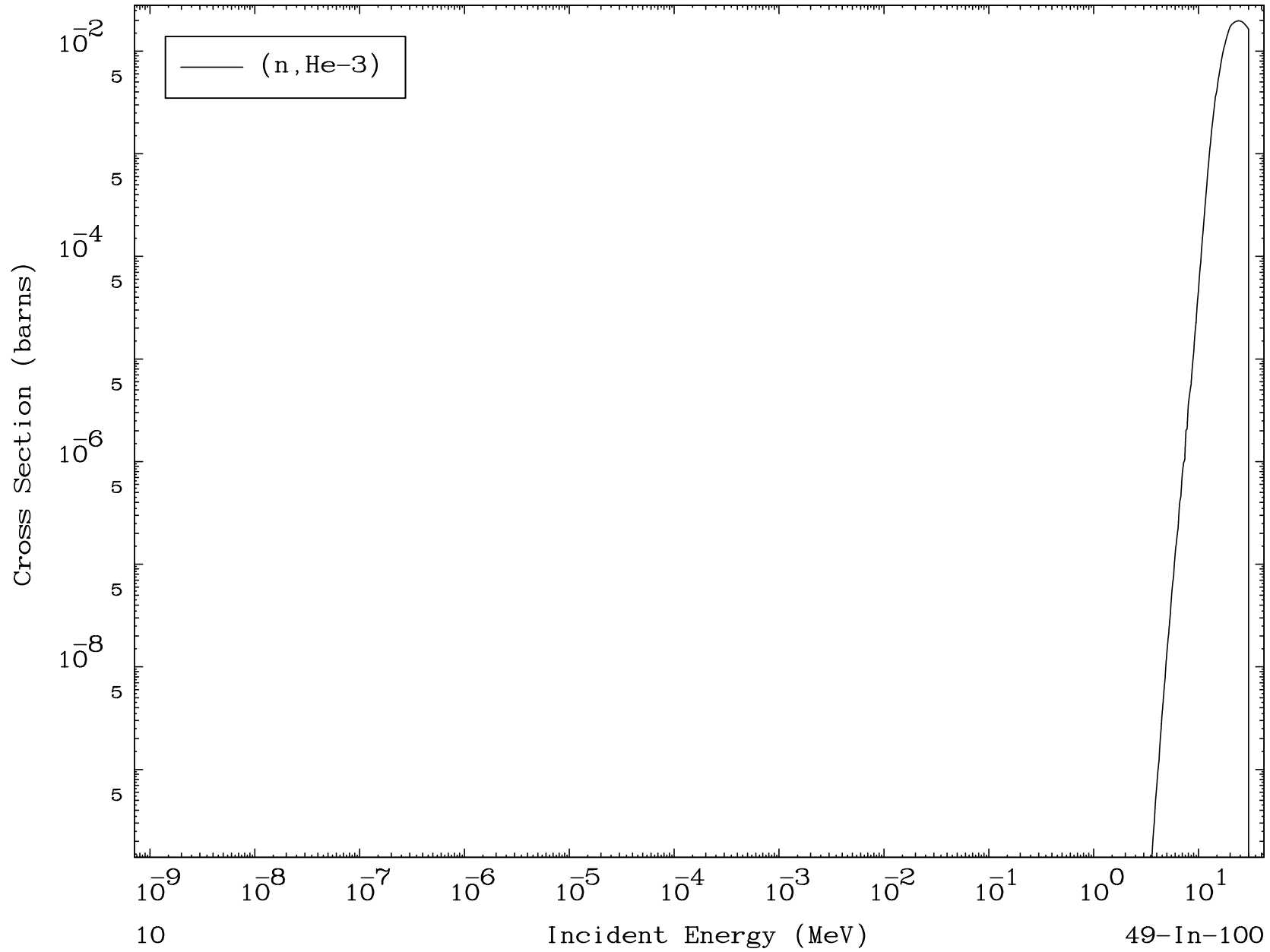




MAT 4886

(n,He3) Levels  
294 Kelvin Cross Sections

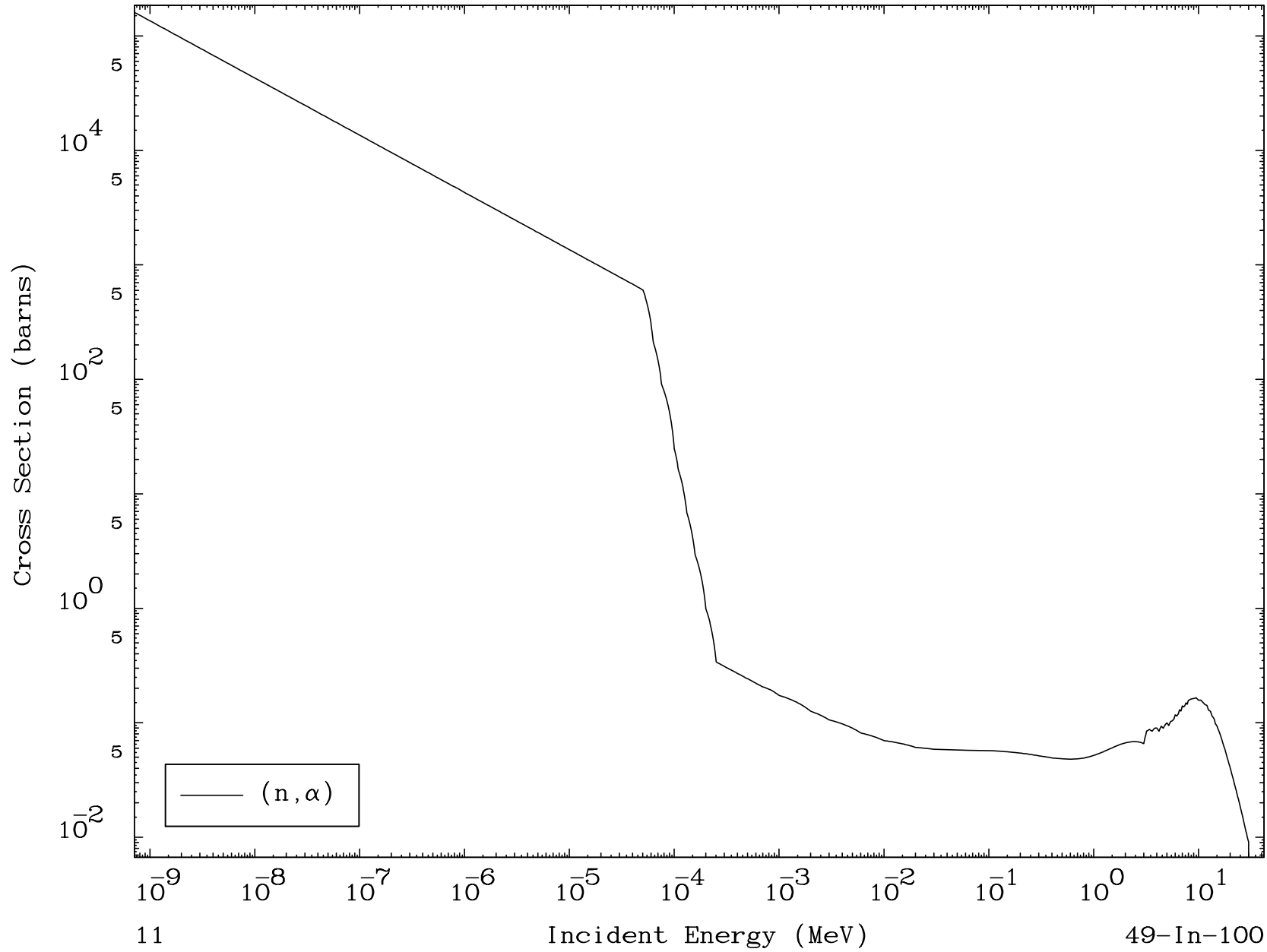
49-In-100



MAT 4886

(n,  $\alpha$ ) Levels  
294 Kelvin Cross Sections

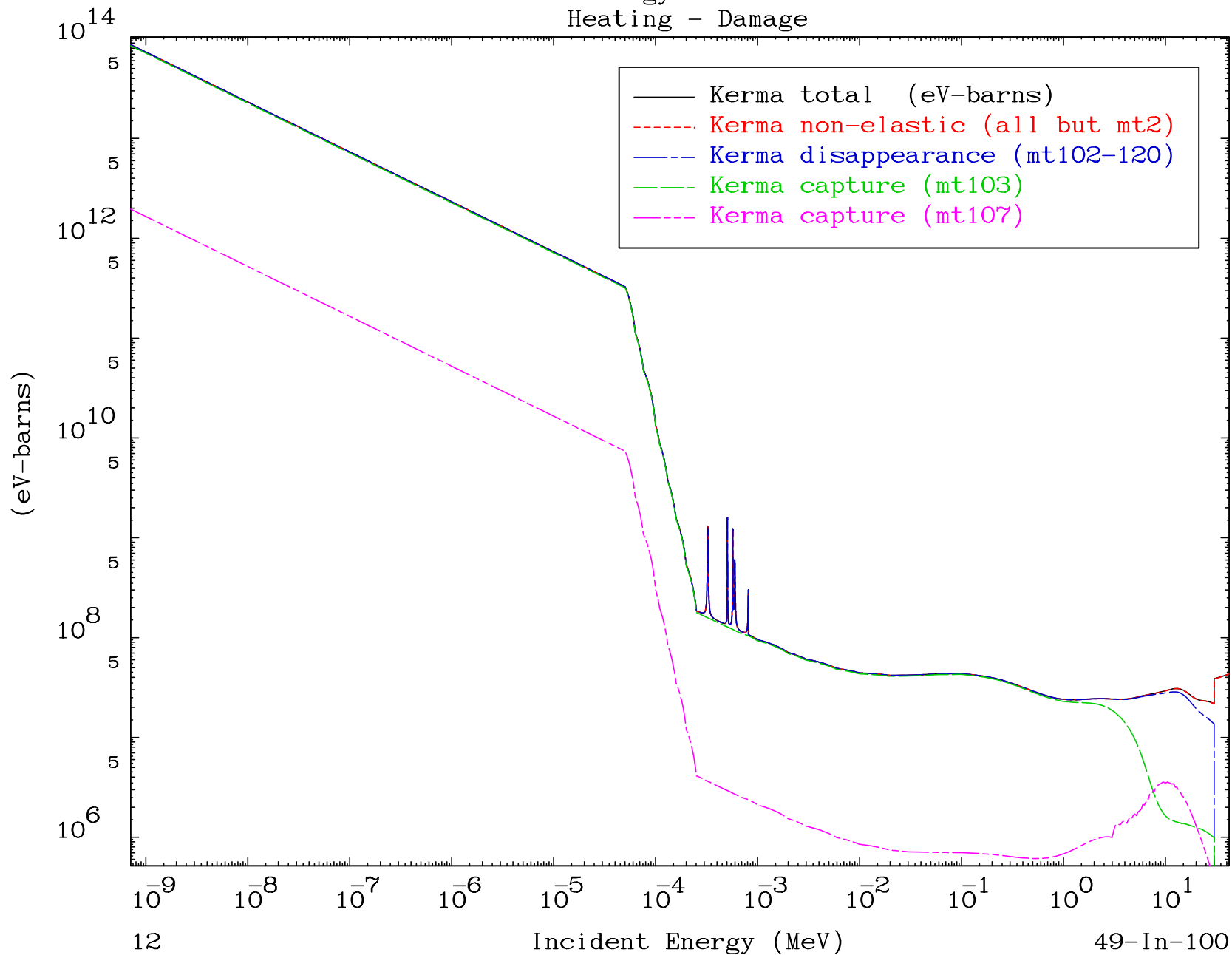
49-In-100



MAT 4886

Energy Release  
Heating - Damage

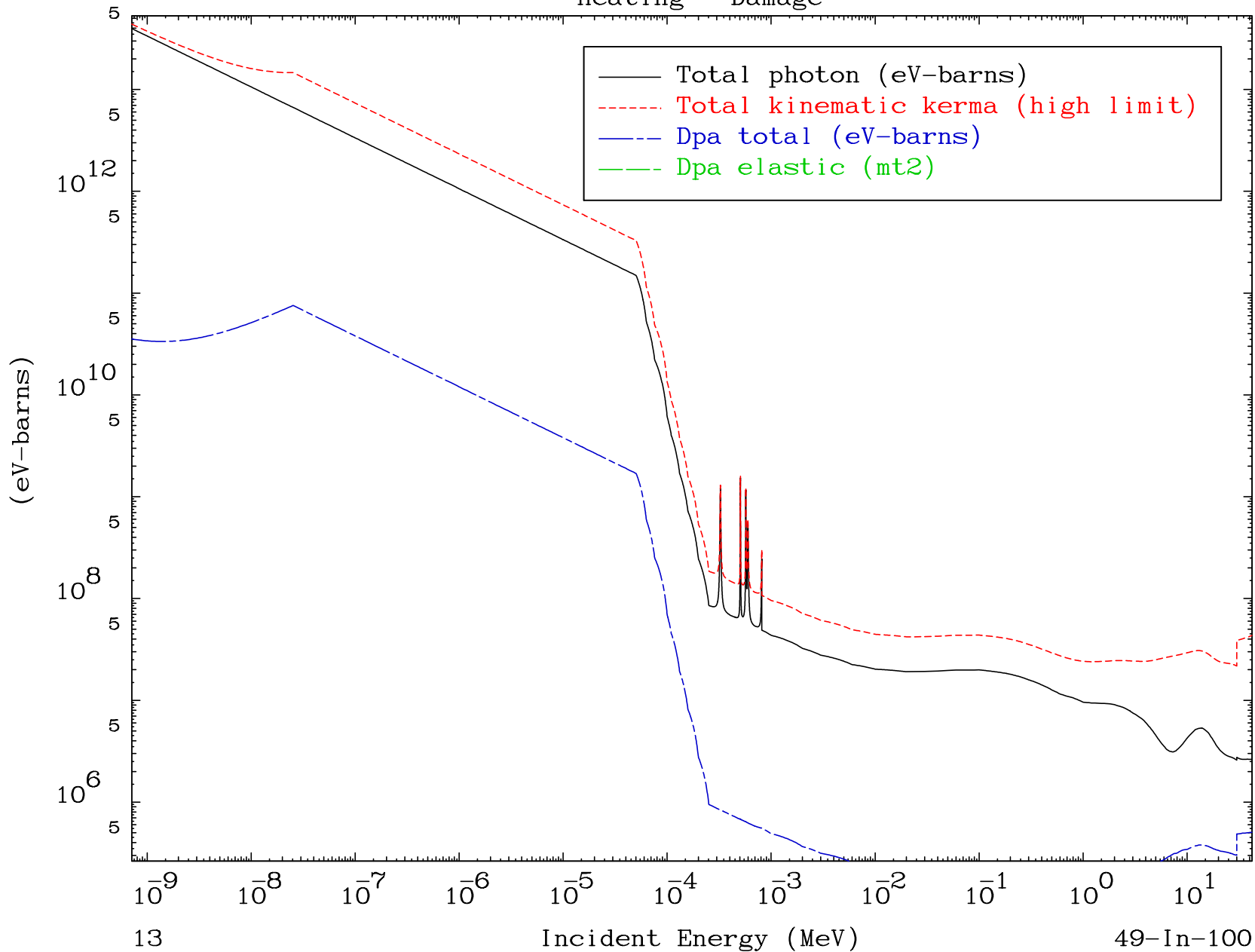
49-In-100

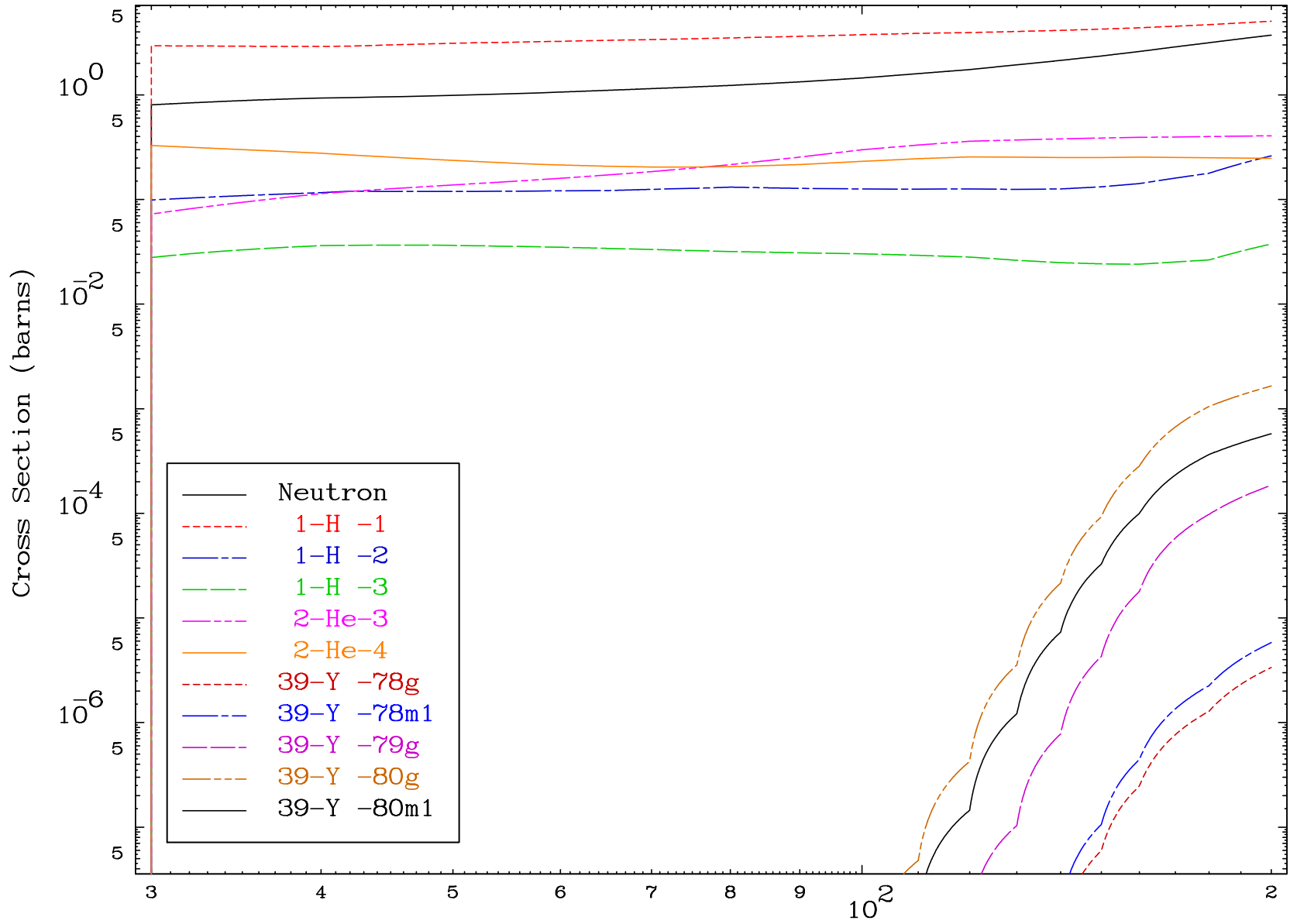


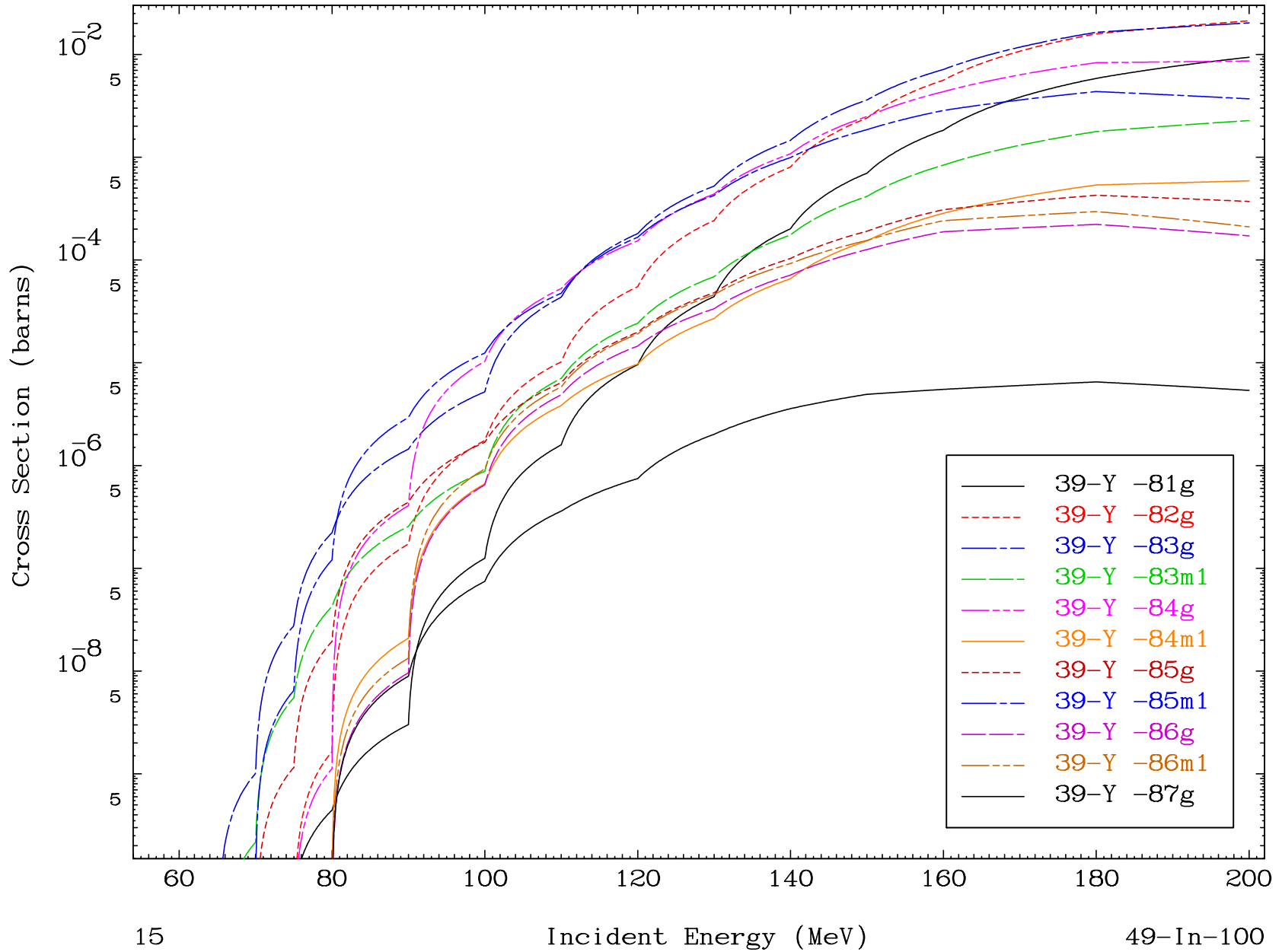
MAT 4886

Energy Release  
Heating - Damage

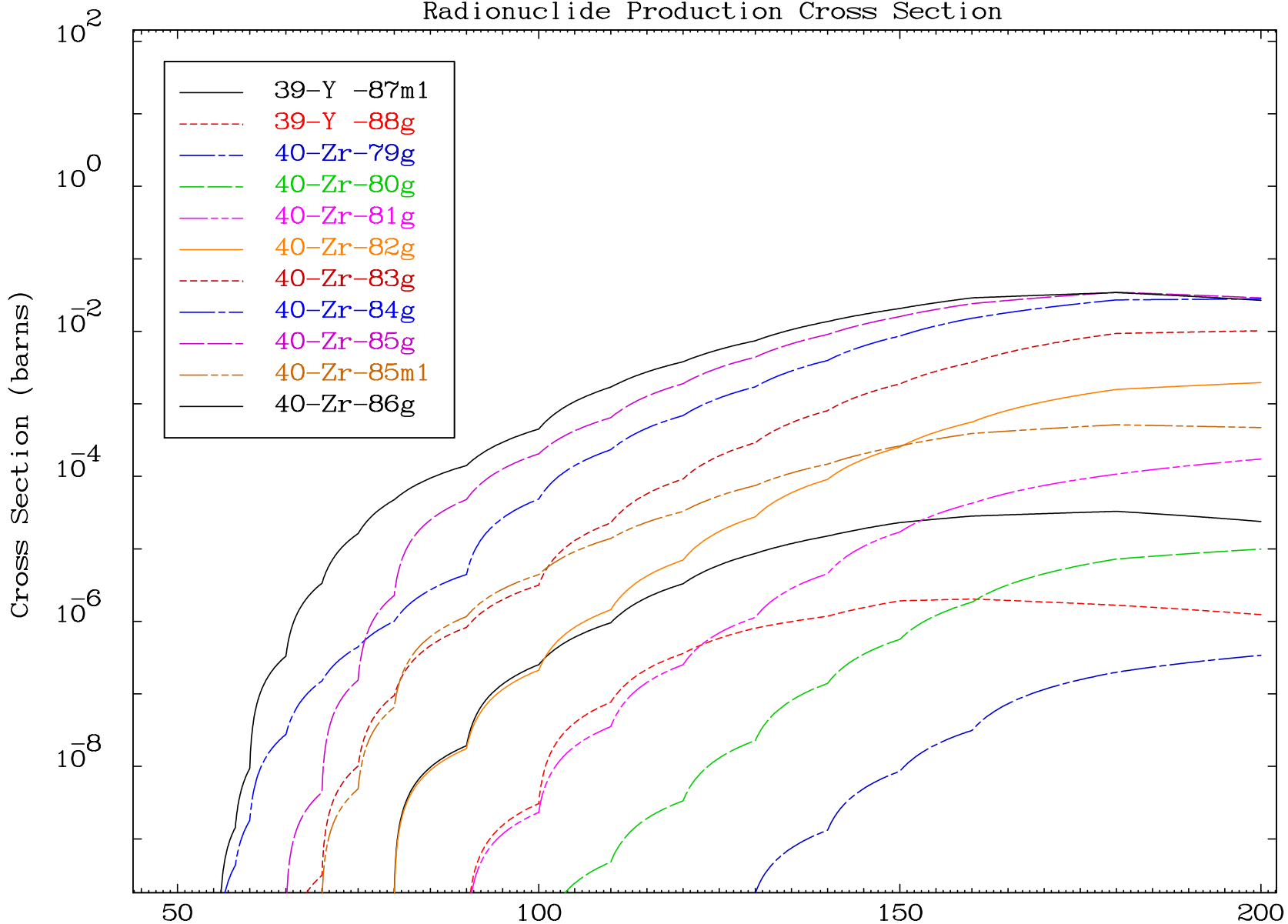
49-In-100





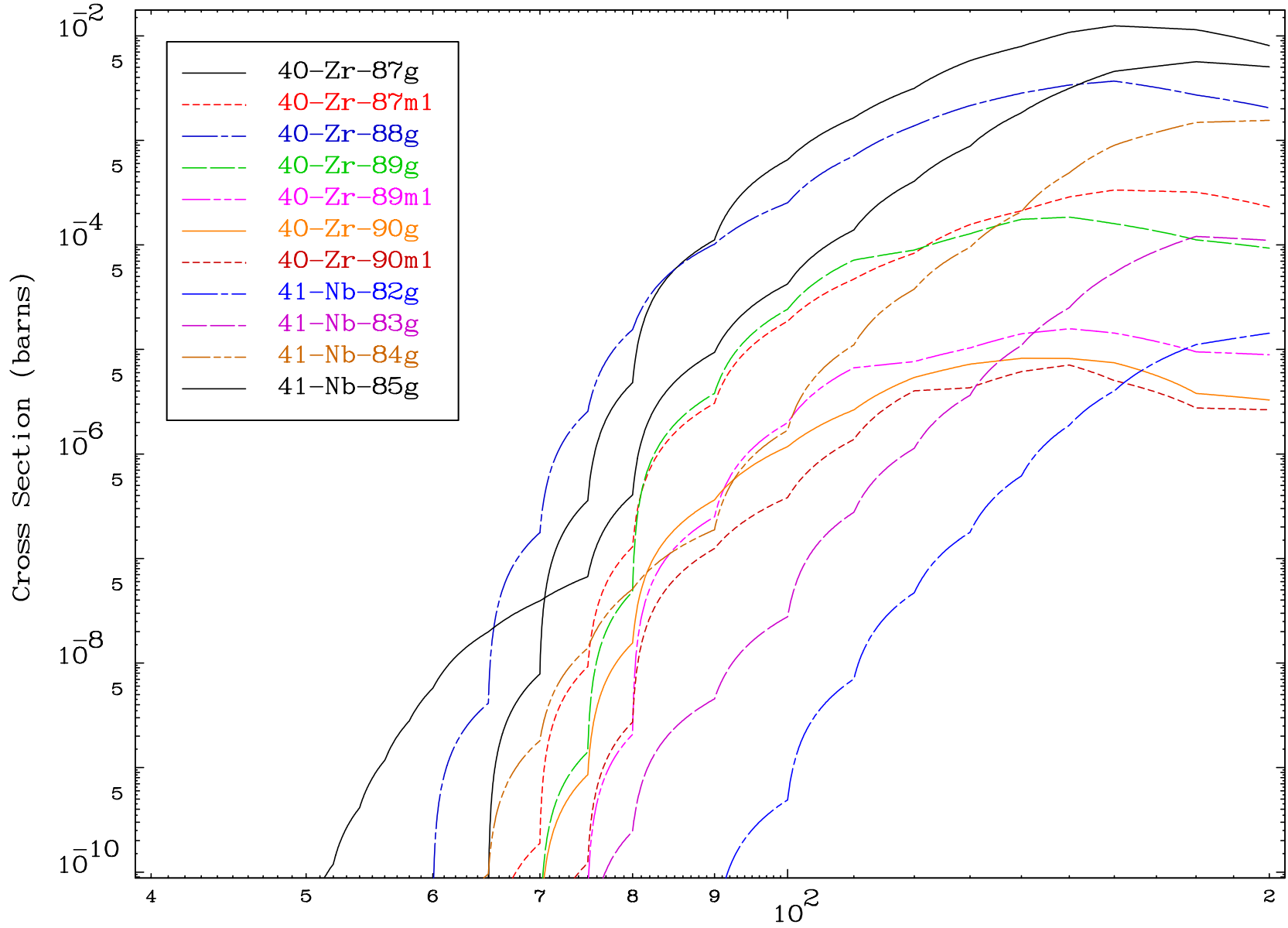


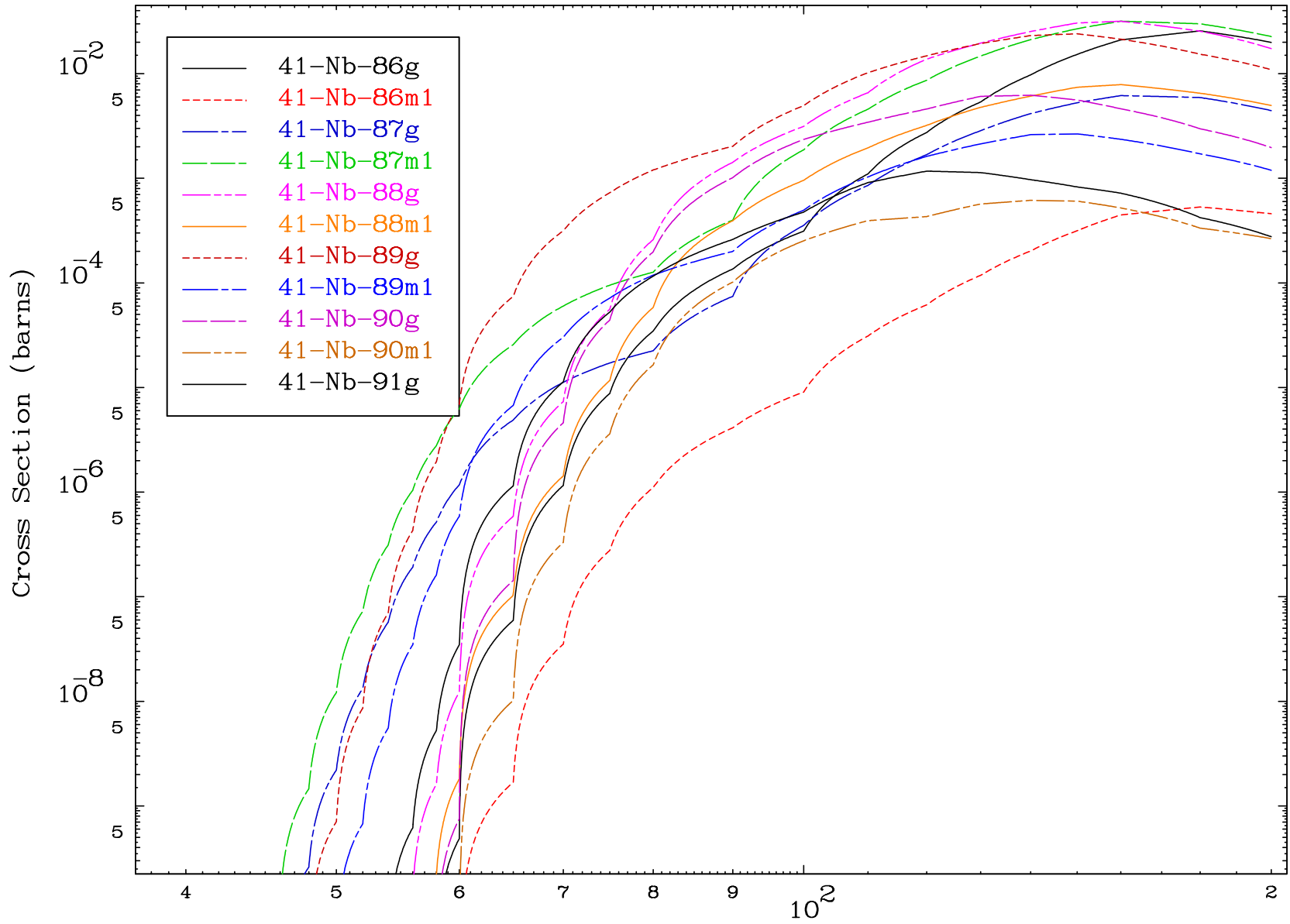
Radionuclide Production Cross Section

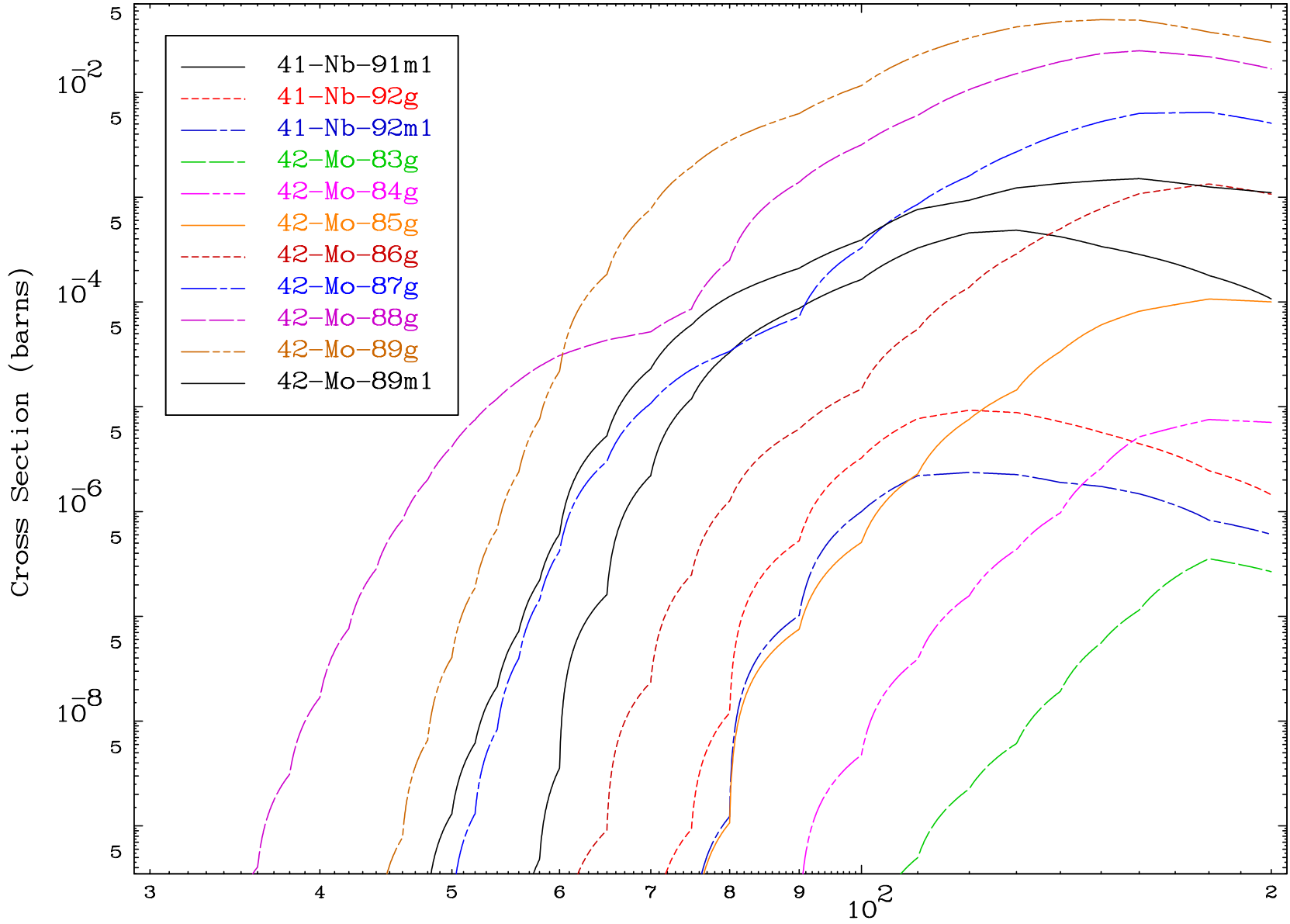




Radionuclide Production Cross Section





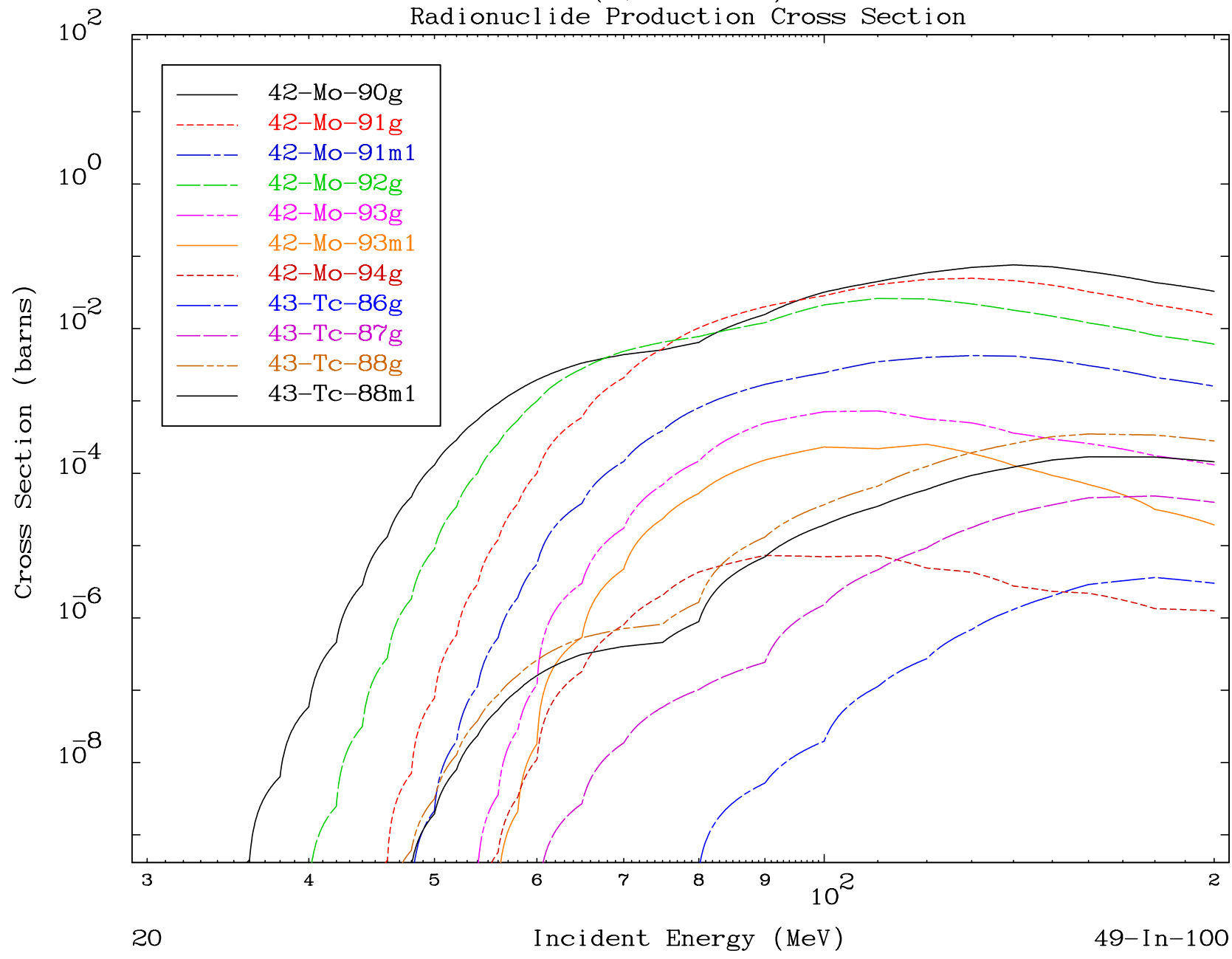


MAT 4886

(n,remainder)

49-In-100

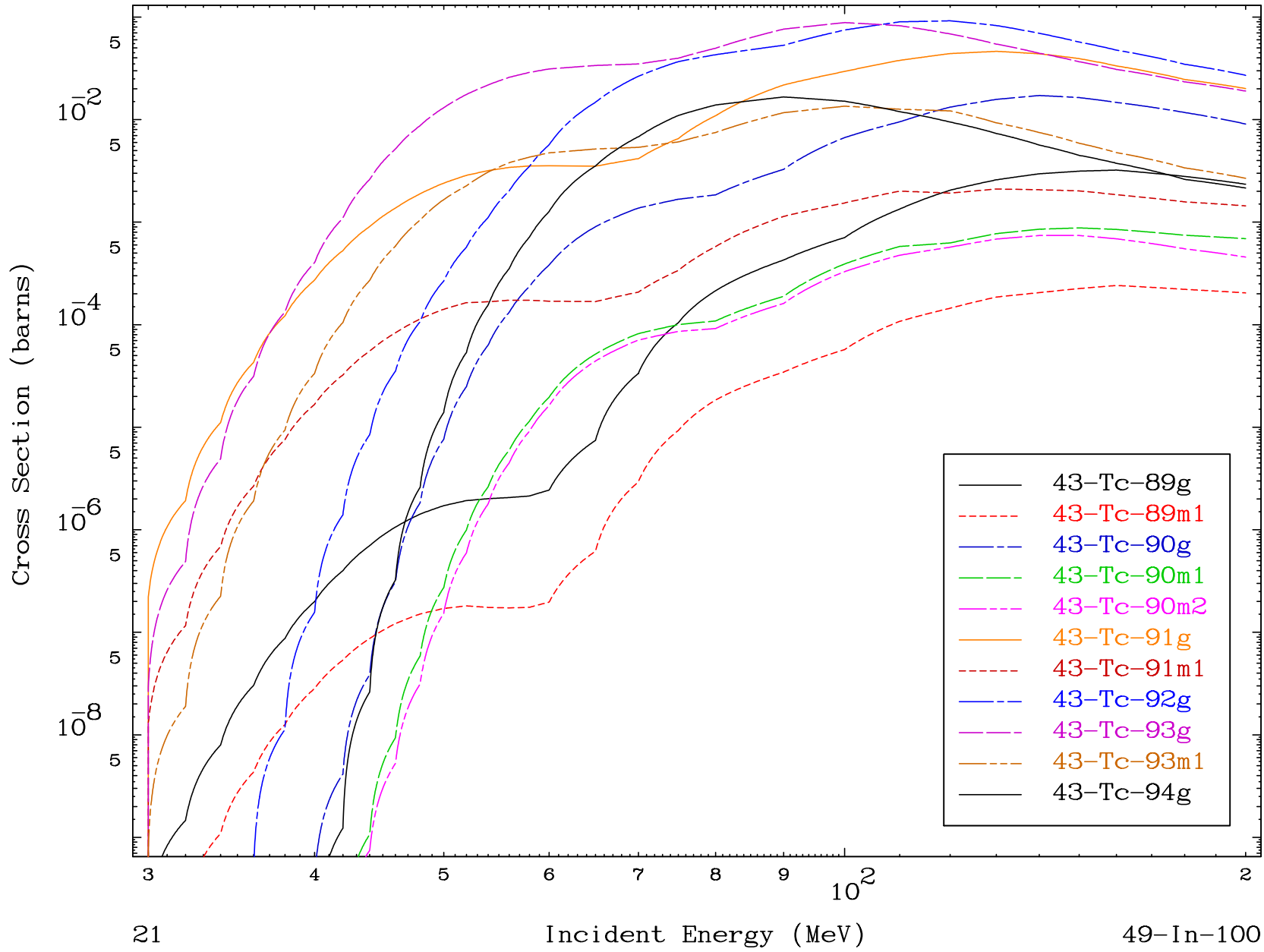
### Radionuclide Production Cross Section



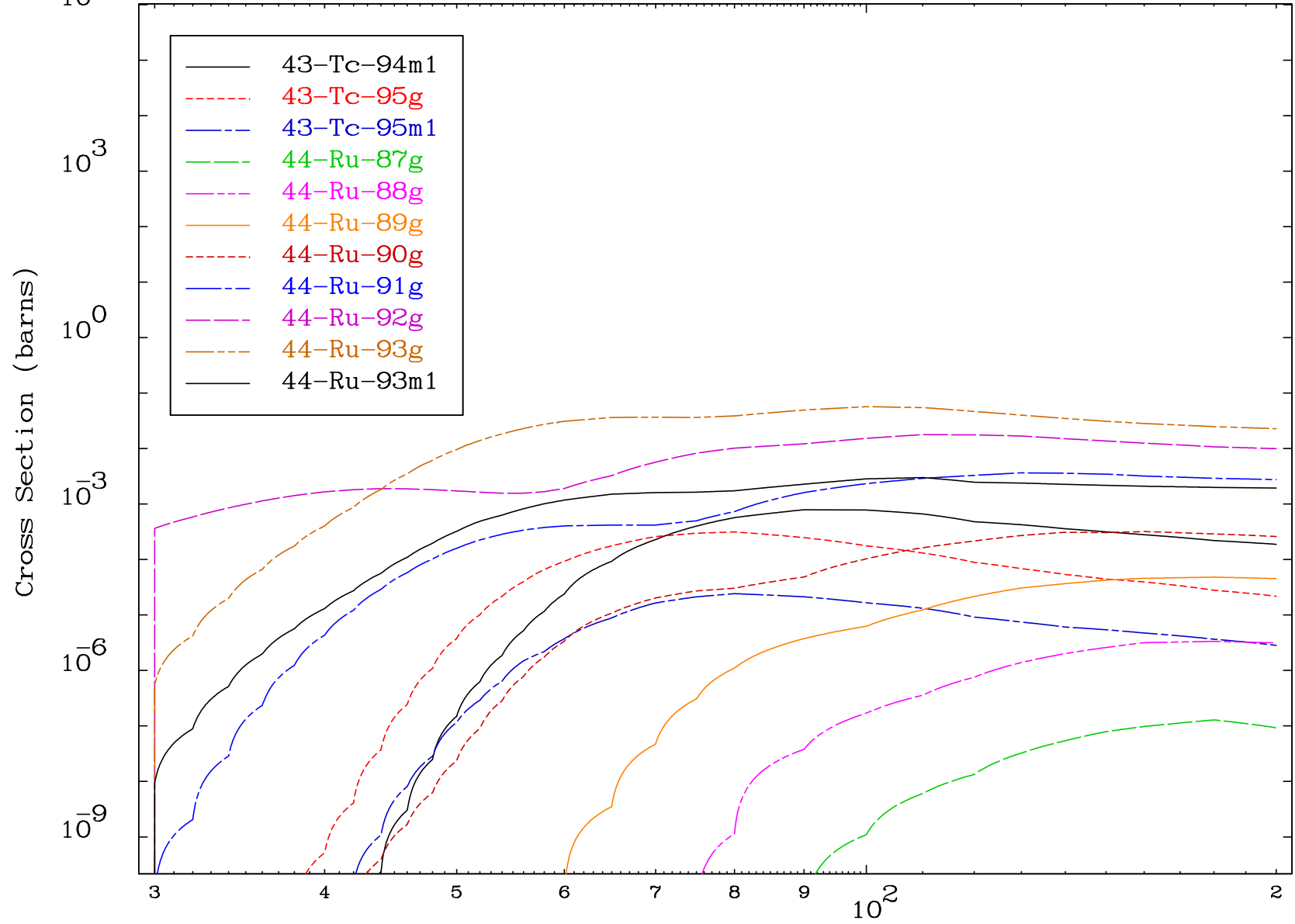
20

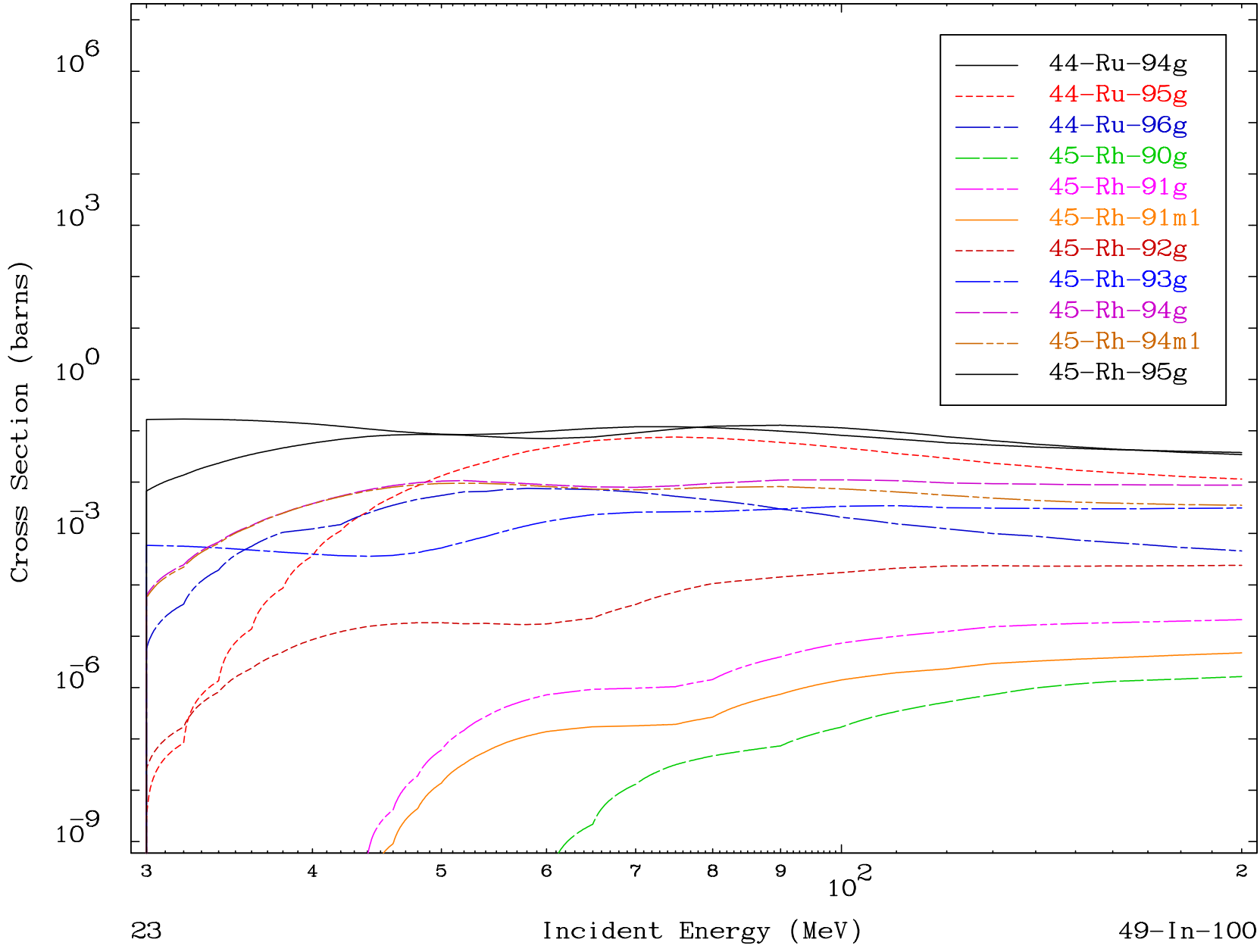
Incident Energy (MeV)

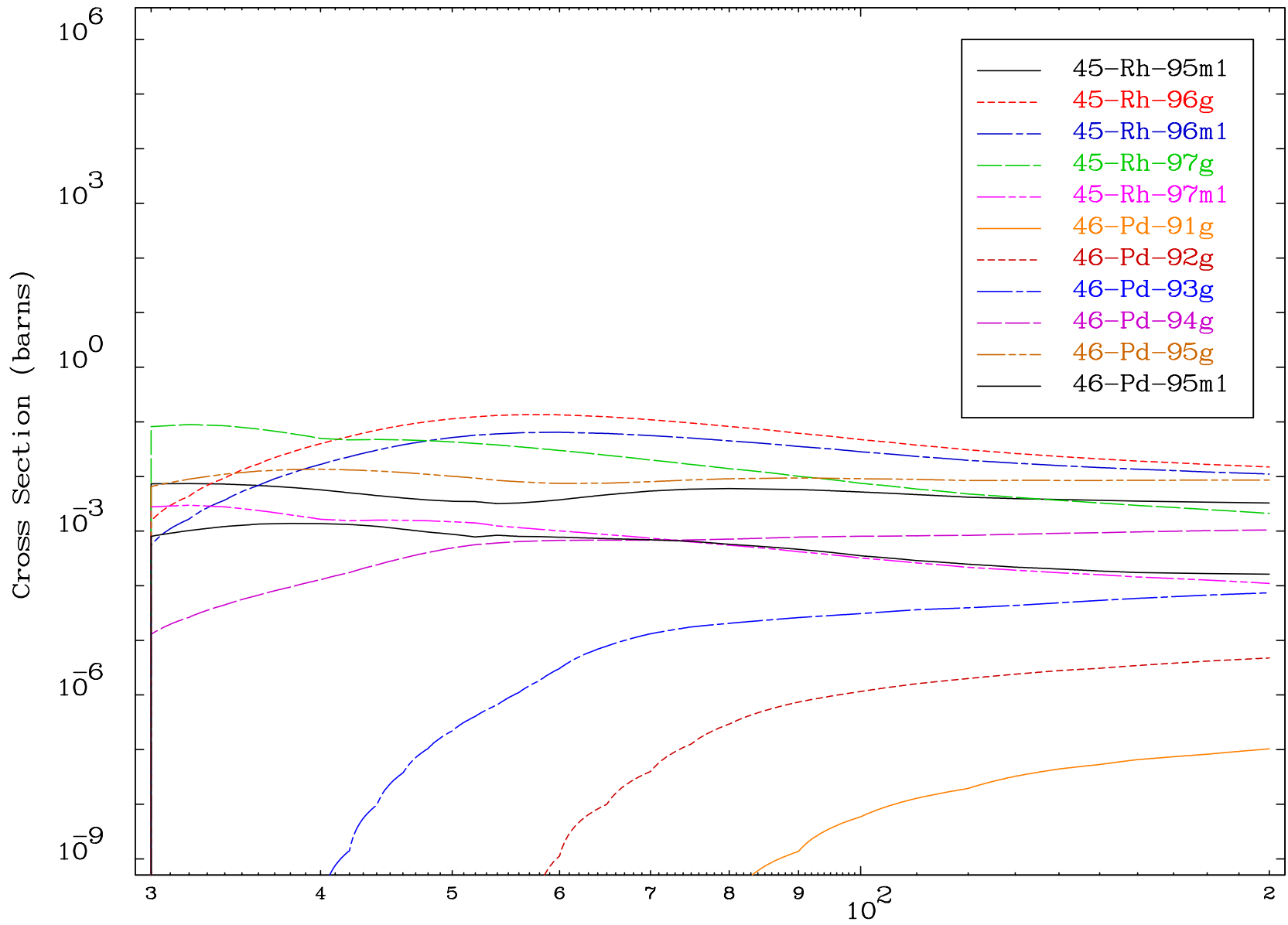
49-In-100



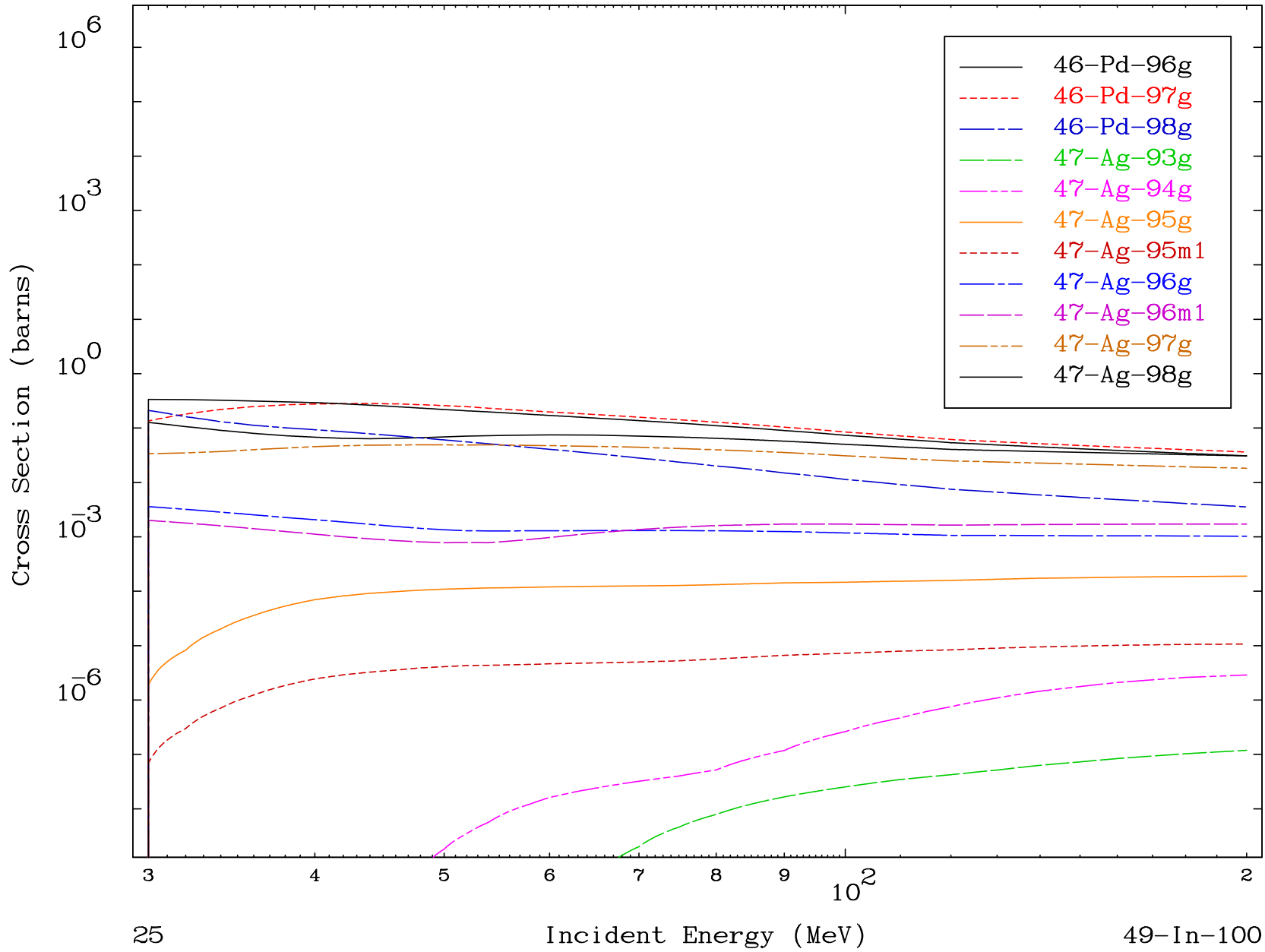
Radionuclide Production Cross Section

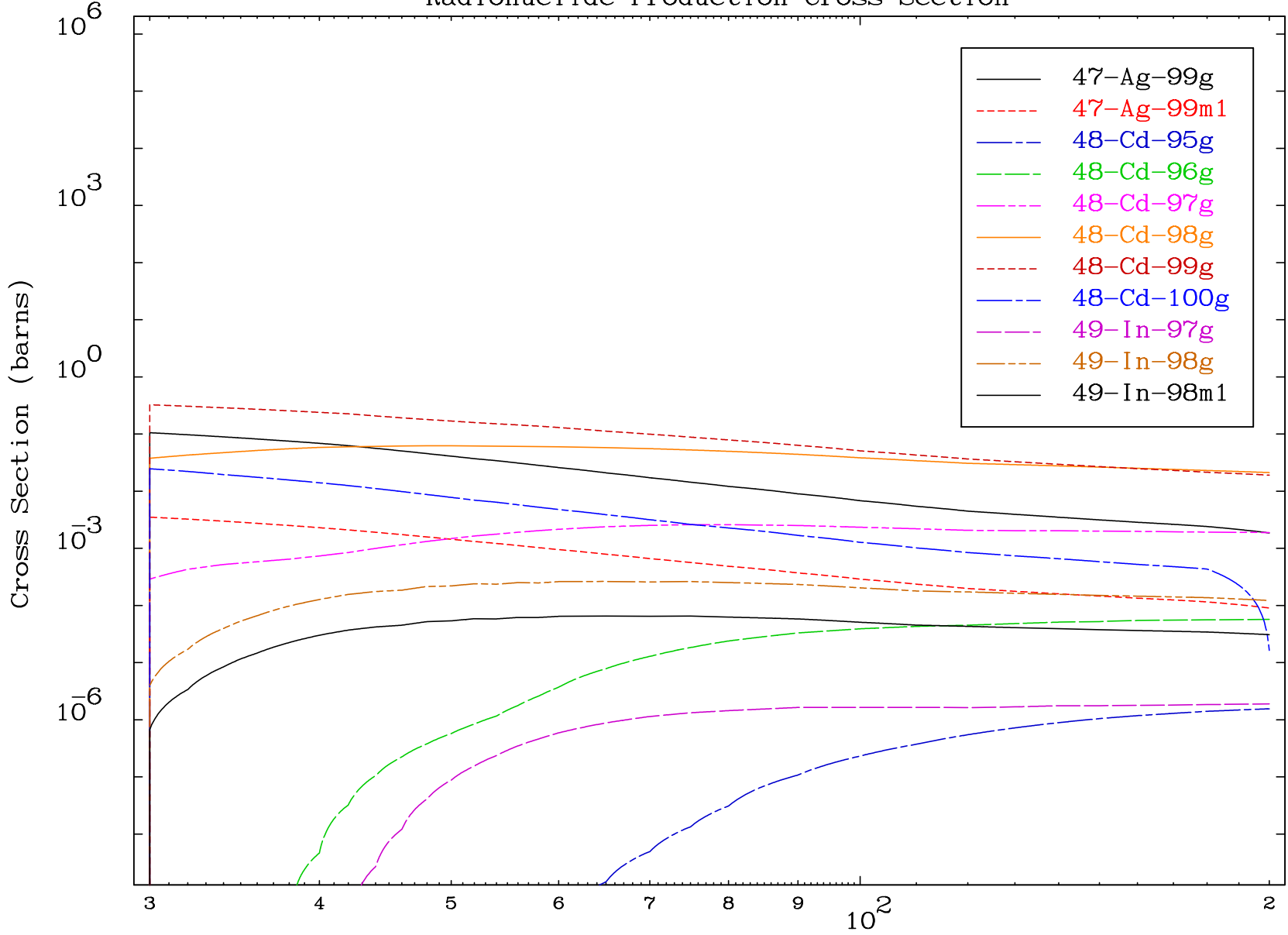


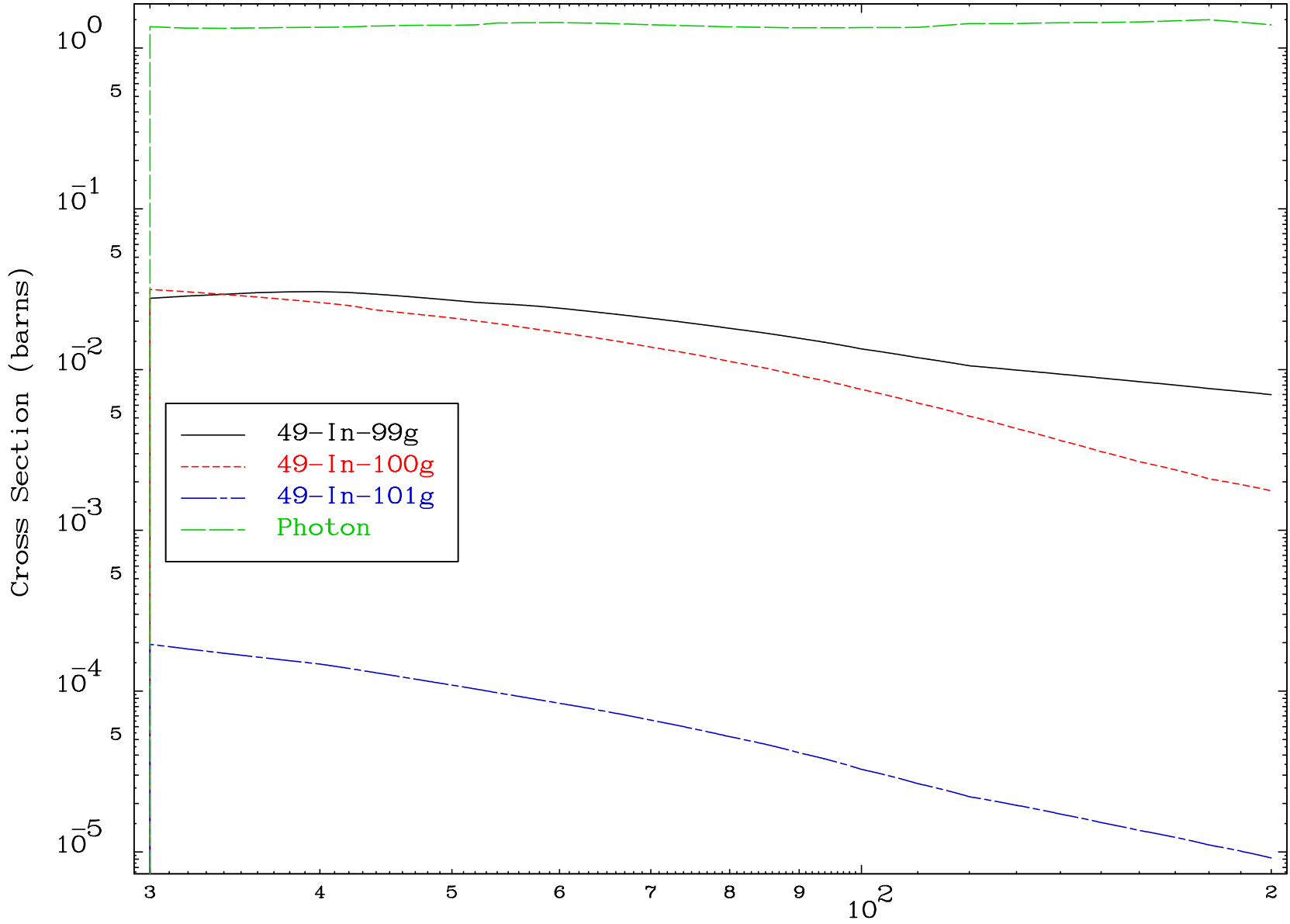




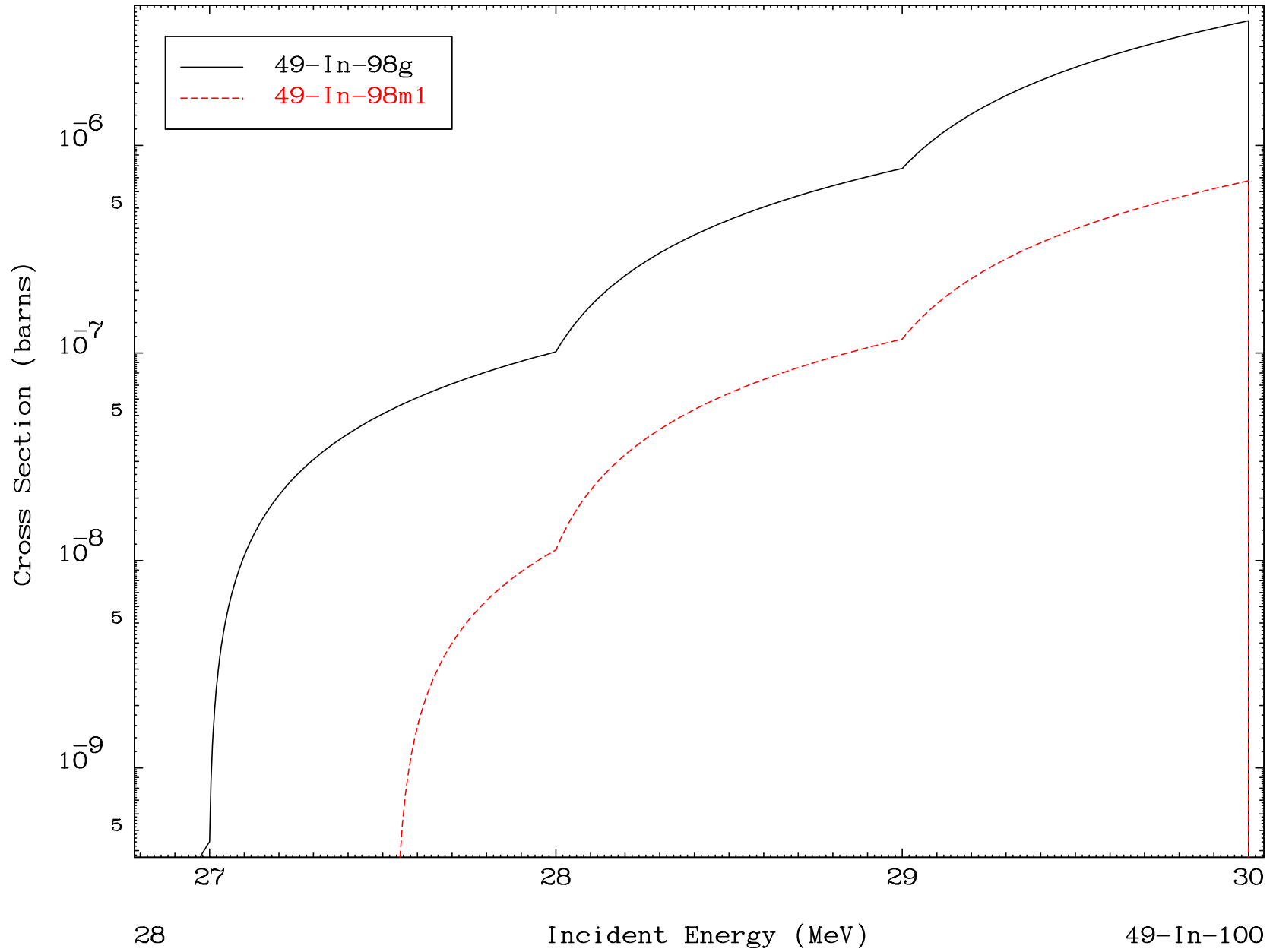




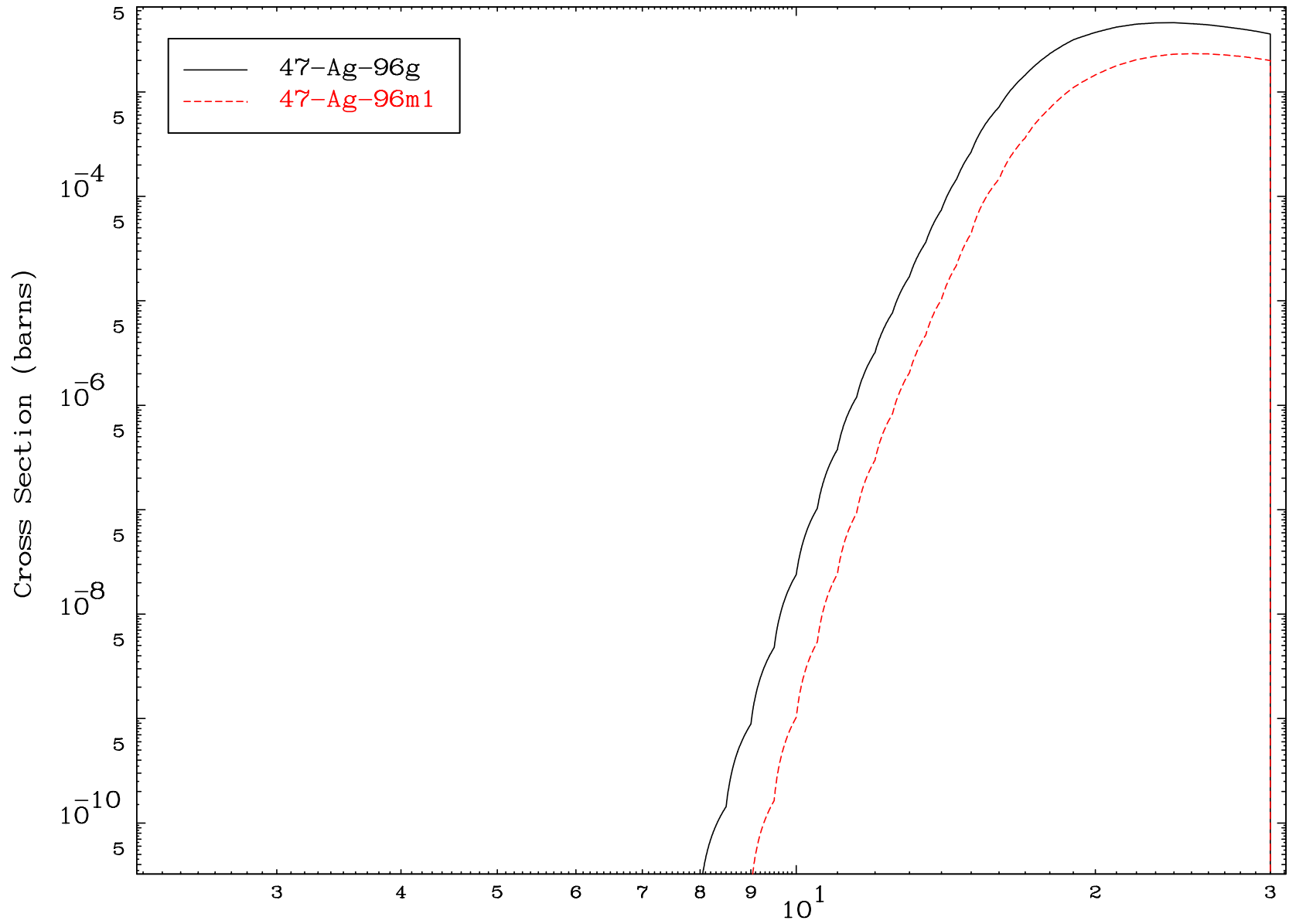




Radionuclide Production Cross Section



Radionuclide Production Cross Section

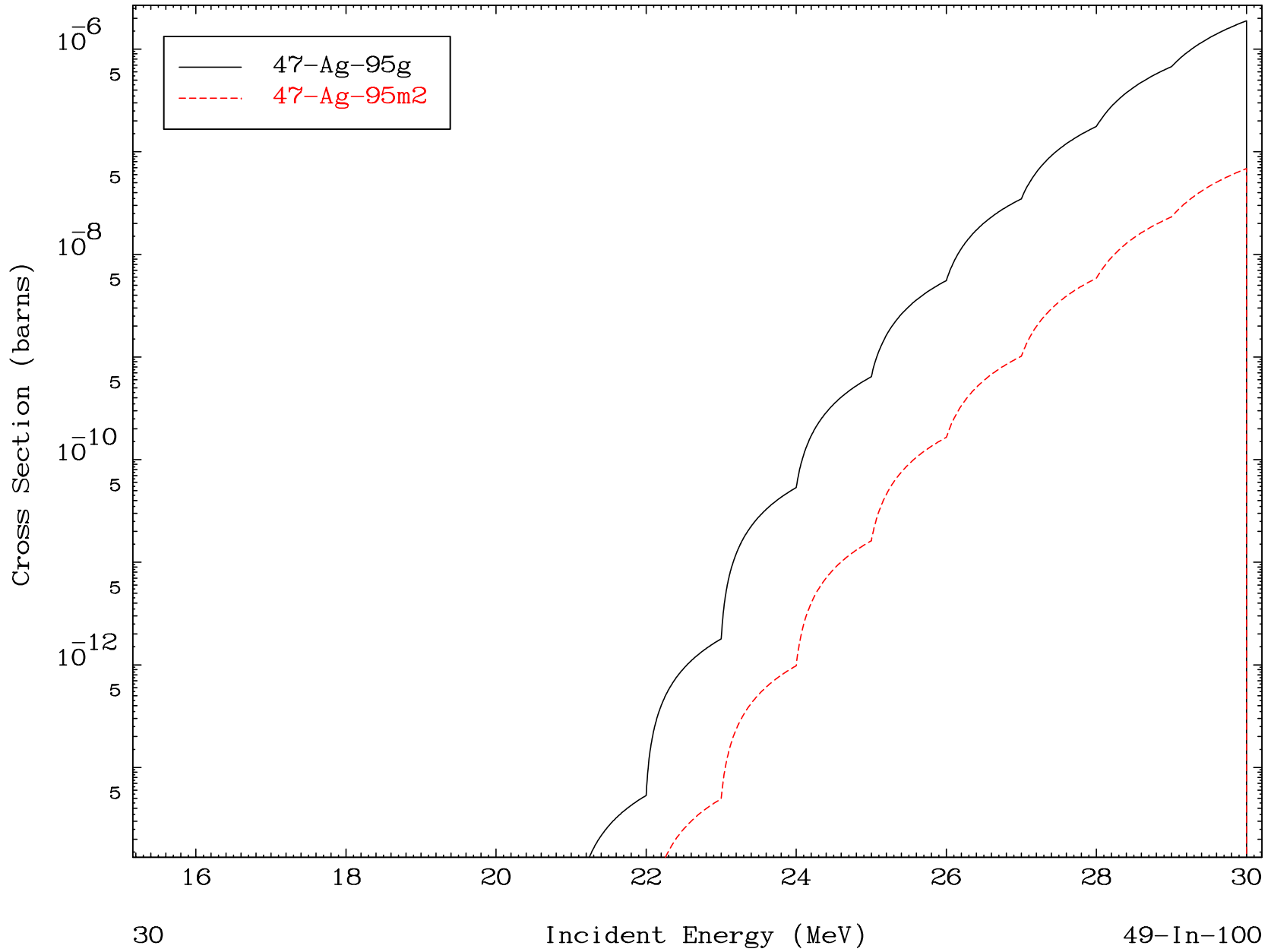


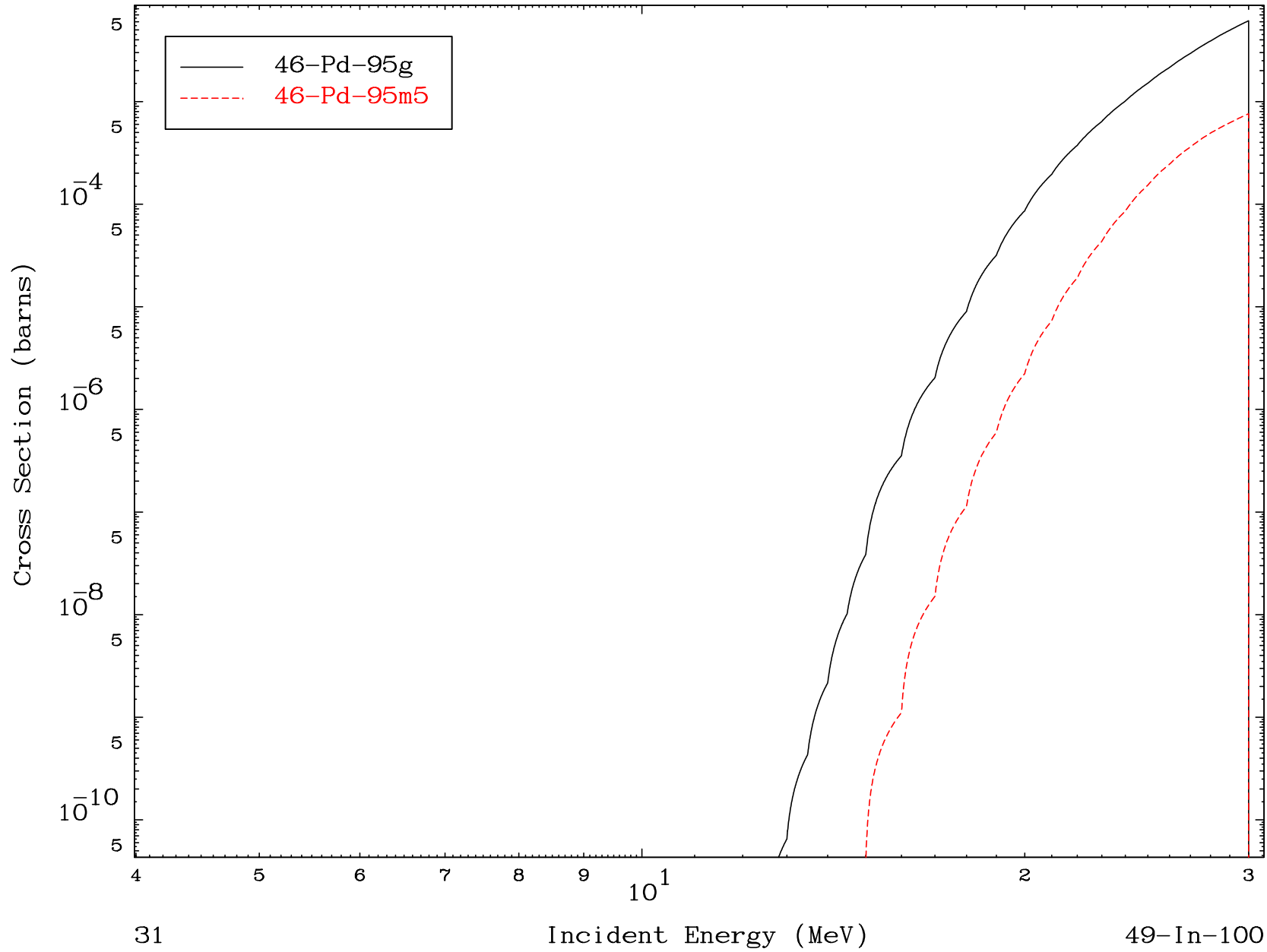
MAT 4886

(n,2n)  $\alpha$

49-In-100

Radionuclide Production Cross Section



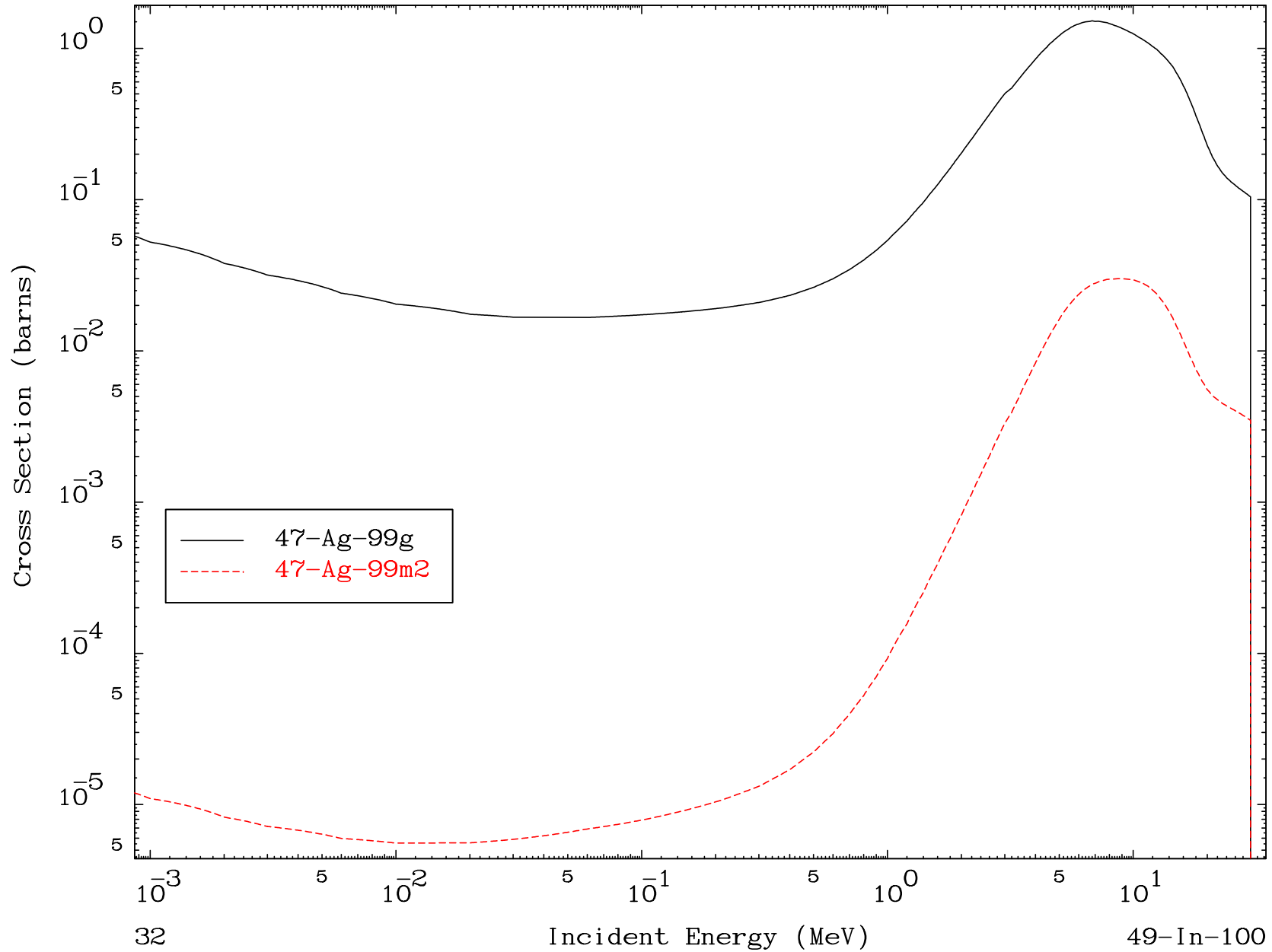


MAT 4886

(n,2p)

49-In-100

Radionuclide Production Cross Section



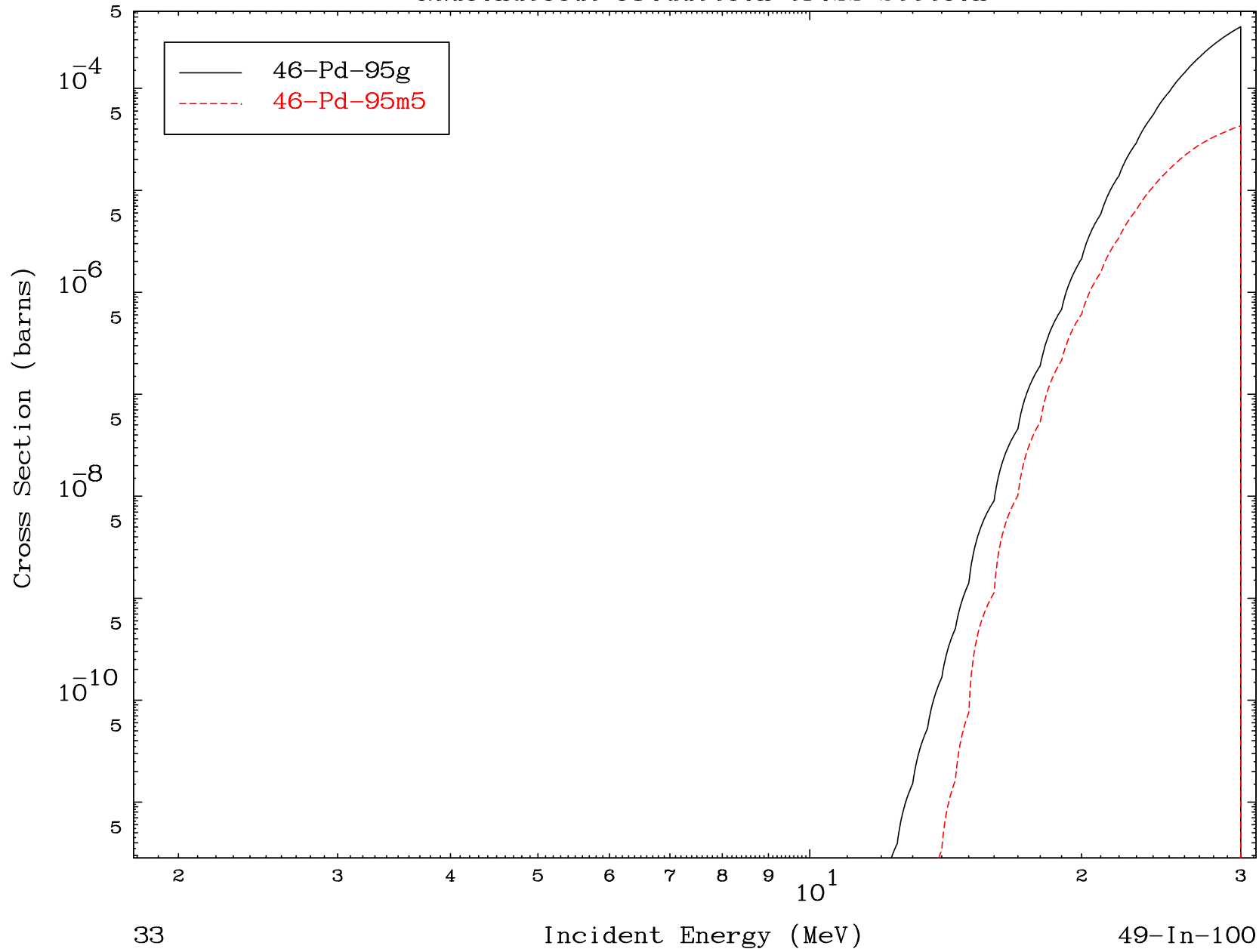


MAT 4886

(n,d)  $\alpha$

49-In-100

Radionuclide Production Cross Section



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Incident Energy (MeV)

49-In-100