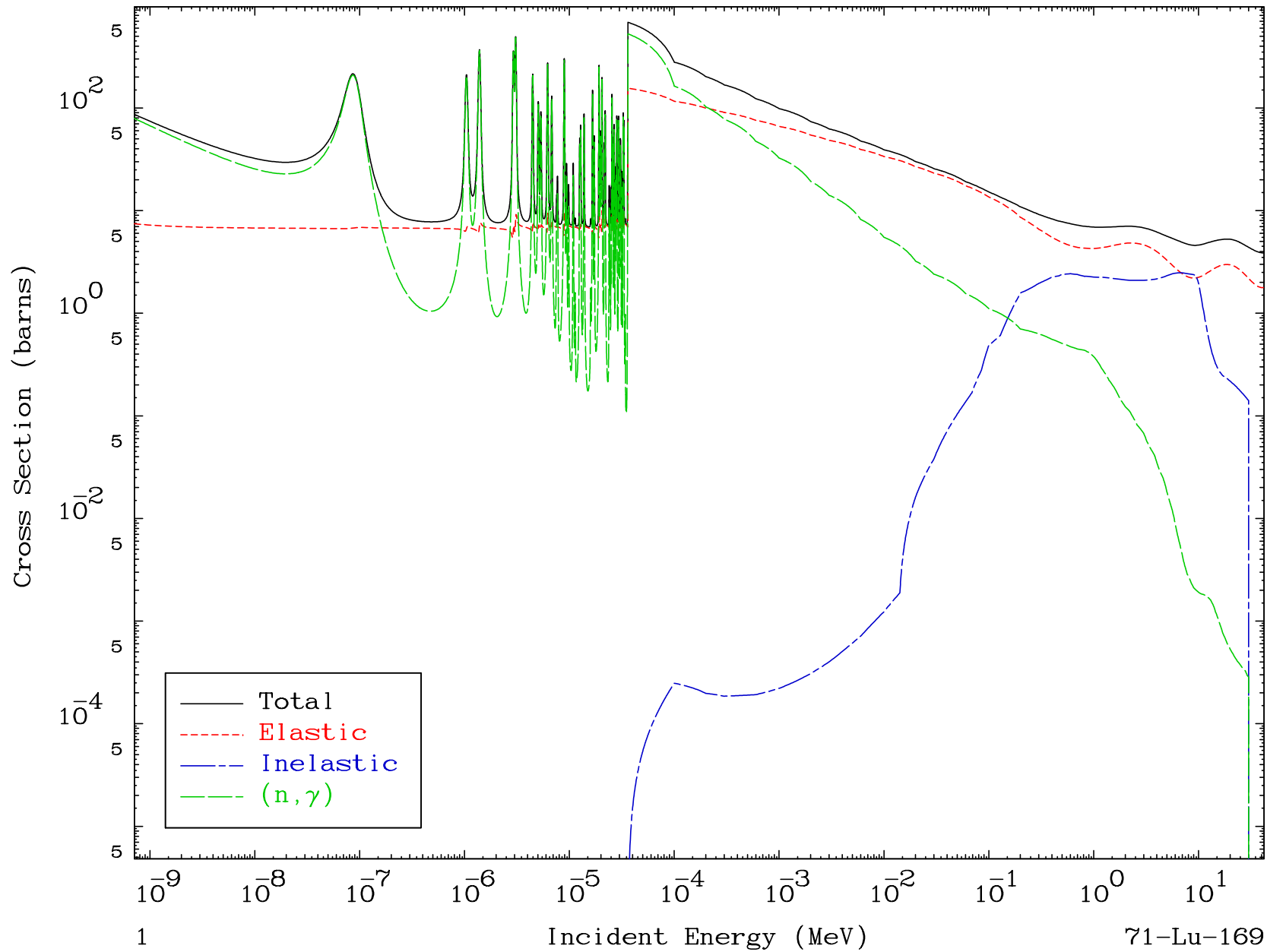
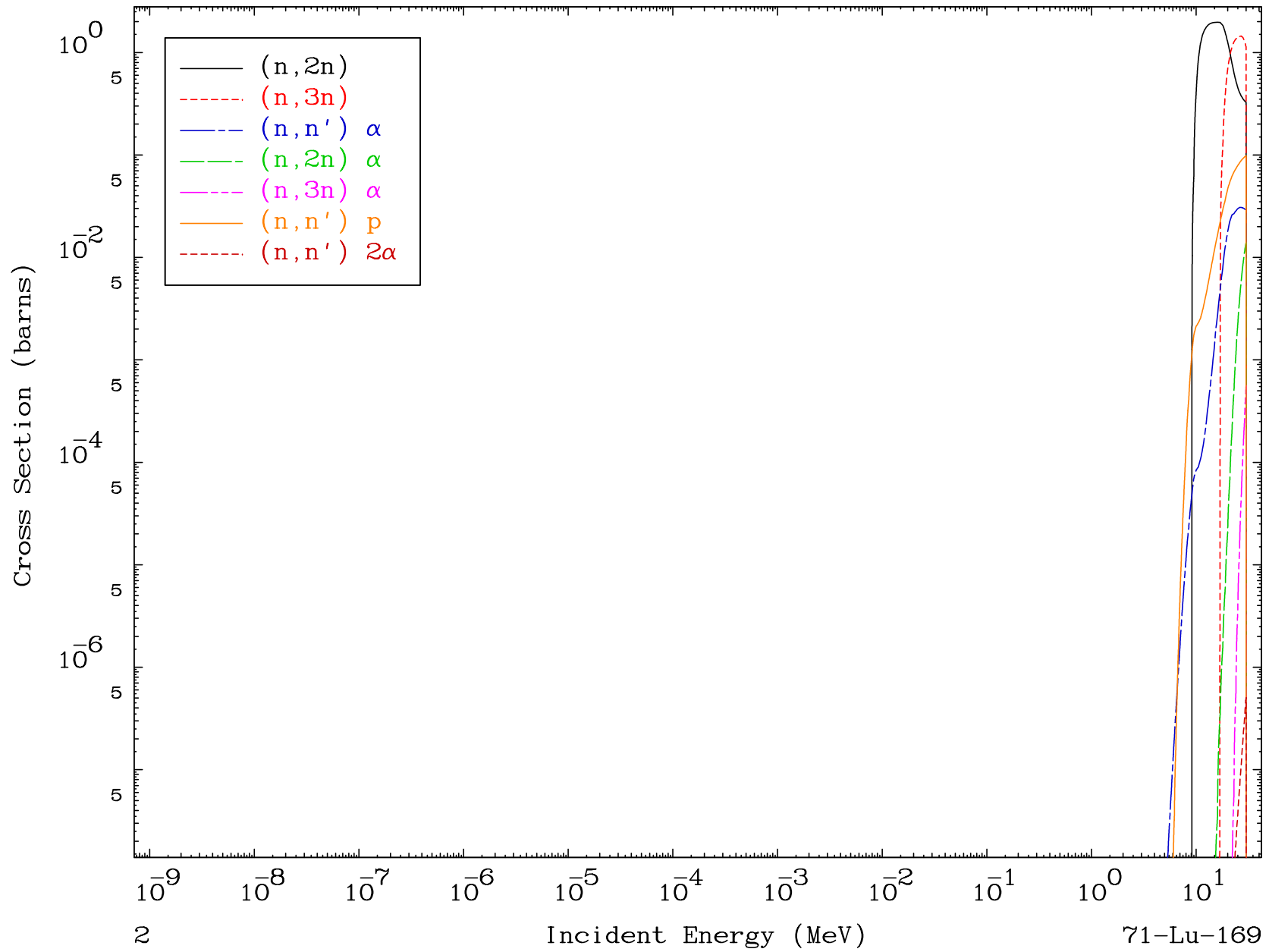


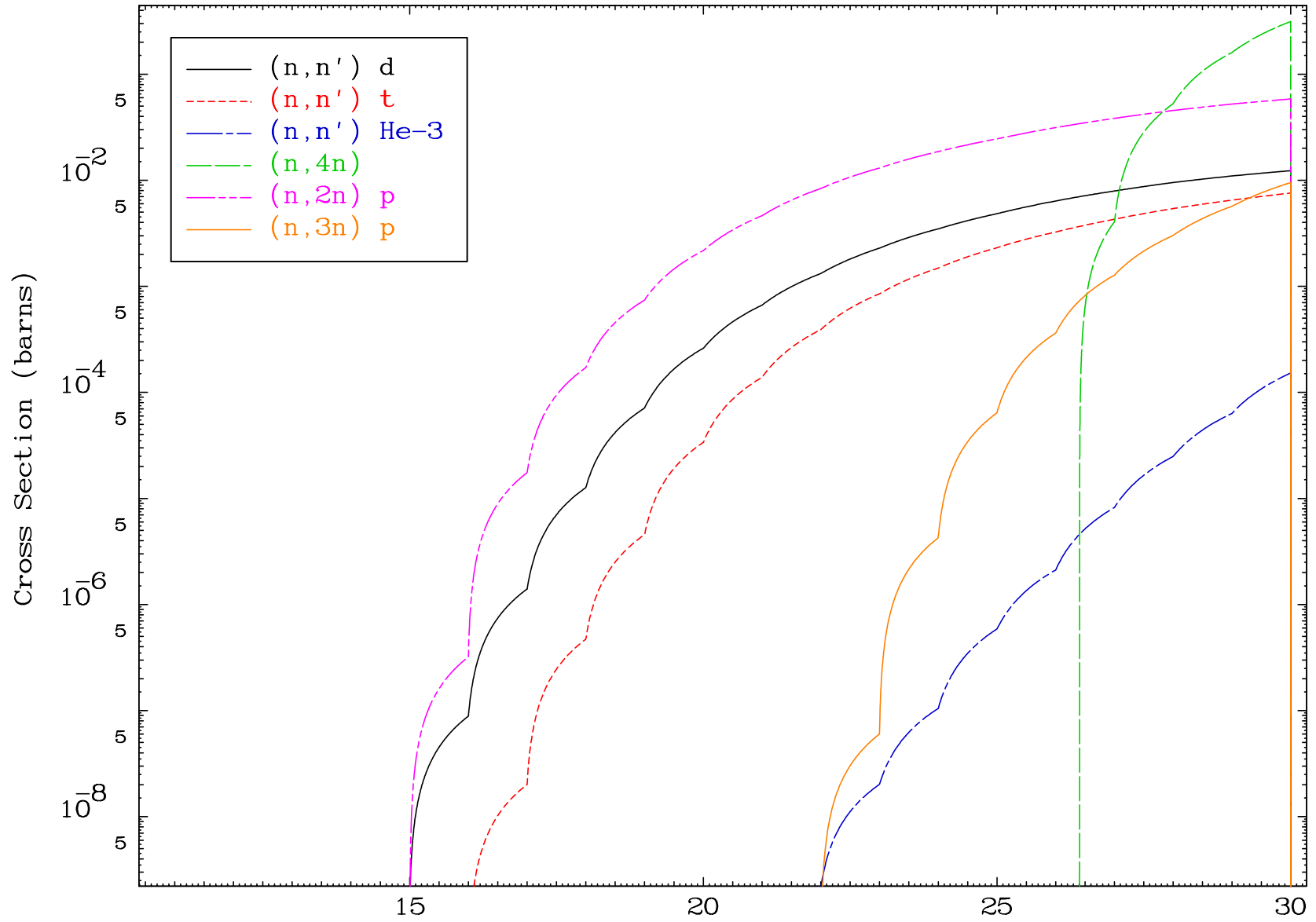
MAT 7108

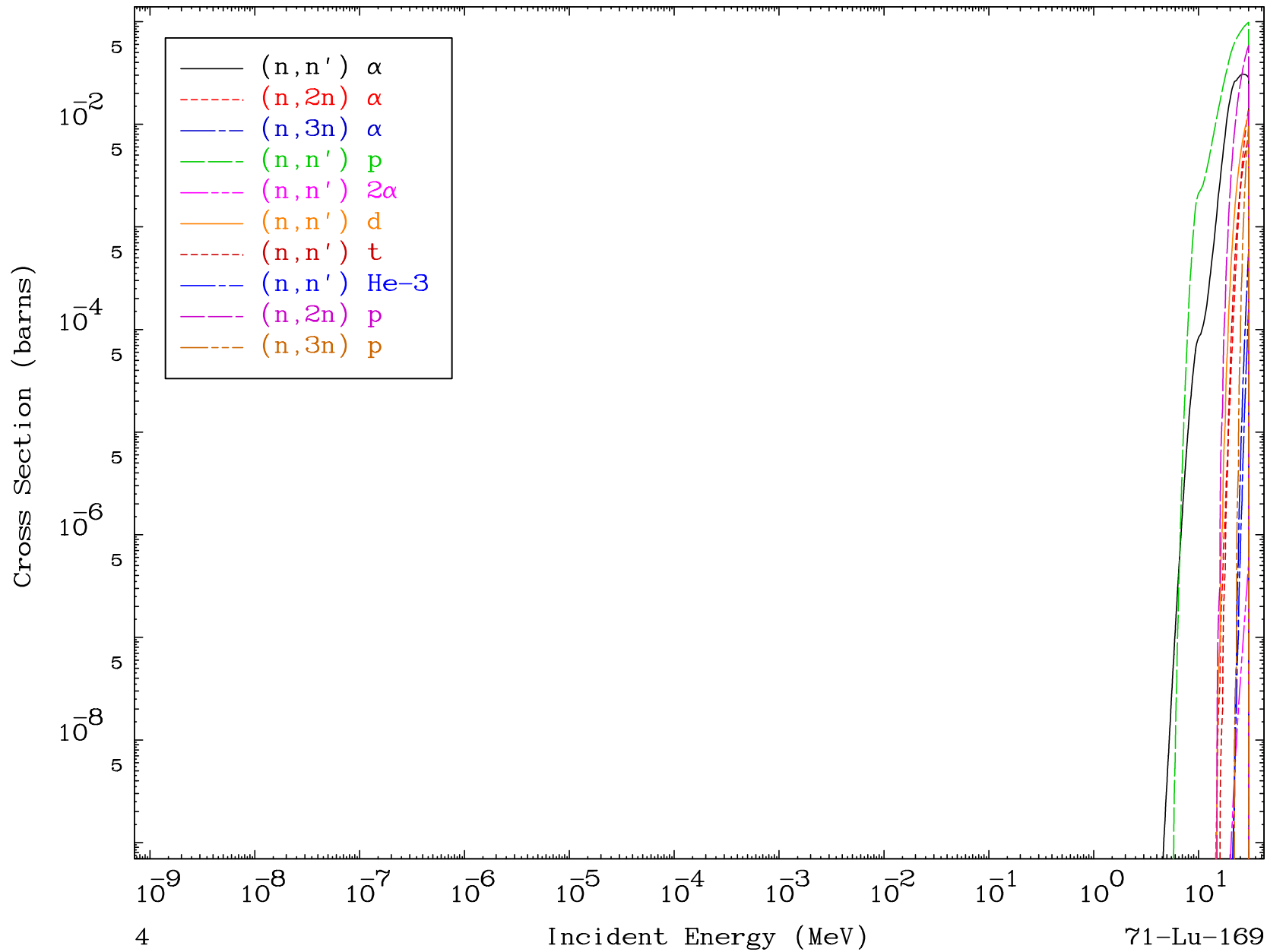
Major  
294 Kelvin Cross Sections

71-Lu-169

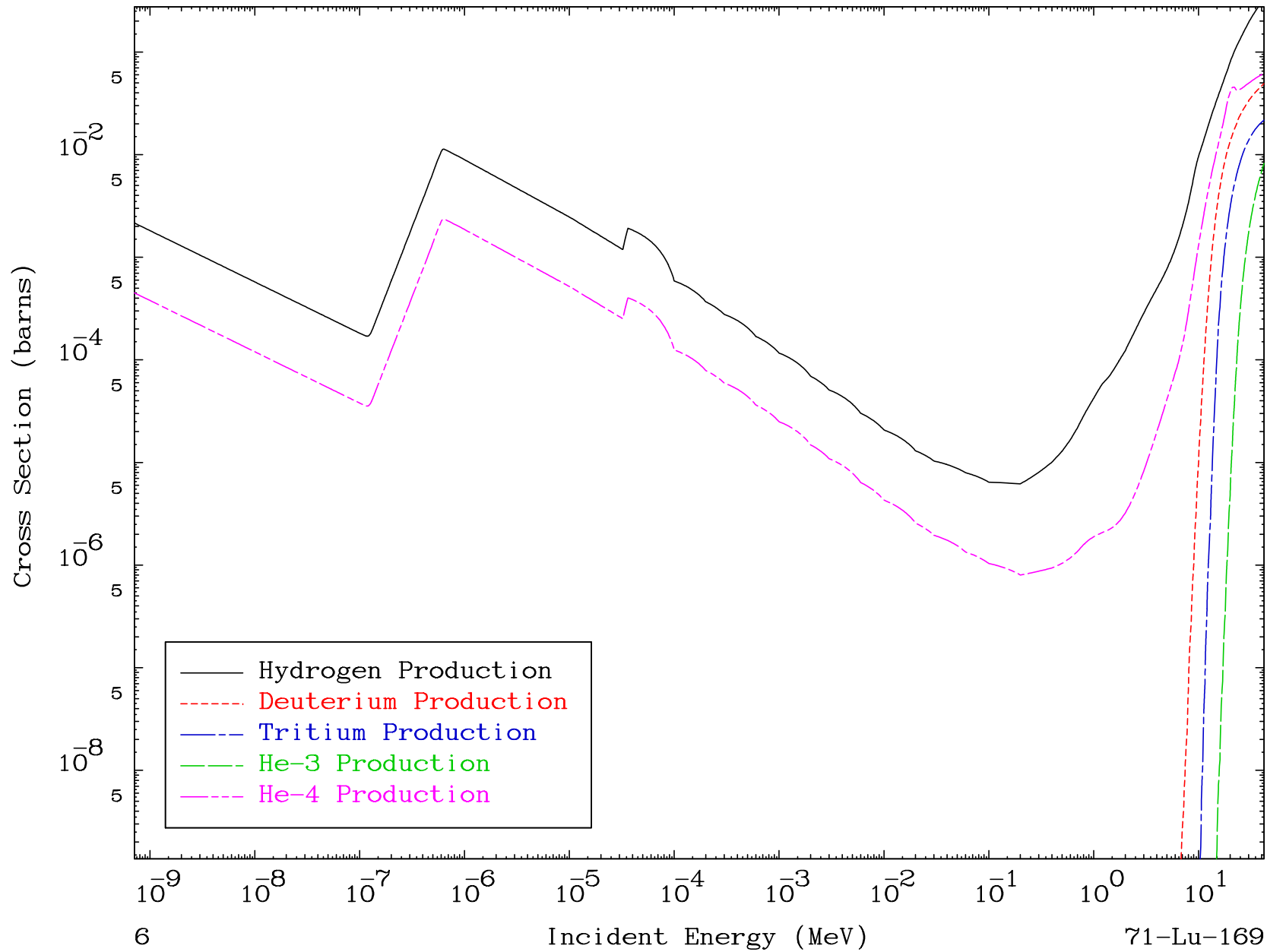


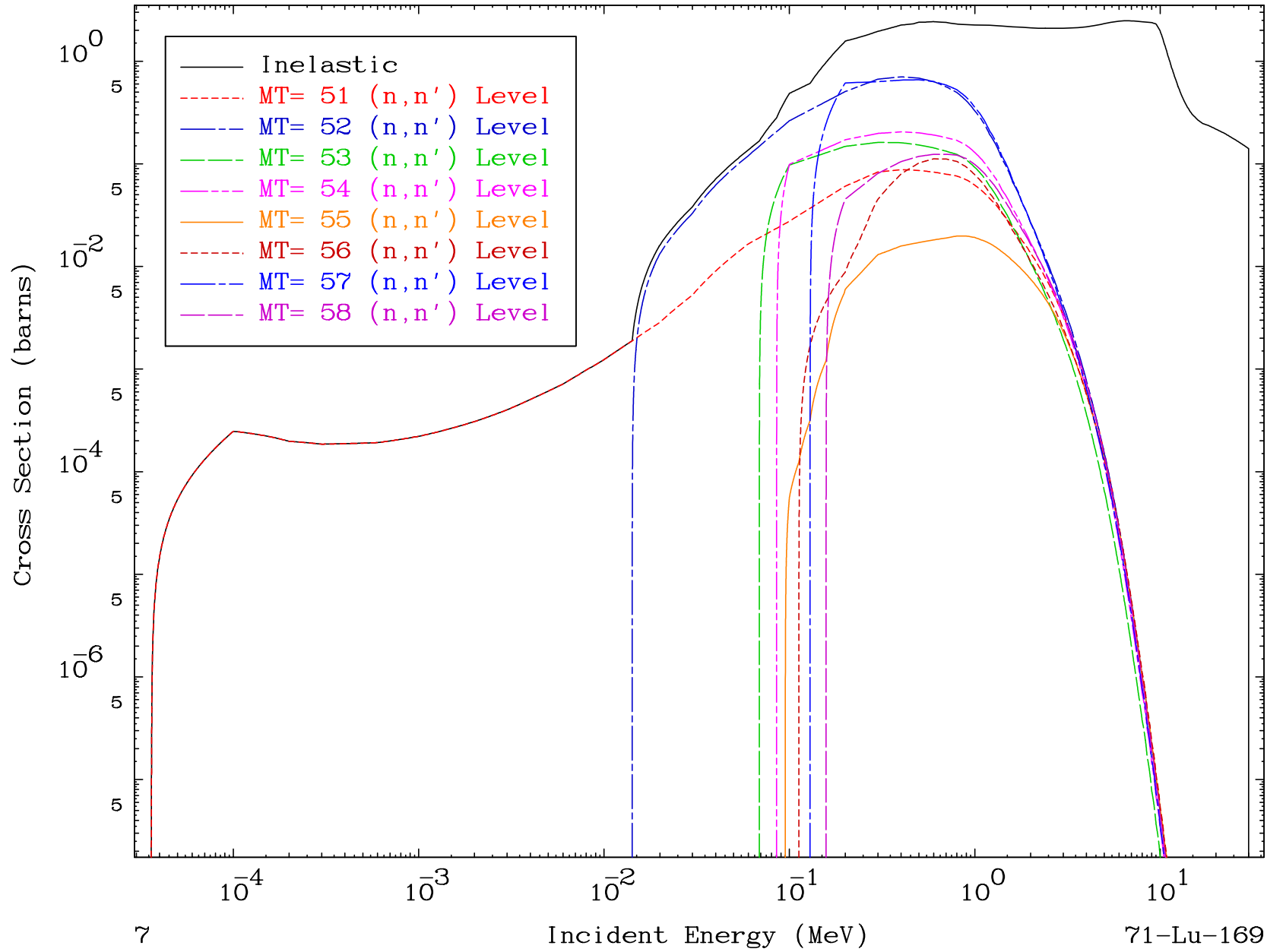


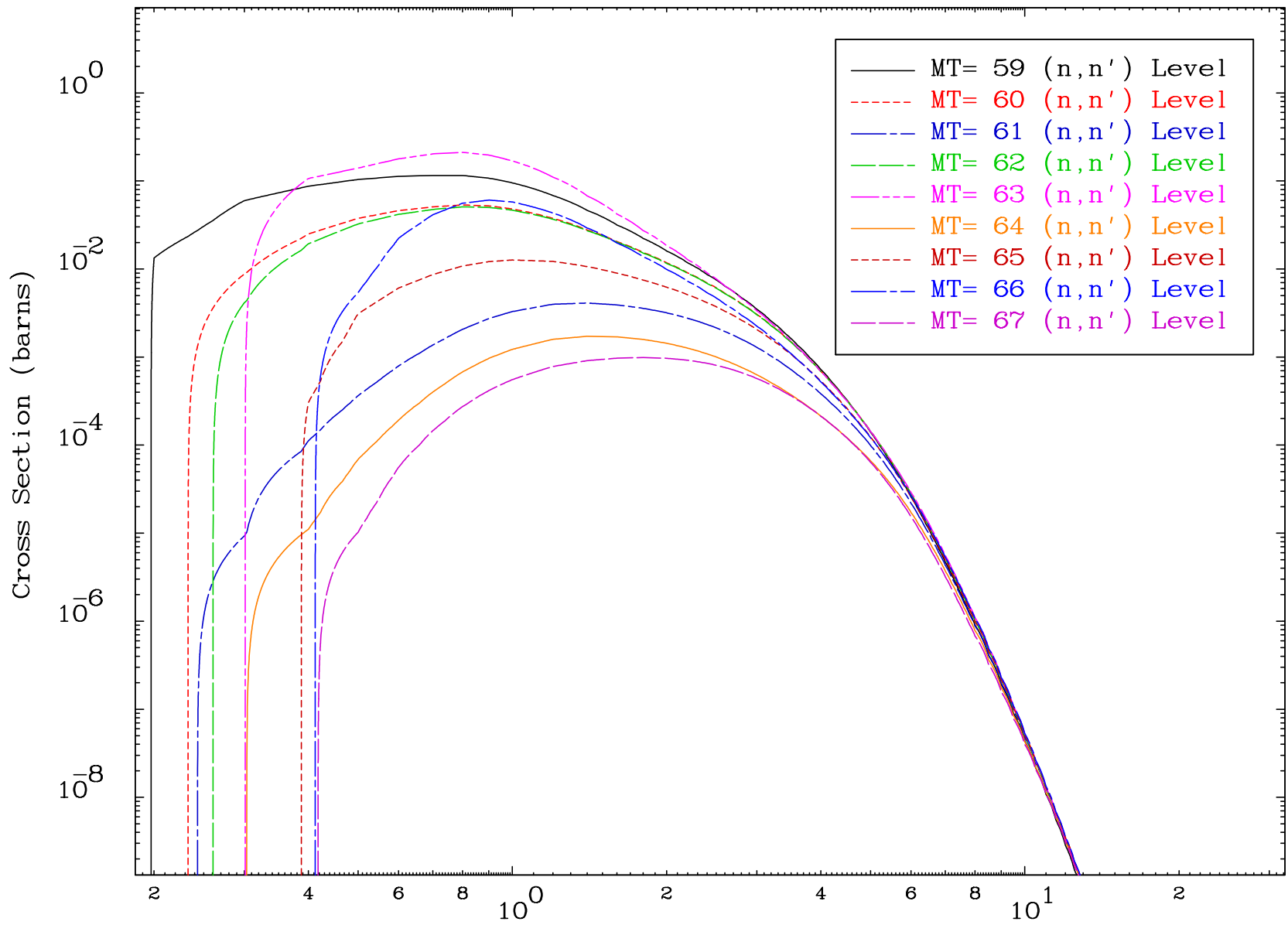




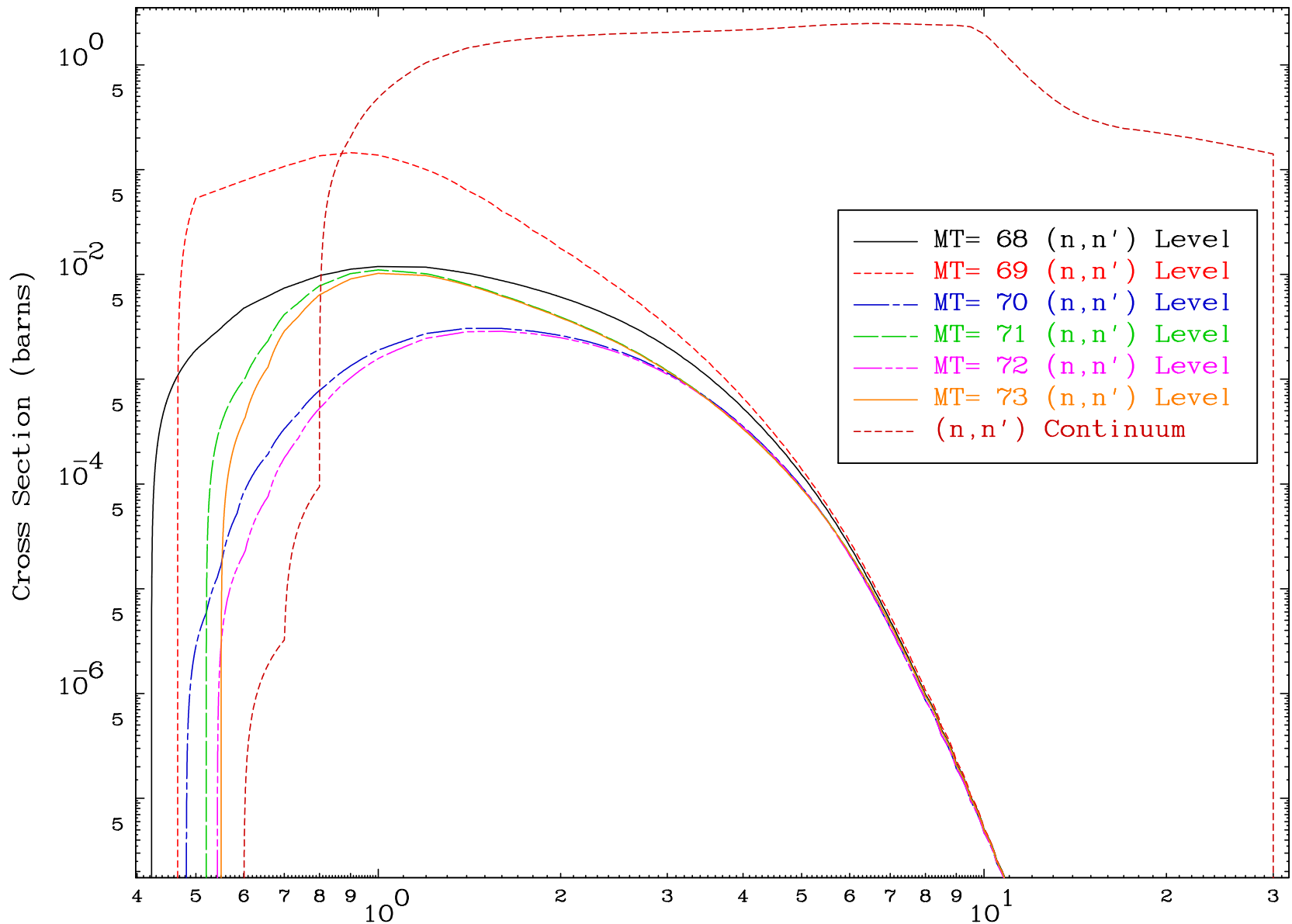








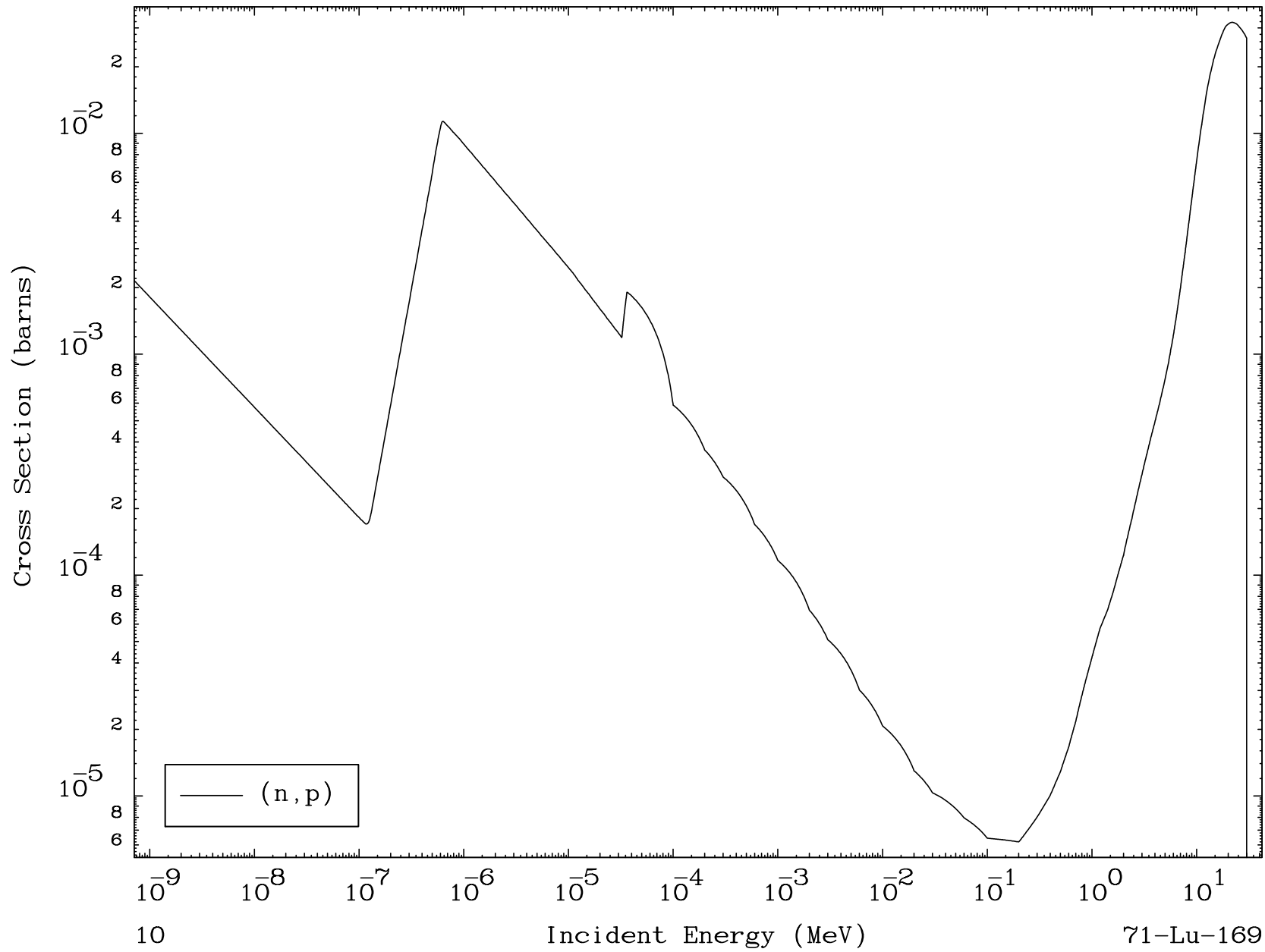


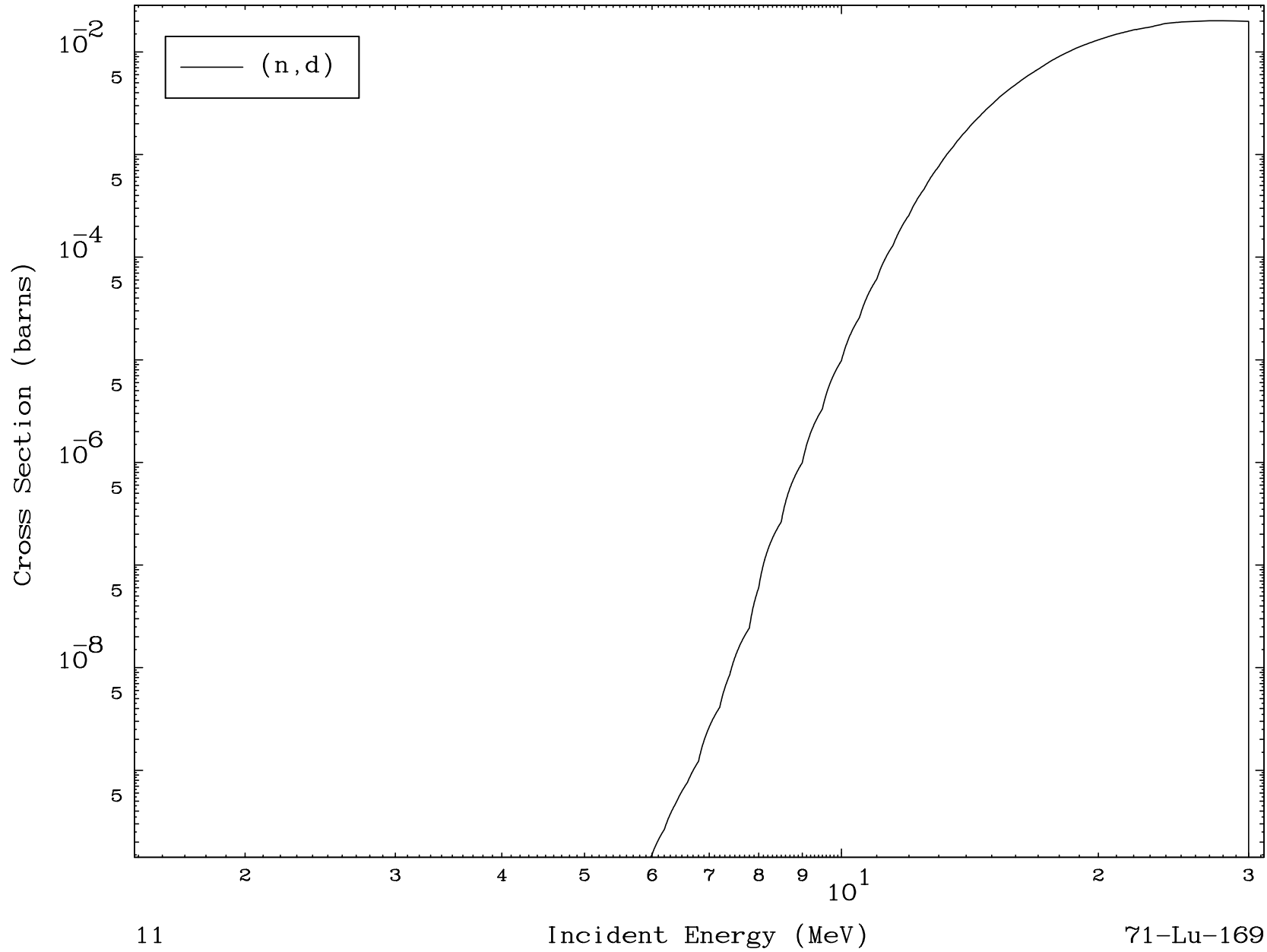


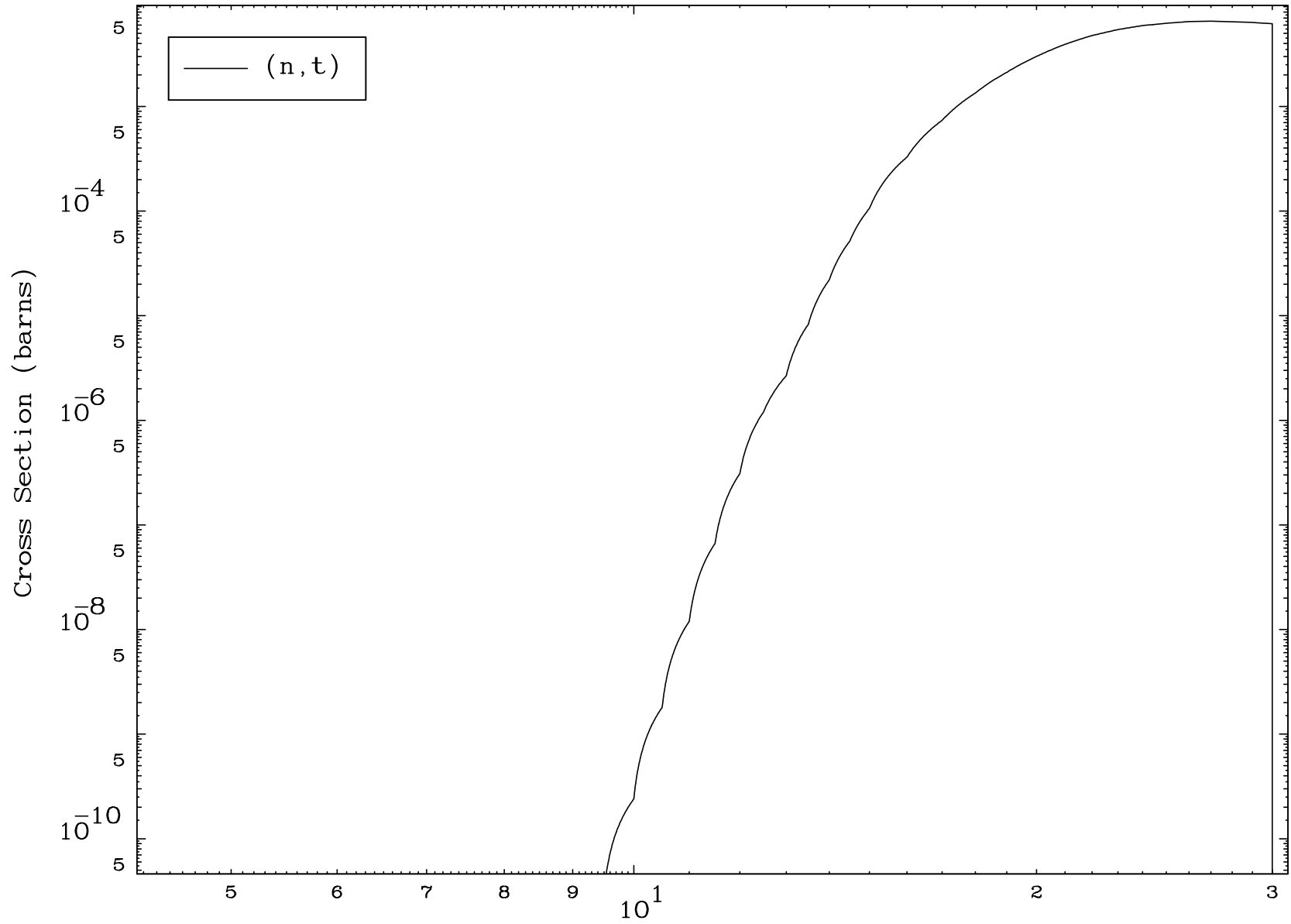
MAT 7108

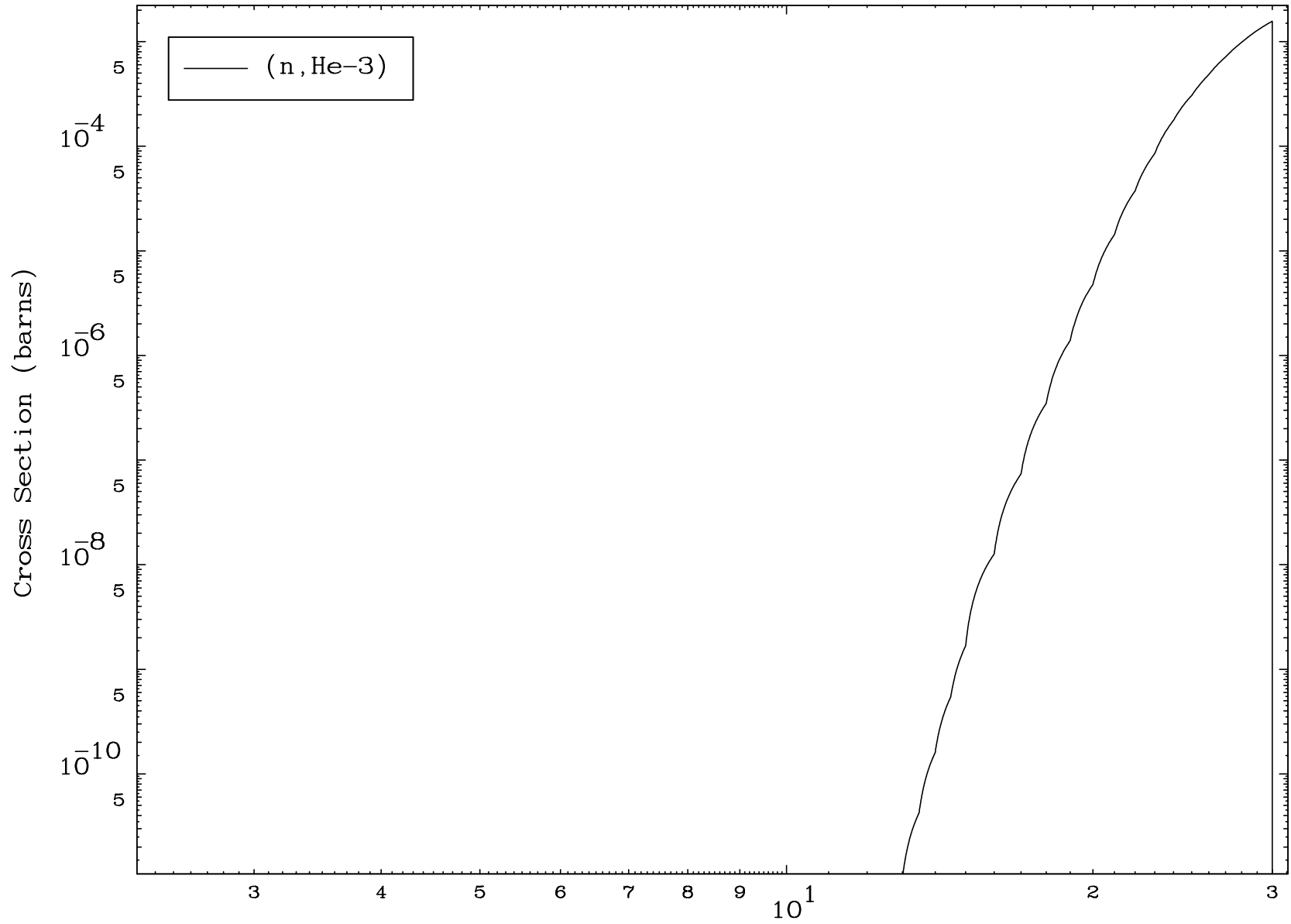
(n,p) Levels  
294 Kelvin Cross Sections

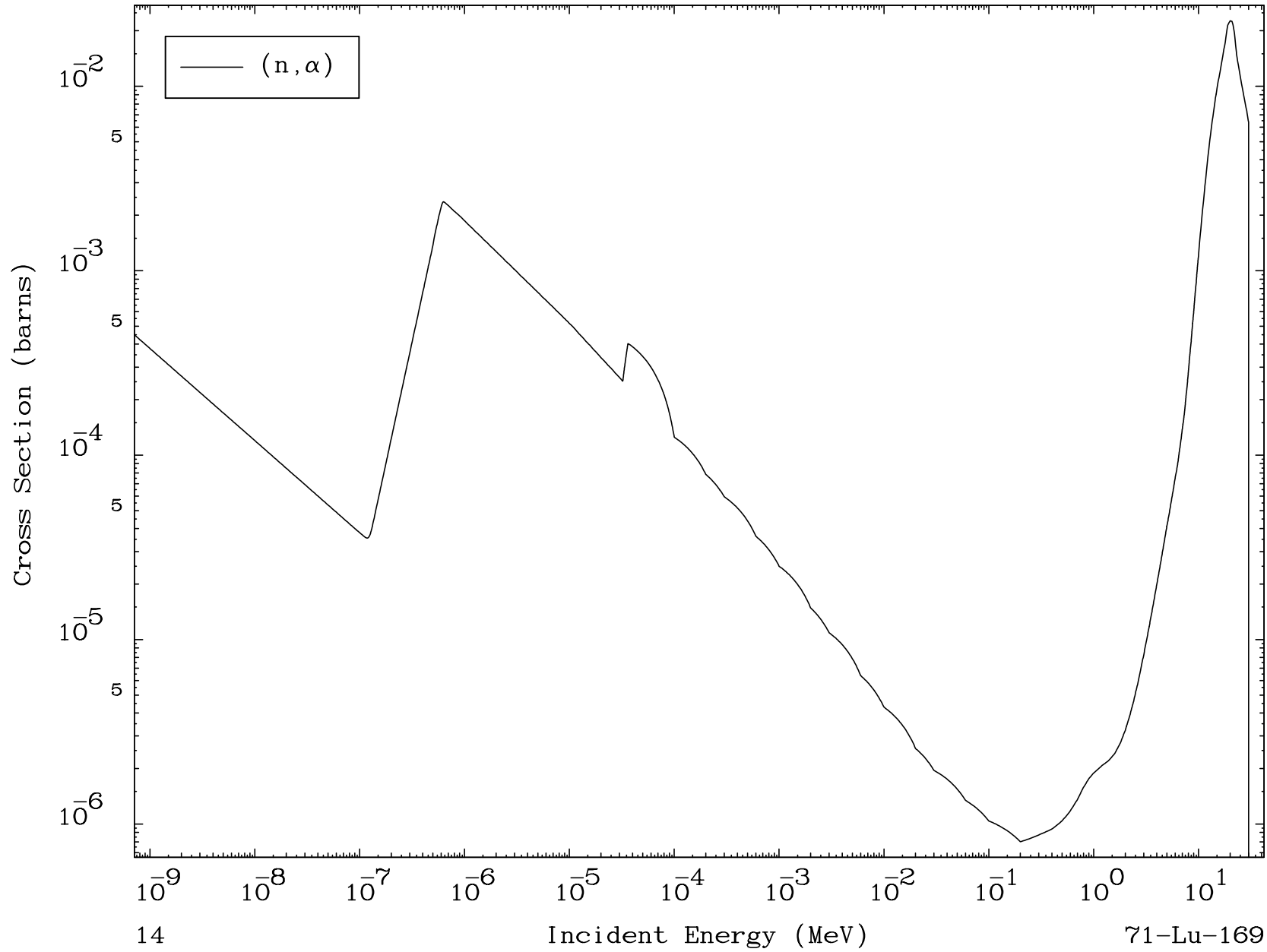
71-Lu-169

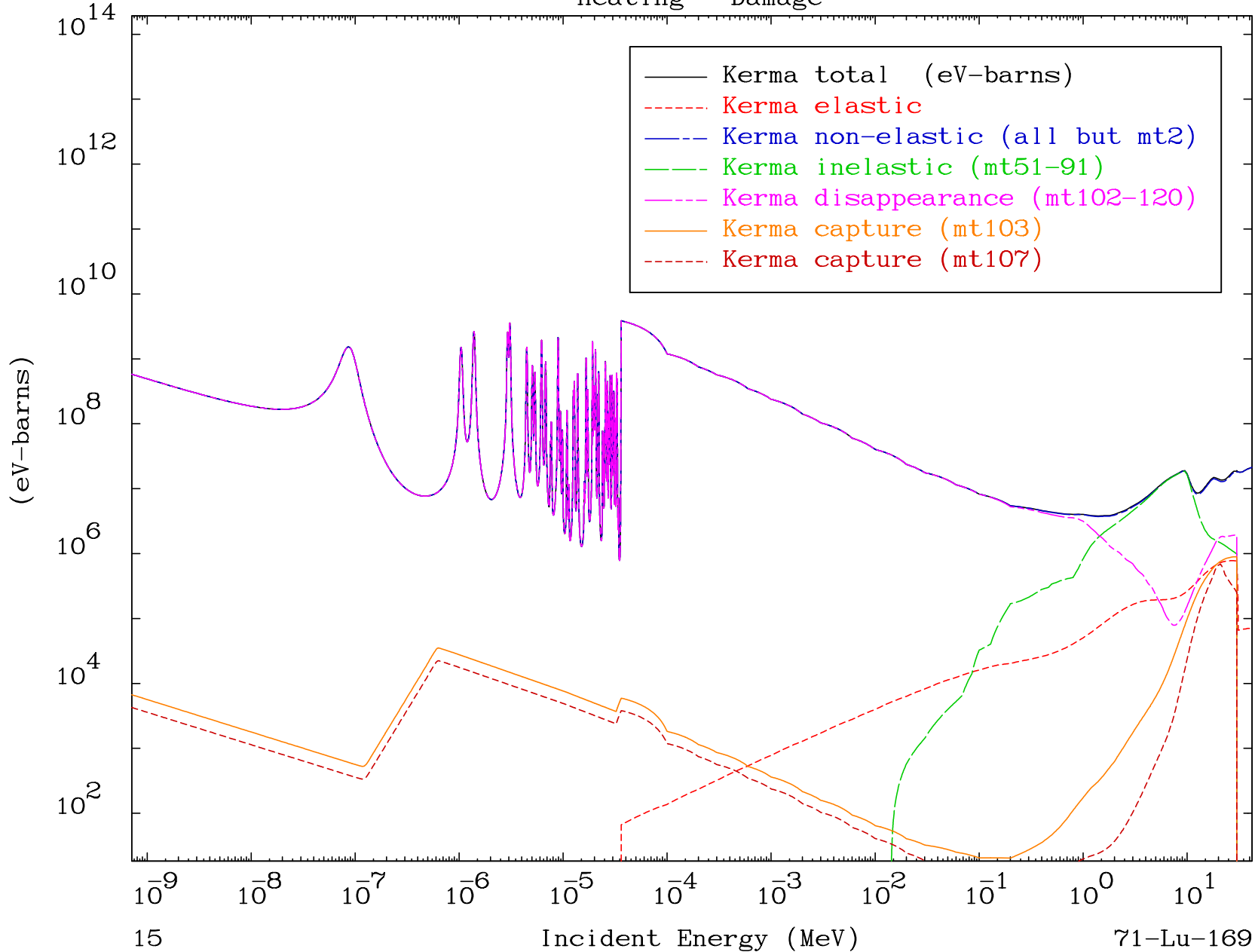


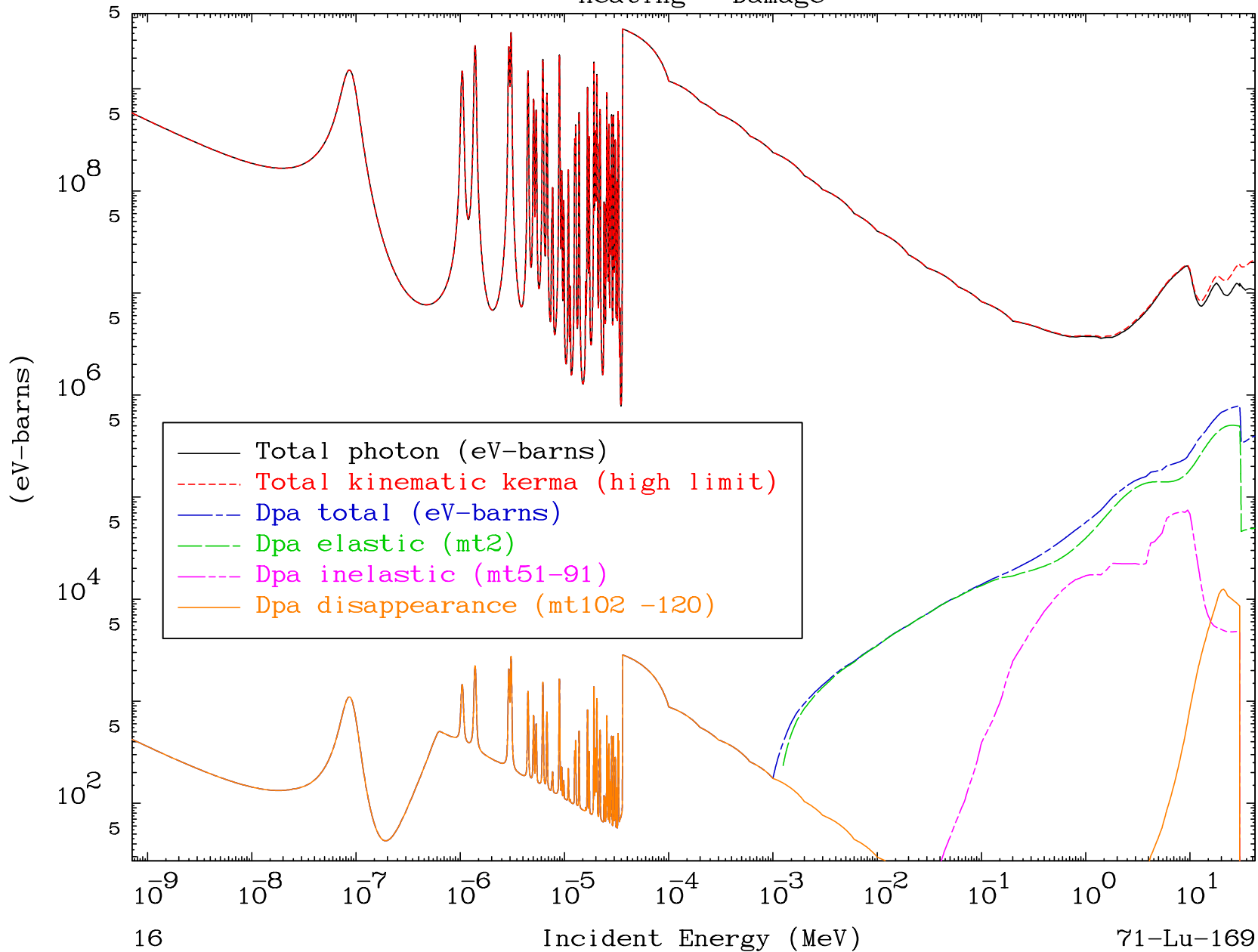




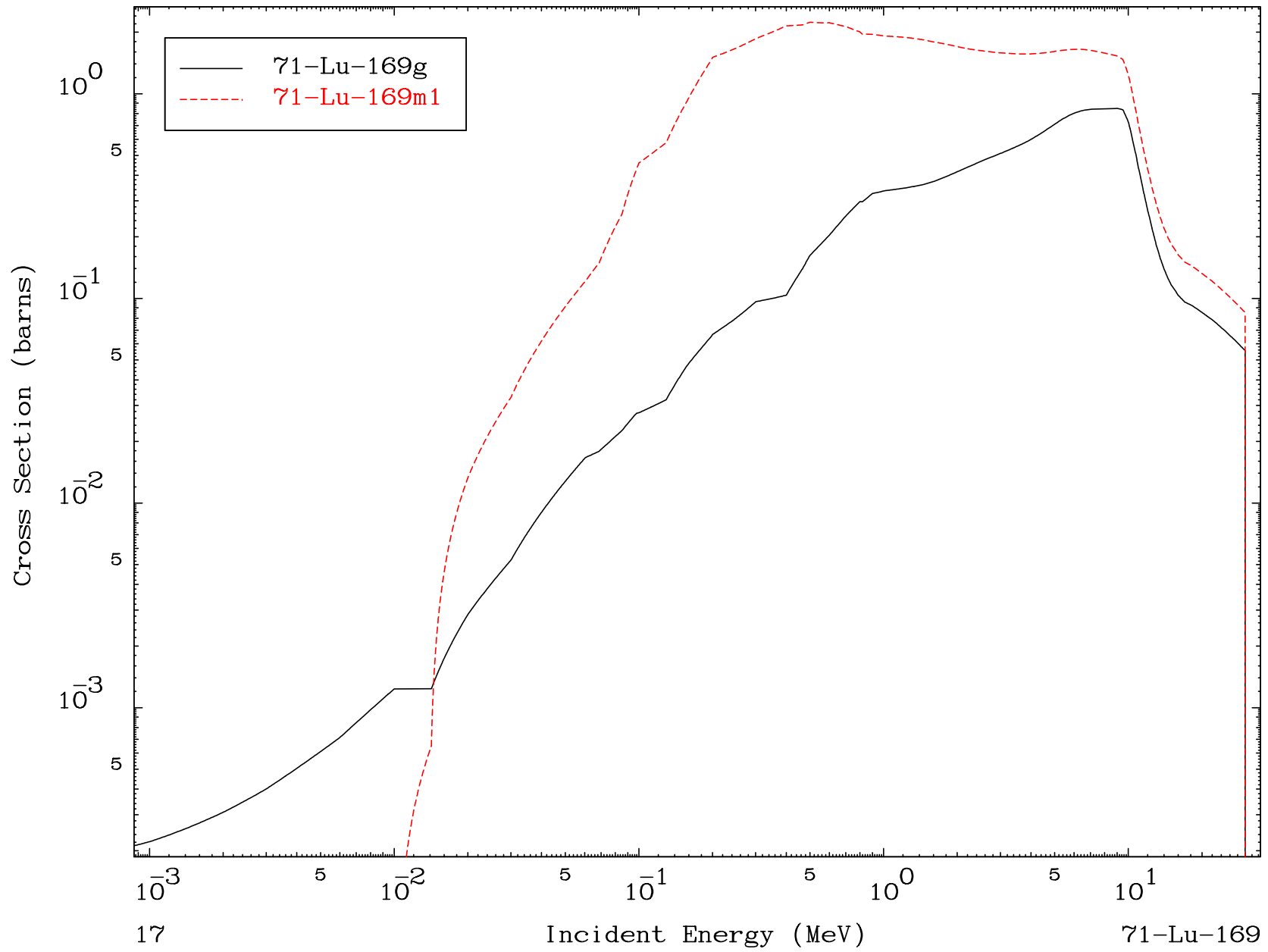




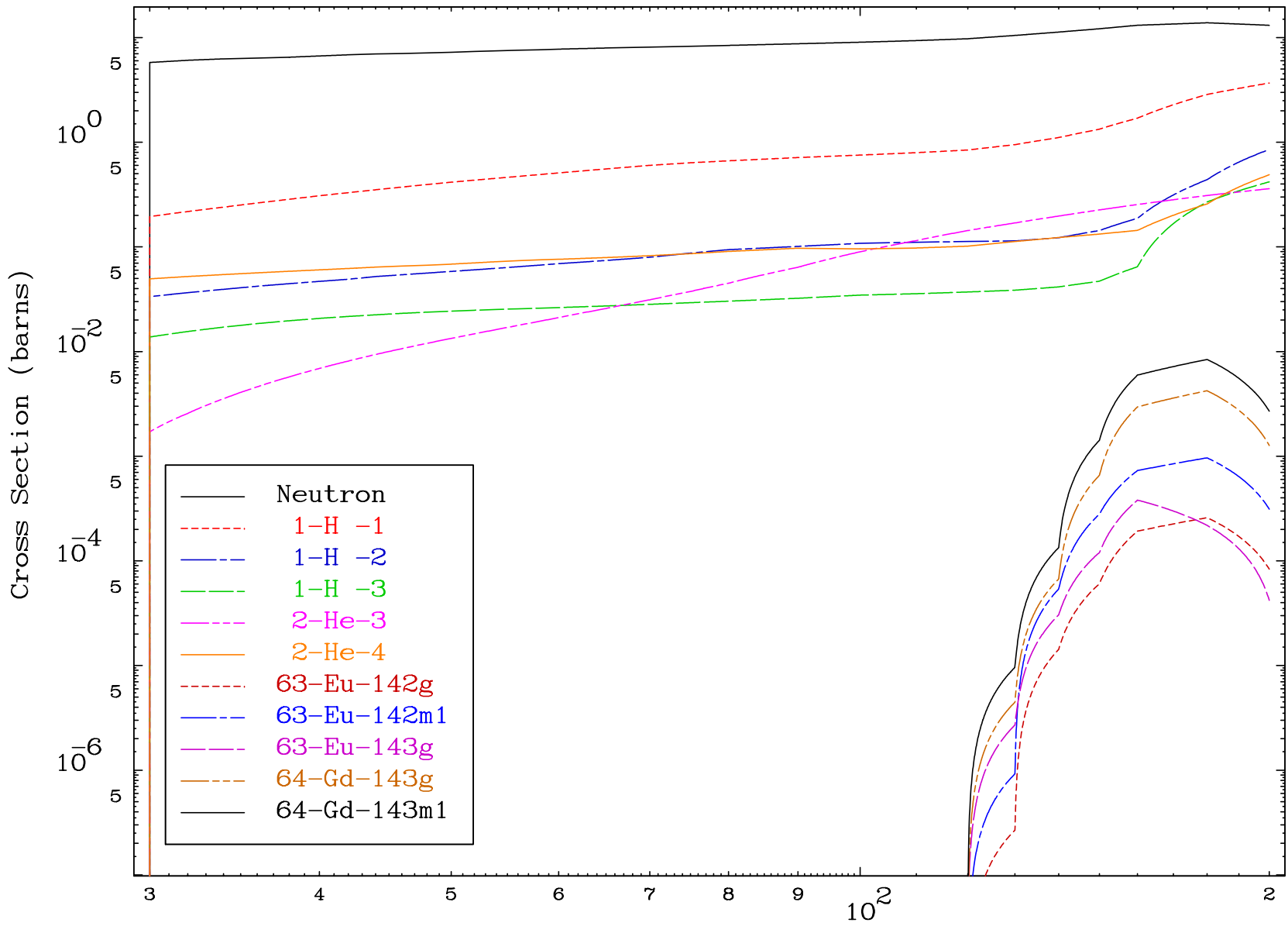




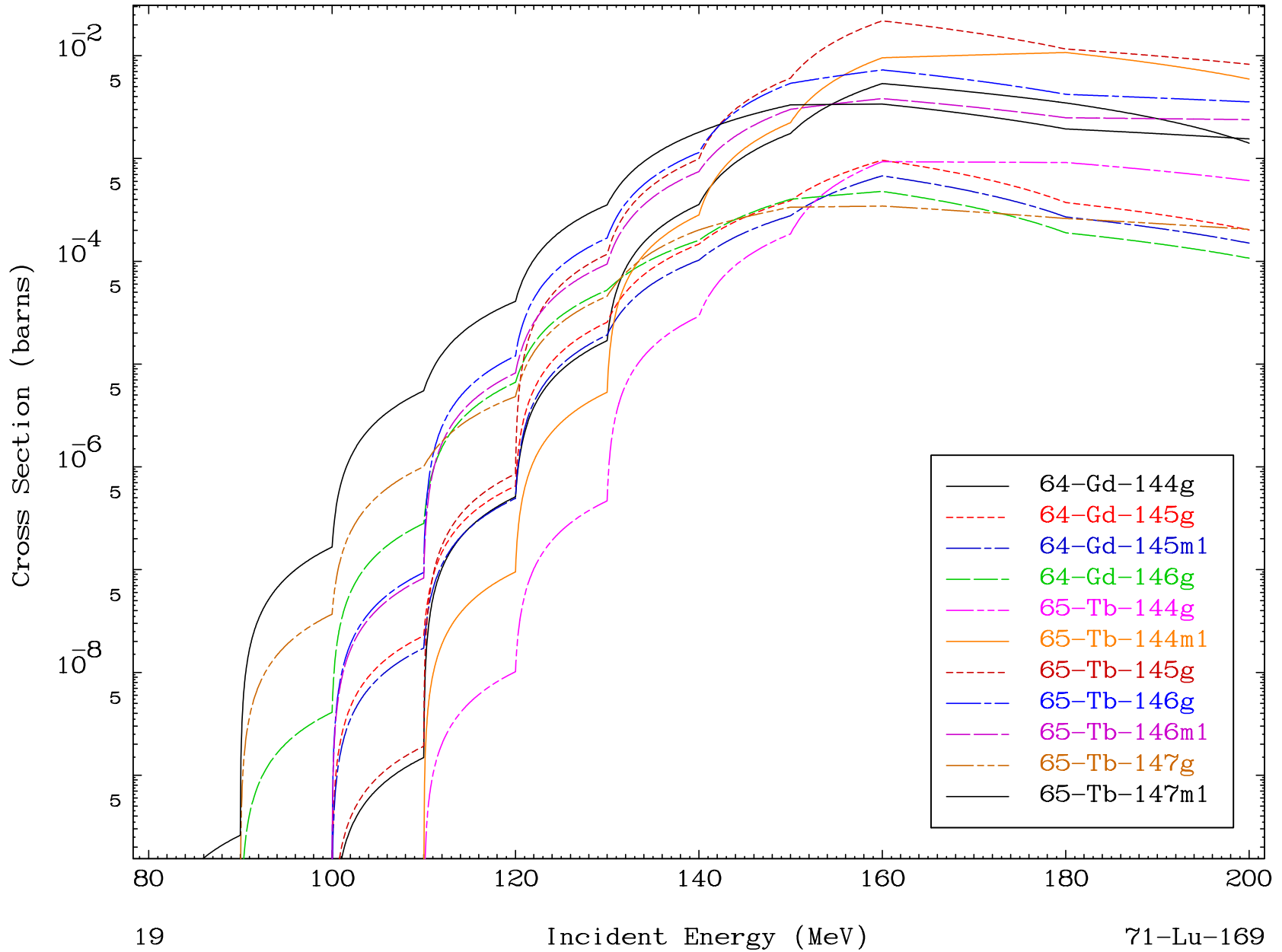


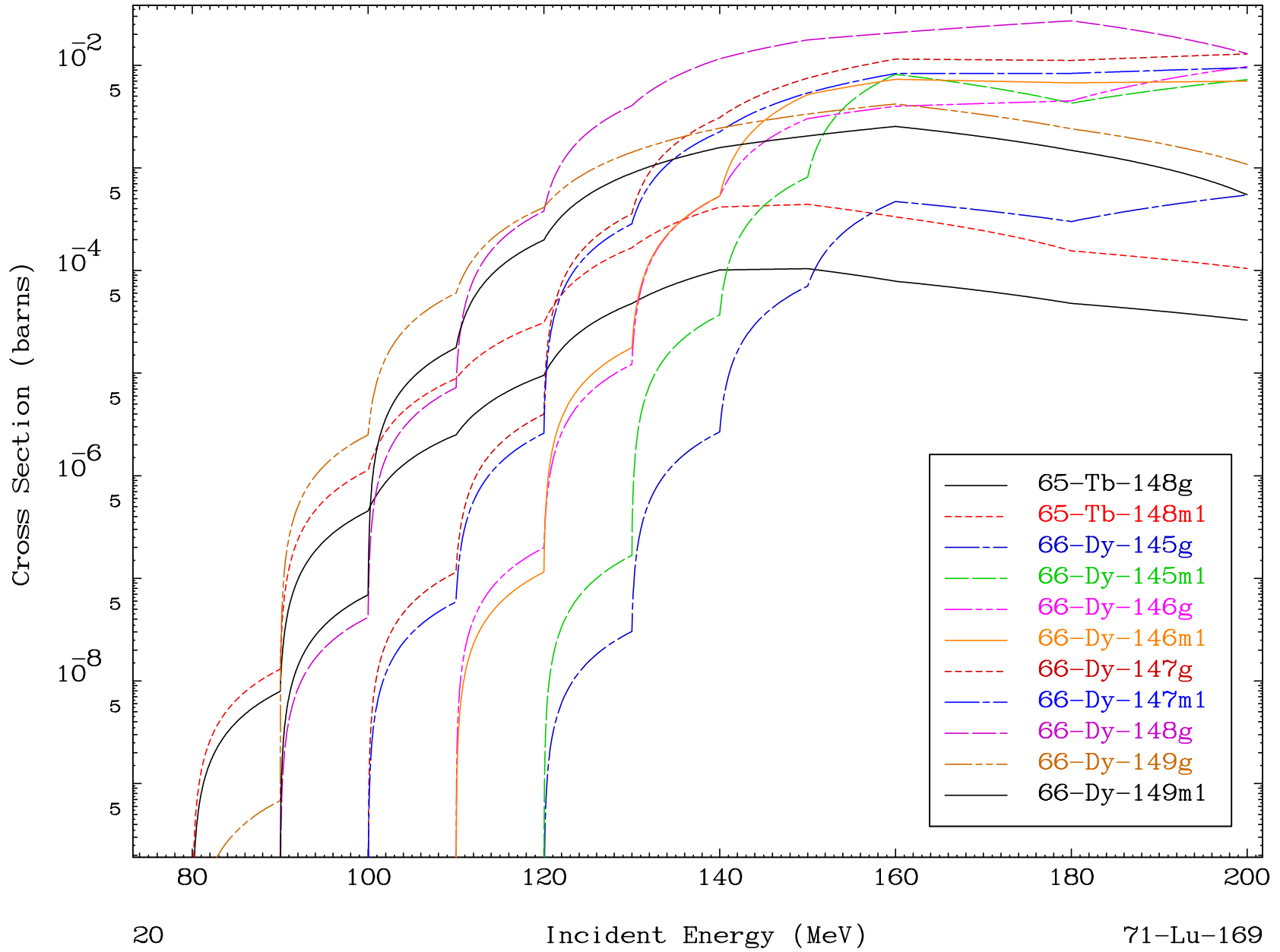


Radionuclide Production Cross Section

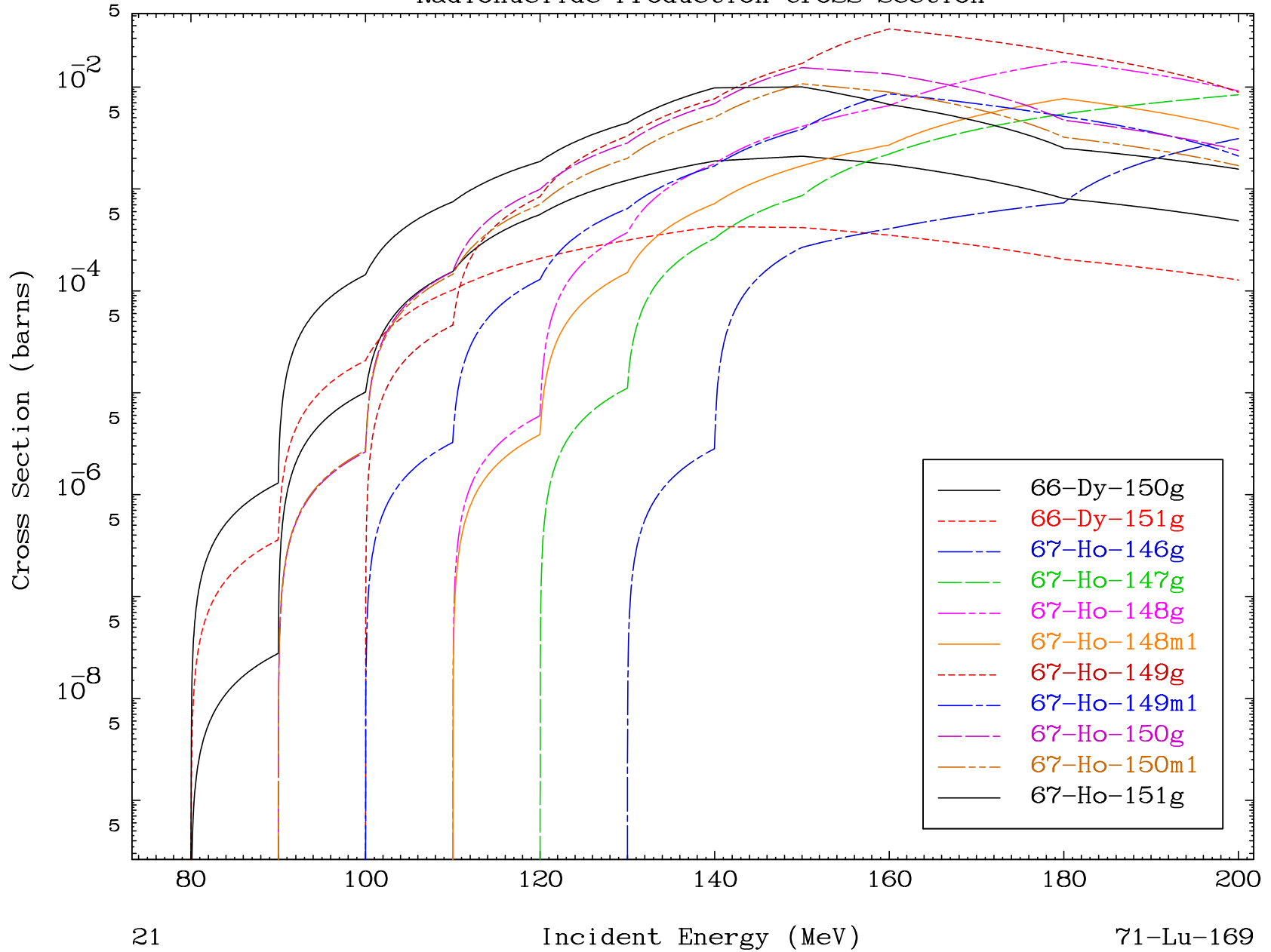


Radionuclide Production Cross Section

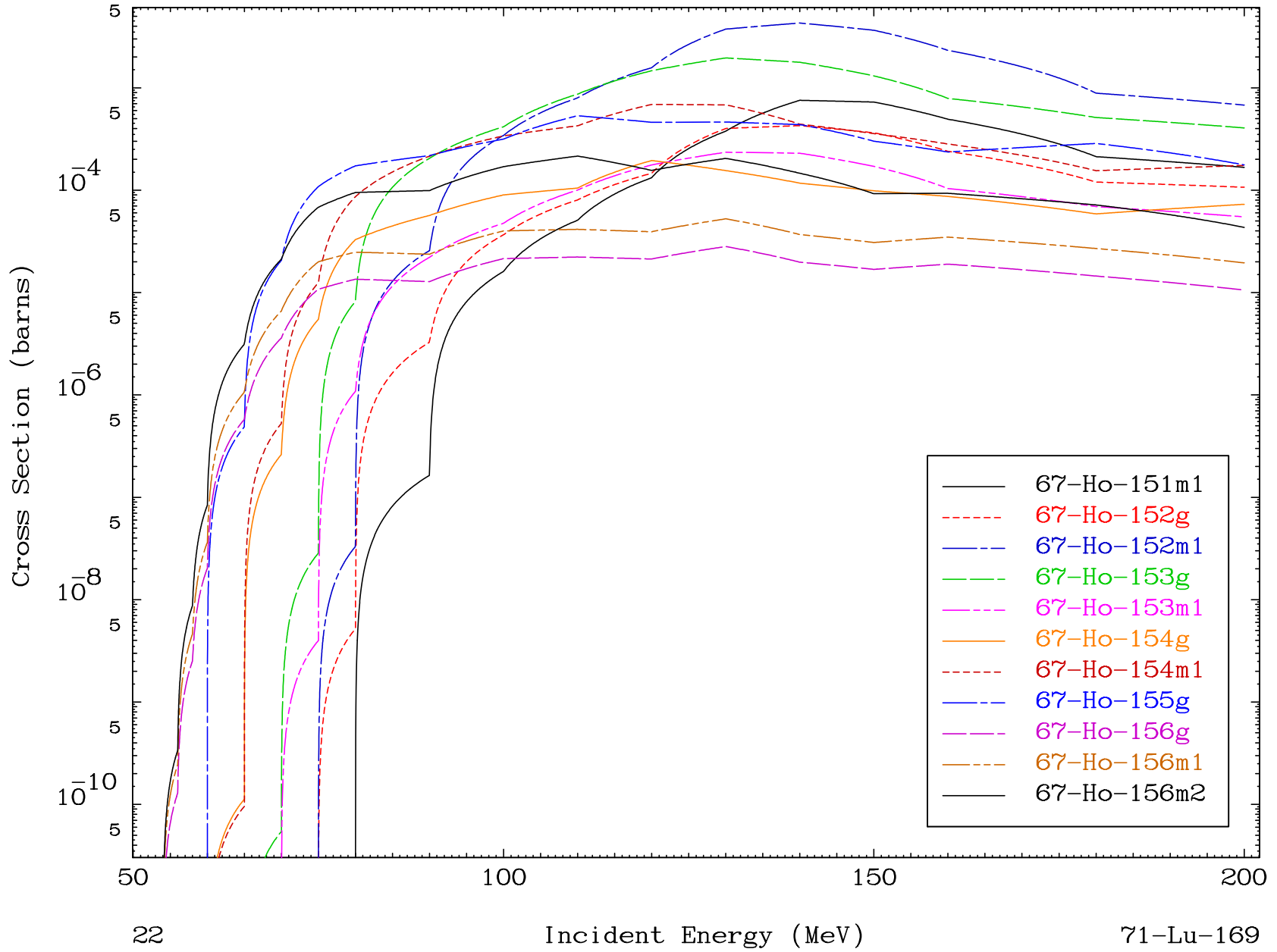




Radionuclide Production Cross Section



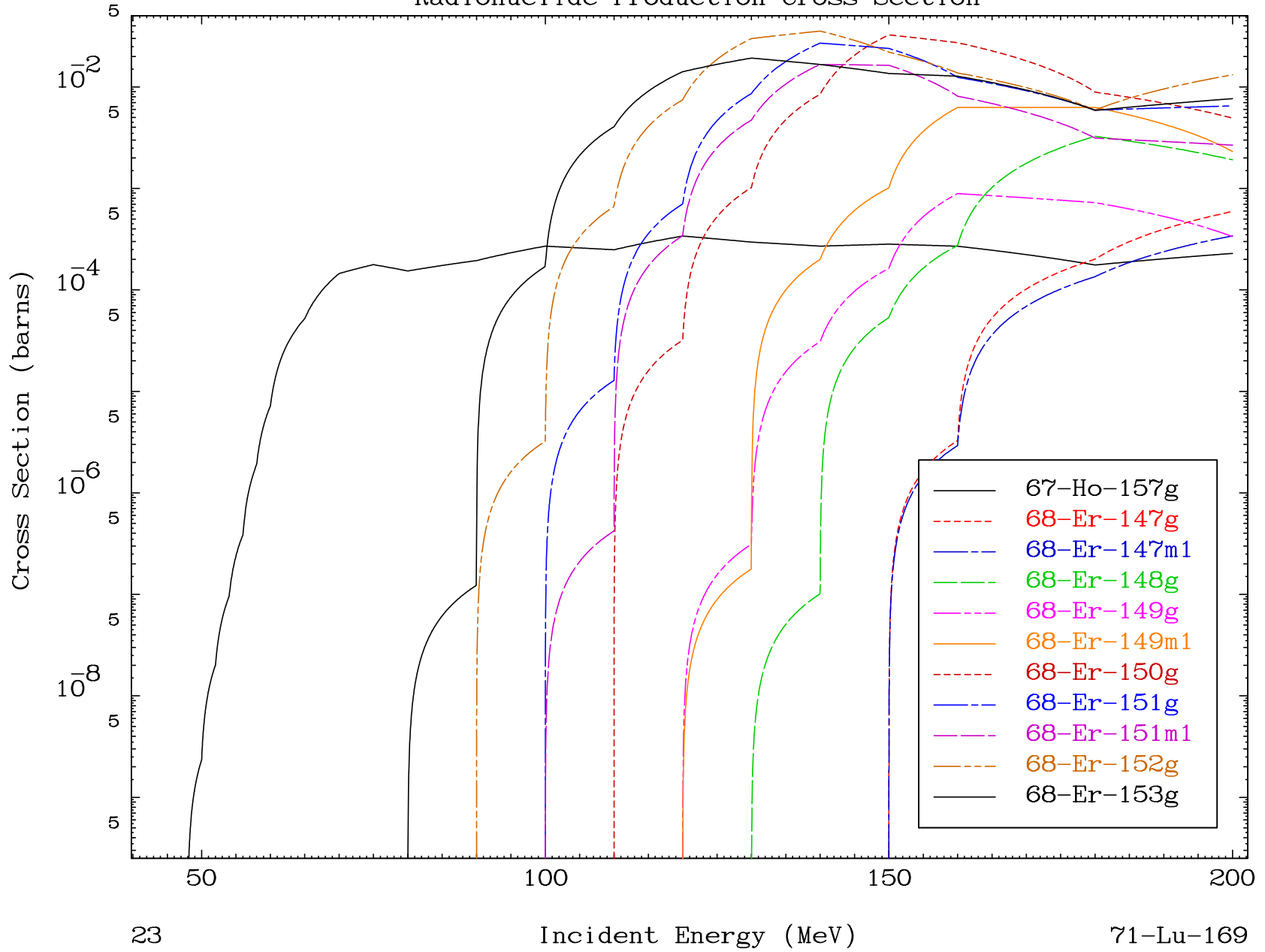
Radionuclide Production Cross Section



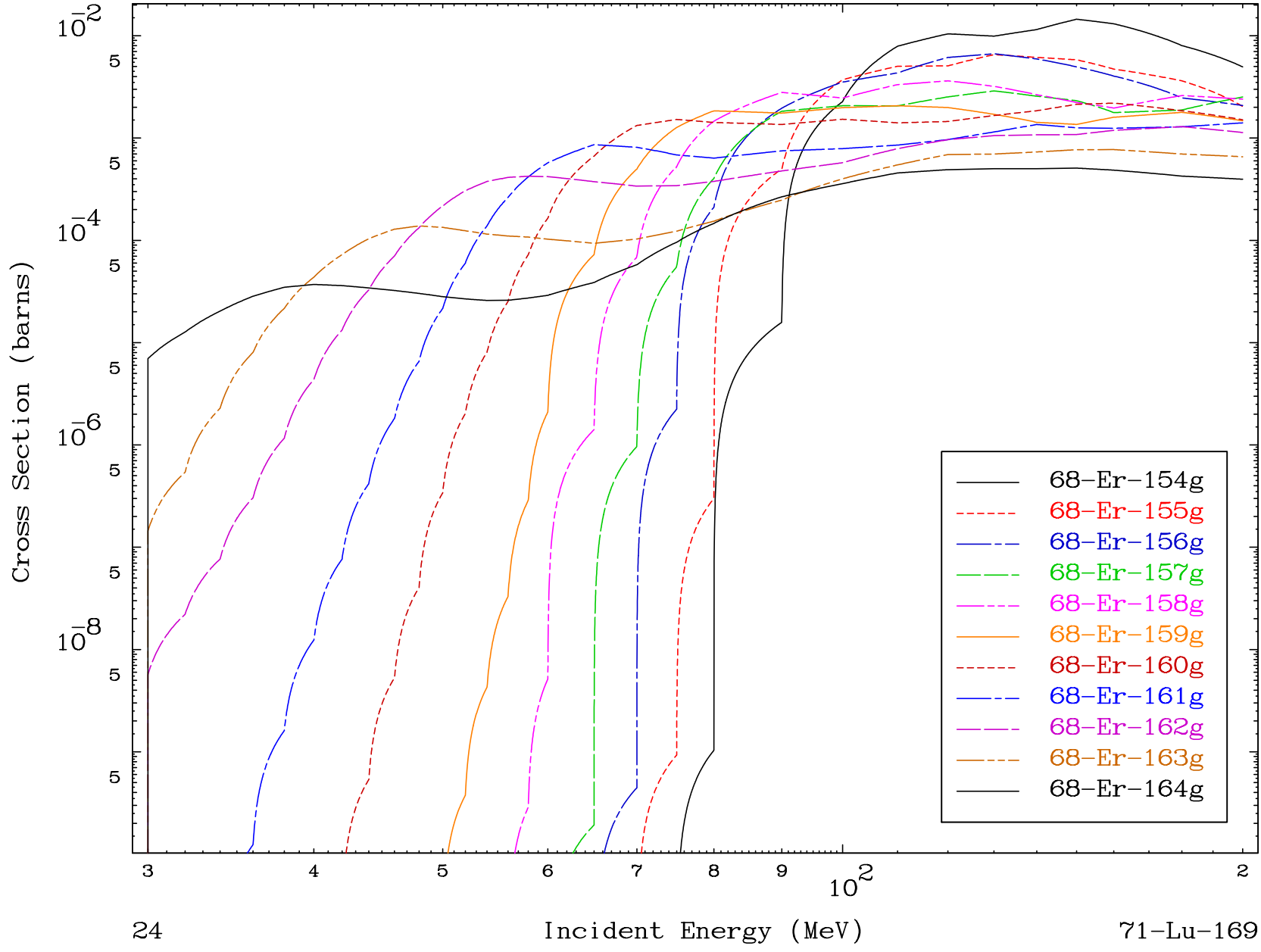
MAT 7108

(n,remainder)  
Radionuclide Production Cross Section

71-Lu-169

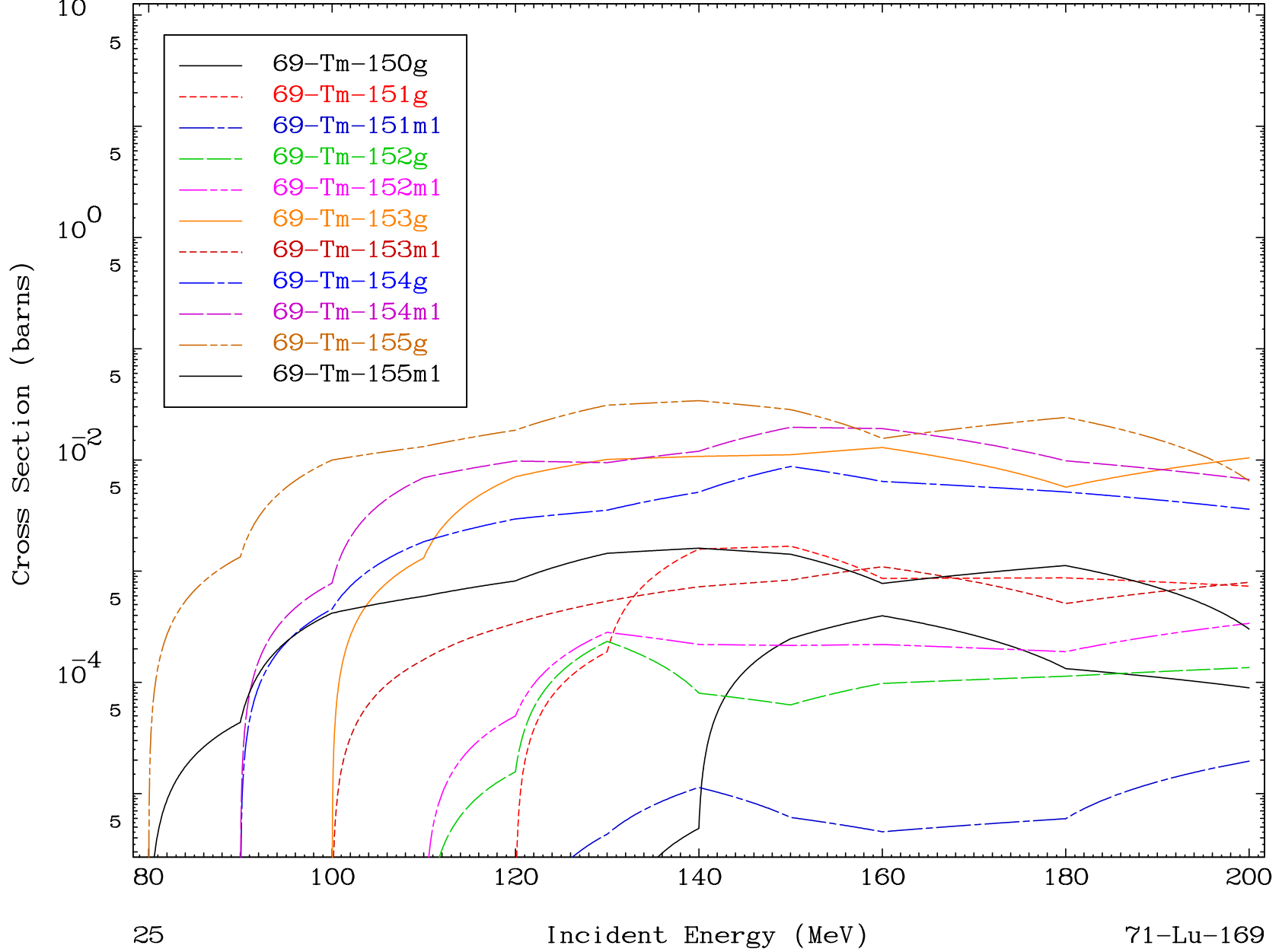


Radionuclide Production Cross Section





Radionuclide Production Cross Section

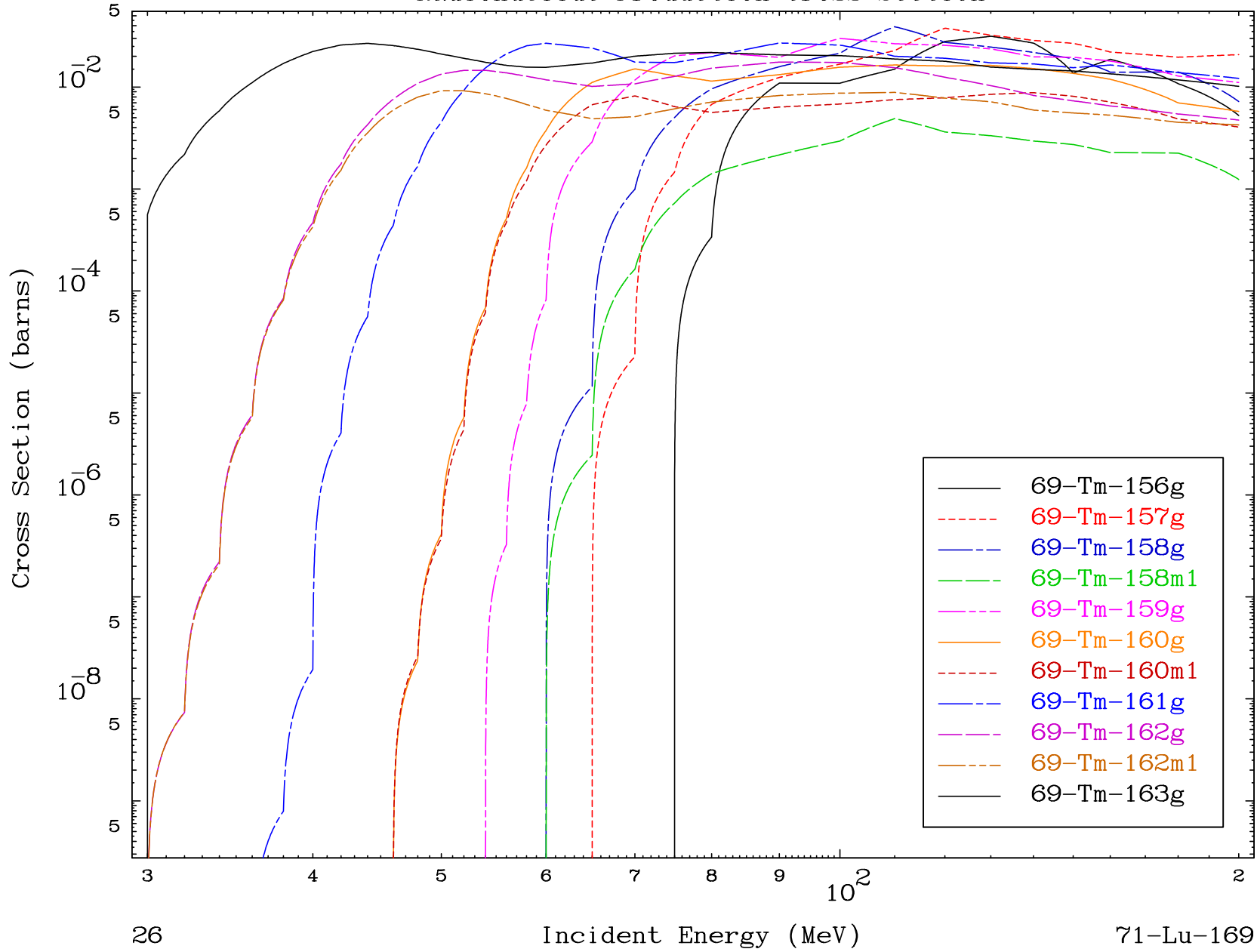


MAT 7108

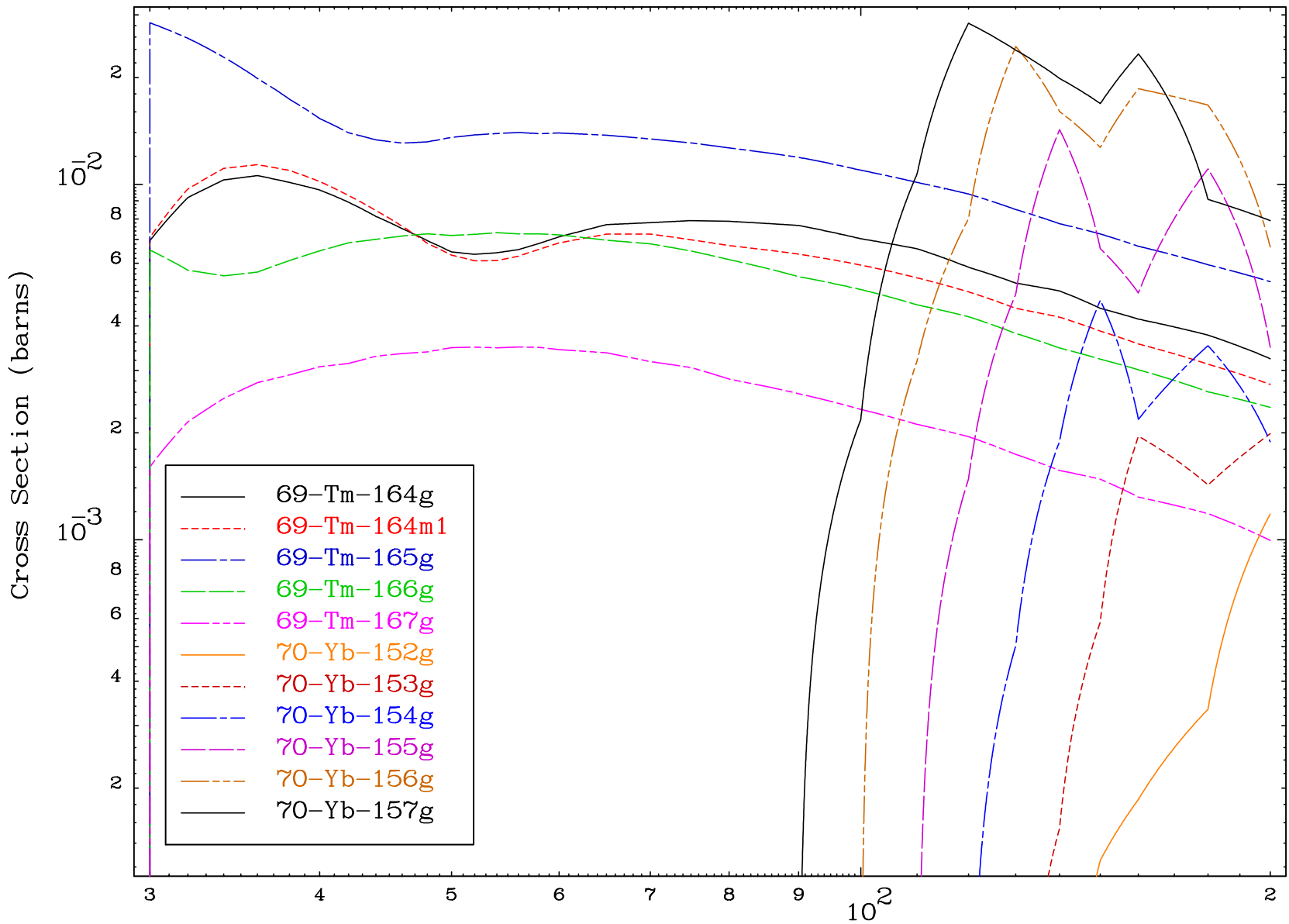
(n,remainder)

71-Lu-169

### Radionuclide Production Cross Section



Radionuclide Production Cross Section

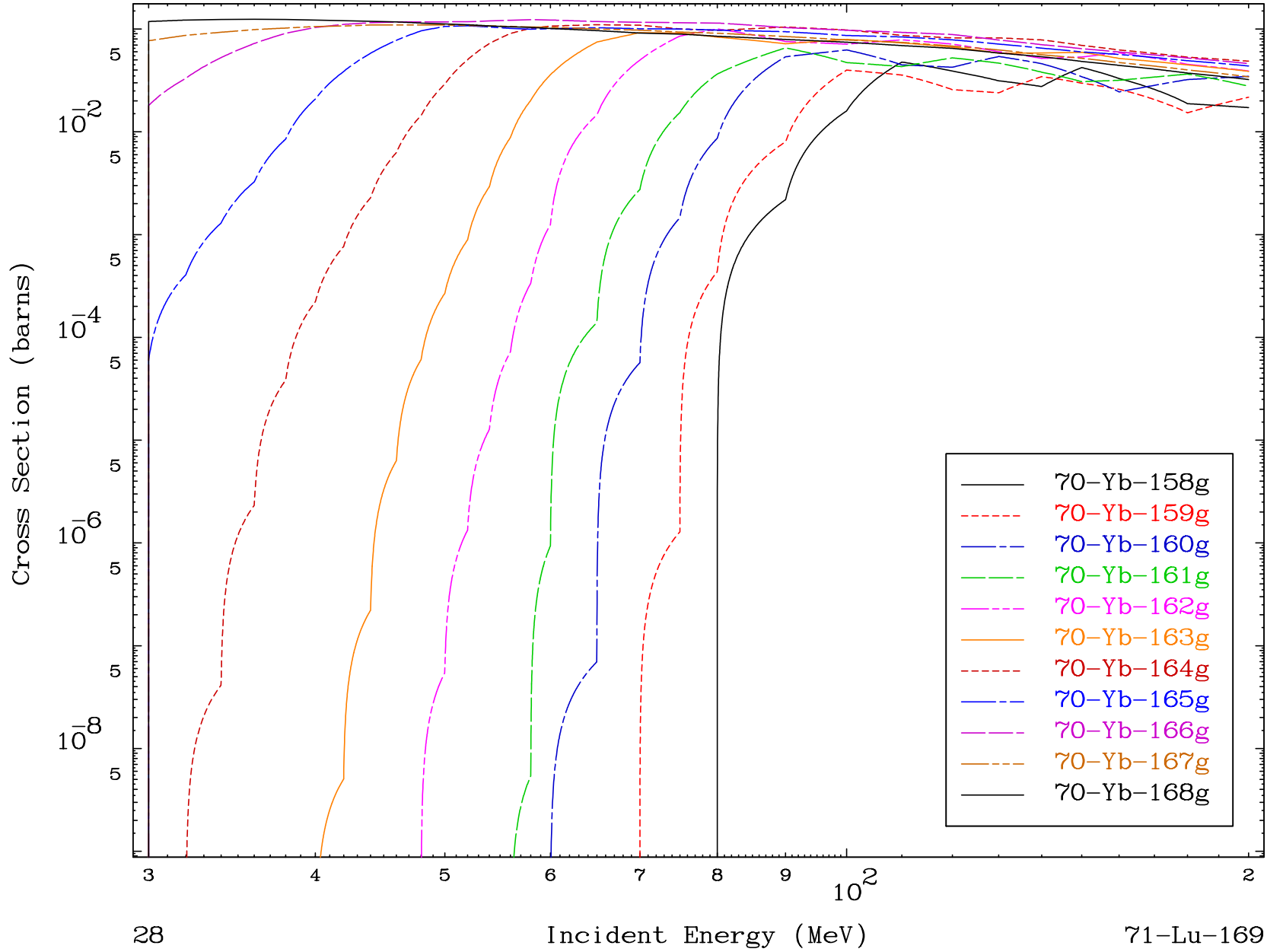


MAT 7108

(n,remainder)

71-Lu-169

### Radionuclide Production Cross Section



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Incident Energy (MeV)

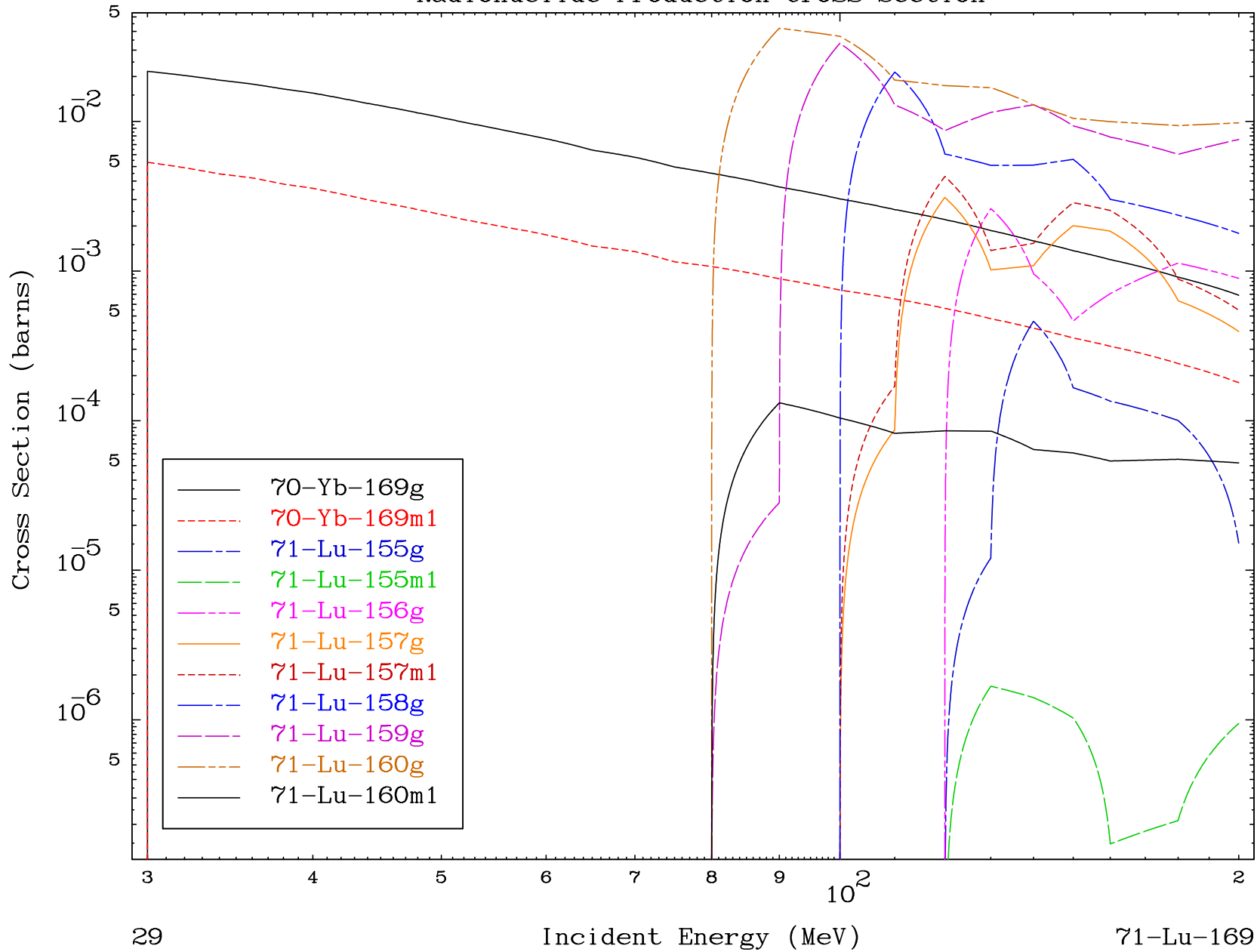
71-Lu-169

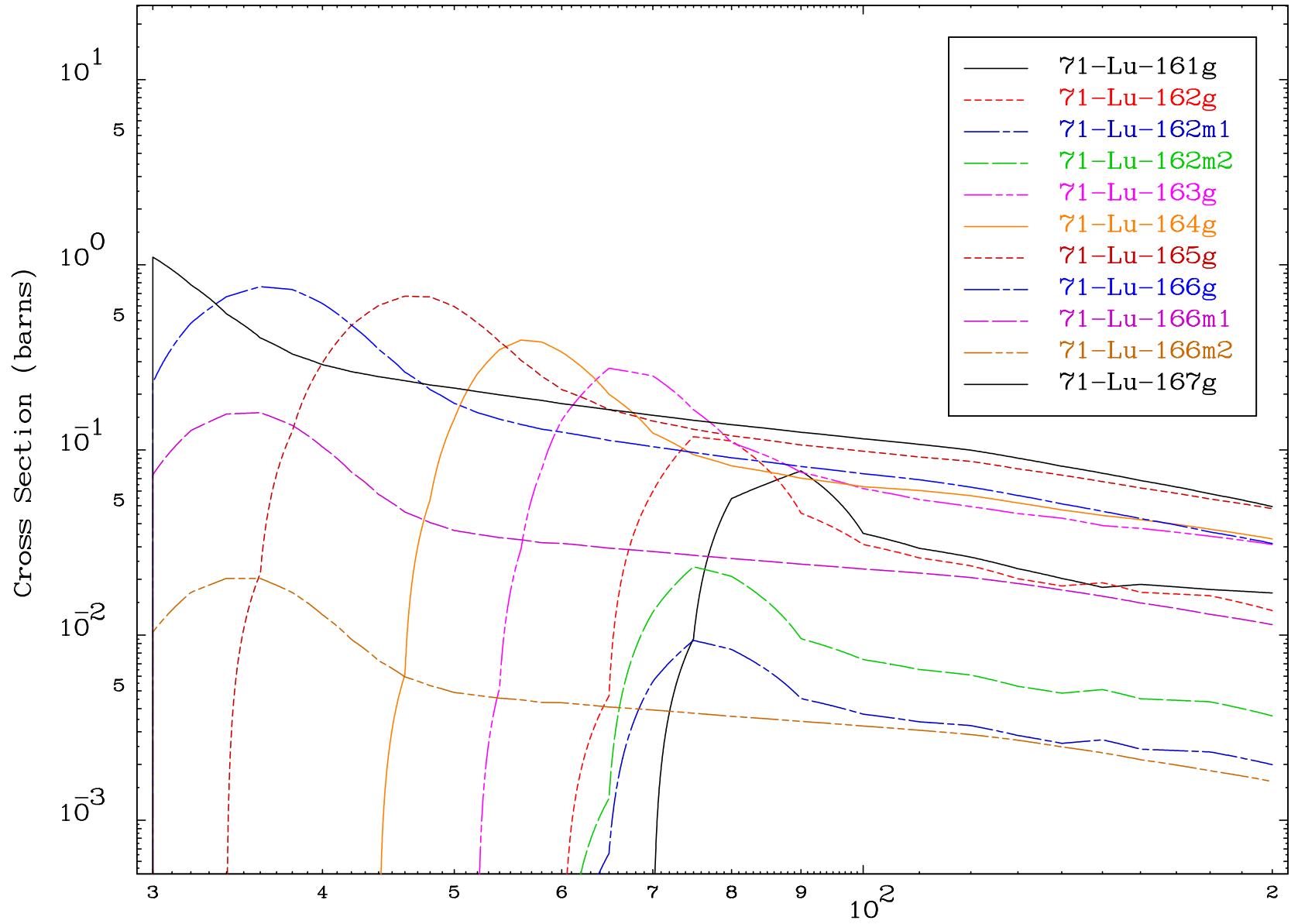
MAT 7108

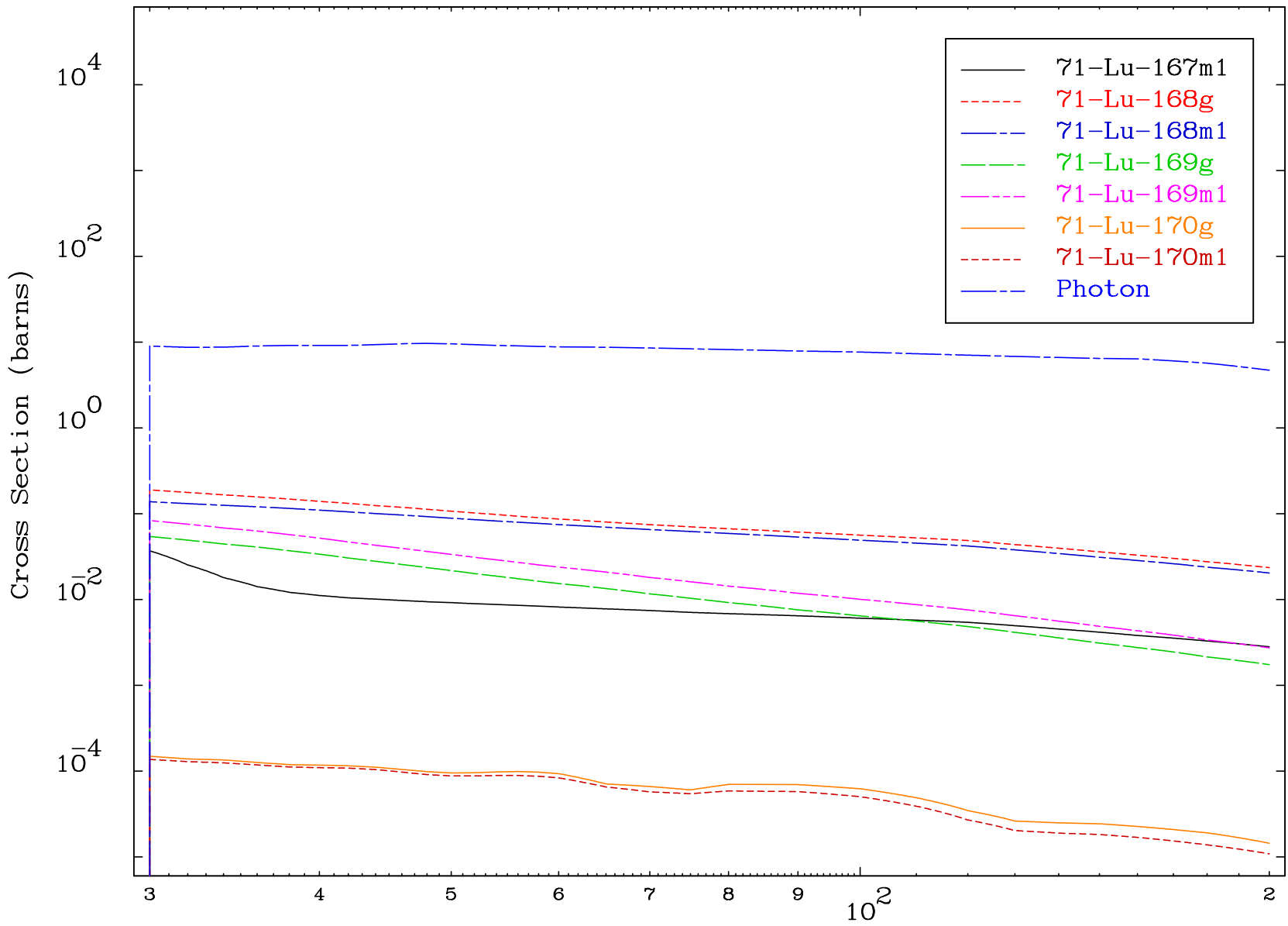
(n,remainder)

71-Lu-169

### Radionuclide Production Cross Section





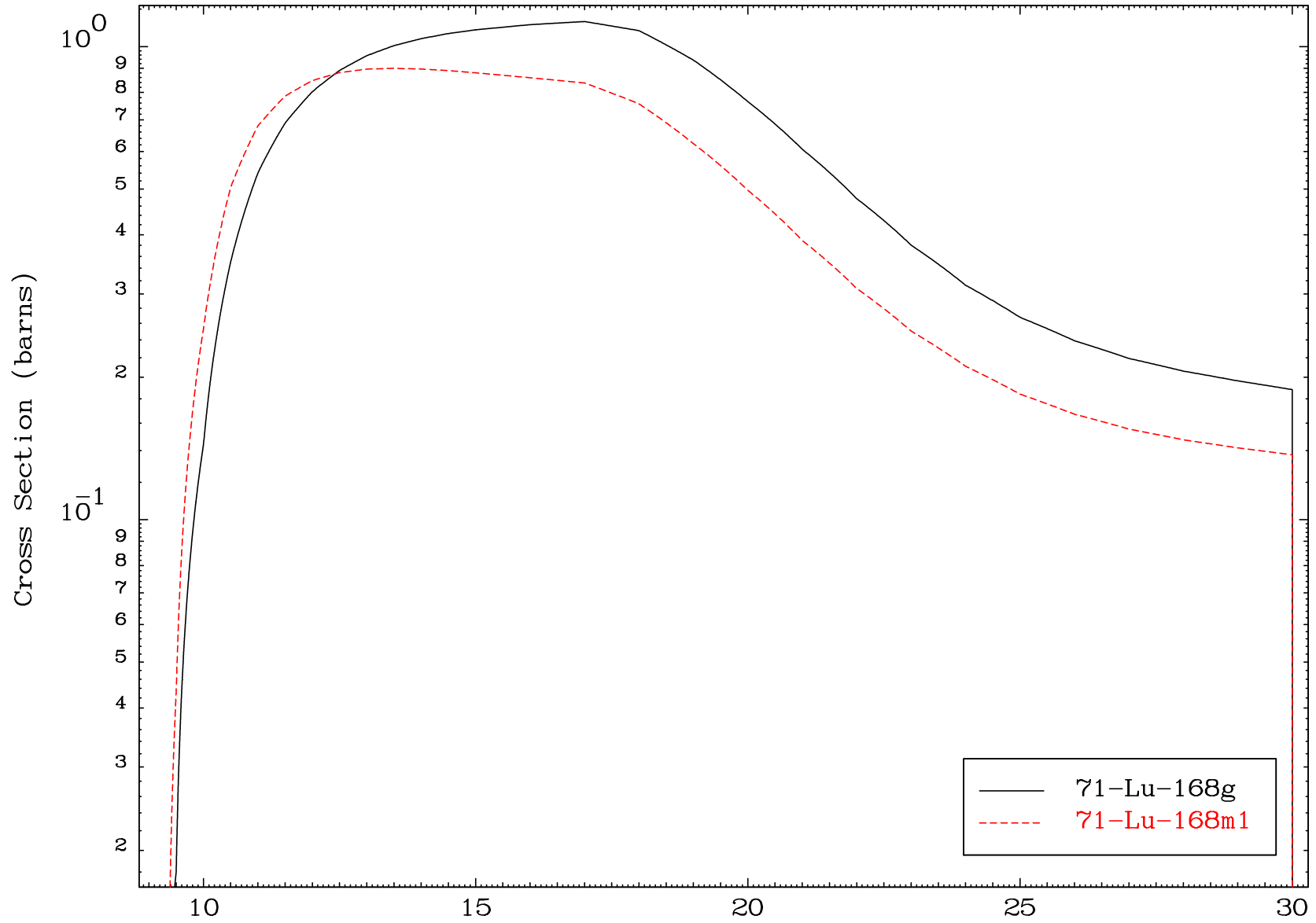


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(n,2n)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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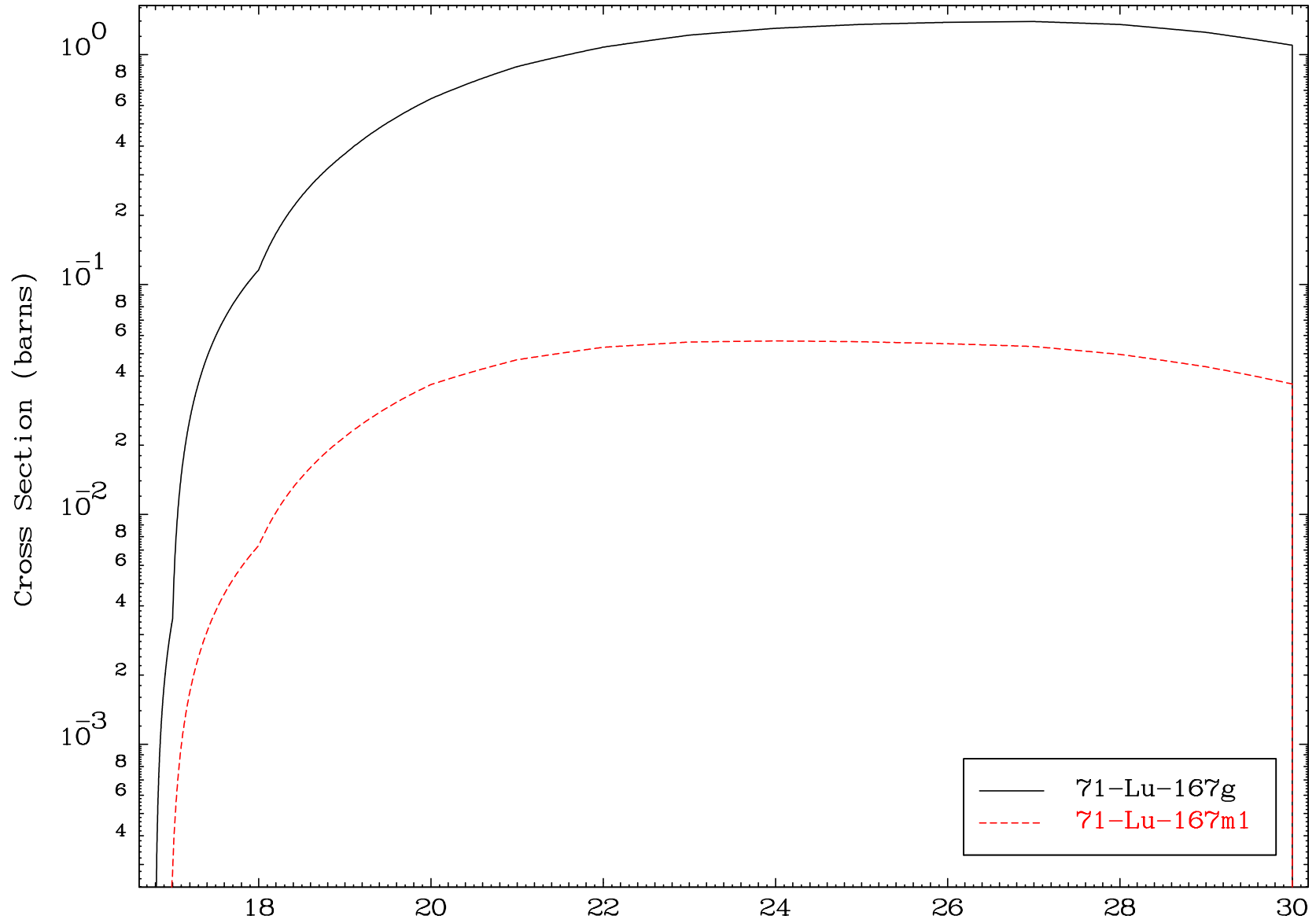


MAT 7108

(n,3n)

71-Lu-169

Radionuclide Production Cross Section



33

Incident Energy (MeV)

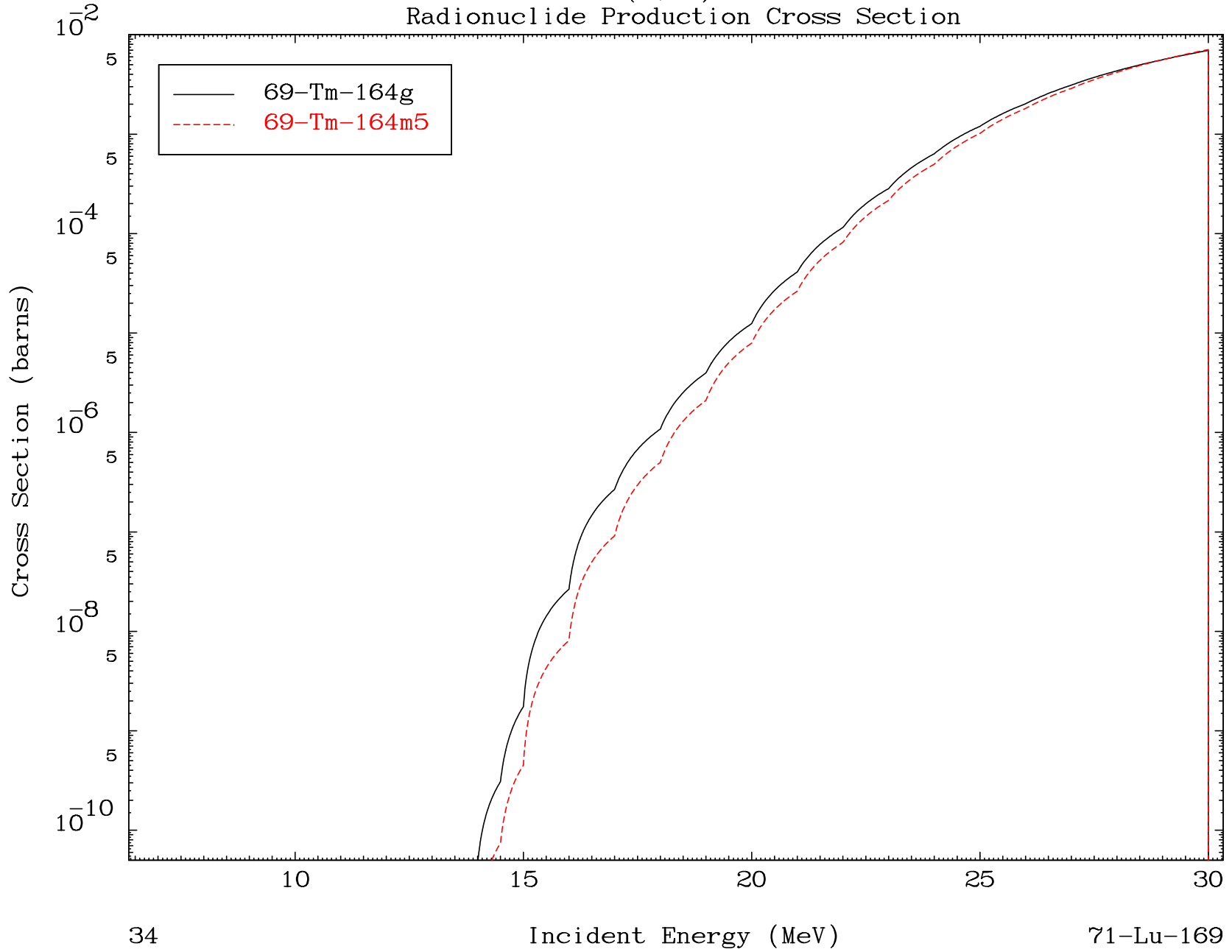
71-Lu-169

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(n,2n)  $\alpha$

71-Lu-169

Radionuclide Production Cross Section

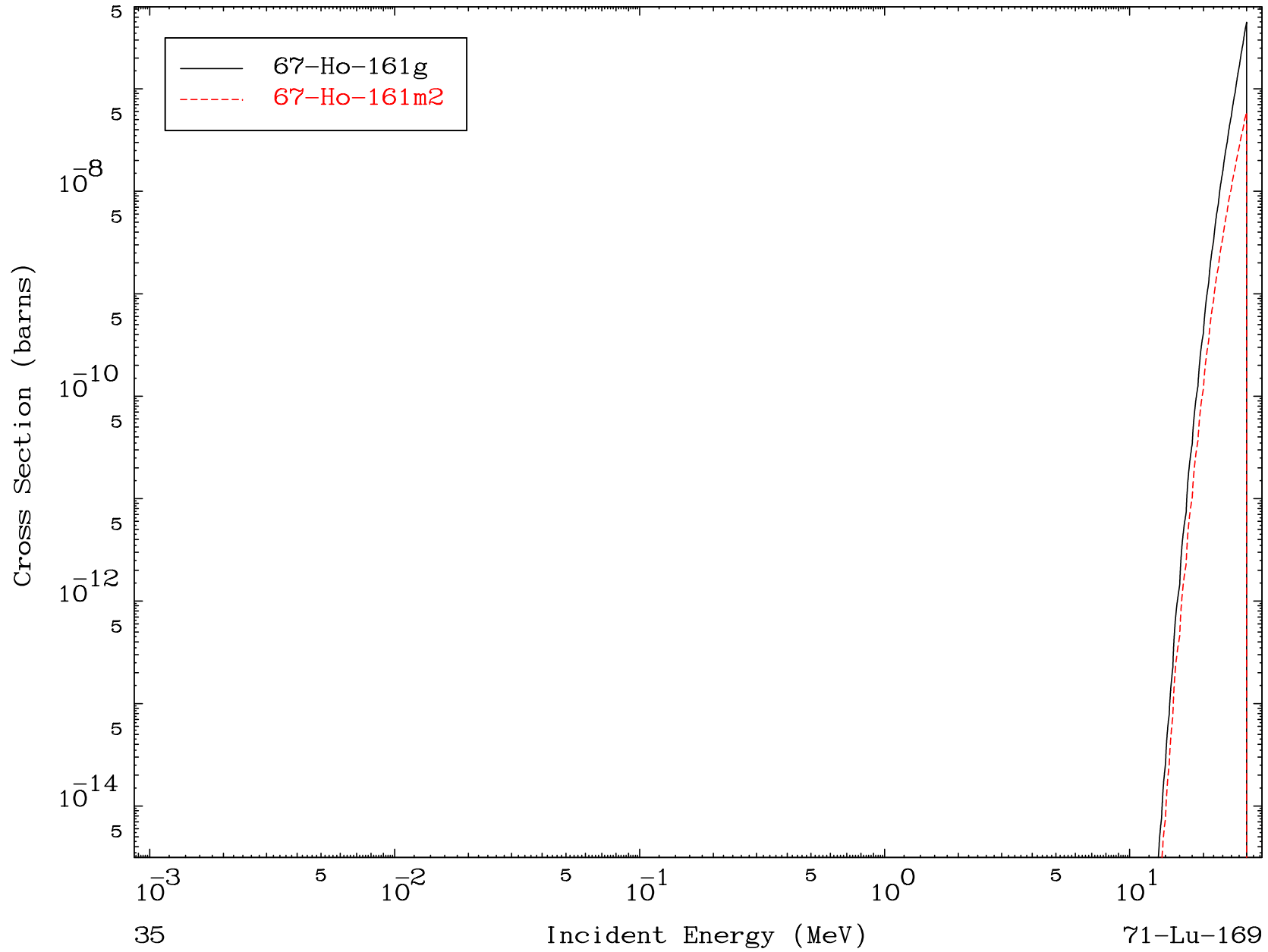


MAT 7108

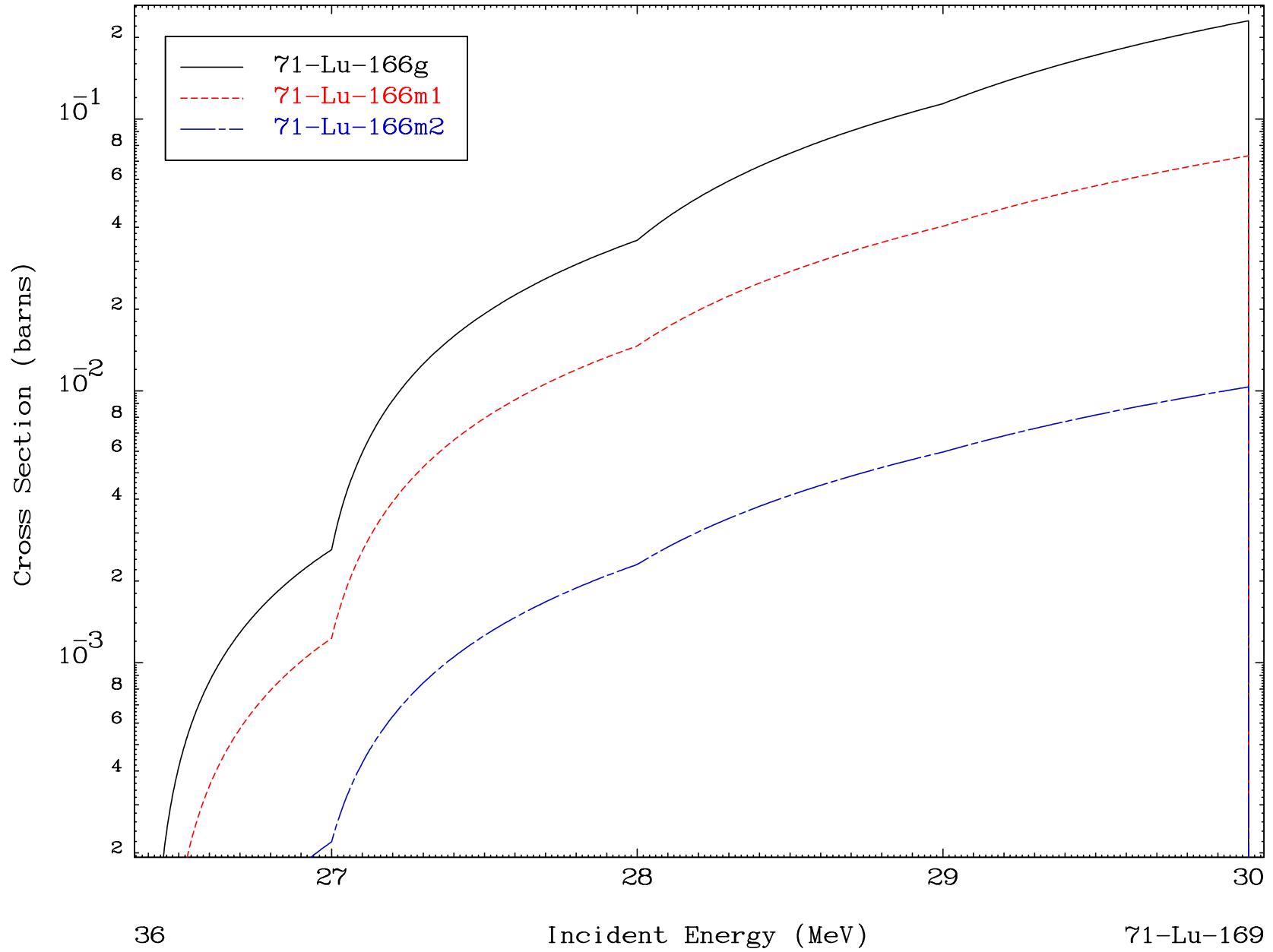
(n,n') 2 $\alpha$

71-Lu-169

Radionuclide Production Cross Section



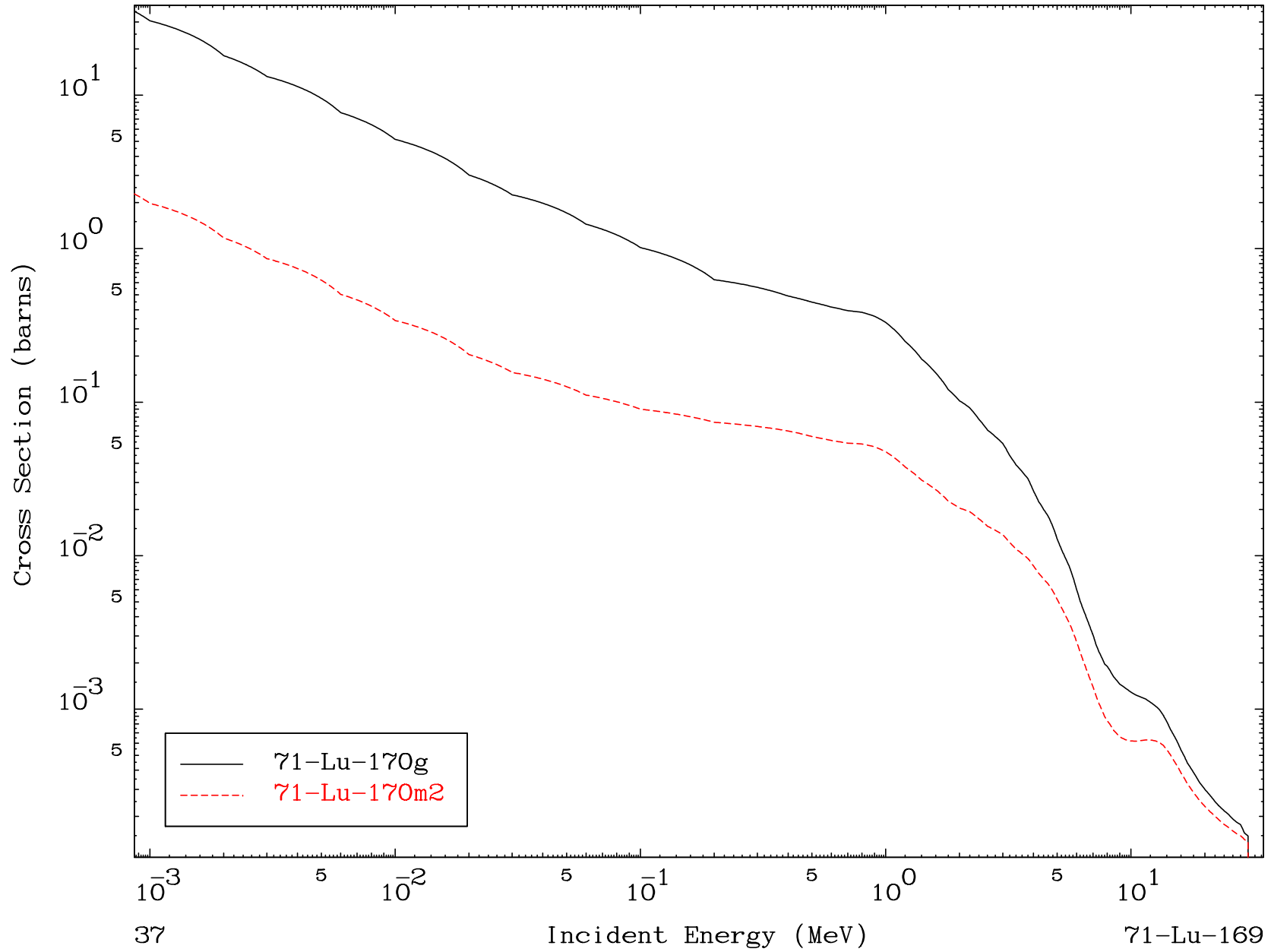
Radionuclide Production Cross Section



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(n,  $\gamma$ )  
Radionuclide Production Cross Section

<sup>71</sup>Lu-169

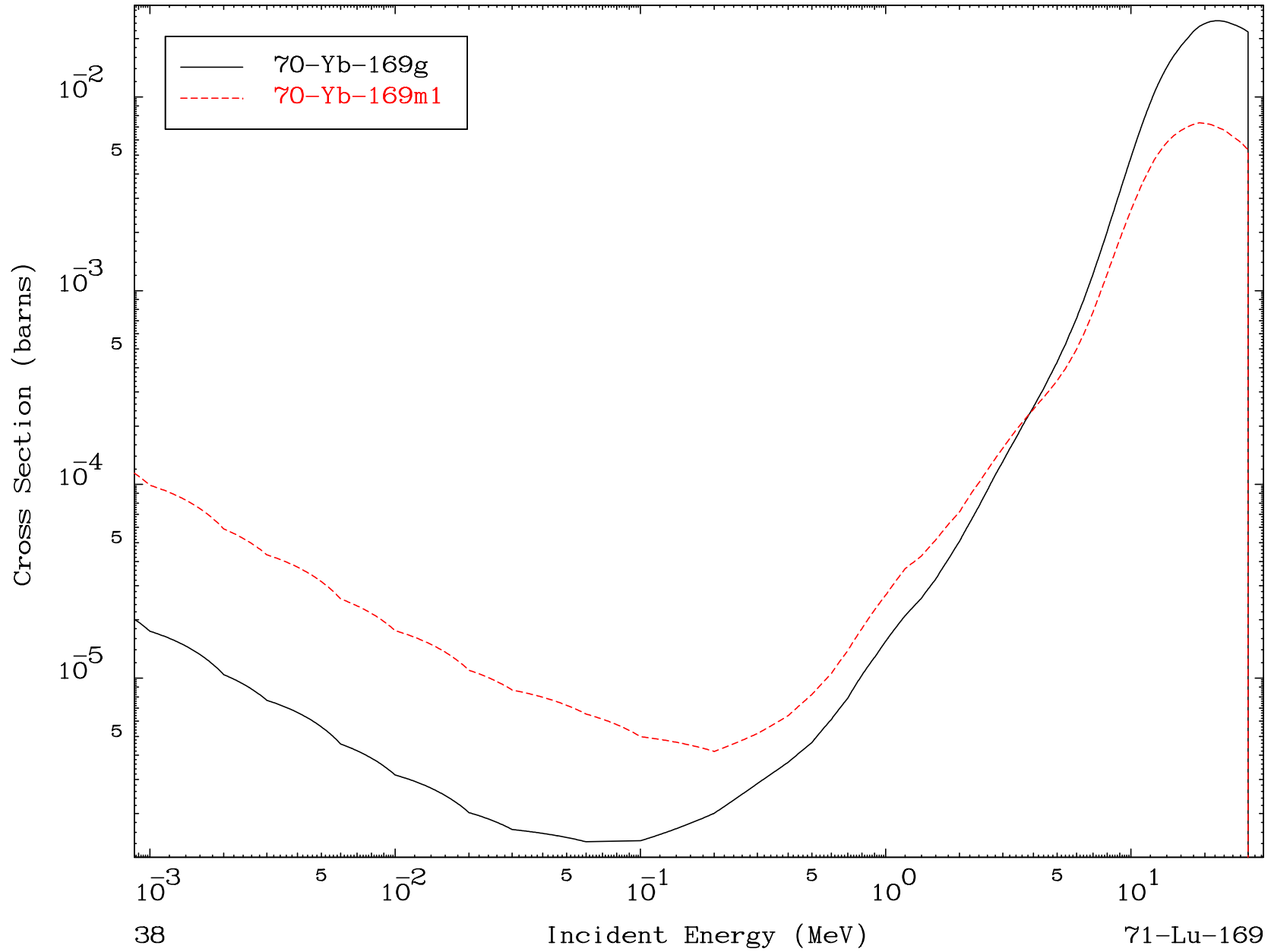


MAT 7108

(n,p)

71-Lu-169

Radionuclide Production Cross Section



MAT 7108

(n,2 $\alpha$ )

71-Lu-169

Radionuclide Production Cross Section

