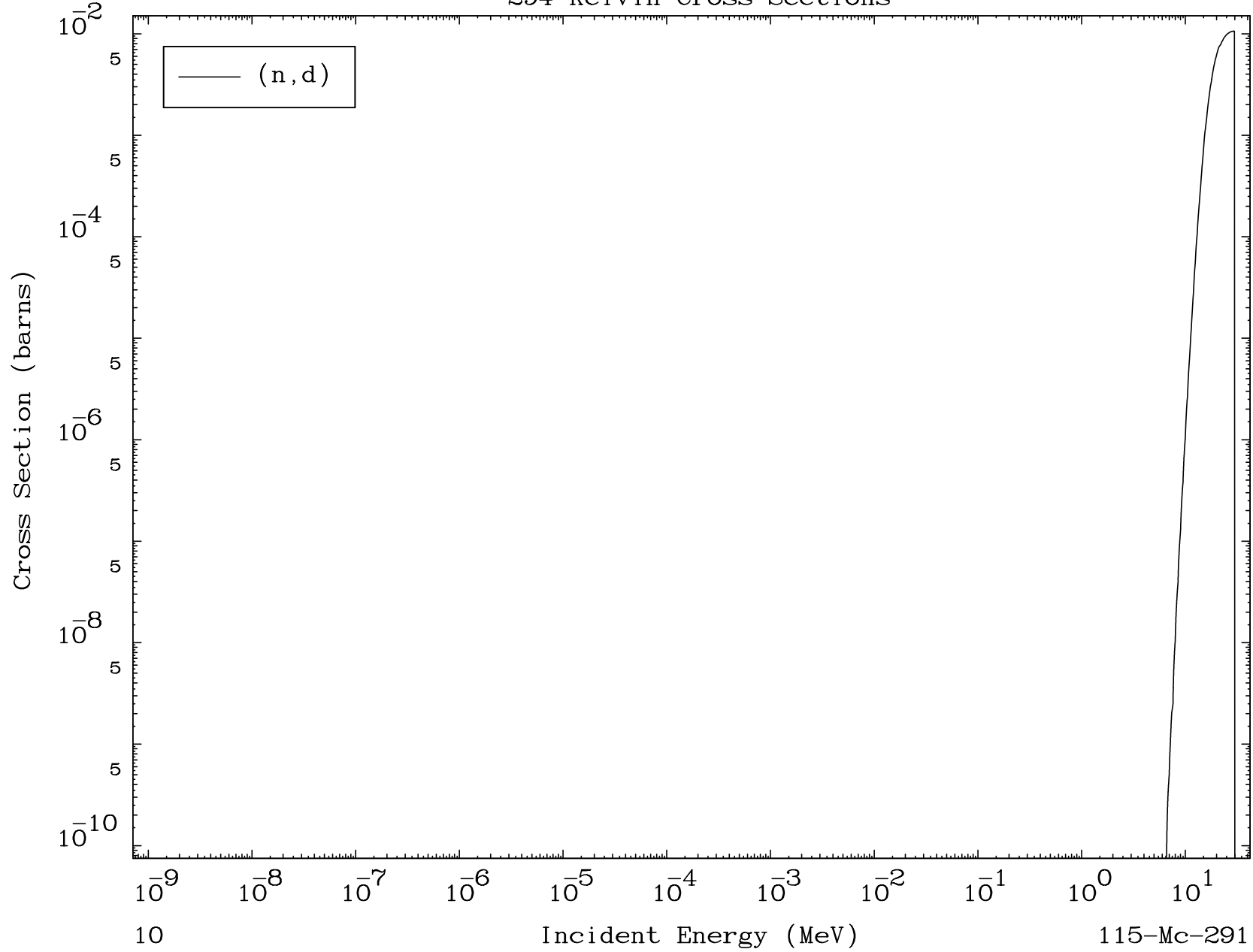
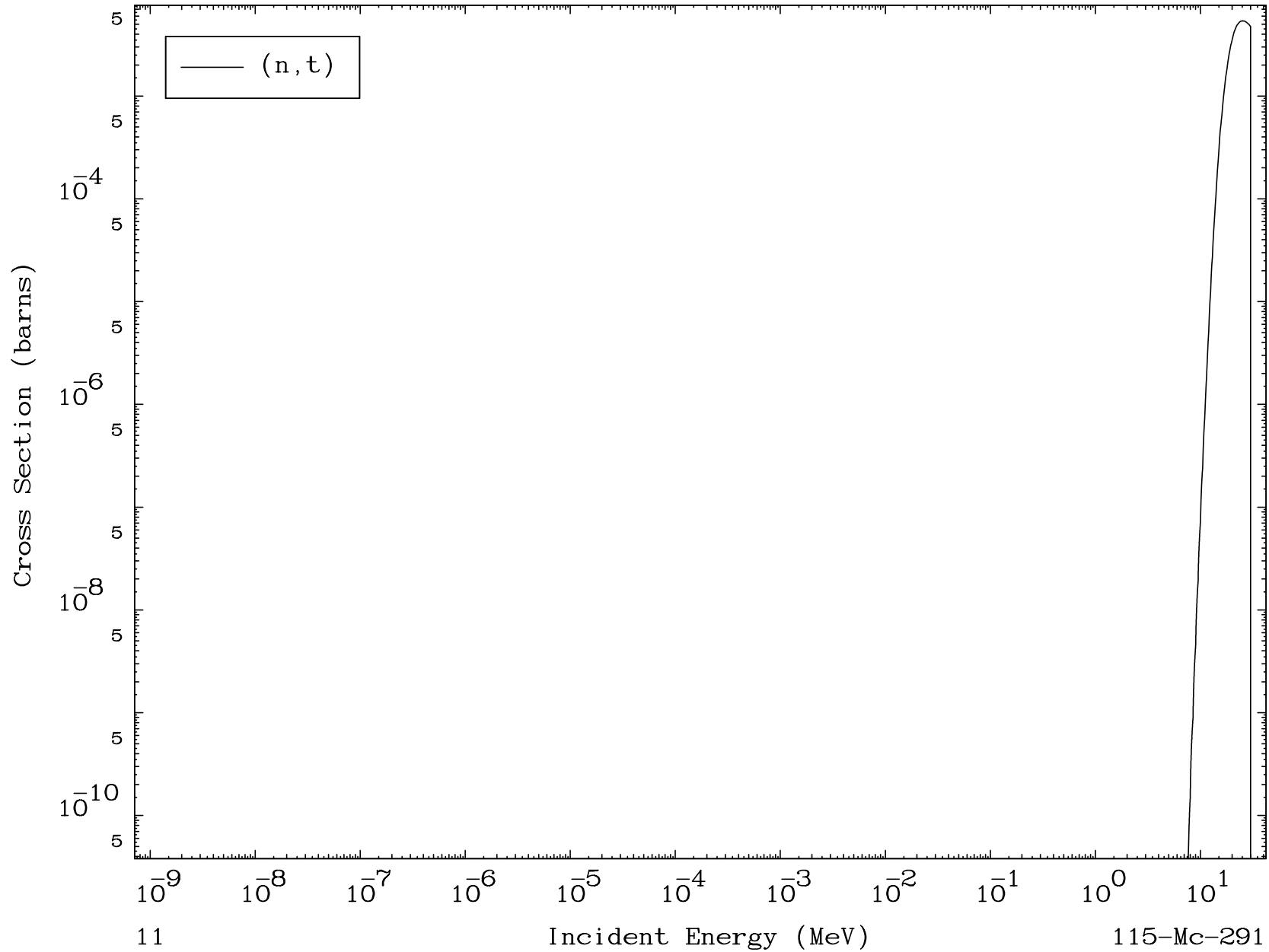


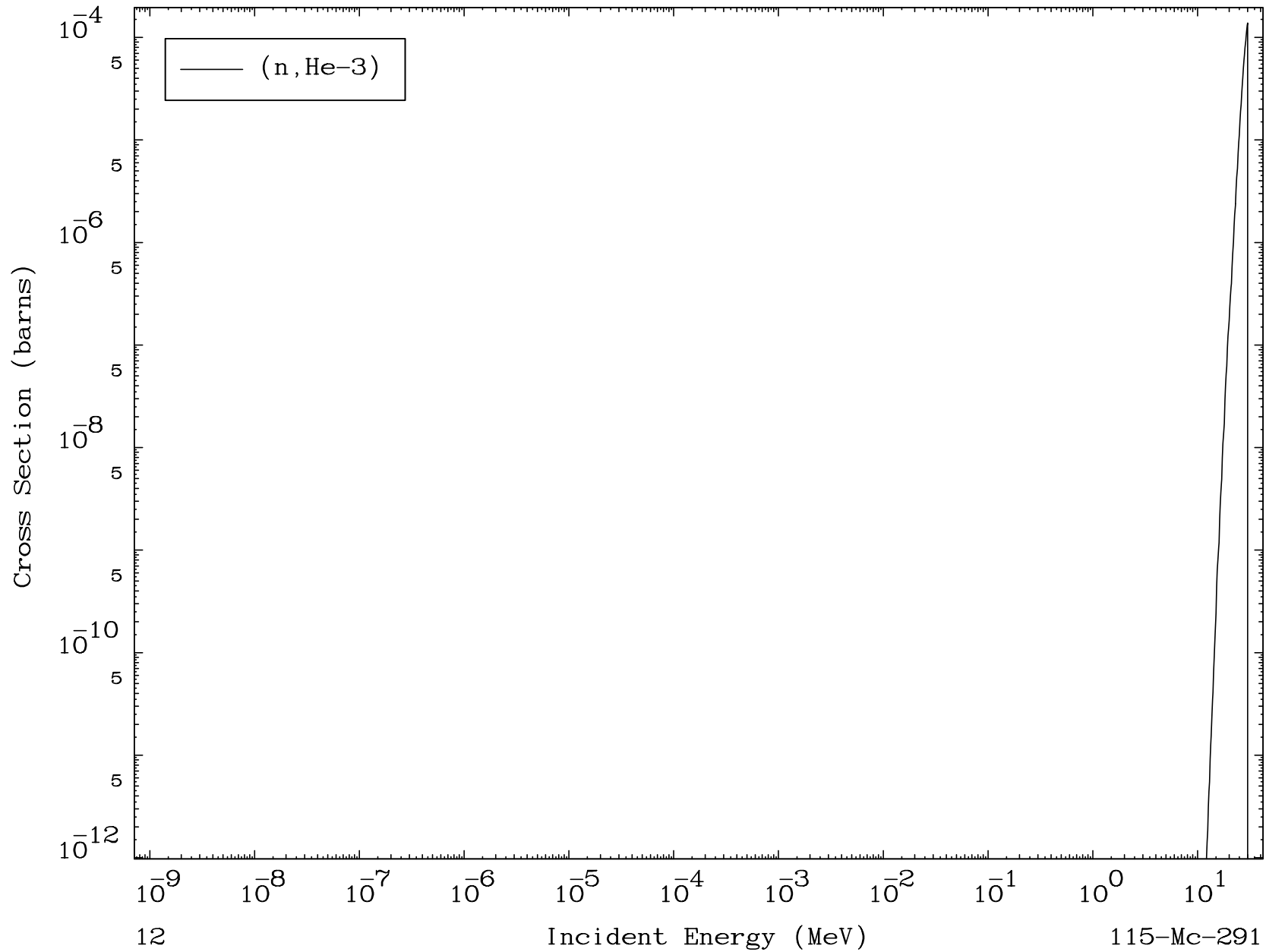
MAT 1591

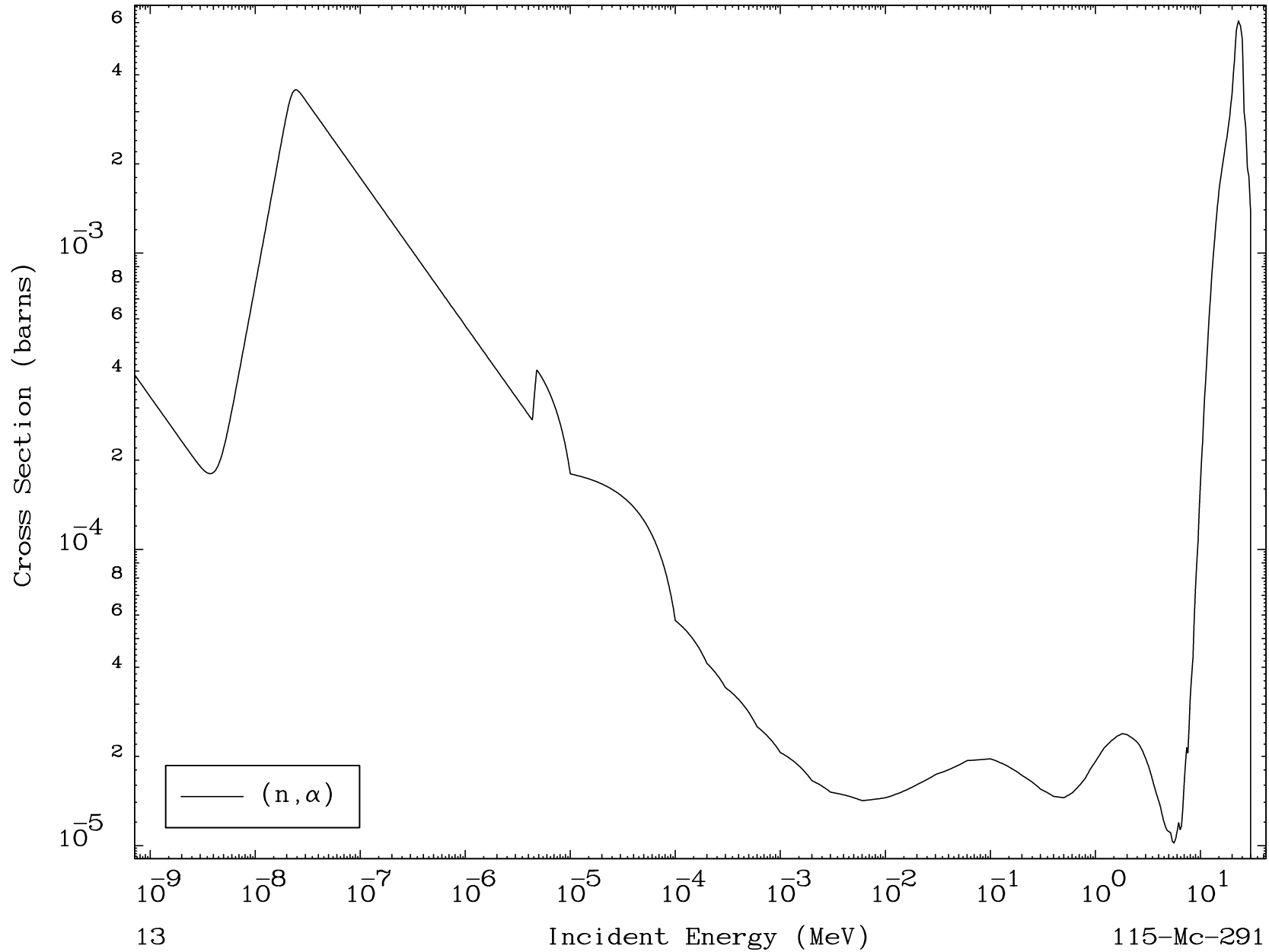
(n,d) Levels  
294 Kelvin Cross Sections

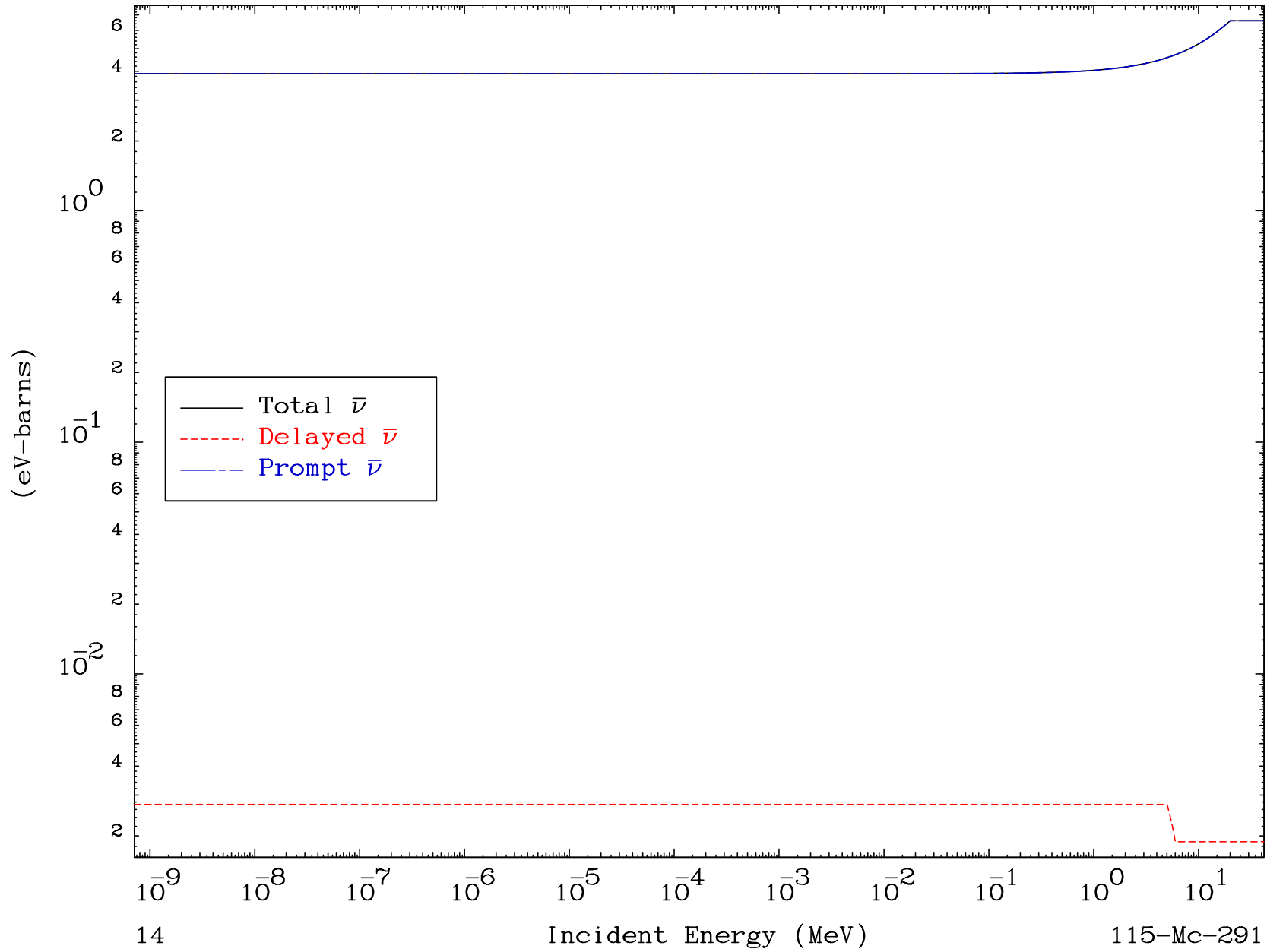
115-Mc-291

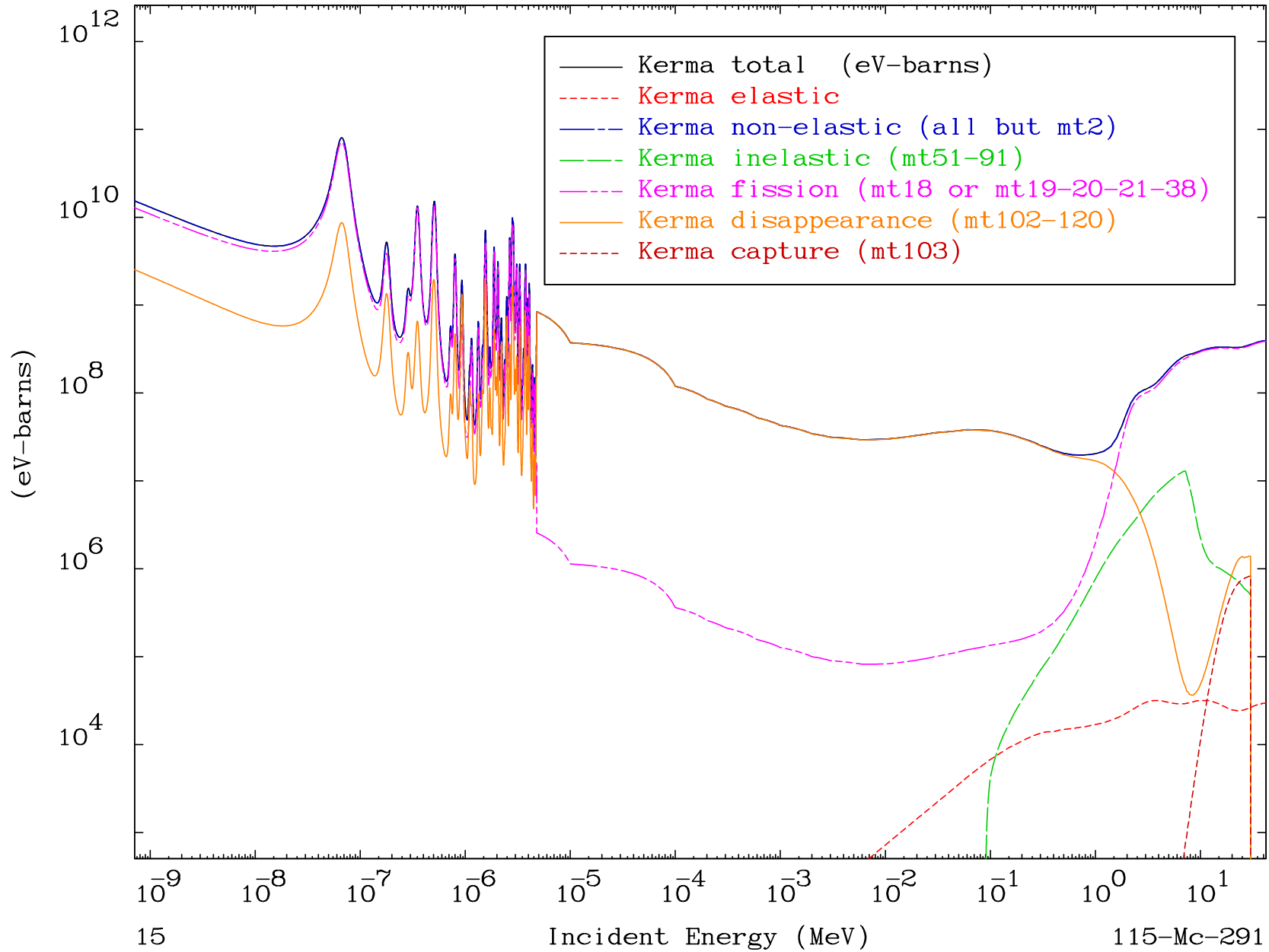


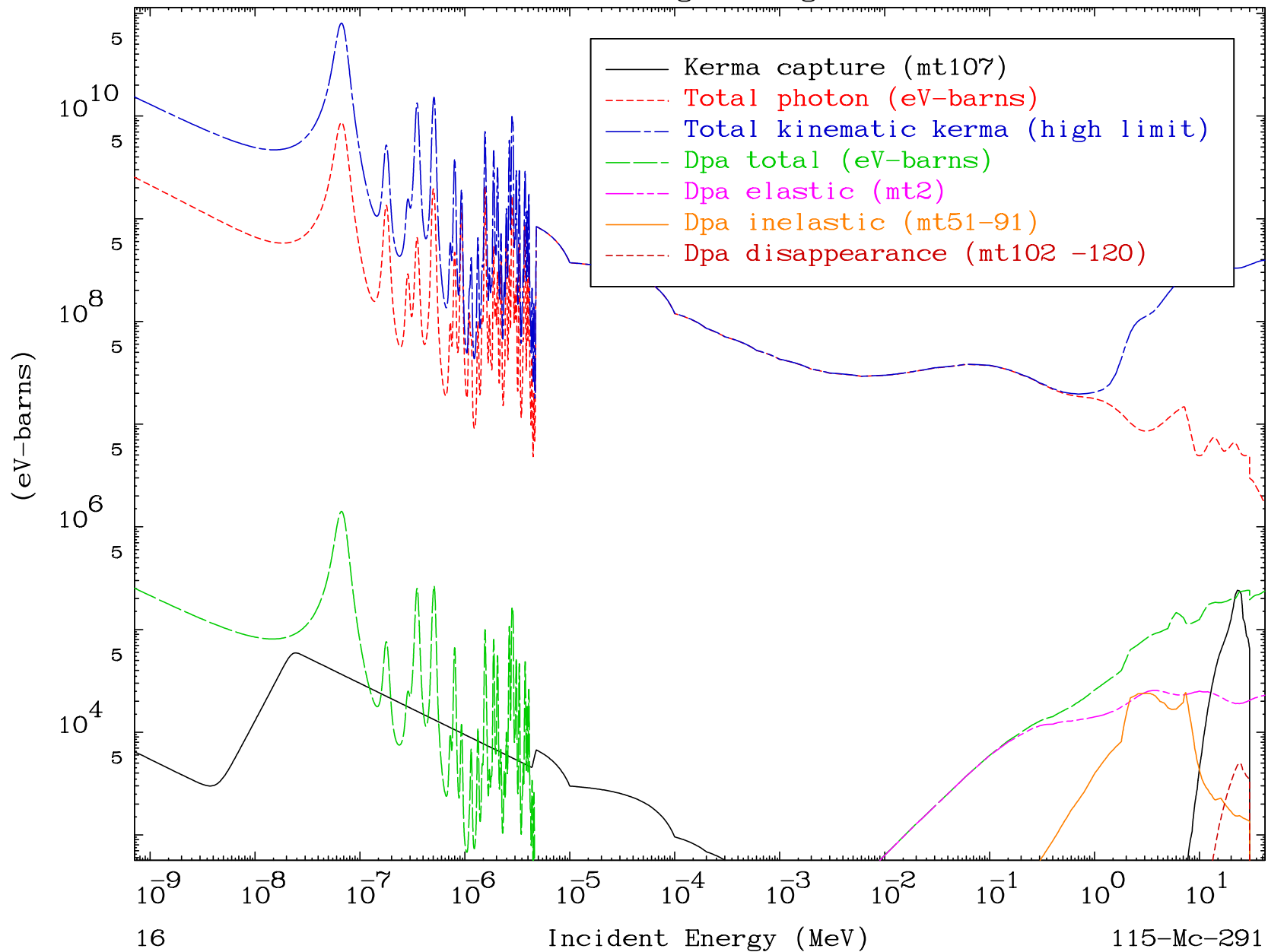






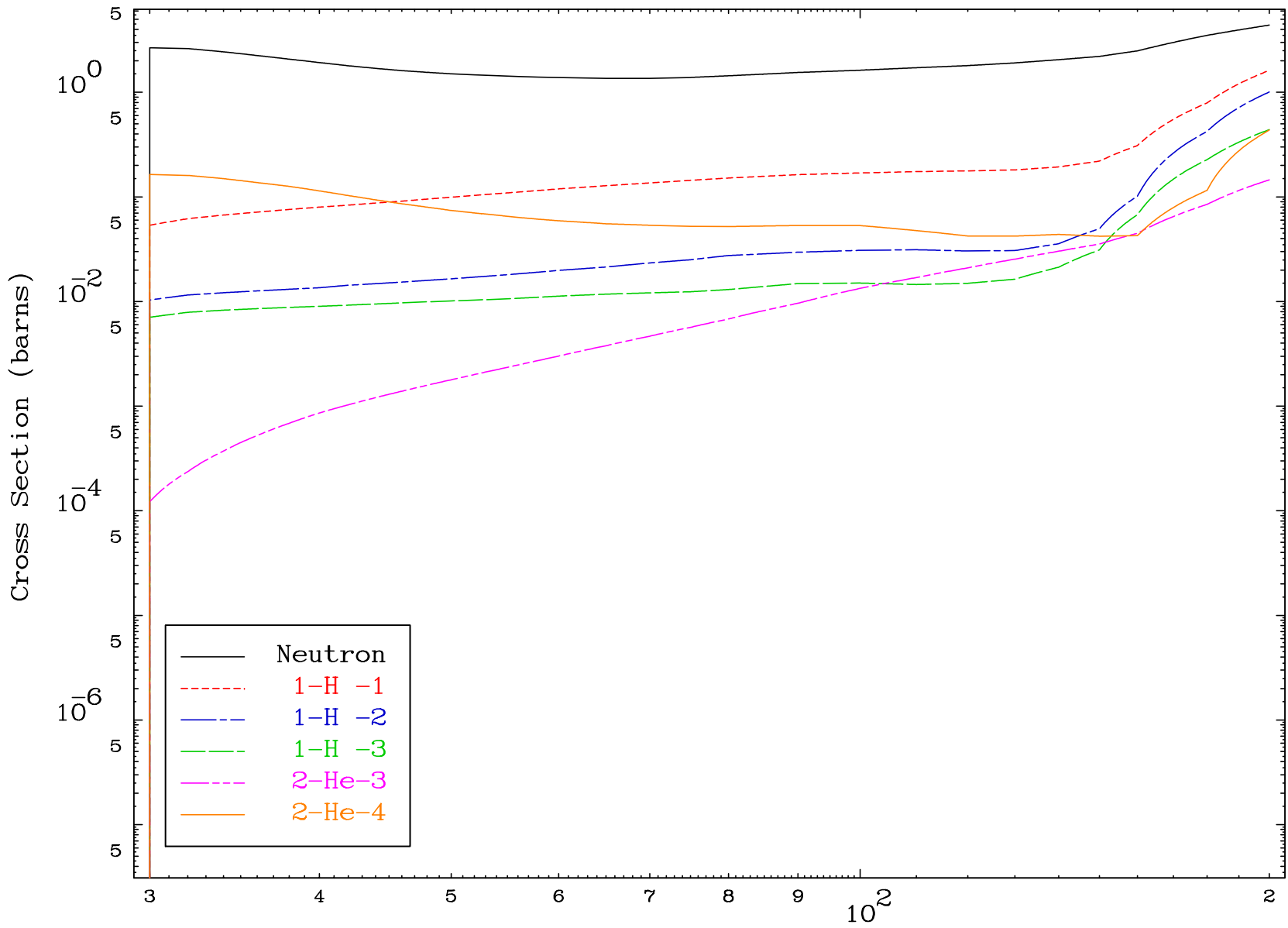


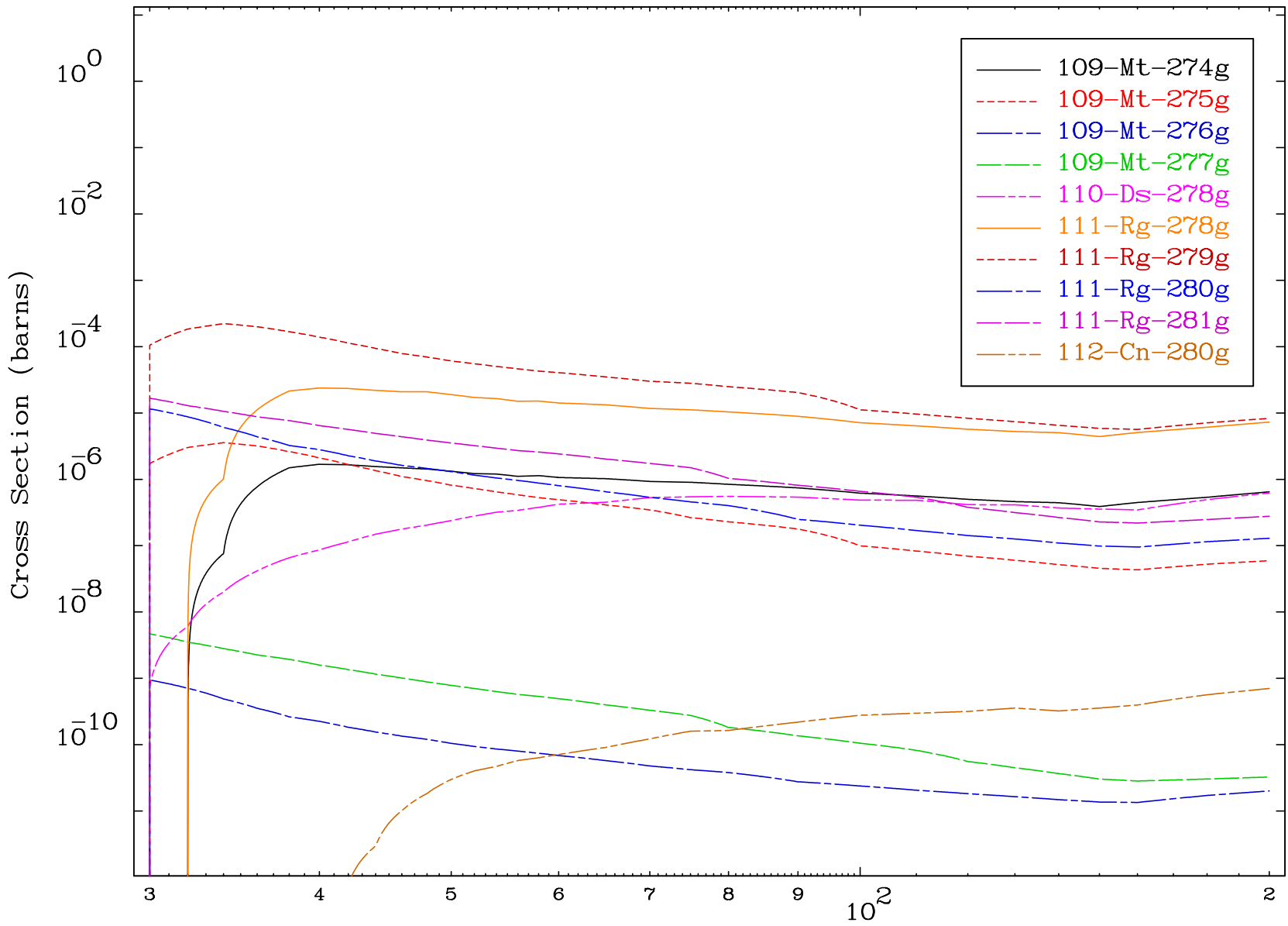


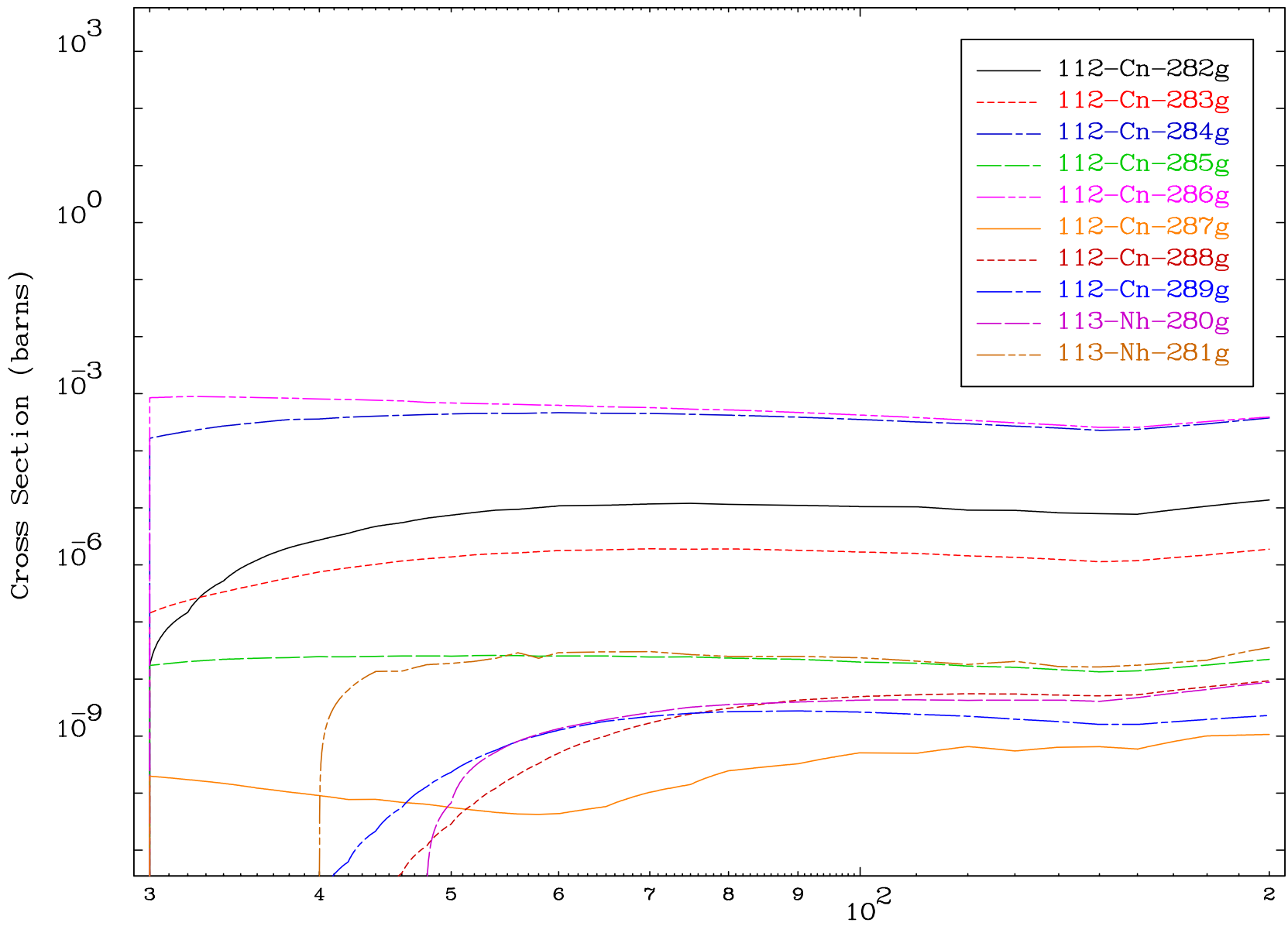




Radionuclide Production Cross Section





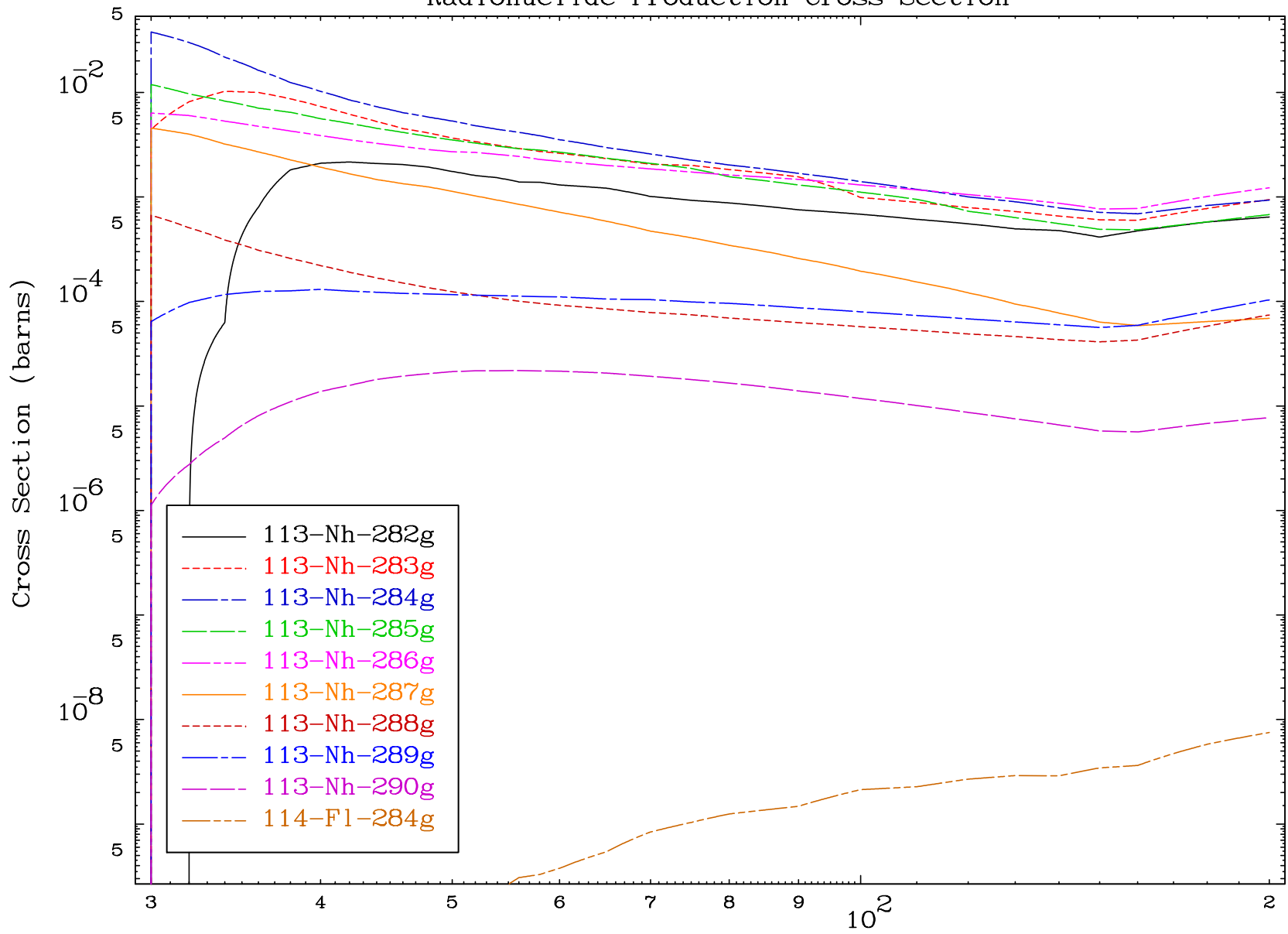


MAT 1591

(n,remainder)

115-Mc-291

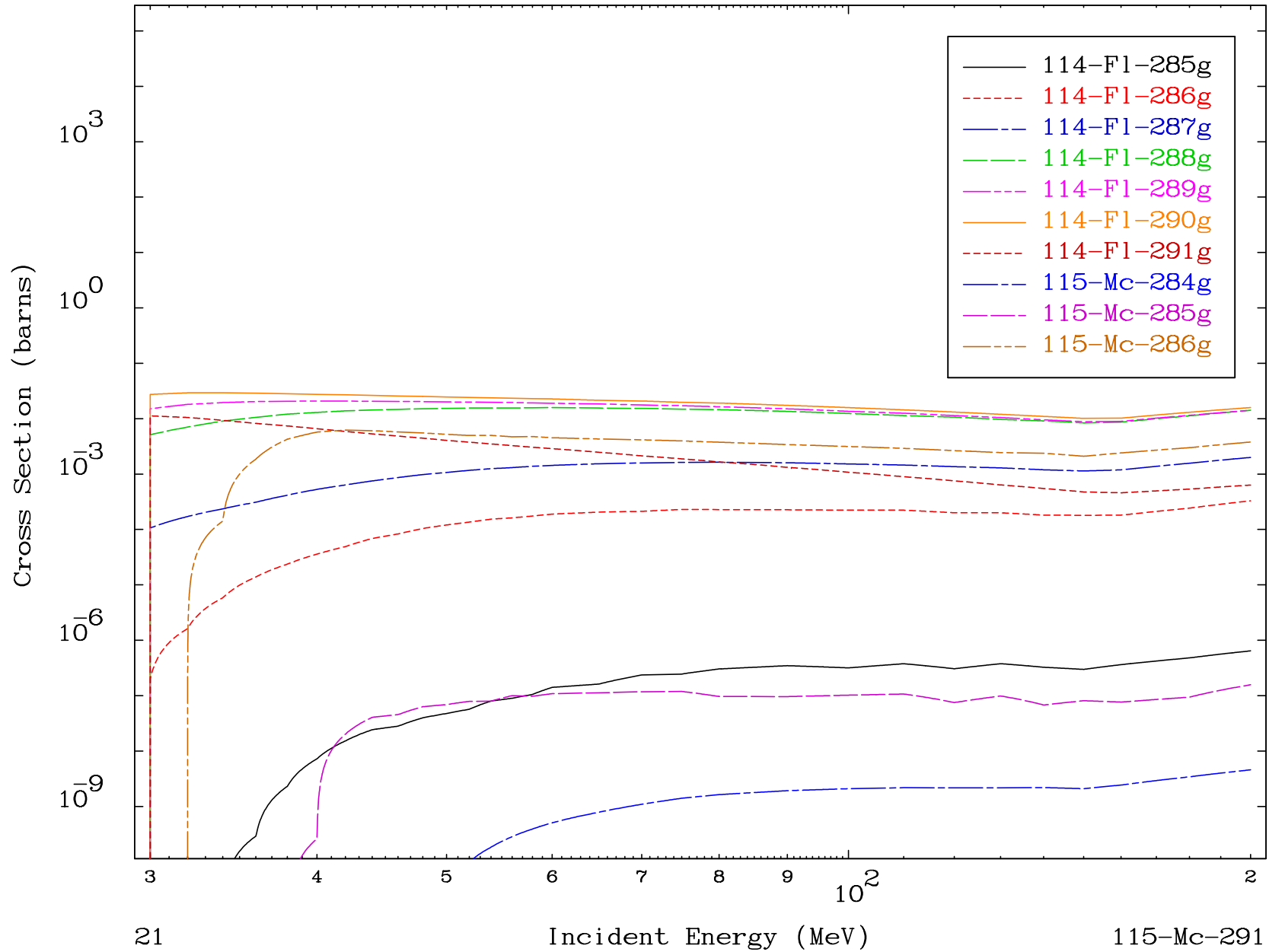
### Radionuclide Production Cross Section



20

Incident Energy (MeV)

115-Mc-291

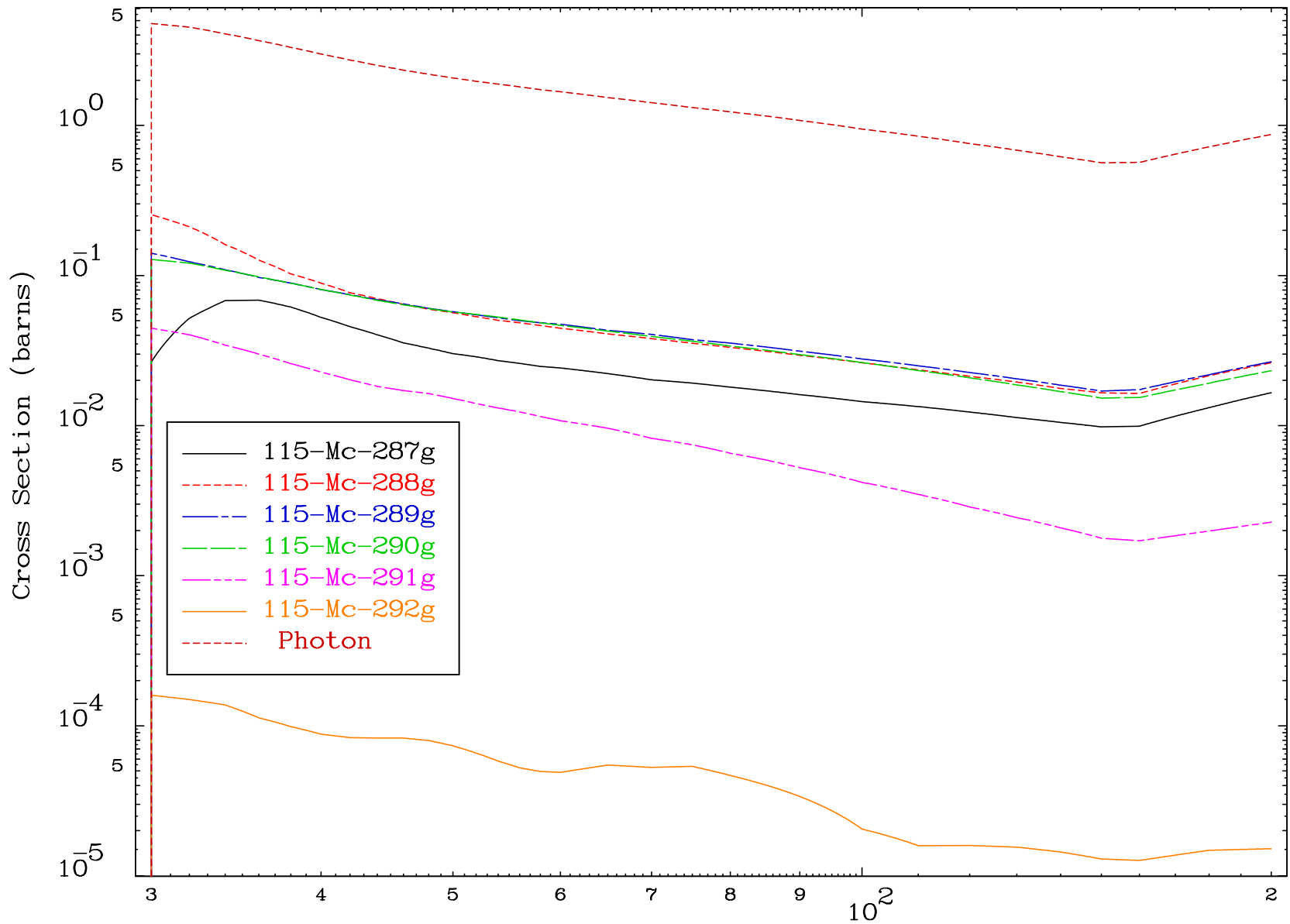


MAT 1591

(n,remainder)

115-Mc-291

### Radionuclide Production Cross Section



22

Incident Energy (MeV)

115-Mc-291