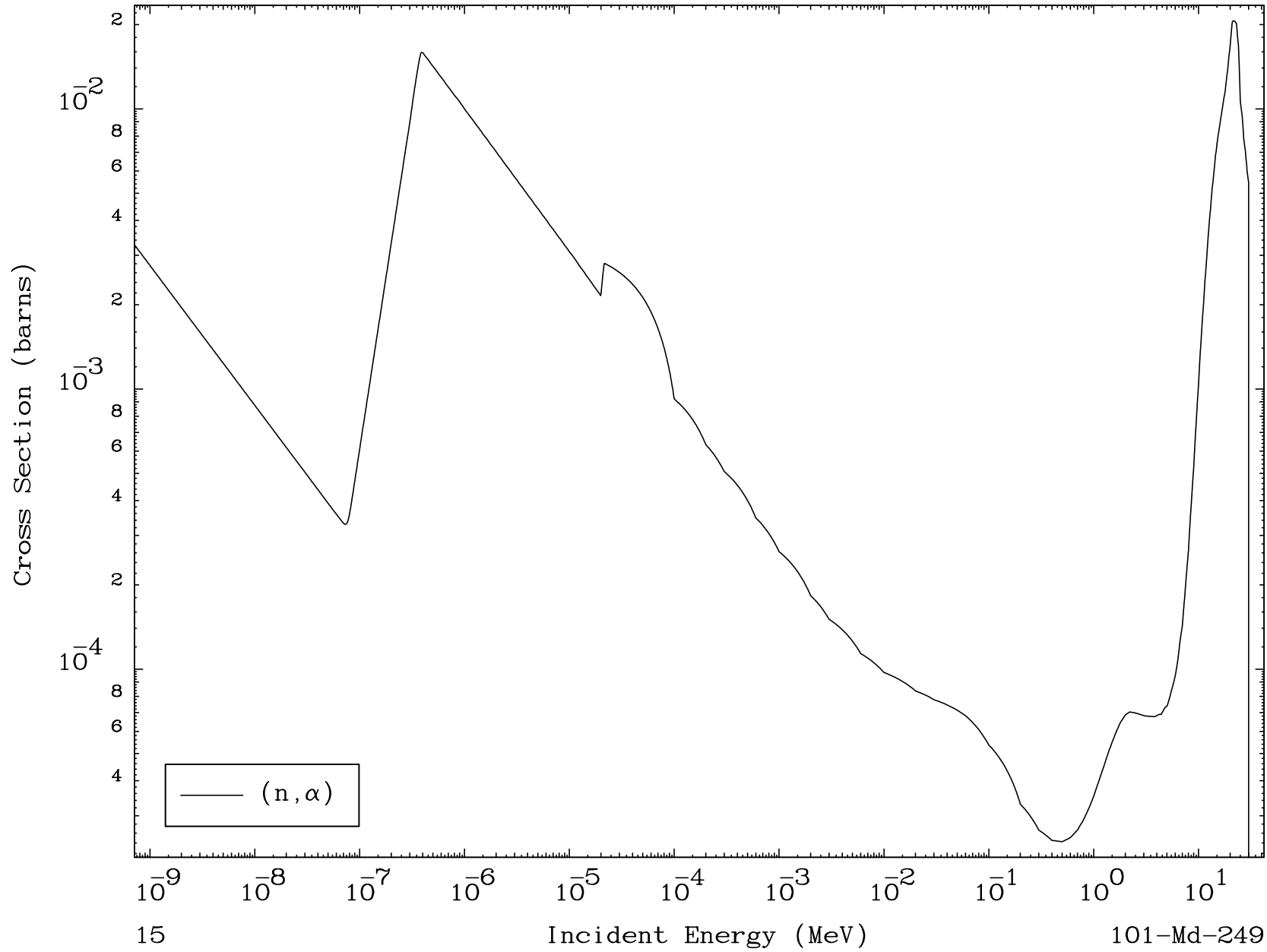
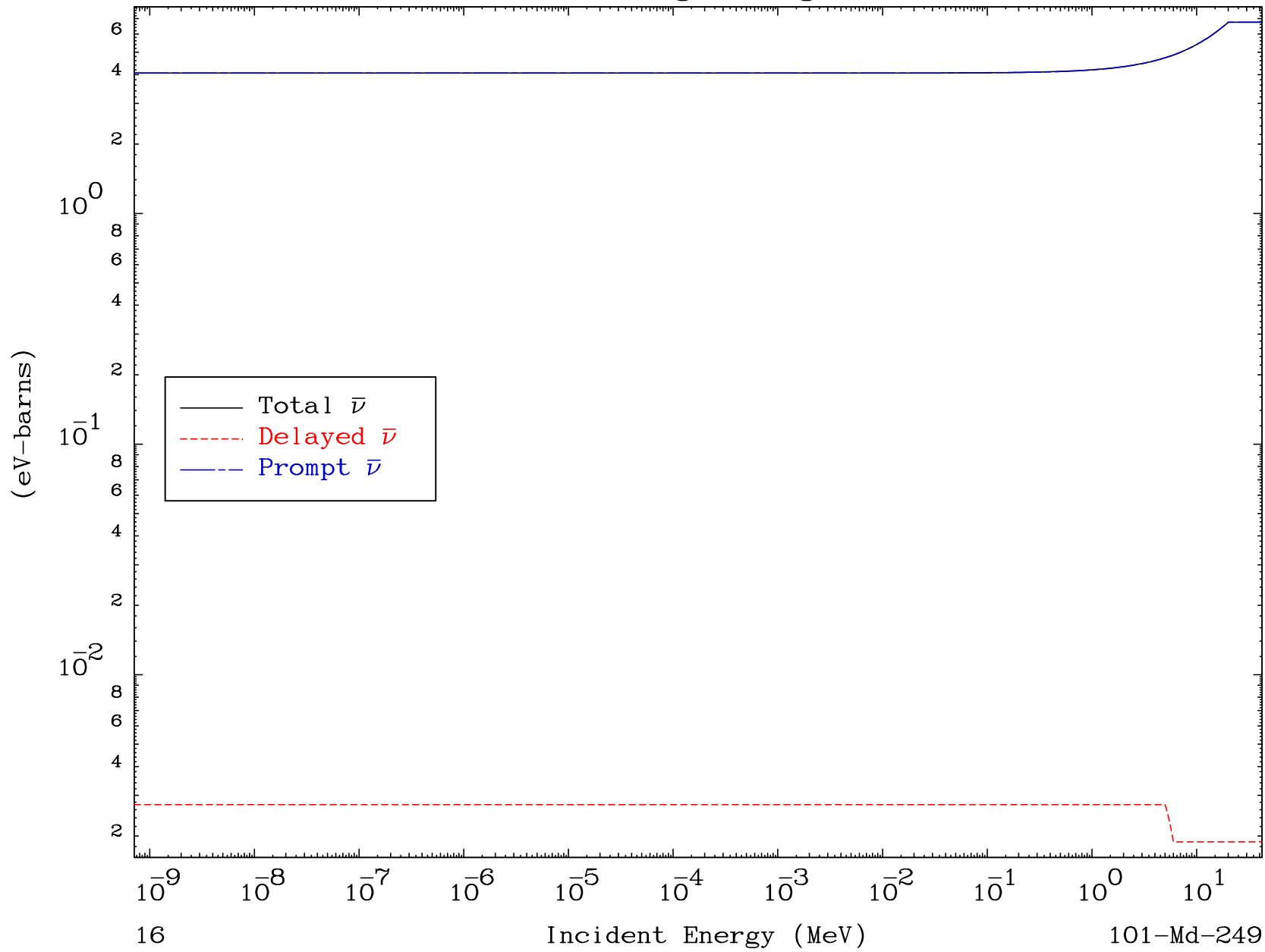


MAT 9955

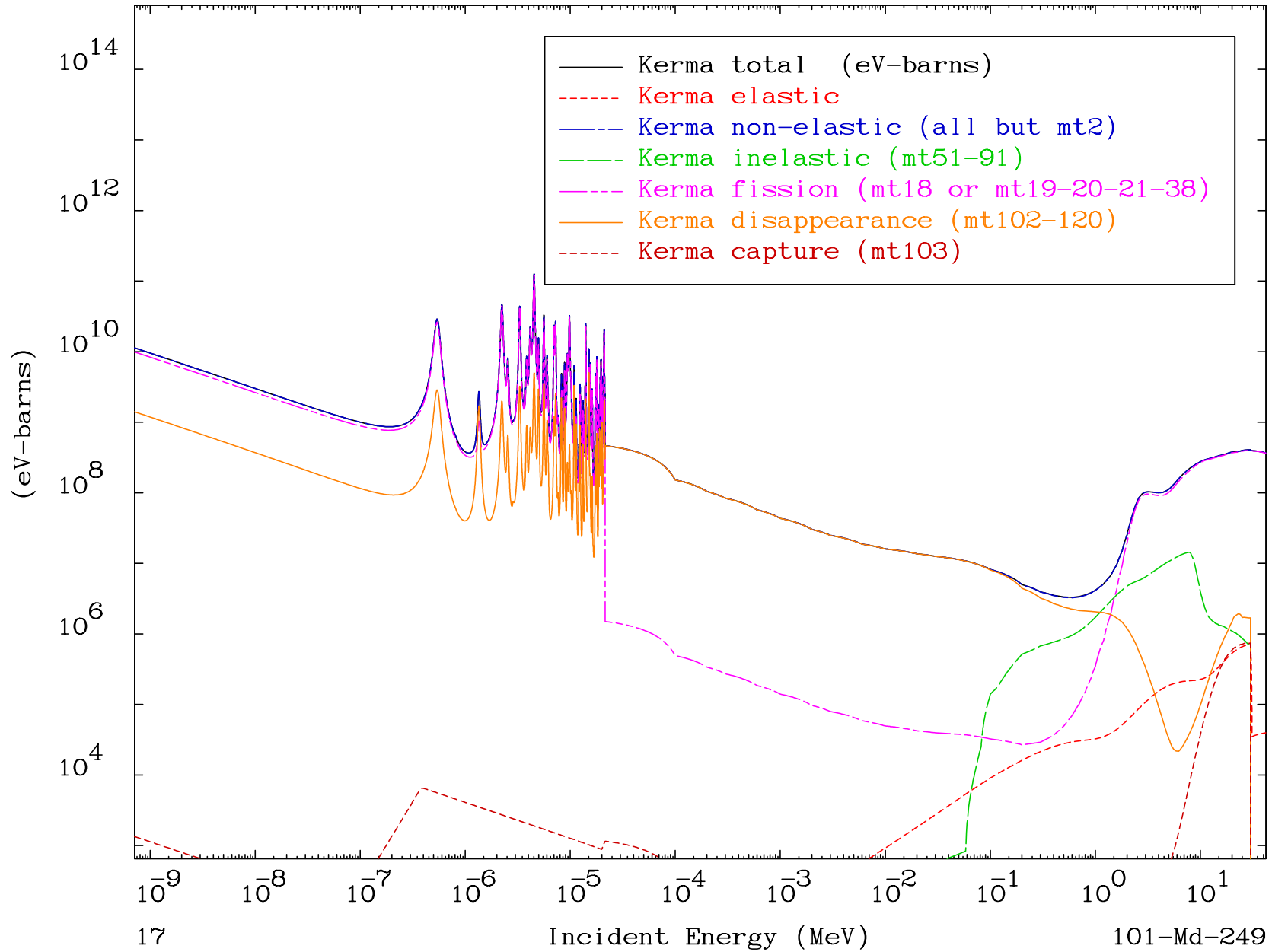
(n,  $\alpha$ ) Levels  
294 Kelvin Cross Sections

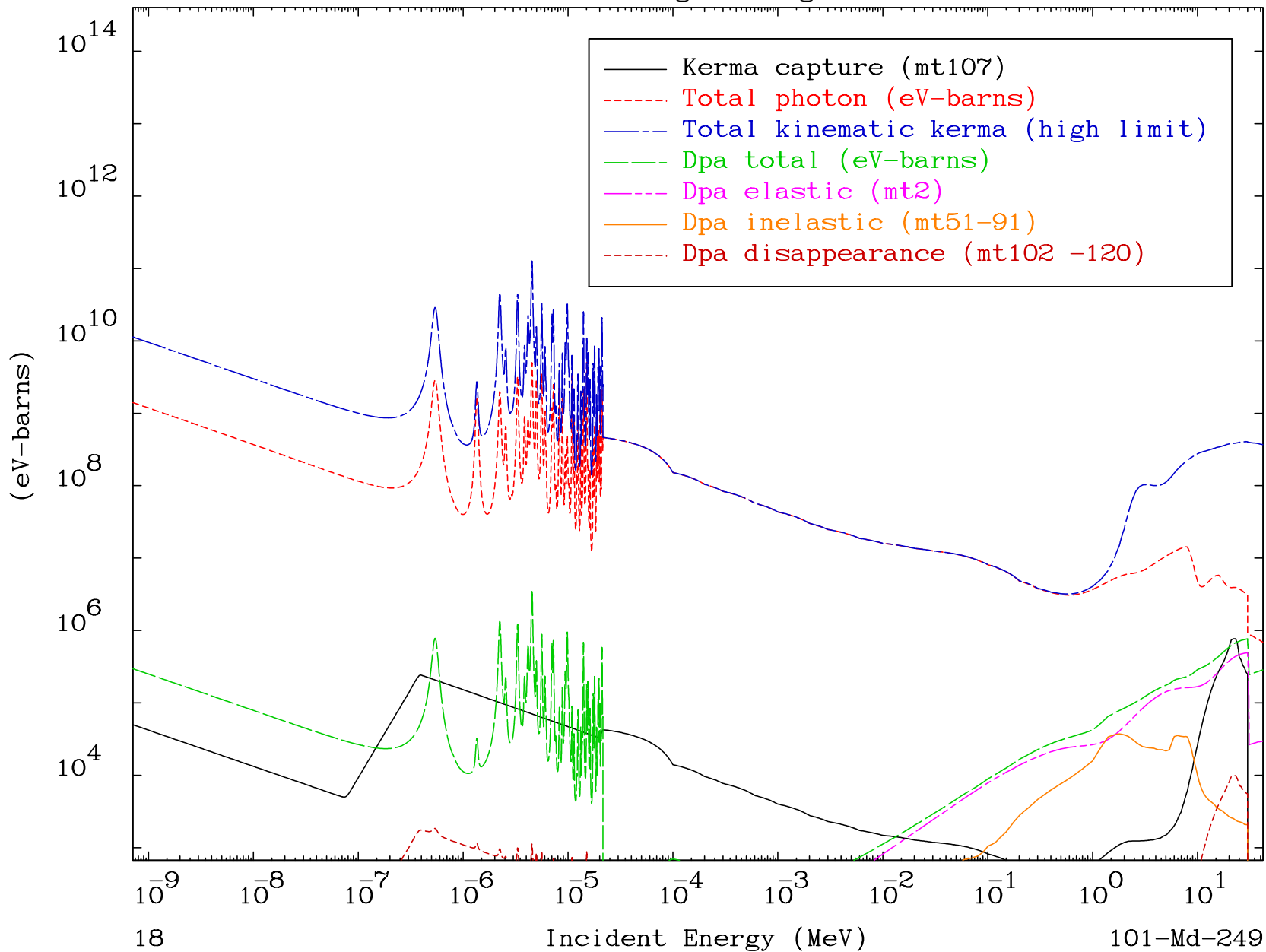
101-Md-249







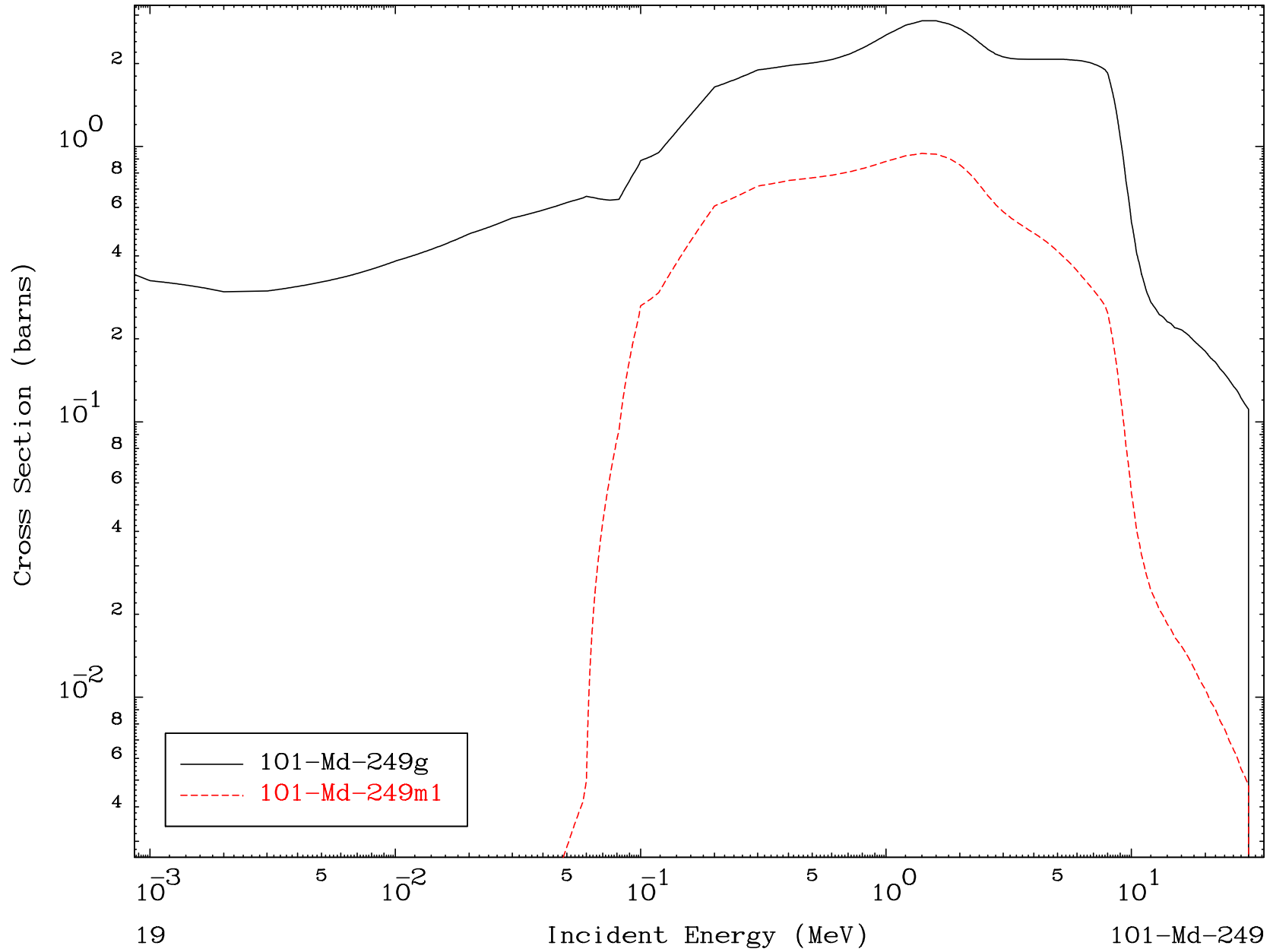




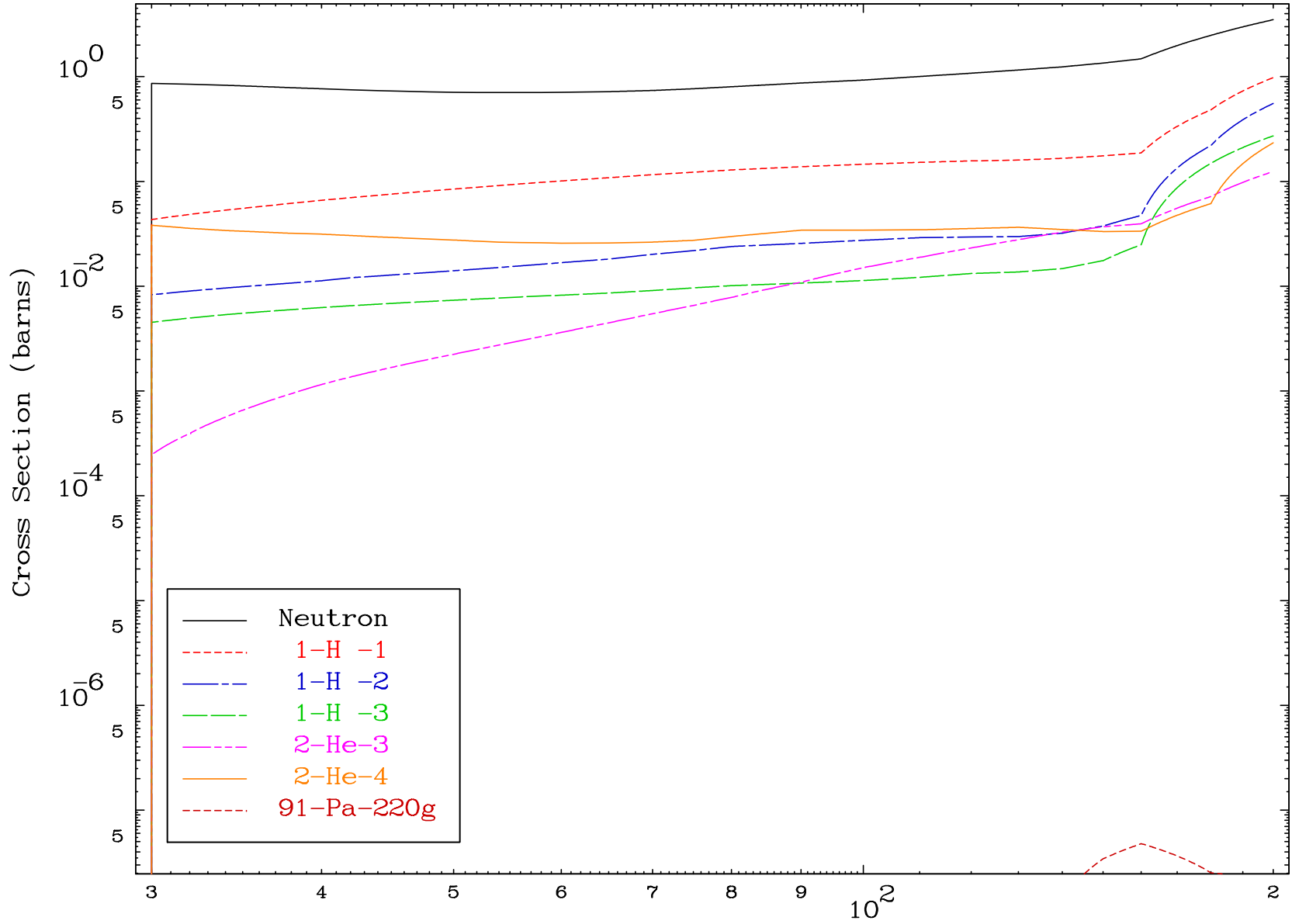
MAT 9955

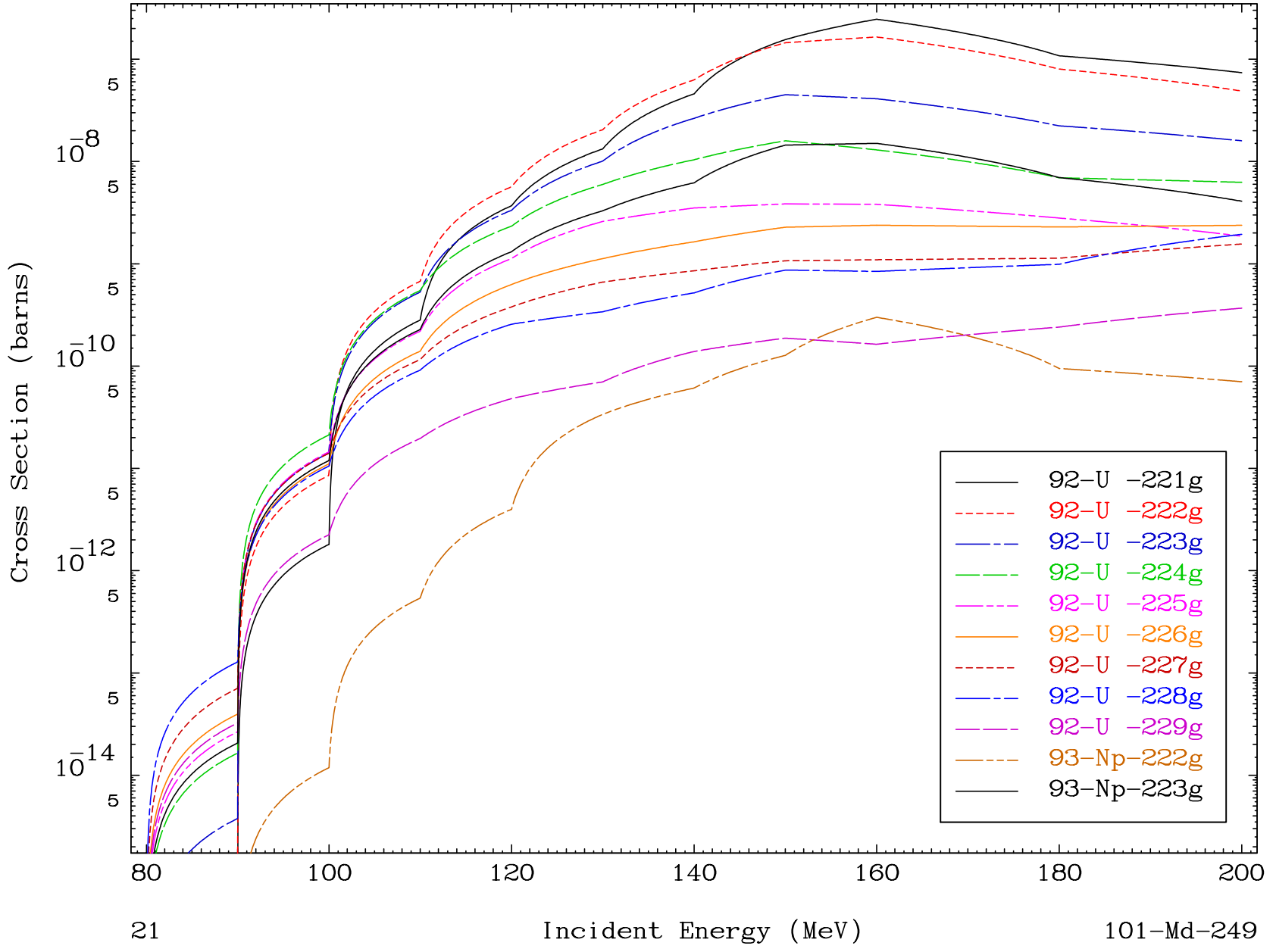
Inelastic  
Radionuclide Production Cross Section

101-Md-249

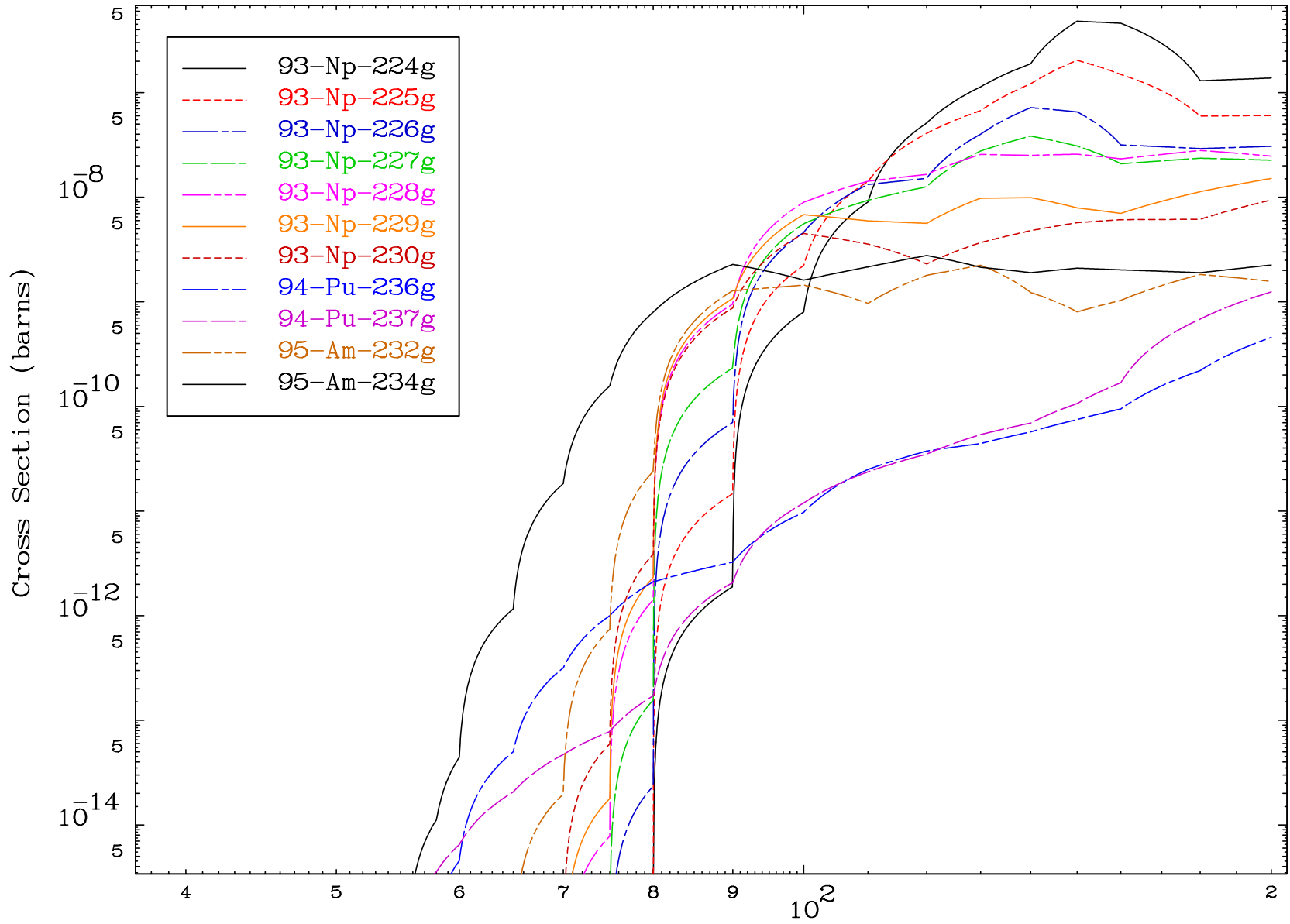


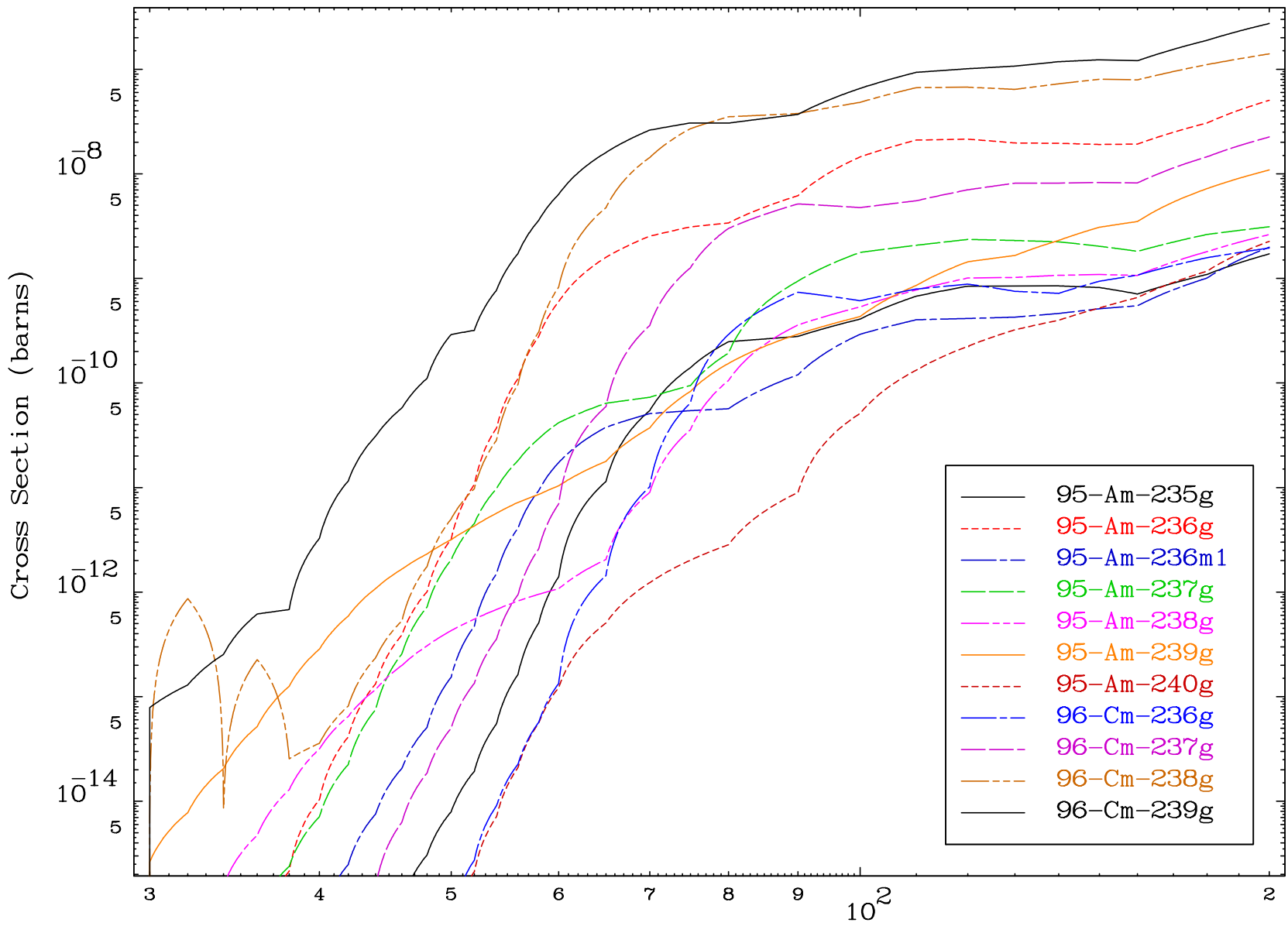
Radionuclide Production Cross Section





Radionuclide Production Cross Section



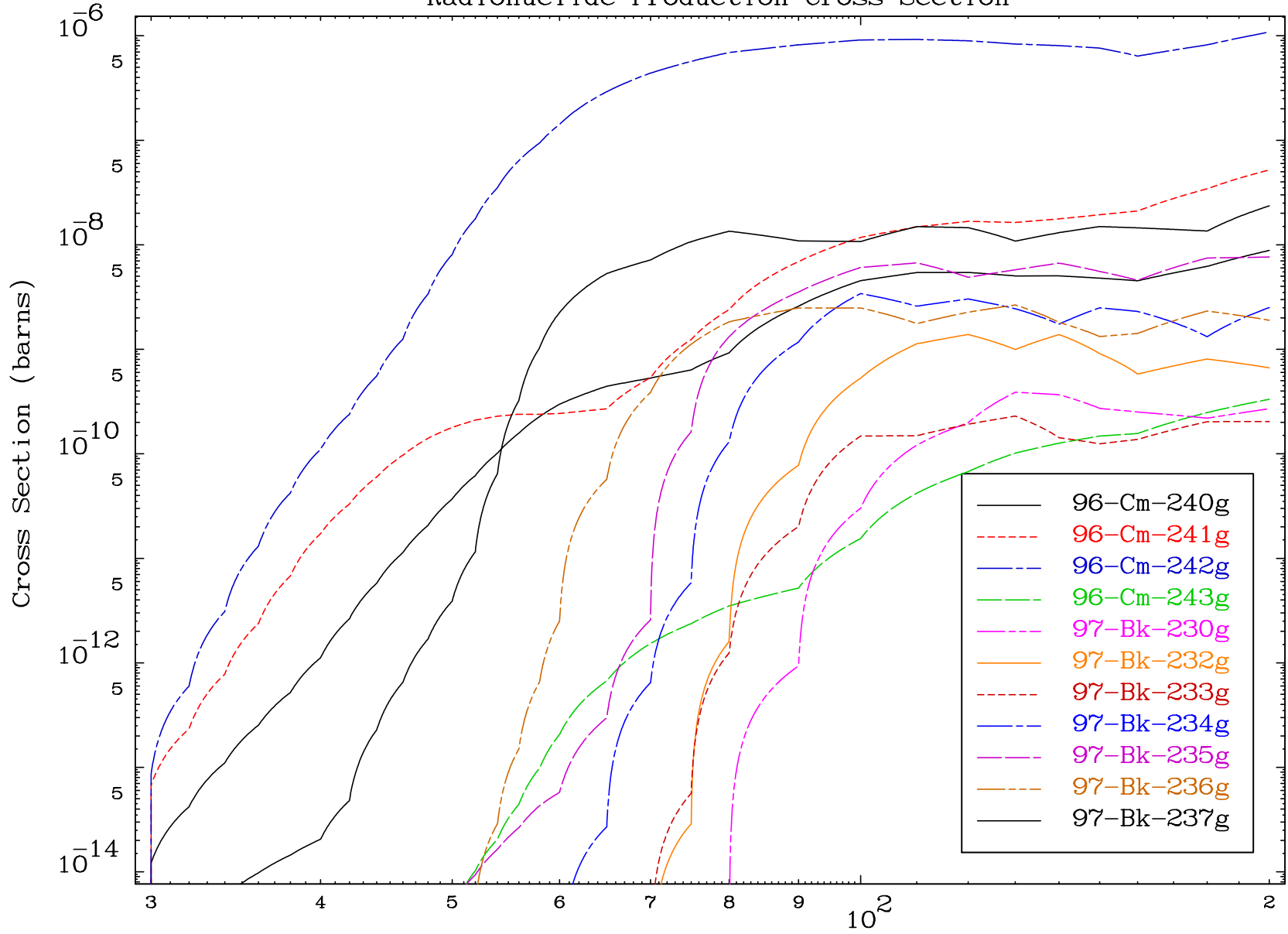


MAT 9955

(n,remainder)

101-Md-249

### Radionuclide Production Cross Section



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Incident Energy (MeV)

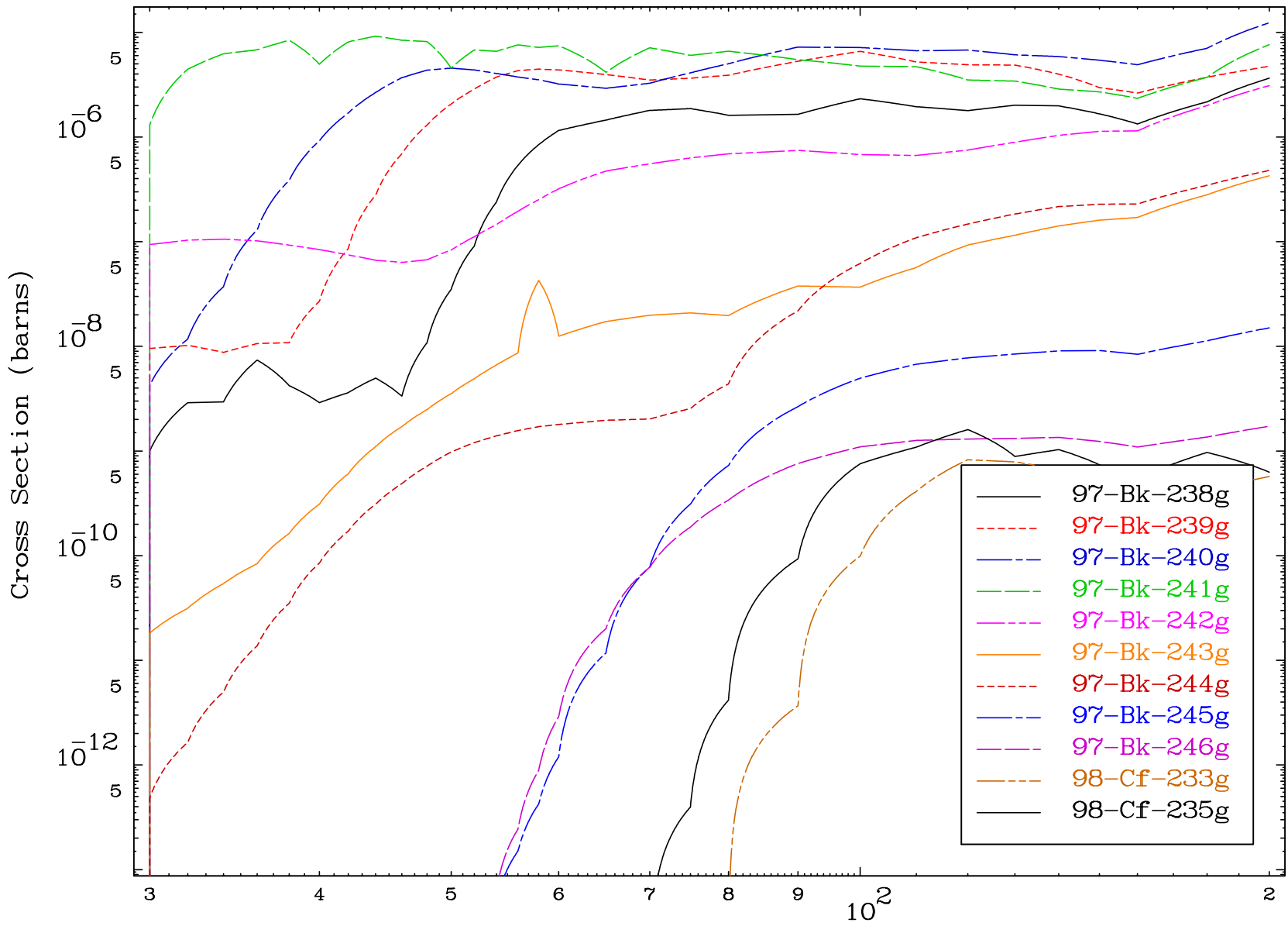
101-Md-249



MAT 9955

(n,remainder)  
Radionuclide Production Cross Section

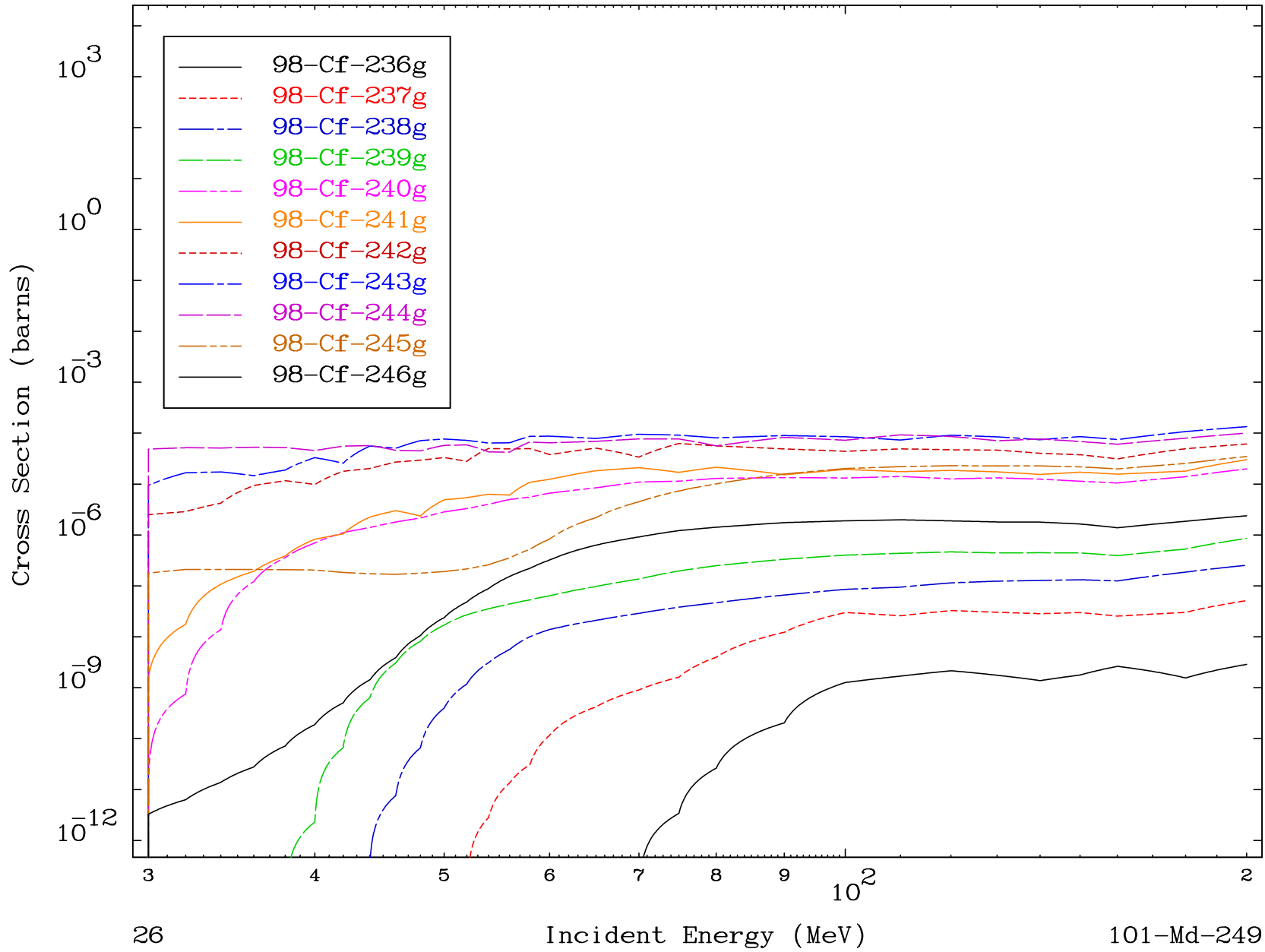
101-Md-249

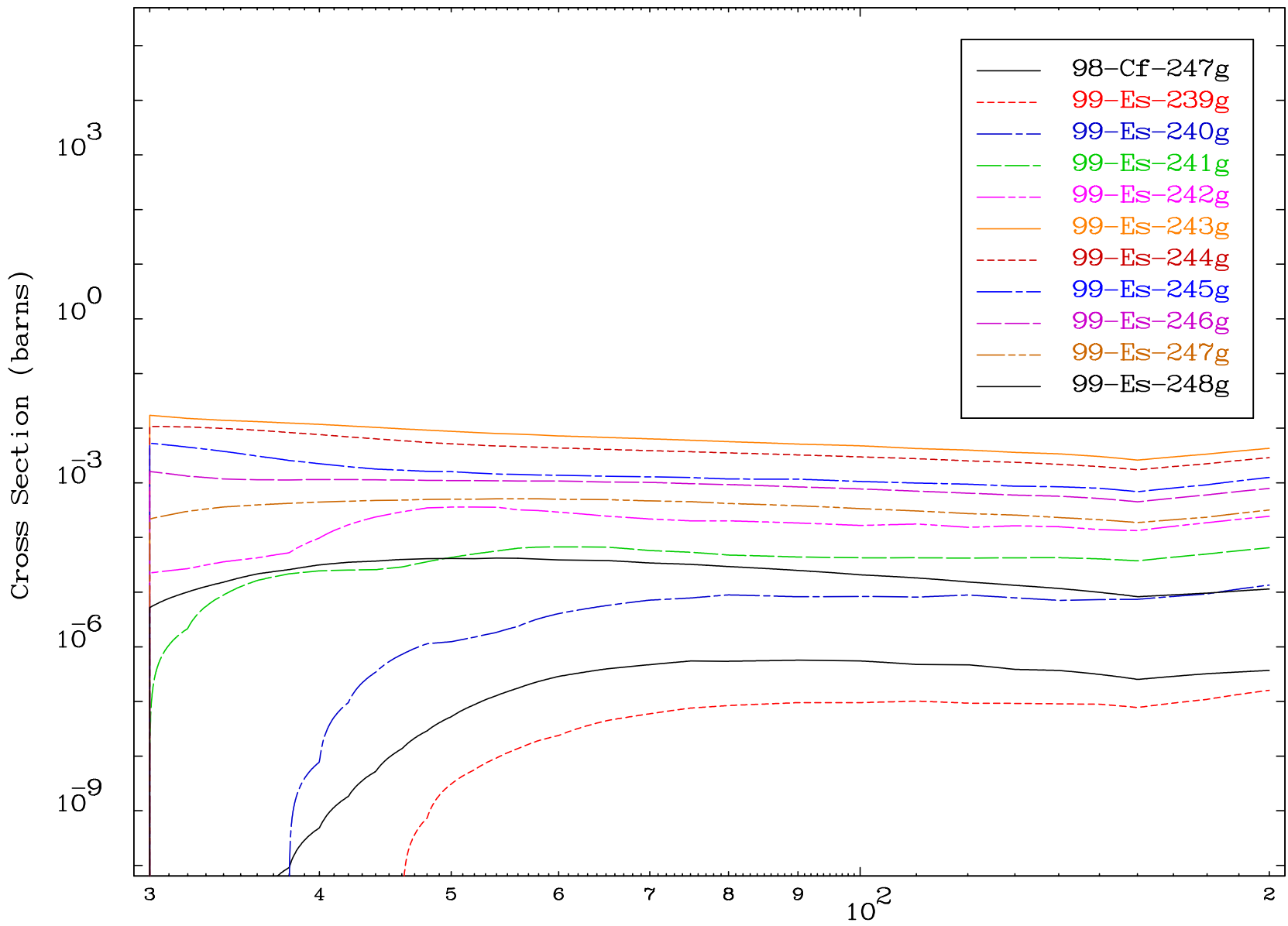


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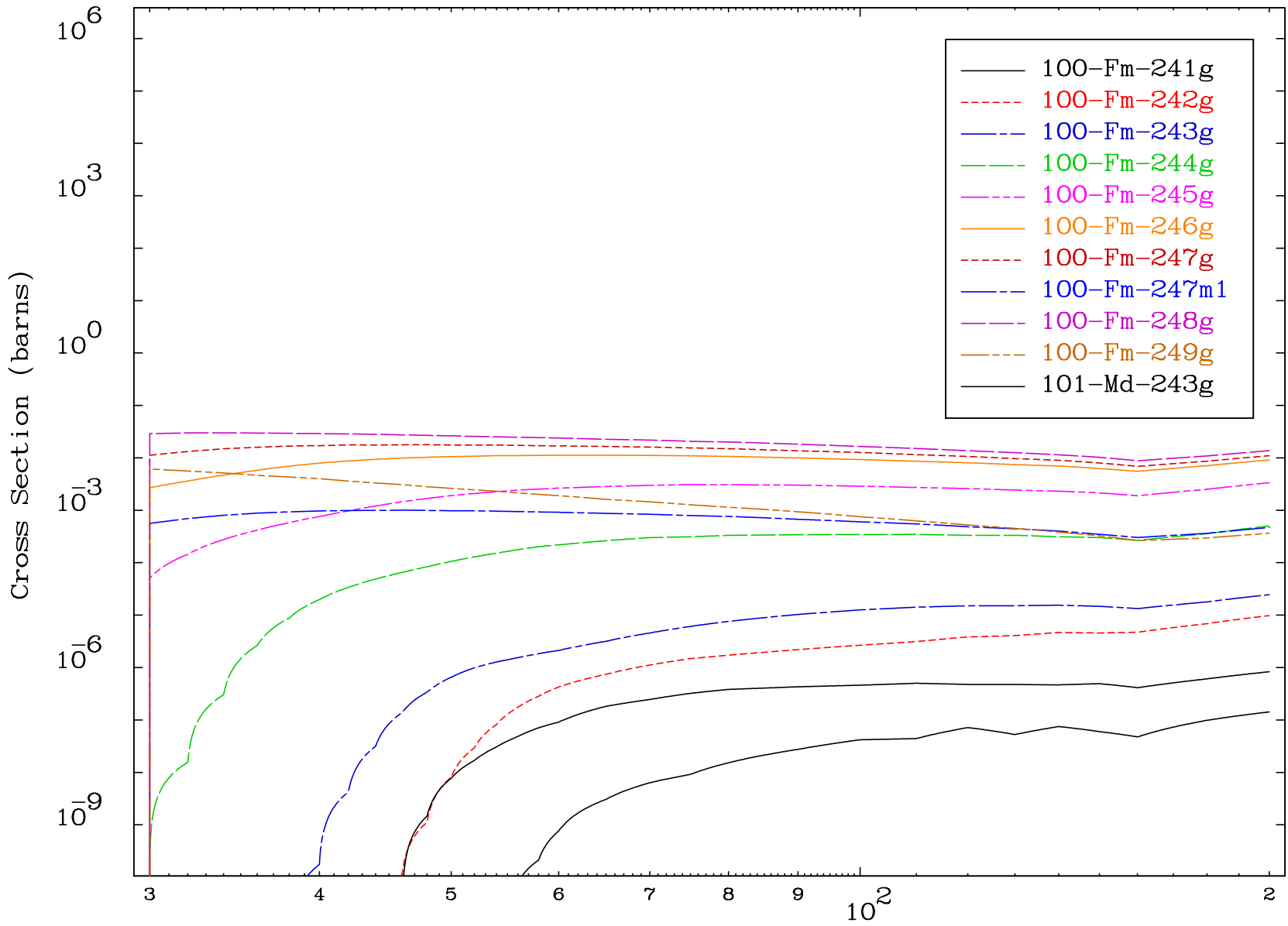
Incident Energy (MeV)

101-Md-249

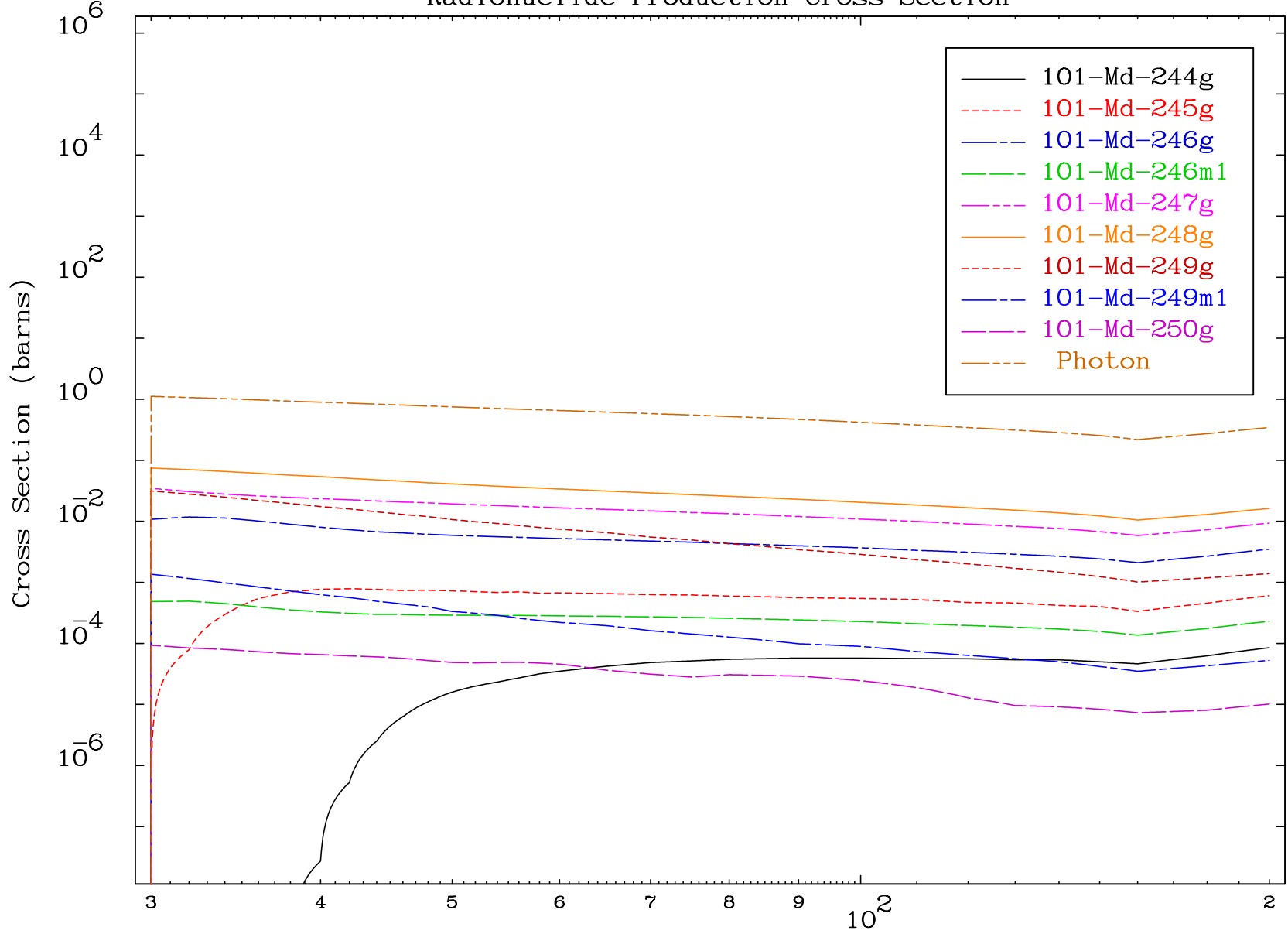




Radionuclide Production Cross Section



Radionuclide Production Cross Section

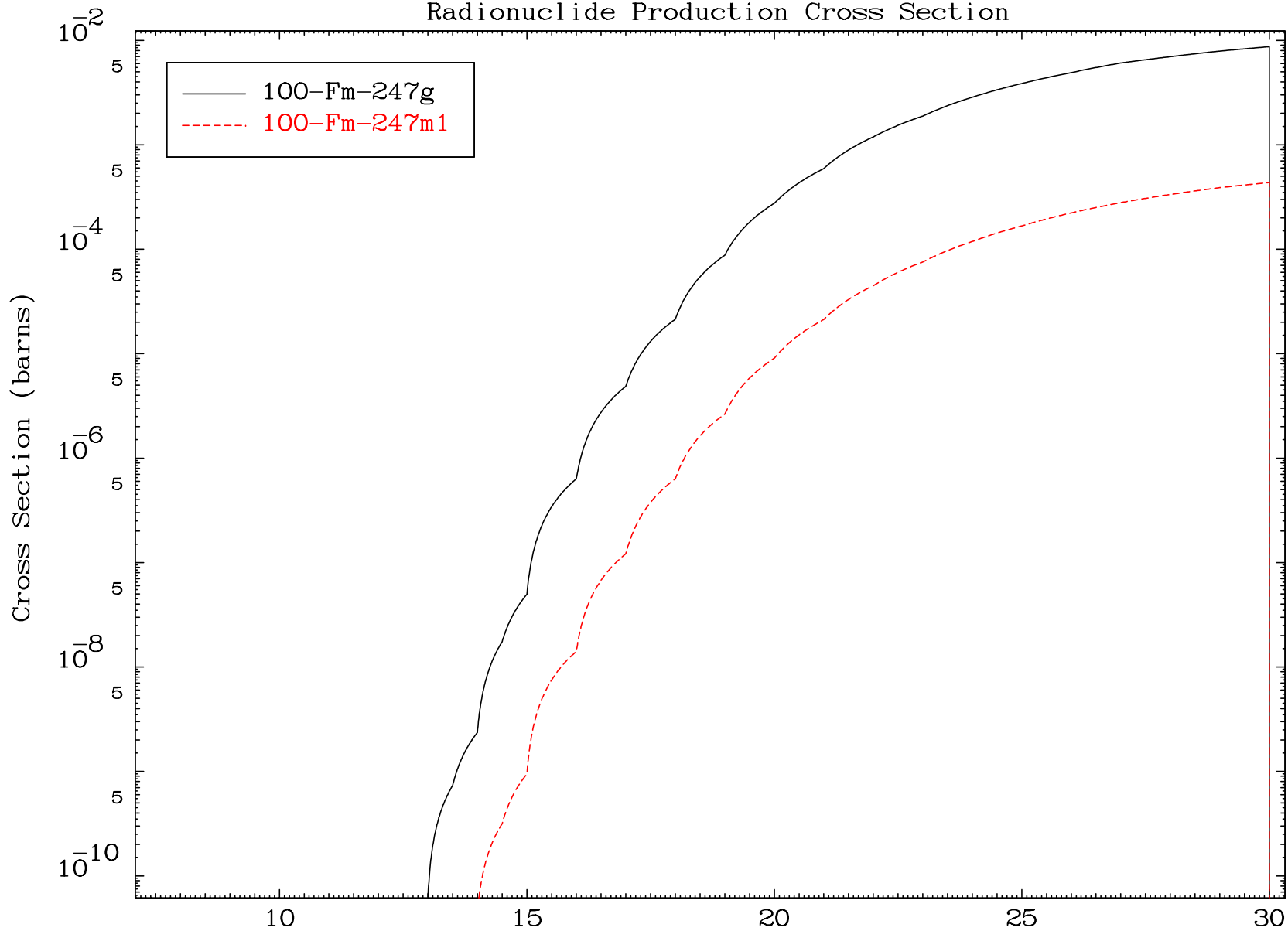


MAT 9955

(n,n') d

101-Md-249

Radionuclide Production Cross Section



30

Incident Energy (MeV)

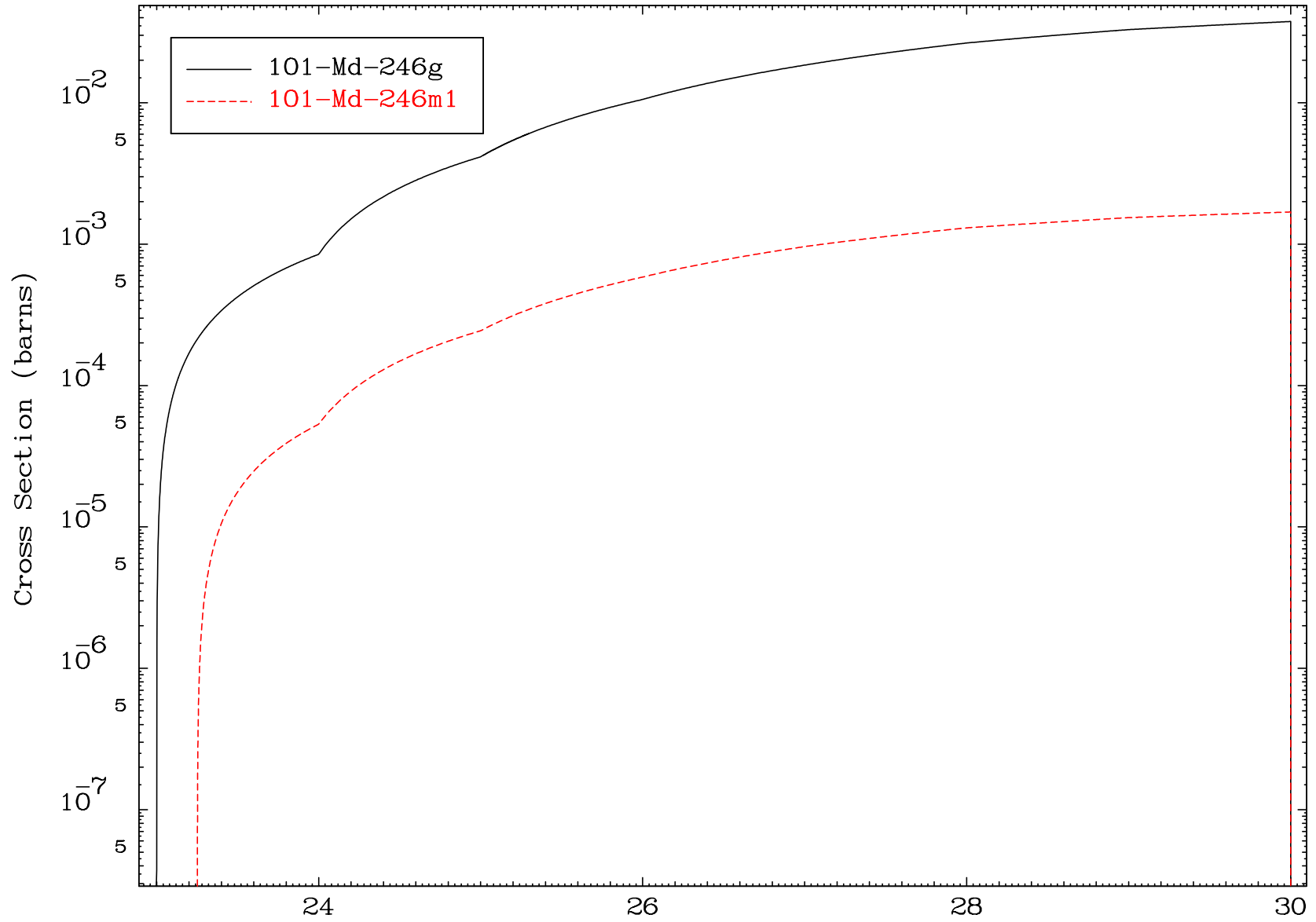
101-Md-249

MAT 9955

(n,4n)

101-Md-249

Radionuclide Production Cross Section

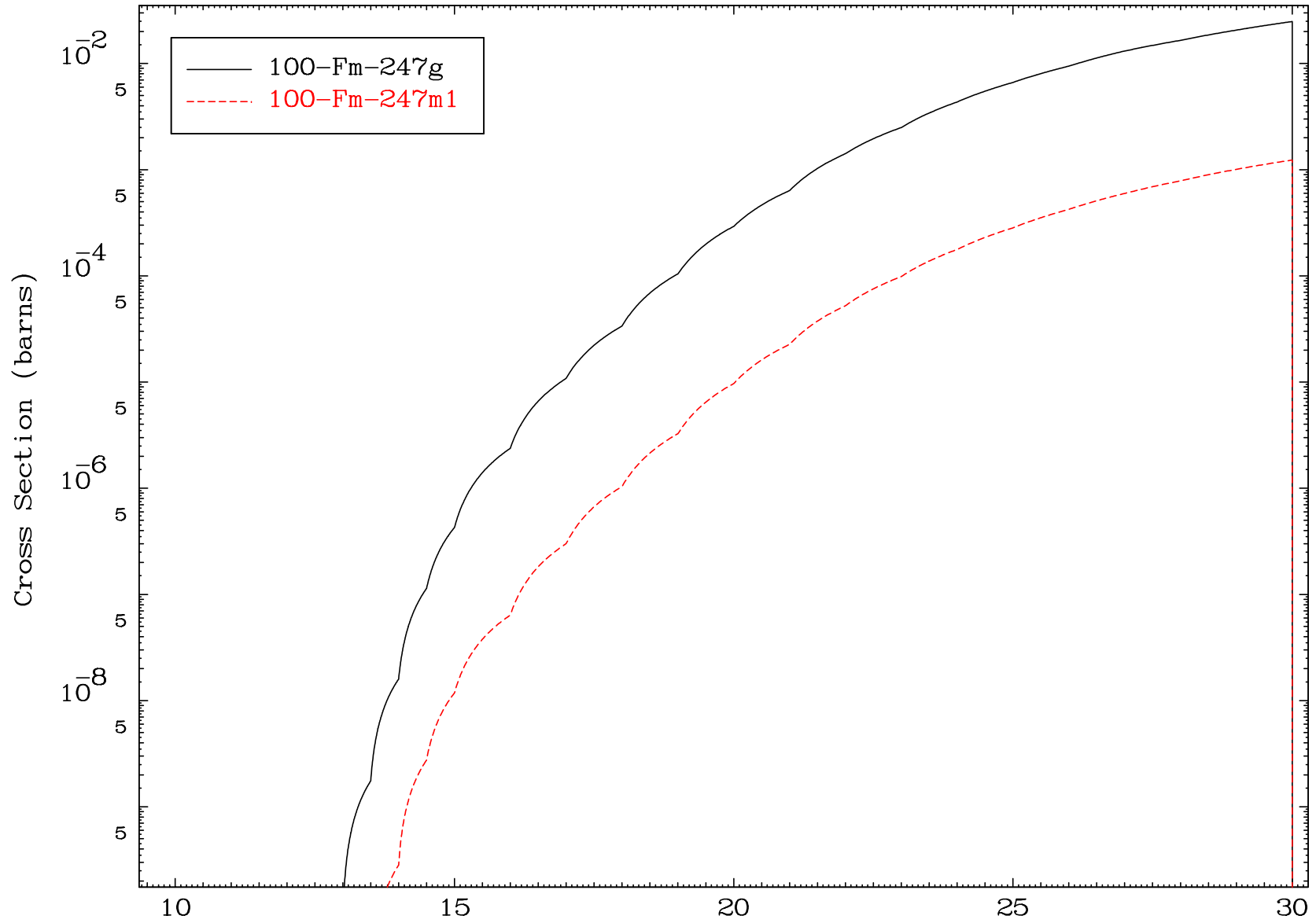


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Incident Energy (MeV)

101-Md-249

Radionuclide Production Cross Section





Radionuclide Production Cross Section

