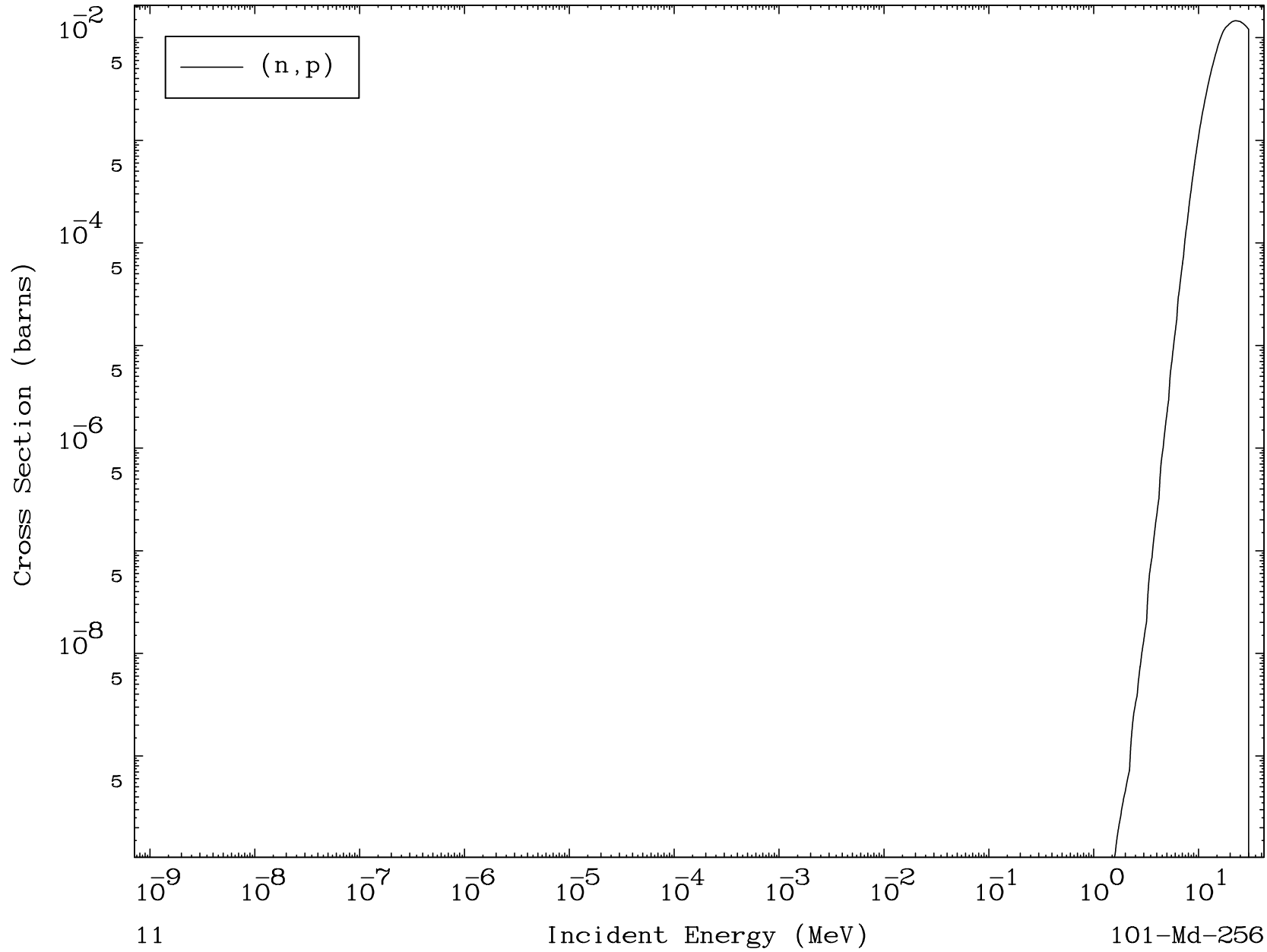


MAT 9957

(n,p) Levels
294 Kelvin Cross Sections

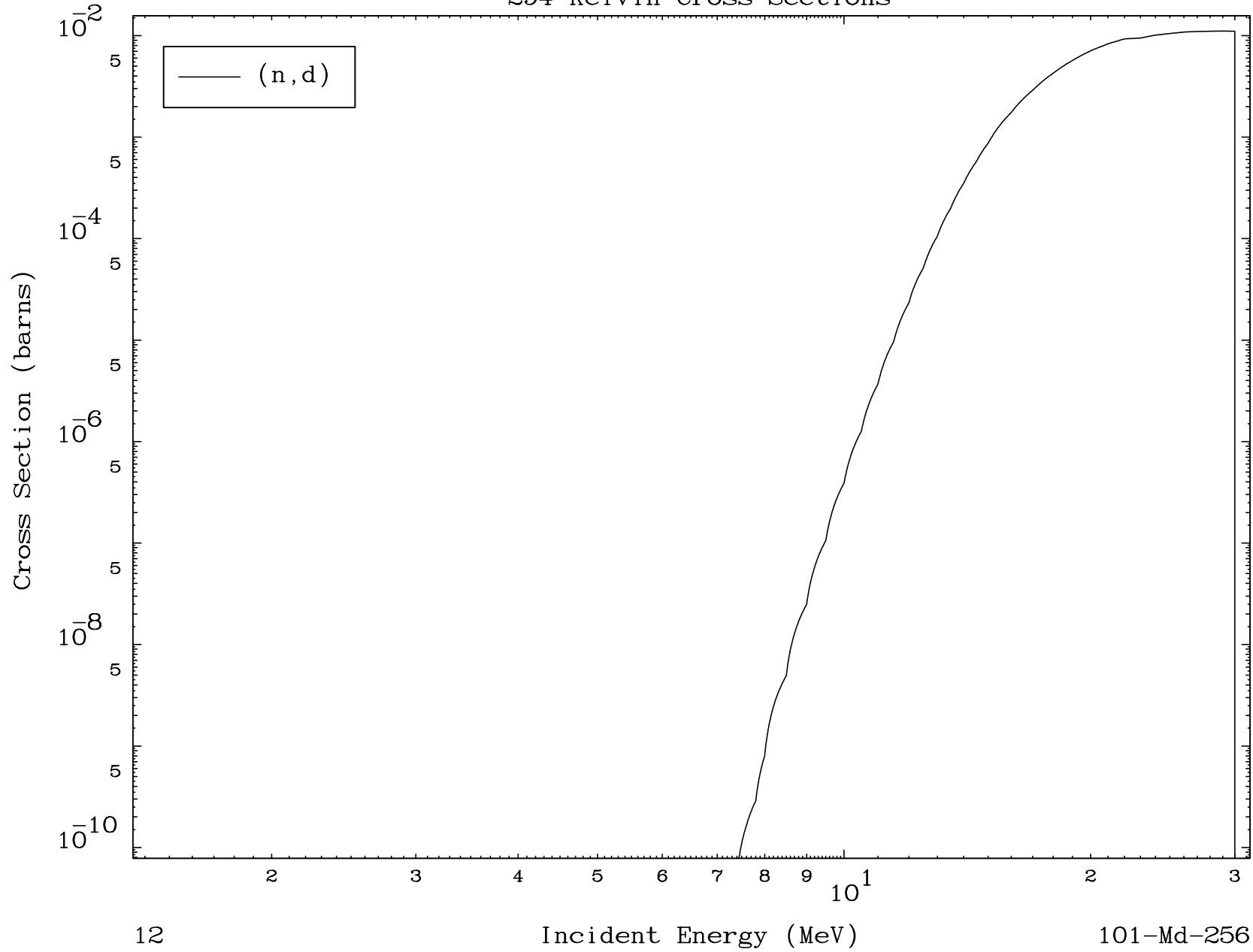
101-Md-256



MAT 9957

(n,d) Levels
294 Kelvin Cross Sections

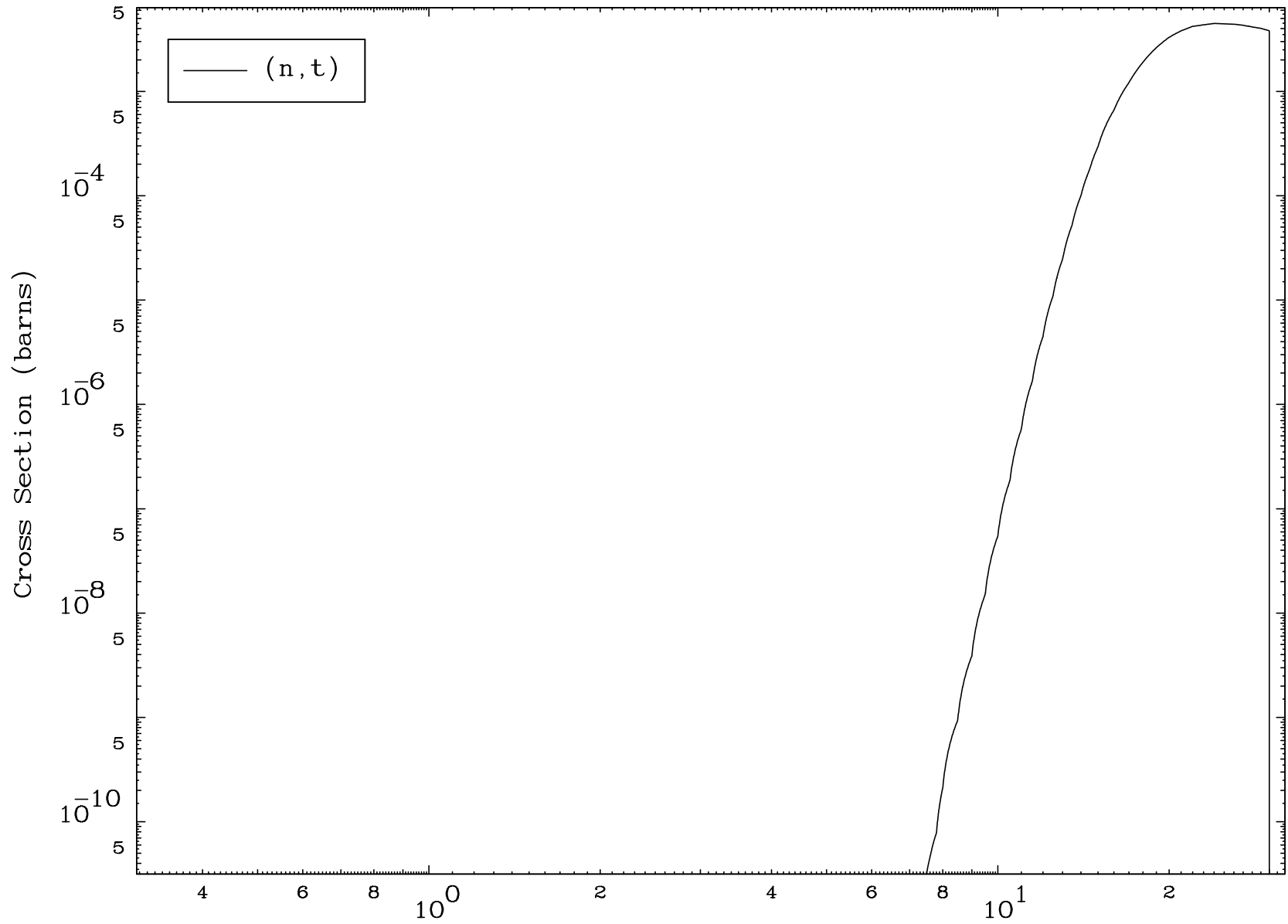
101-Md-256

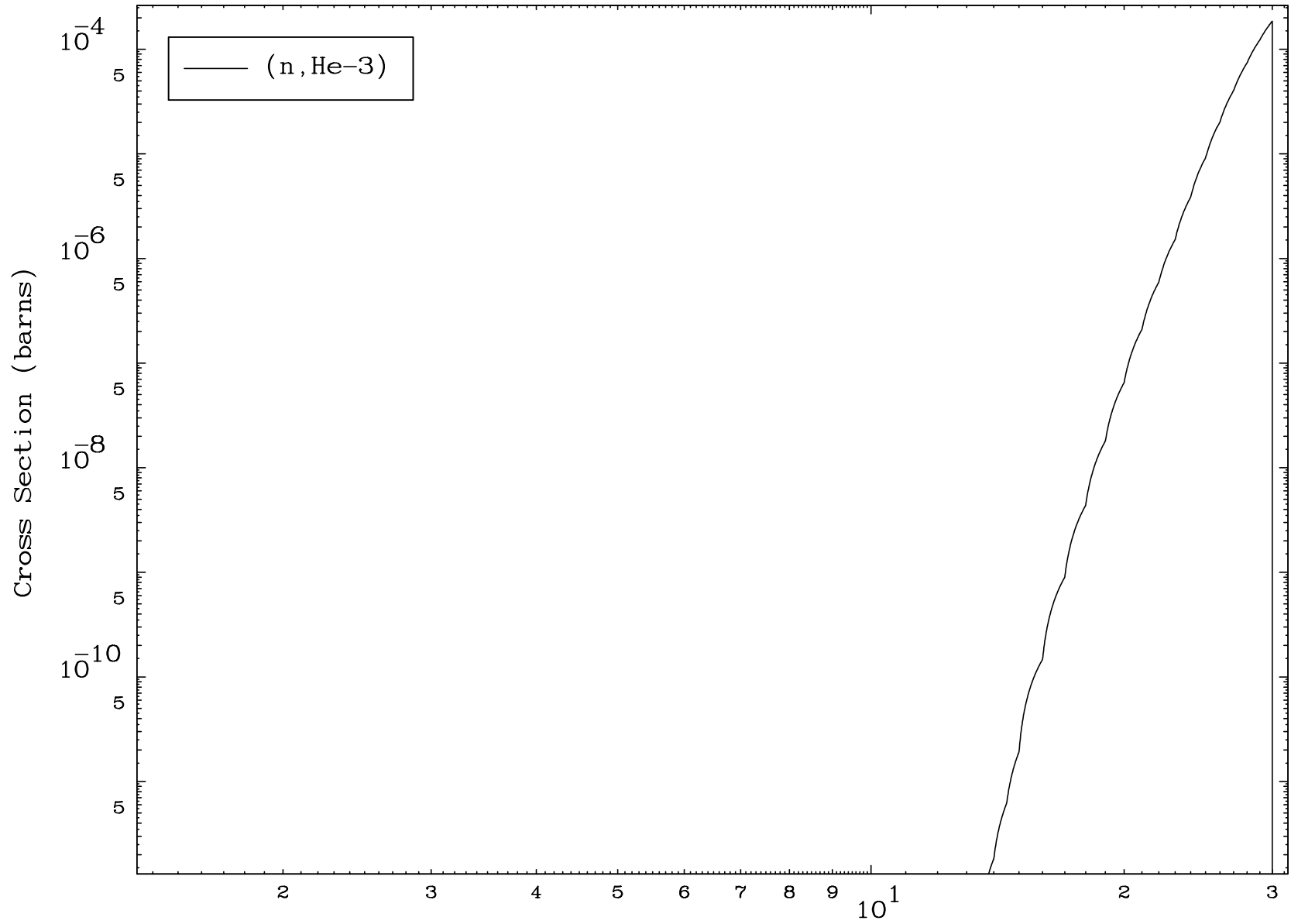


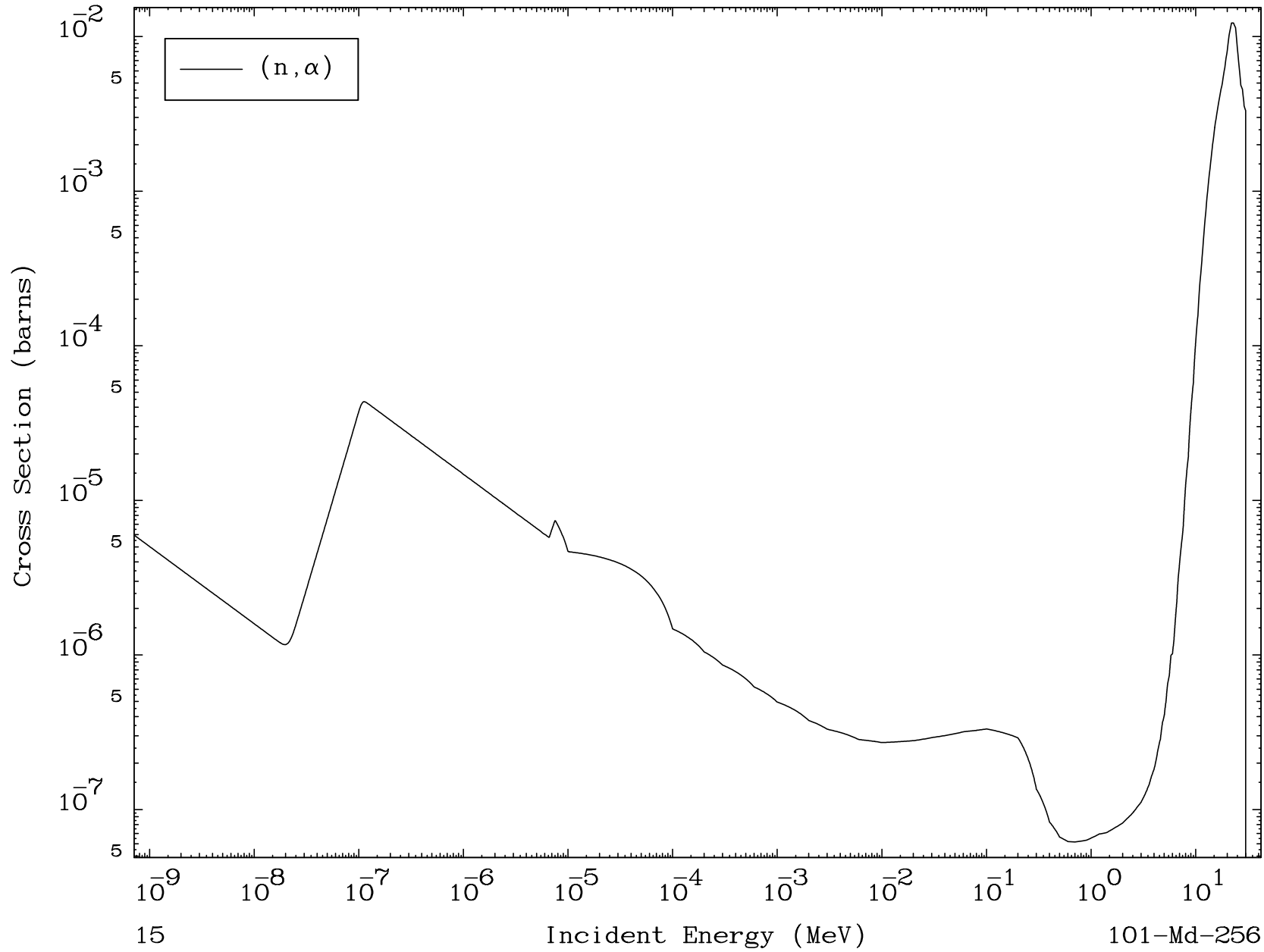
12

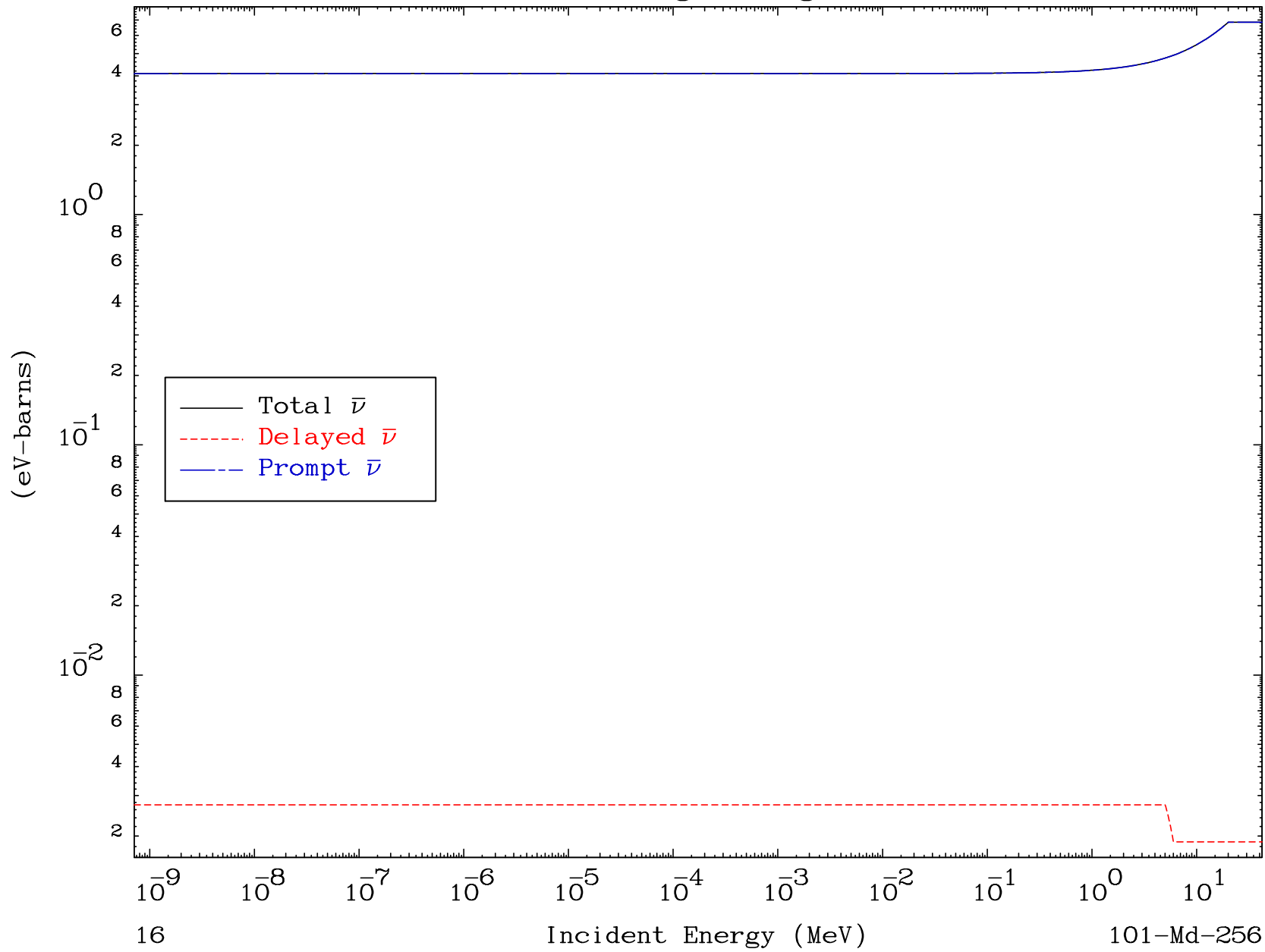
Incident Energy (MeV)

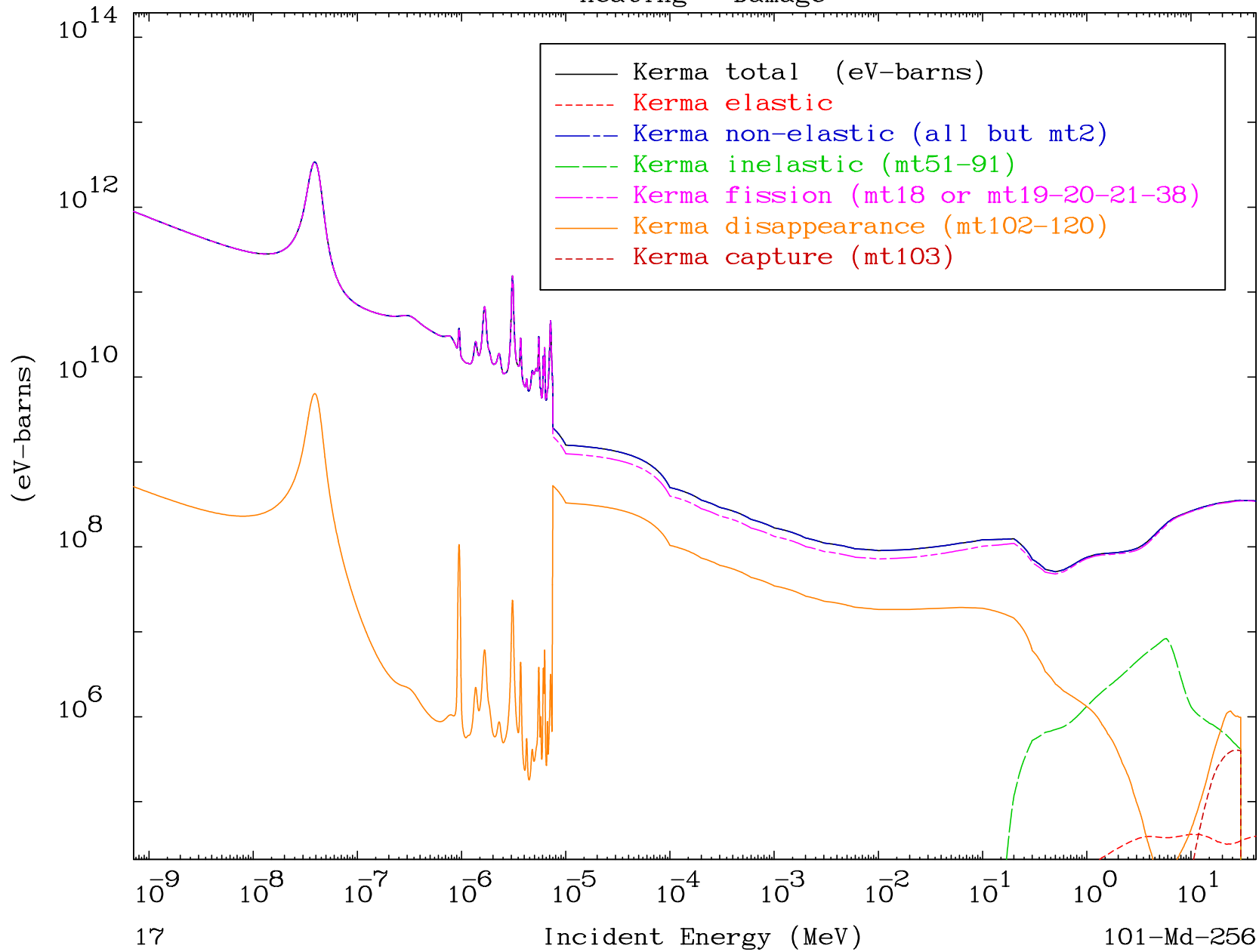
101-Md-256

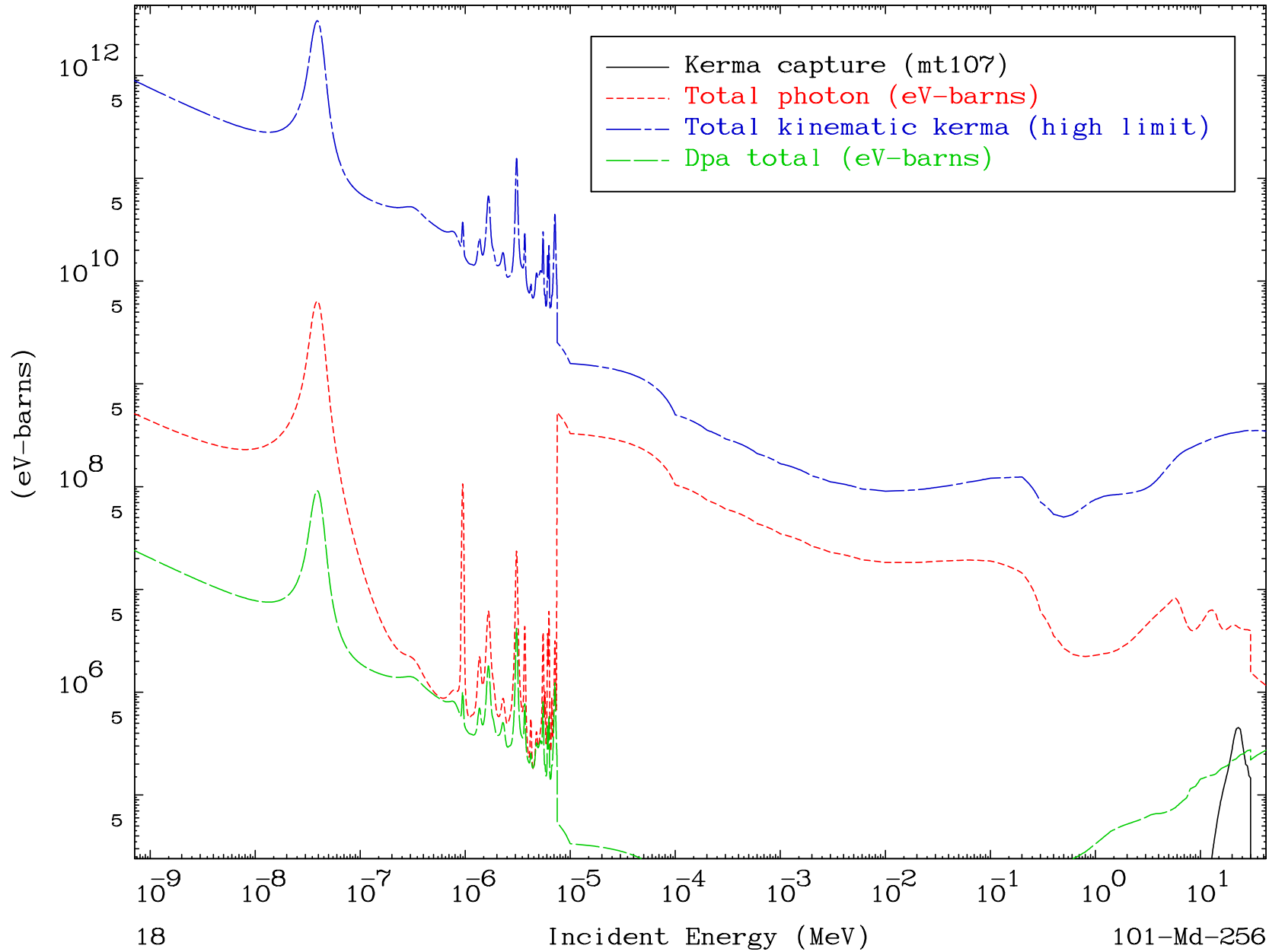


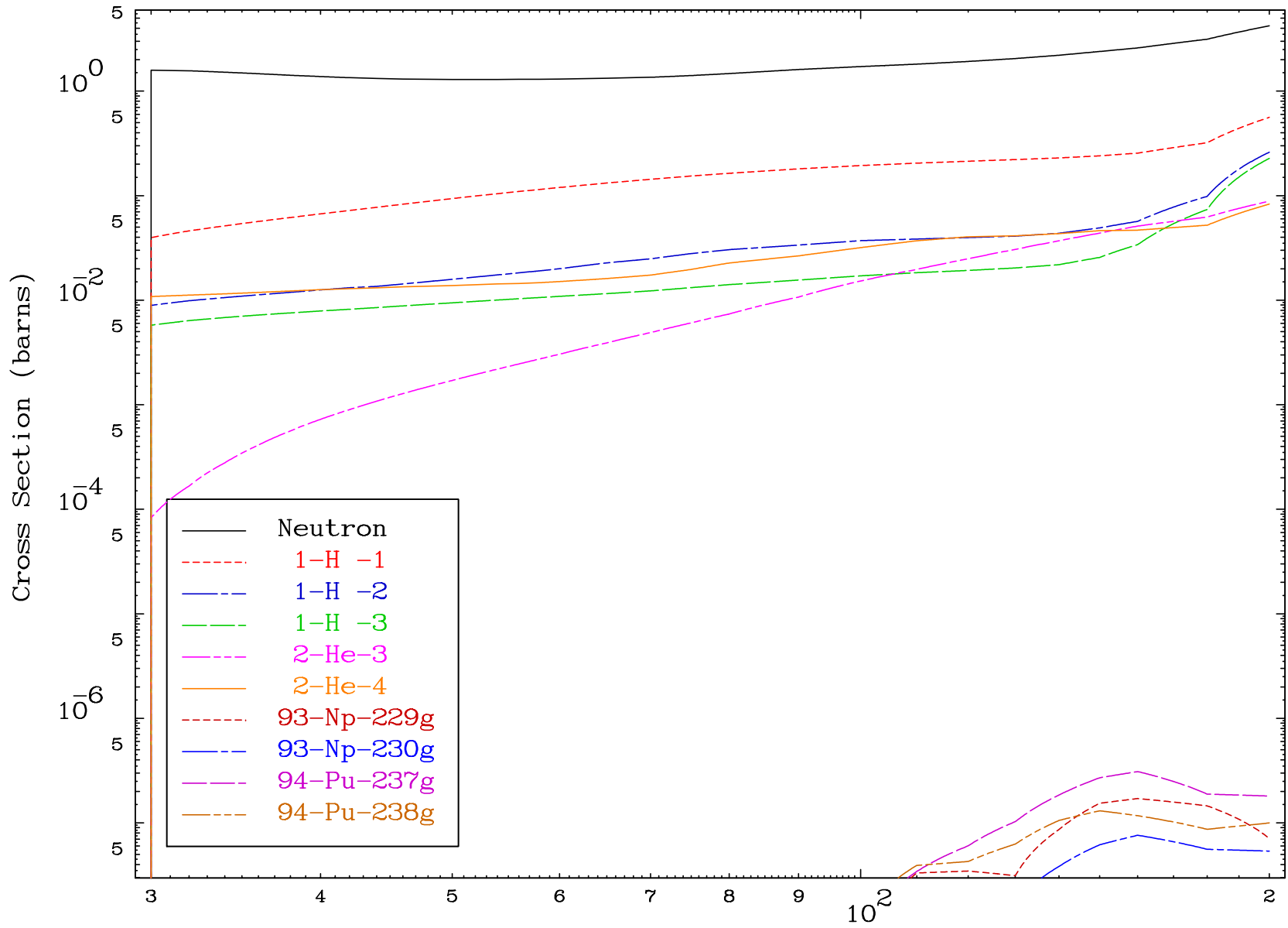










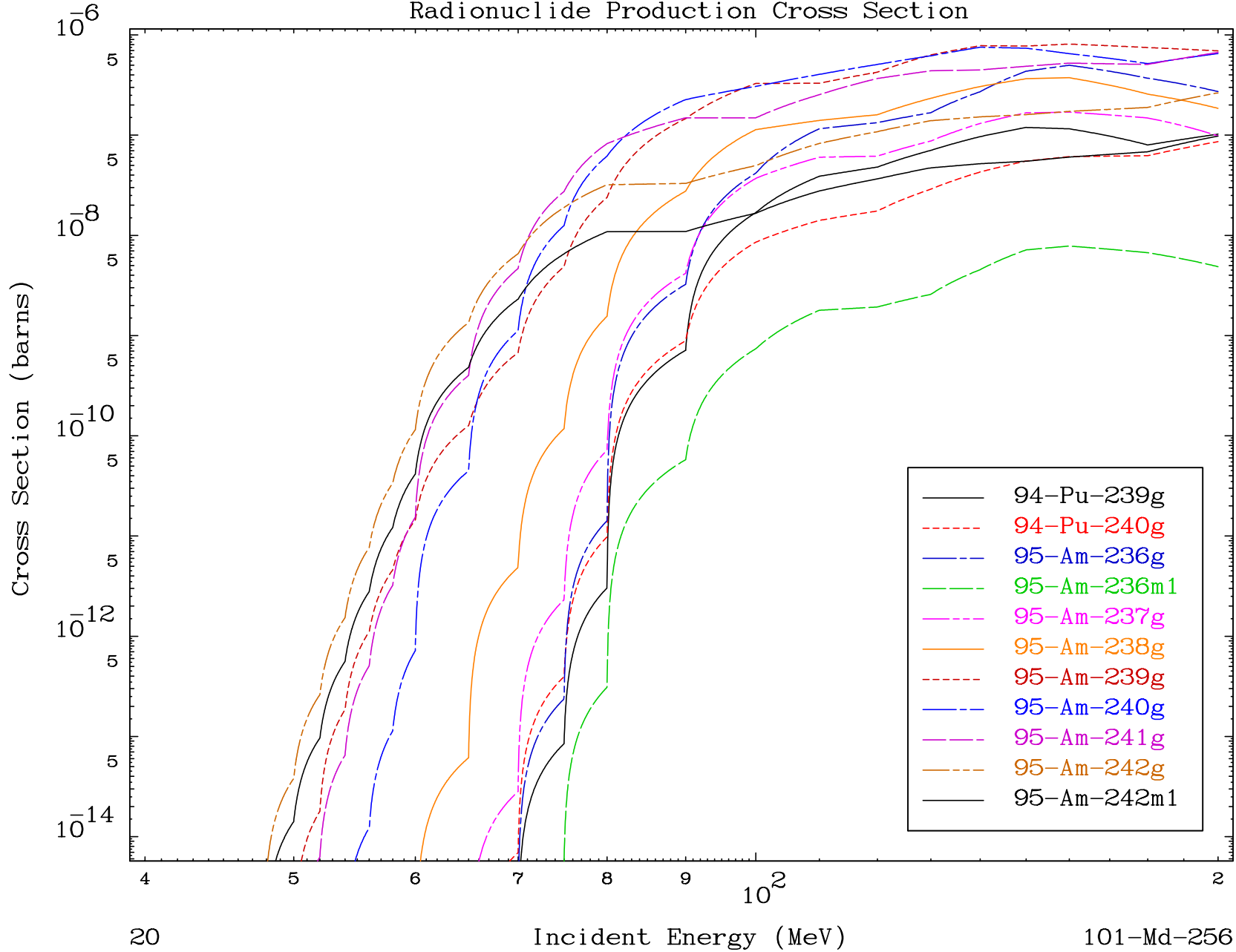


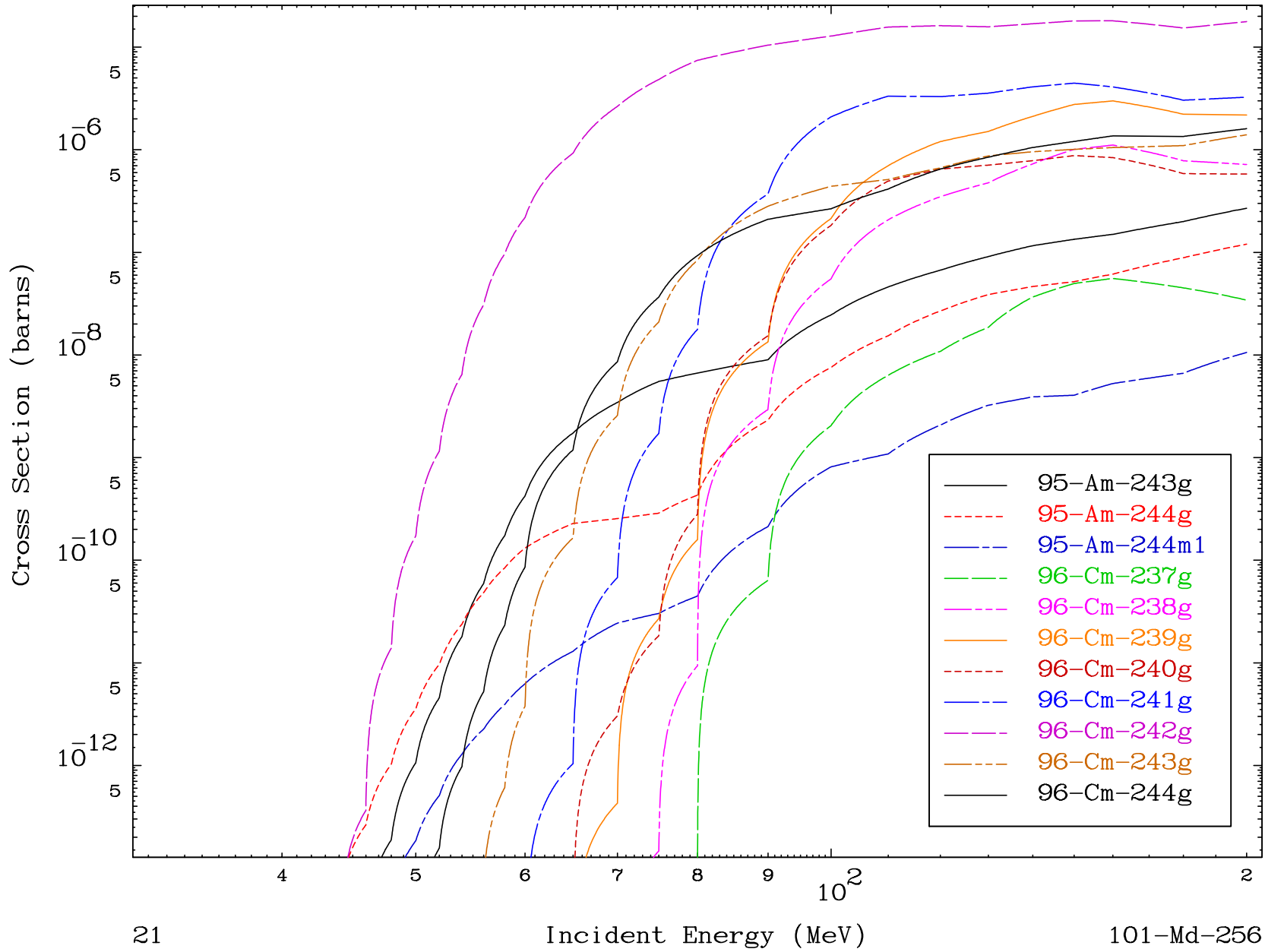
MAT 9957

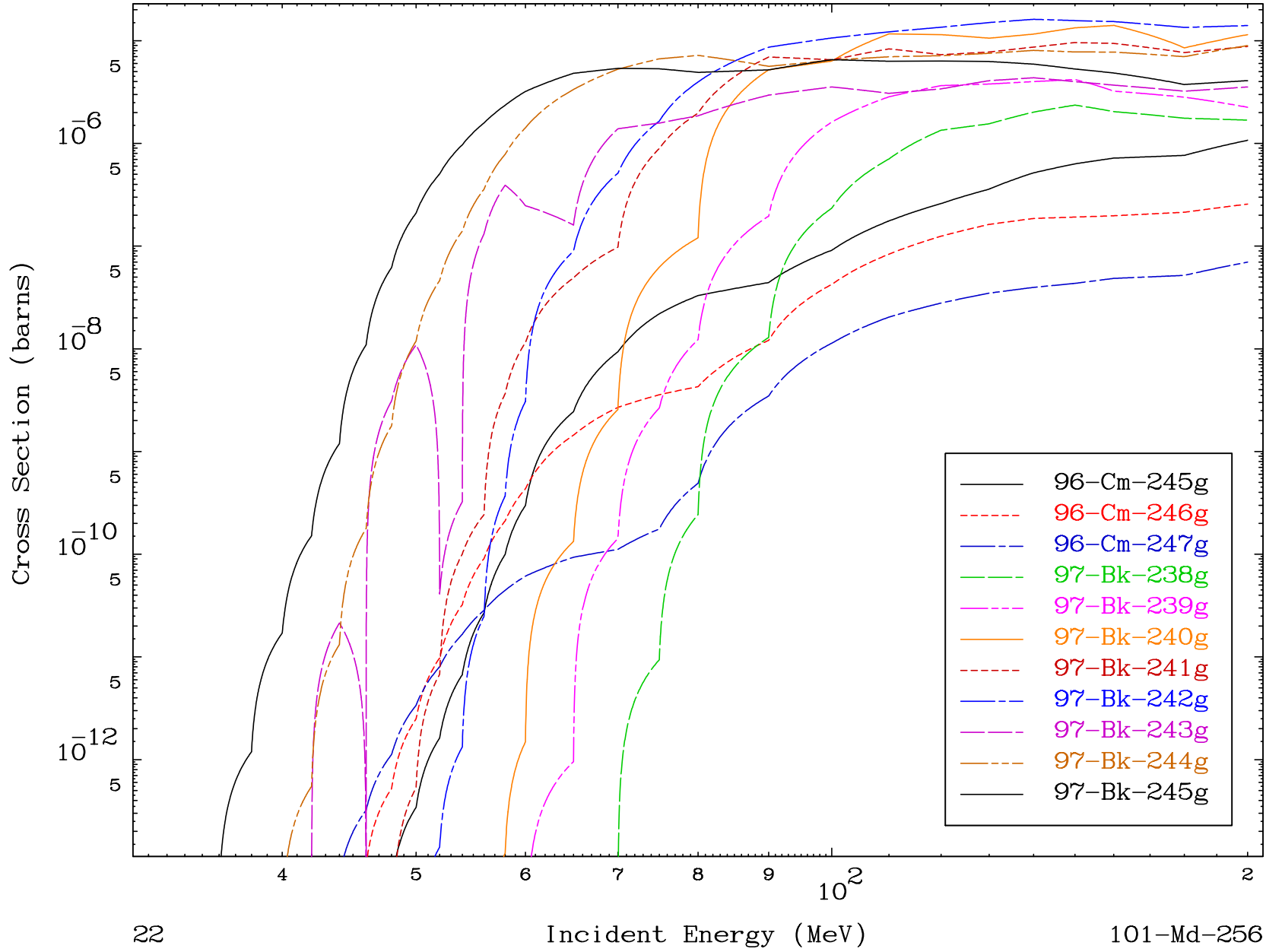
(n,remainder)

101-Md-256

Radionuclide Production Cross Section





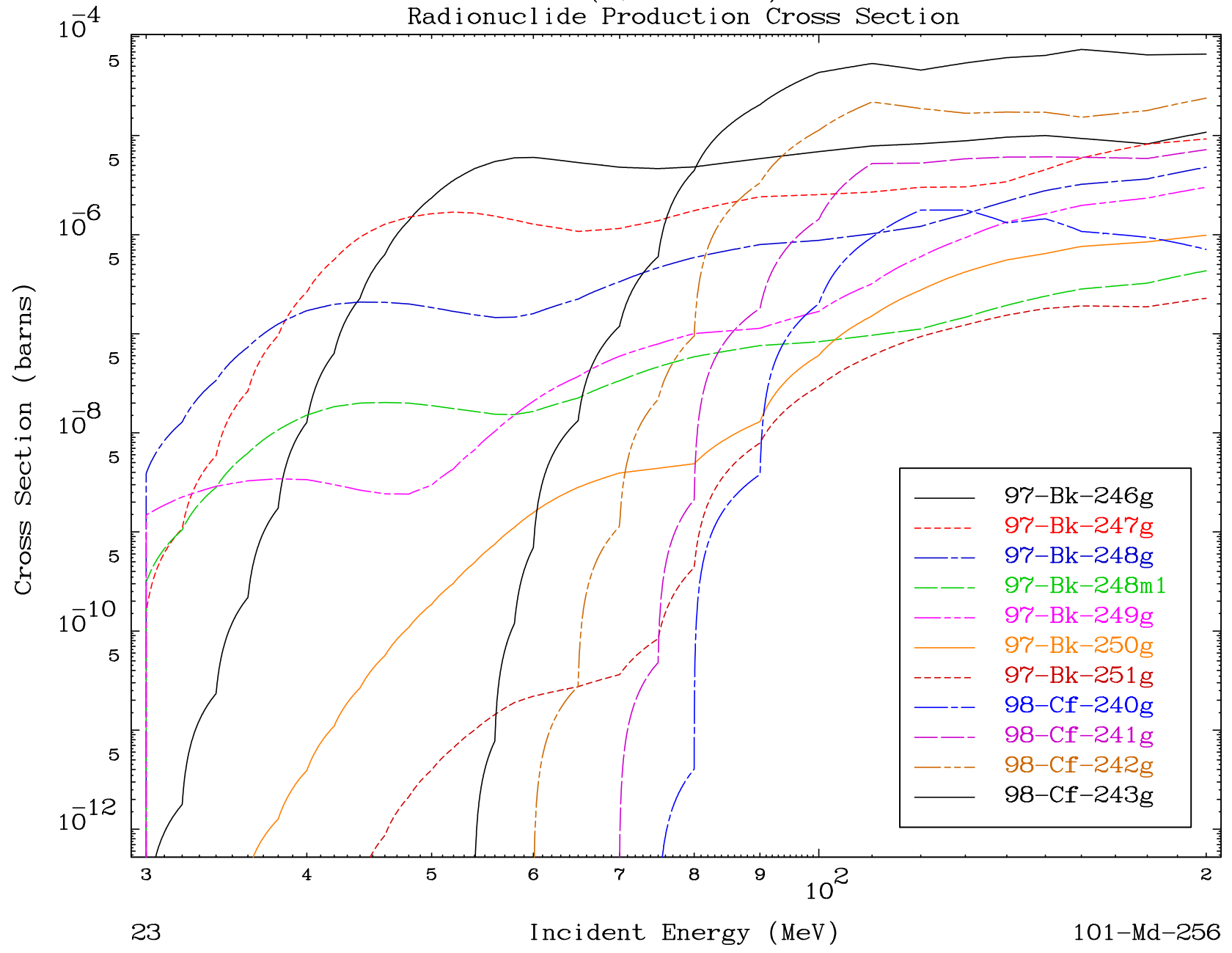


MAT 9957

(n,remainder)

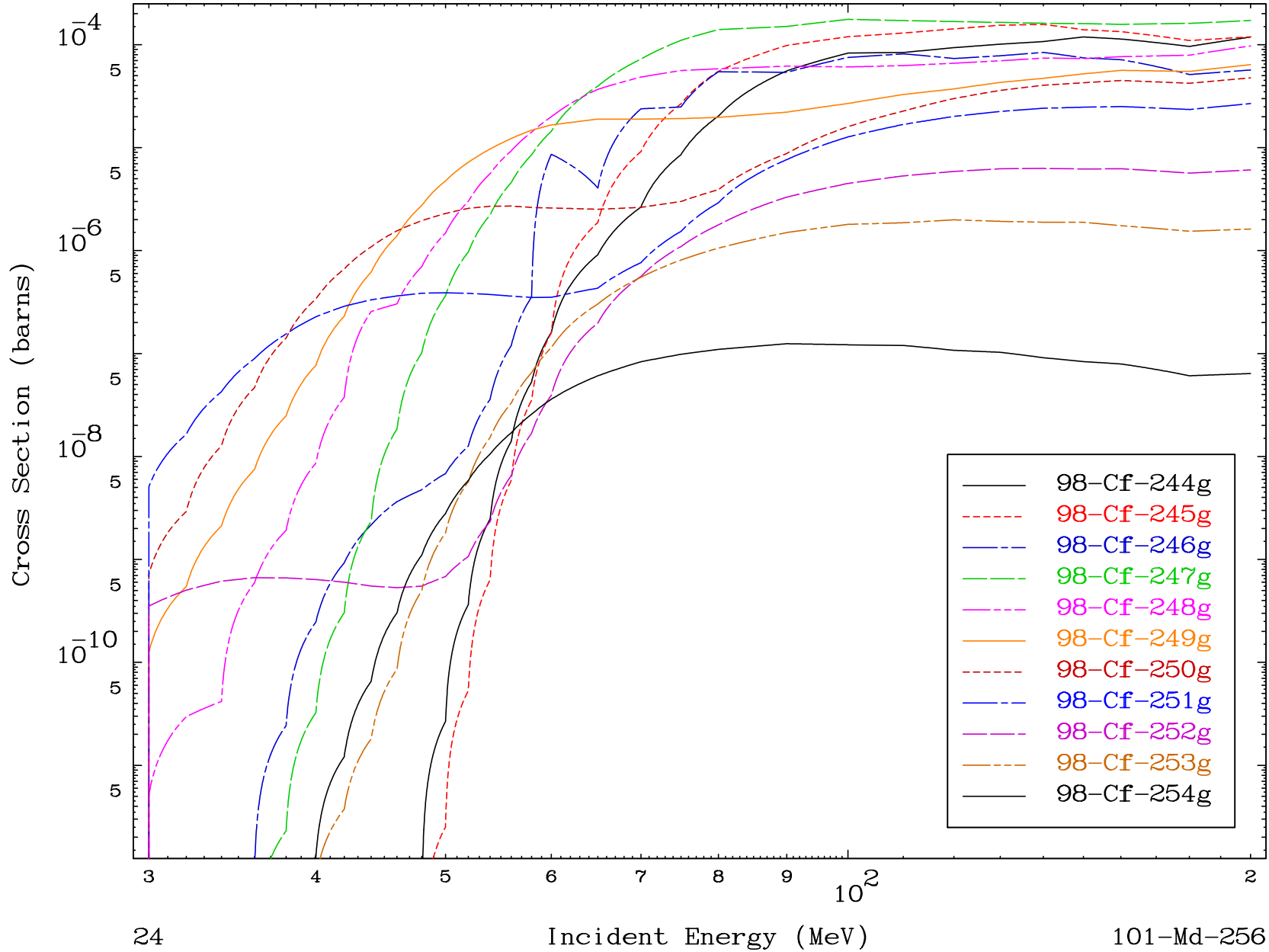
101-Md-256

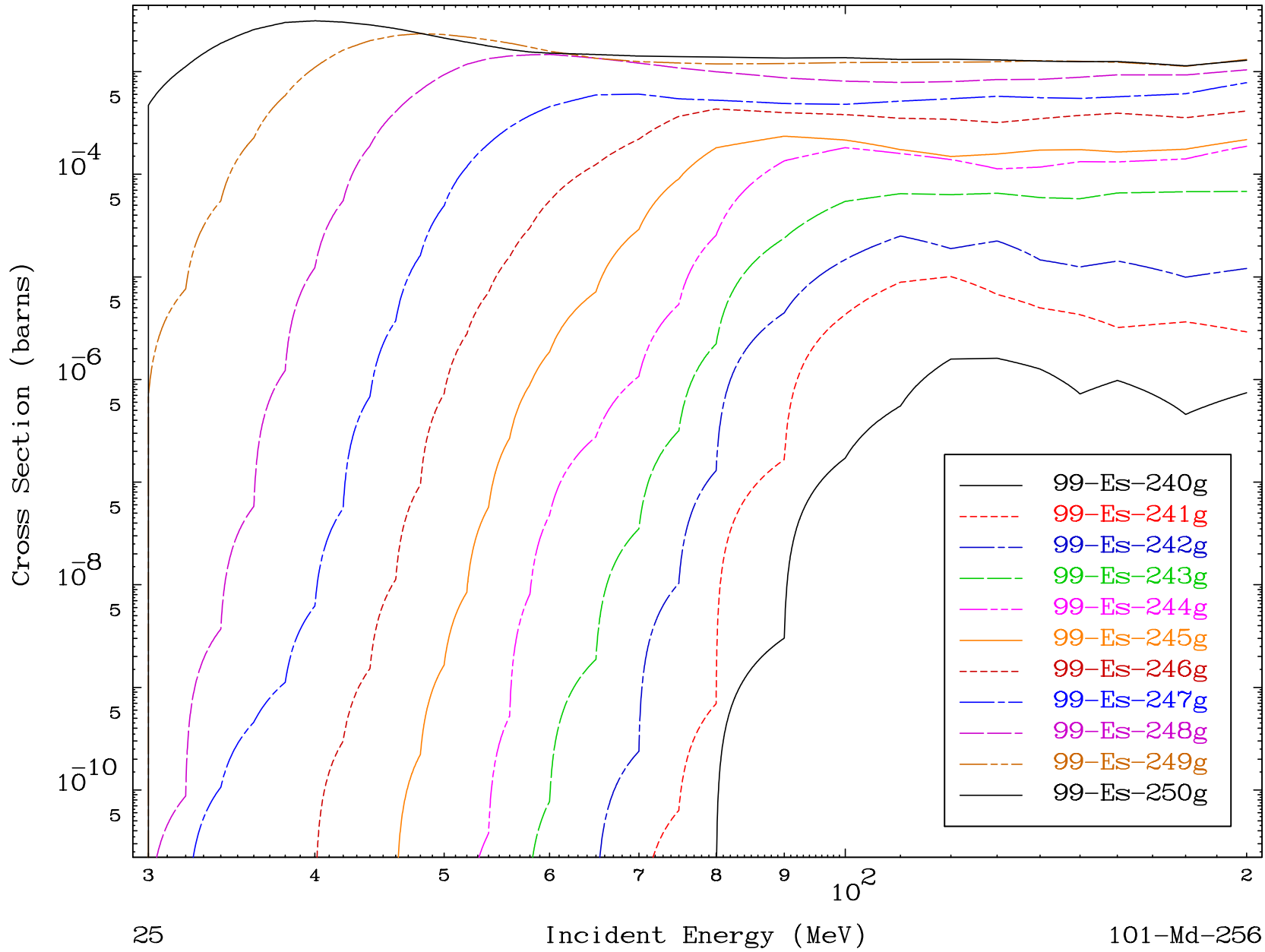
Radionuclide Production Cross Section



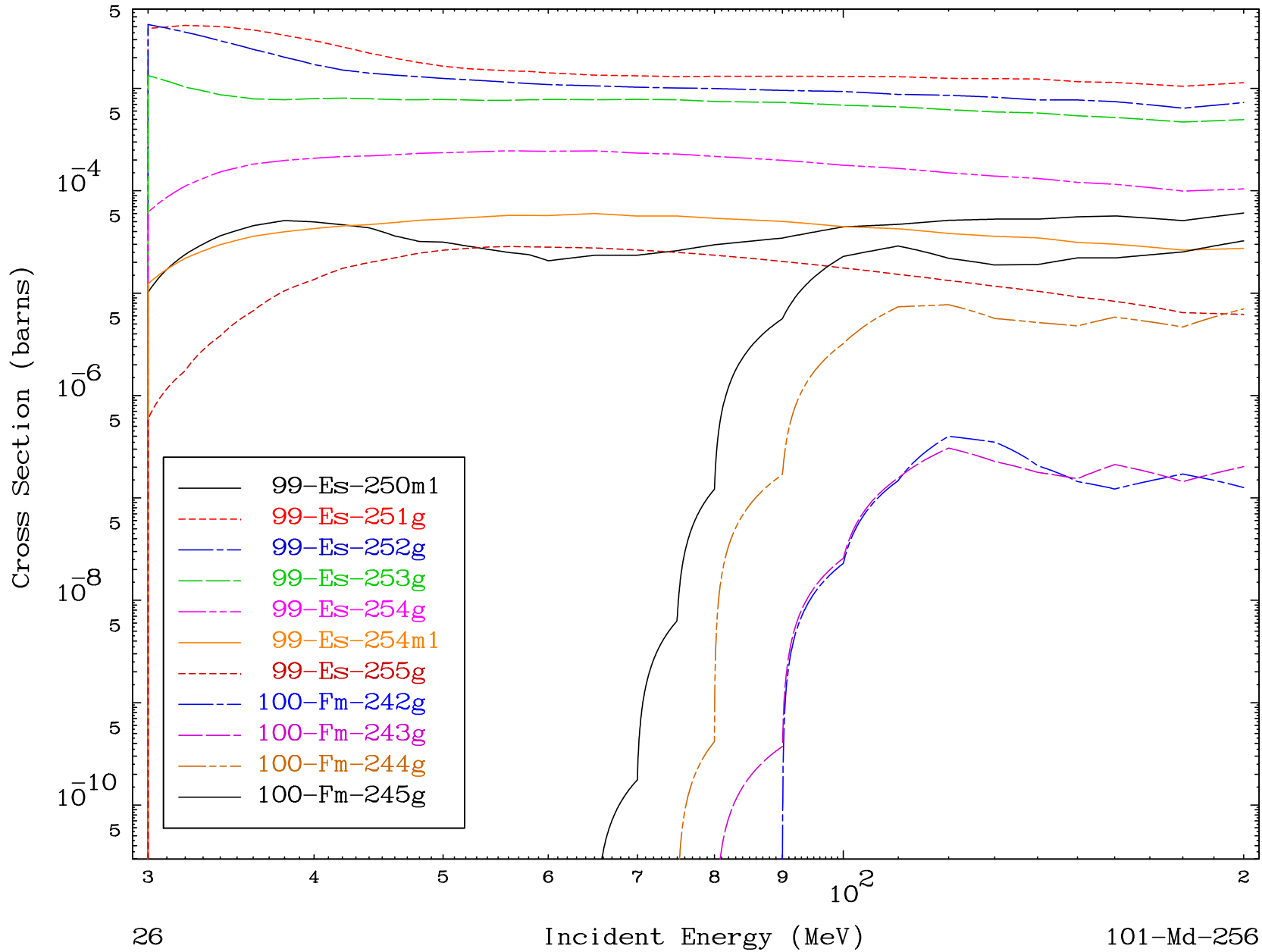
23

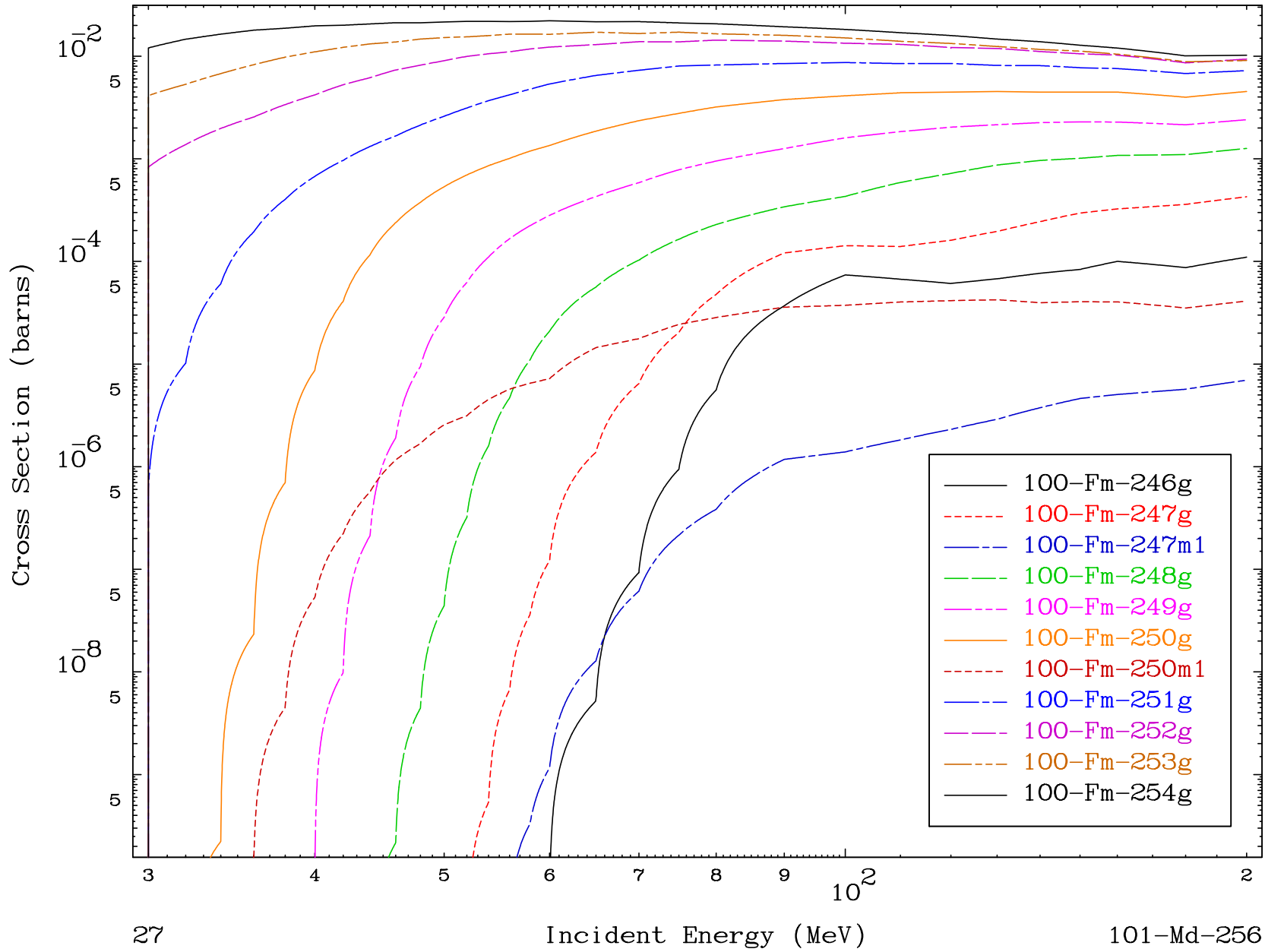
101-Md-256

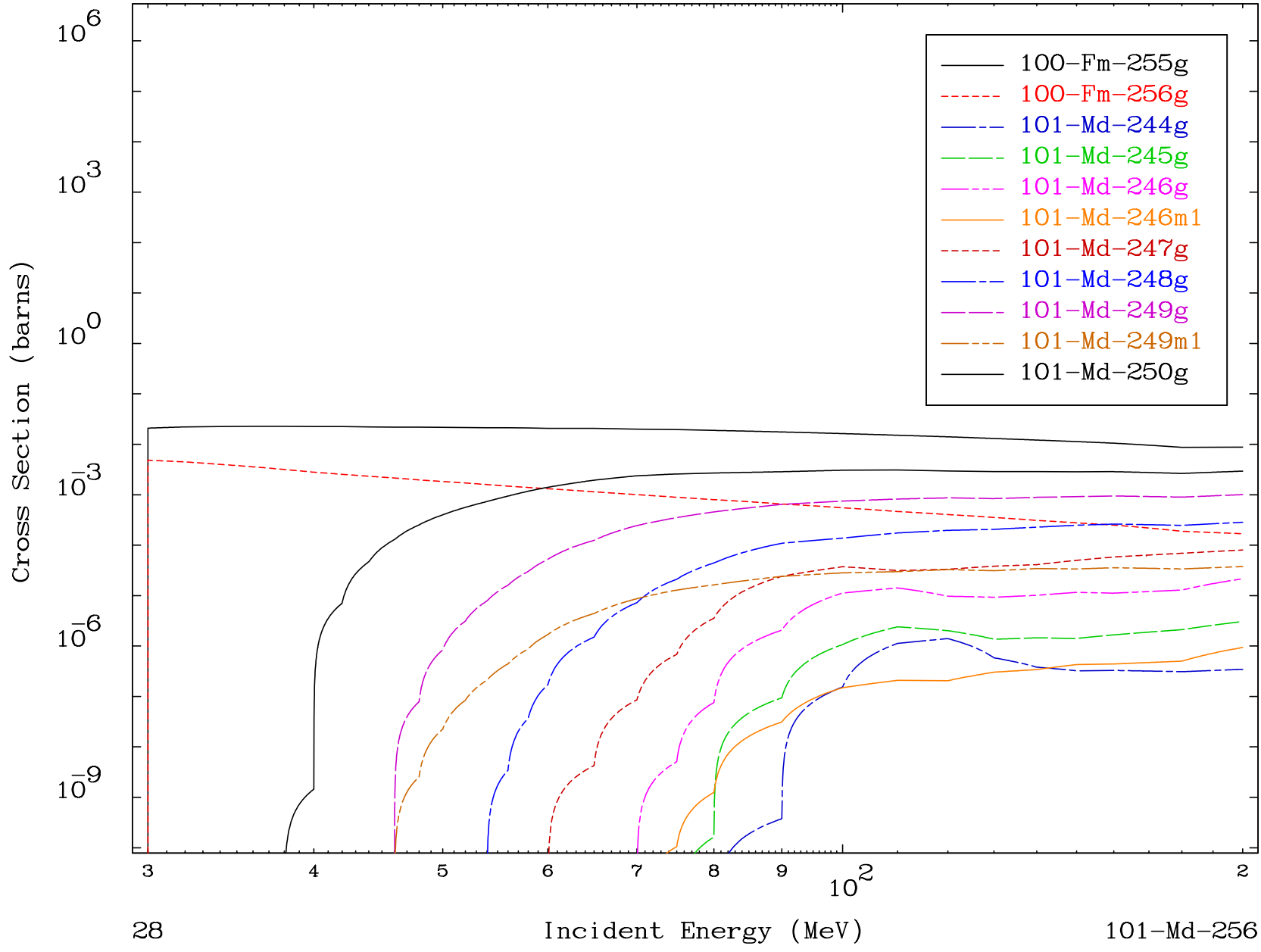




Radionuclide Production Cross Section



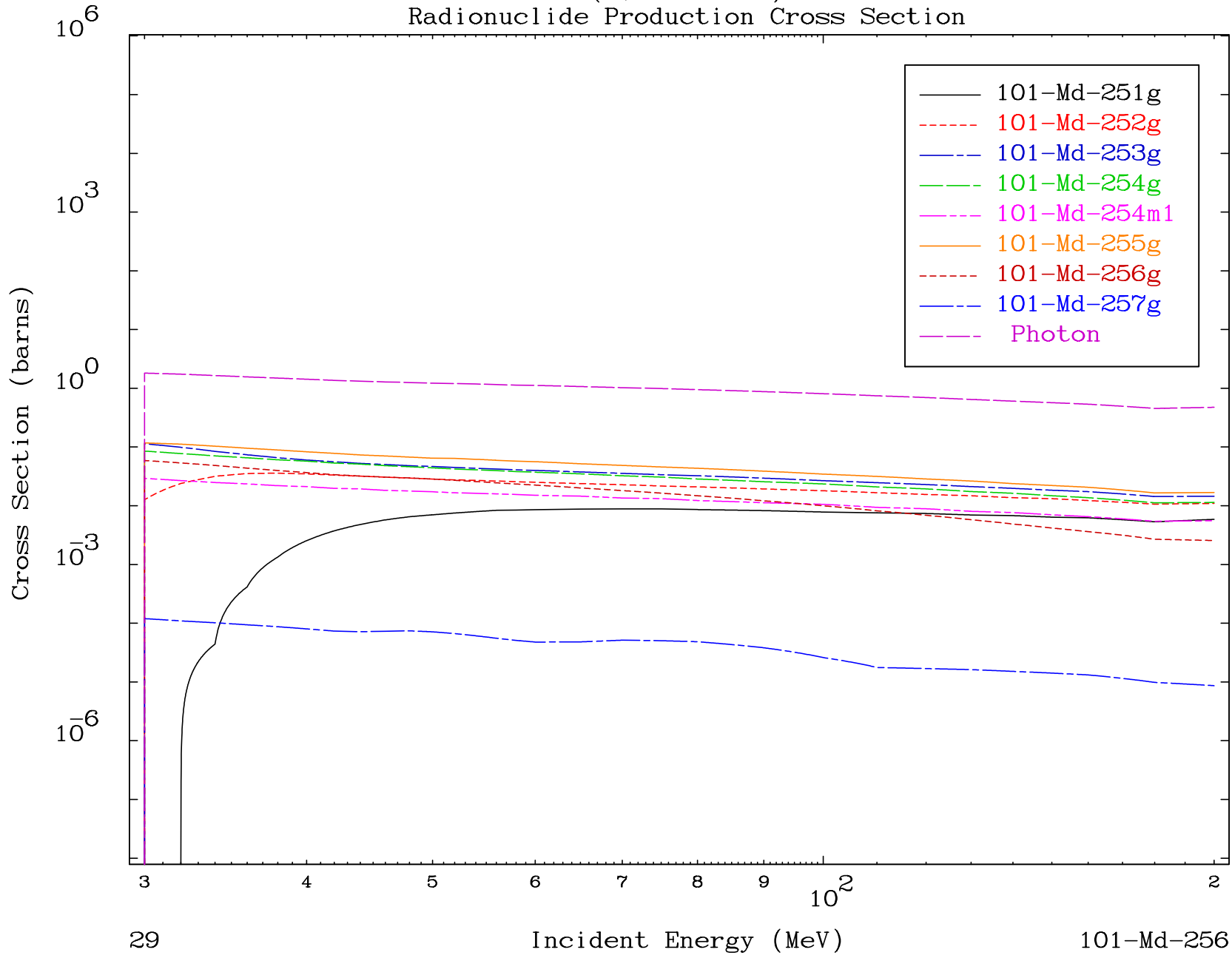




MAT 9957

(n,remainder)
Radionuclide Production Cross Section

101-Md-256



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Incident Energy (MeV)

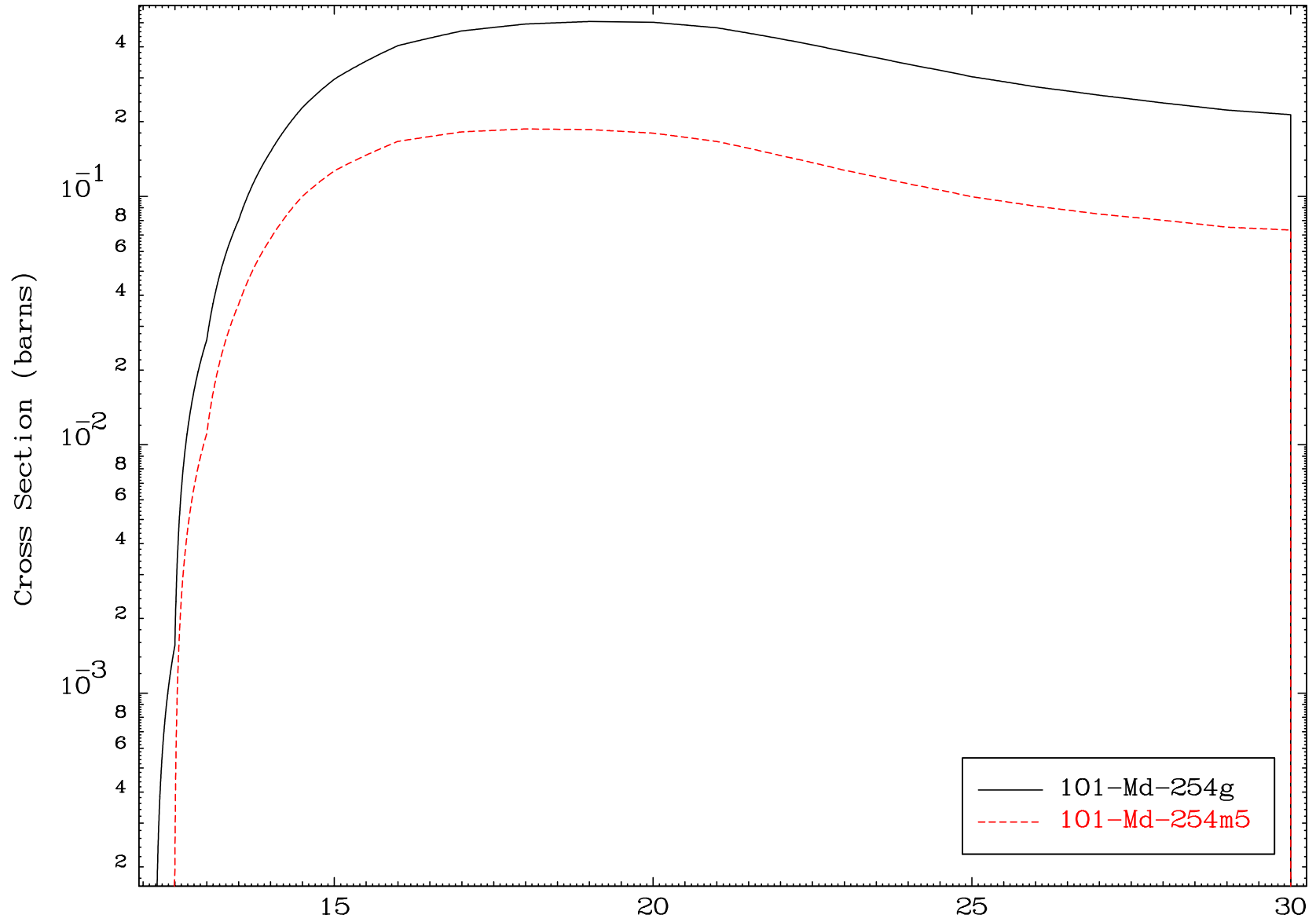
101-Md-256

MAT 9957

(n,3n)

101-Md-256

Radionuclide Production Cross Section



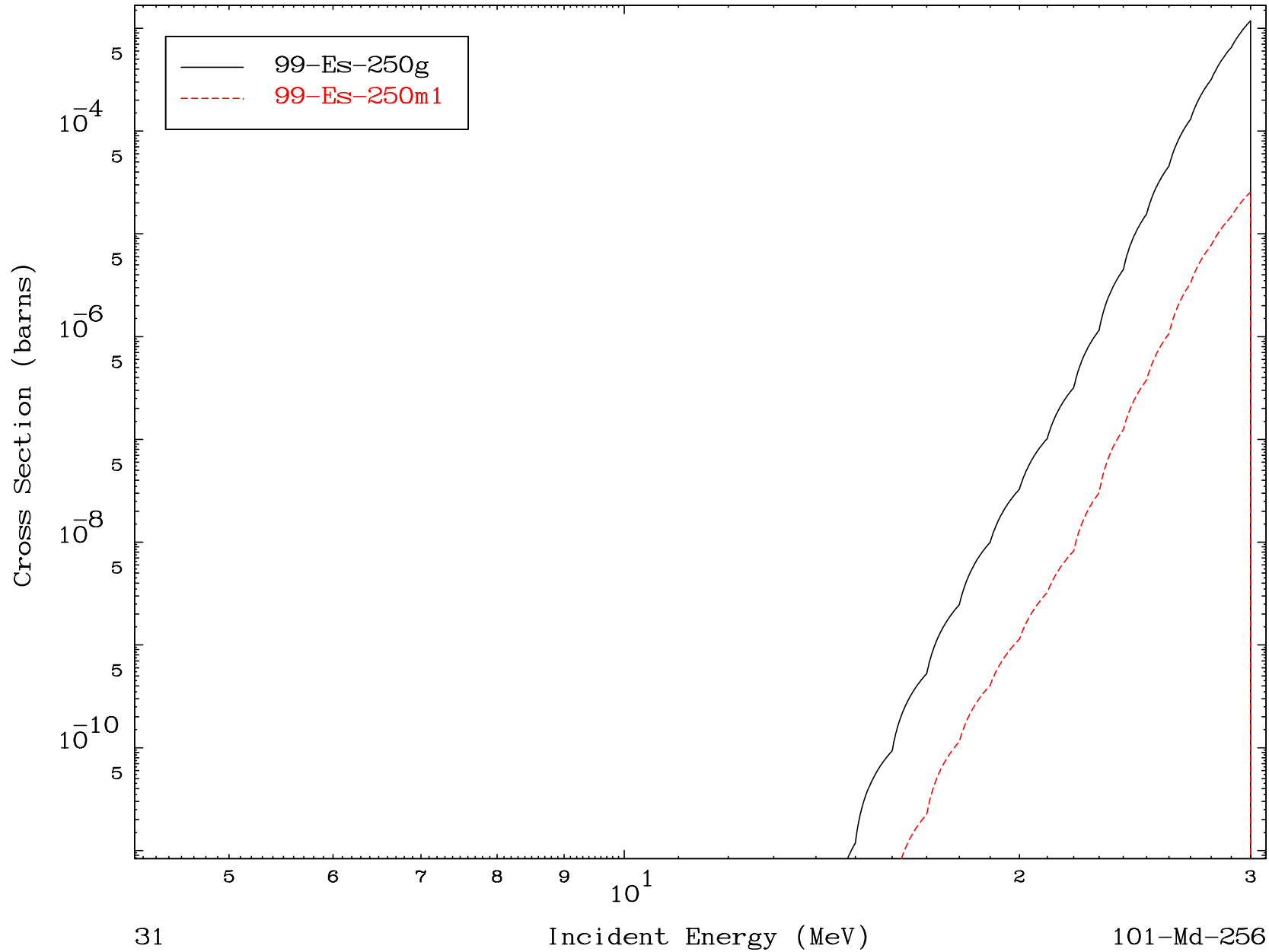
— 101-Md-254g
- - - 101-Md-254m5

30

Incident Energy (MeV)

101-Md-256

Radionuclide Production Cross Section

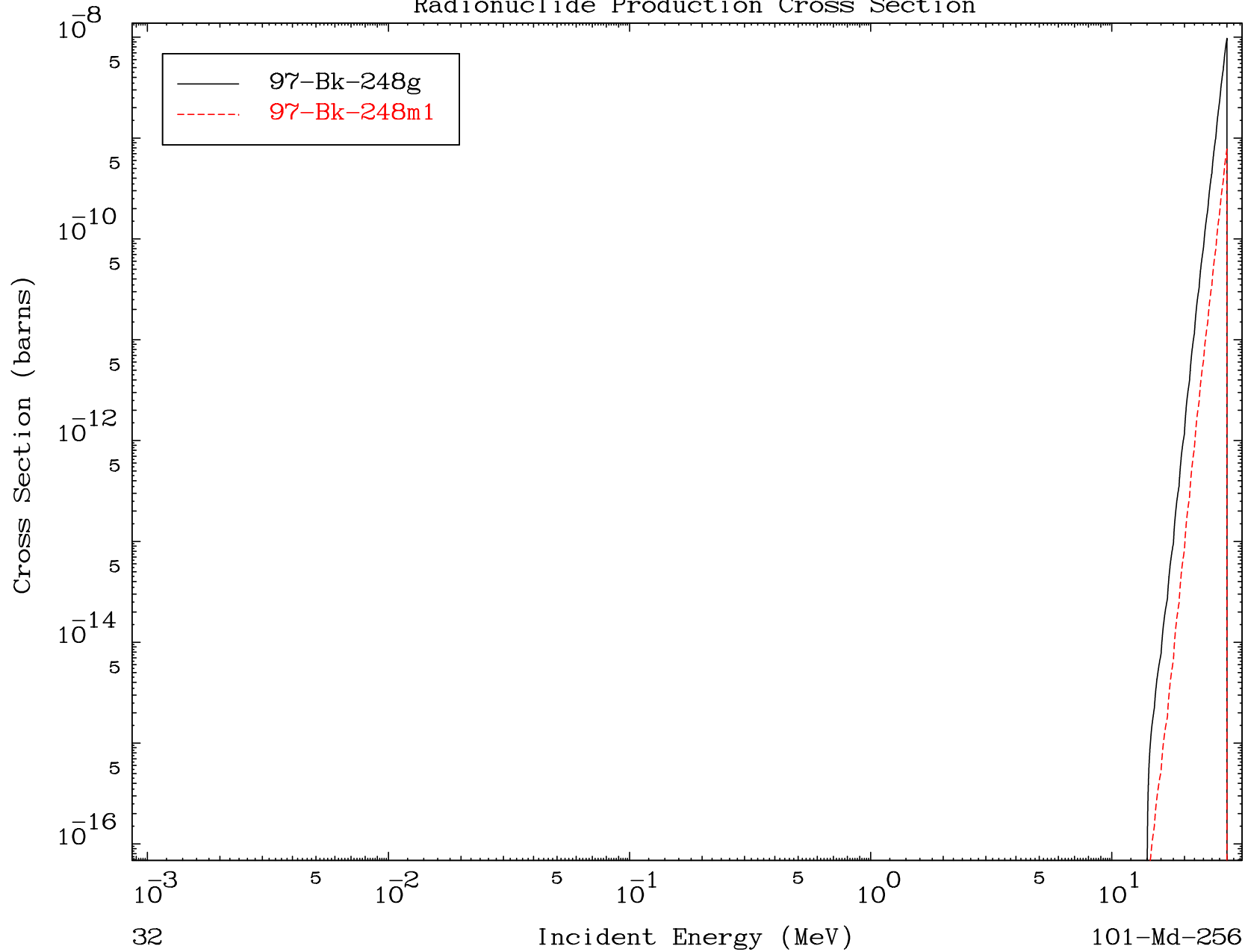


MAT 9957

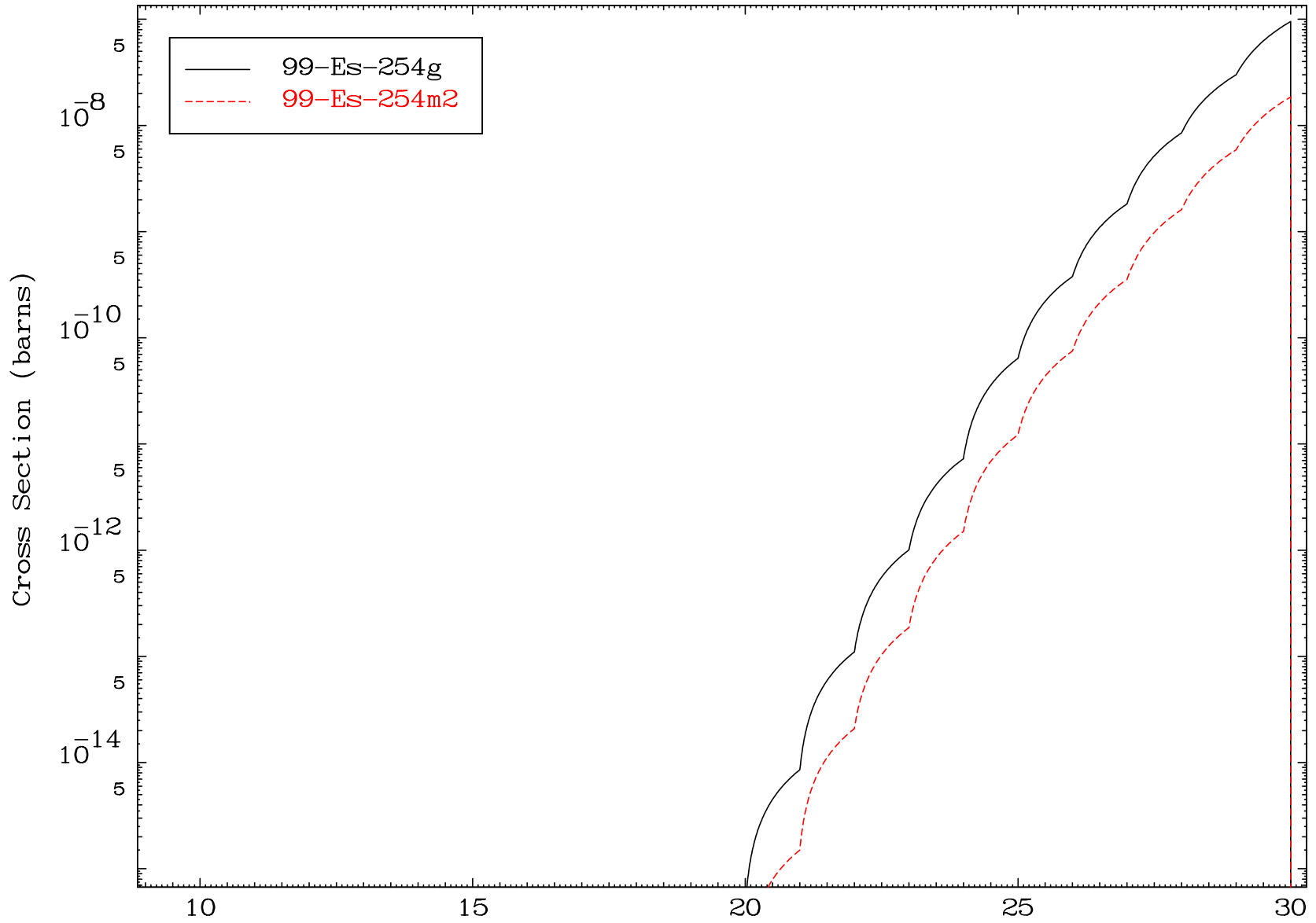
(n,n') 2α

101-Md-256

Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

