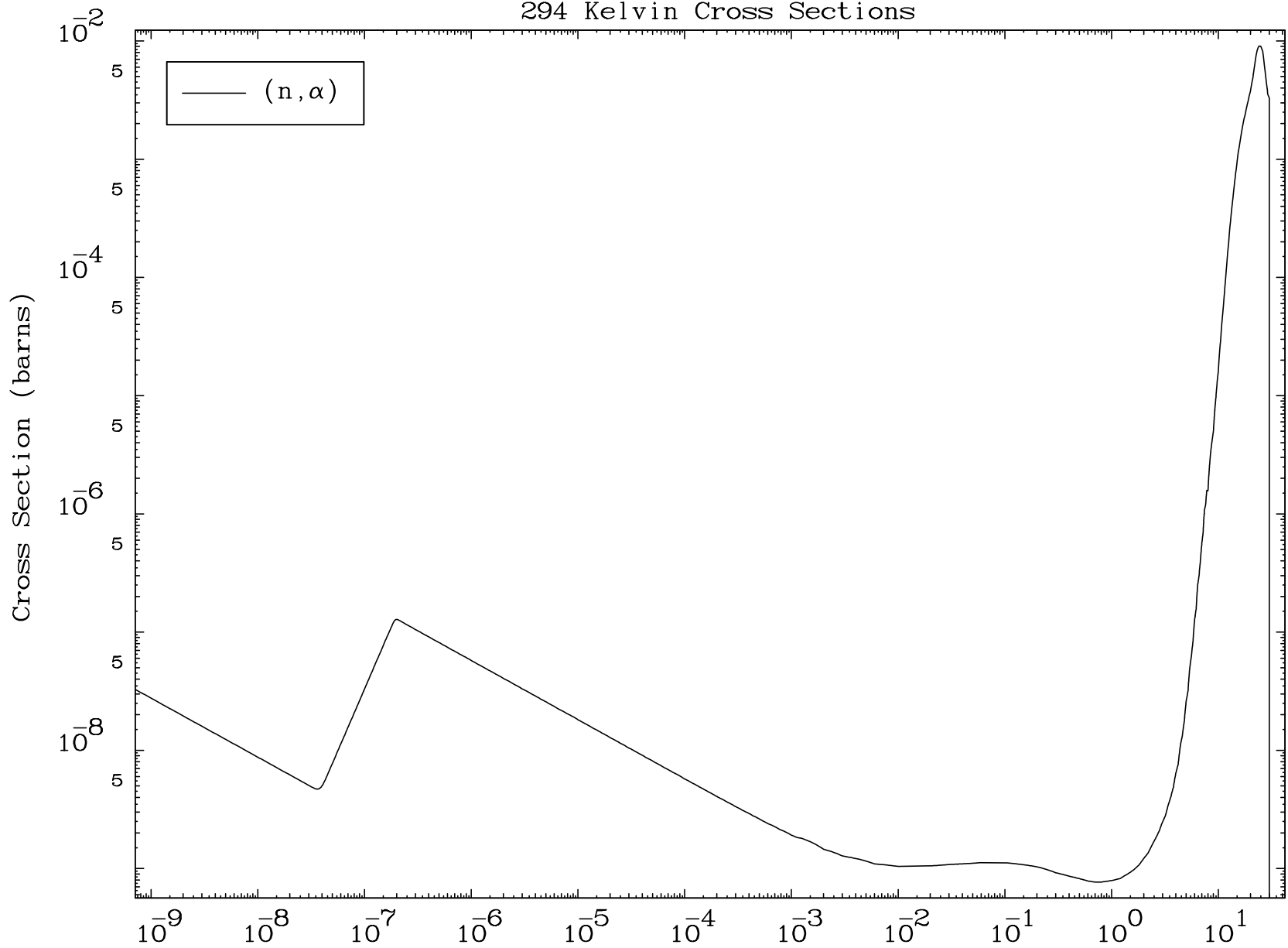


MAT 9960

(n,  $\alpha$ ) Levels  
294 Kelvin Cross Sections

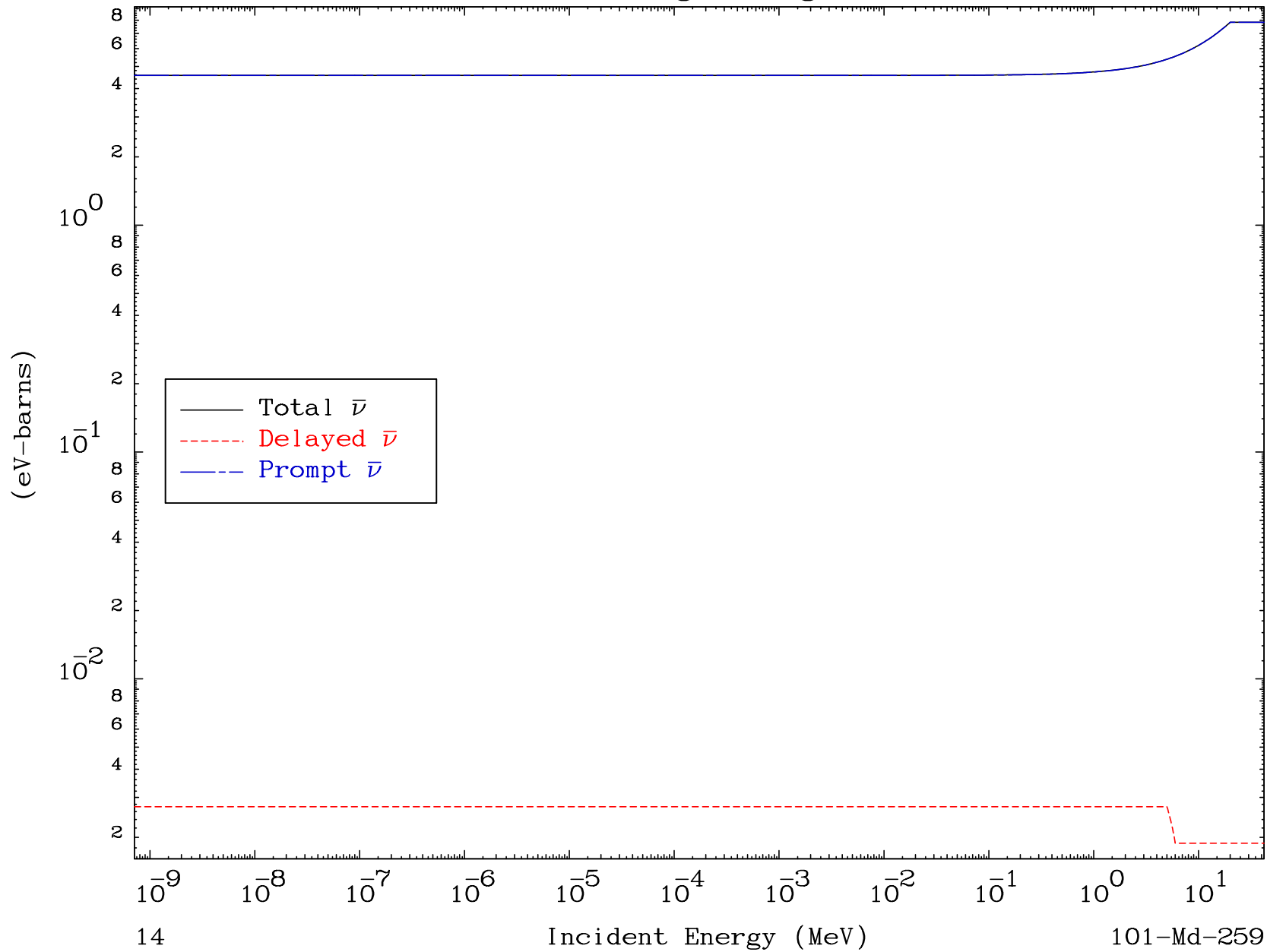
101-Md-259

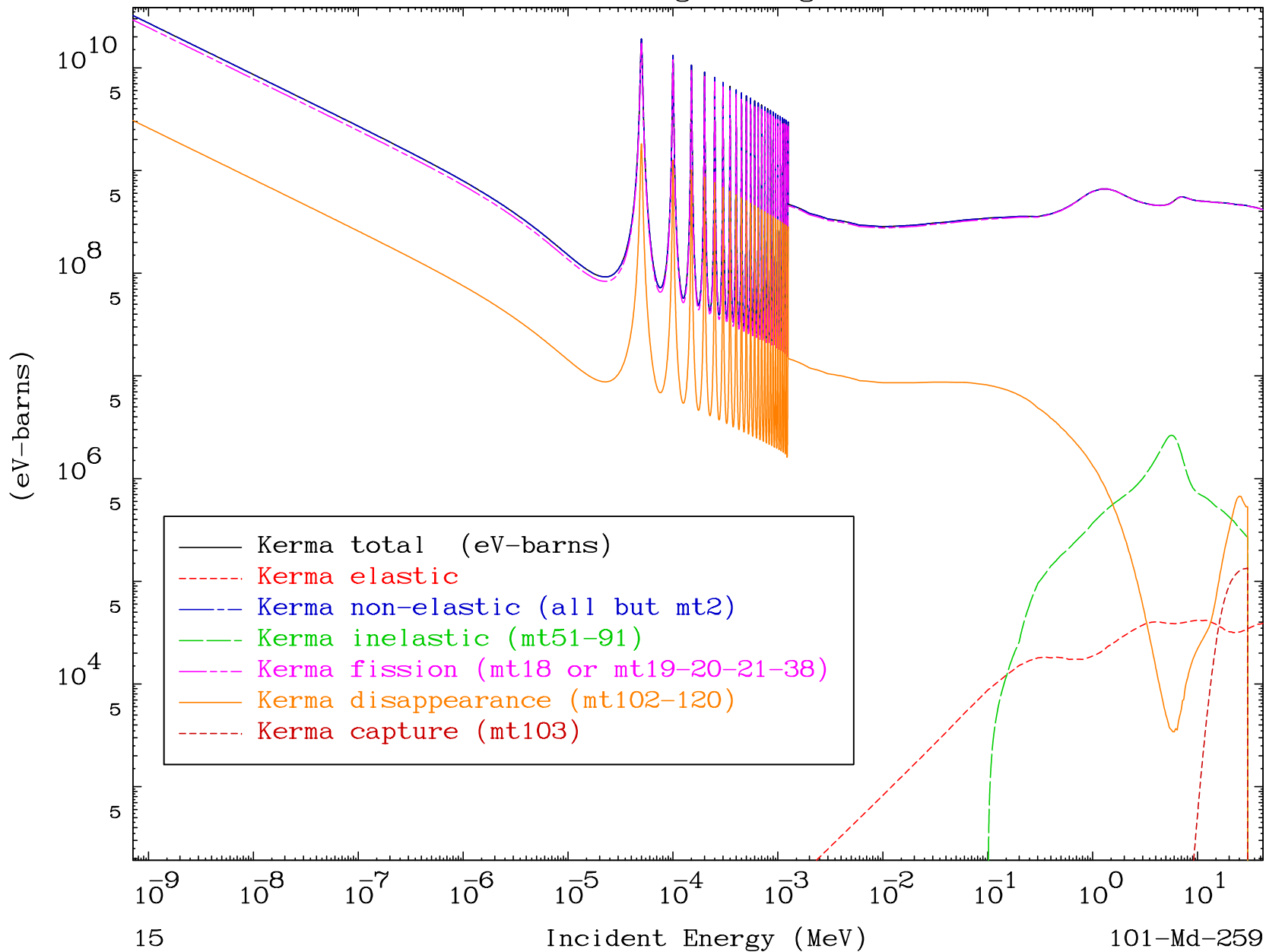


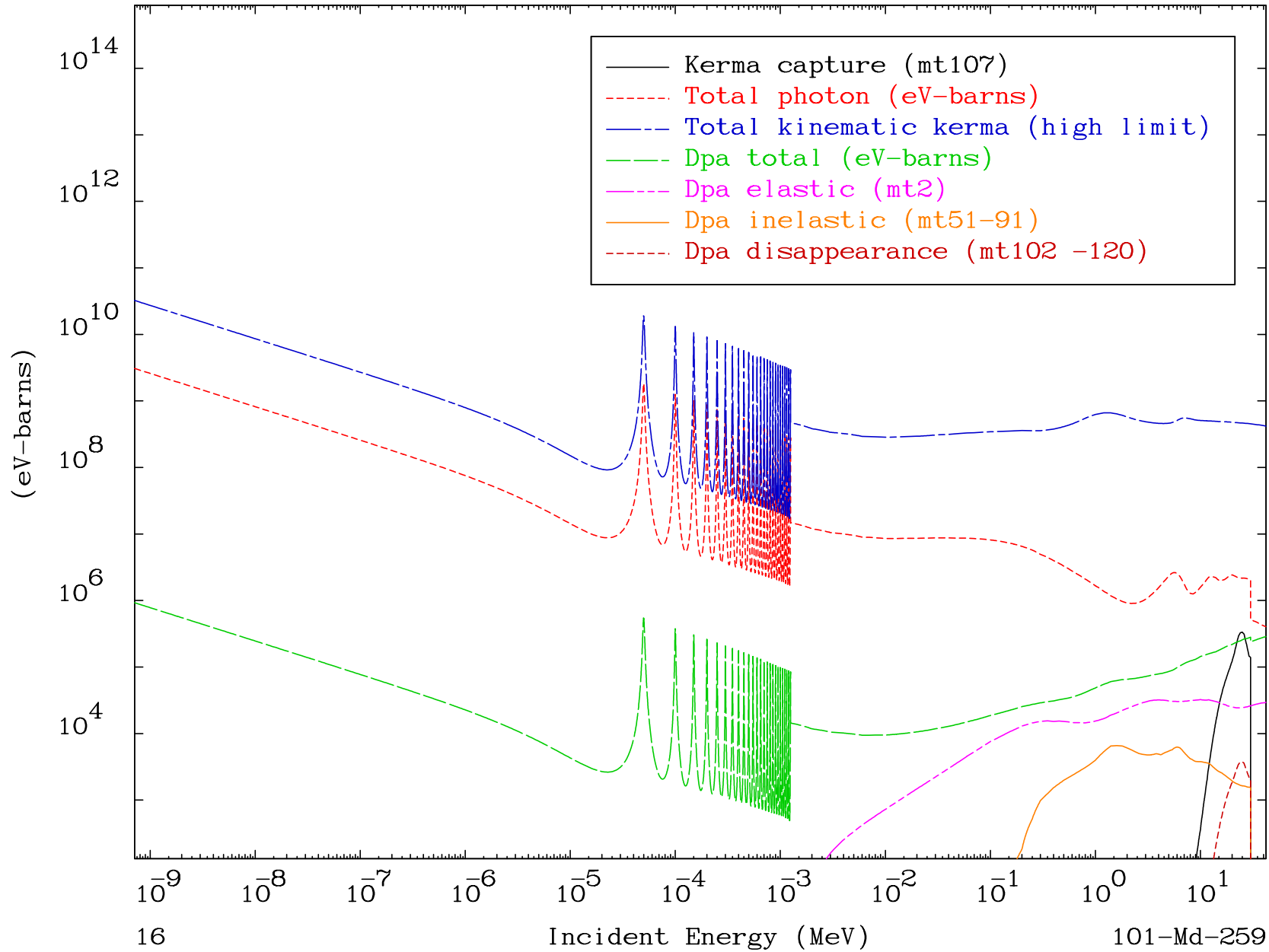
13

Incident Energy (MeV)

101-Md-259

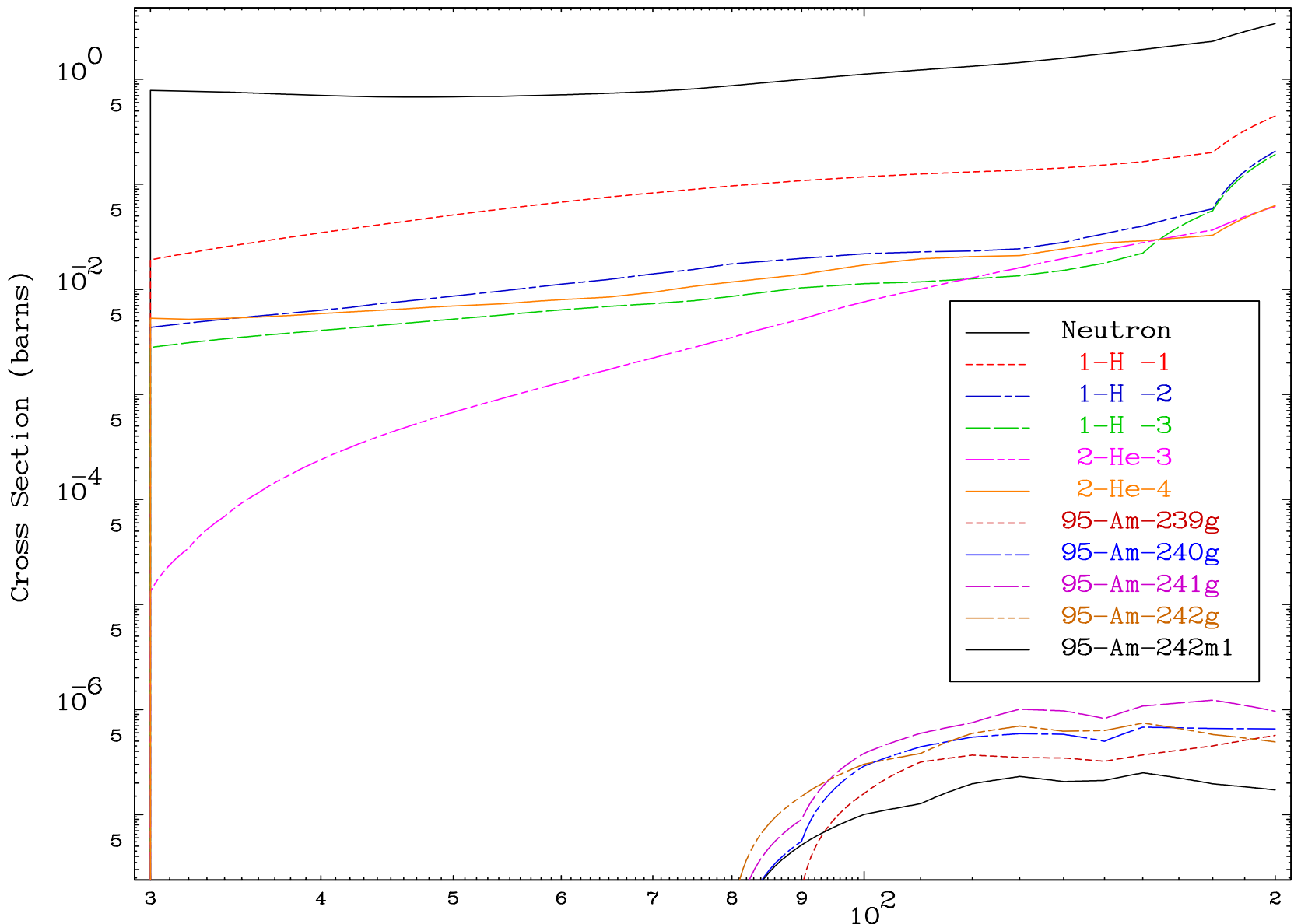




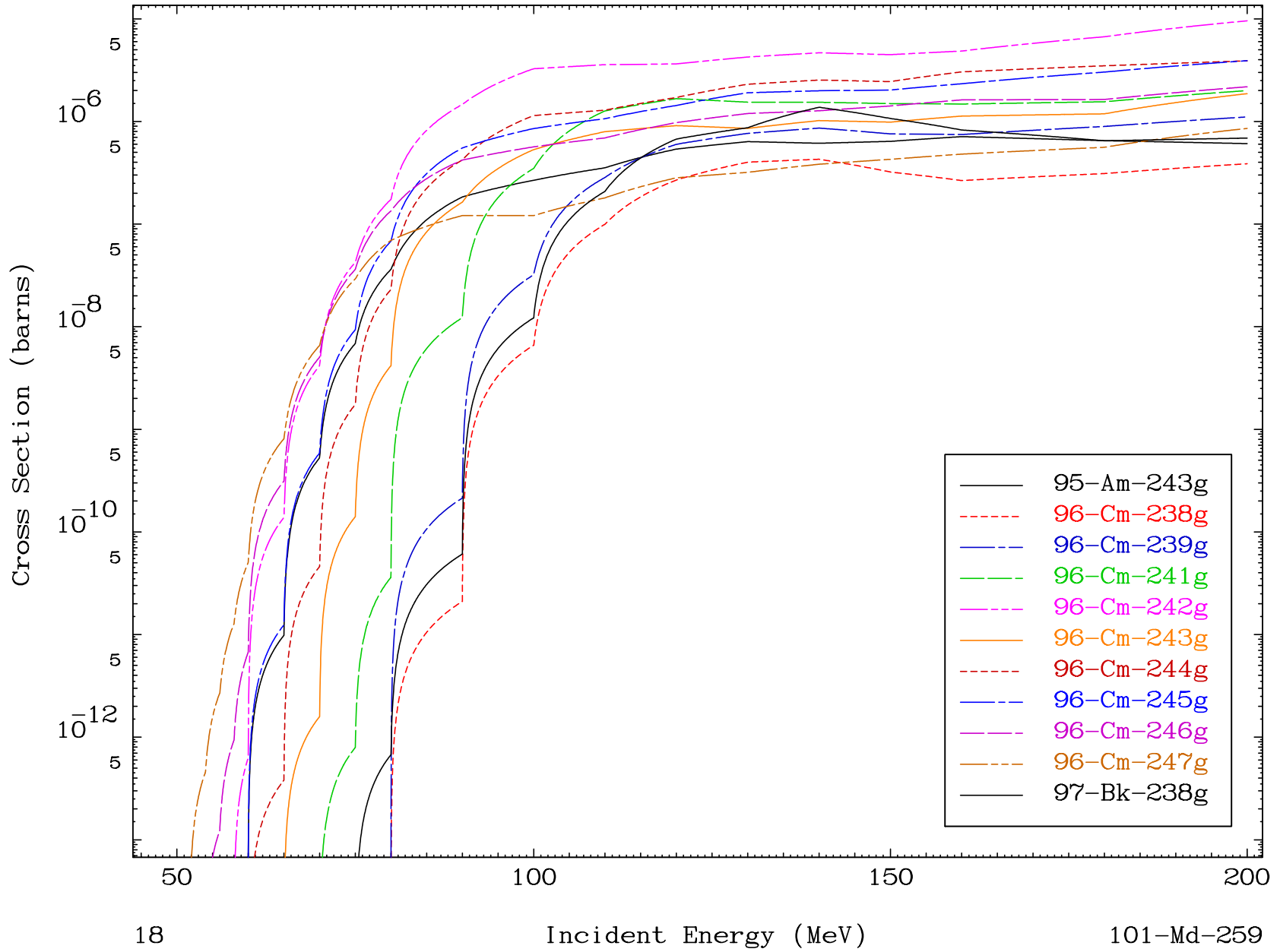


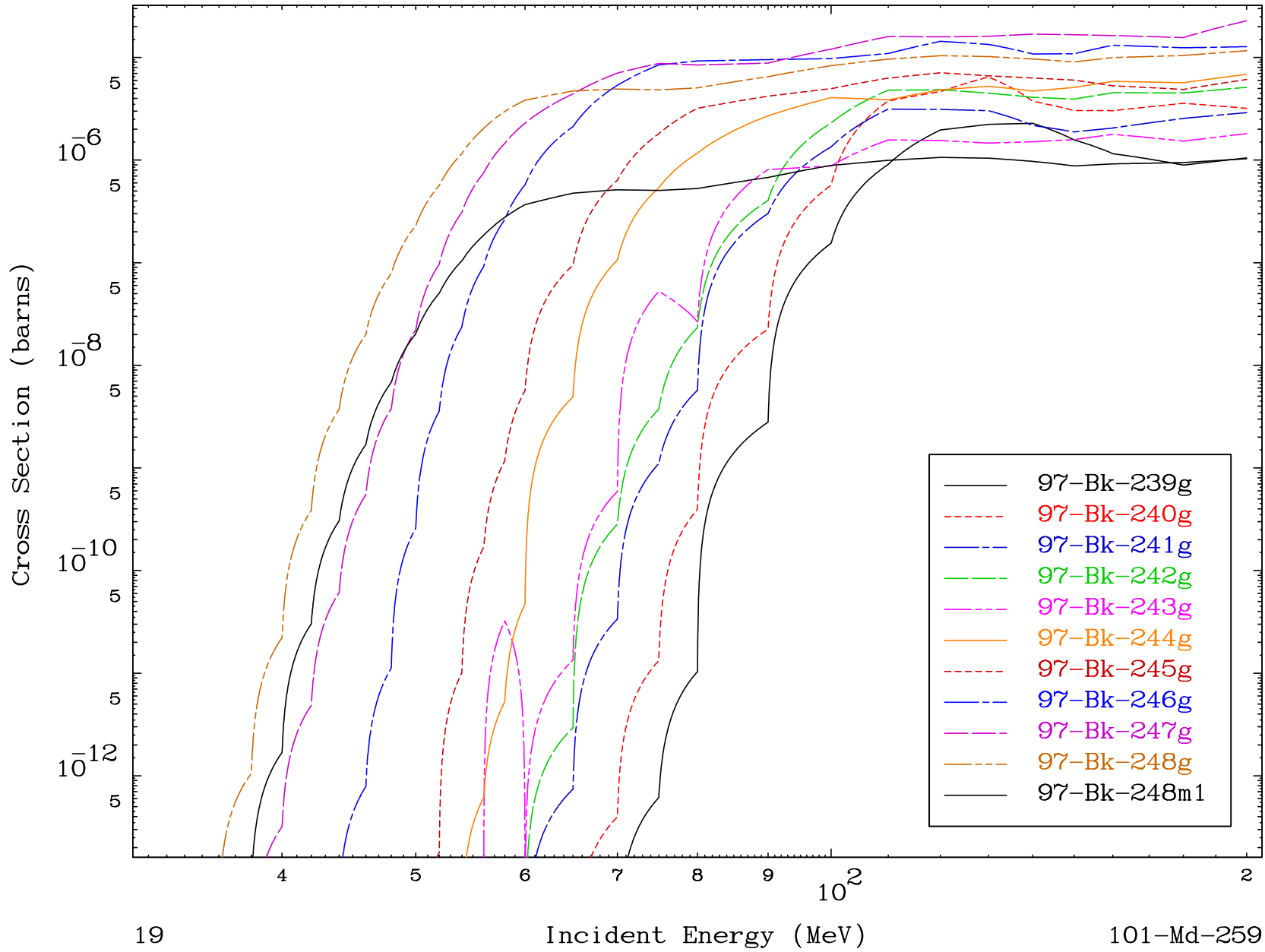


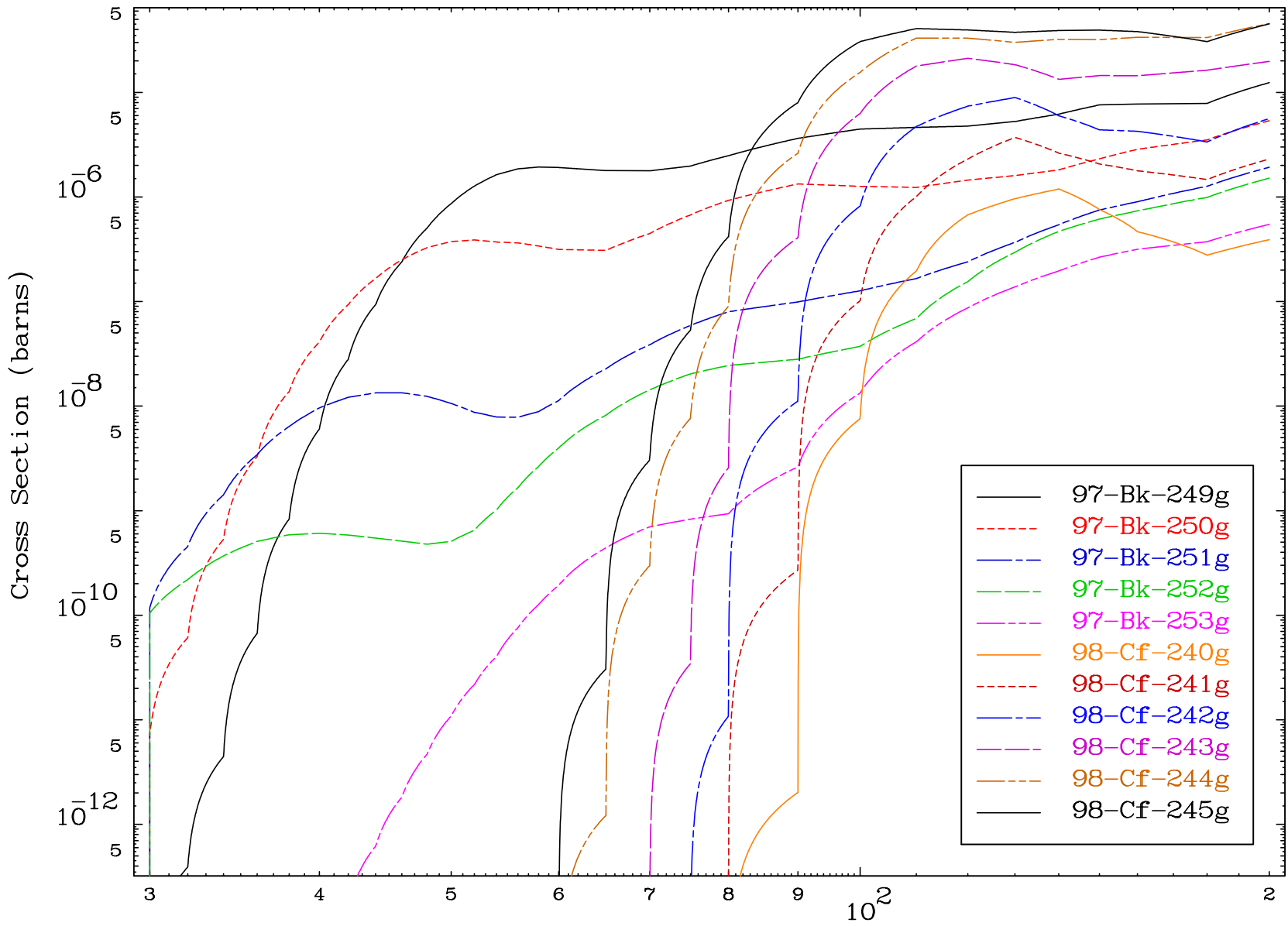
Radionuclide Production Cross Section



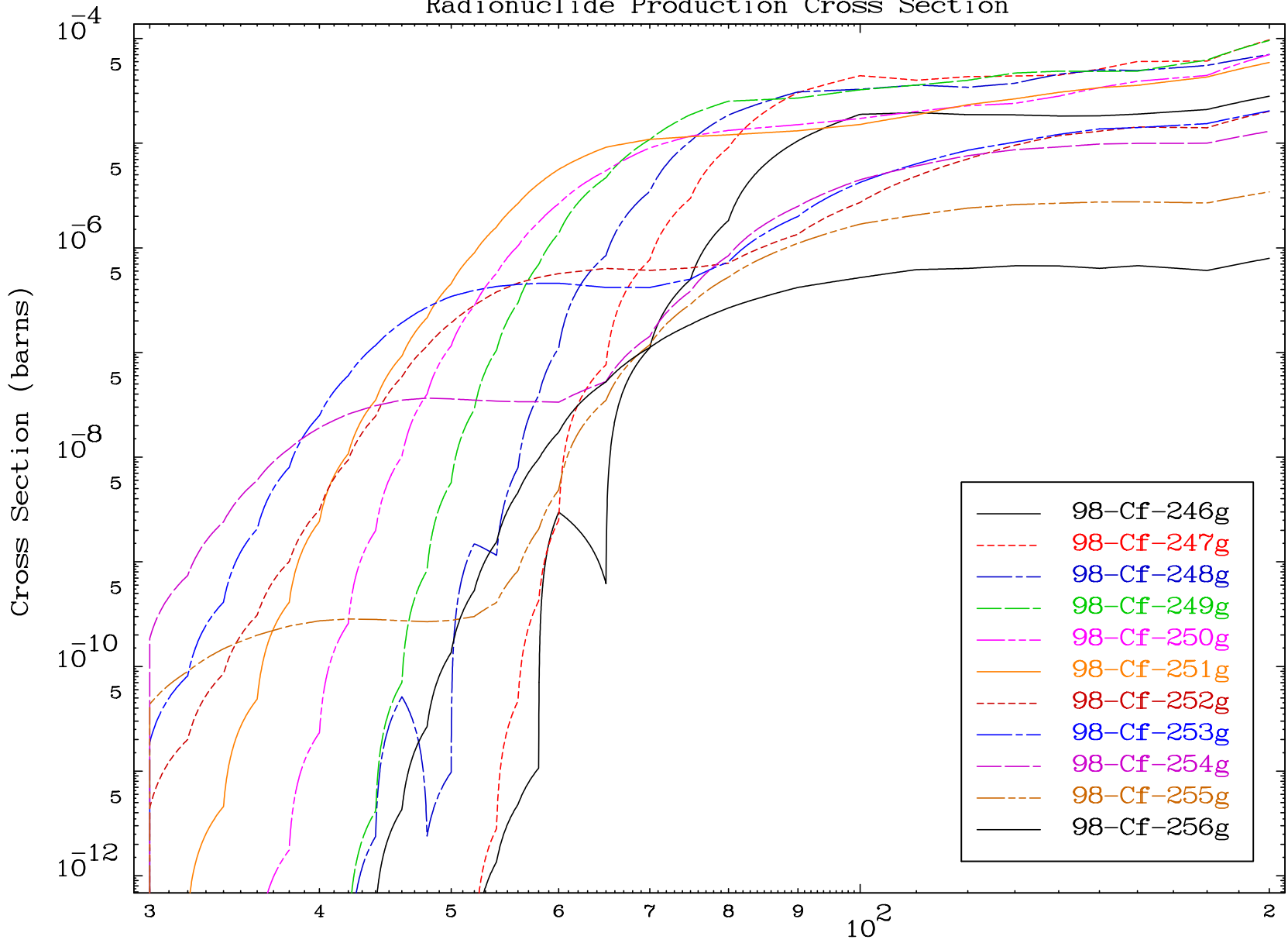
Radionuclide Production Cross Section

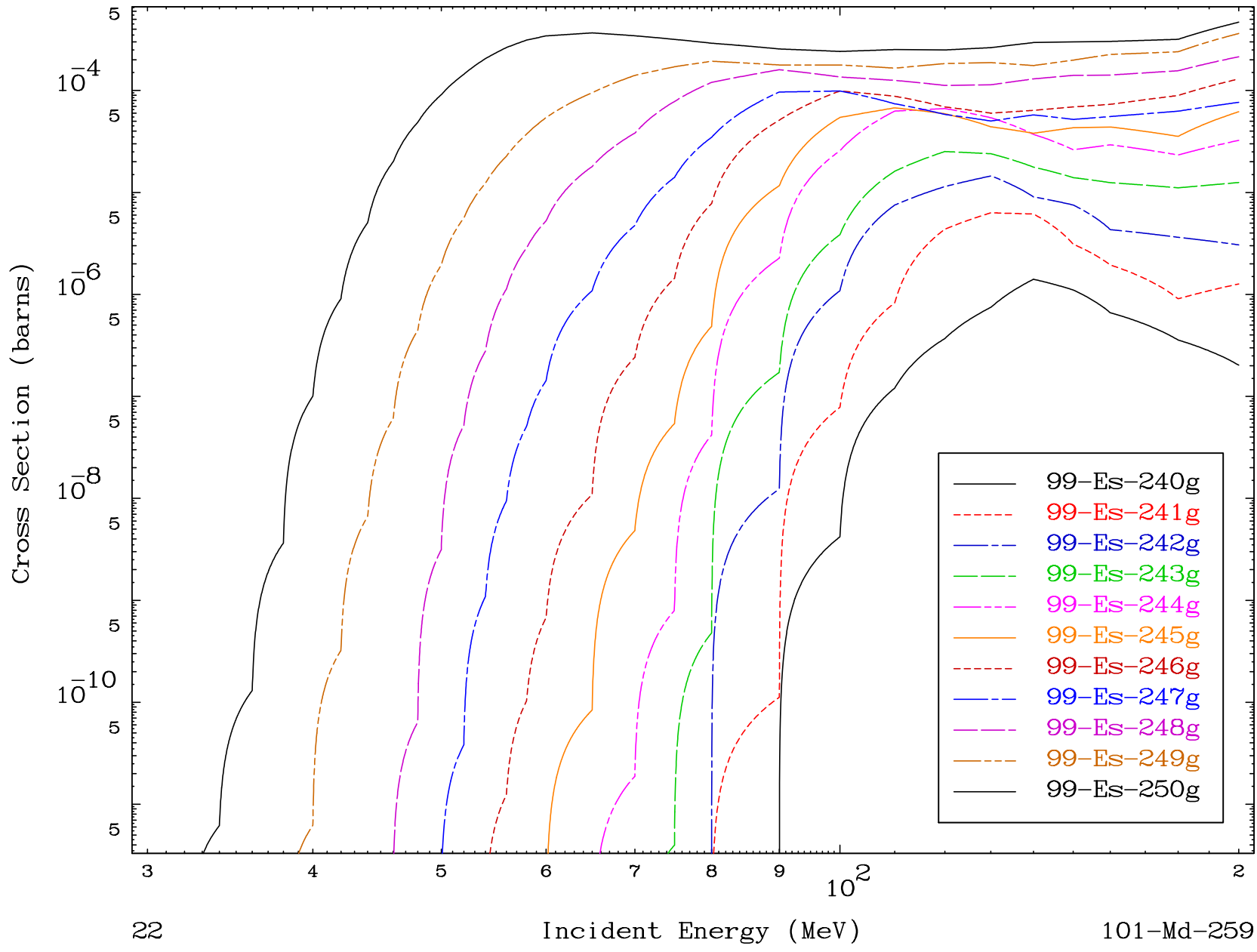




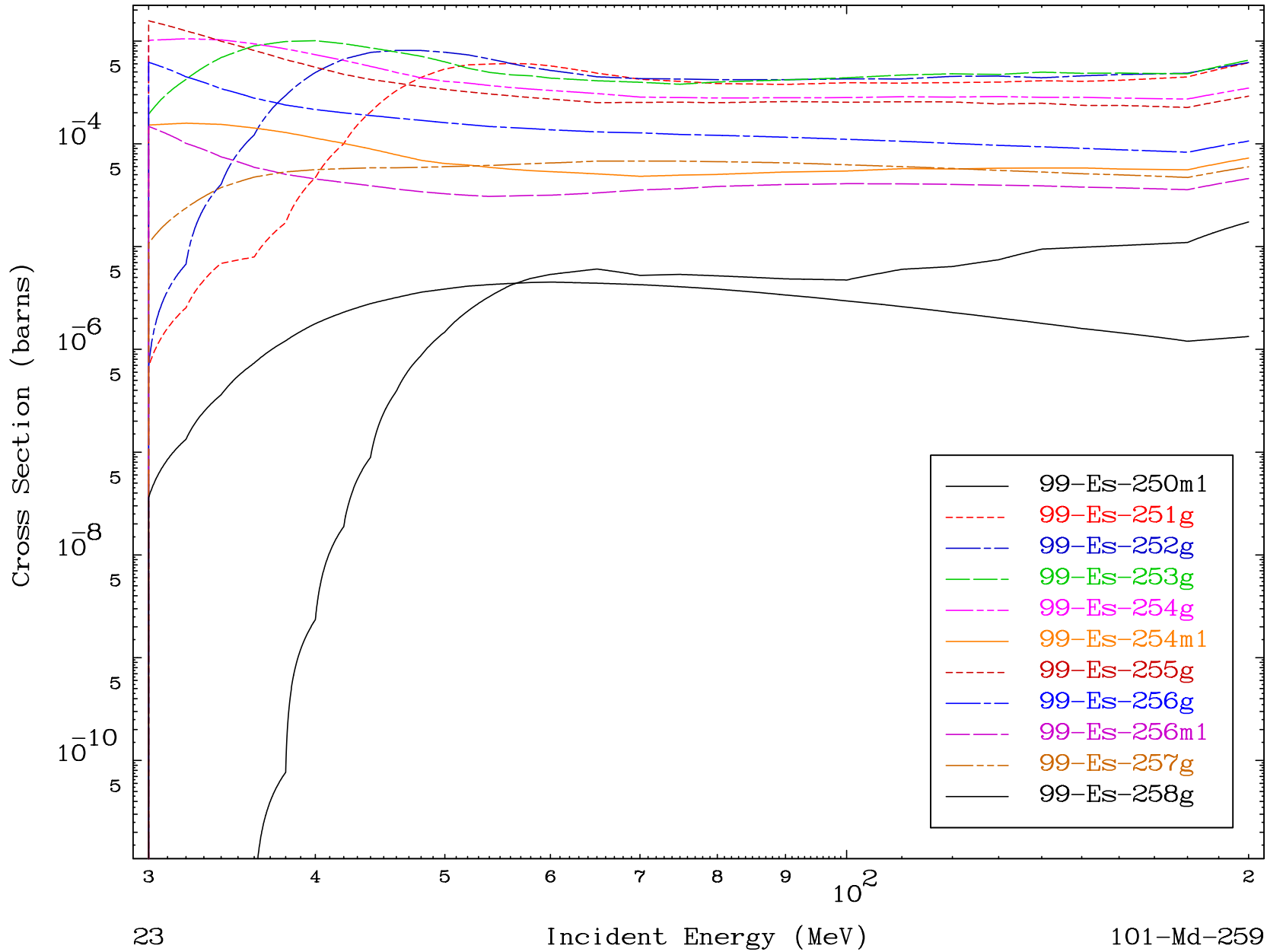


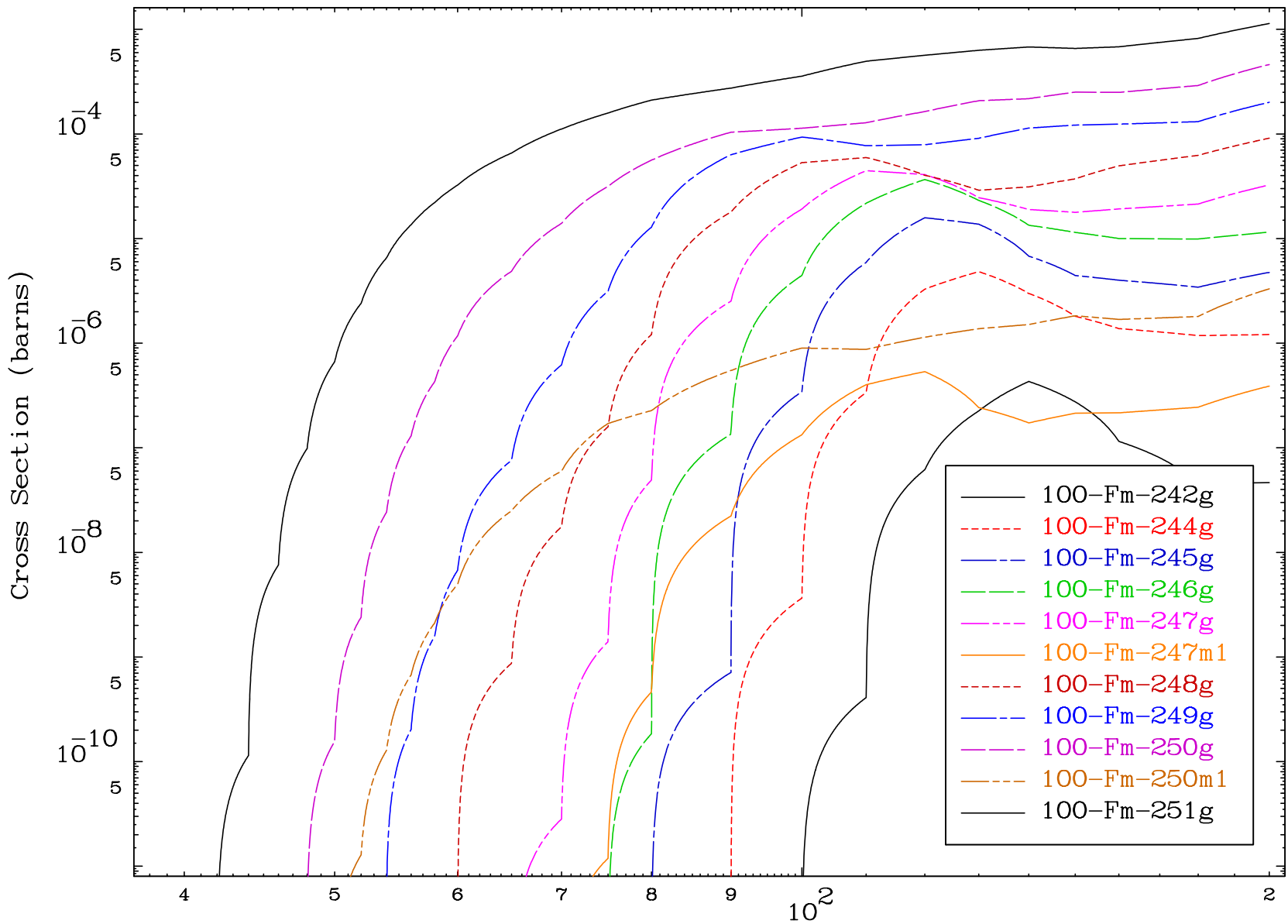
Radionuclide Production Cross Section





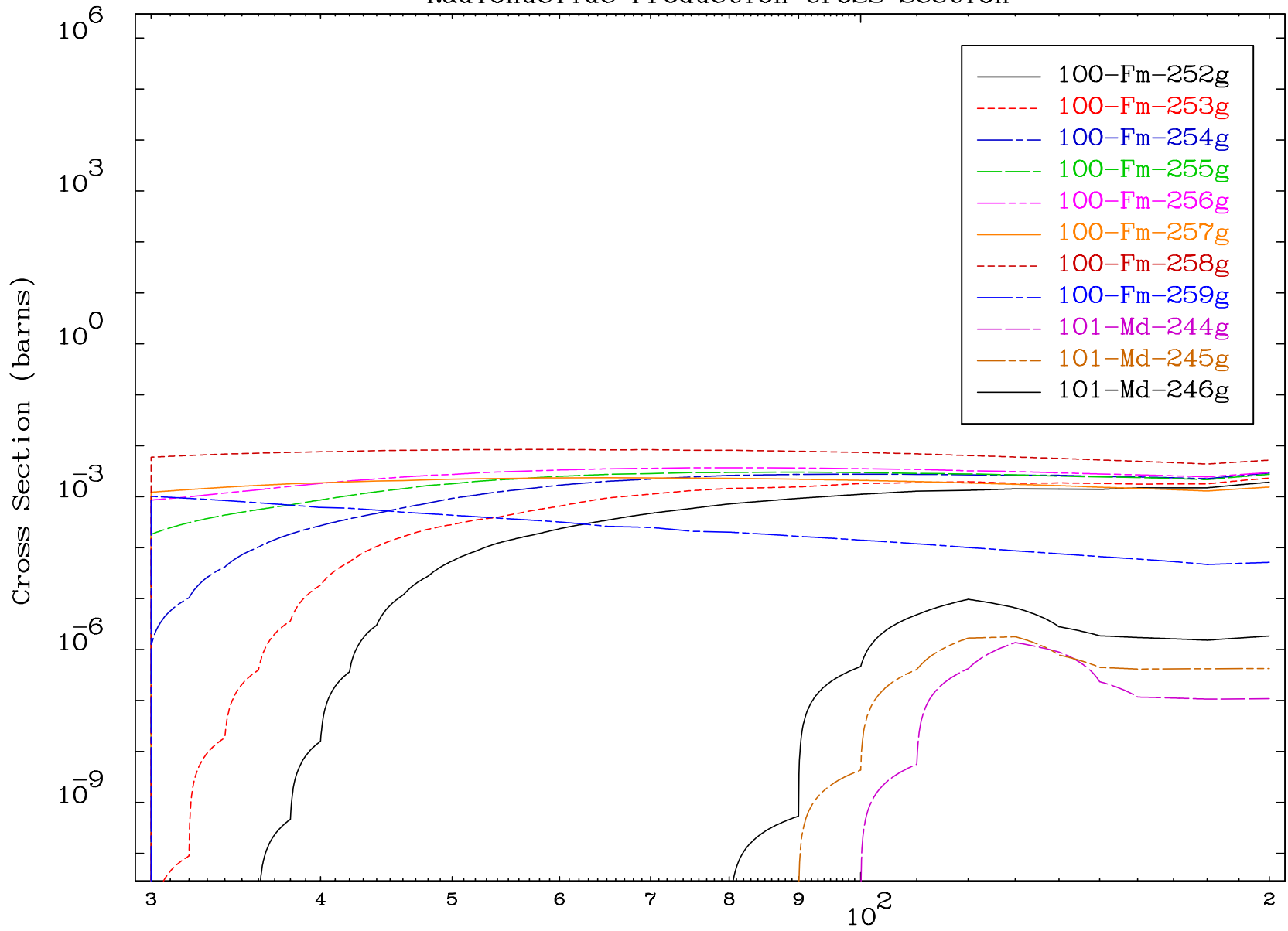
Radionuclide Production Cross Section



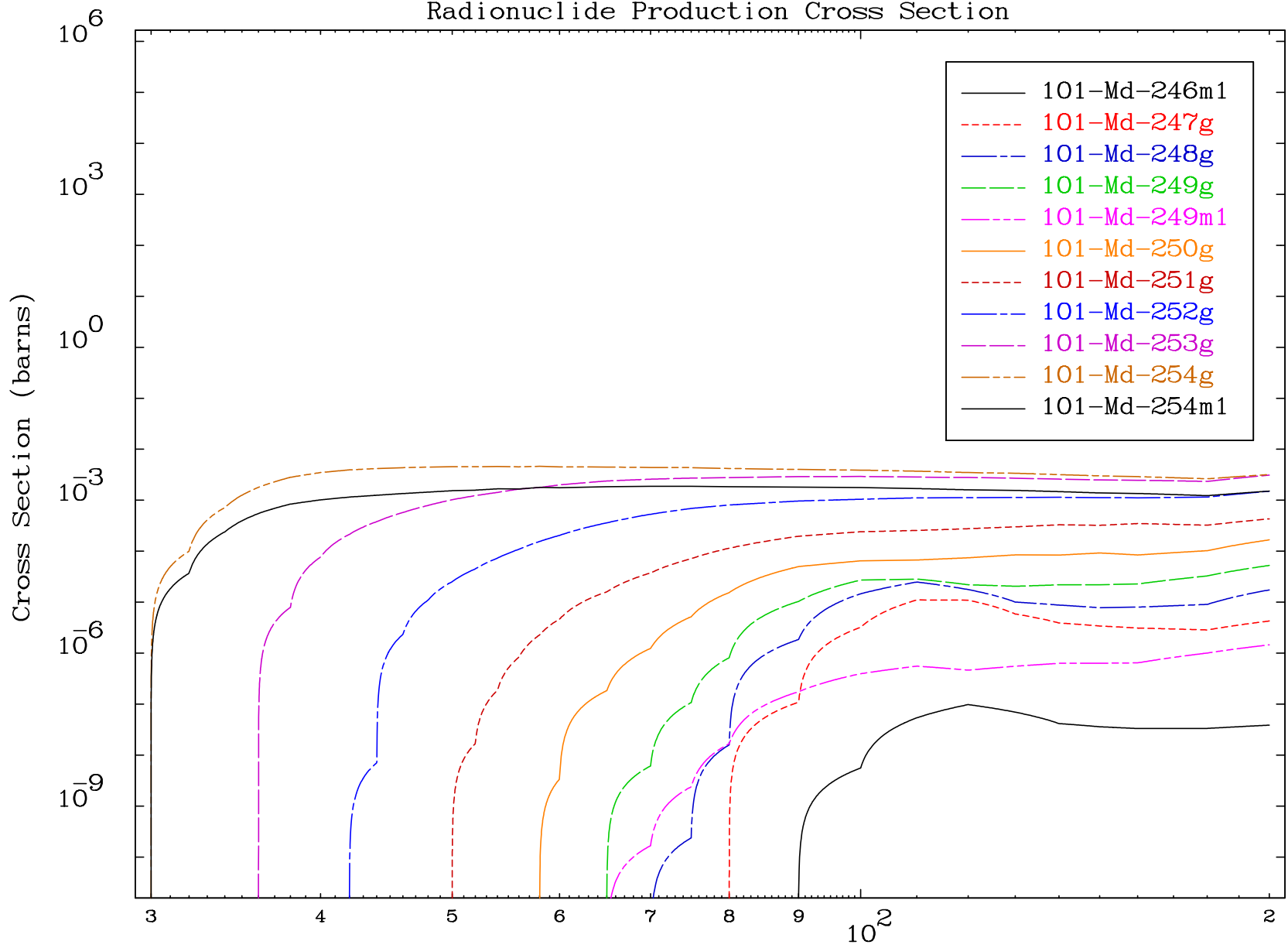


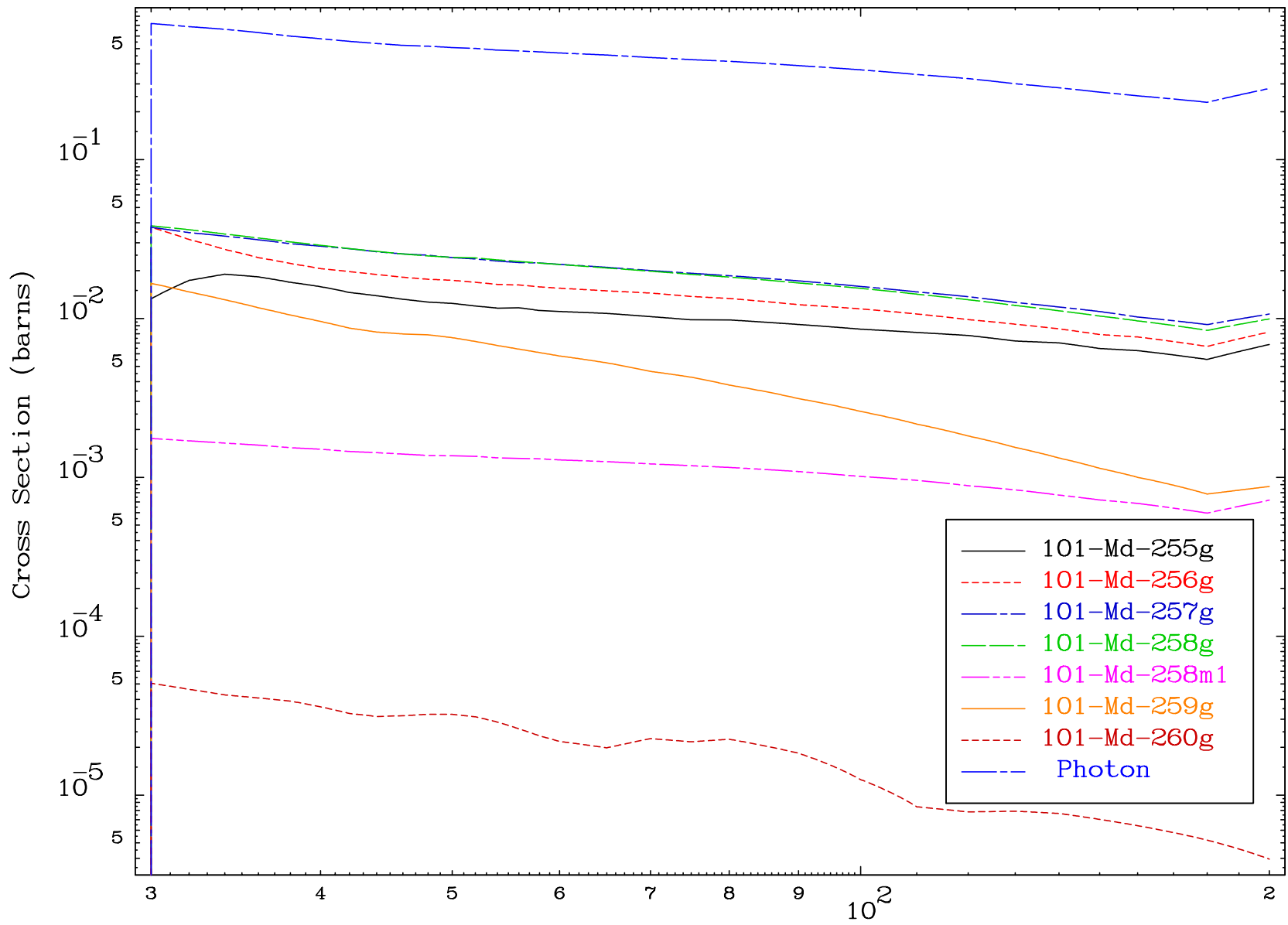


Radionuclide Production Cross Section

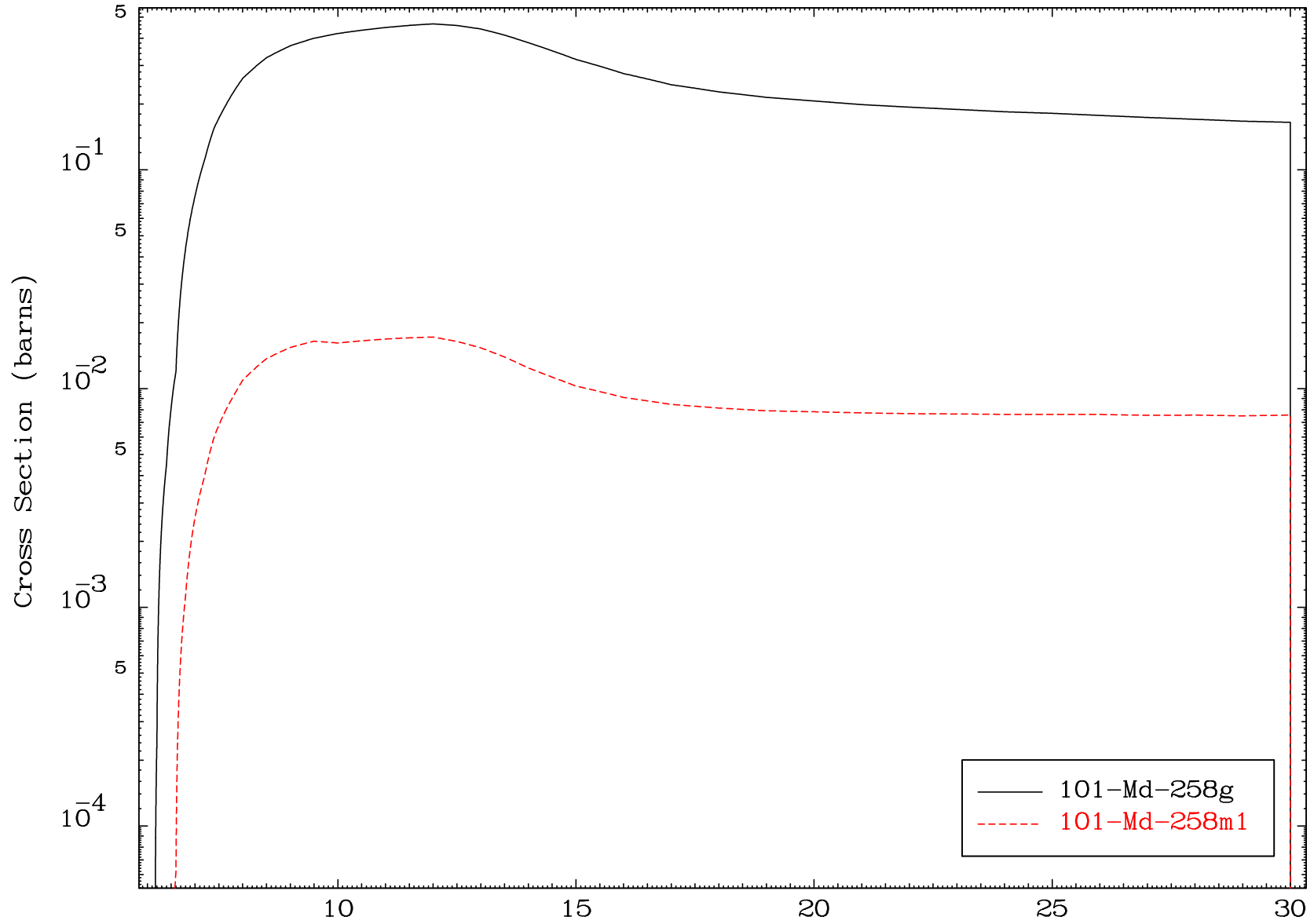


Radionuclide Production Cross Section





Radionuclide Production Cross Section

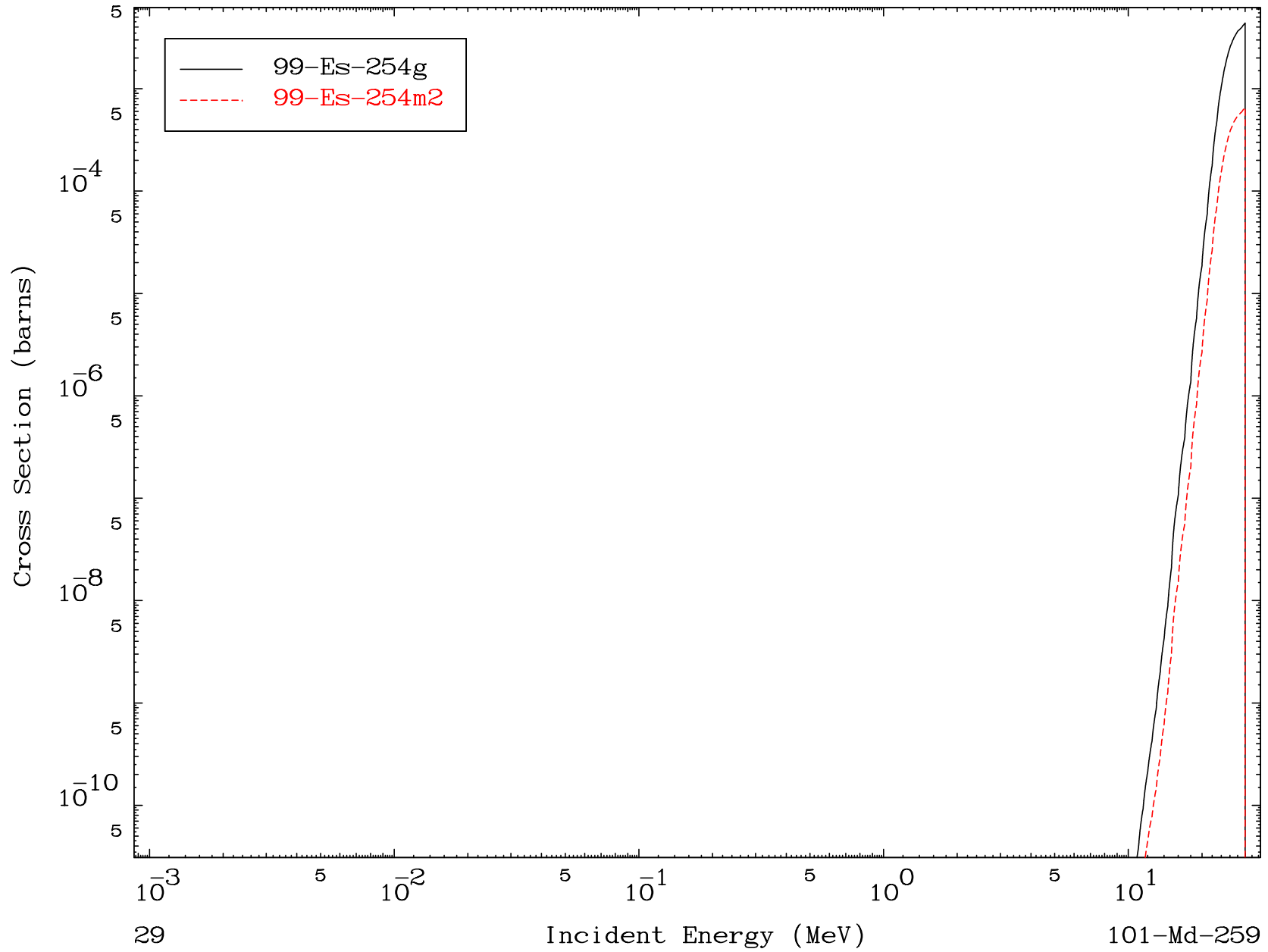


MAT 9960

(n,2n)  $\alpha$

101-Md-259

Radionuclide Production Cross Section

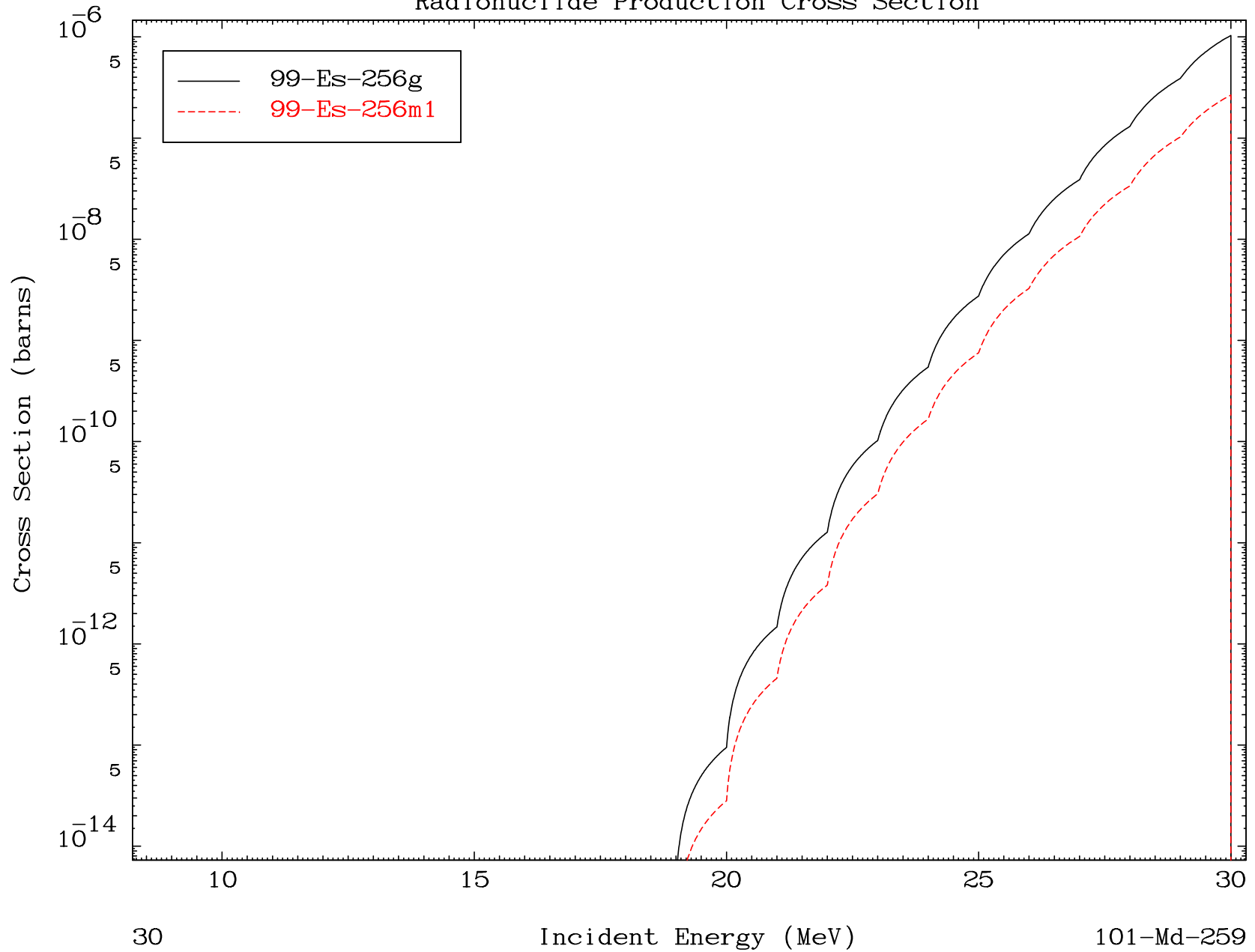


MAT 9960

(n,n') He-3

101-Md-259

Radionuclide Production Cross Section



MAT 9960

(n,  $\alpha$ )

101-Md-259

Radionuclide Production Cross Section

