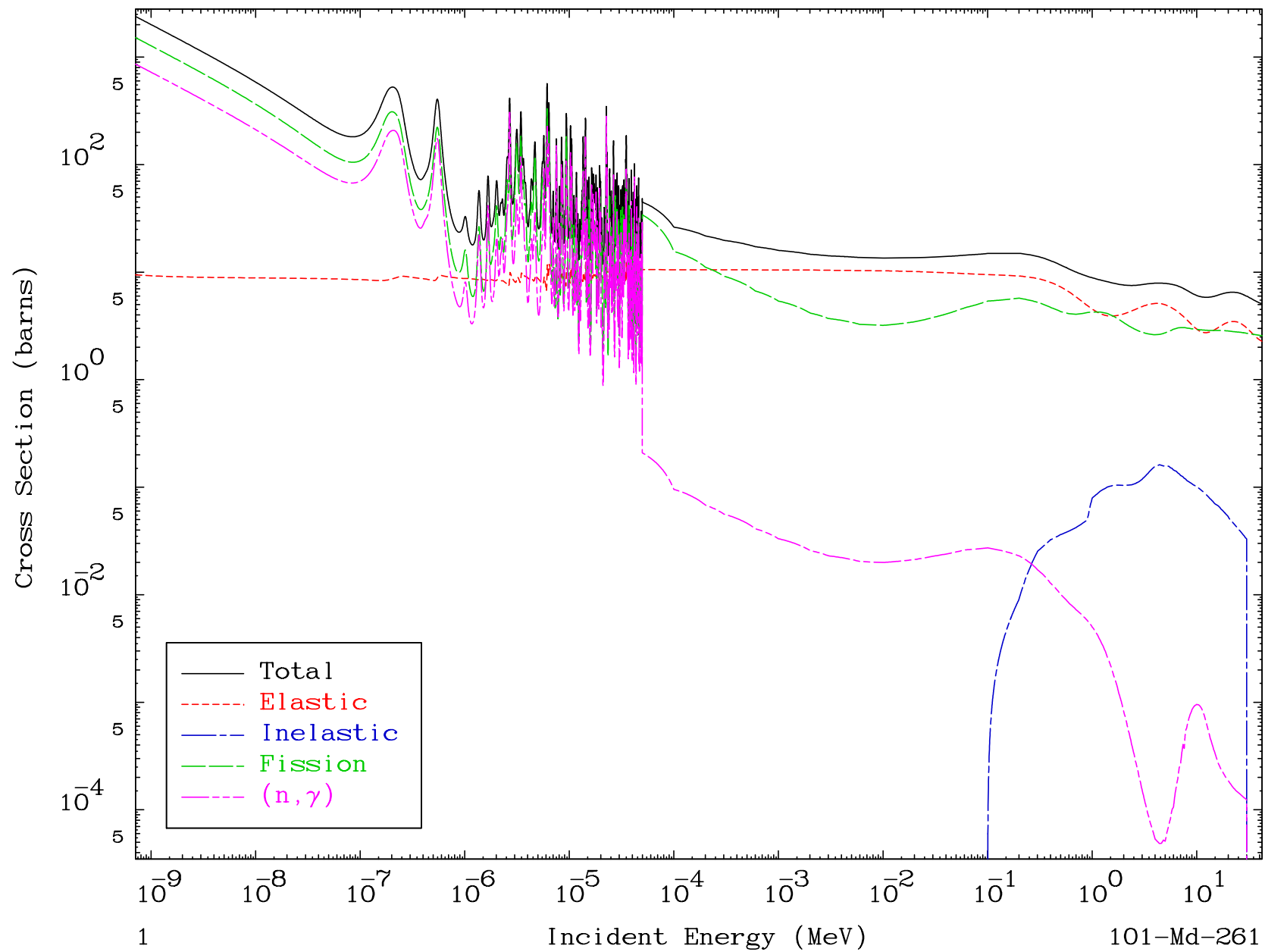
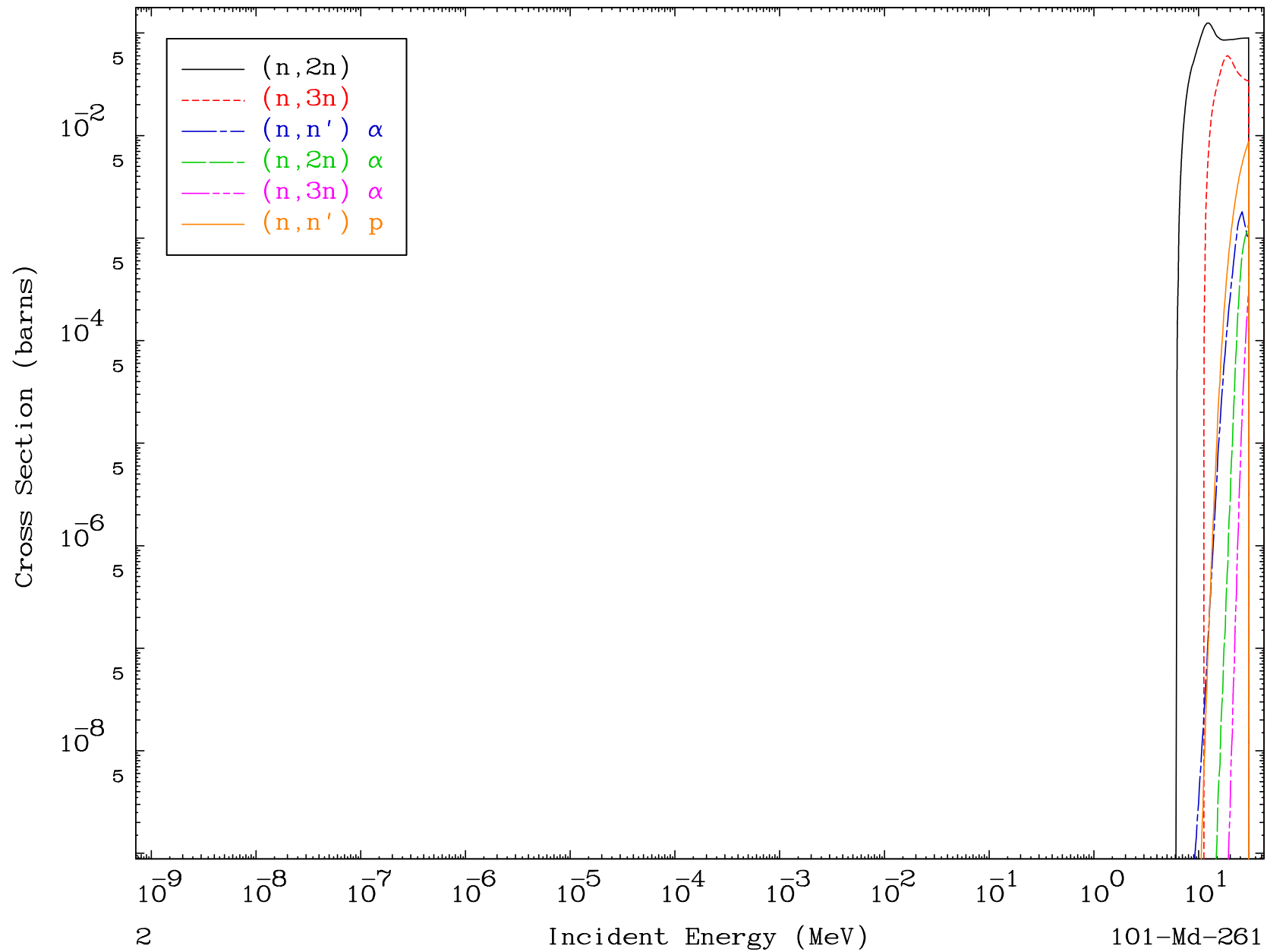


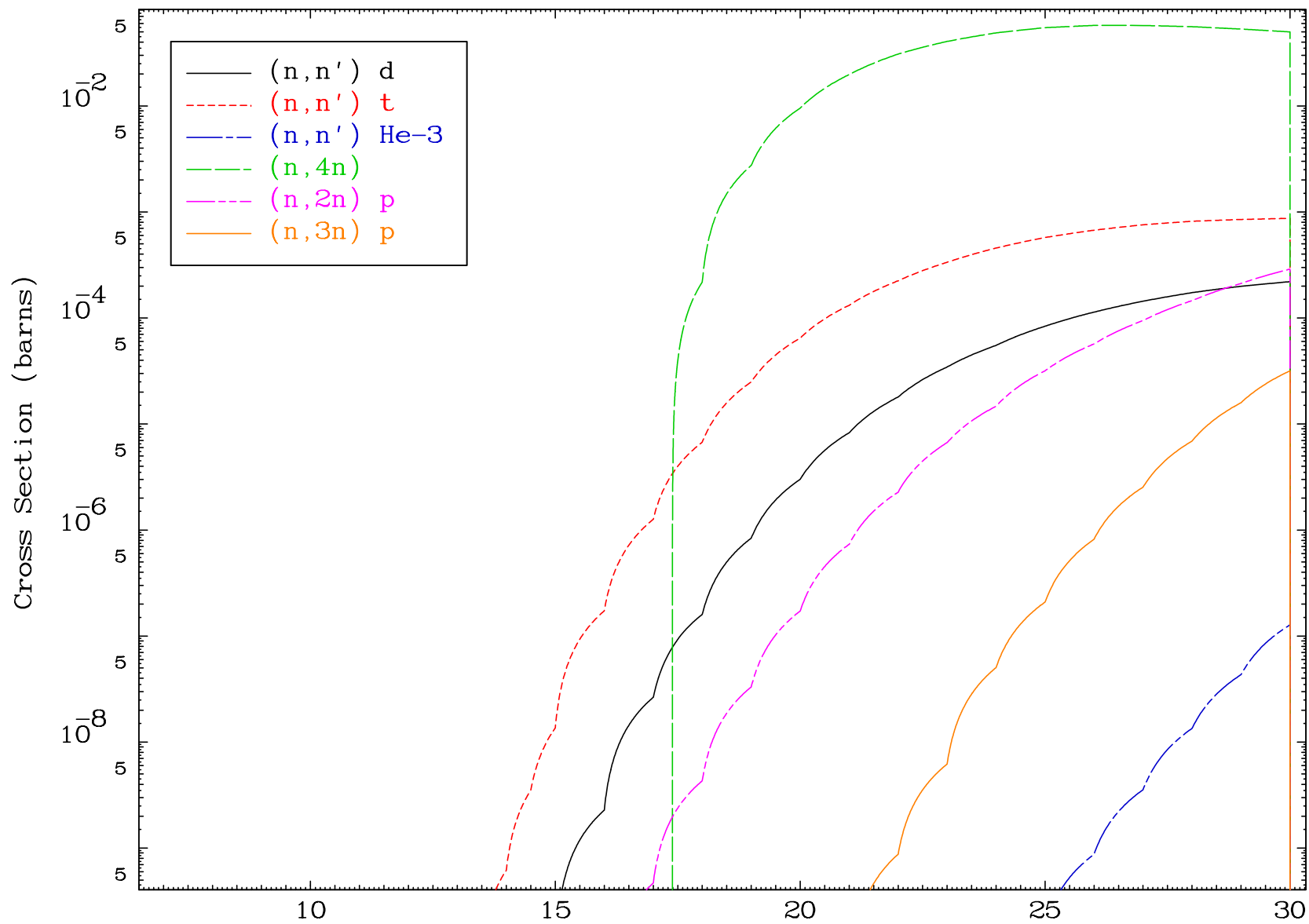
MAT 9962

Major
294 Kelvin Cross Sections

101-Md-261



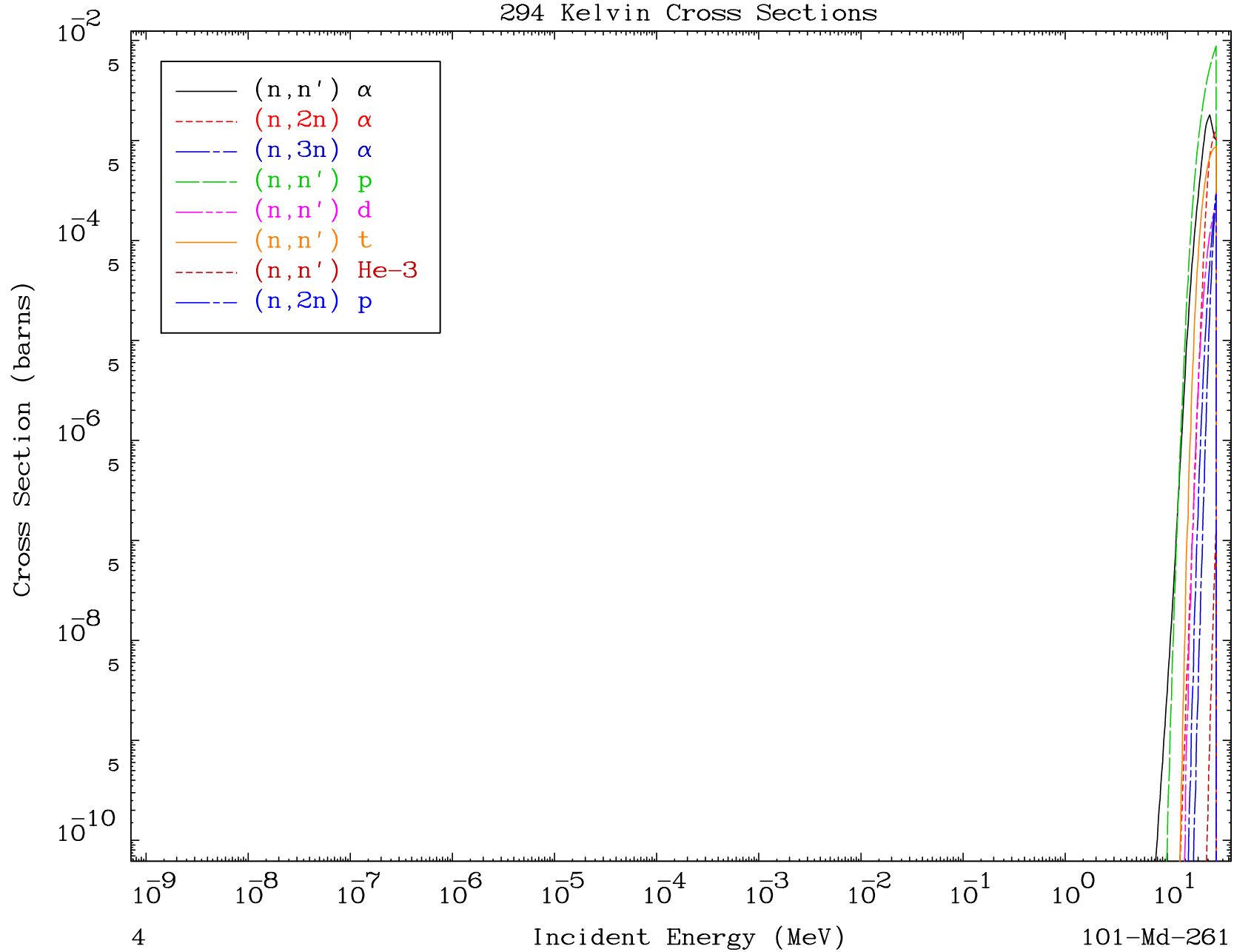


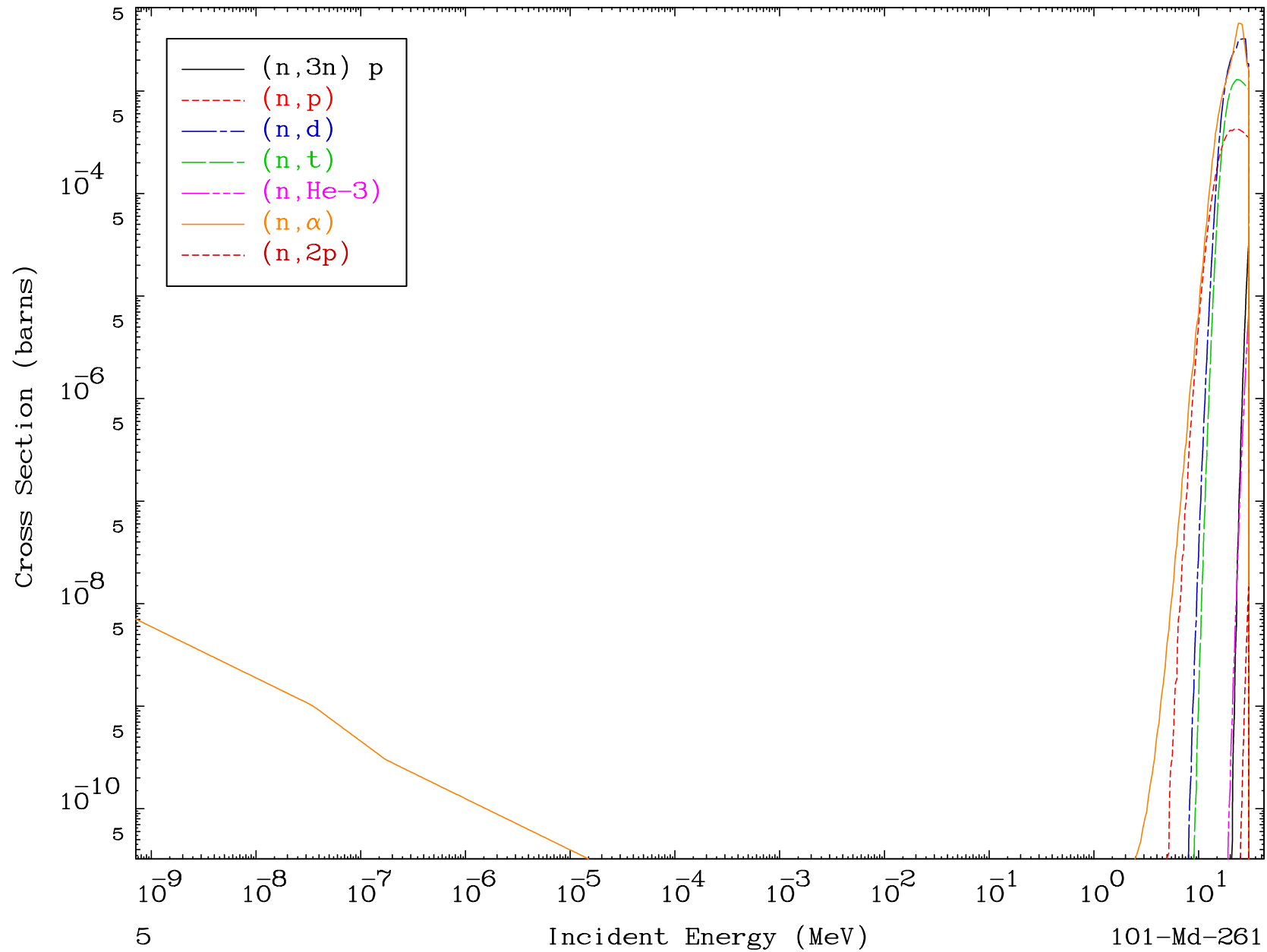


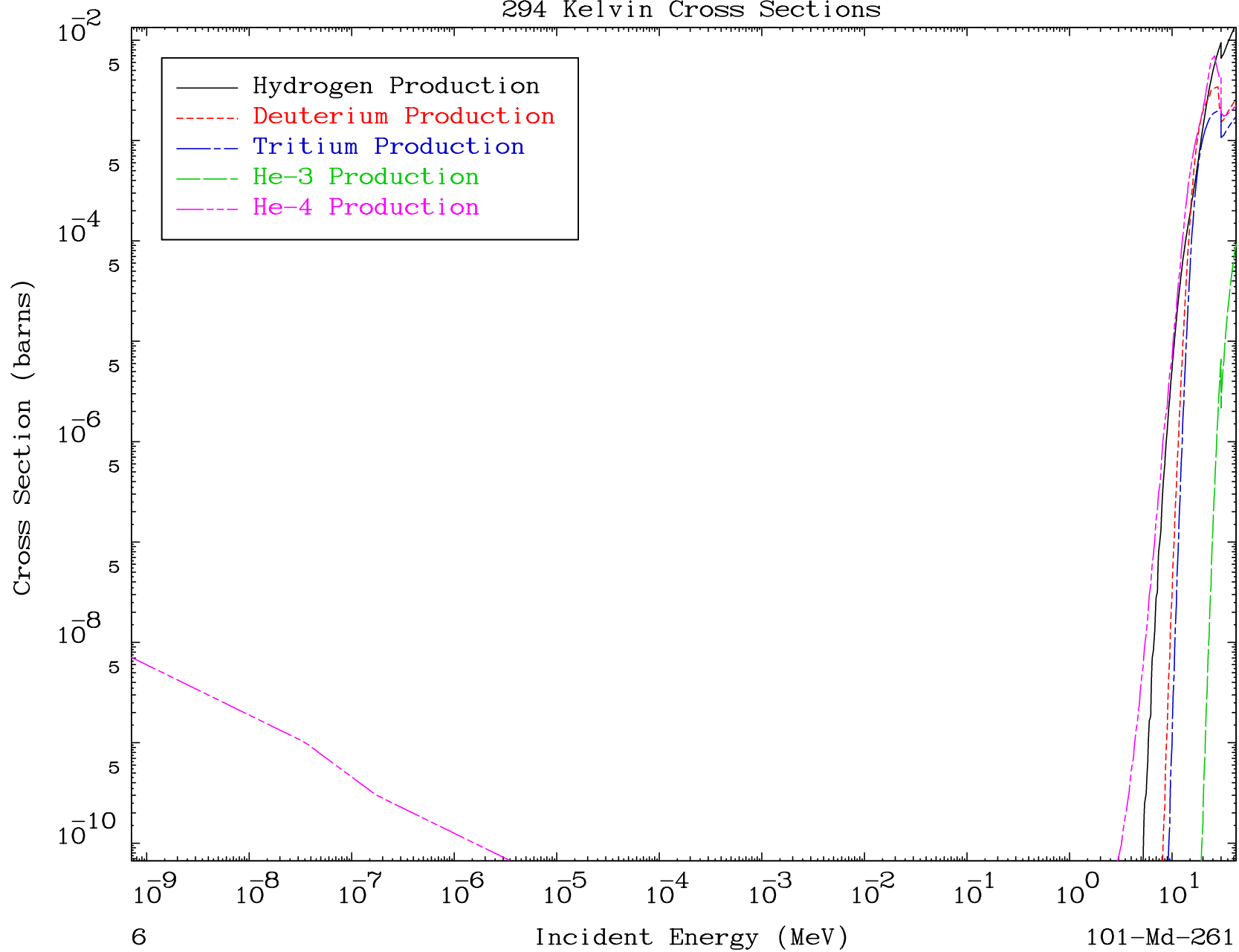
MAT 9962

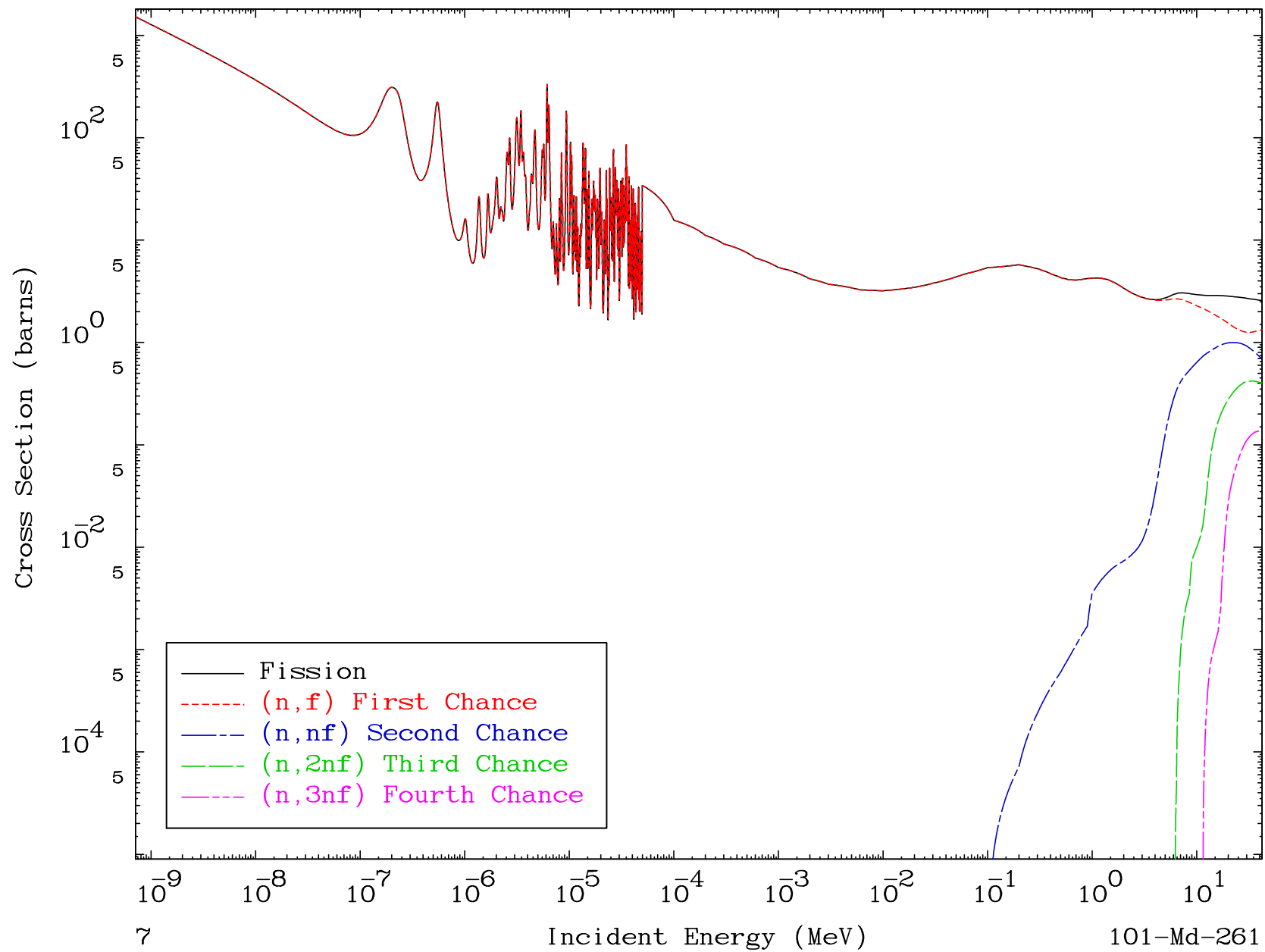
Charged Particle
294 Kelvin Cross Sections

101-Md-261





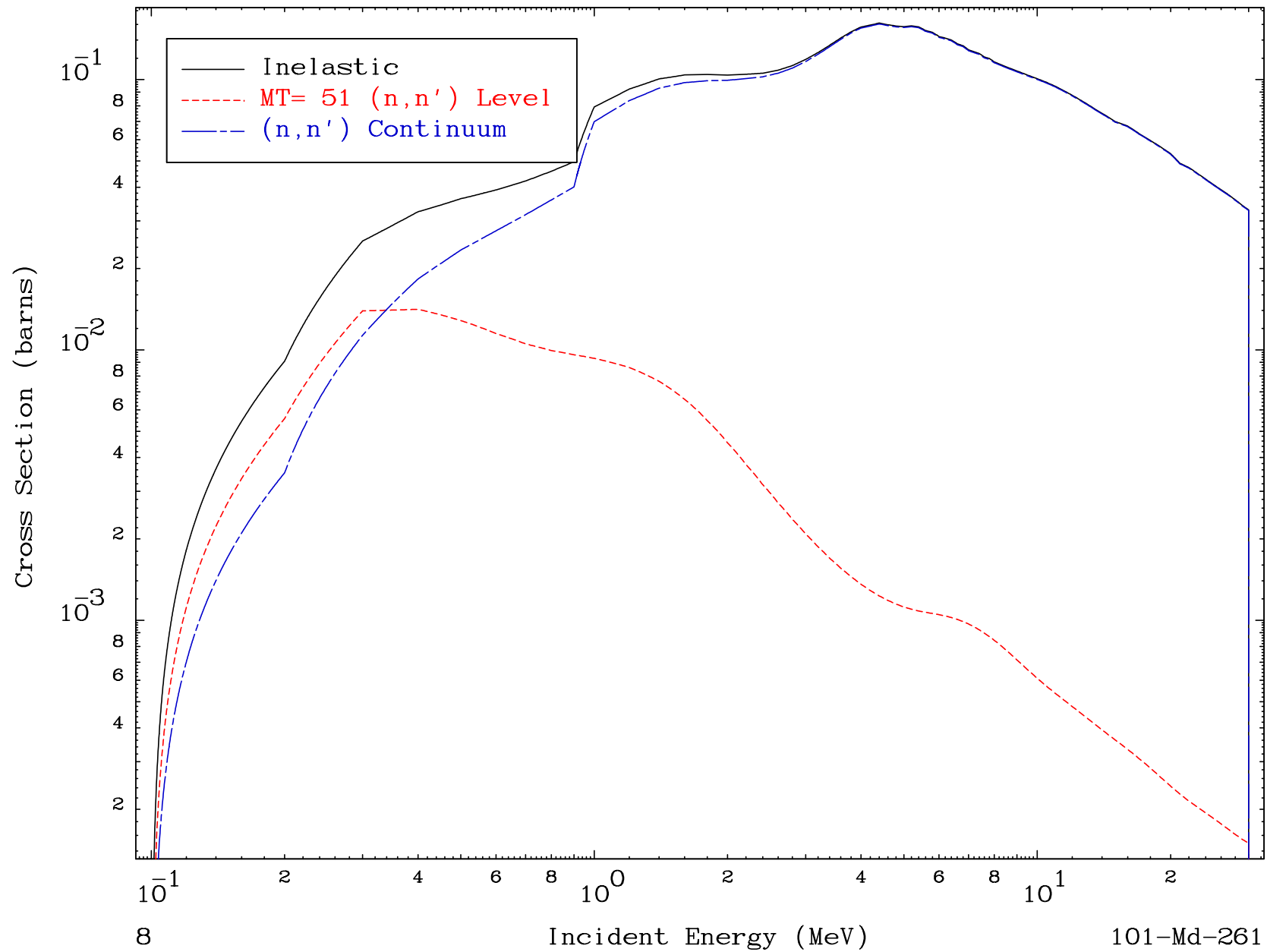




MAT 9962

 (n,n') Level
294 Kelvin Cross Sections

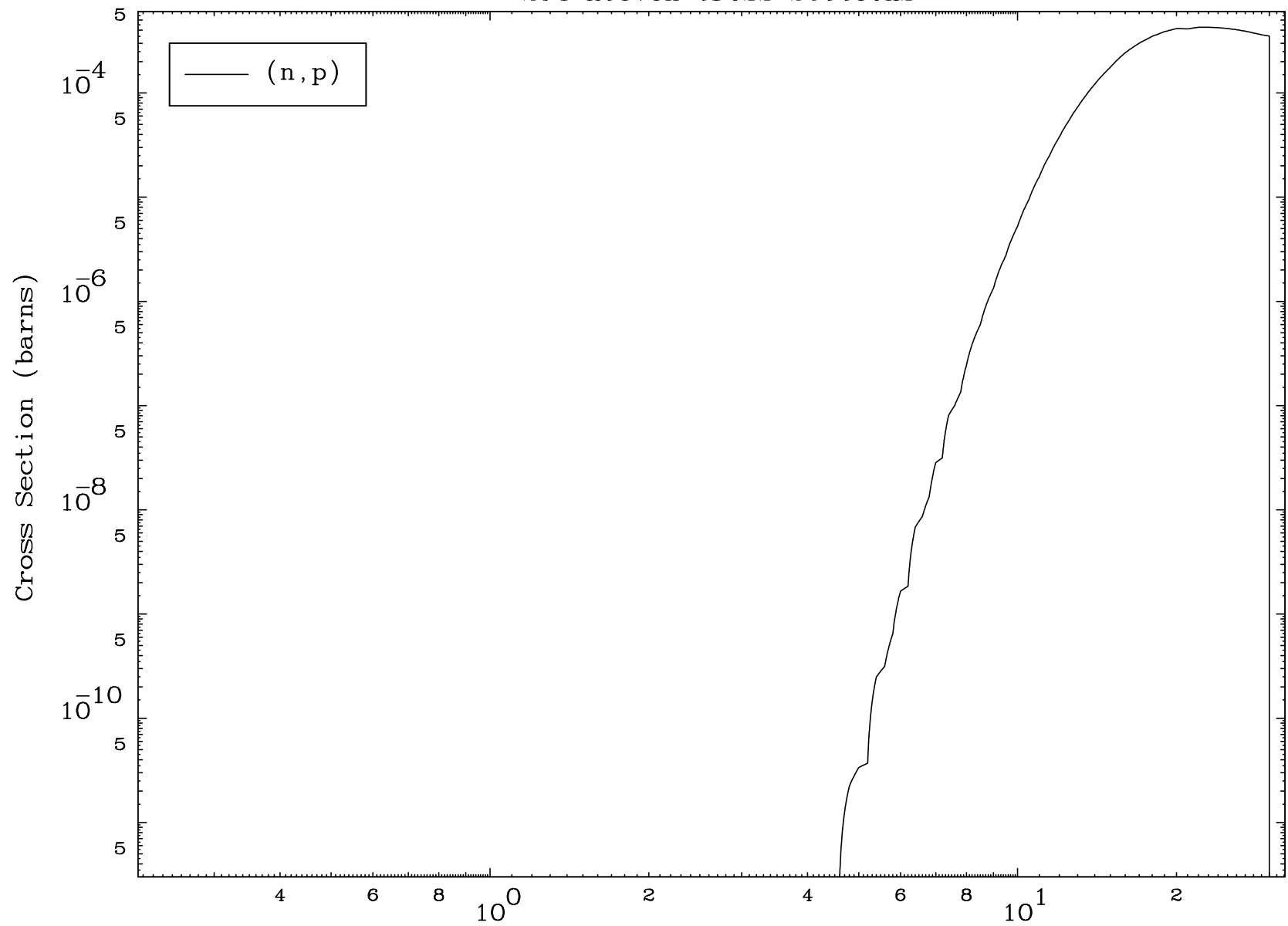
101-Md-261



MAT 9962

(n,p) Levels
294 Kelvin Cross Sections

101-Md-261



9

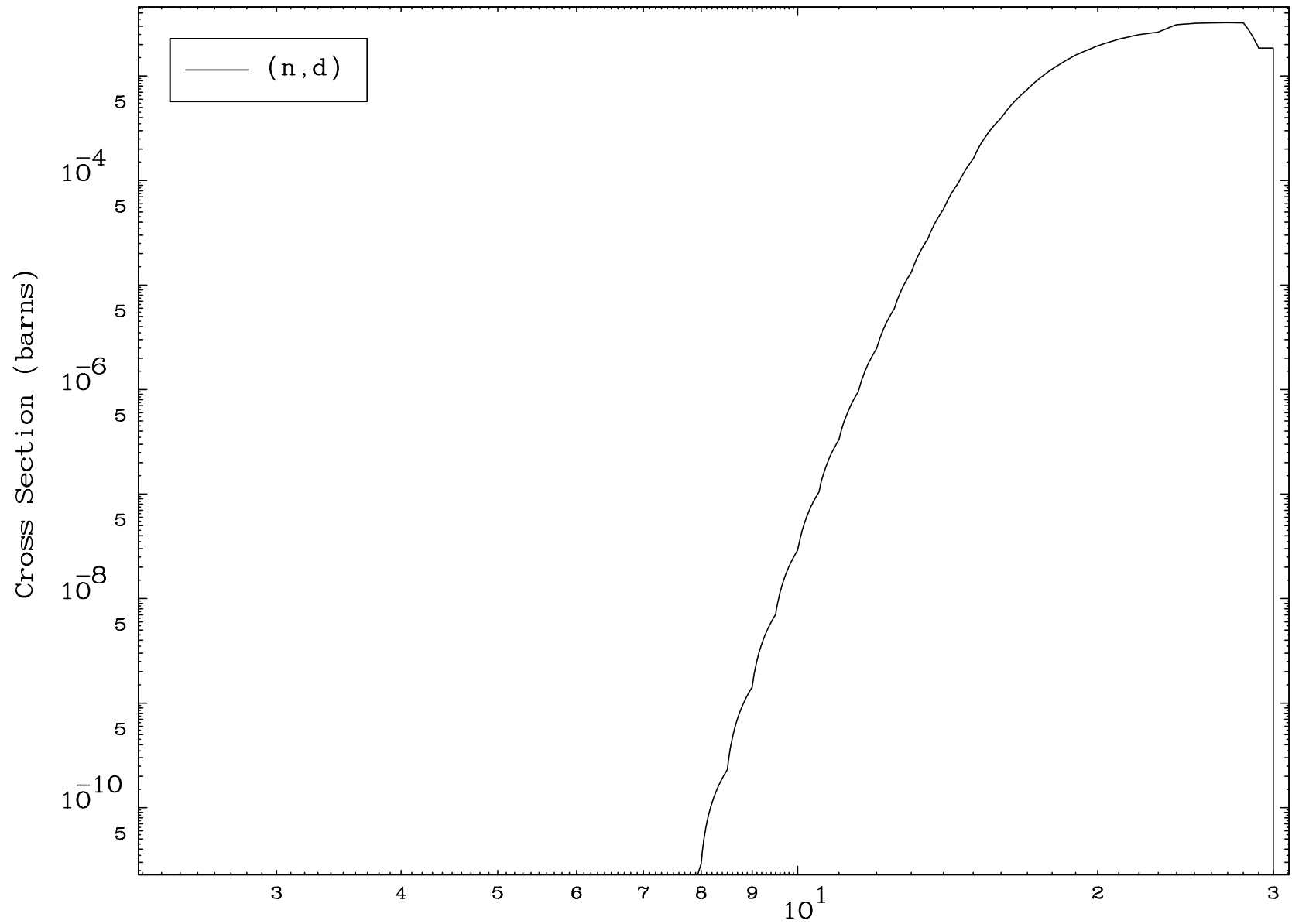
Incident Energy (MeV)

101-Md-261

MAT 9962

(n,d) Levels
294 Kelvin Cross Sections

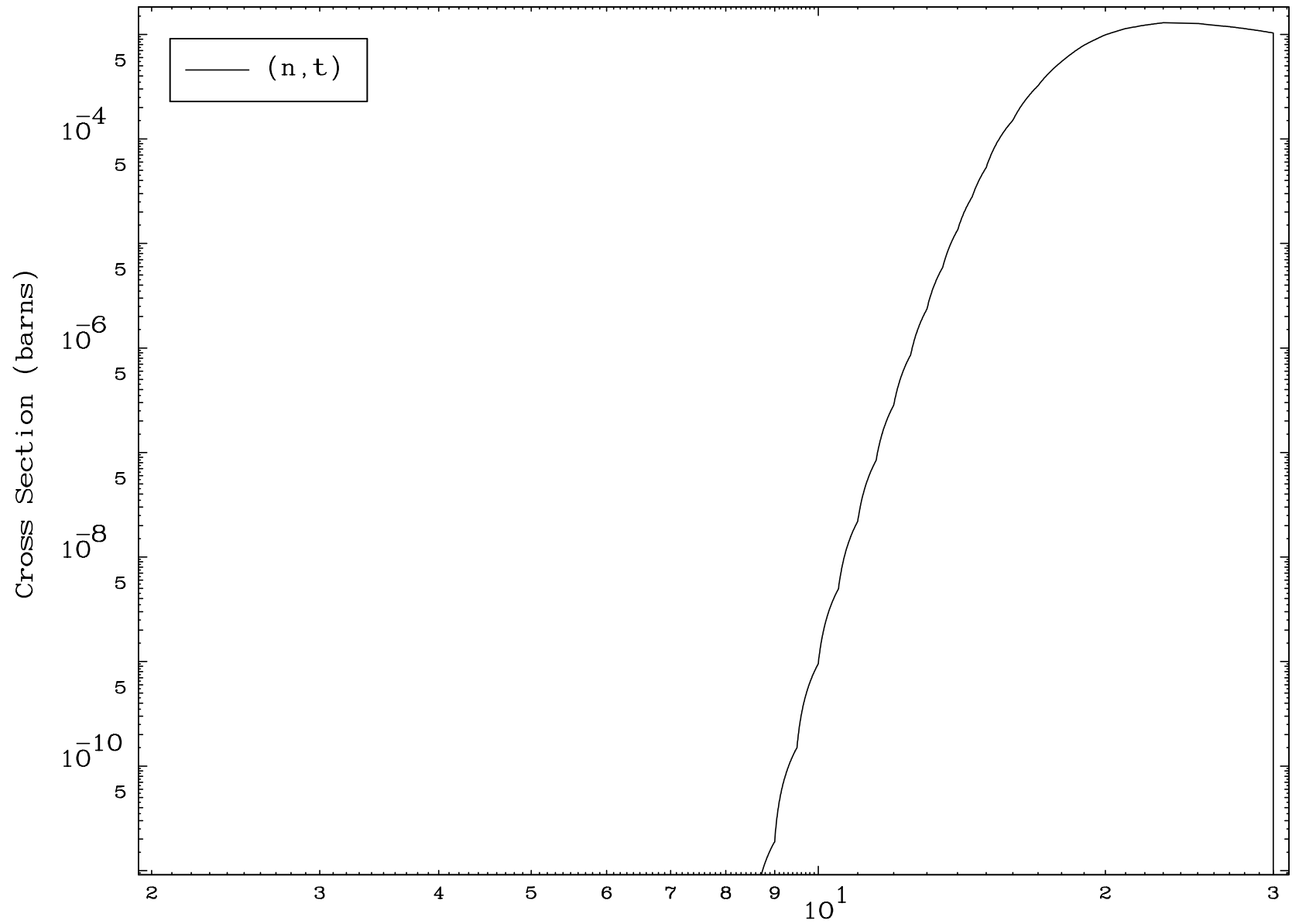
101-Md-261

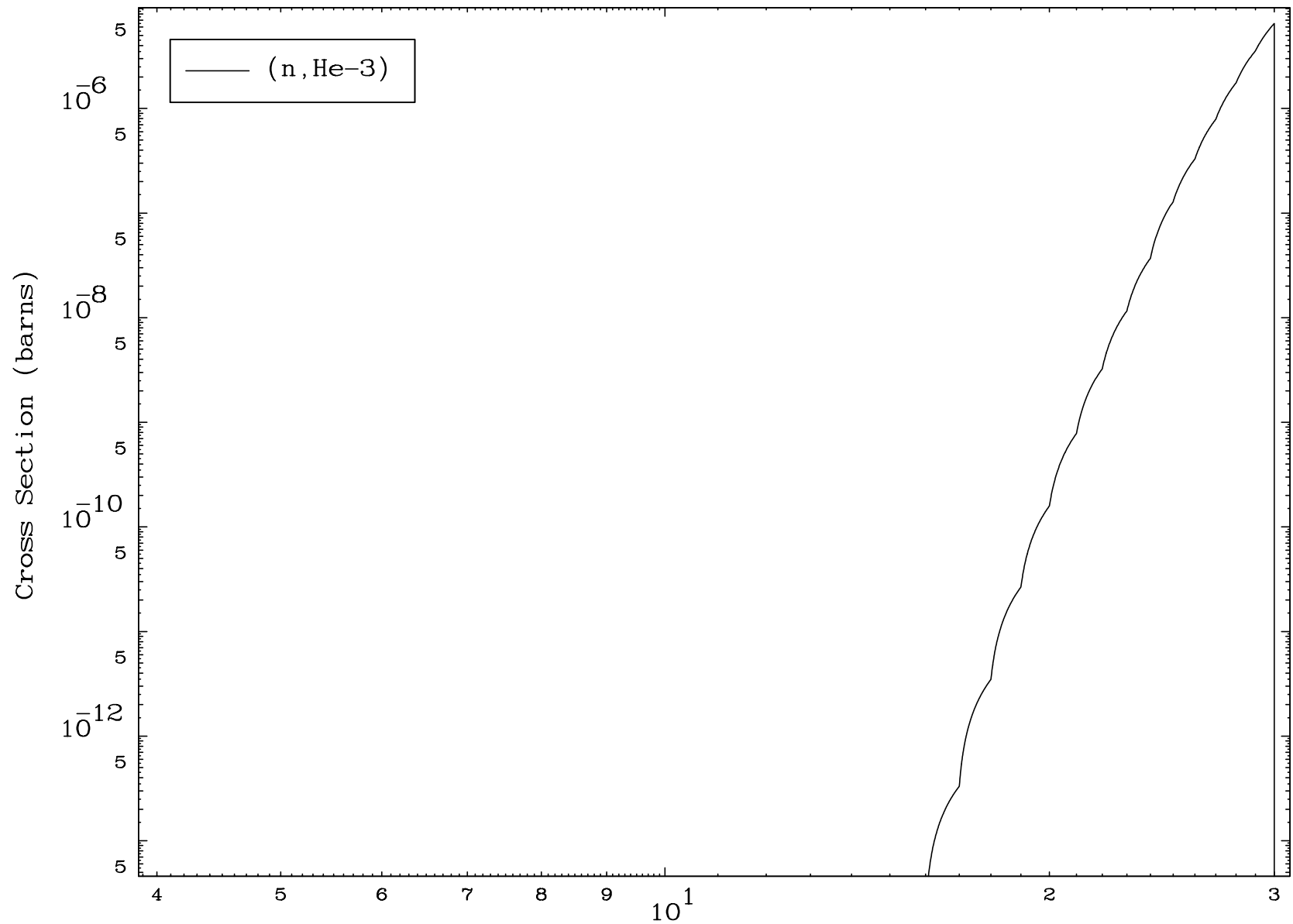


10

Incident Energy (MeV)

101-Md-261

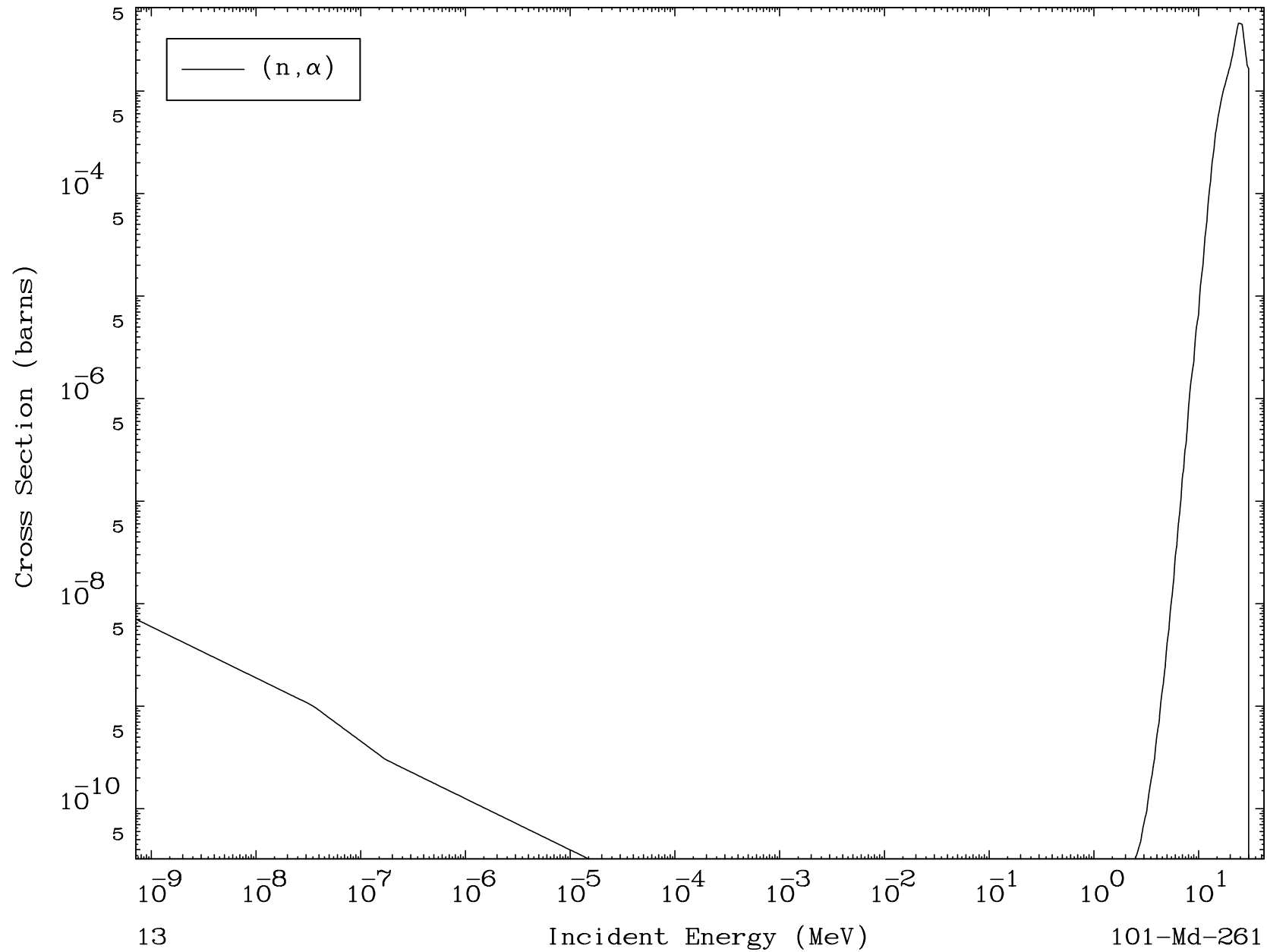




MAT 9962

(n, α) Levels
294 Kelvin Cross Sections

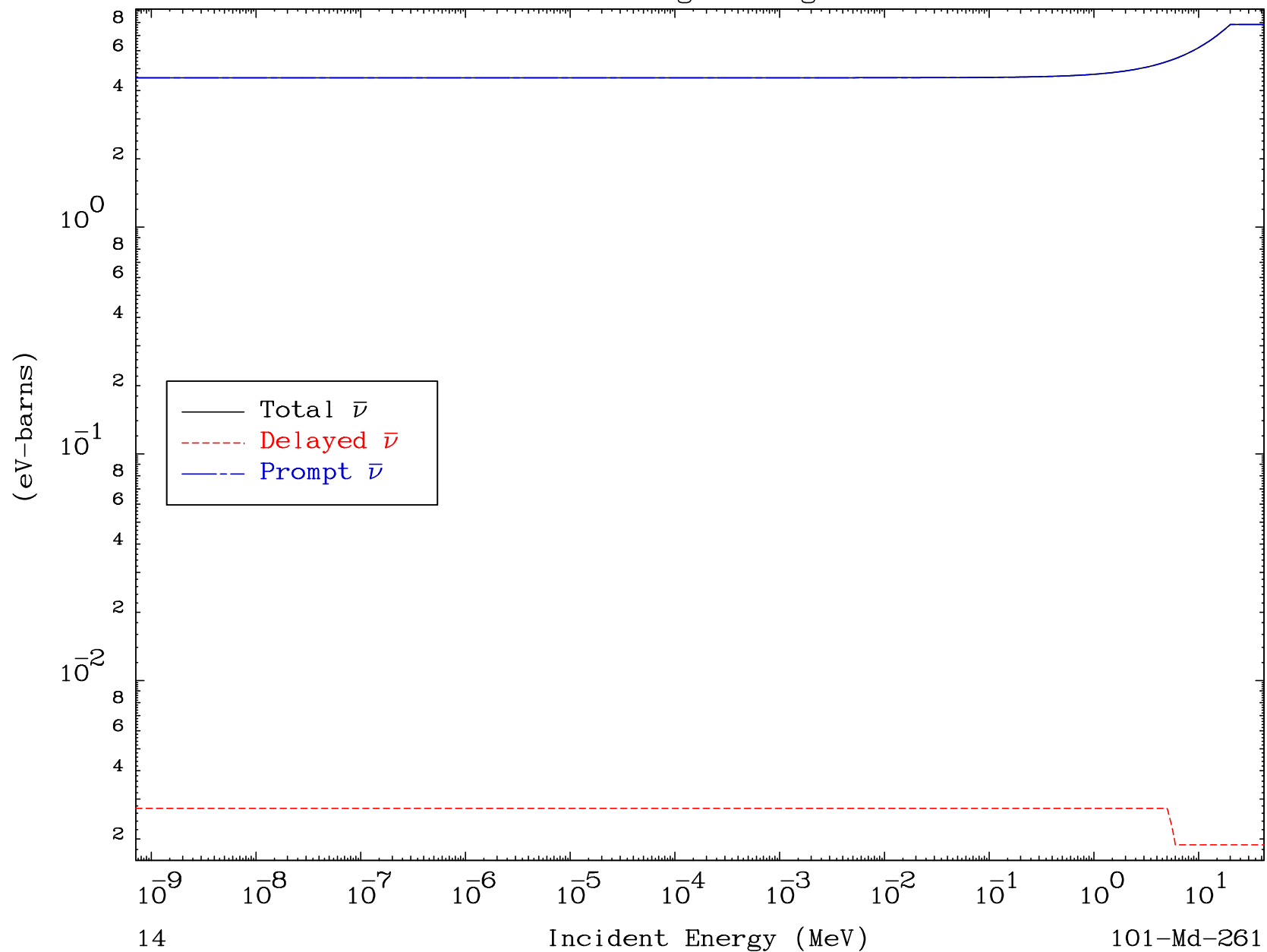
101-Md-261

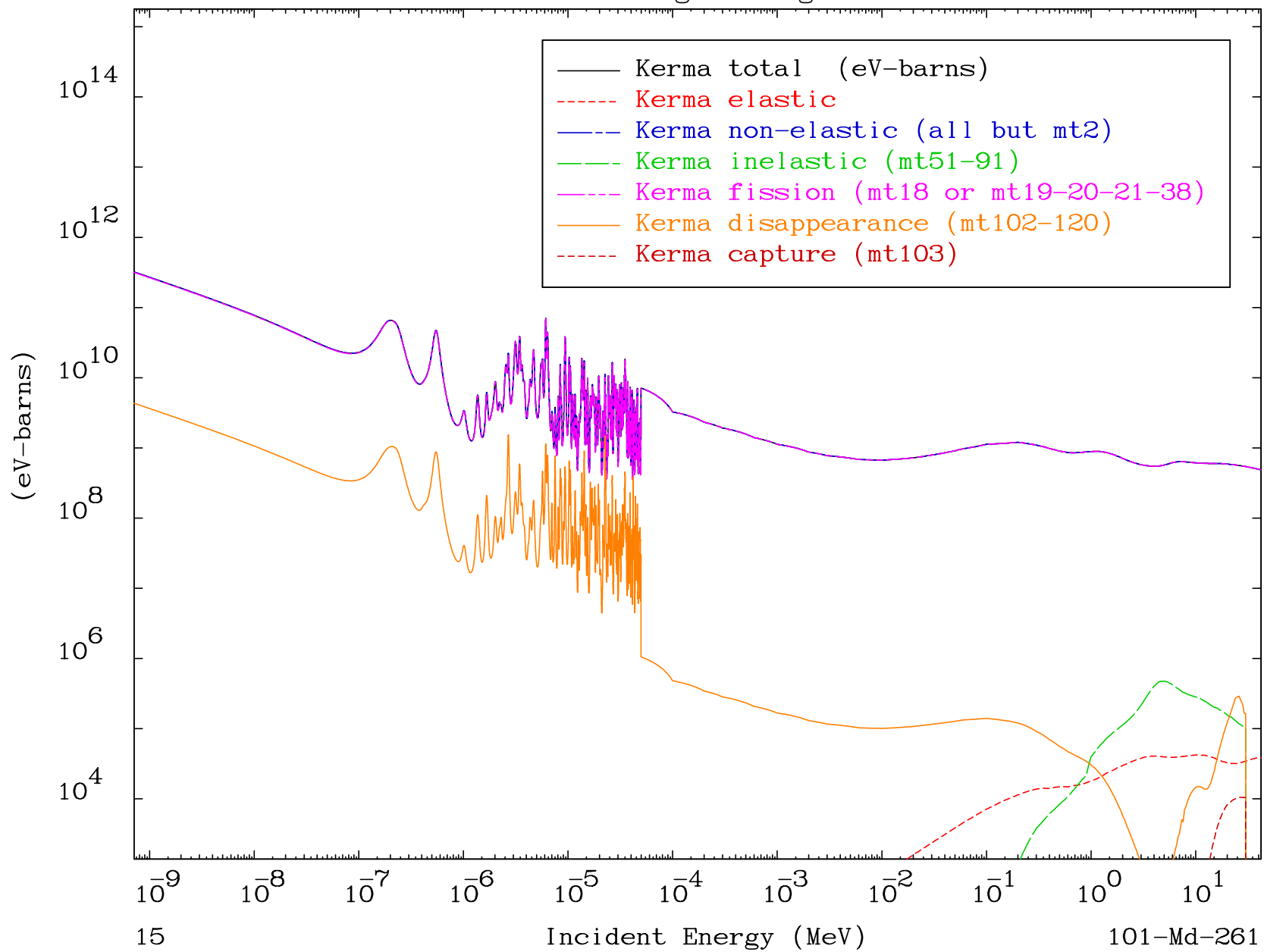


MAT 9962

Energy Release
Heating - Damage

101-Md-261

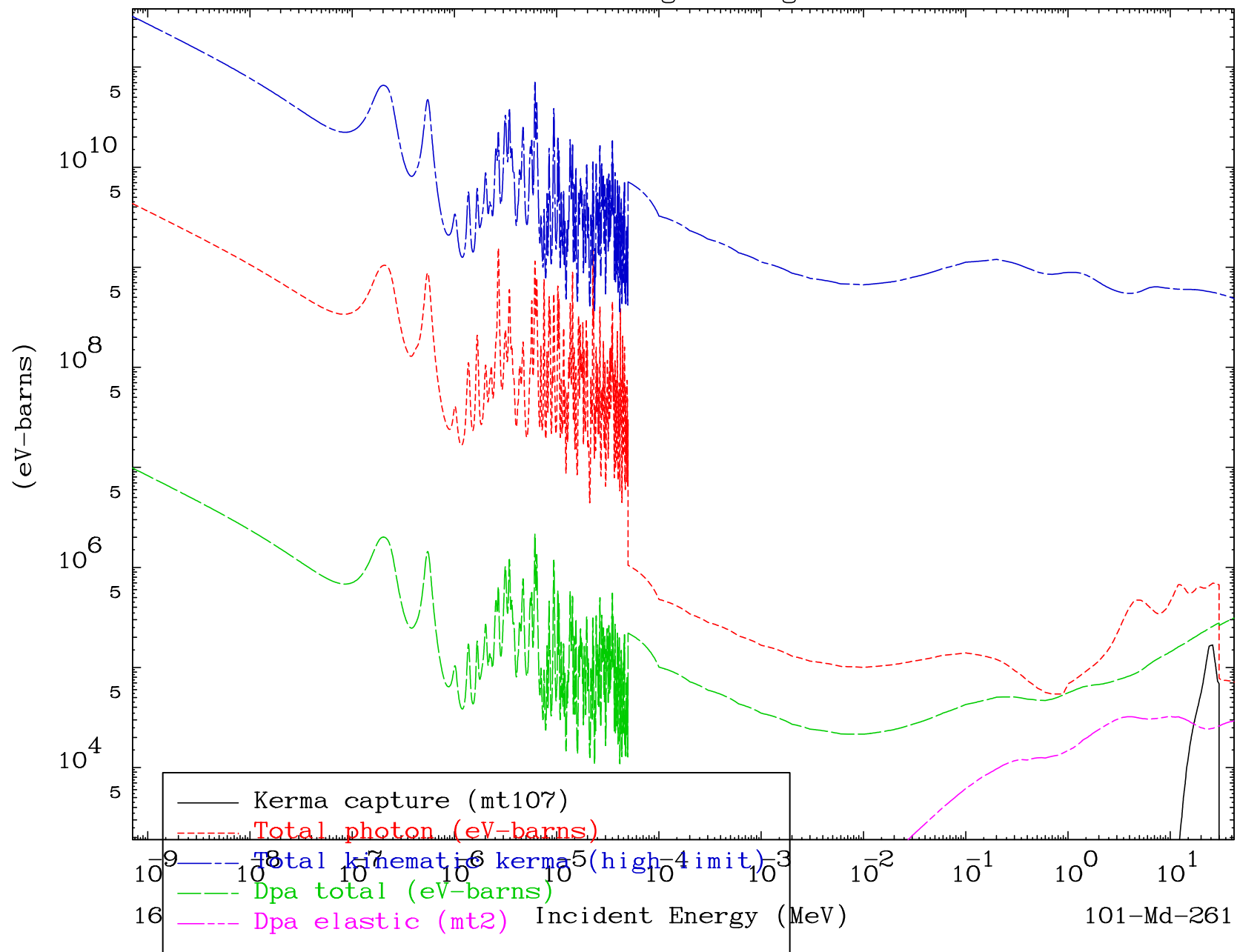




MAT 9962

Energy Release
Heating - Damage

101-Md-261

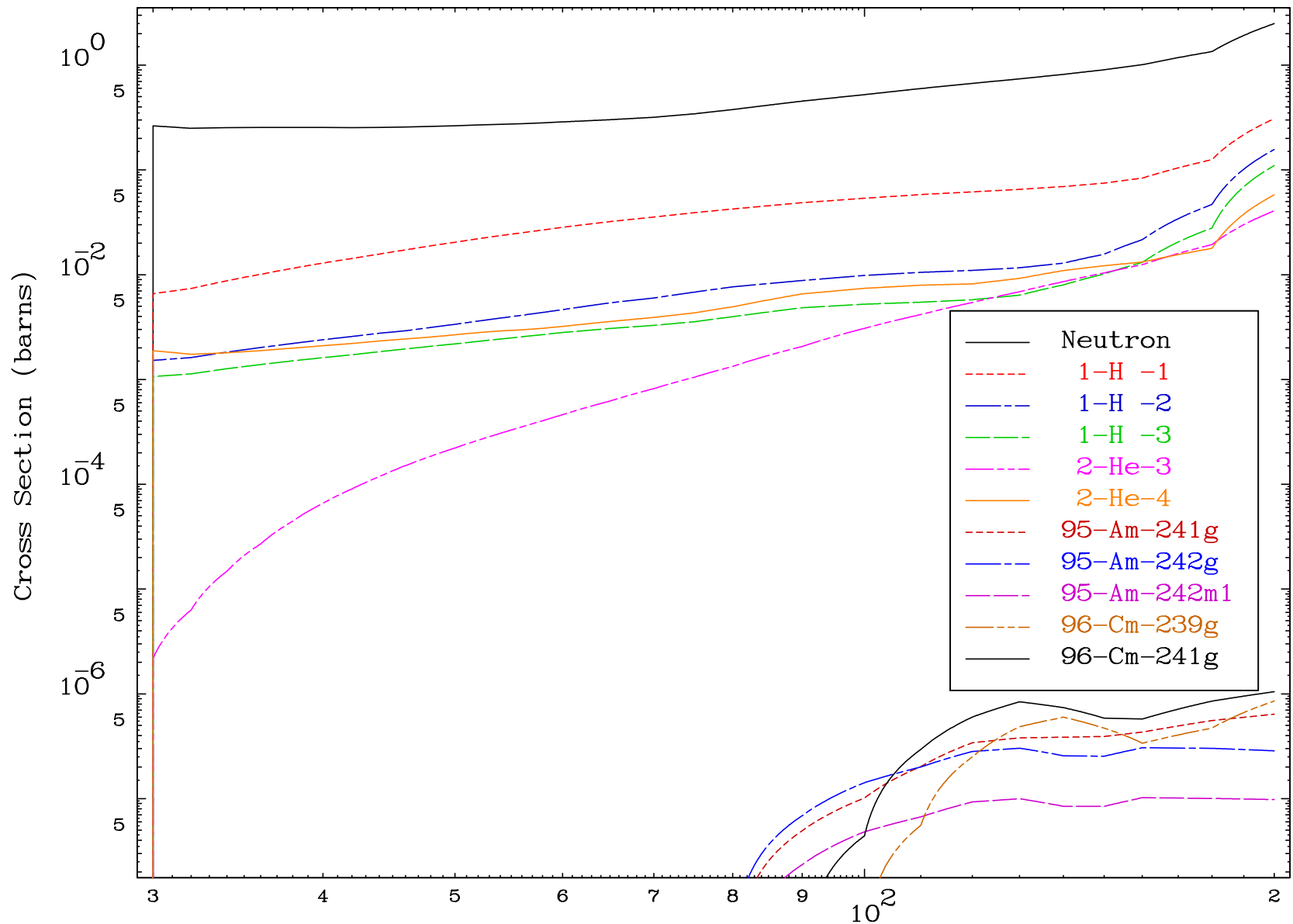


MAT 9962

(n,remainder)

101-Md-261

Radionuclide Production Cross Section



17

Incident Energy (MeV)

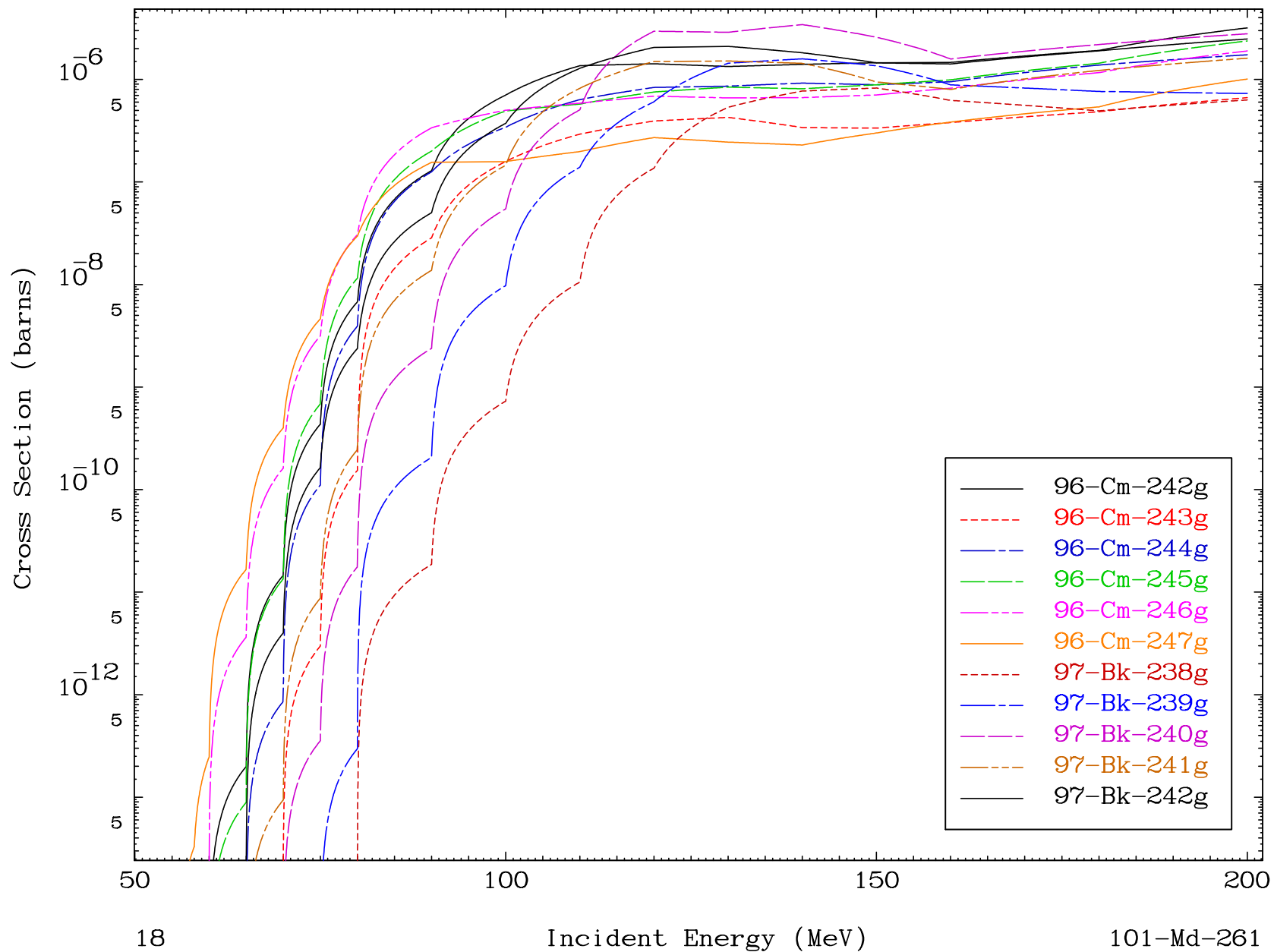
101-Md-261

MAT 9962

(n,remainder)

101-Md-261

Radionuclide Production Cross Section

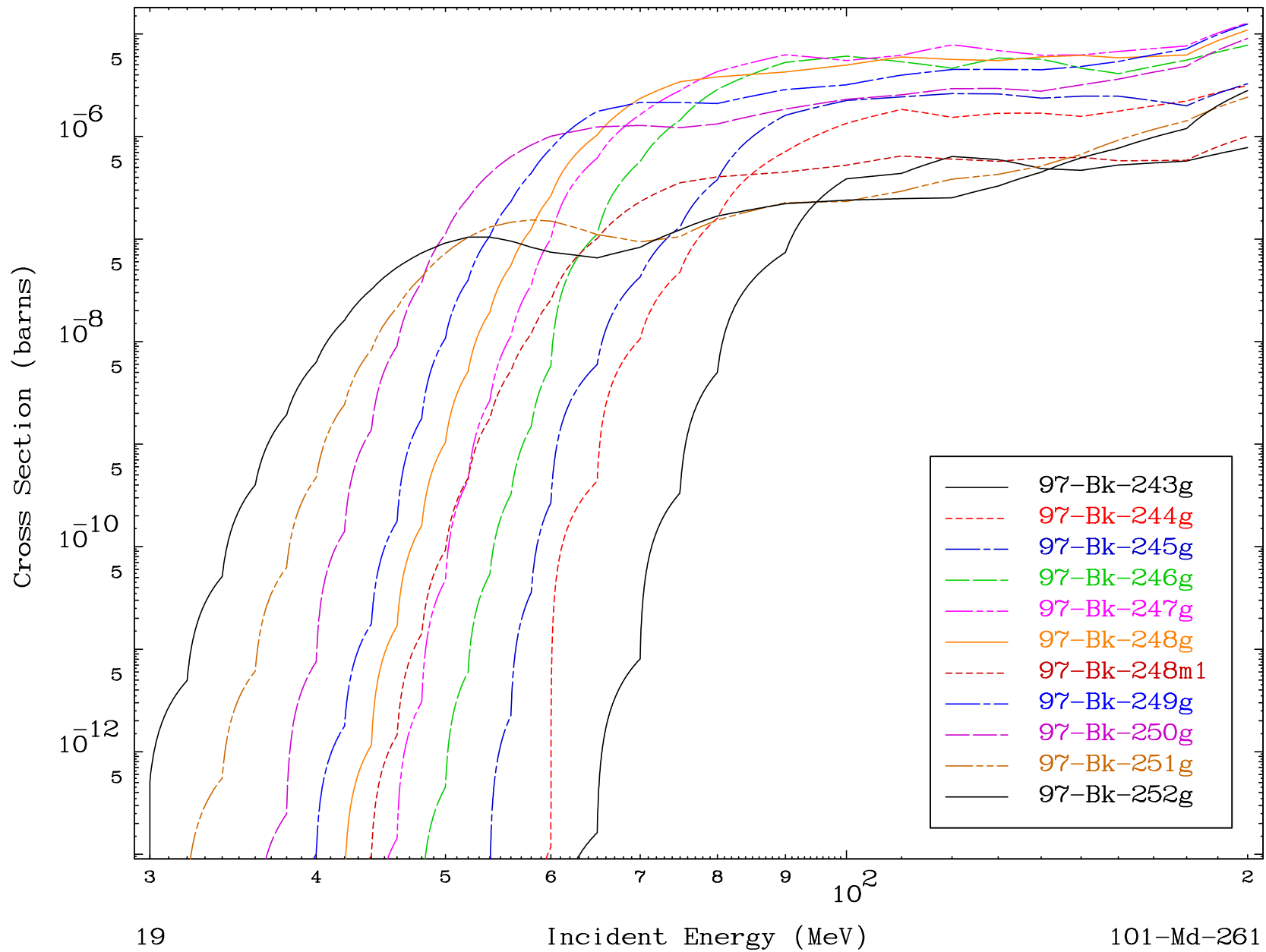


MAT 9962

(n,remainder)

101-Md-261

Radionuclide Production Cross Section

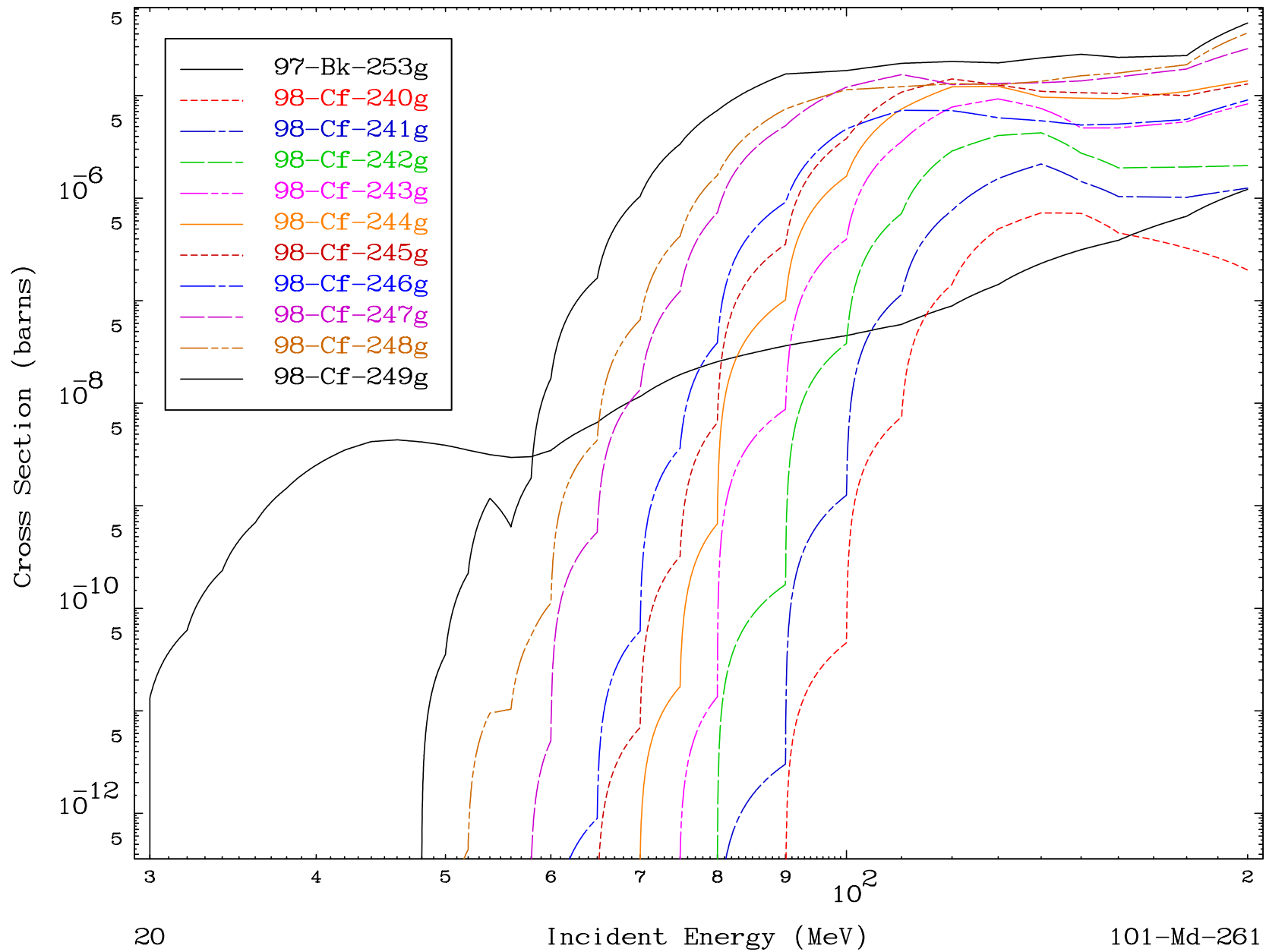


MAT 9962

(n,remainder)

101-Md-261

Radionuclide Production Cross Section

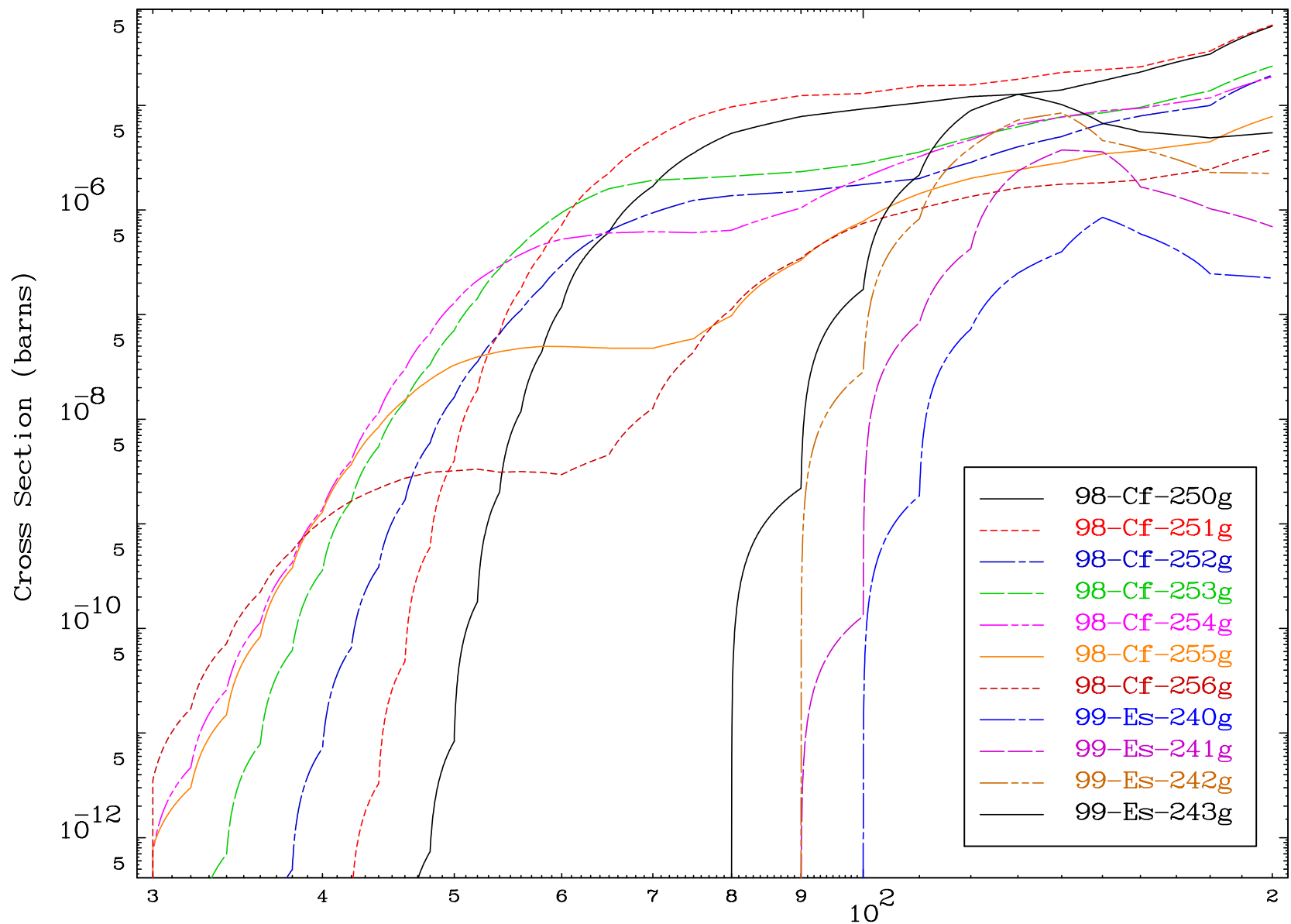


MAT 9962

(n,remainder)

101-Md-261

Radionuclide Production Cross Section



21

Incident Energy (MeV)

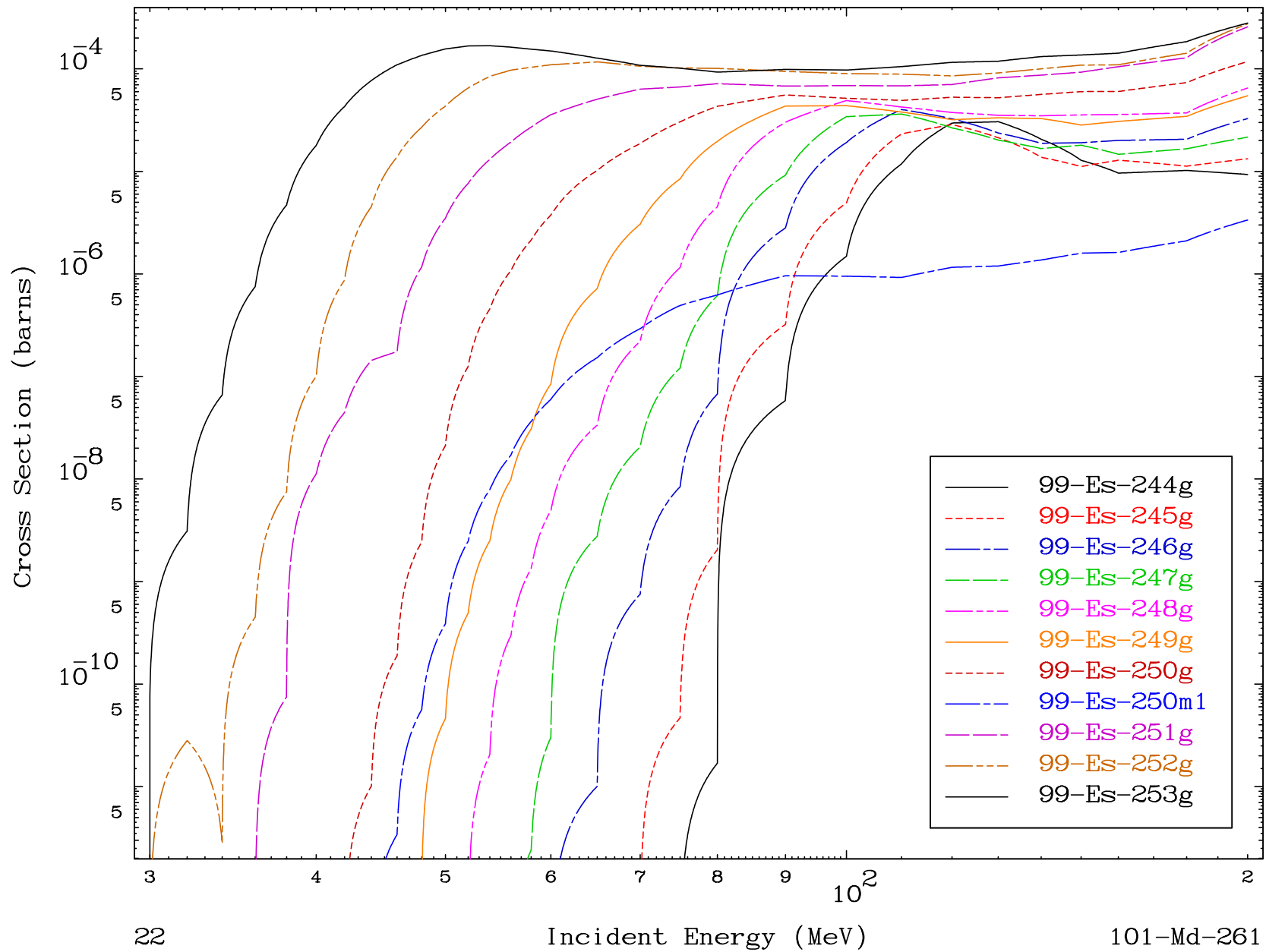
101-Md-261

MAT 9962

(n,remainder)

101-Md-261

Radionuclide Production Cross Section

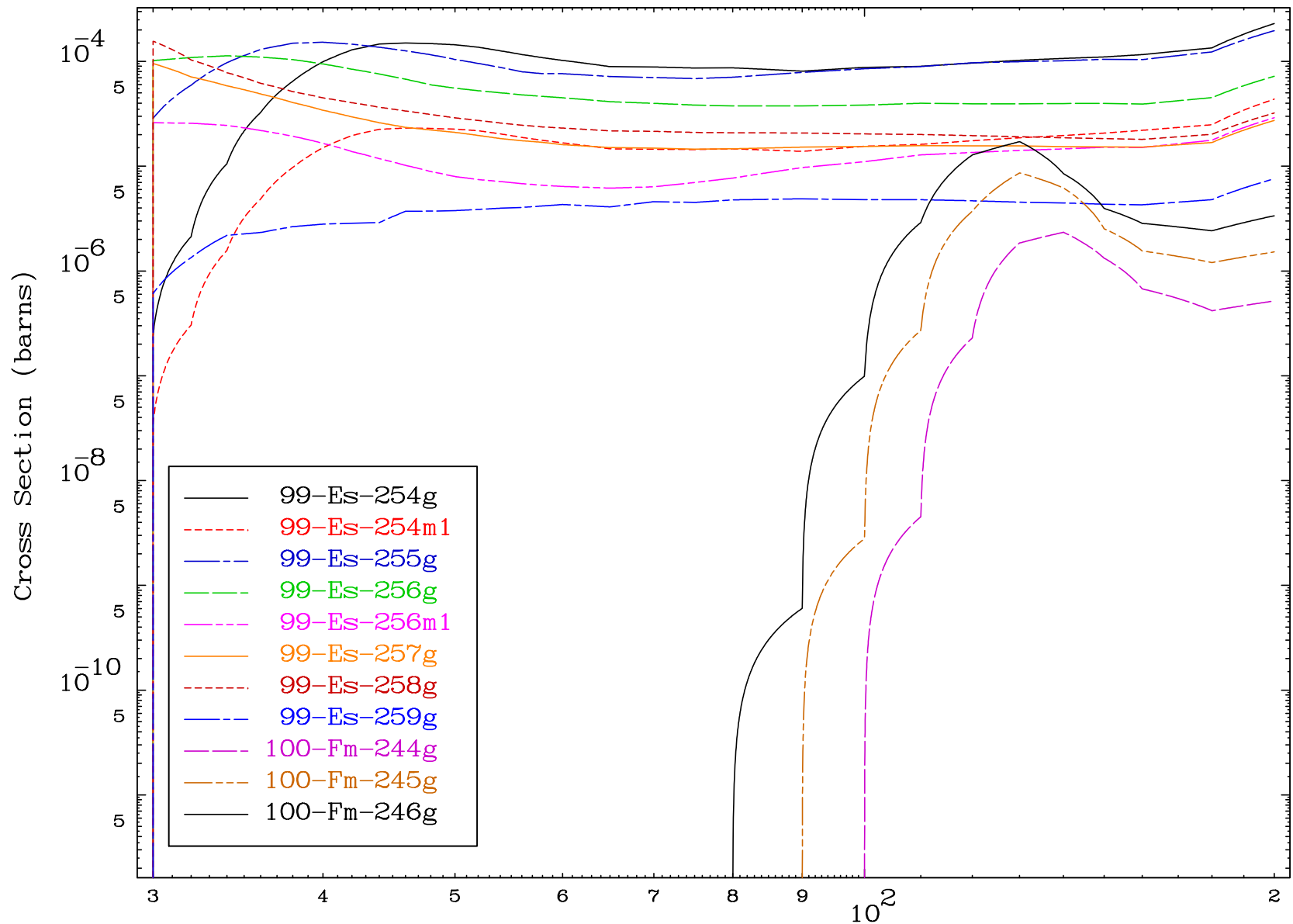


MAT 9962

(n,remainder)

101-Md-261

Radionuclide Production Cross Section



23

Incident Energy (MeV)

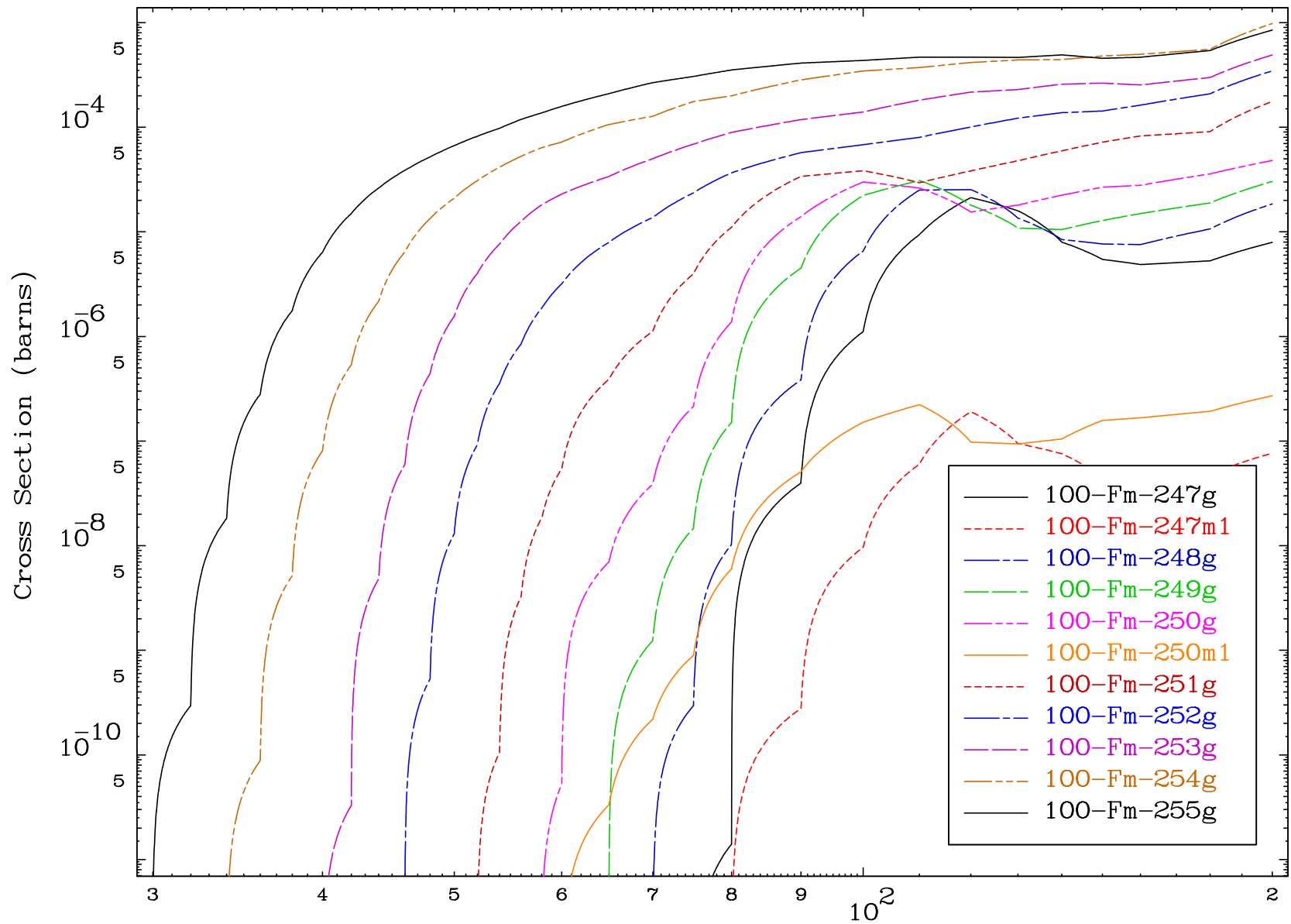
101-Md-261

MAT 9962

(n,remainder)

101-Md-261

Radionuclide Production Cross Section



24

Incident Energy (MeV)

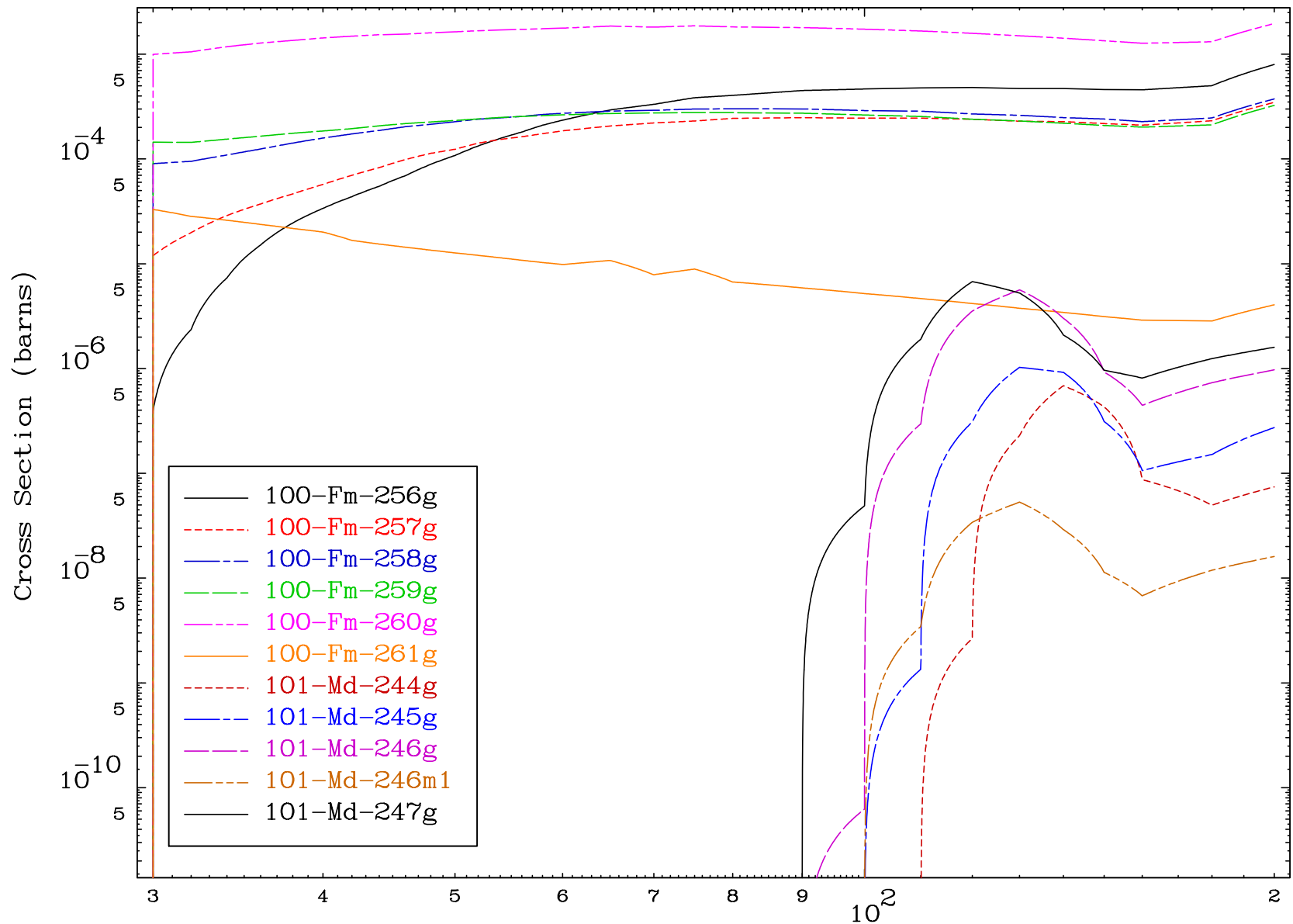
101-Md-261

MAT 9962

(n,remainder)

101-Md-261

Radionuclide Production Cross Section

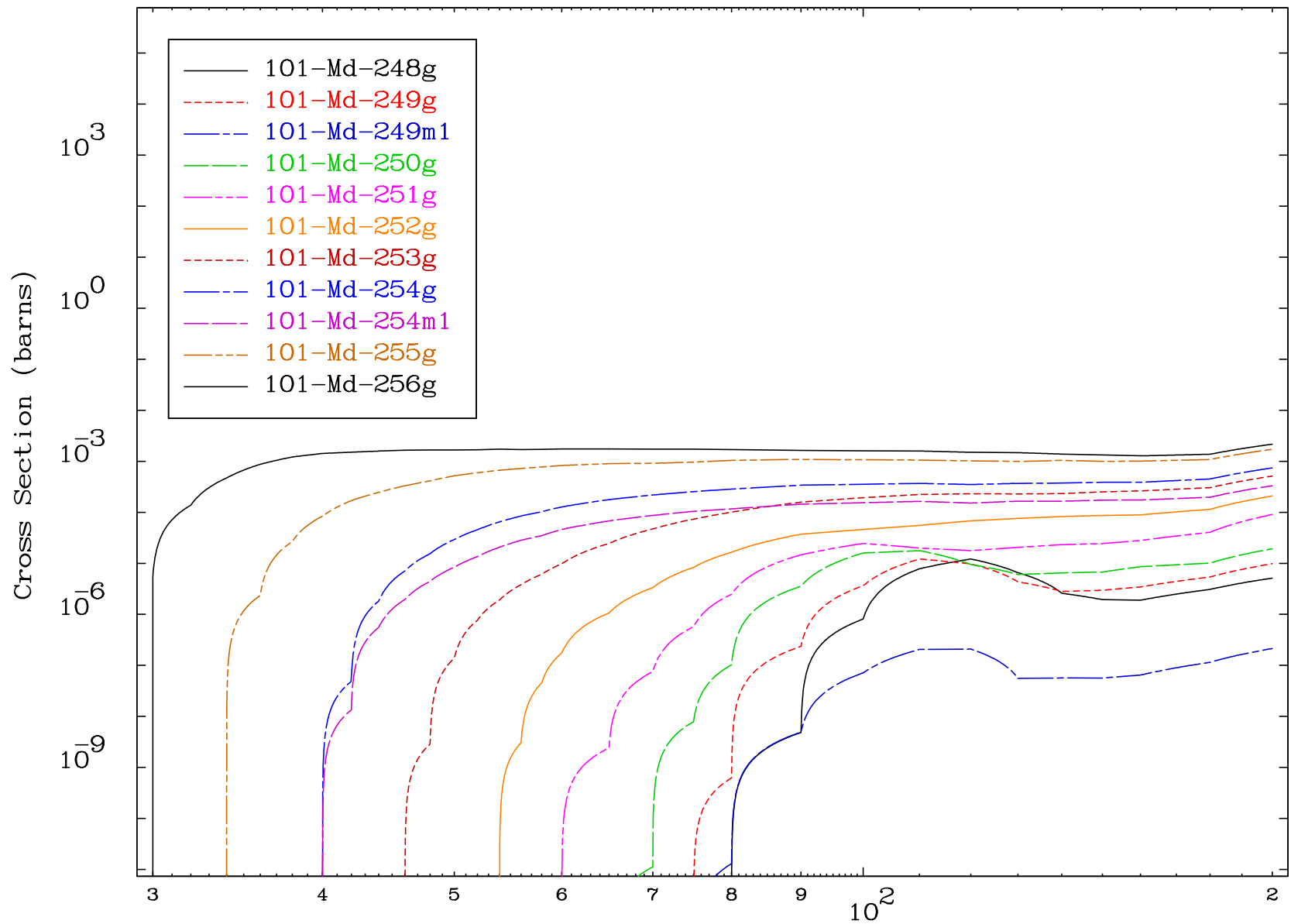


25

Incident Energy (MeV)

101-Md-261

Radionuclide Production Cross Section

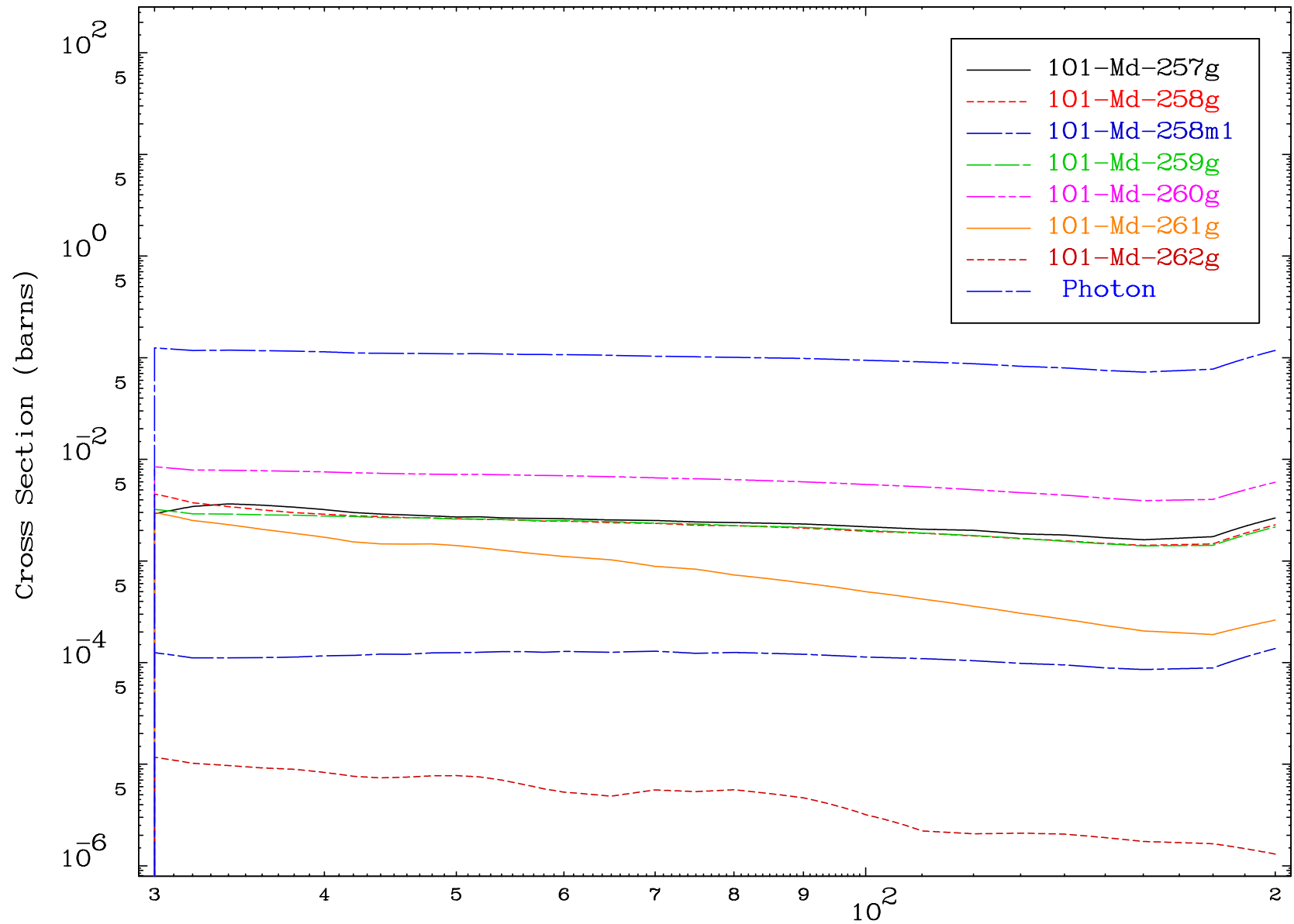


MAT 9962

(n,remainder)

101-Md-261

Radionuclide Production Cross Section



27

Incident Energy (MeV)

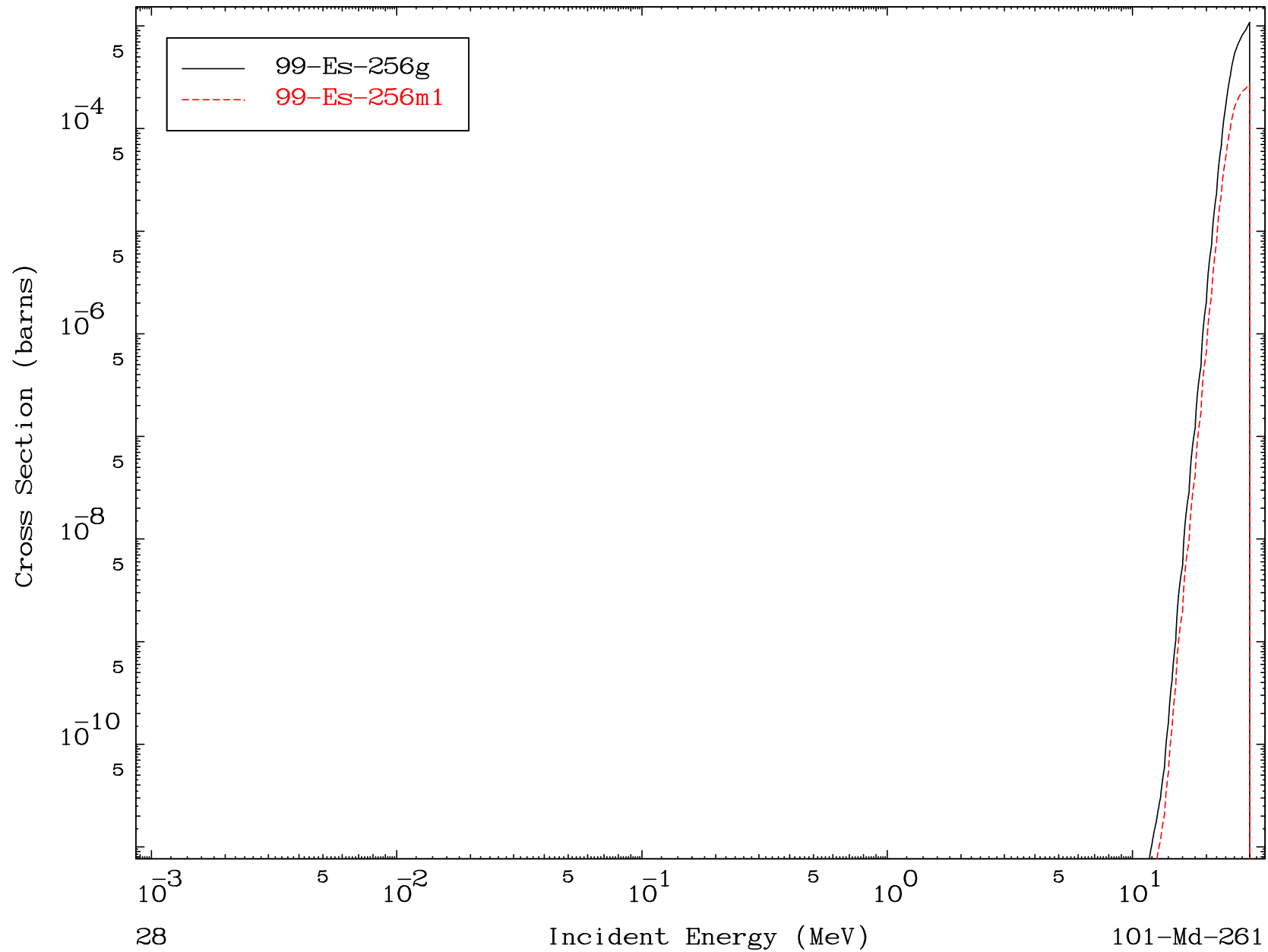
101-Md-261

MAT 9962

 $(n,2n) \alpha$

101-Md-261

Radionuclide Production Cross Section



MAT 9962

(n,4n)

101-Md-261

Radionuclide Production Cross Section

