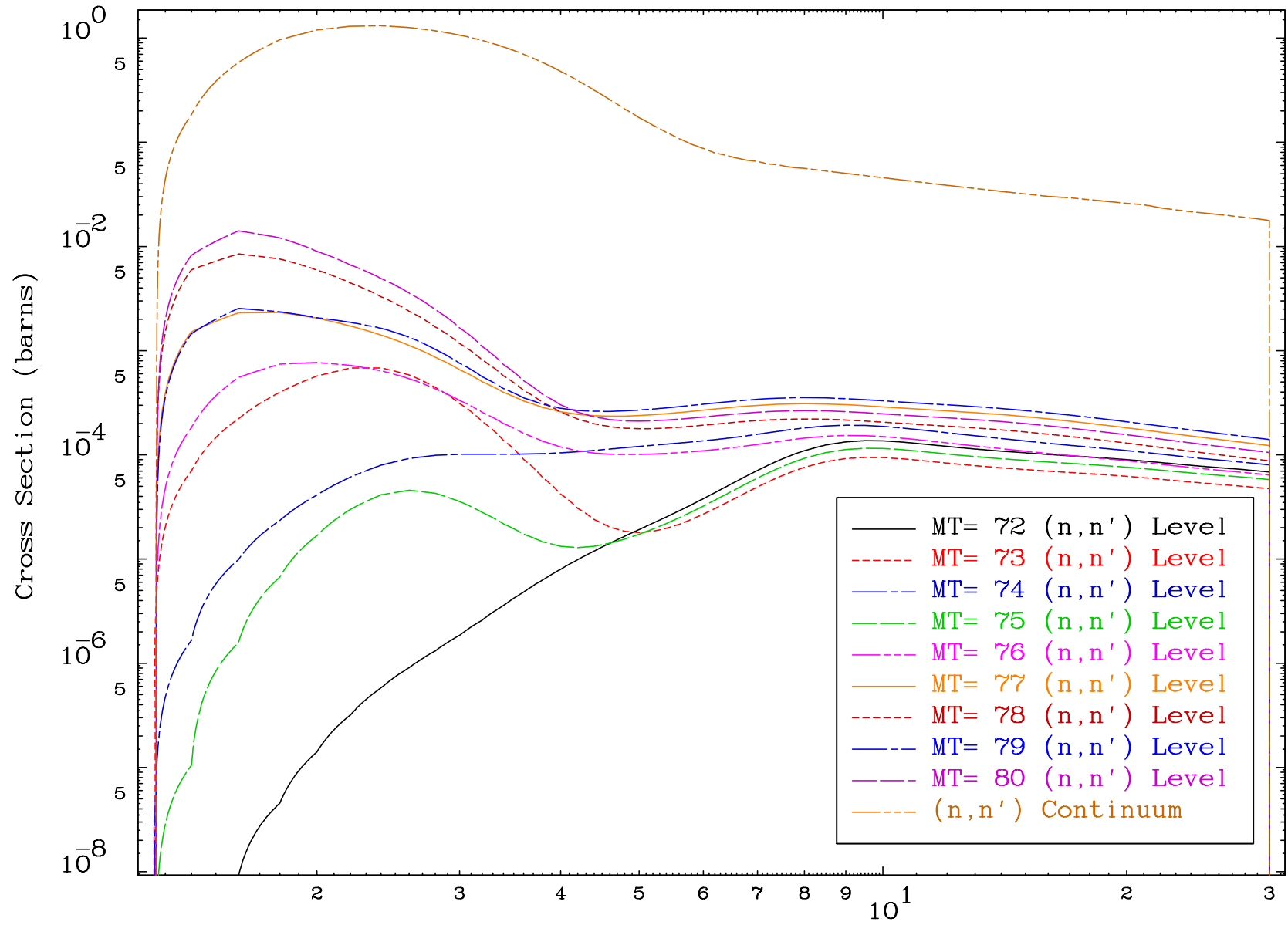


MAT 9334

(n,n') Level  
294 Kelvin Cross Sections

93-Np-233



10

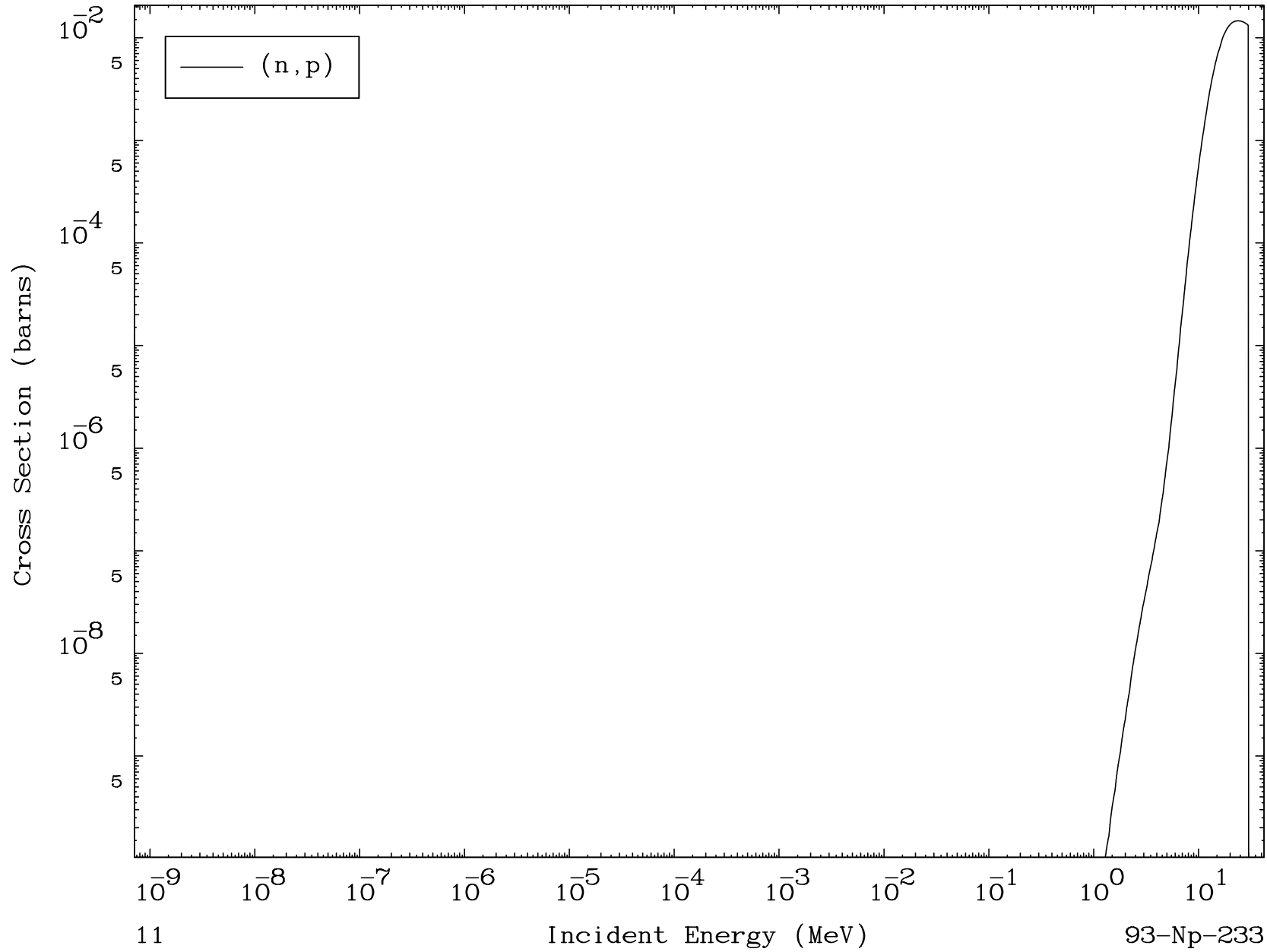
Incident Energy (MeV)

93-Np-233

MAT 9334

(n,p) Levels  
294 Kelvin Cross Sections

93-Np-233



11

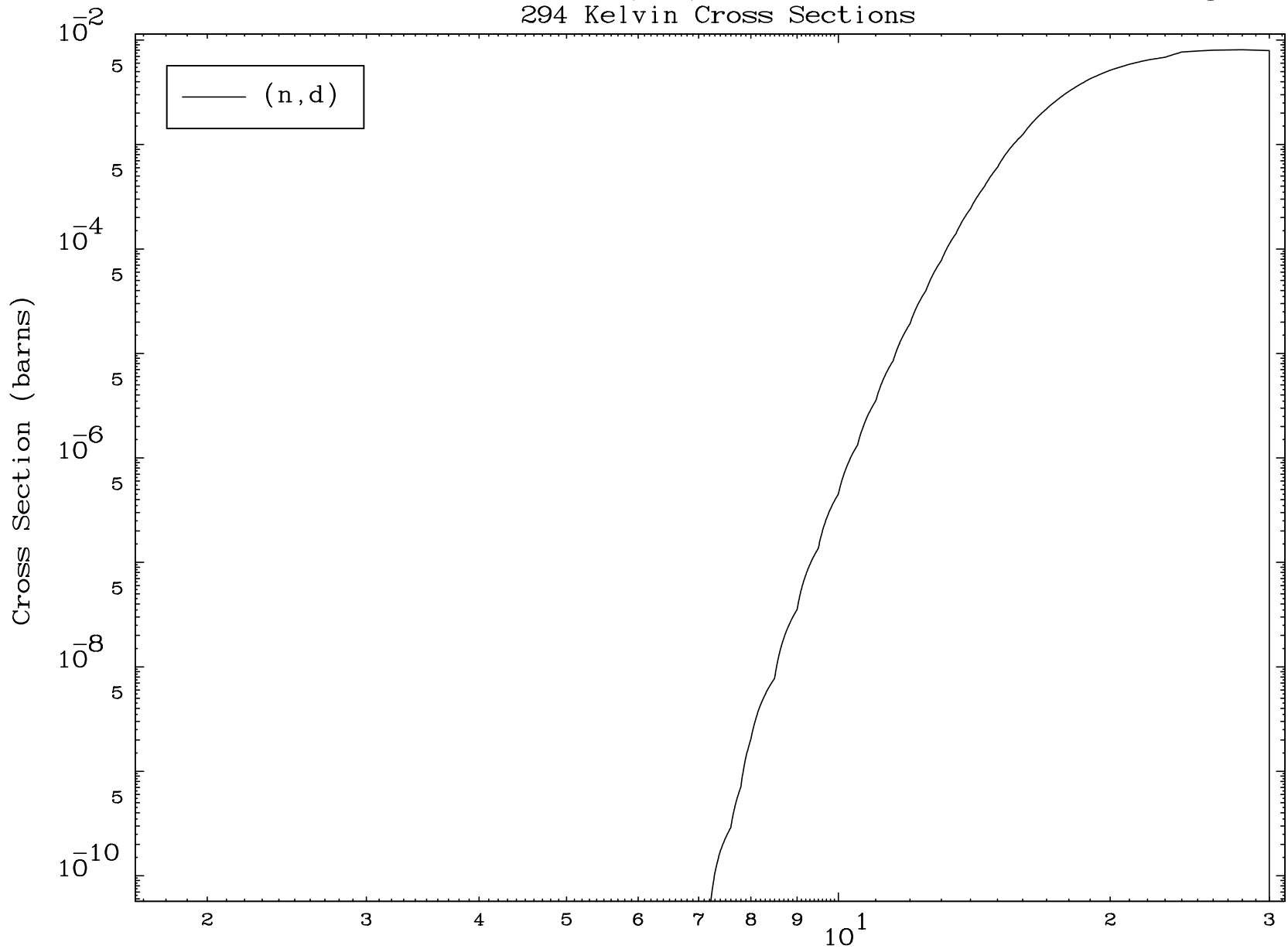
Incident Energy (MeV)

93-Np-233

MAT 9334

(n,d) Levels  
294 Kelvin Cross Sections

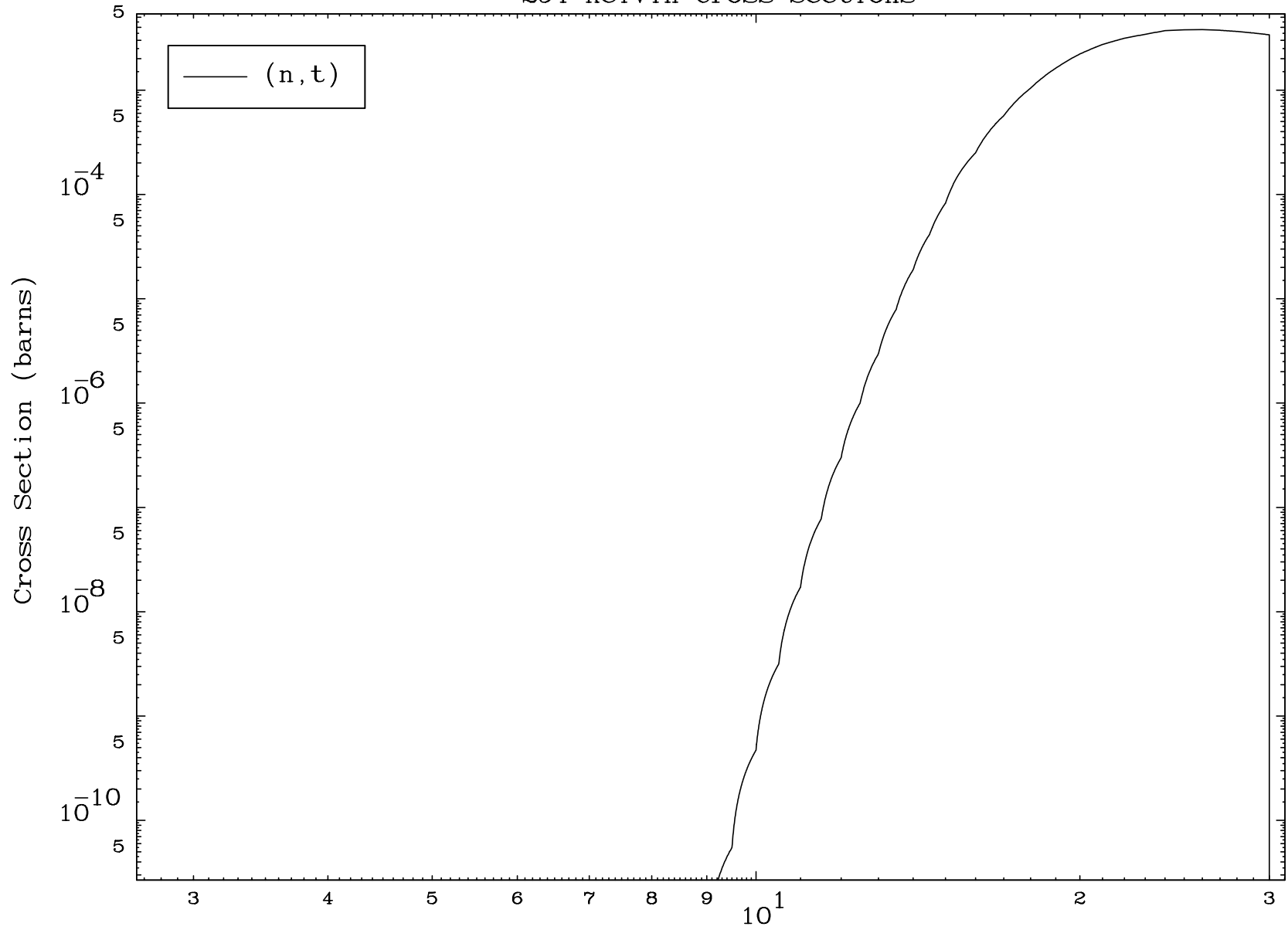
93-Np-233

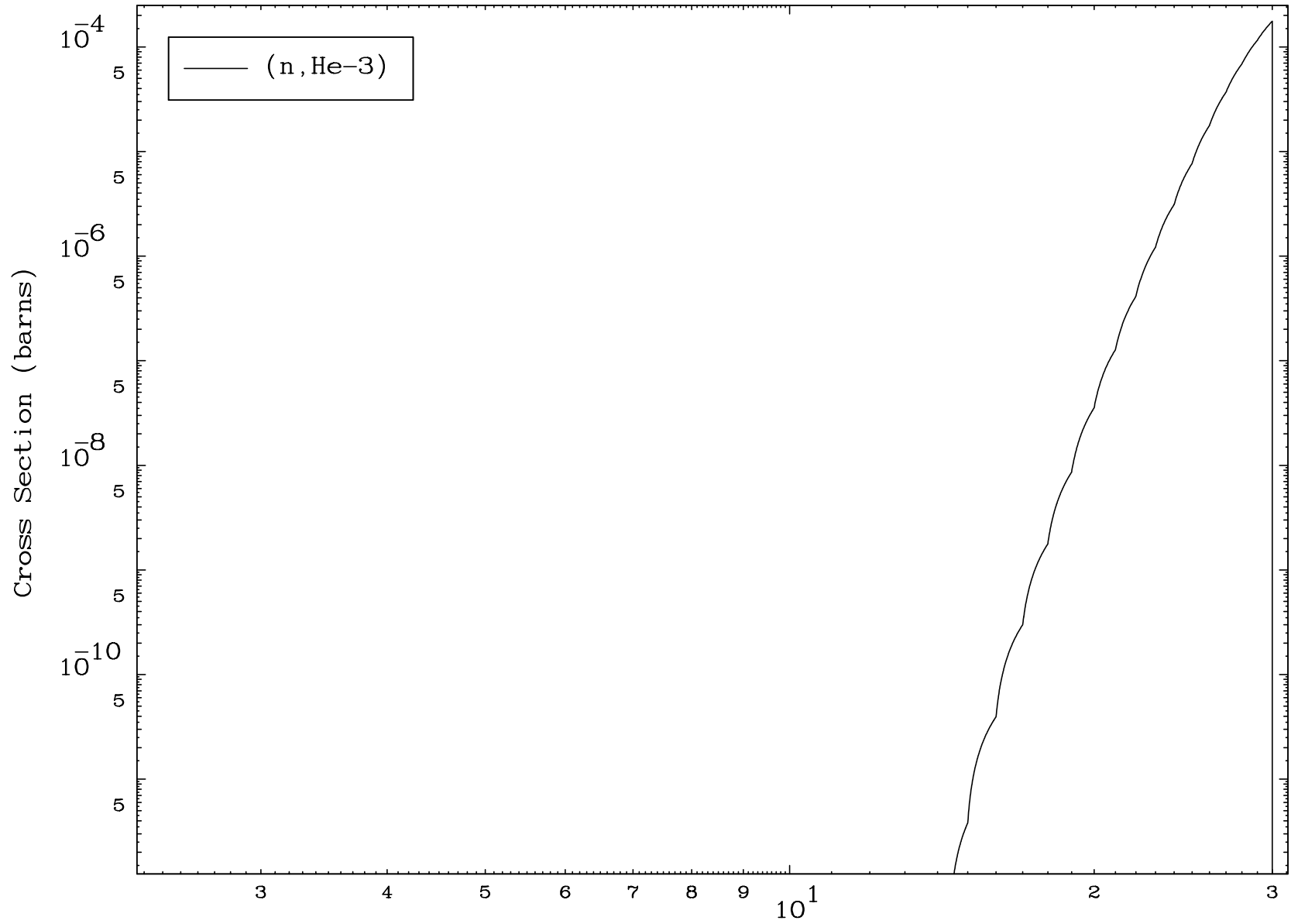


12

Incident Energy (MeV)

93-Np-233

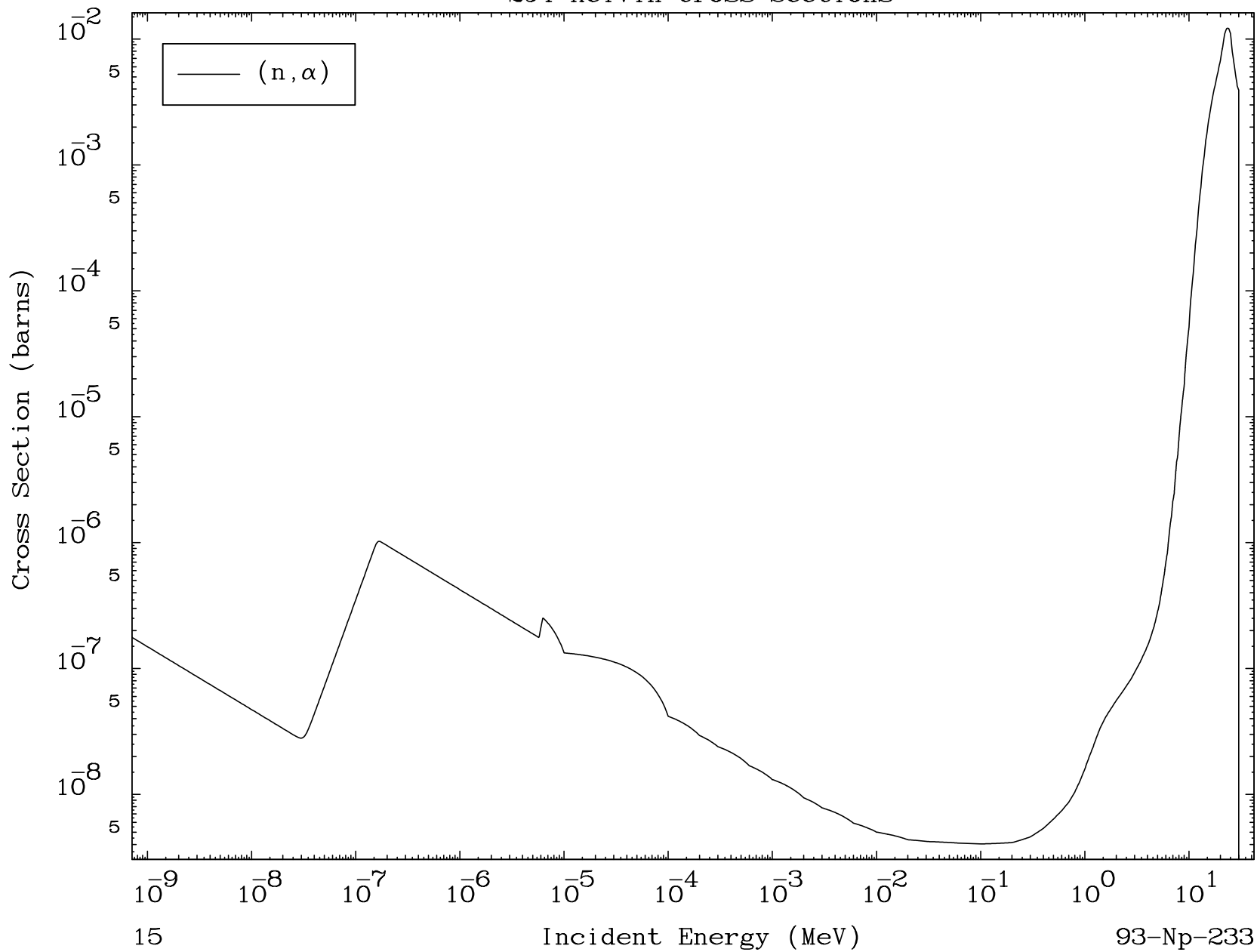




MAT 9334

(n,  $\alpha$ ) Levels  
294 Kelvin Cross Sections

93-Np-233



15

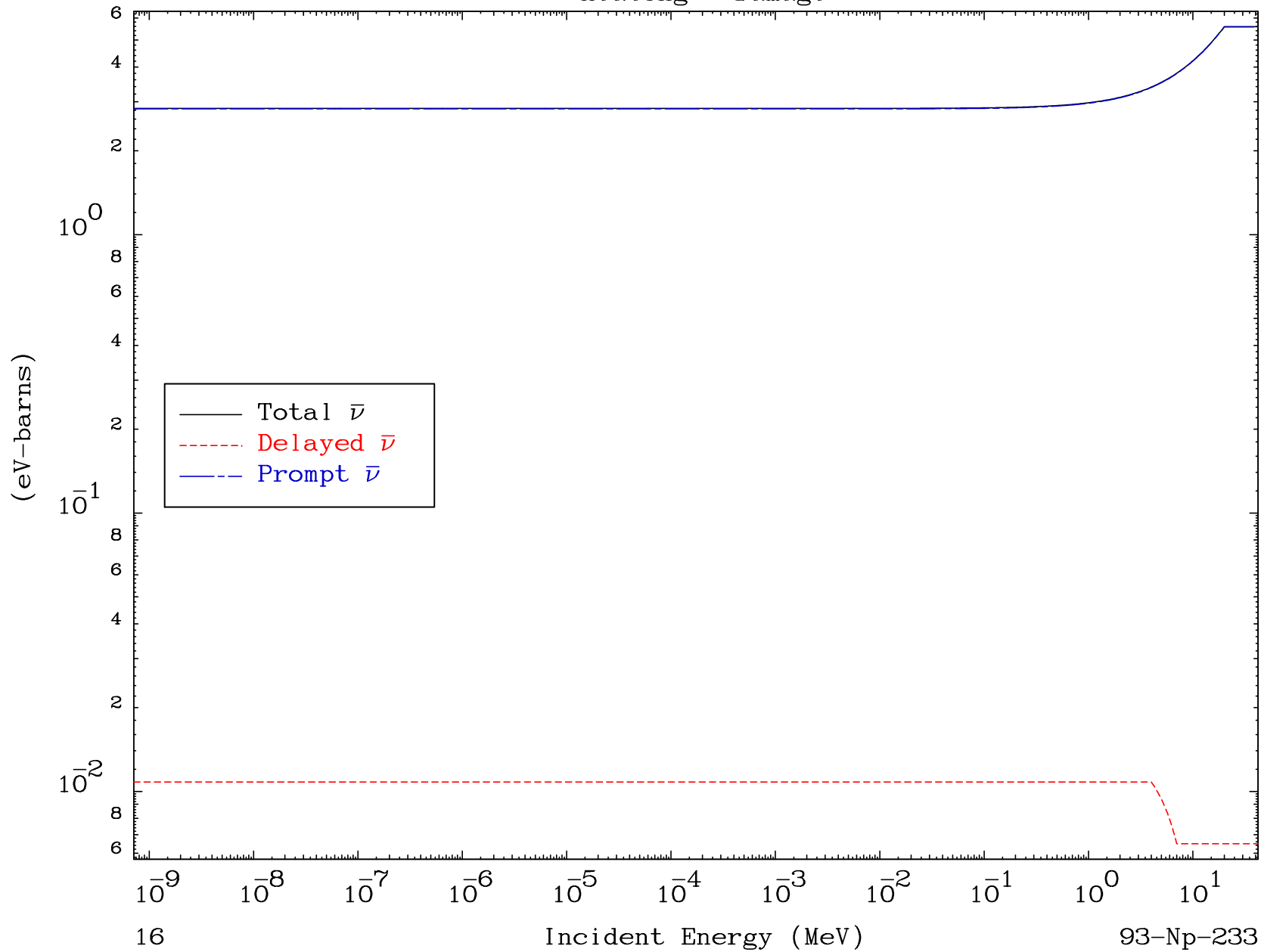
Incident Energy (MeV)

93-Np-233

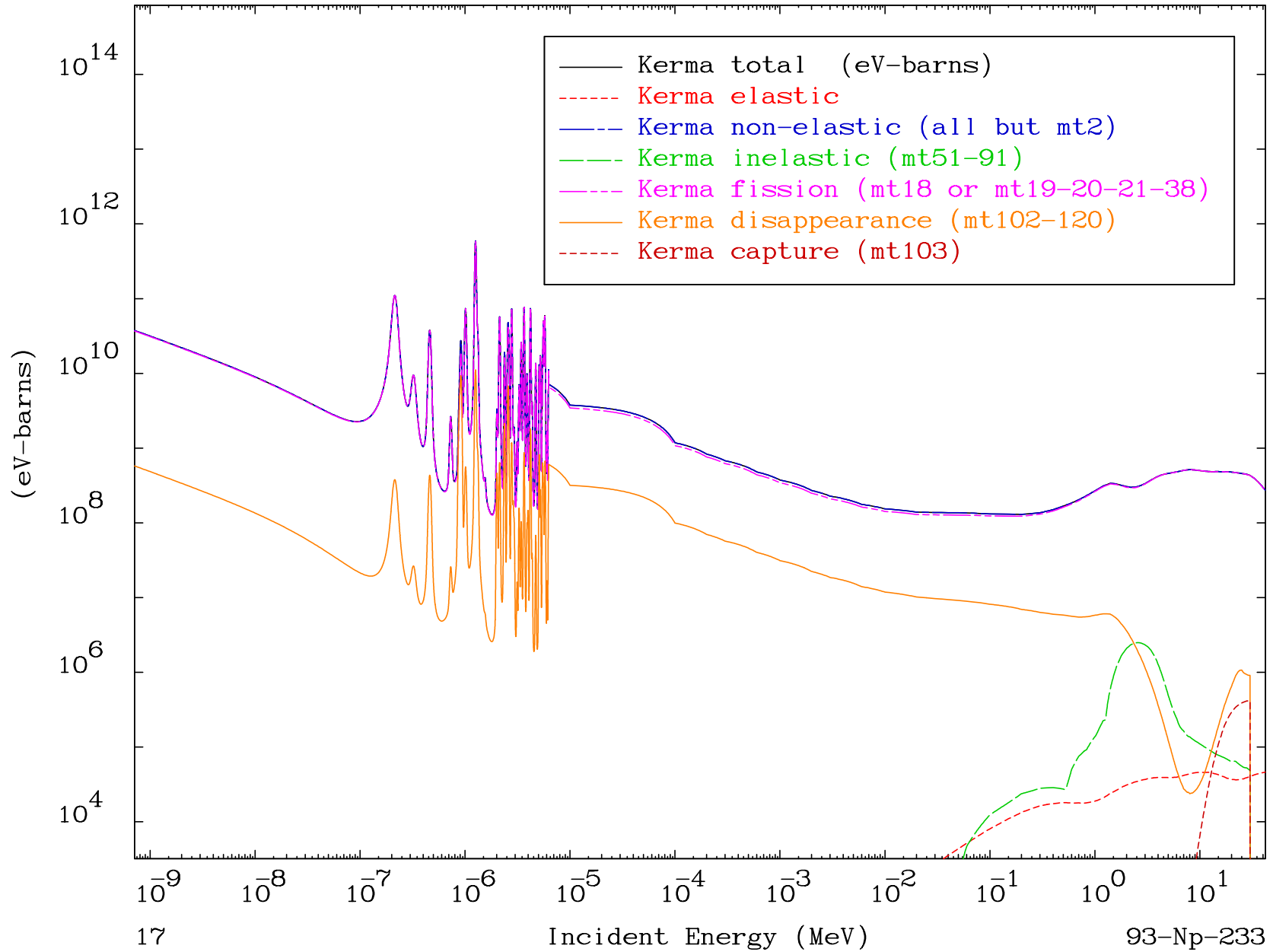
MAT 9334

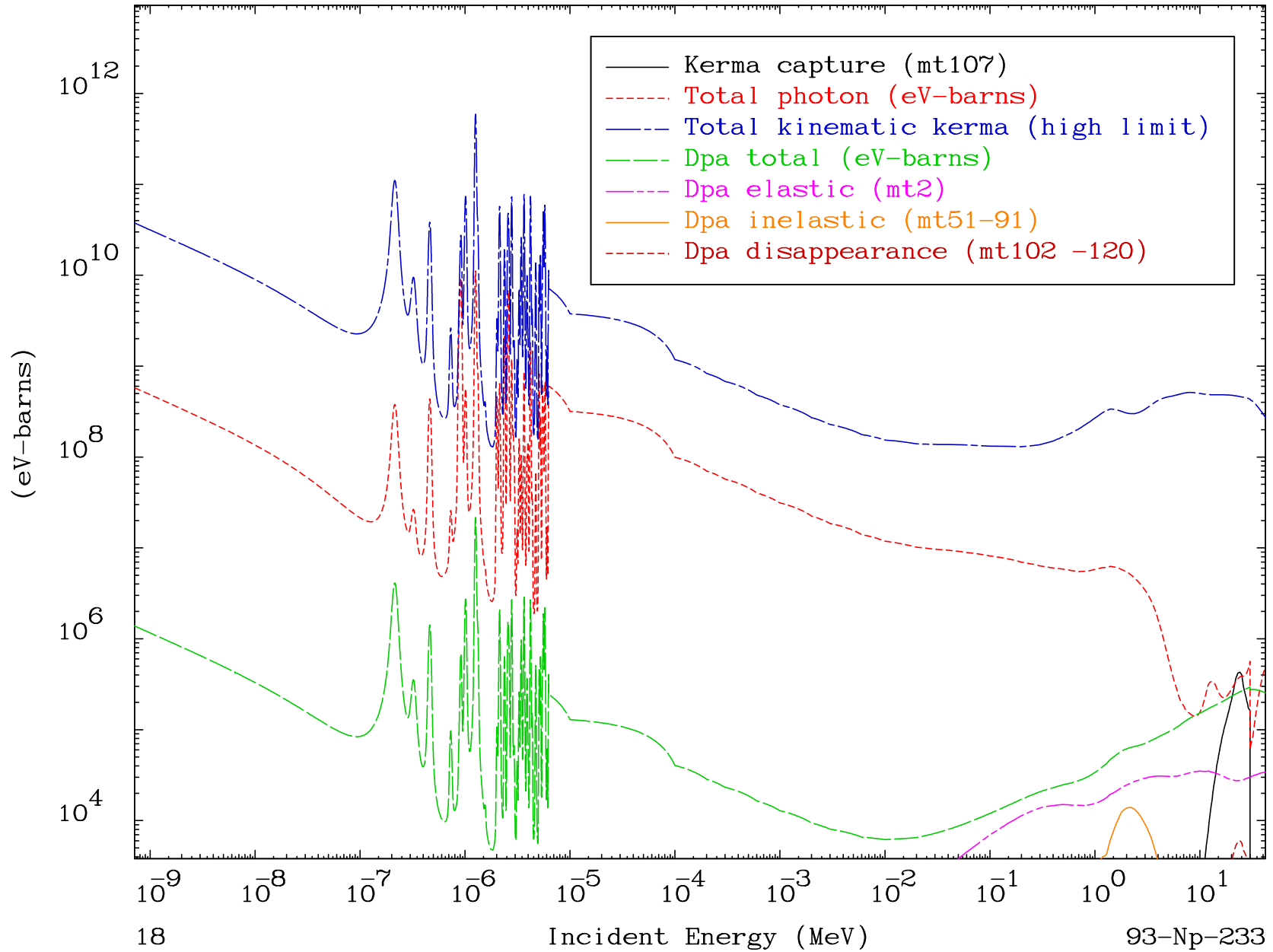
Energy Release  
Heating - Damage

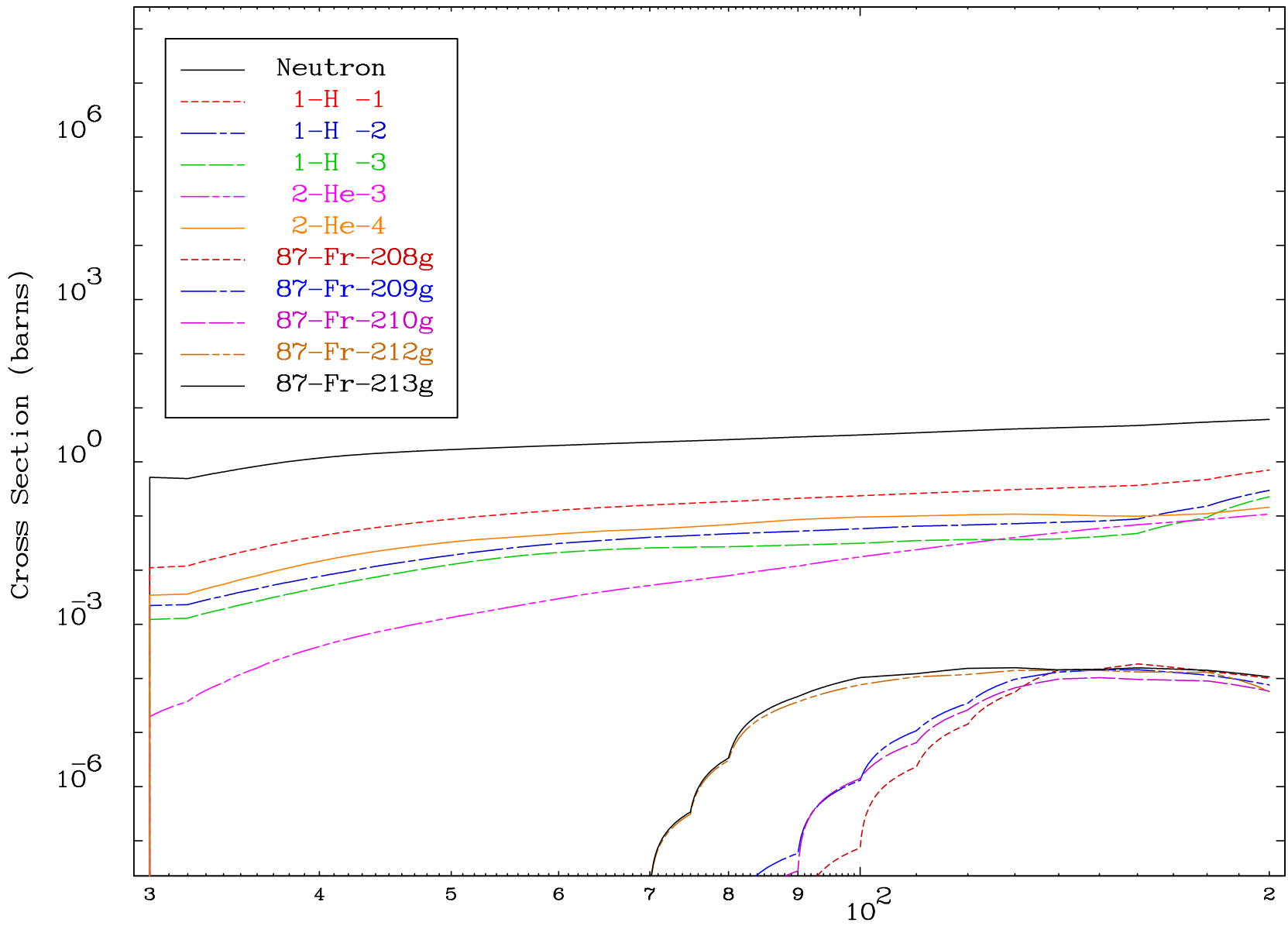
93-Np-233









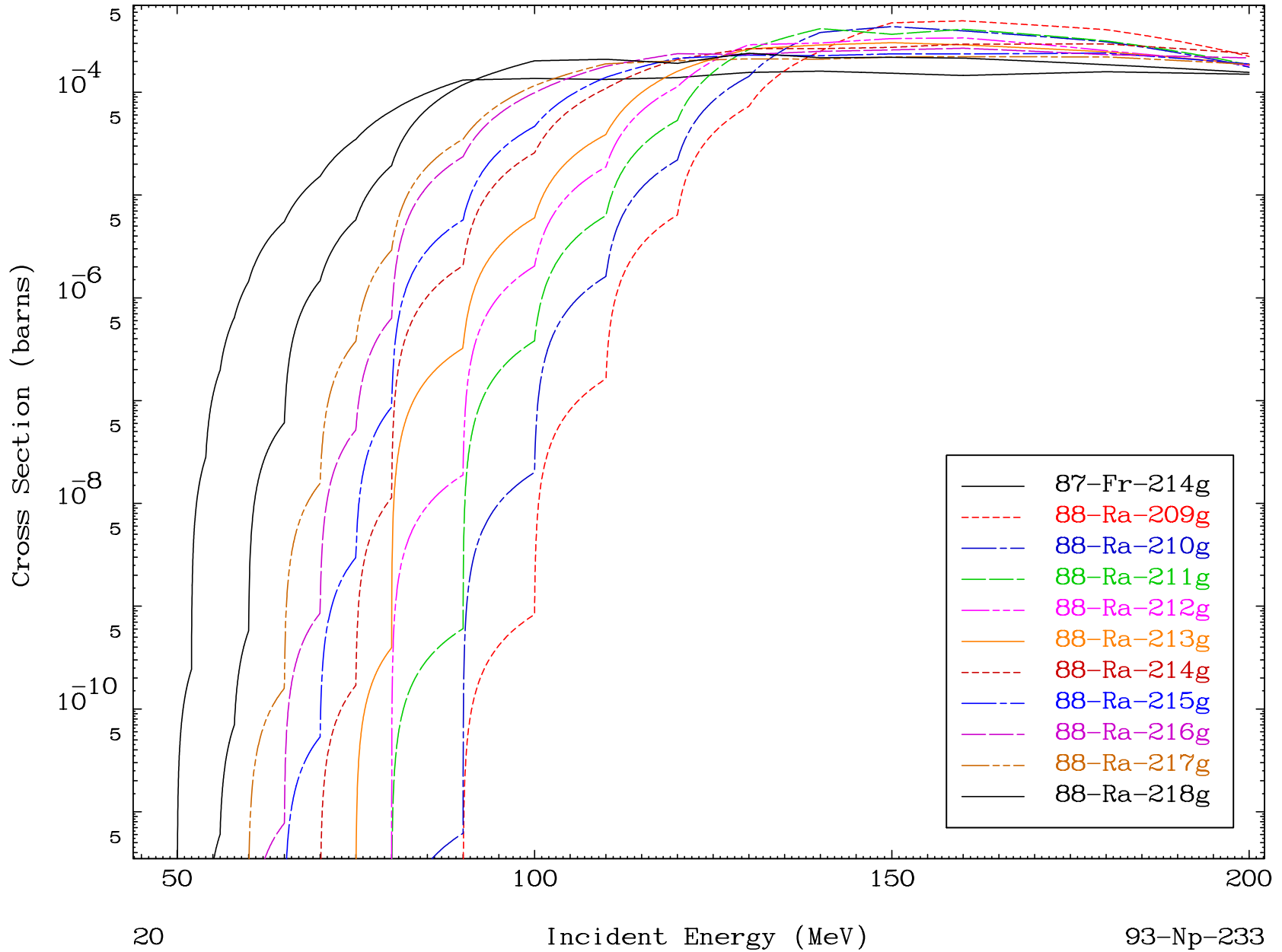


MAT 9334

(n,remainder)

93-Np-233

### Radionuclide Production Cross Section

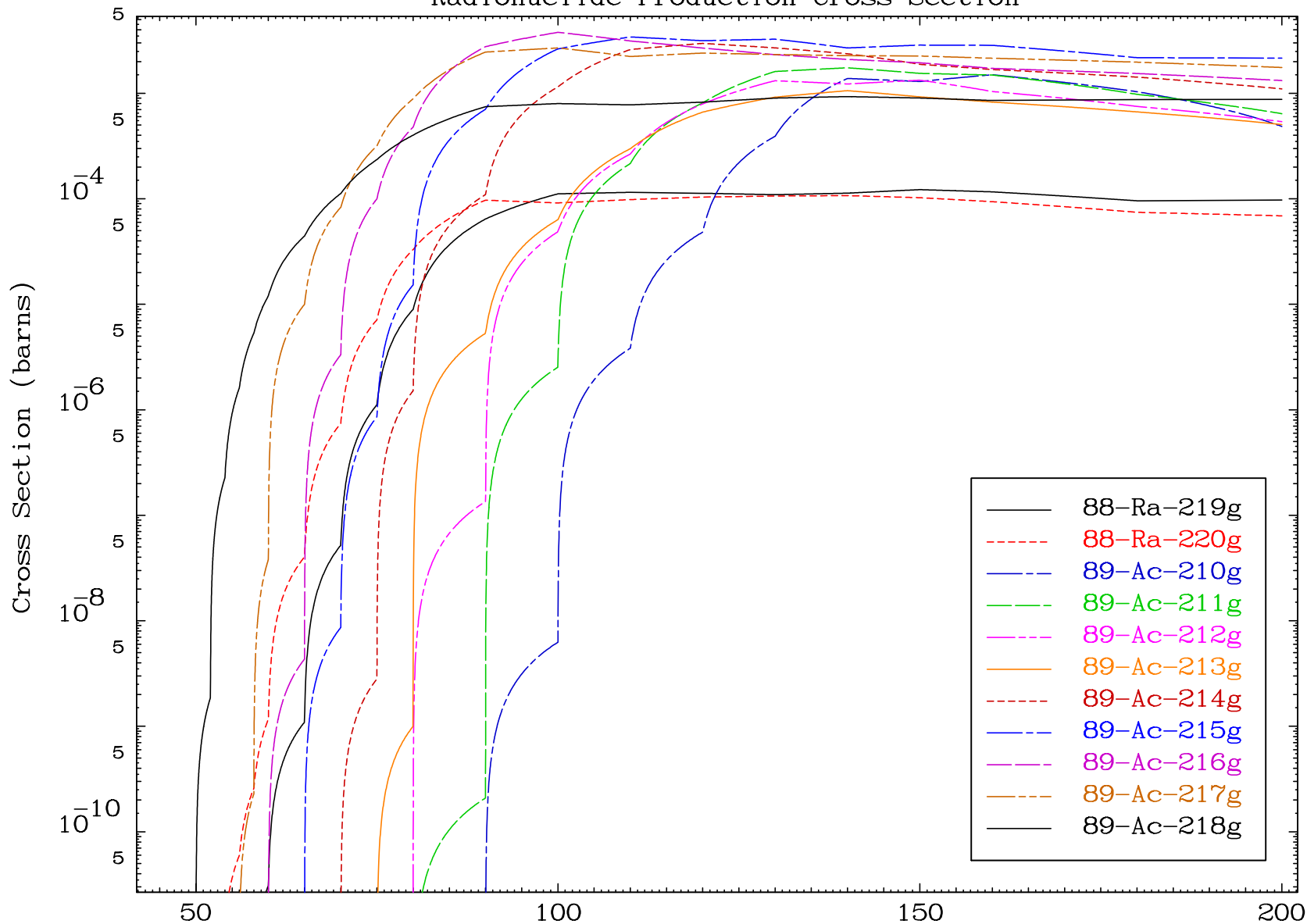


MAT 9334

(n,remainder)

93-Np-233

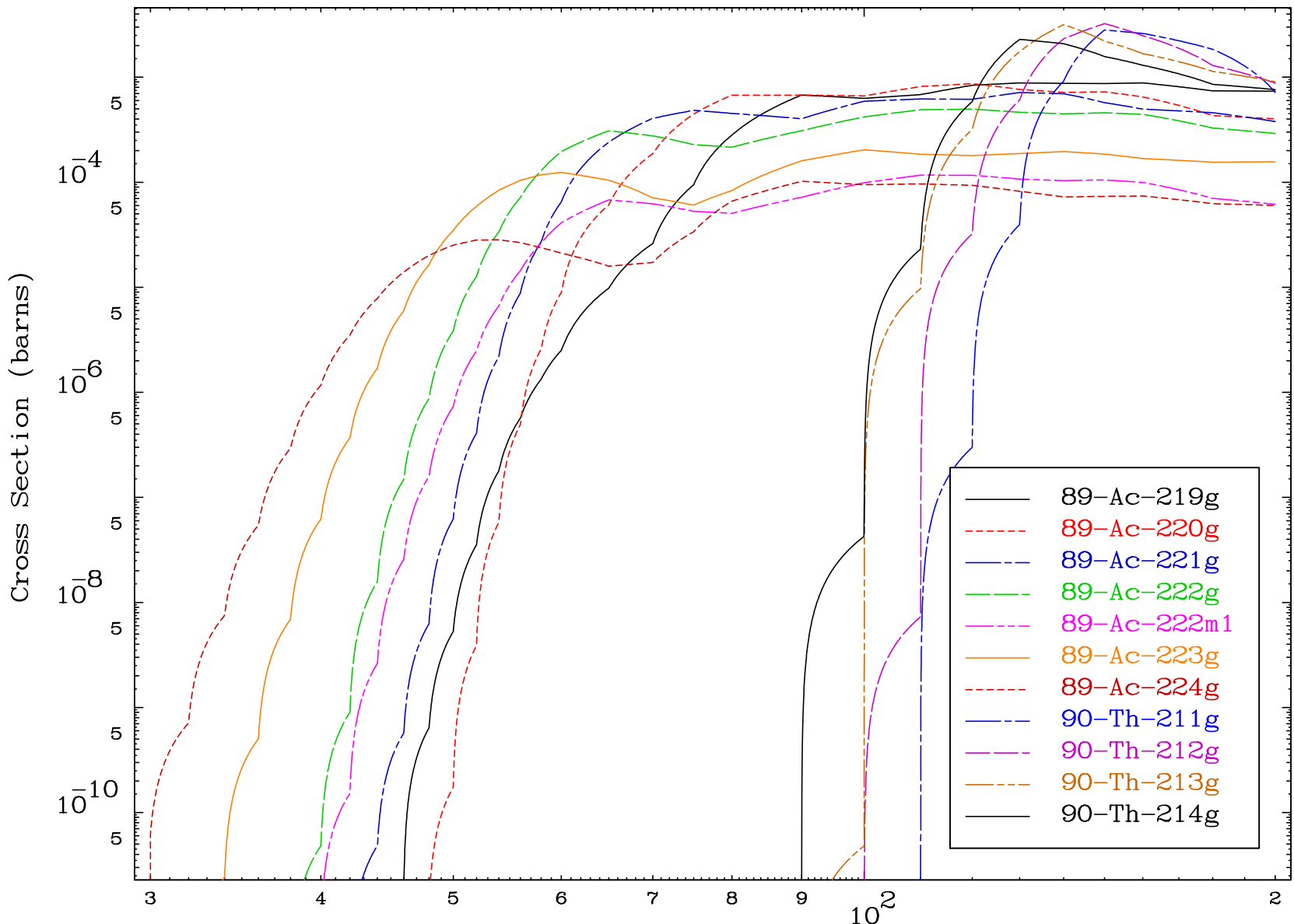
### Radionuclide Production Cross Section



21

Incident Energy (MeV)

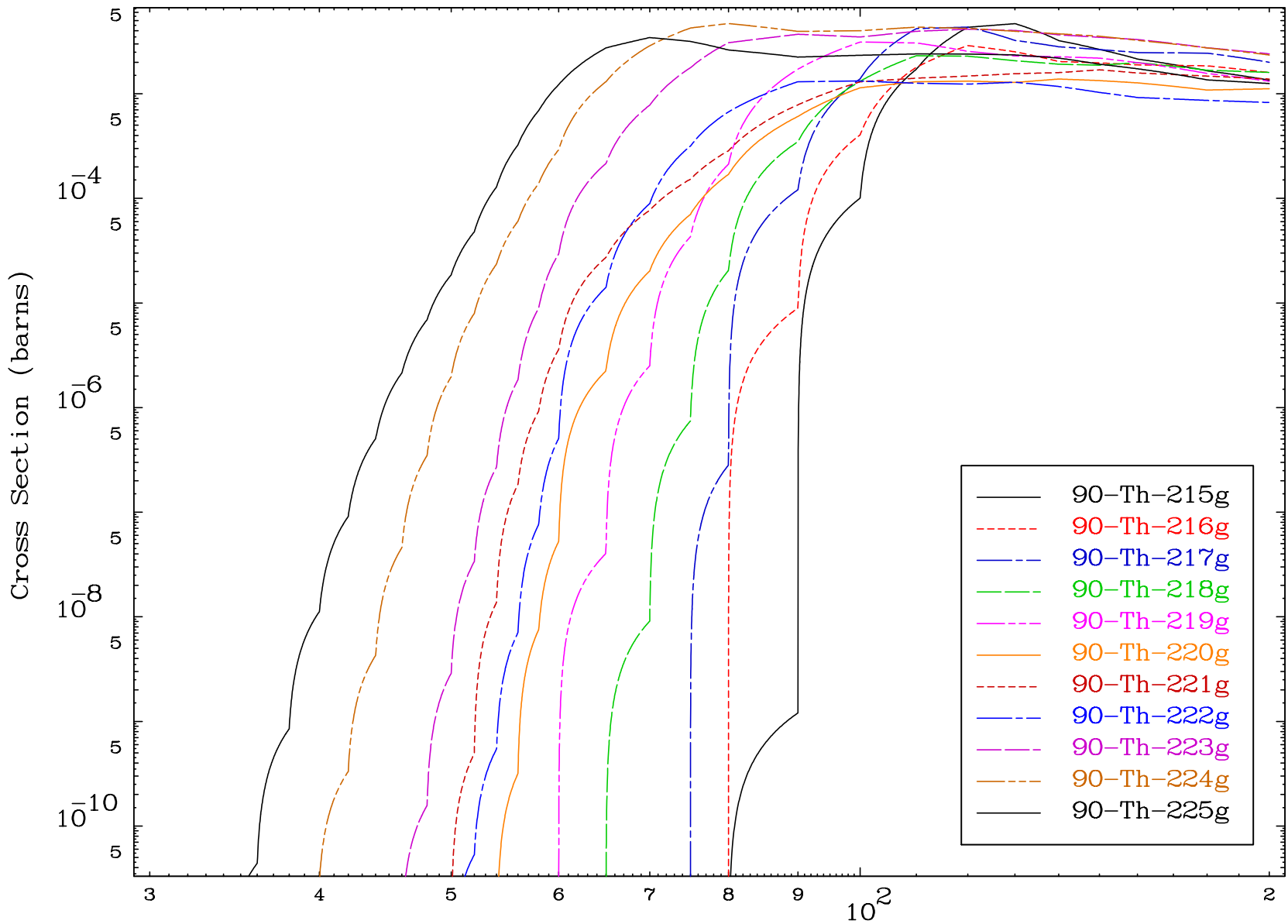
93-Np-233



MAT 9334

(n,remainder)  
Radionuclide Production Cross Section

93-Np-233



23

Incident Energy (MeV)

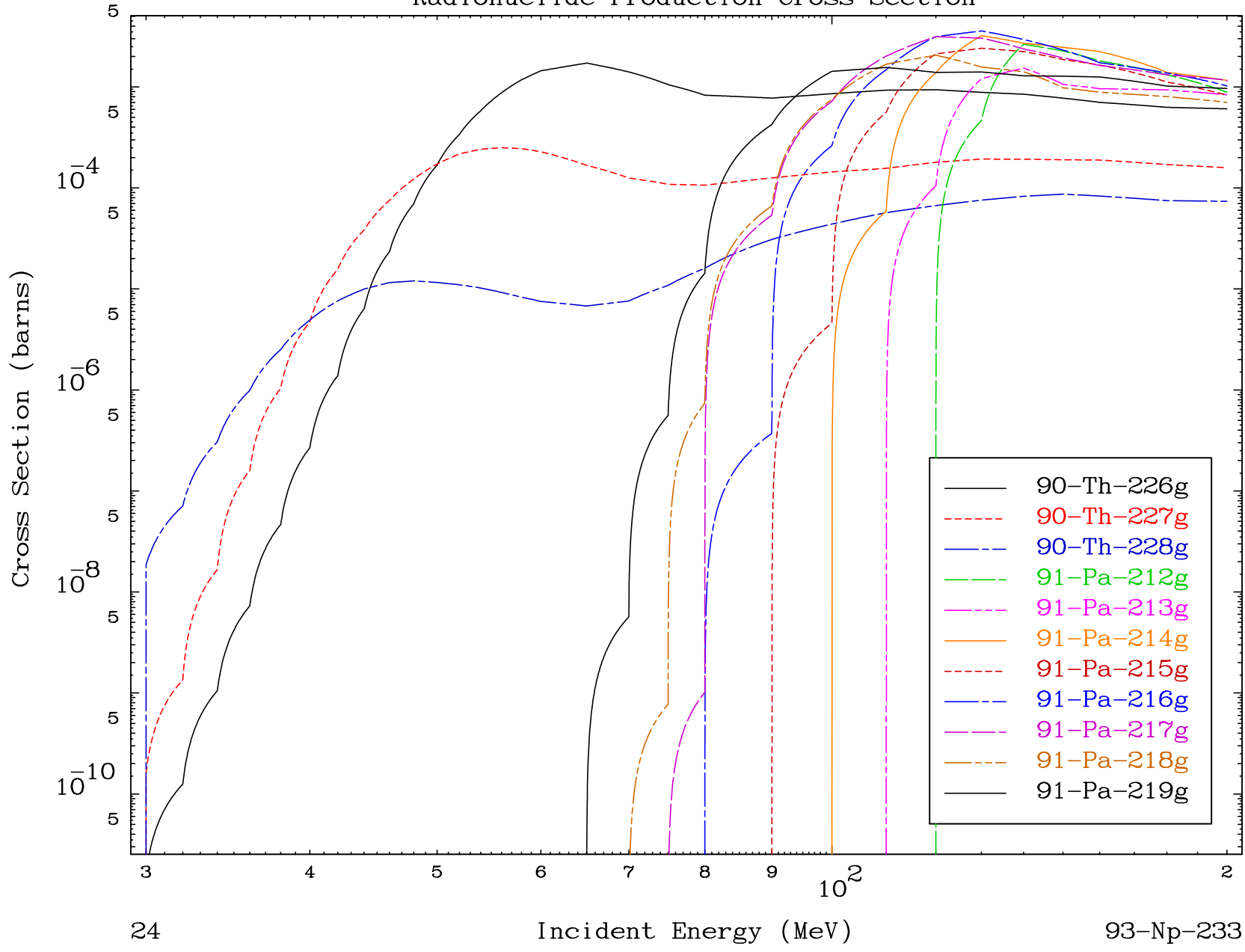
93-Np-233

MAT 9334

(n,remainder)

93-Np-233

Radionuclide Production Cross Section



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93-Np-233

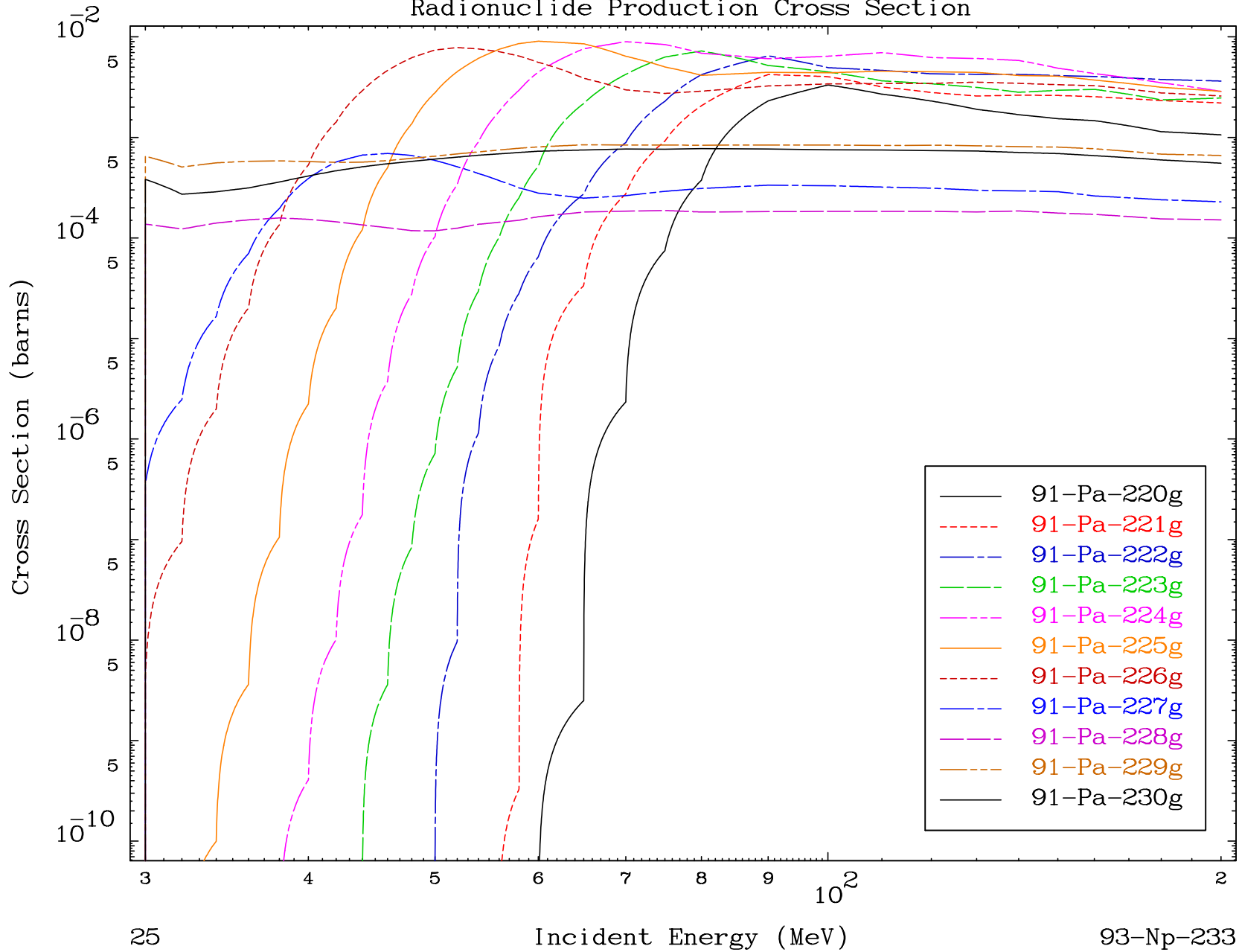


MAT 9334

(n,remainder)

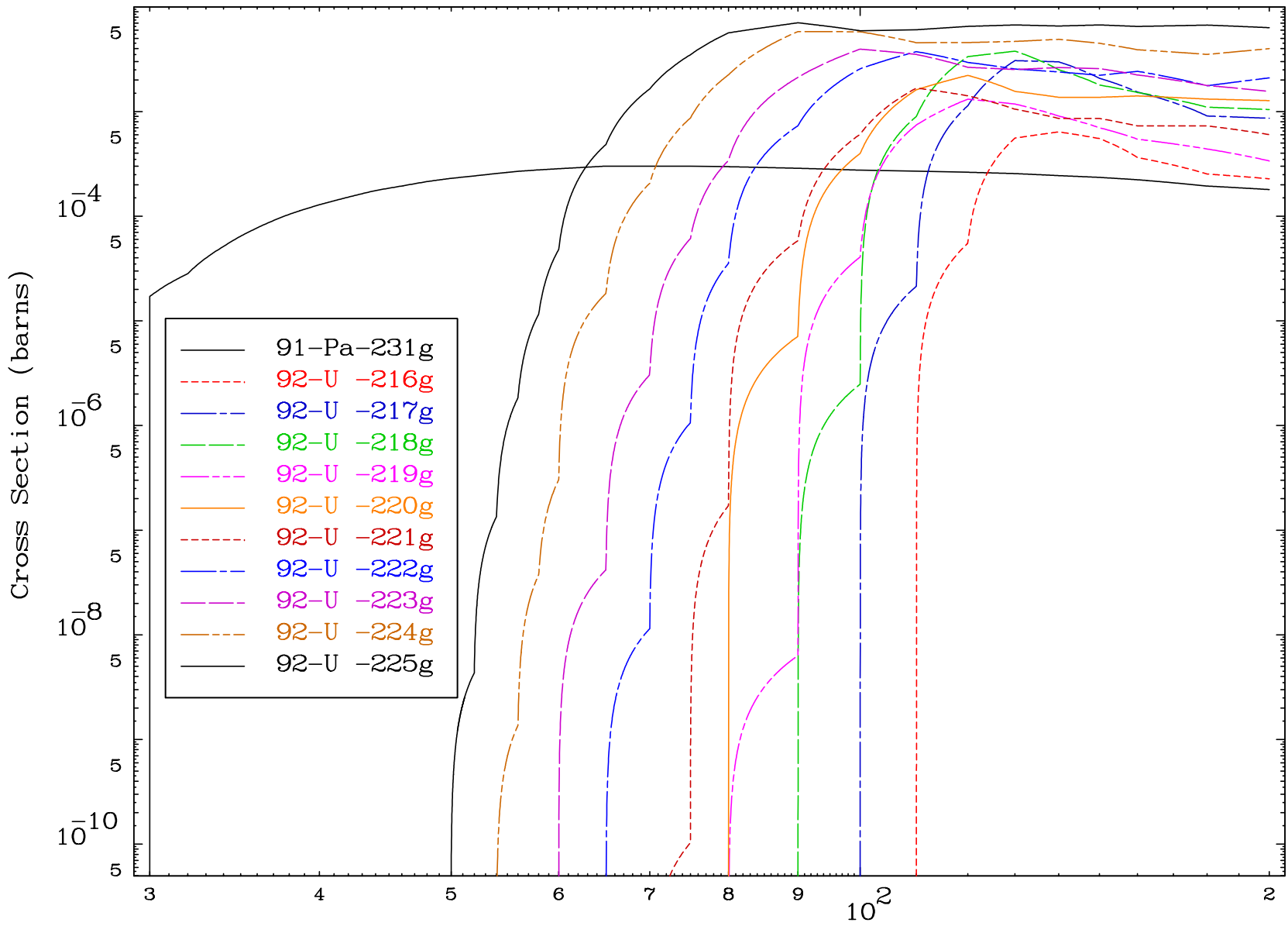
93-Np-233

### Radionuclide Production Cross Section



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93-Np-233

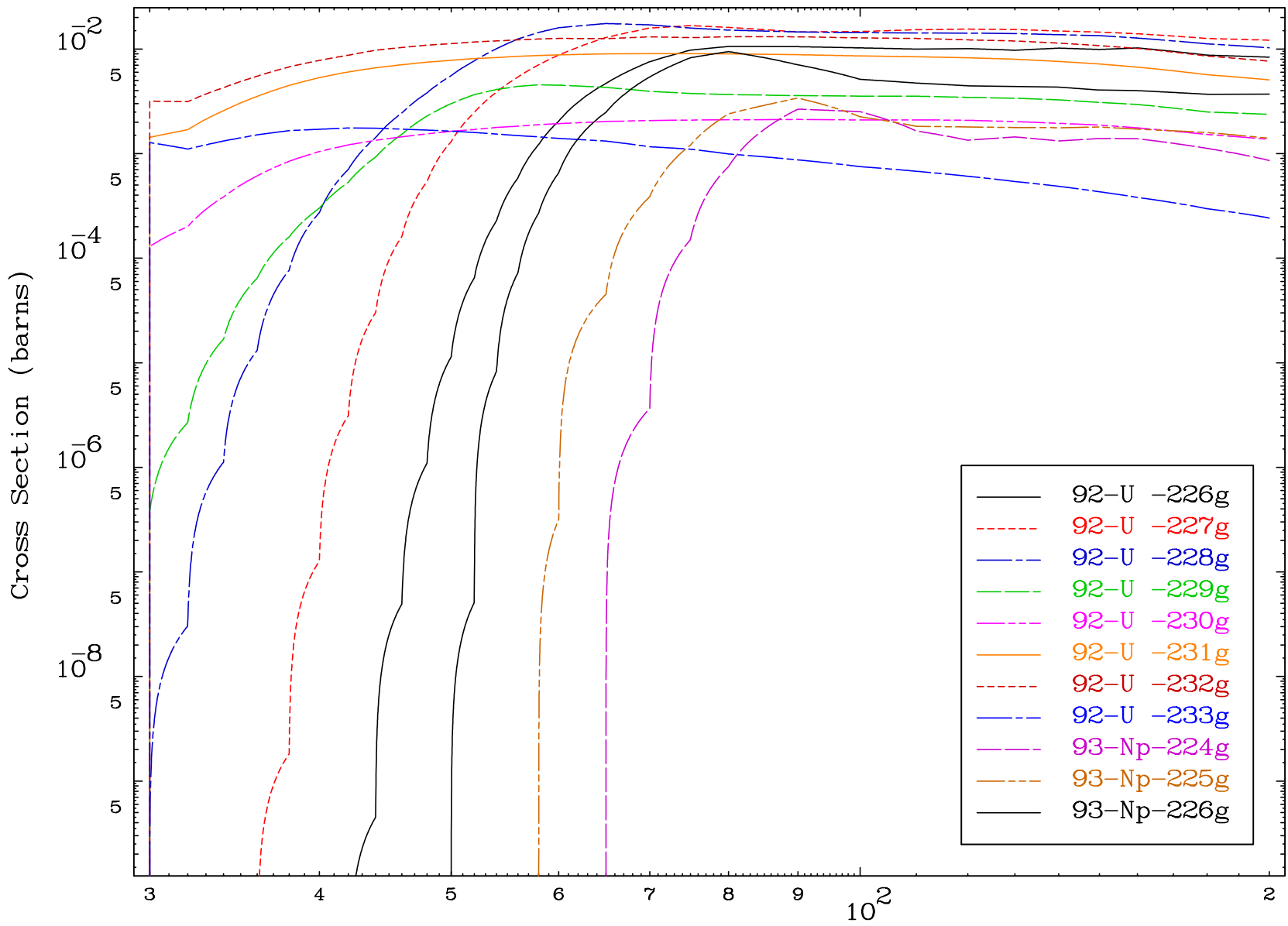


MAT 9334

(n,remainder)

93-Np-233

### Radionuclide Production Cross Section



27

Incident Energy (MeV)

93-Np-233

