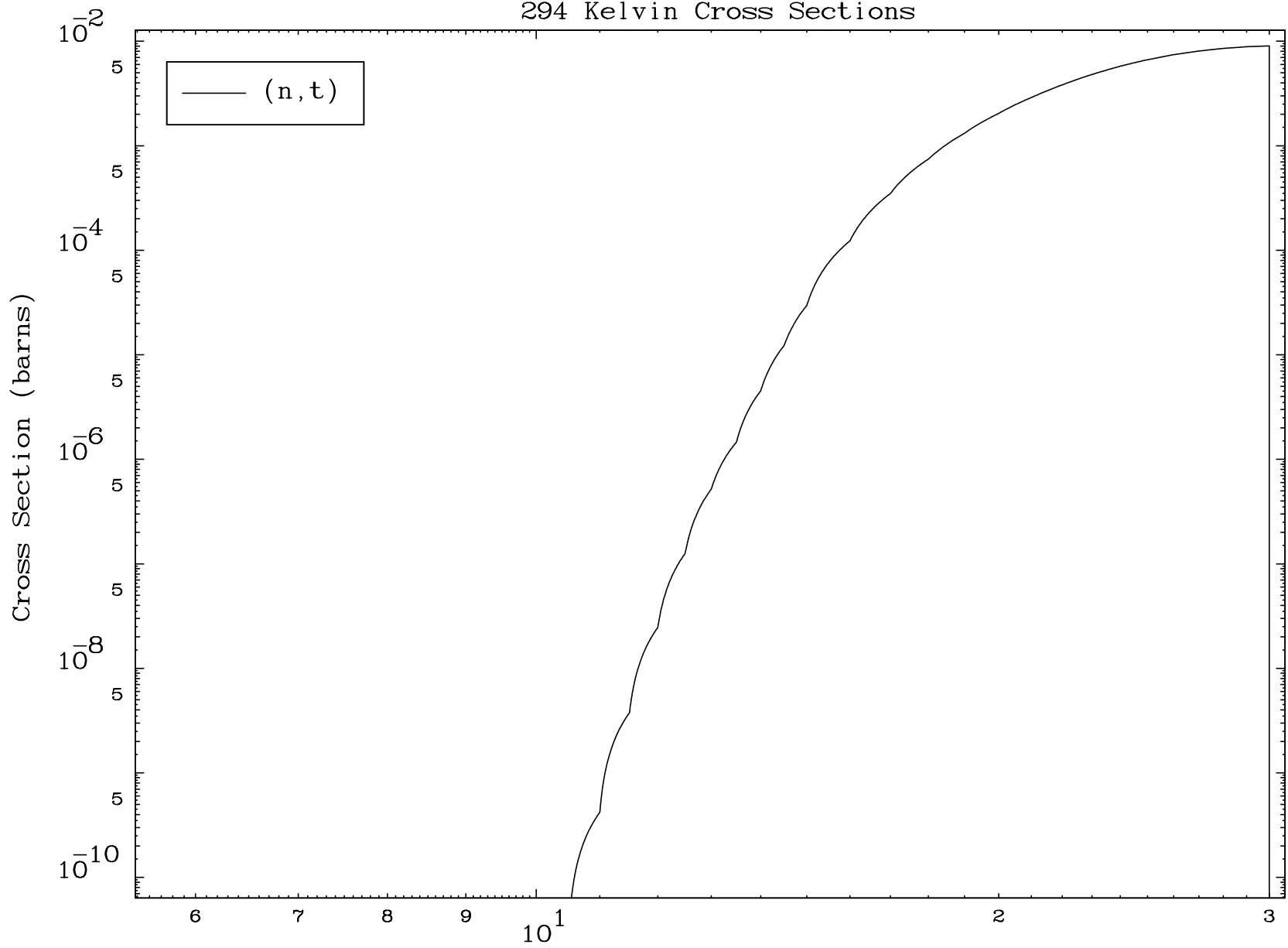


MAT 7613

(n,t) Levels  
294 Kelvin Cross Sections

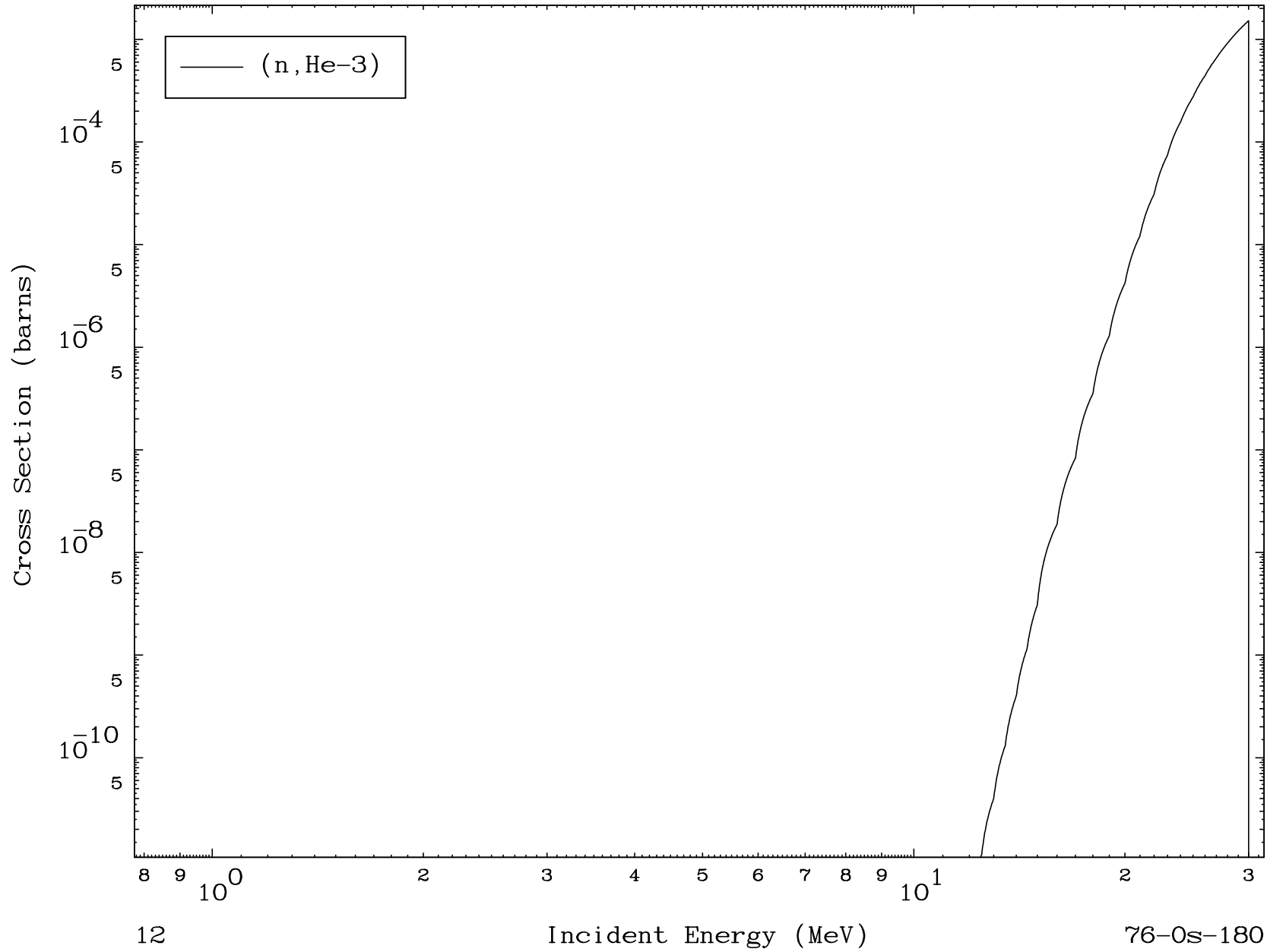
76-0s-180

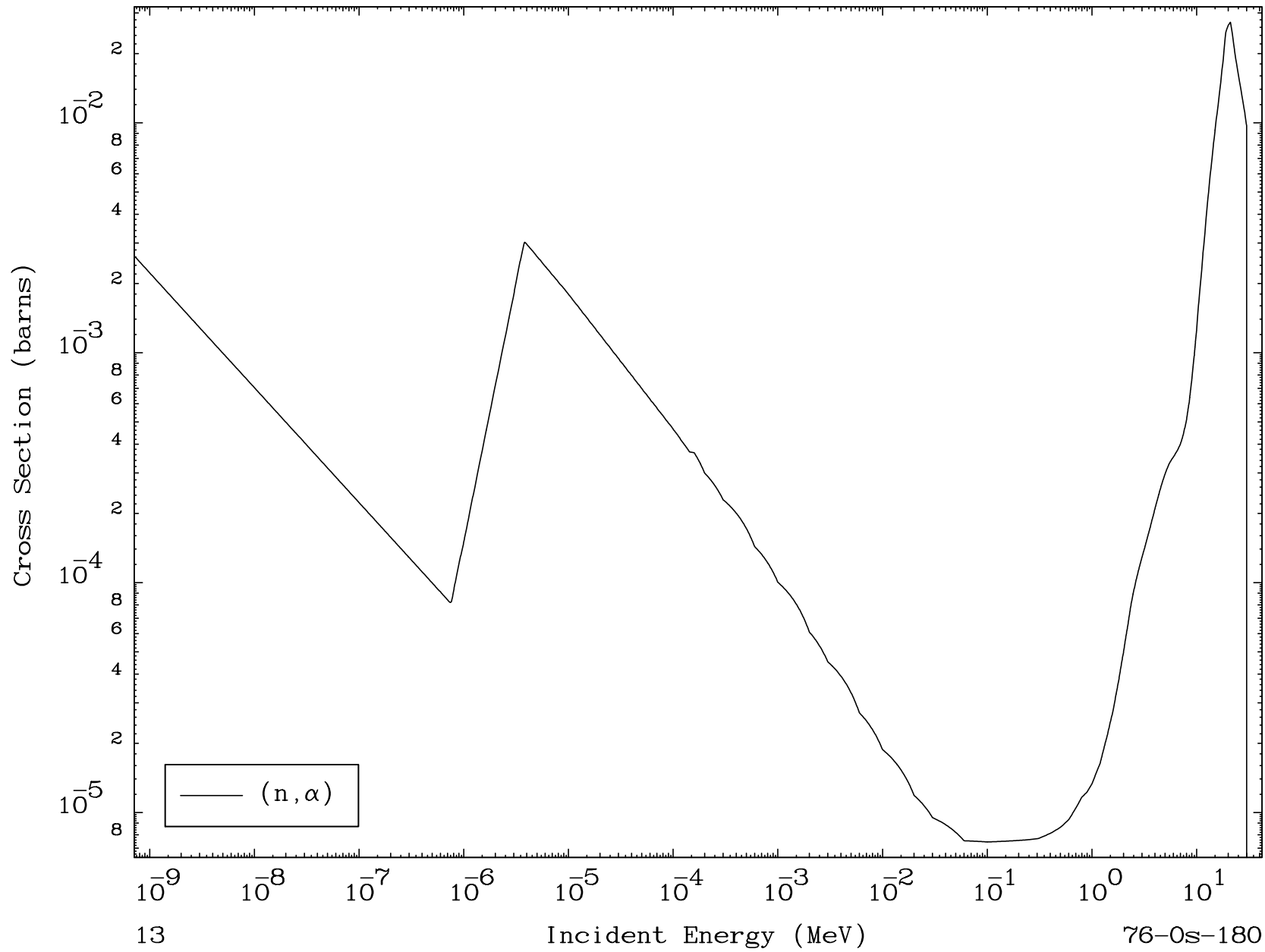


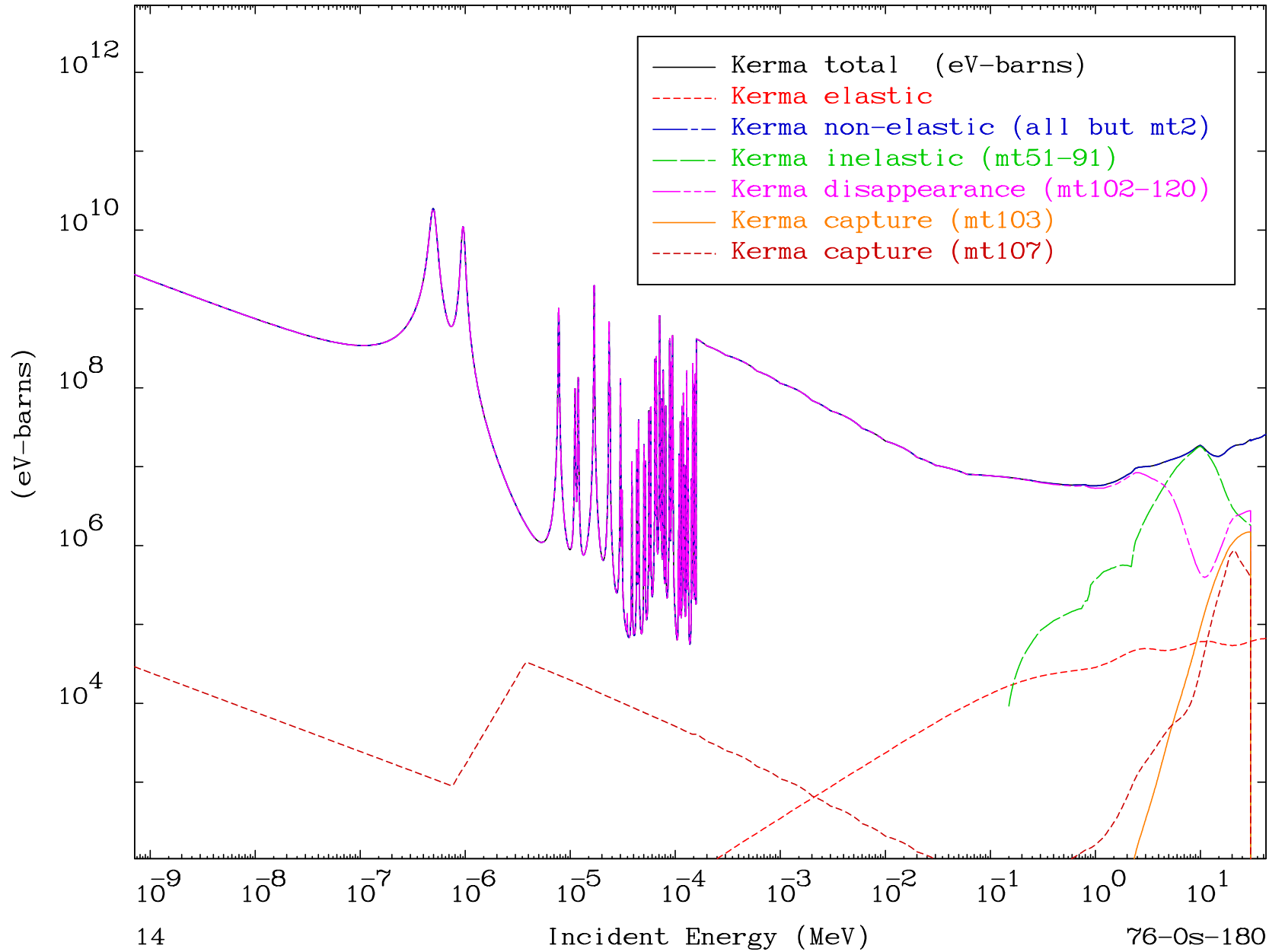
11

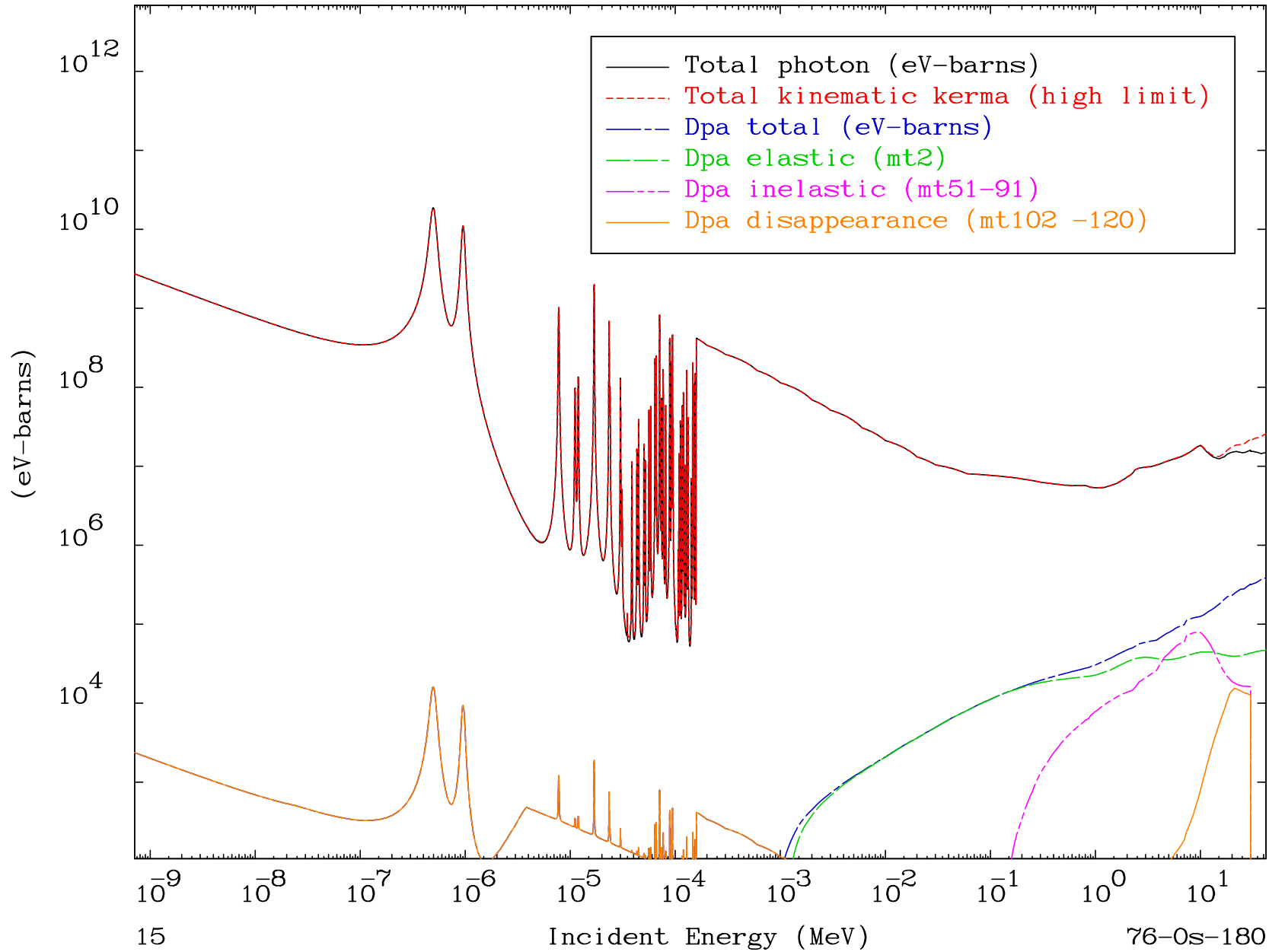
Incident Energy (MeV)

76-0s-180

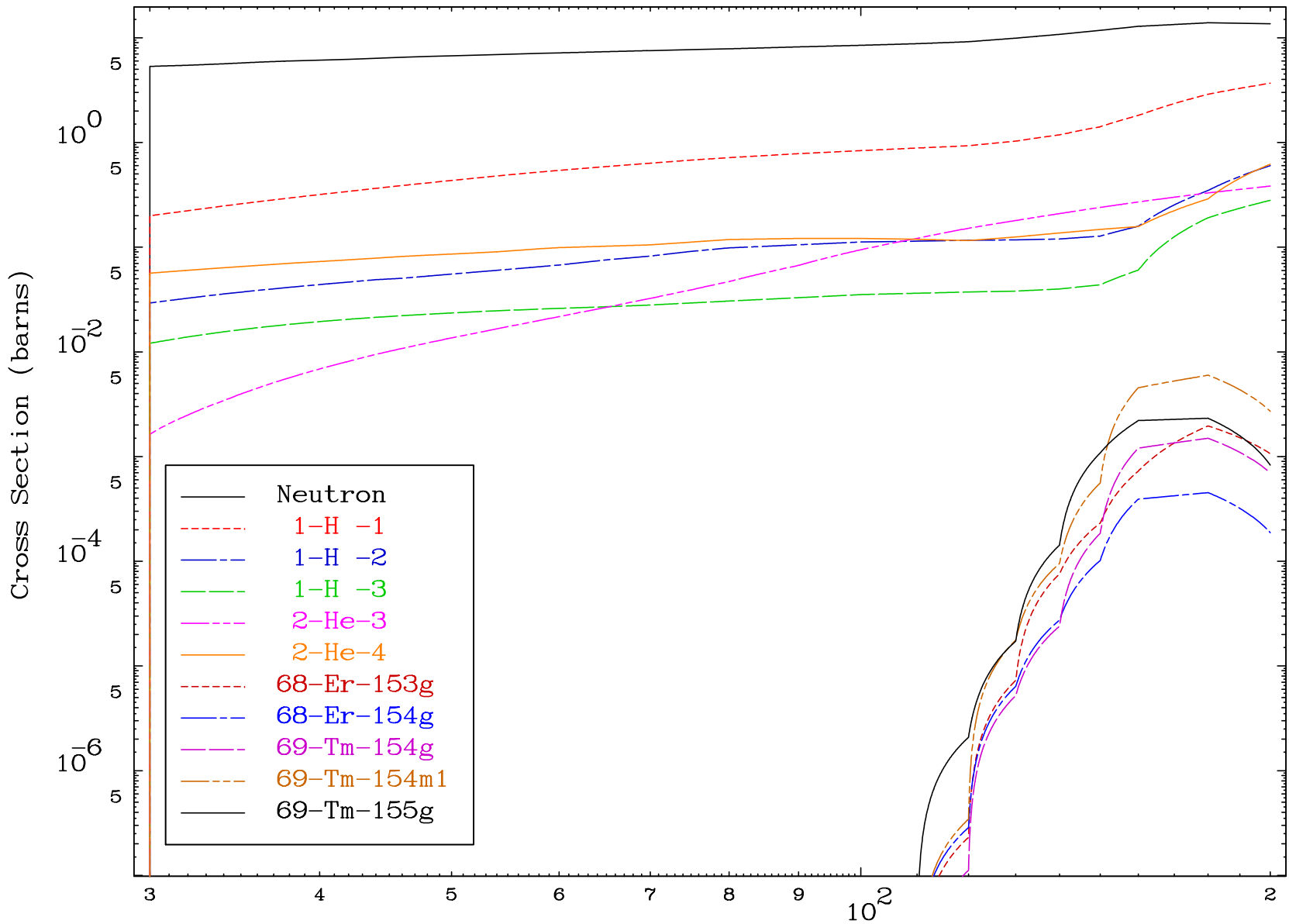




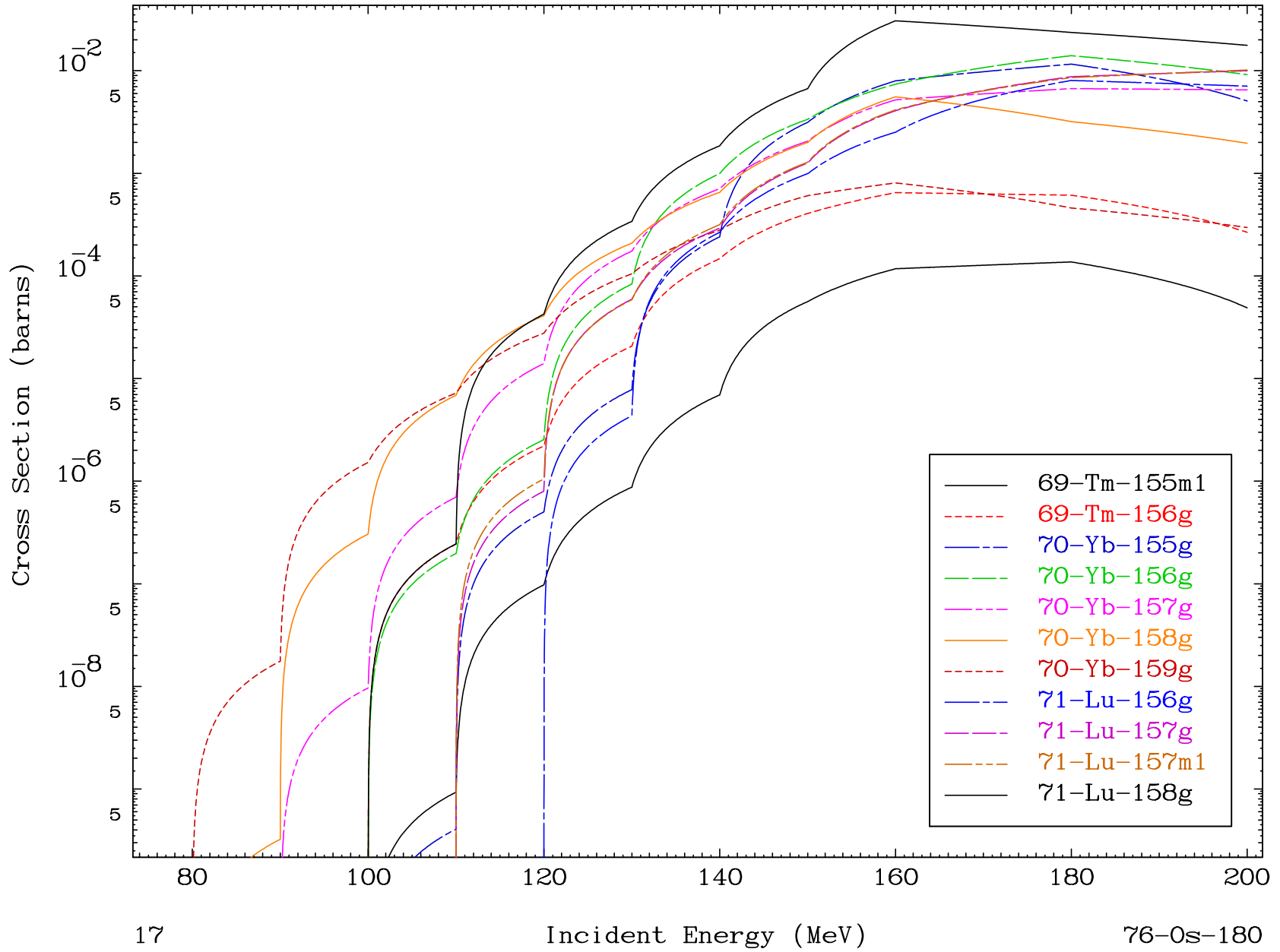


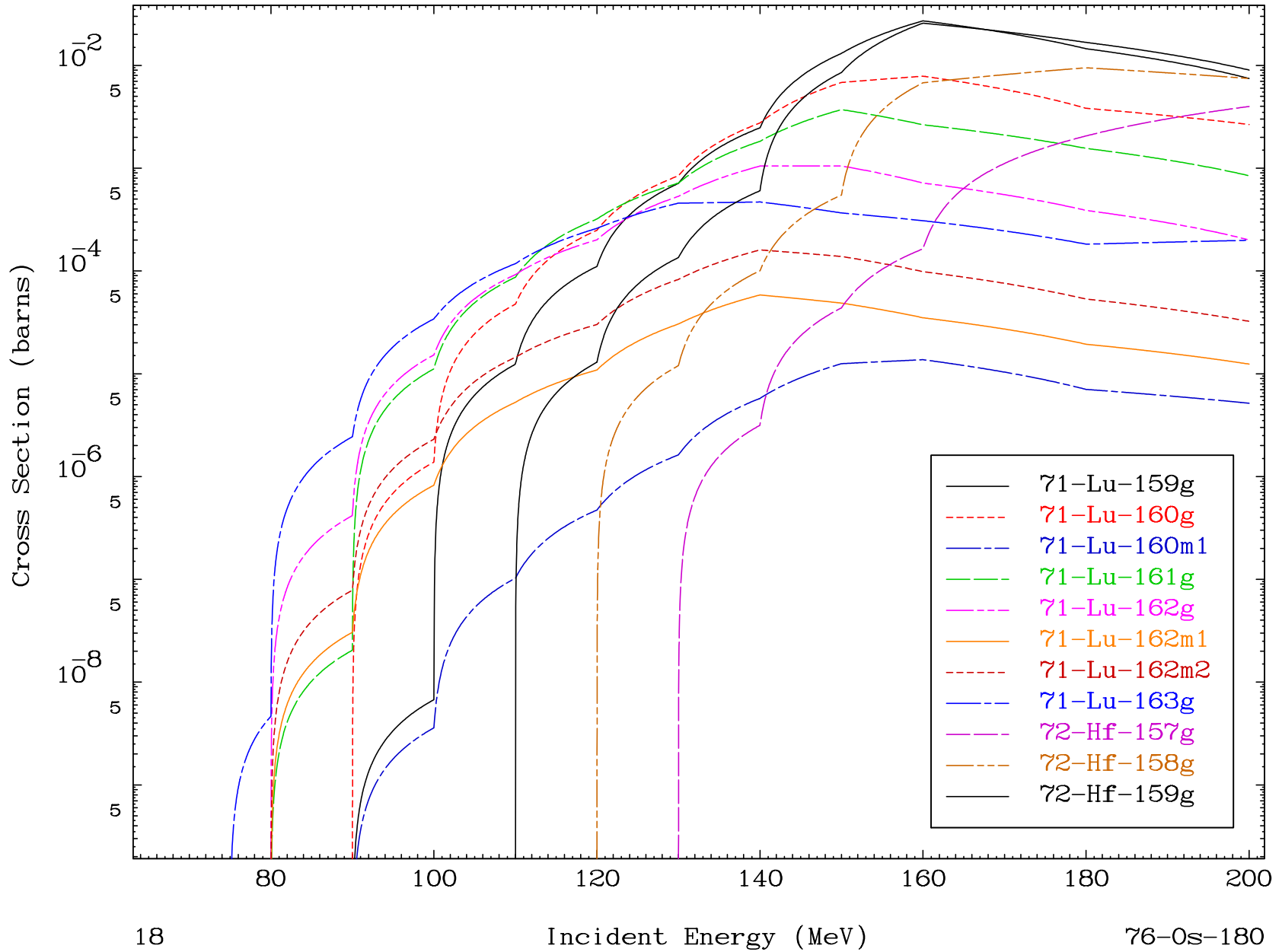


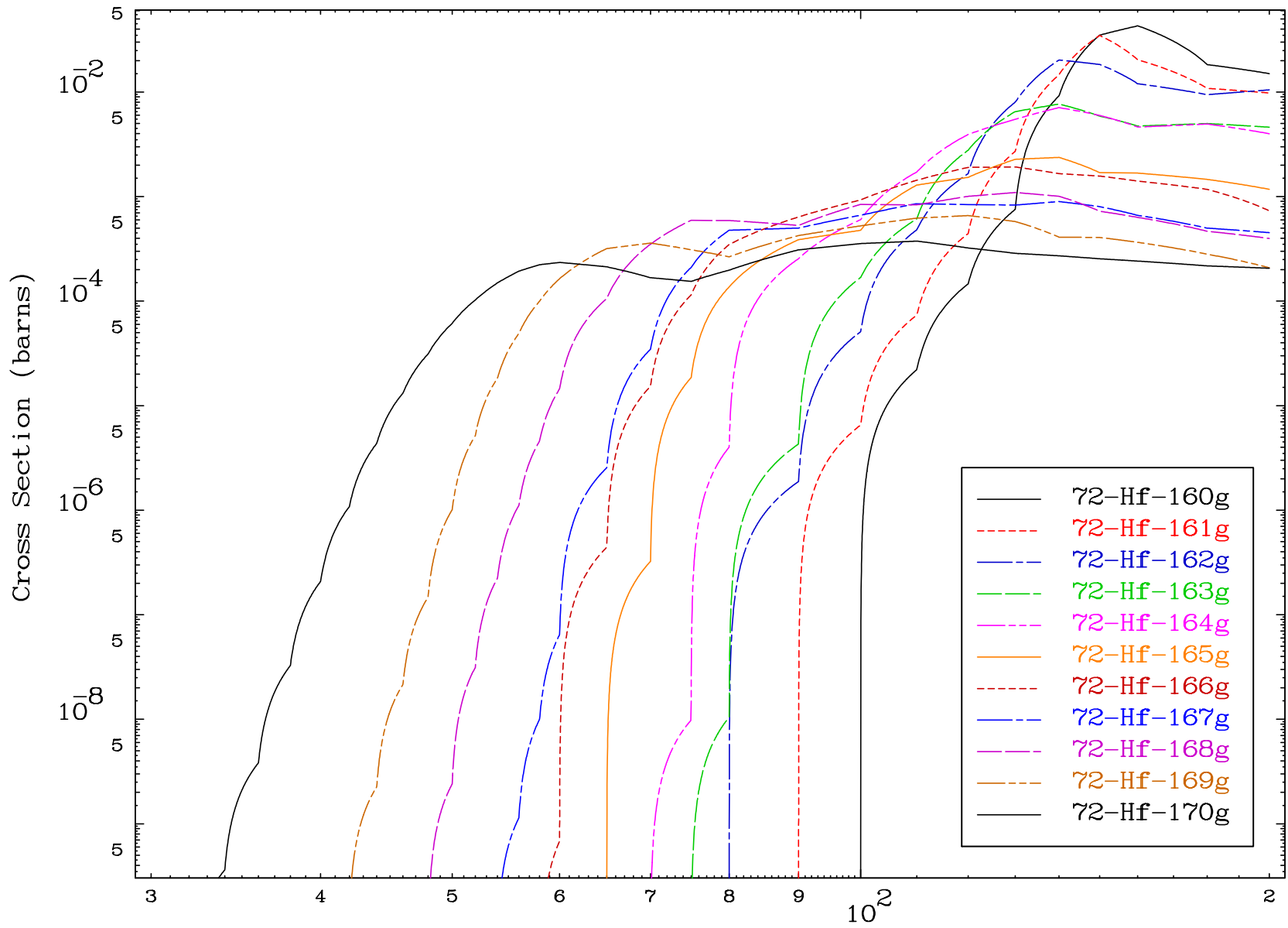
Radionuclide Production Cross Section









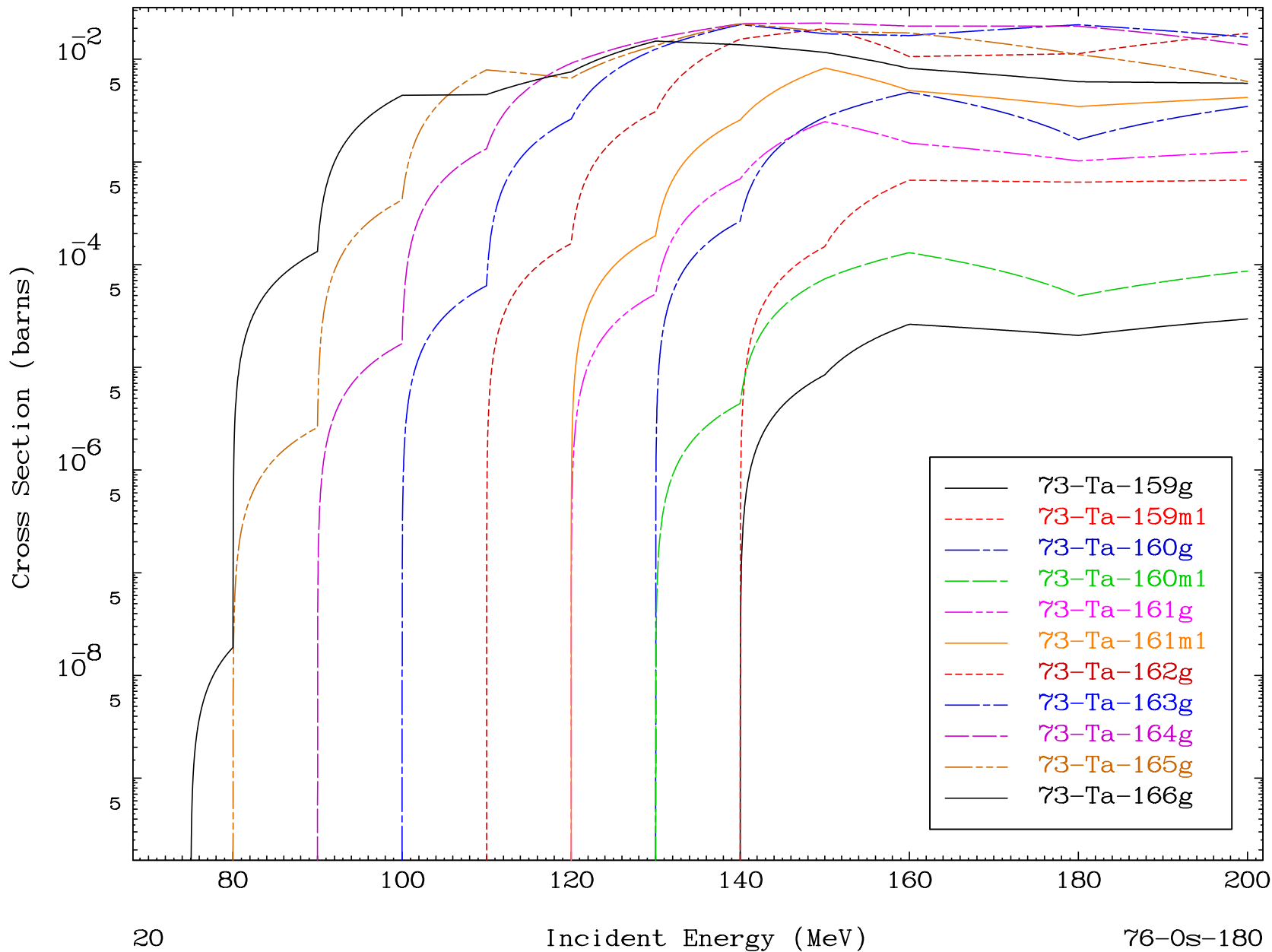


MAT 7613

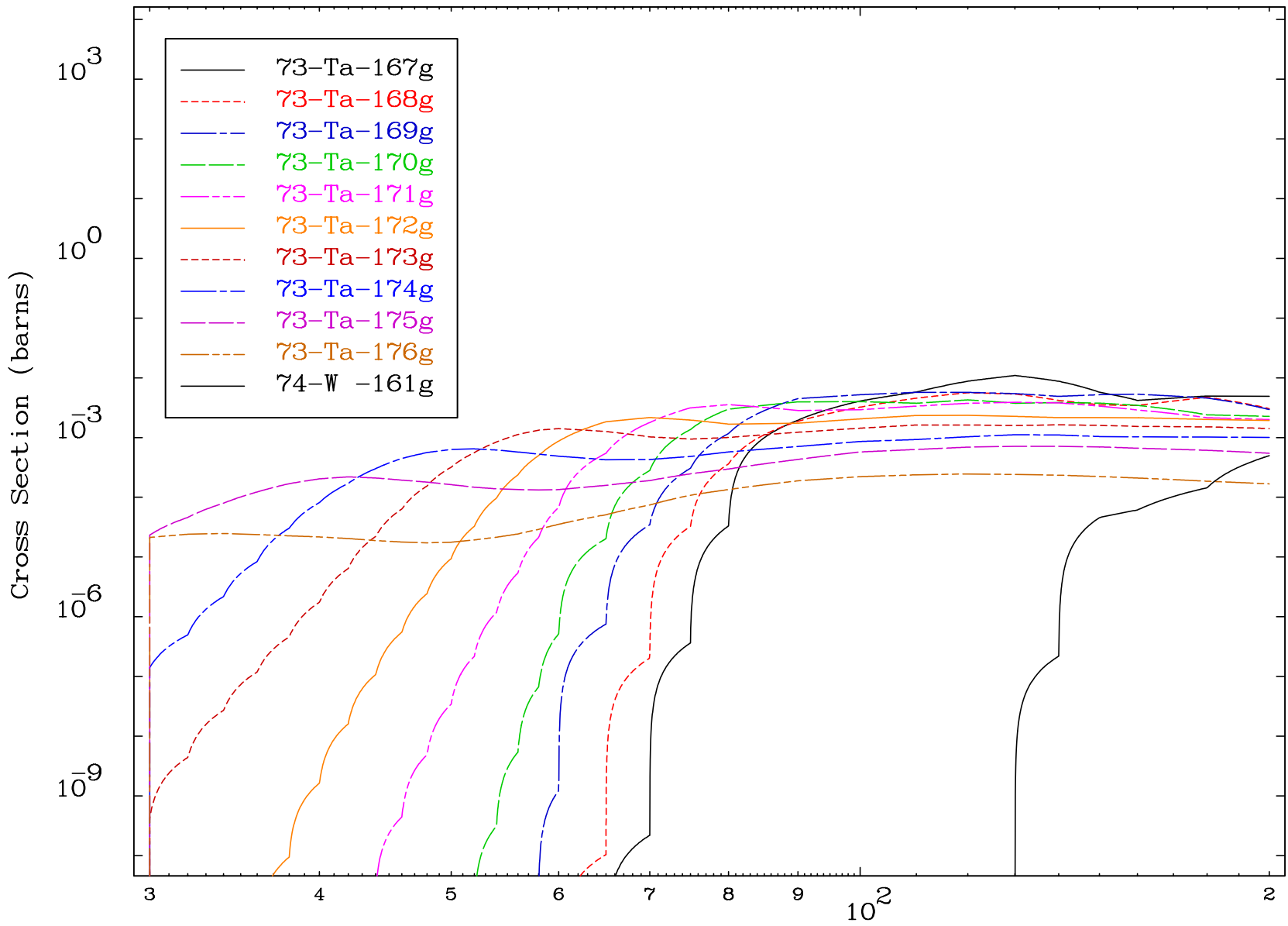
(n,remainder)

76-0s-180

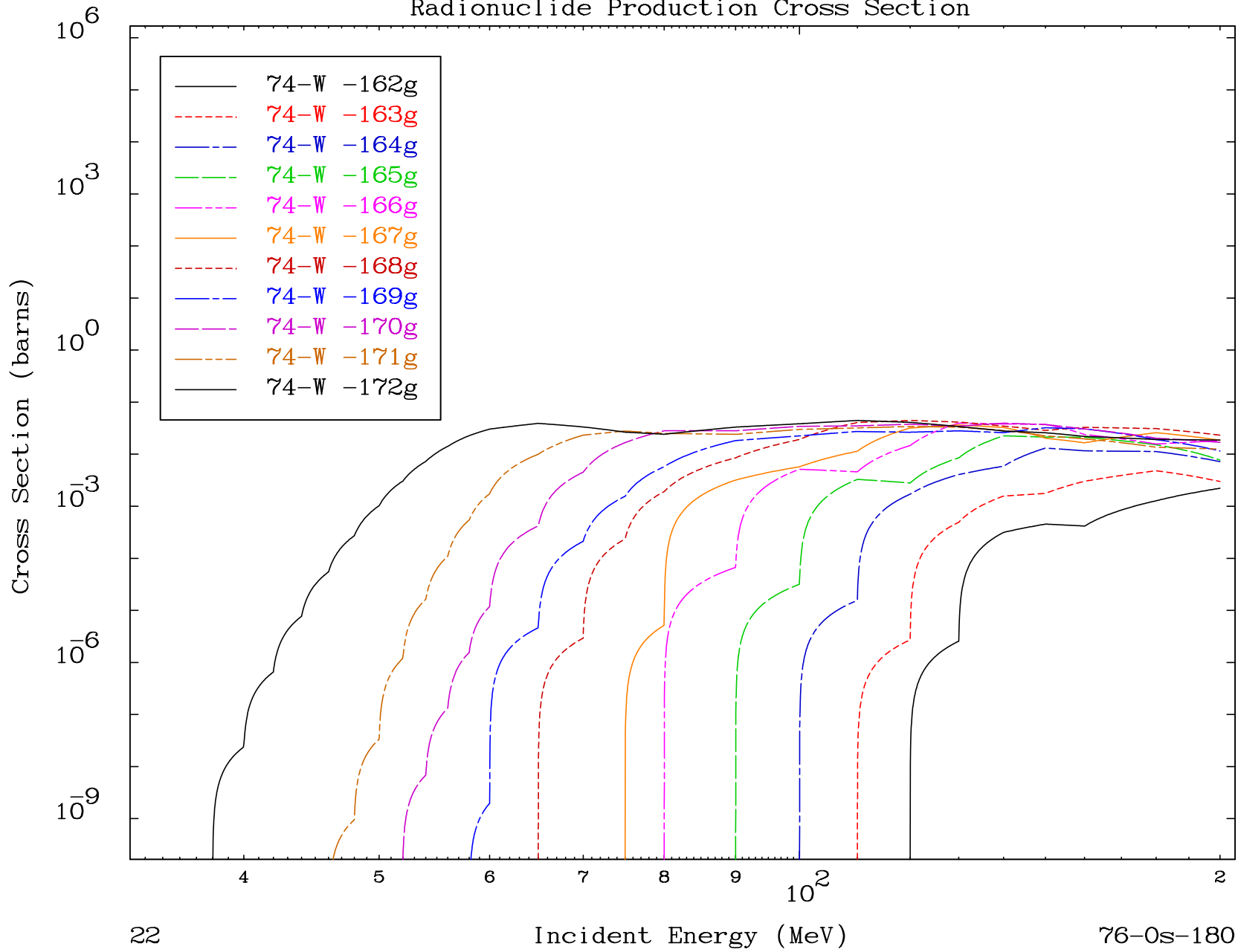
### Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

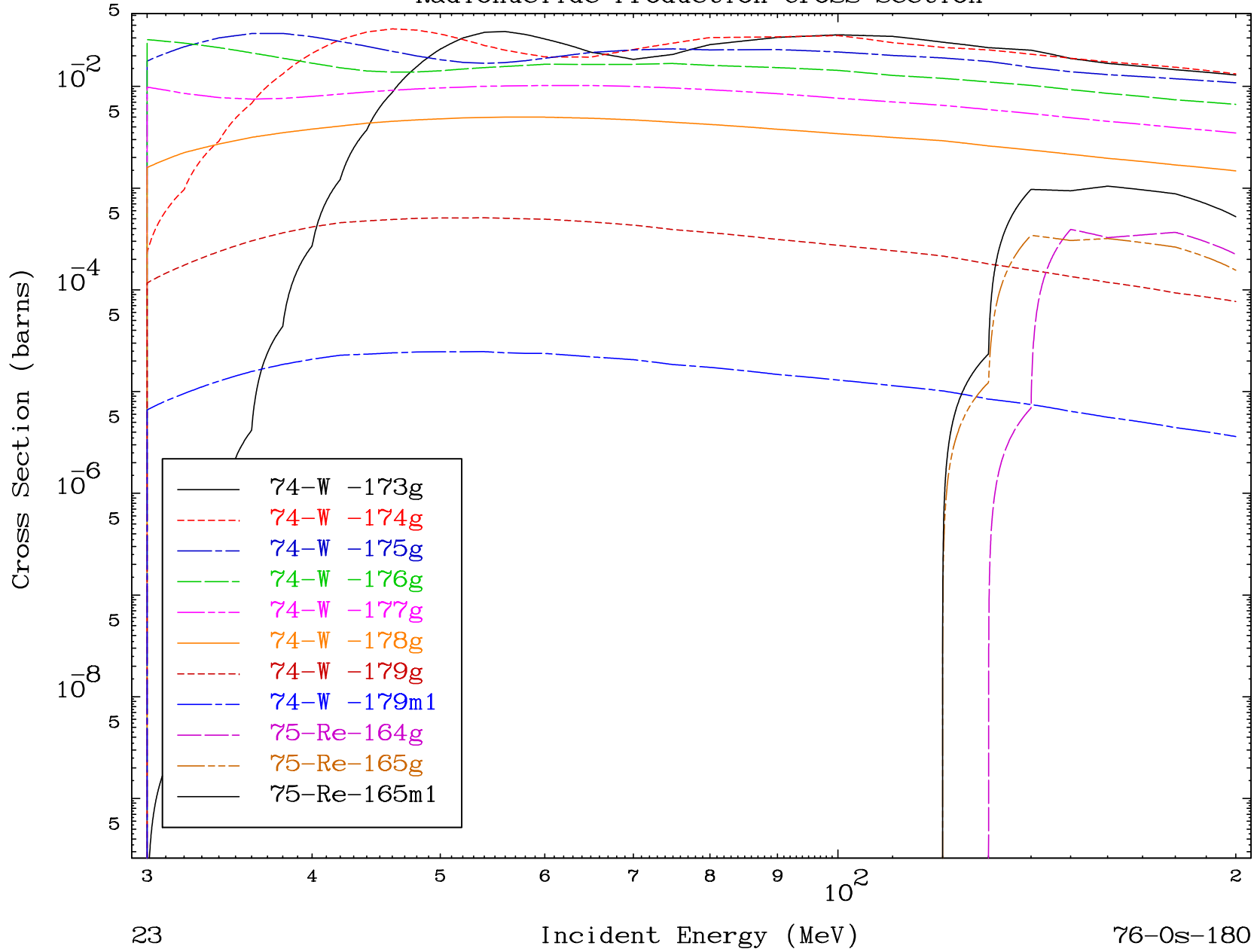


MAT 7613

(n,remainder)

76-0s-180

### Radionuclide Production Cross Section

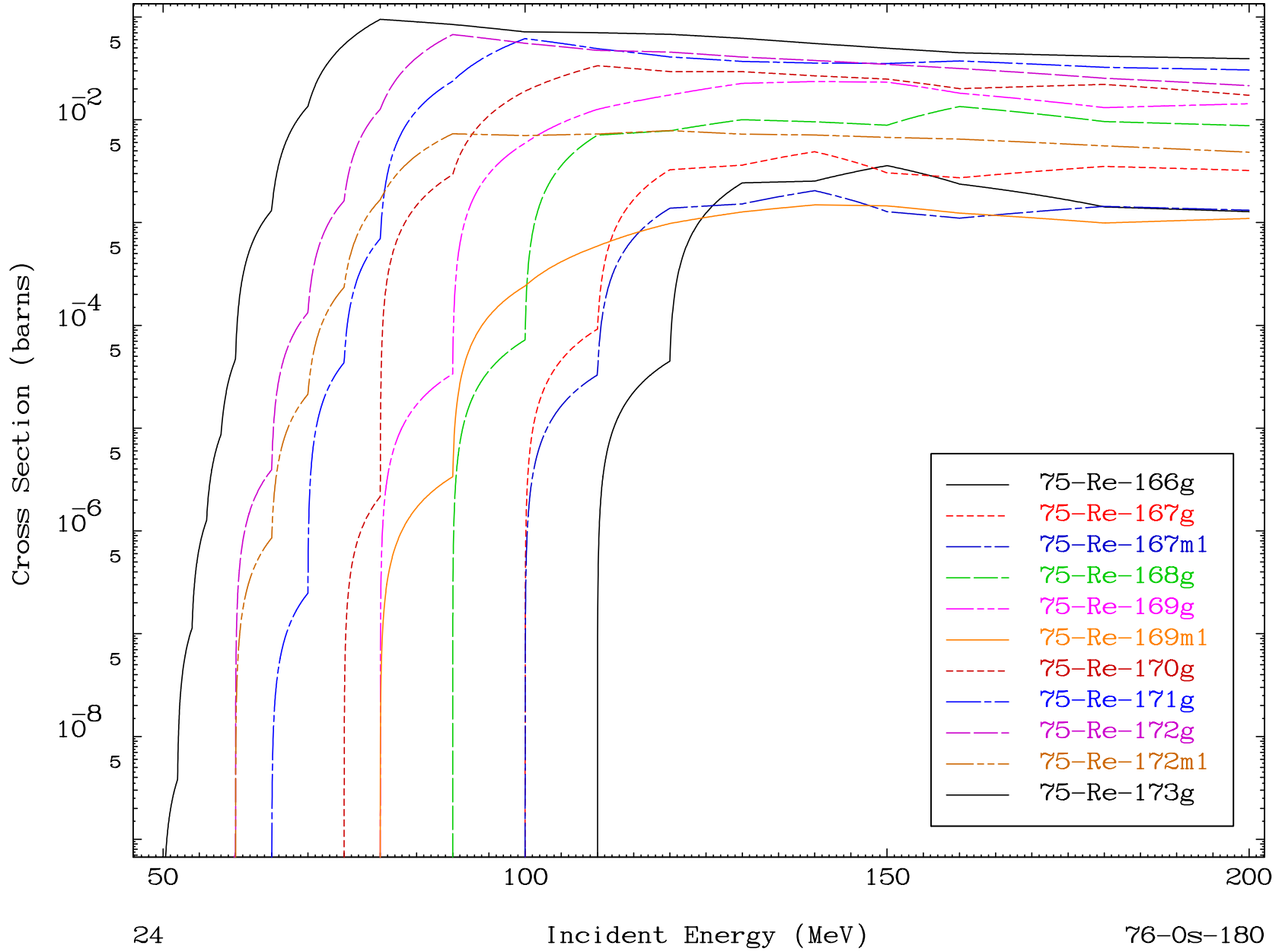


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Incident Energy (MeV)

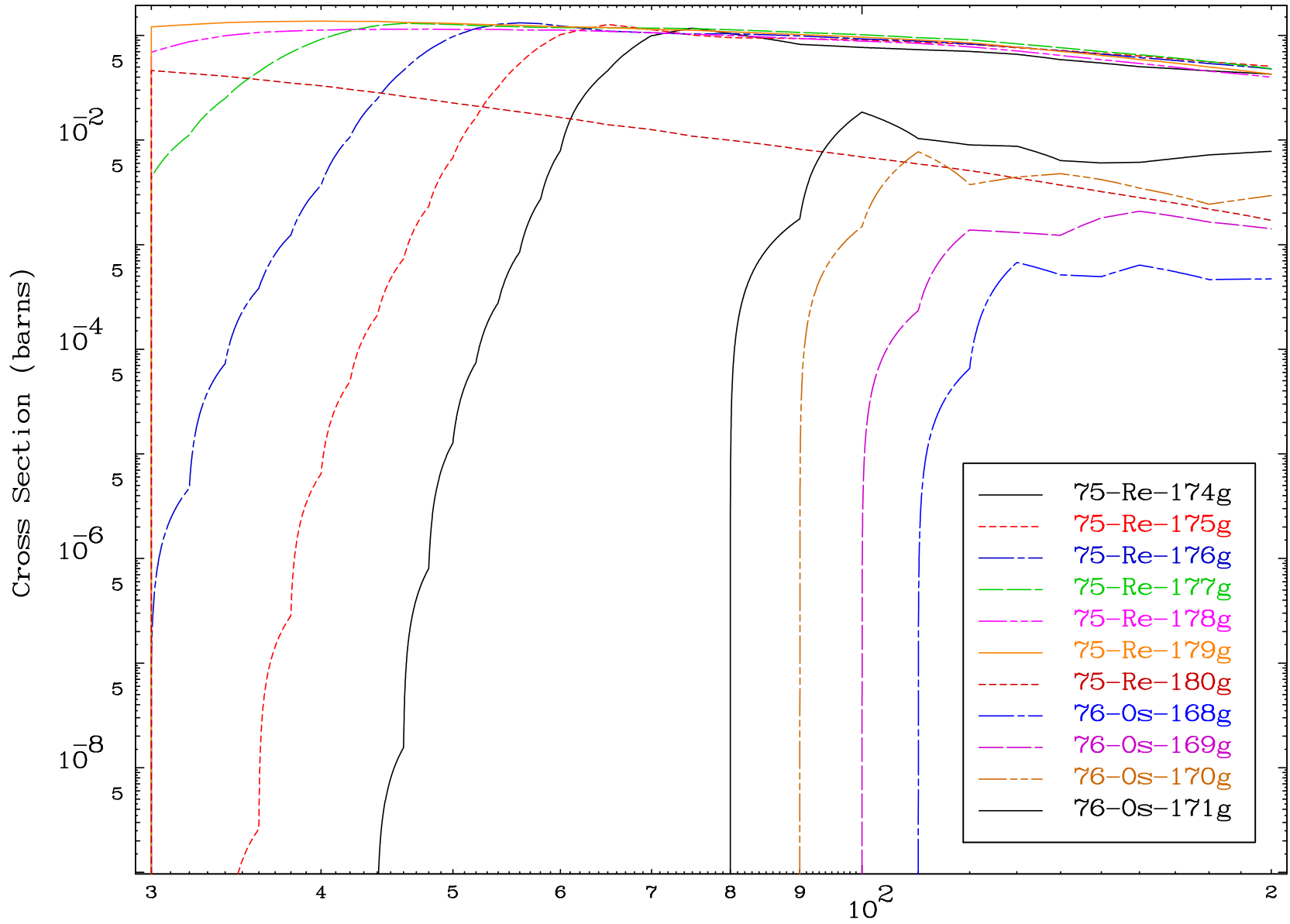
76-0s-180

Radionuclide Production Cross Section

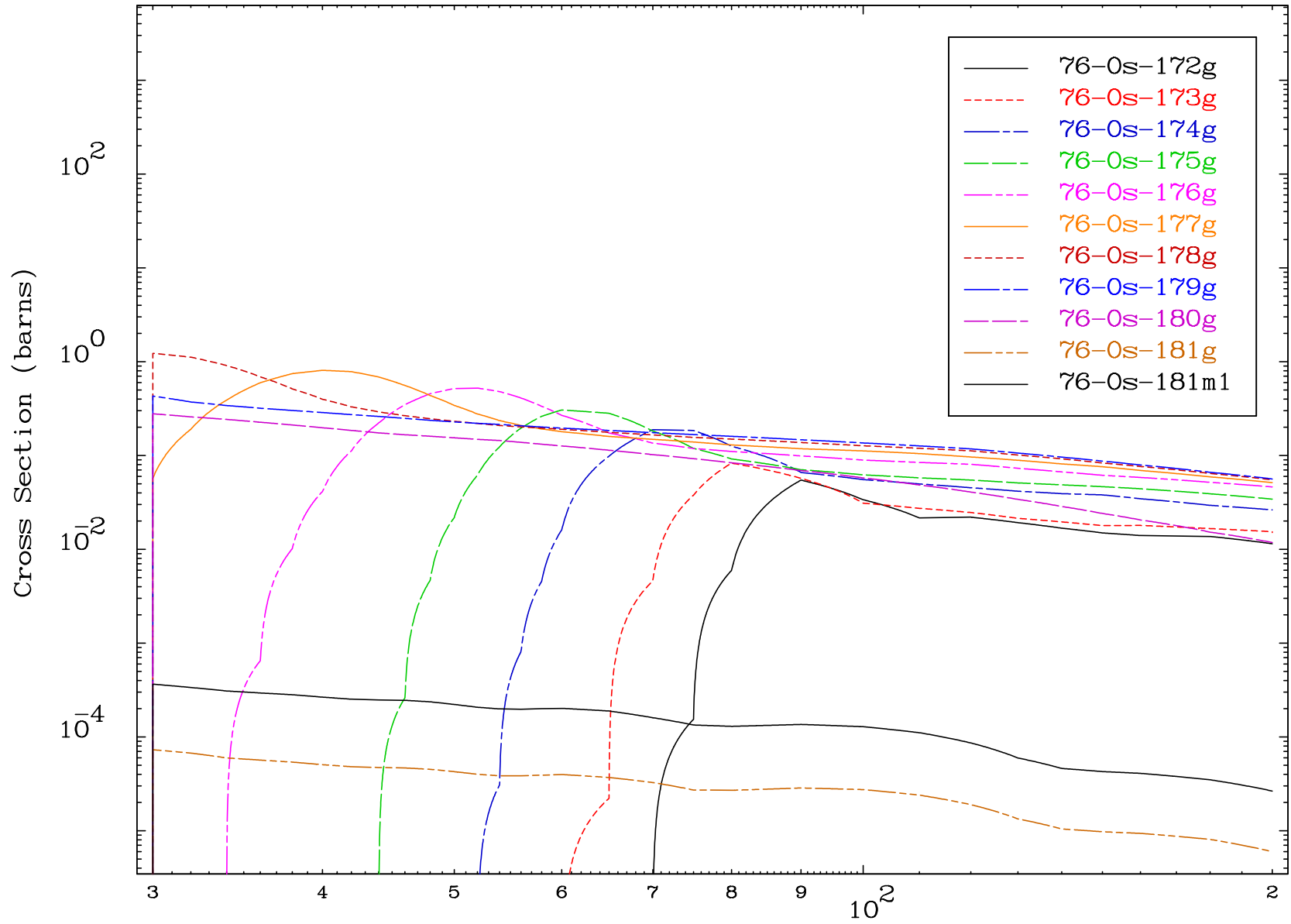




Radionuclide Production Cross Section



Radionuclide Production Cross Section

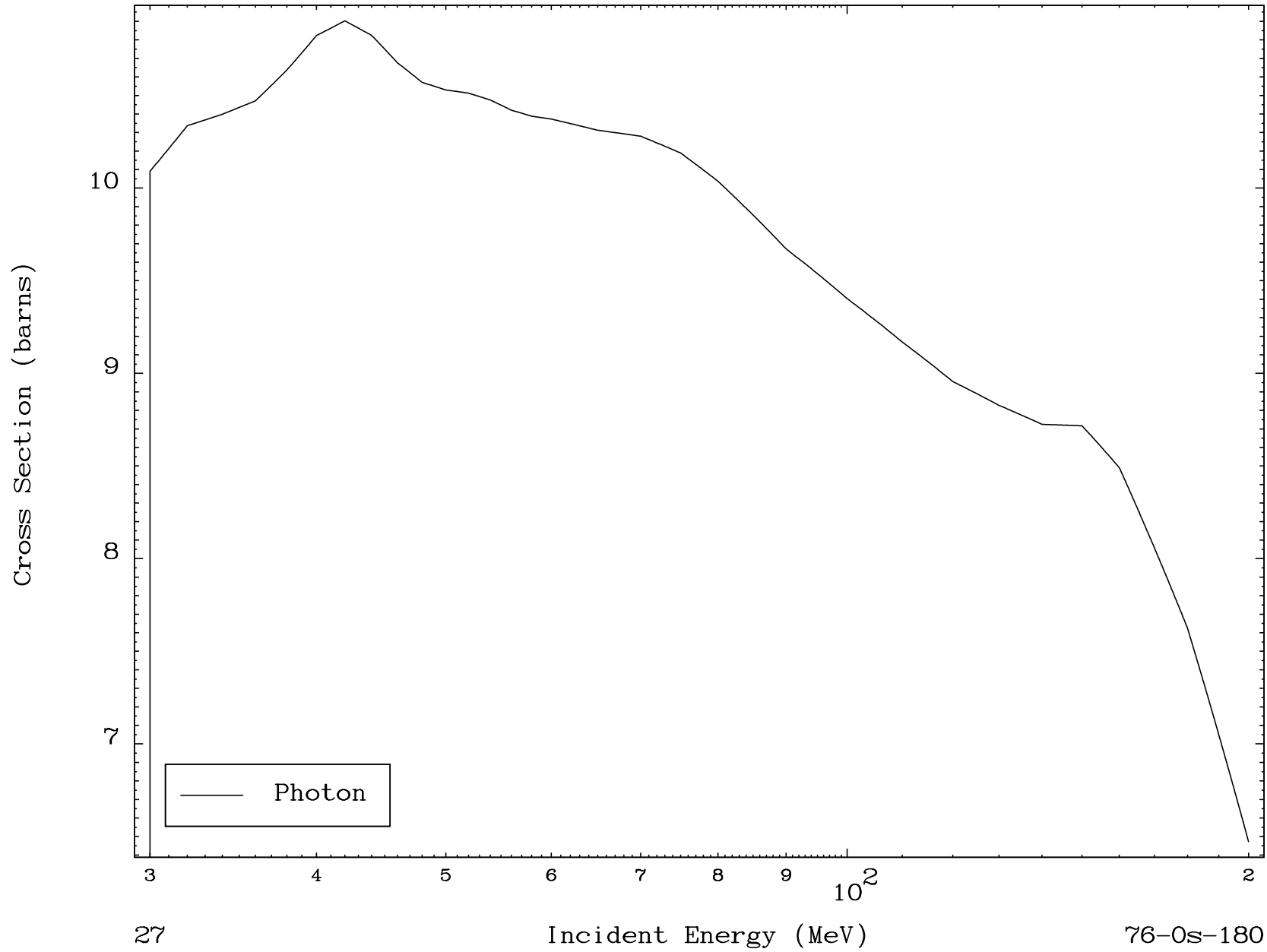


MAT 7613

(n,remainder)

76-0s-180

Radionuclide Production Cross Section



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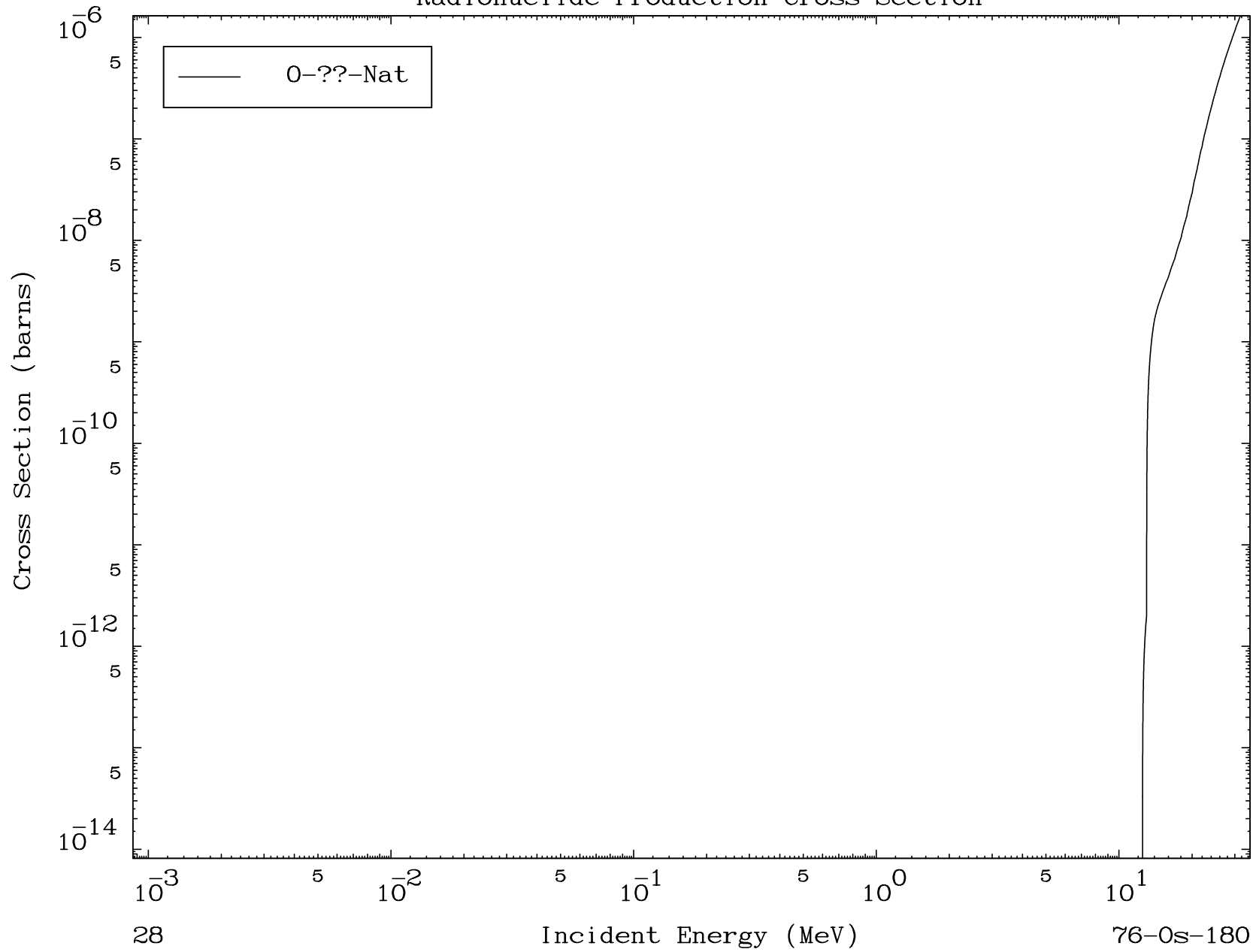
Incident Energy (MeV)

76-0s-180

MAT 7613

Fission  
Radionuclide Production Cross Section

76-0s-180

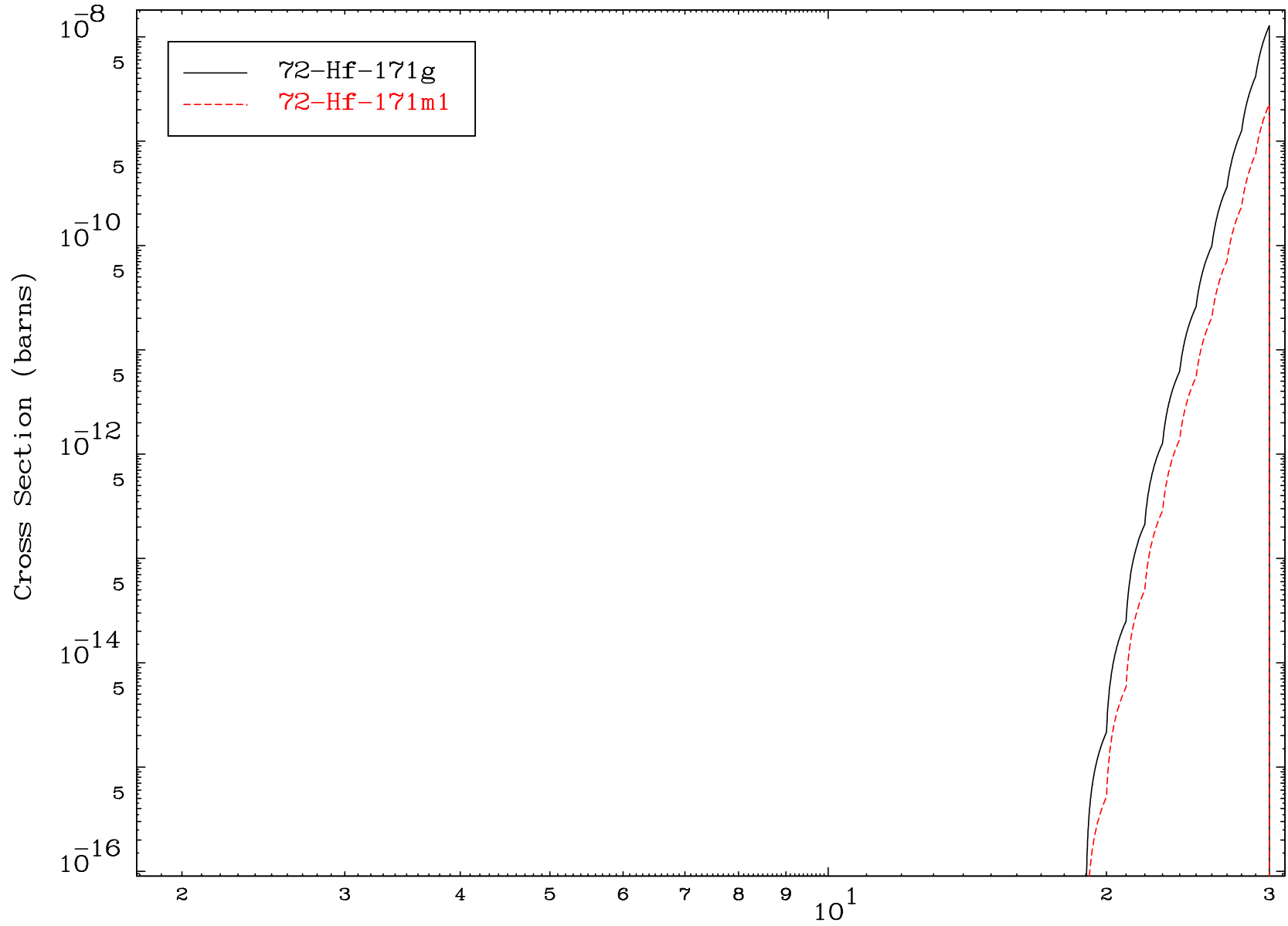


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(n,2n) 2 $\alpha$

76-0s-180

Radionuclide Production Cross Section



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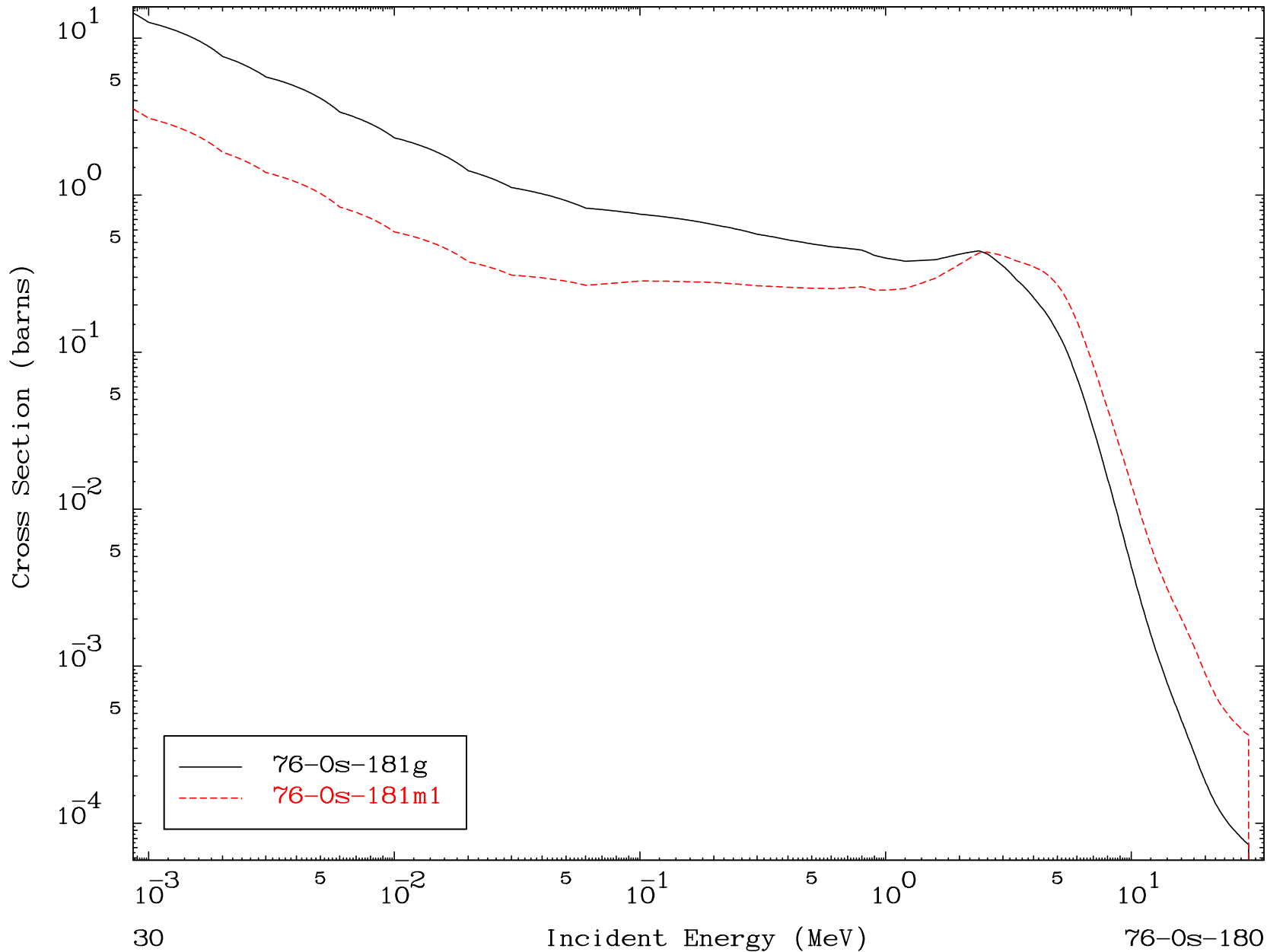
Incident Energy (MeV)

76-0s-180

MAT 7613

(n,  $\gamma$ )  
Radionuclide Production Cross Section

76-0s-180



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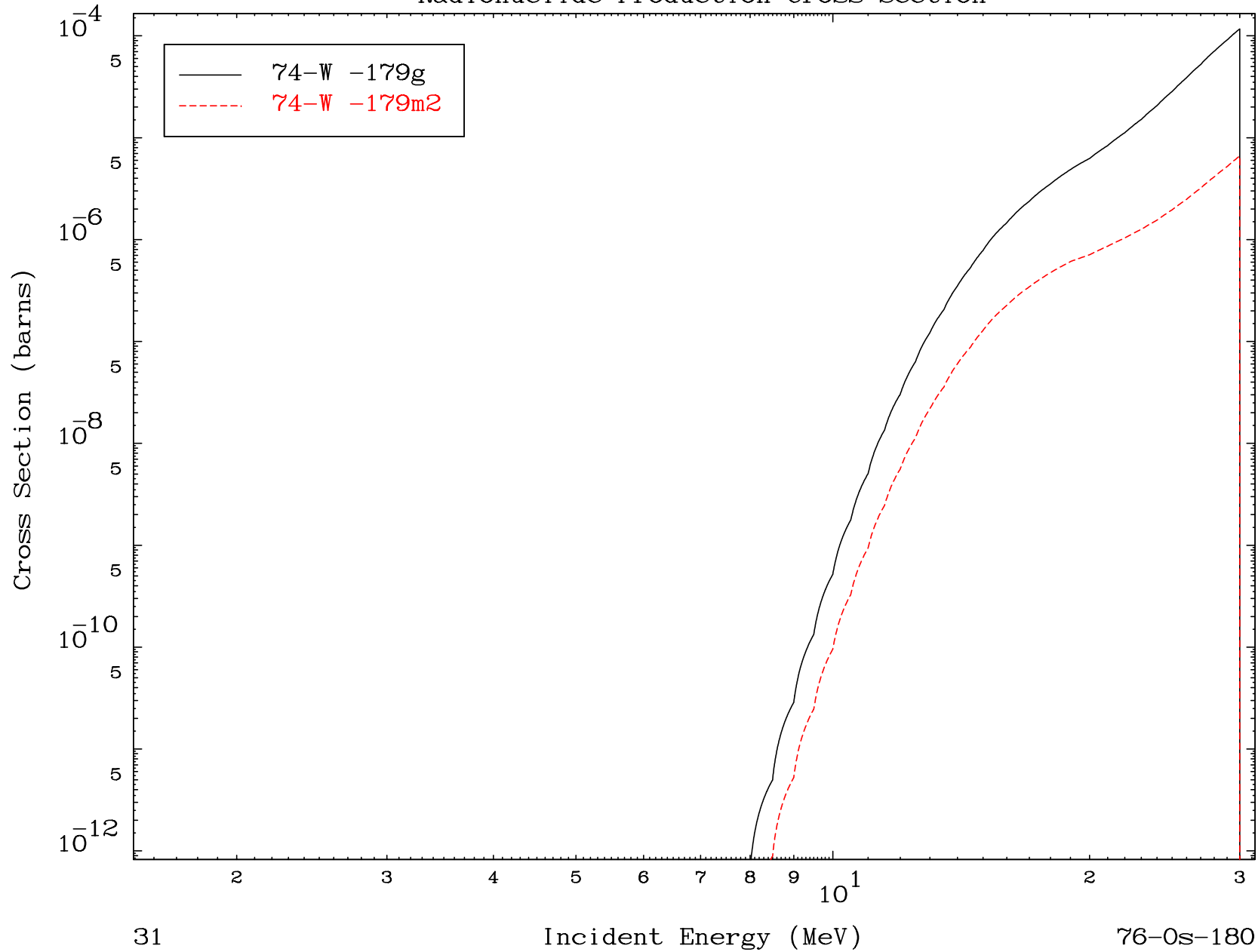
76-0s-180

MAT 7613

(n,2p)

76-0s-180

Radionuclide Production Cross Section



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Incident Energy (MeV)

76-0s-180