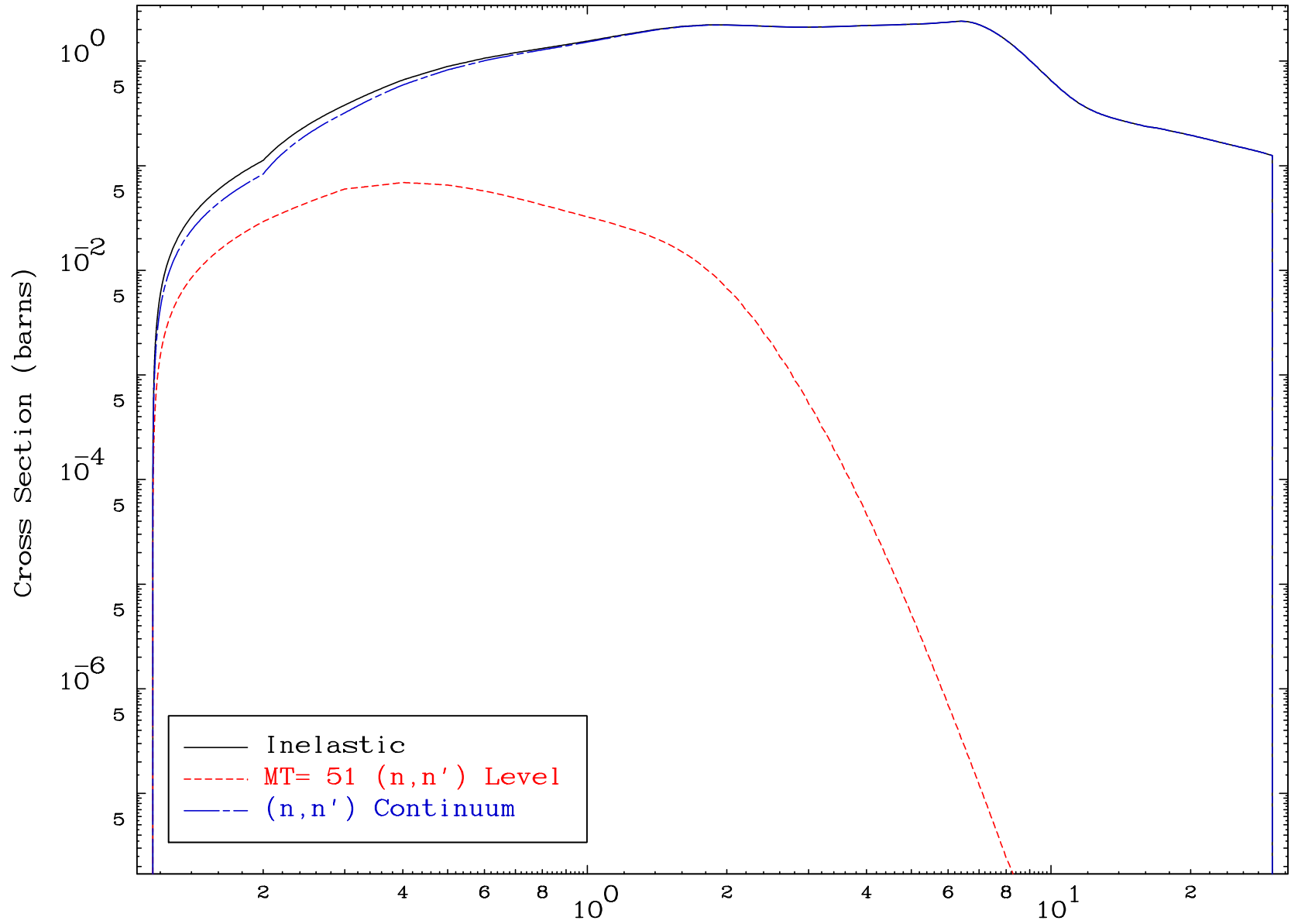


MAT 9116

(n,n') Level  
294 Kelvin Cross Sections

91-Pa-226

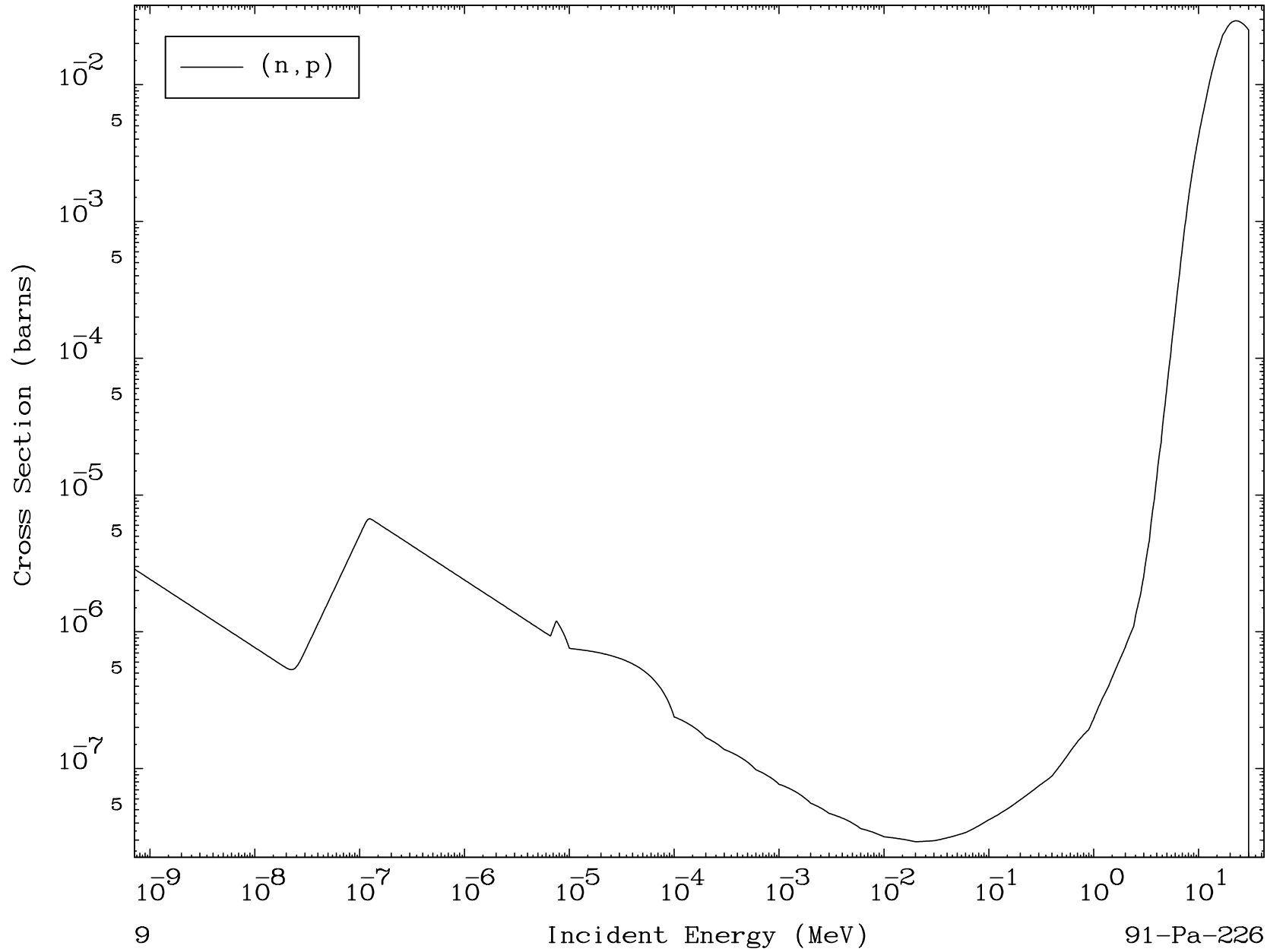


8

Incident Energy (MeV)

91-Pa-226

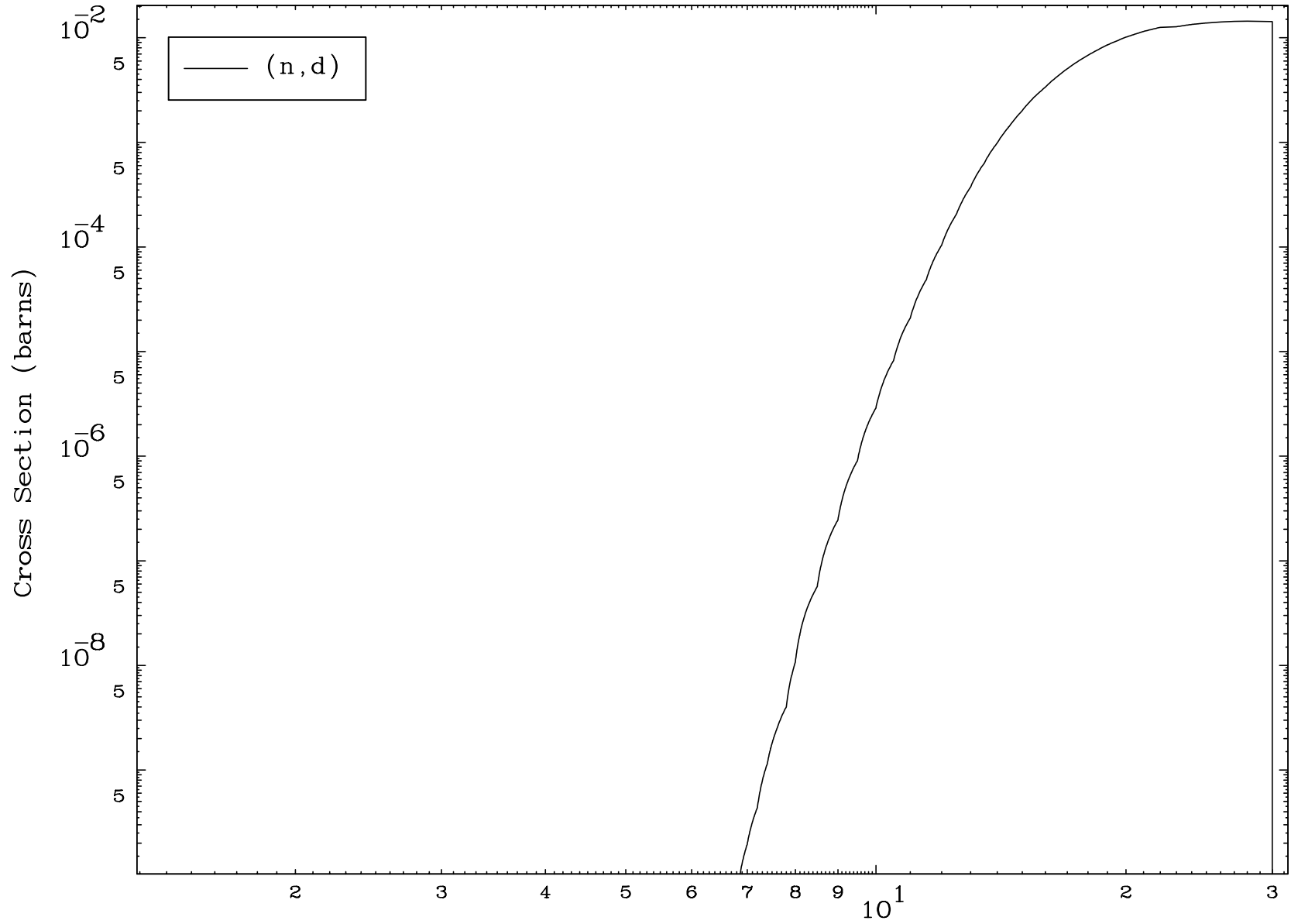




MAT 9116

(n,d) Levels  
294 Kelvin Cross Sections

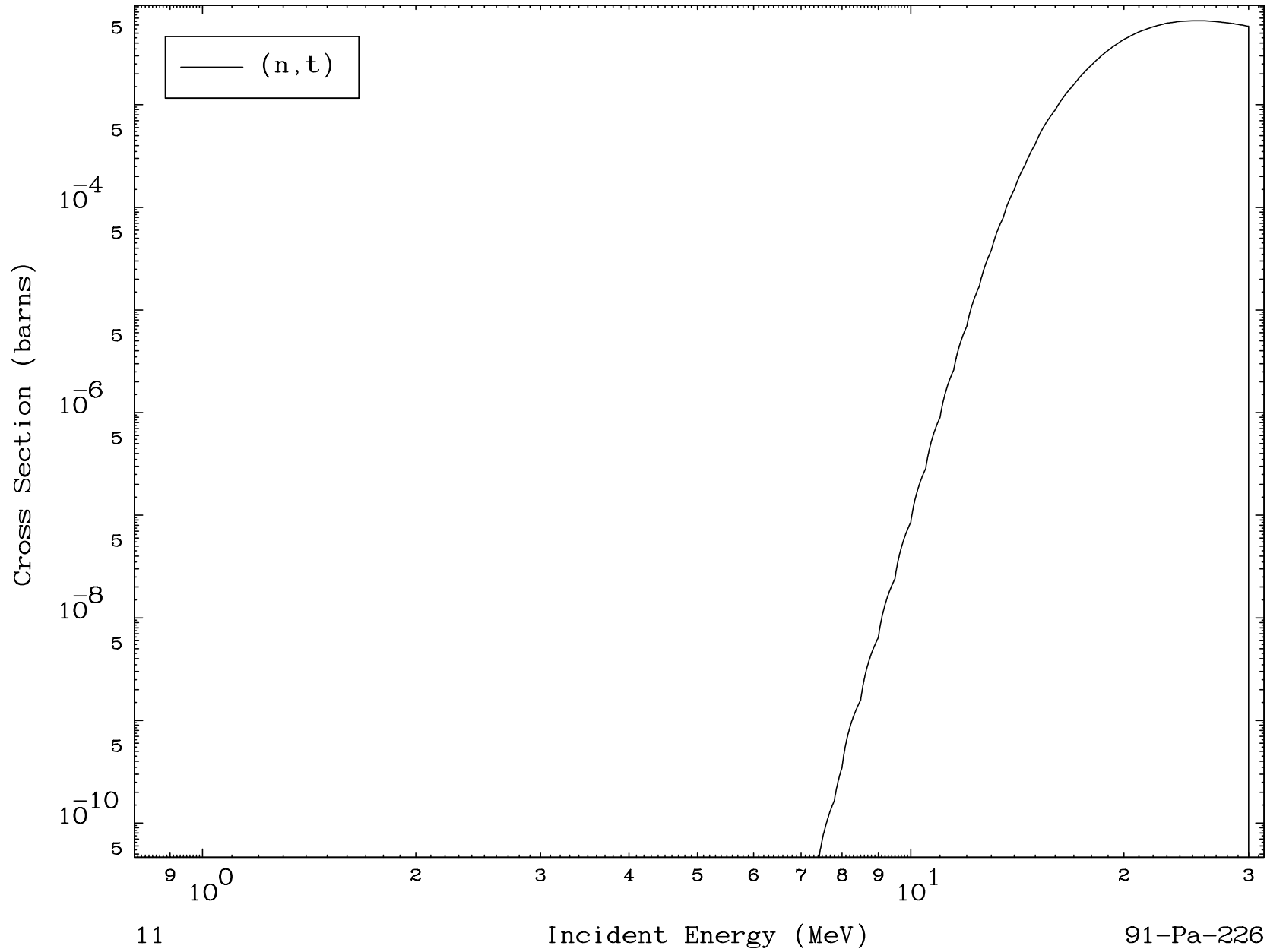
91-Pa-226

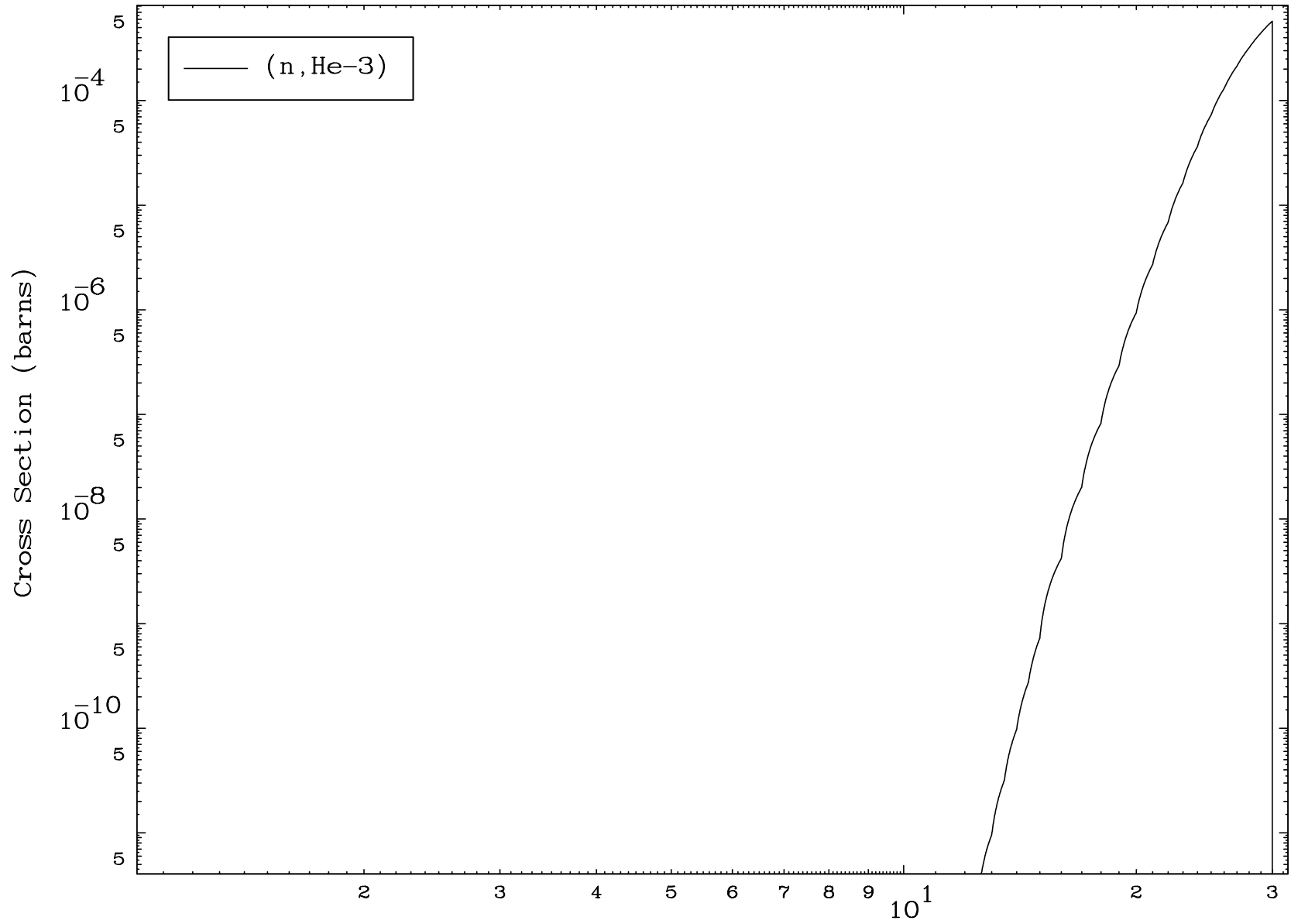


10

Incident Energy (MeV)

91-Pa-226

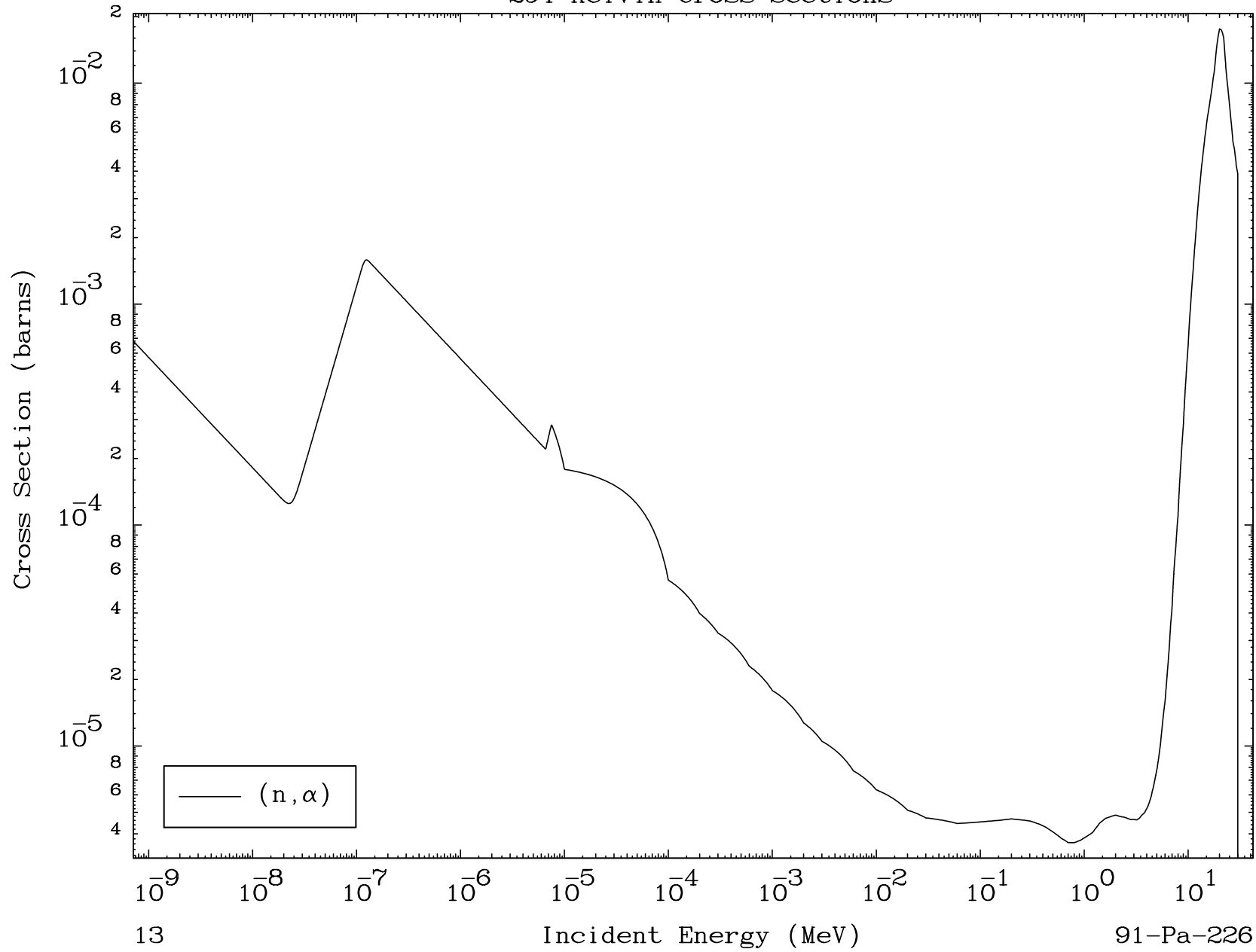




MAT 9116

(n,  $\alpha$ ) Levels  
294 Kelvin Cross Sections

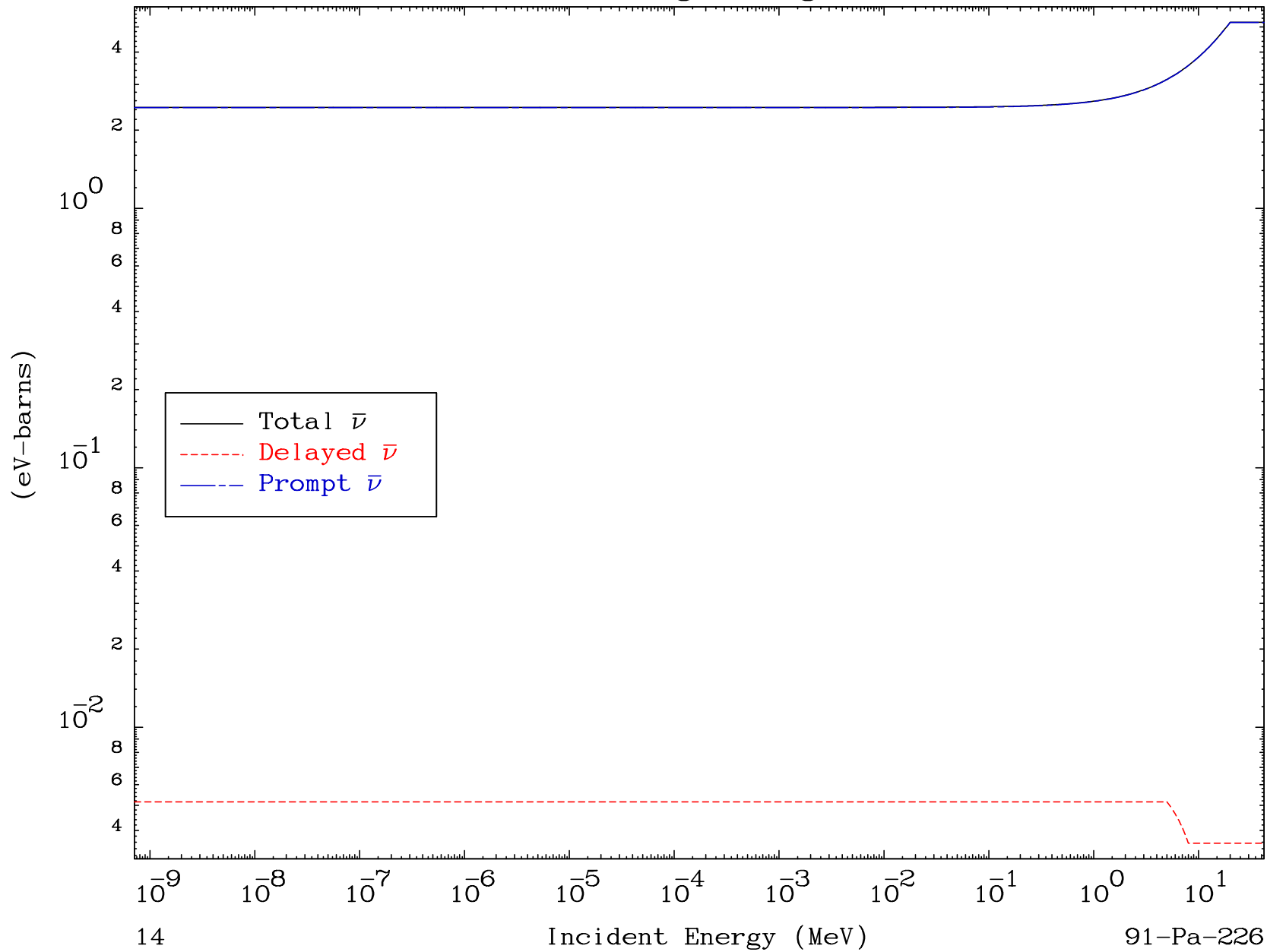
91-Pa-226

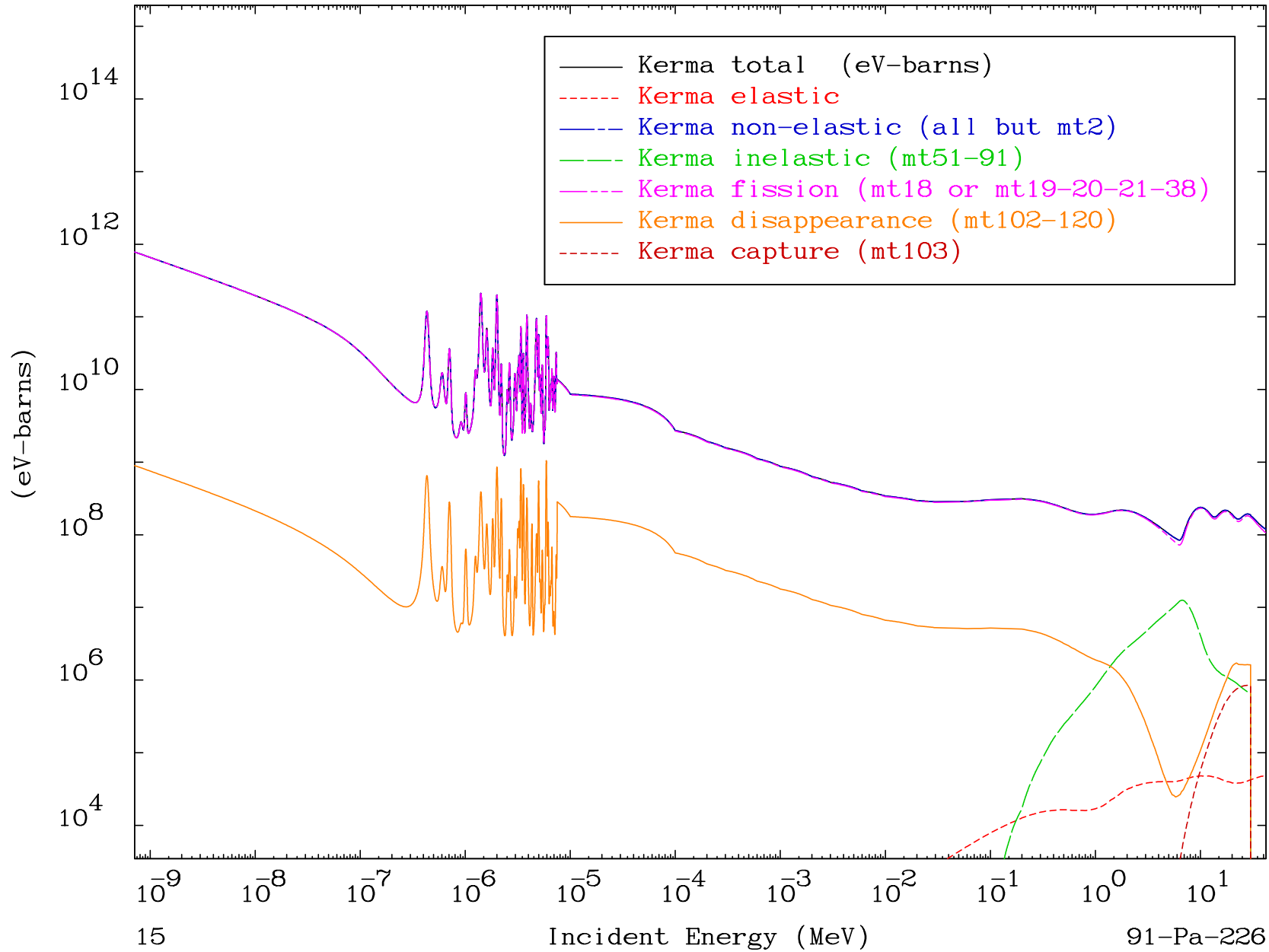


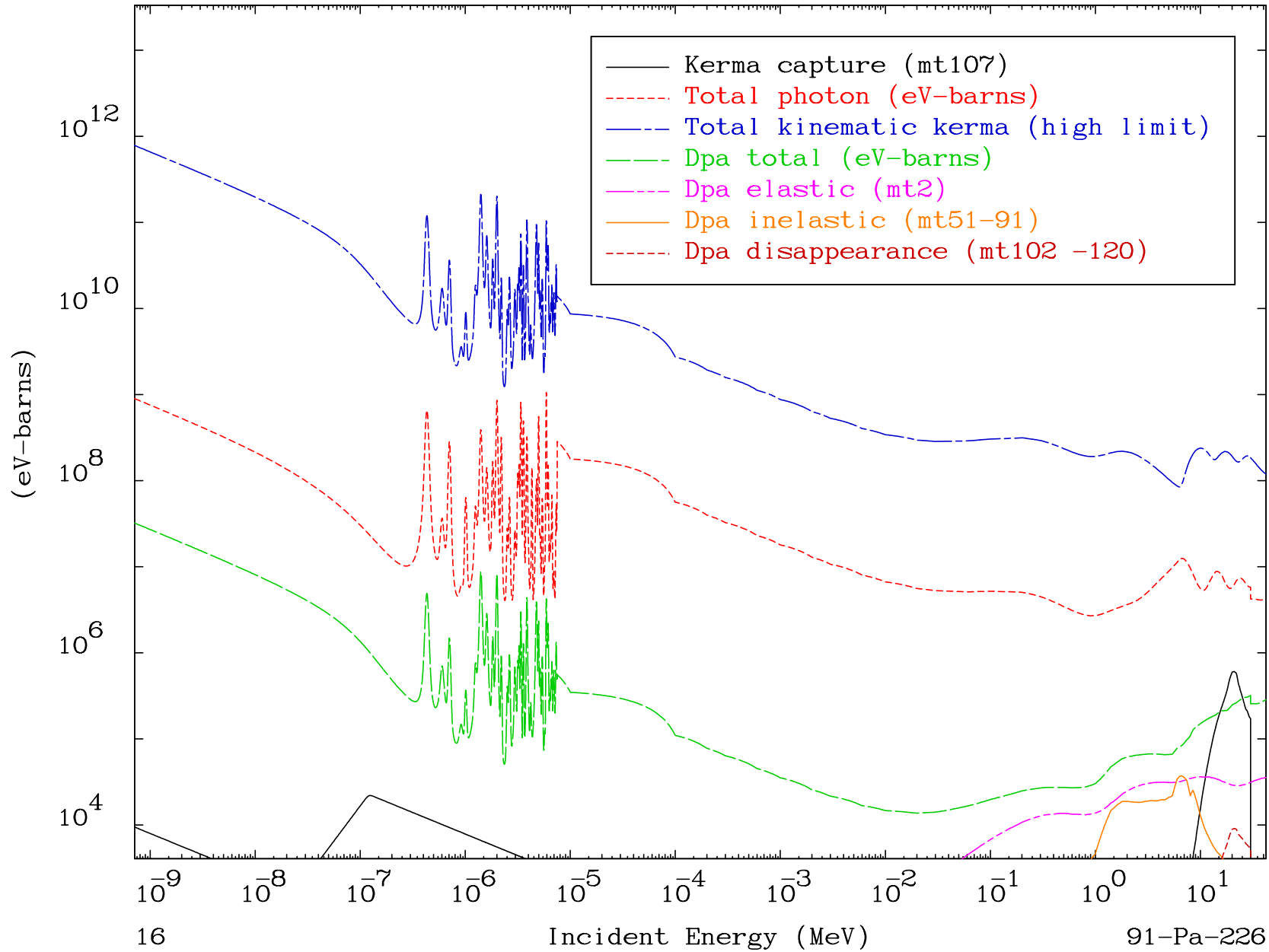
13

Incident Energy (MeV)

91-Pa-226

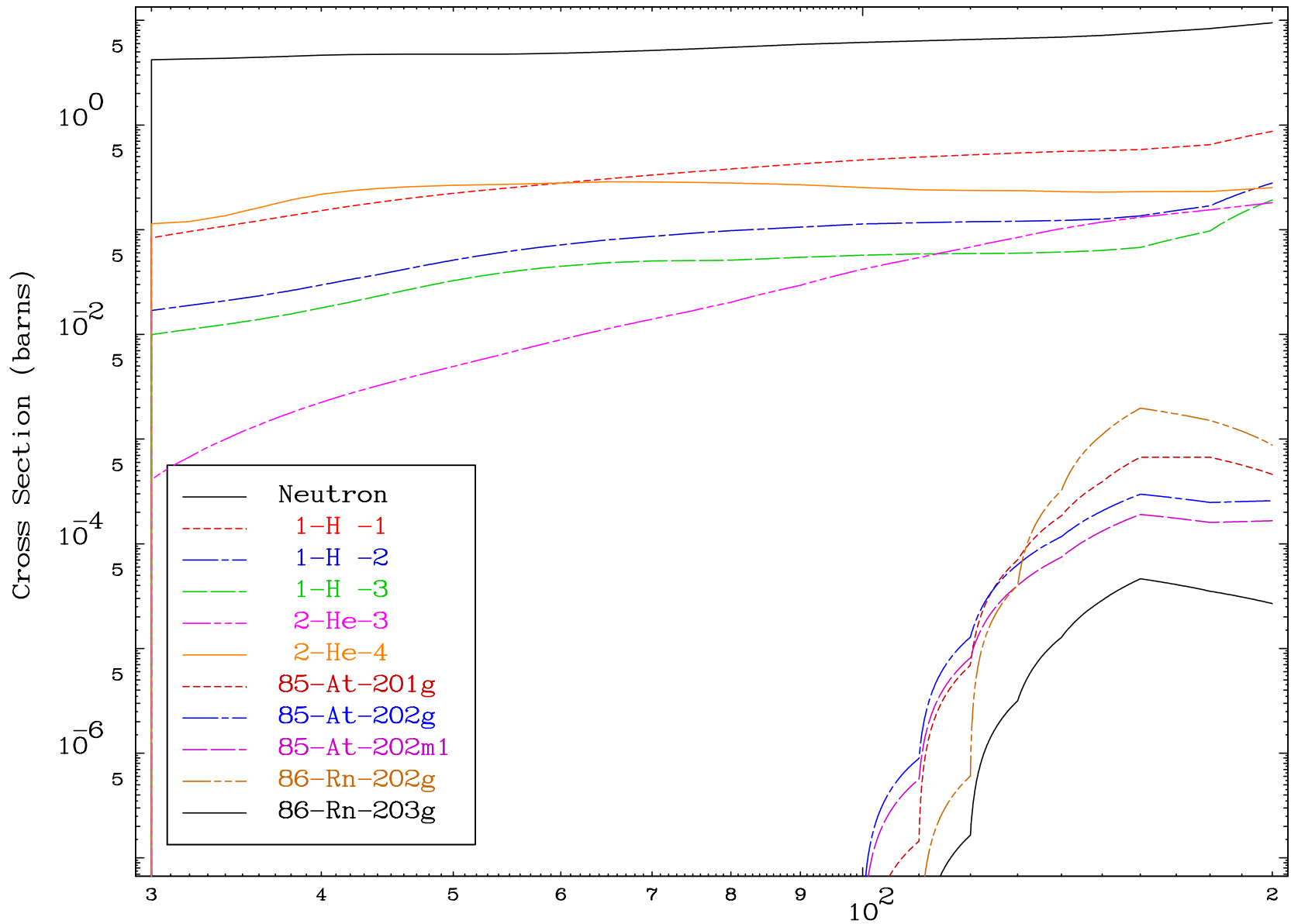


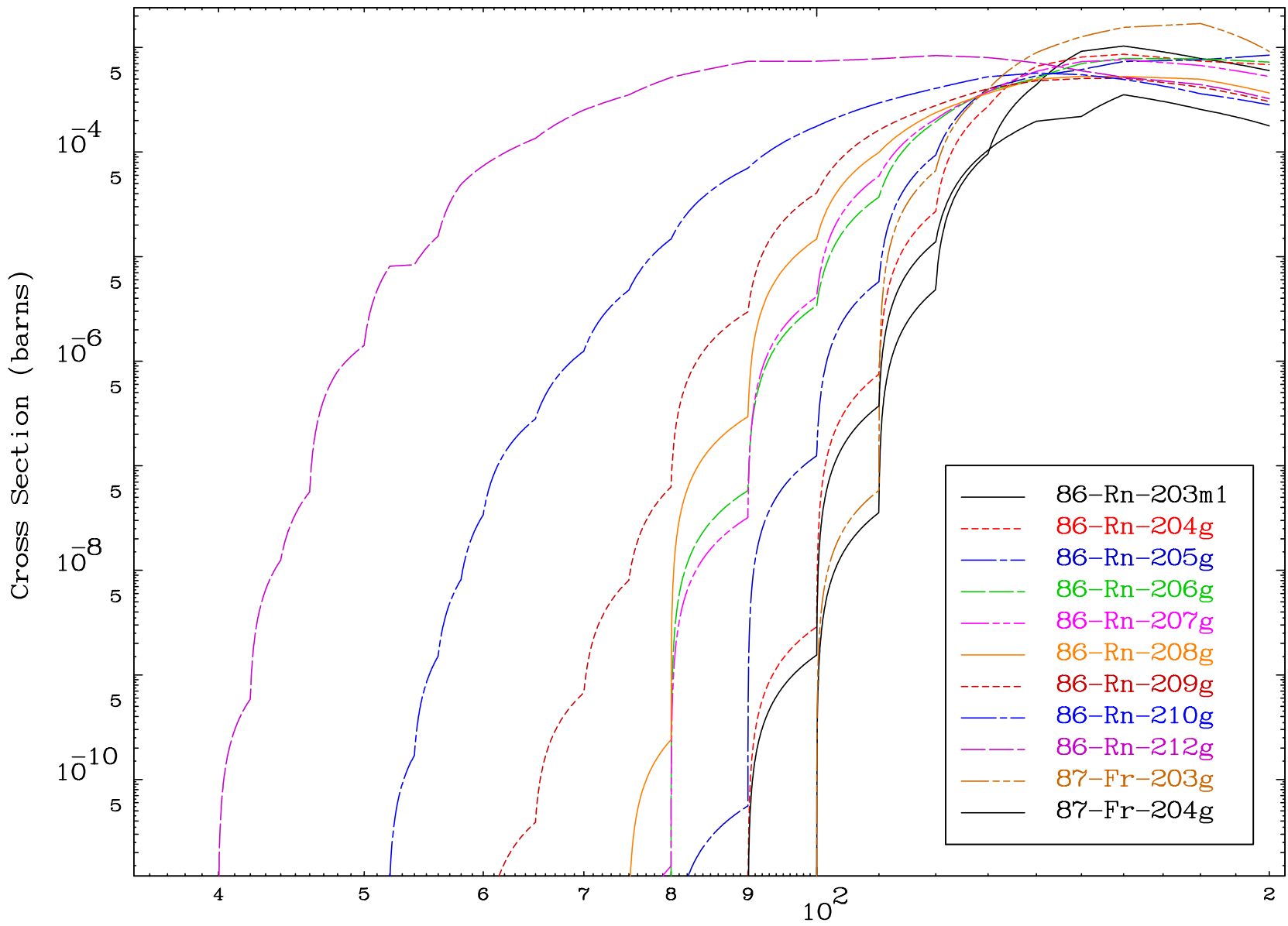


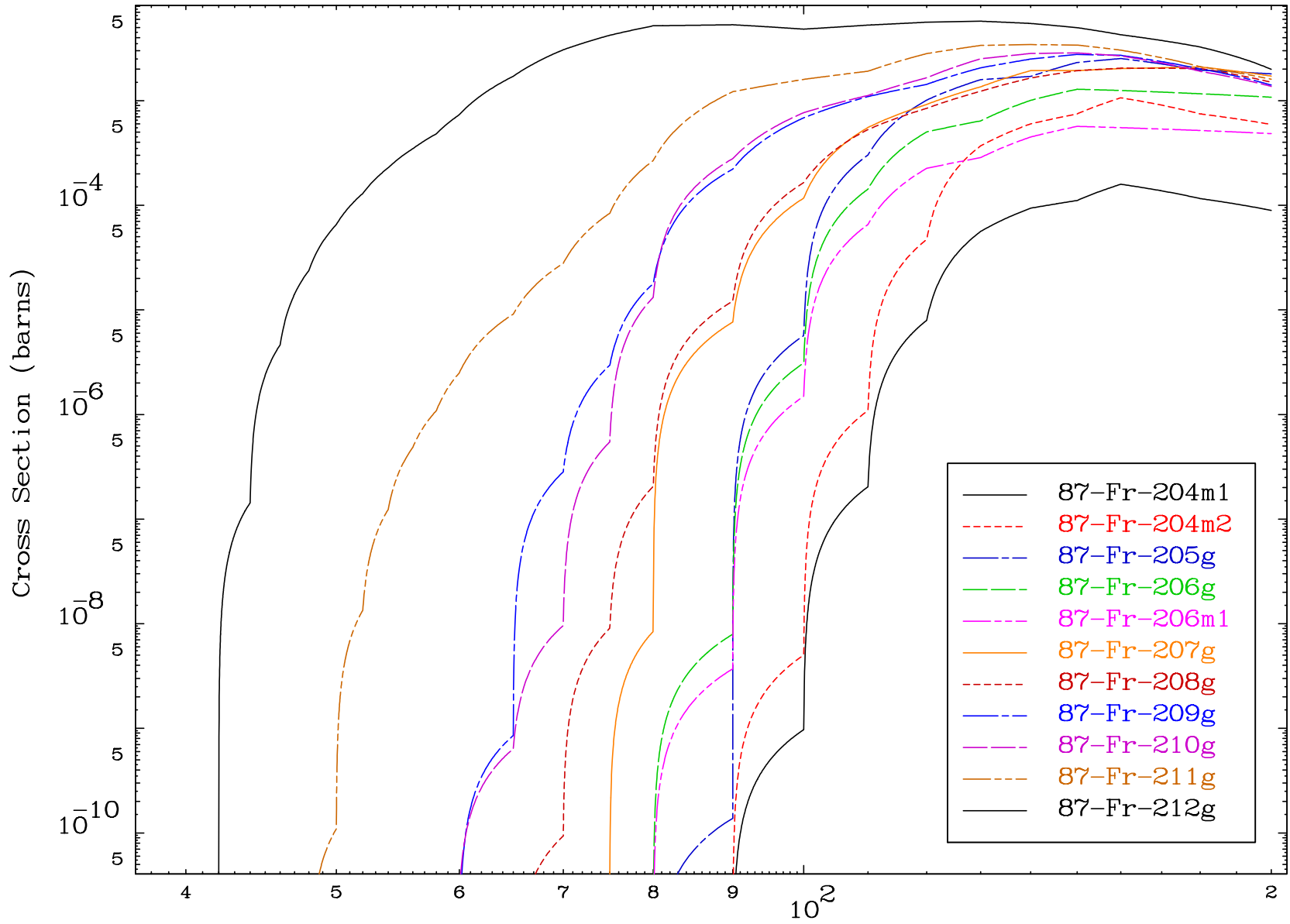


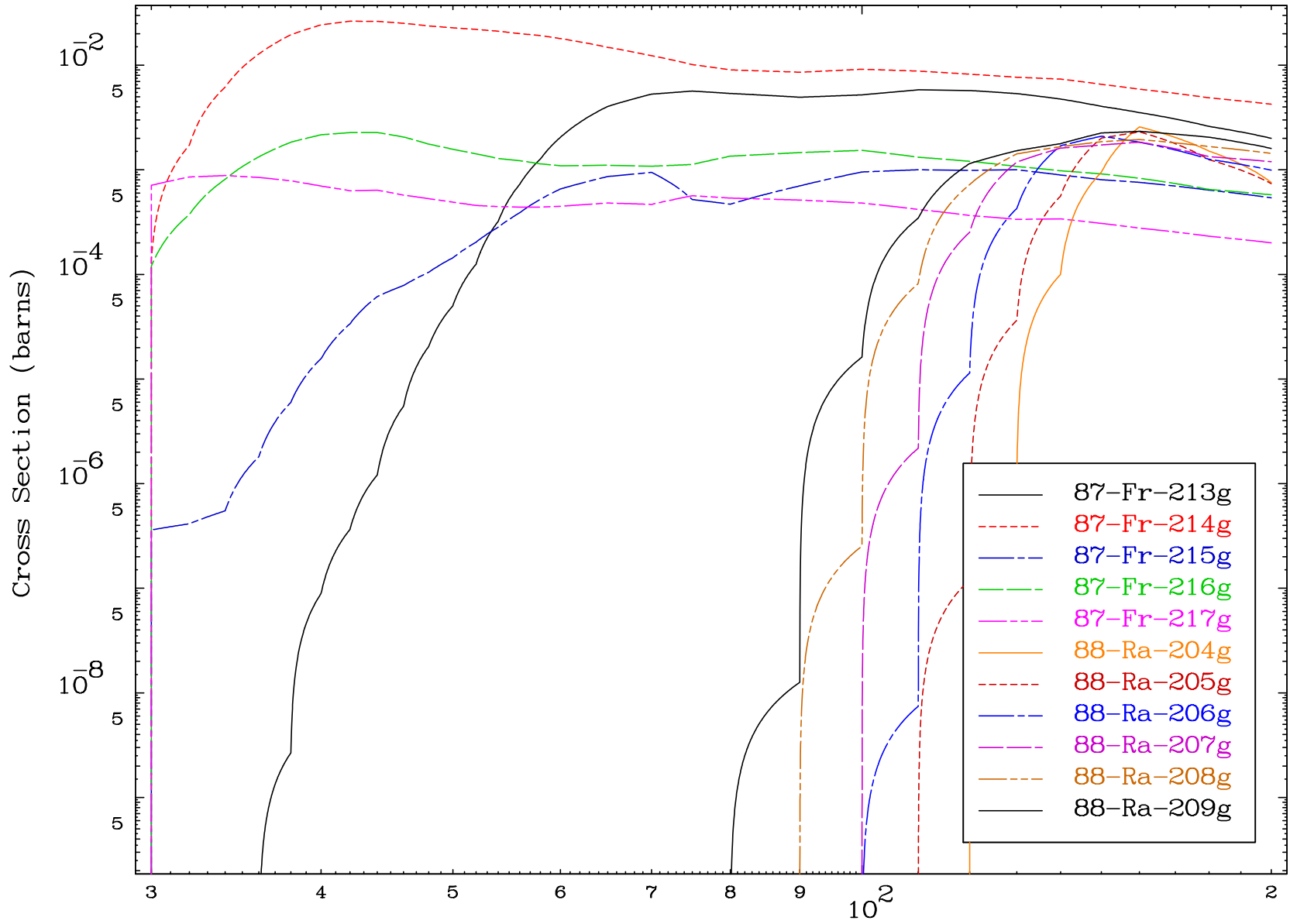


Radionuclide Production Cross Section

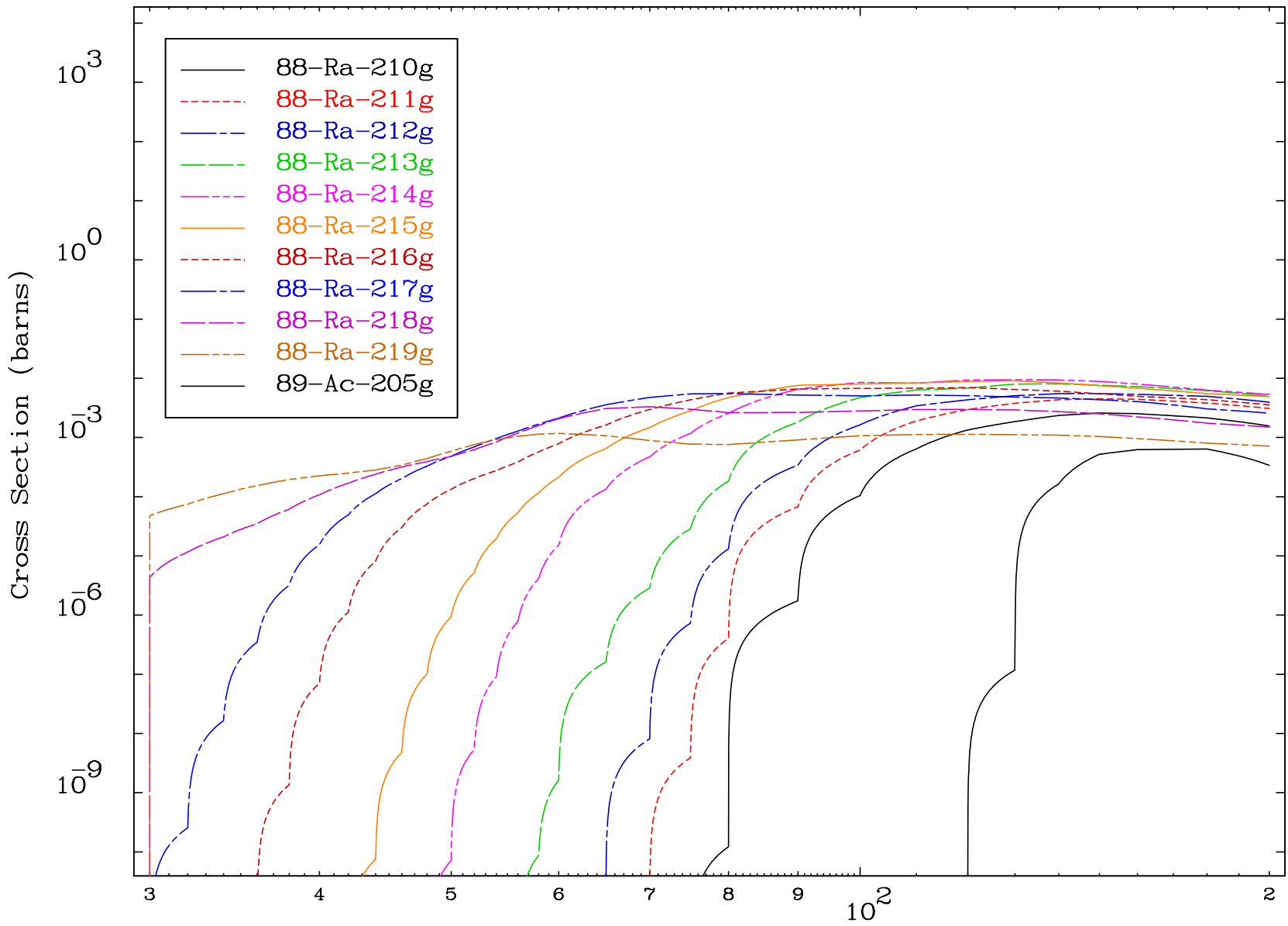




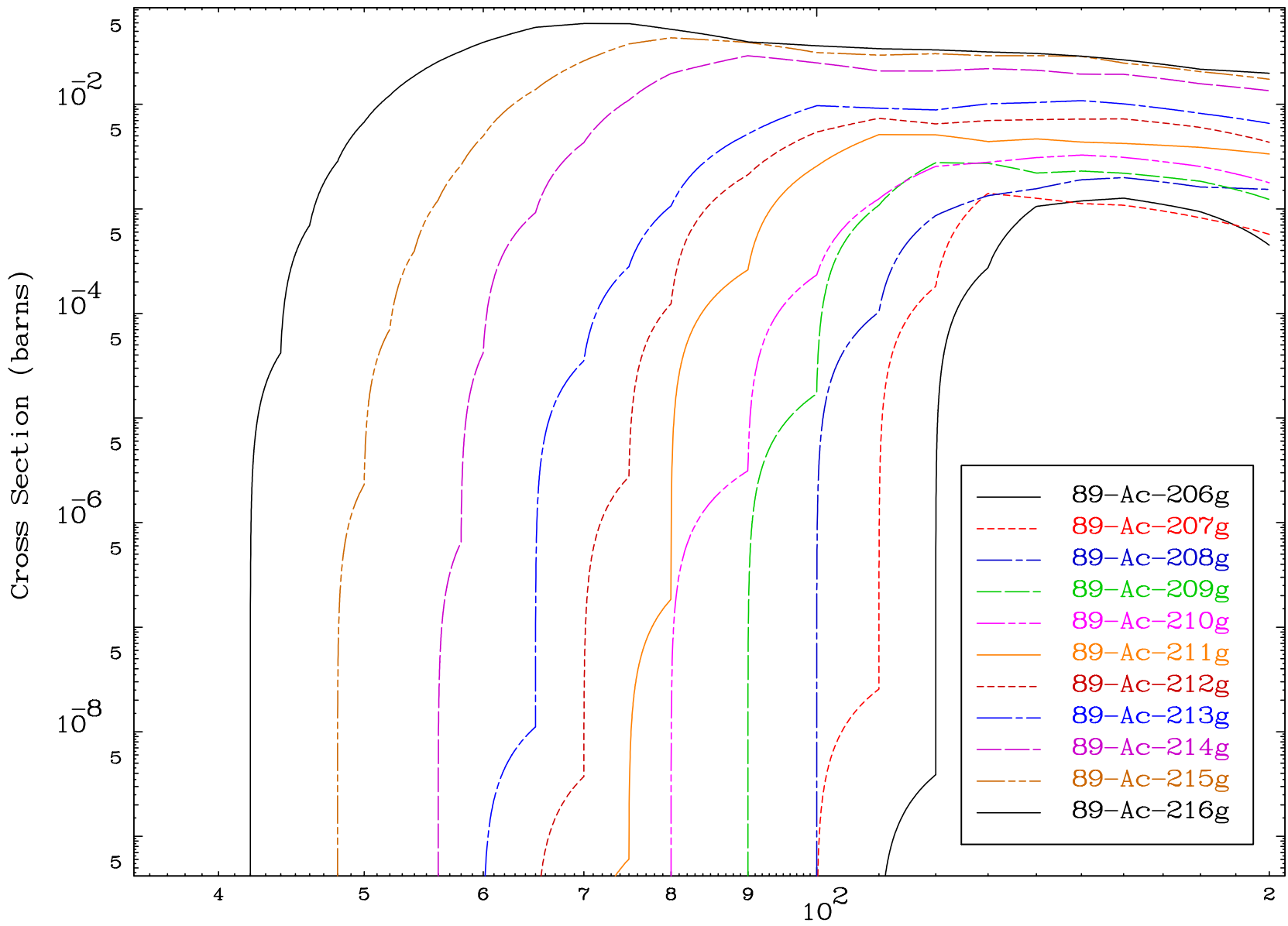




Radionuclide Production Cross Section



Radionuclide Production Cross Section

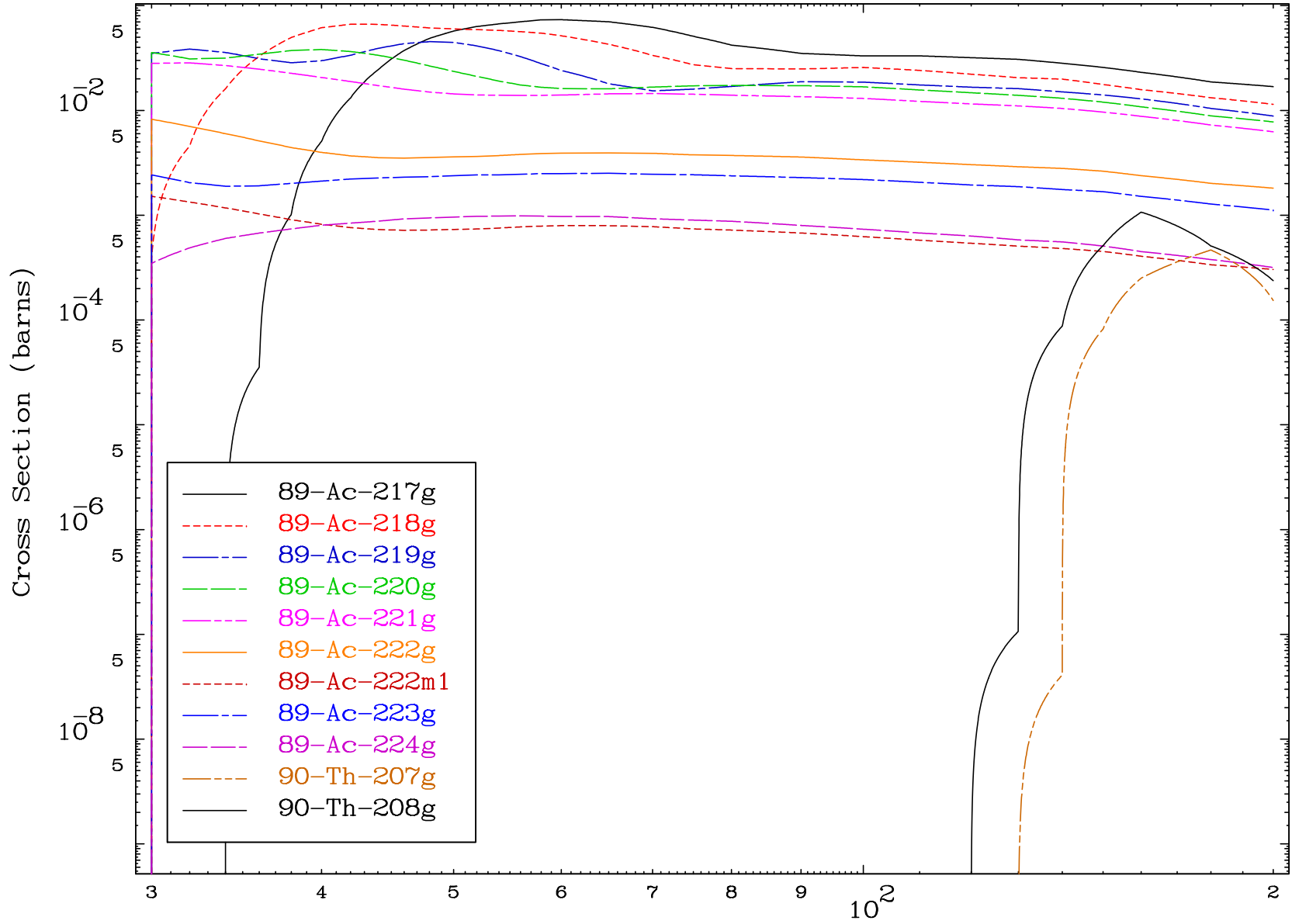


MAT 9116

(n,remainder)

91-Pa-226

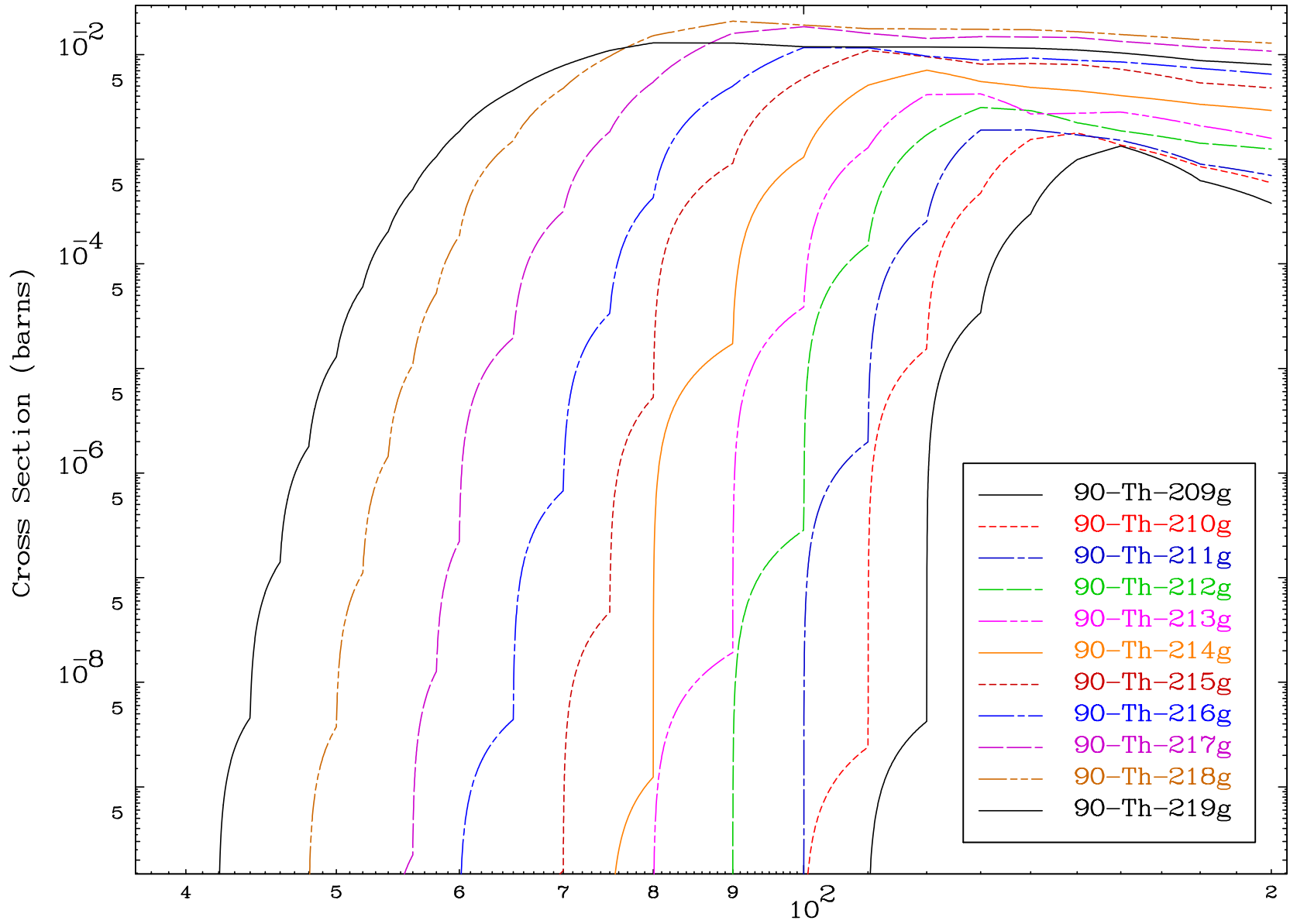
### Radionuclide Production Cross Section



23

Incident Energy (MeV)

91-Pa-226



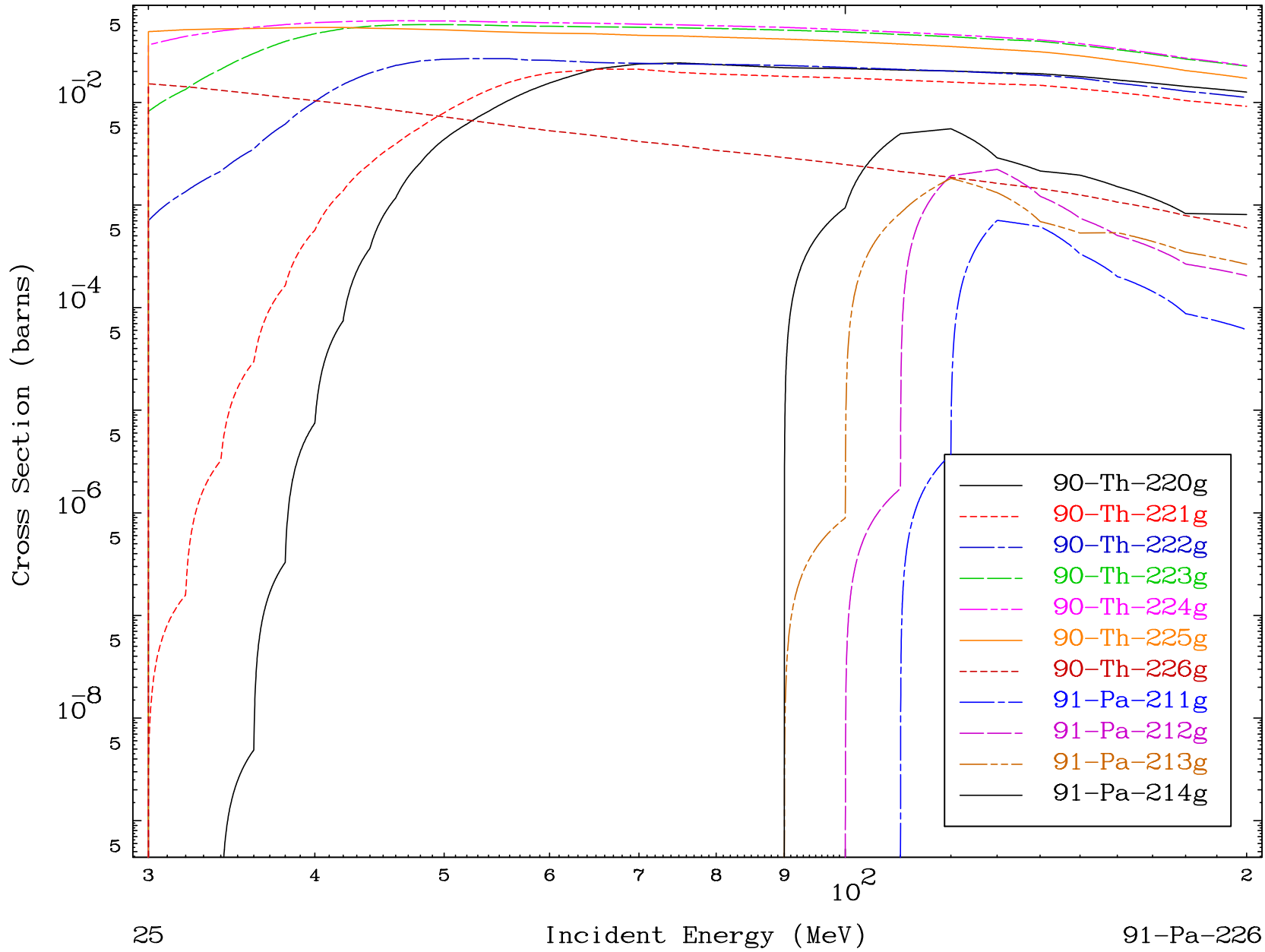


MAT 9116

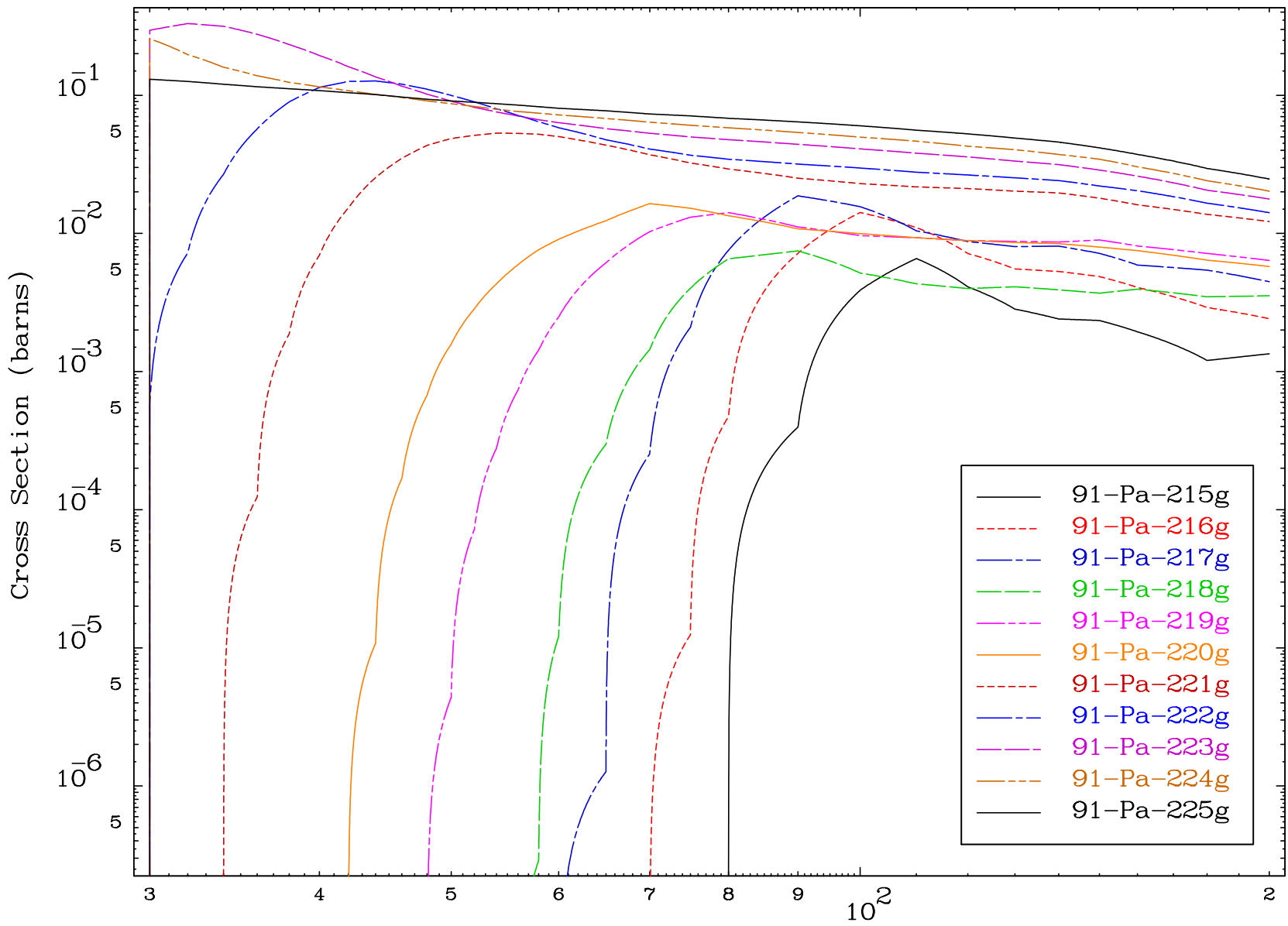
(n,remainder)

91-Pa-226

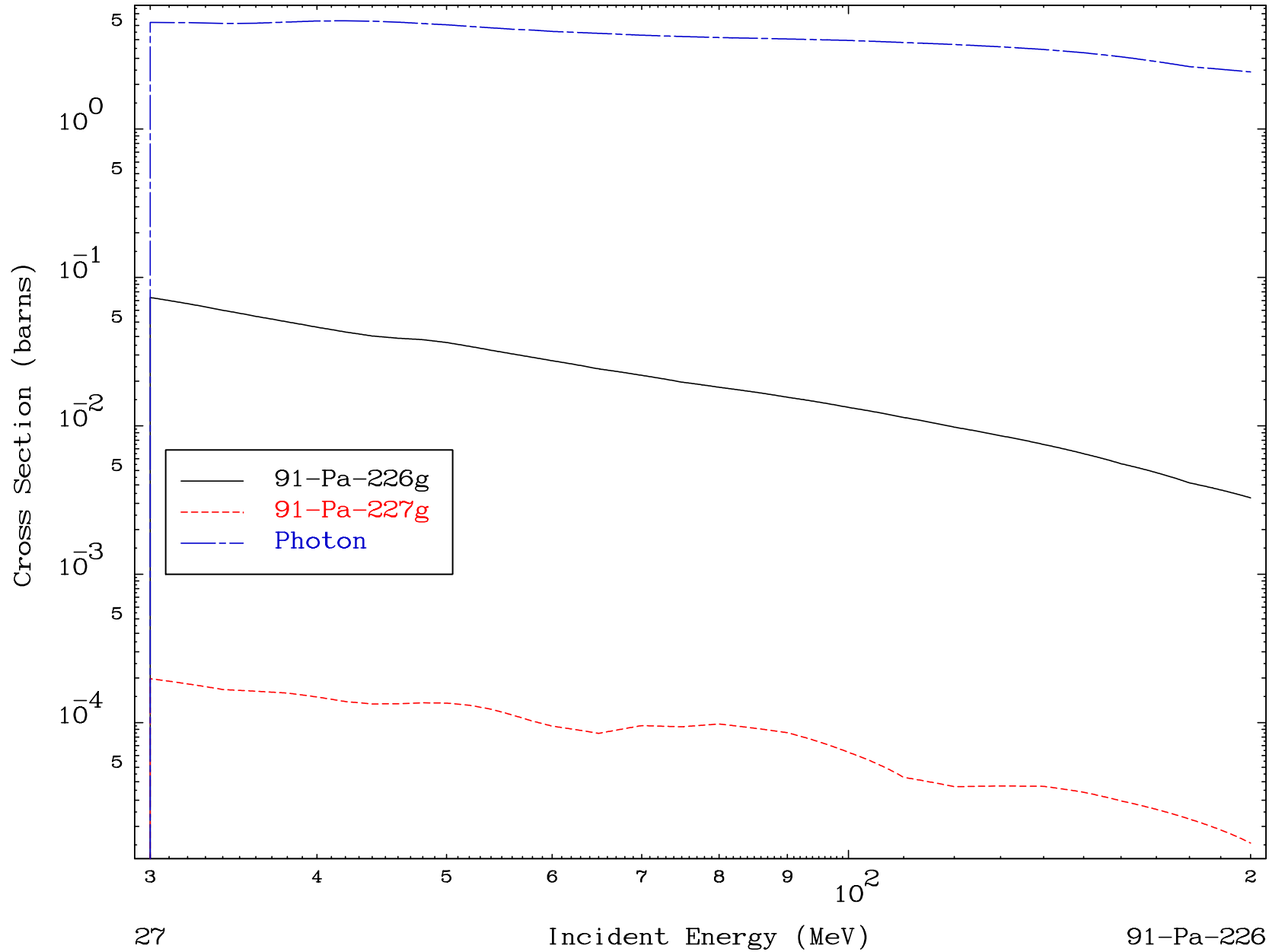
### Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section



MAT 9116

(n,n')  $\alpha$

91-Pa-226

Radionuclide Production Cross Section

