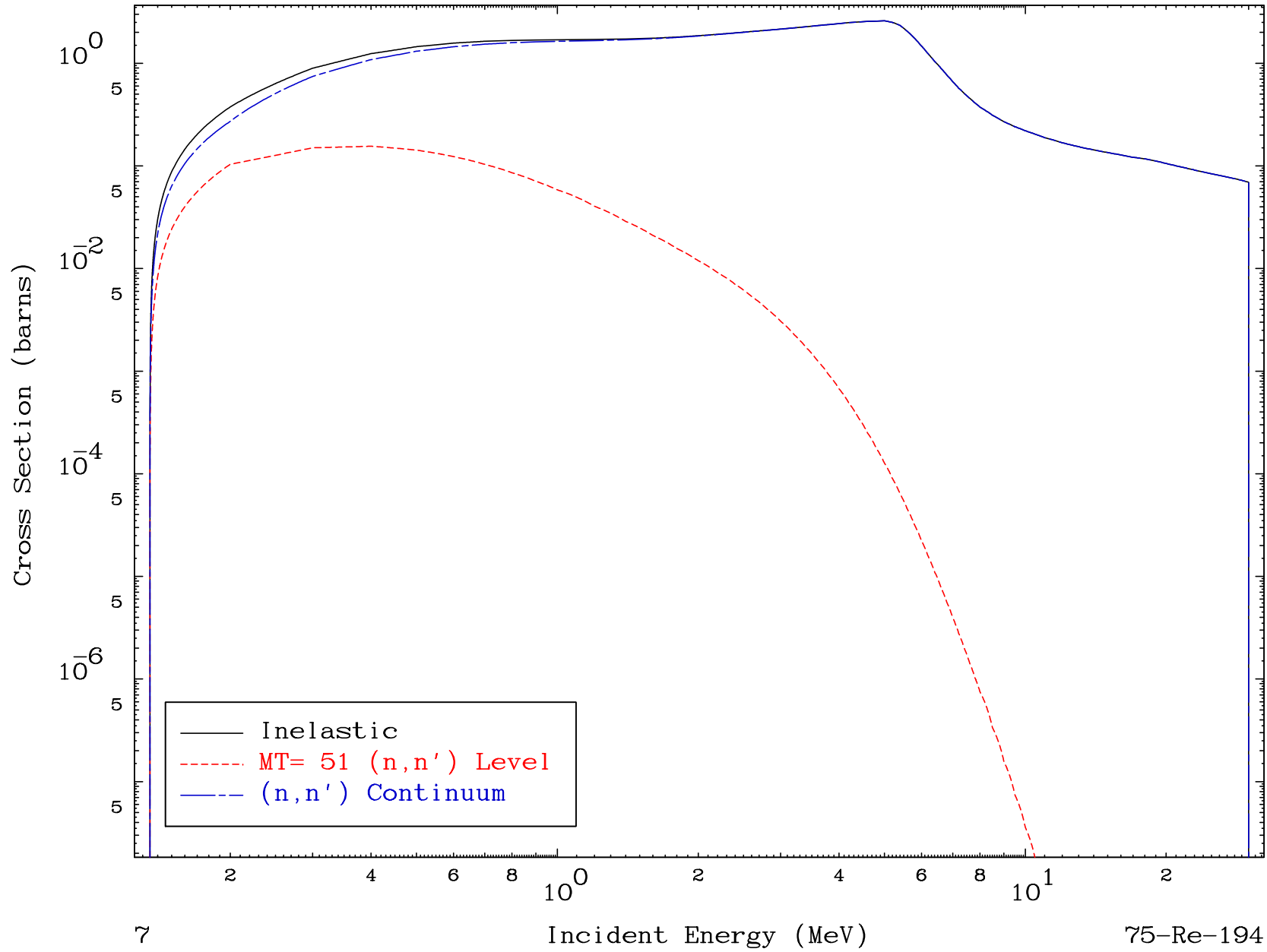


MAT 7552

(n,n') Level  
294 Kelvin Cross Sections

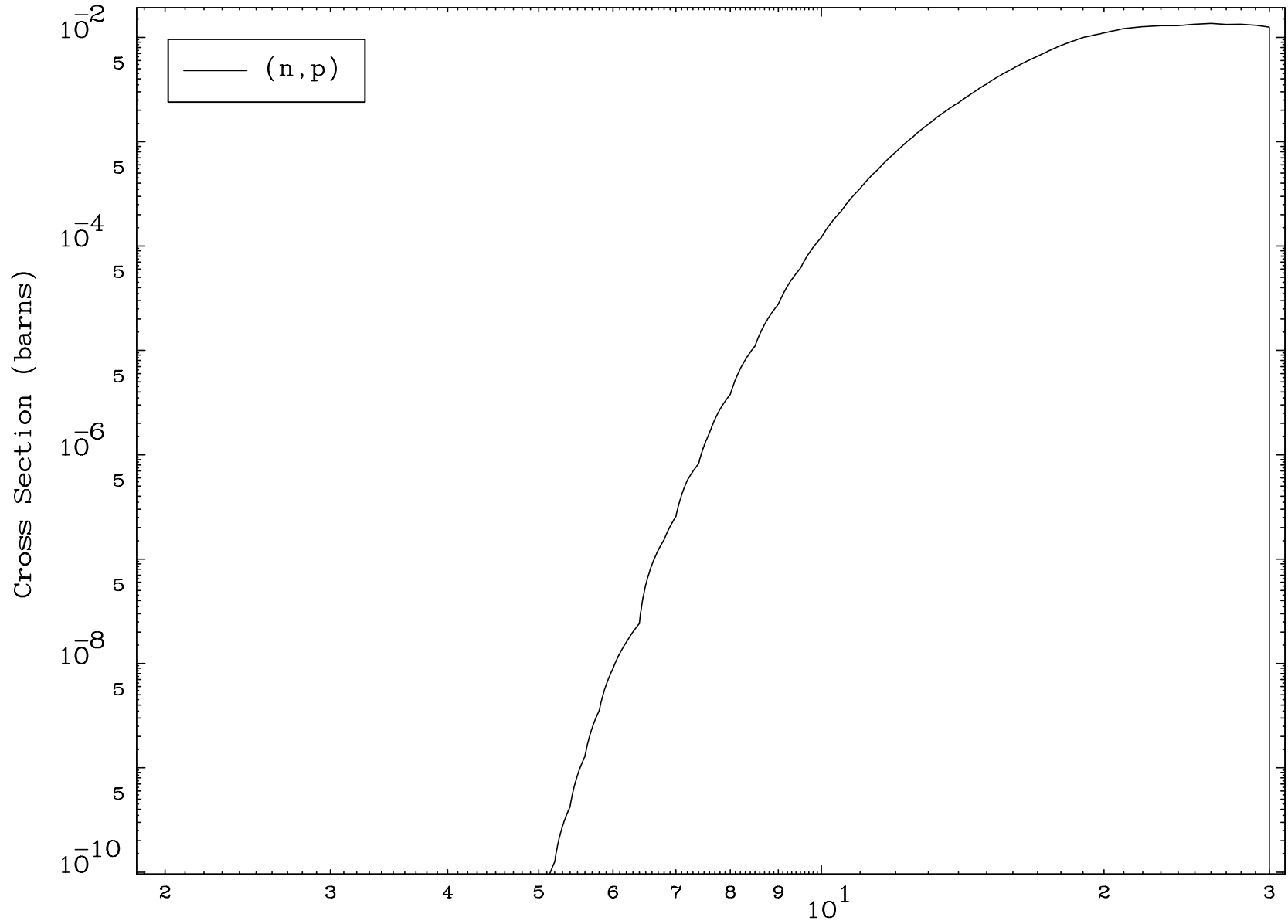
75-Re-194



MAT 7552

(n,p) Levels  
294 Kelvin Cross Sections

75-Re-194

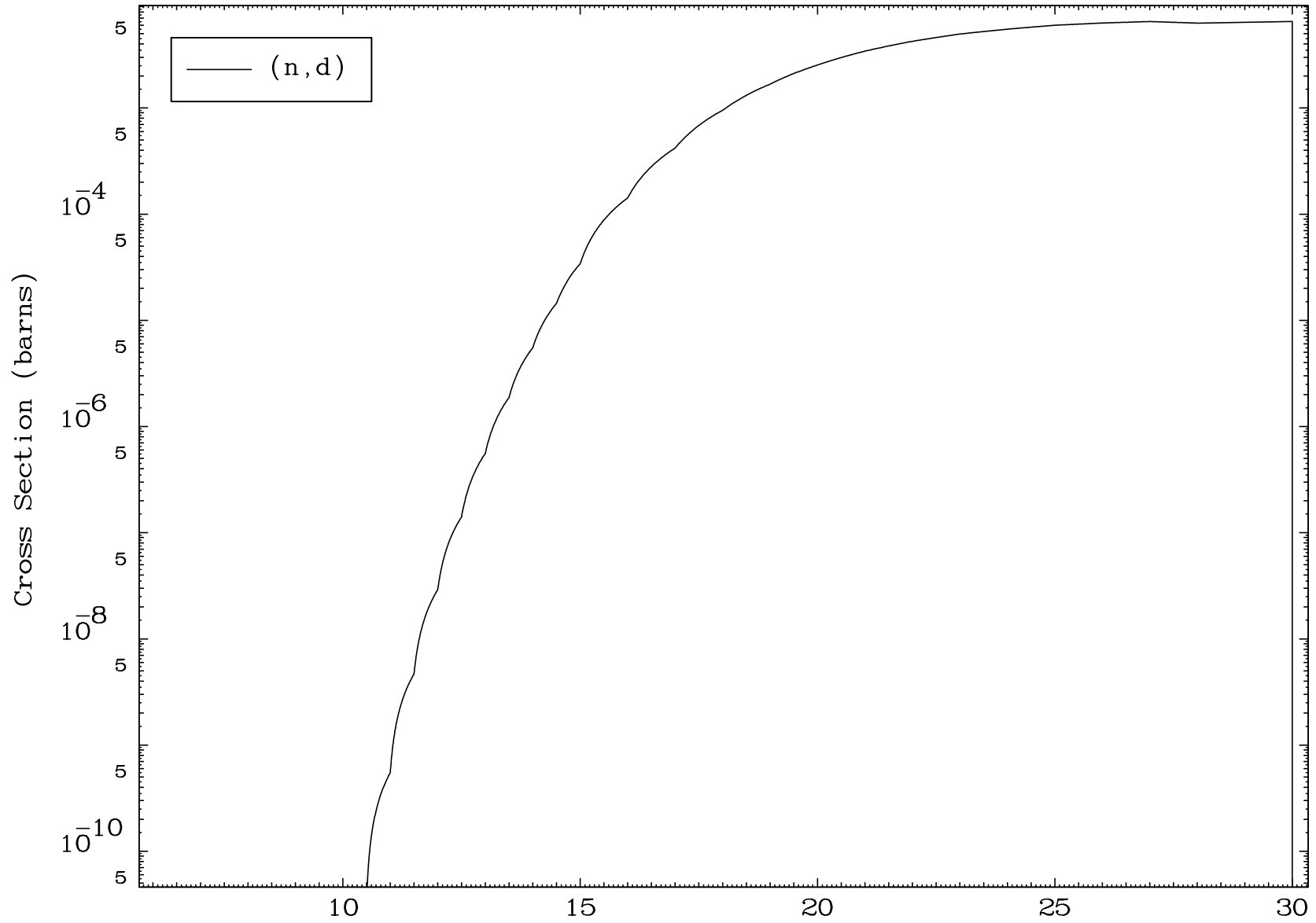


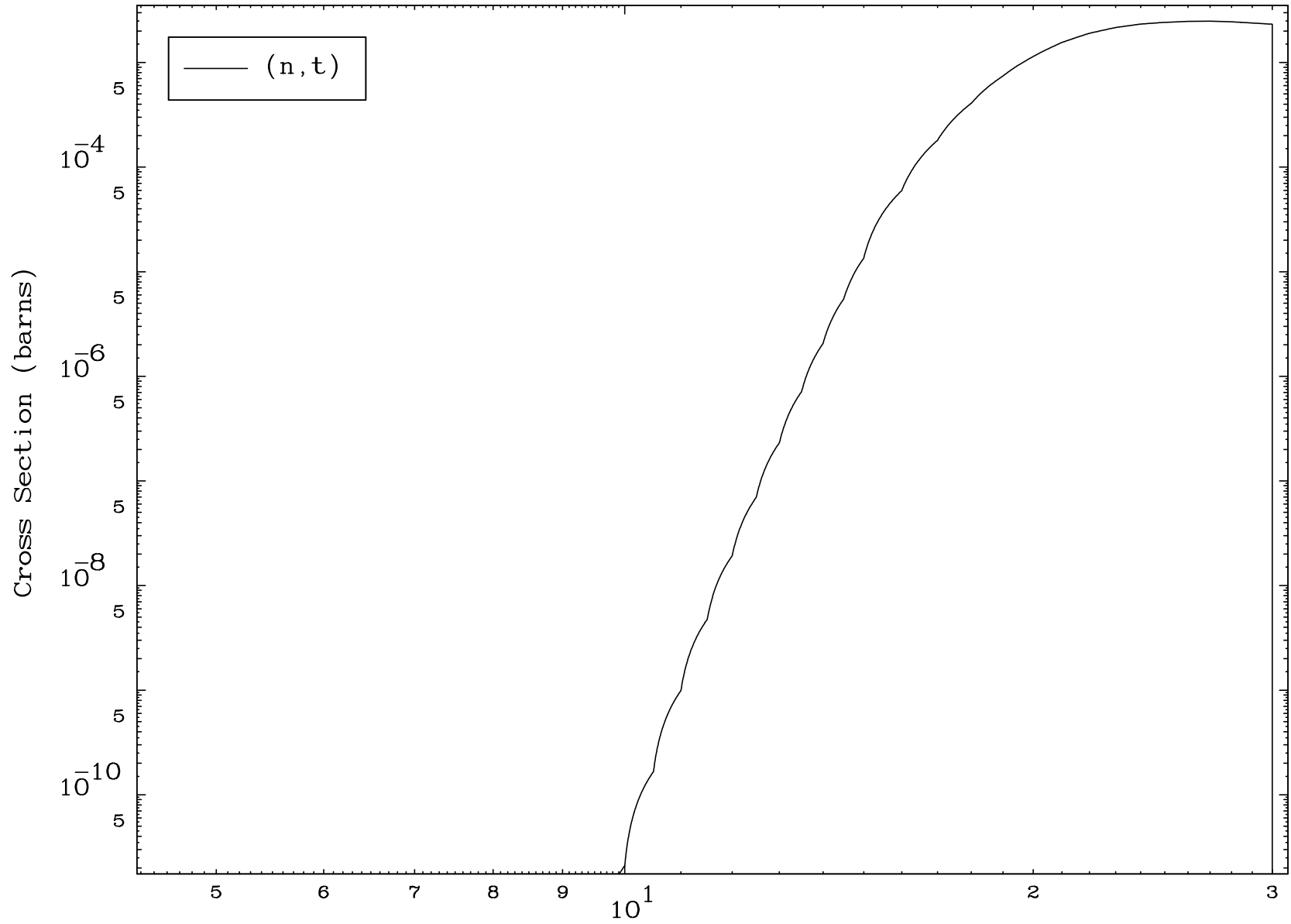
8

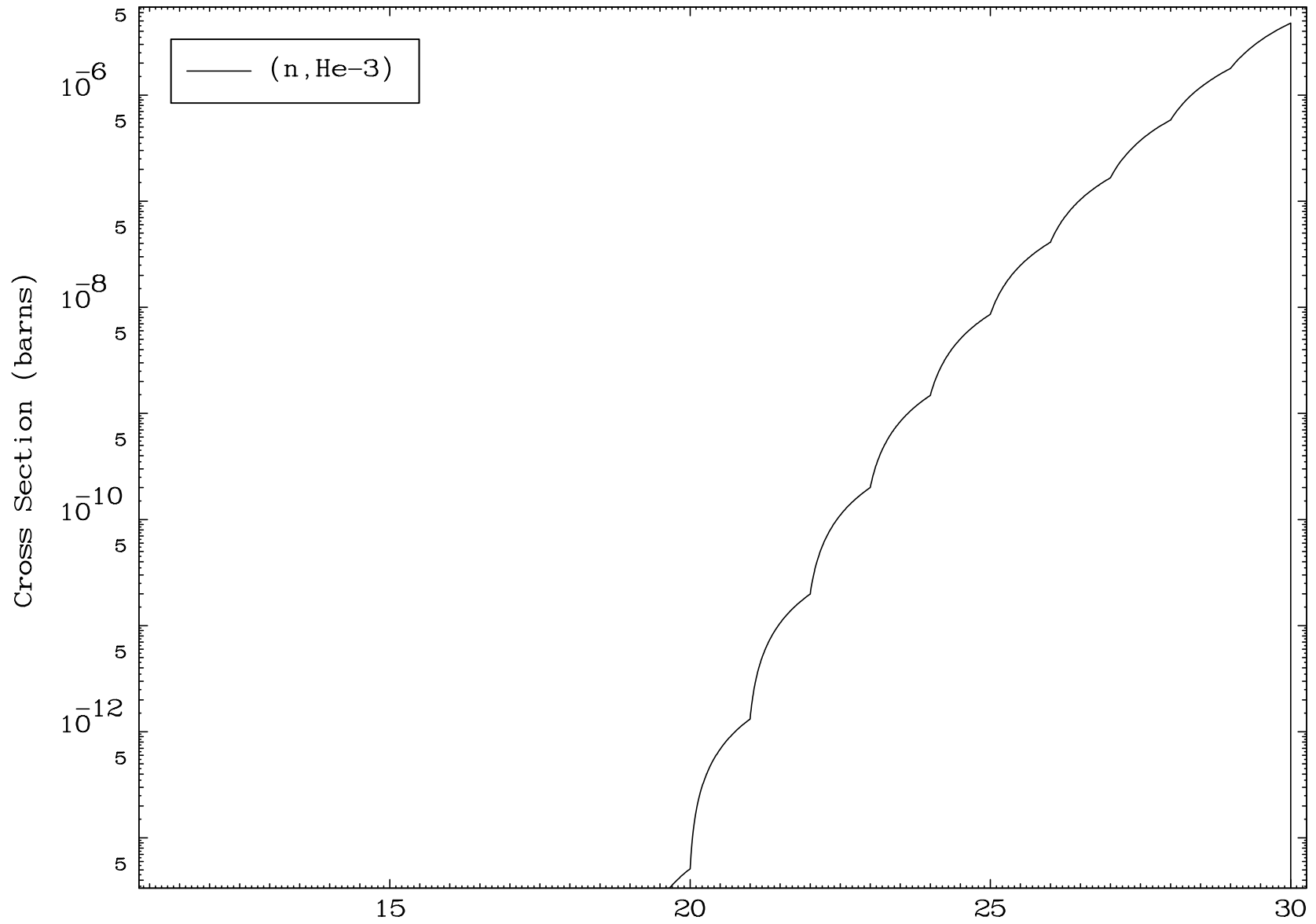
Incident Energy (MeV)

75-Re-194





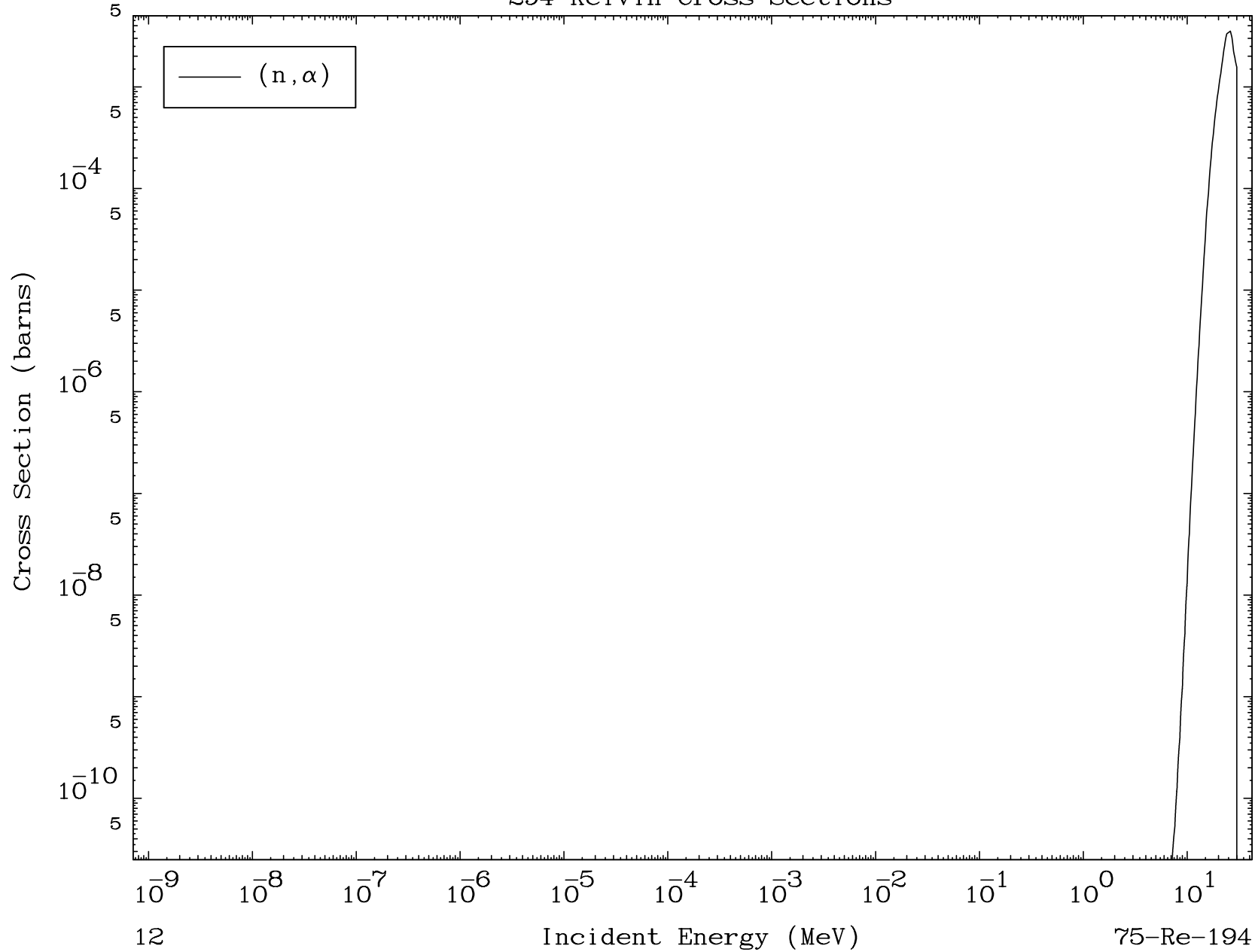




MAT 7552

(n,  $\alpha$ ) Levels  
294 Kelvin Cross Sections

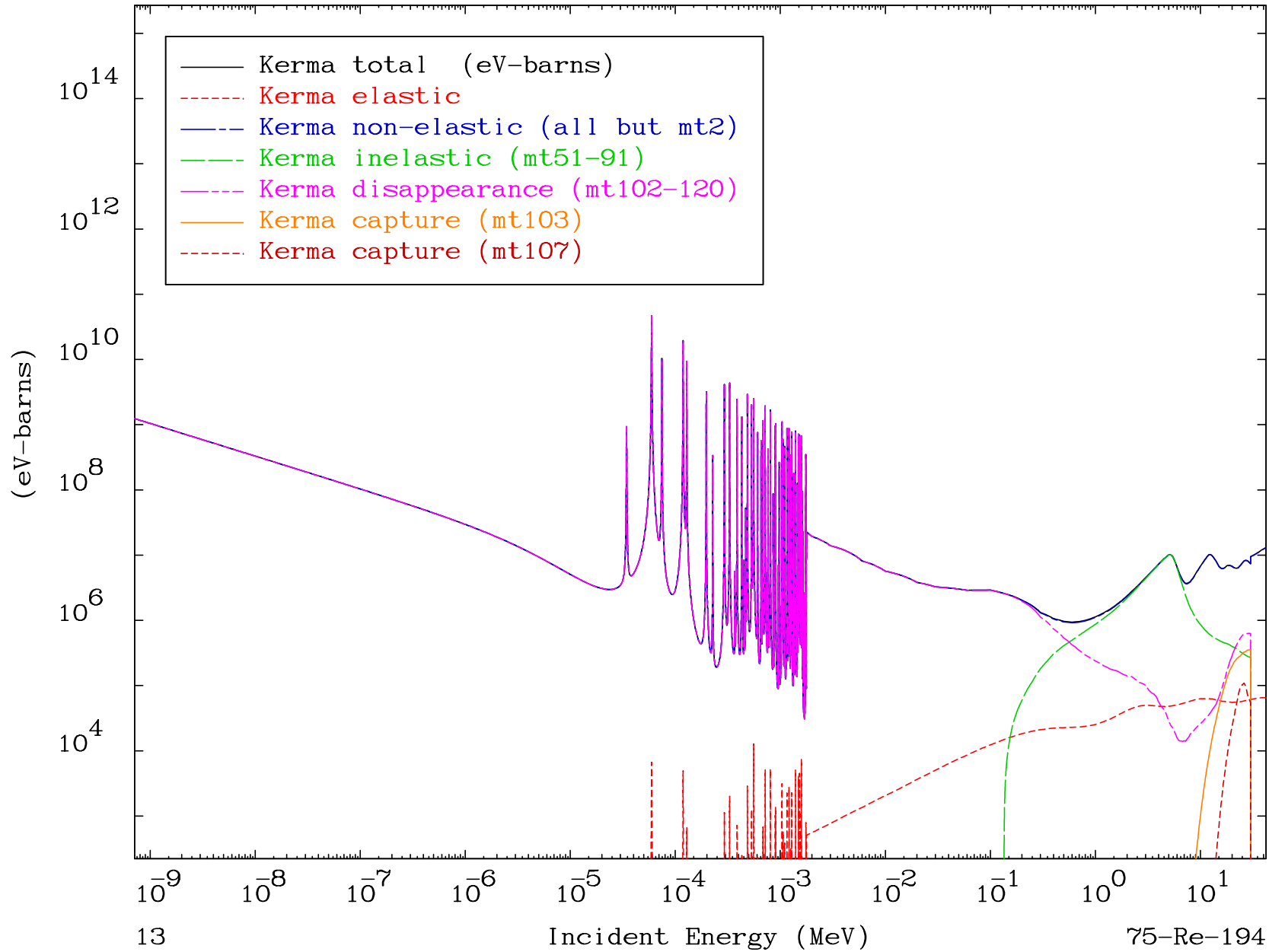
75-Re-194

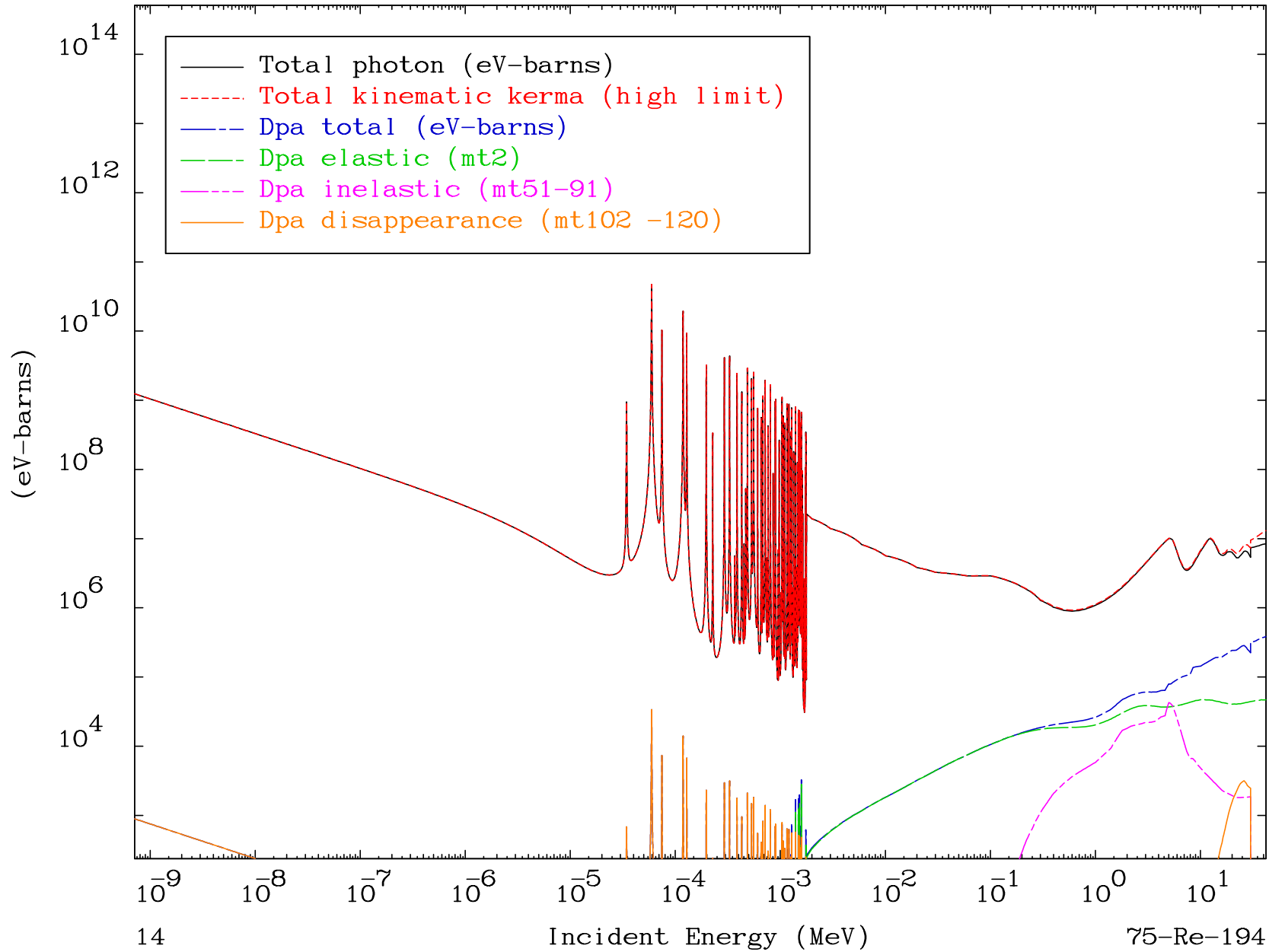


12

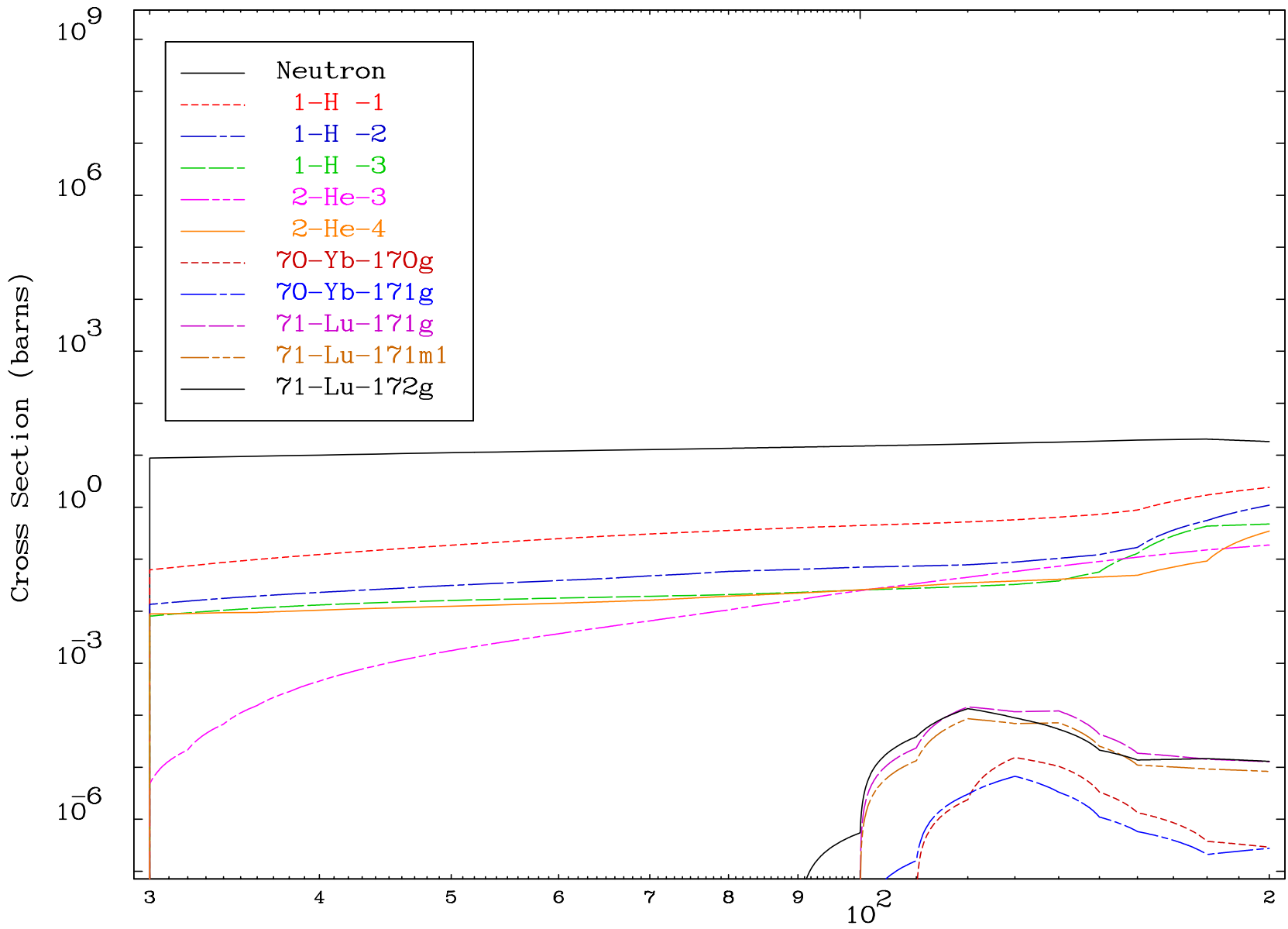
Incident Energy (MeV)

75-Re-194

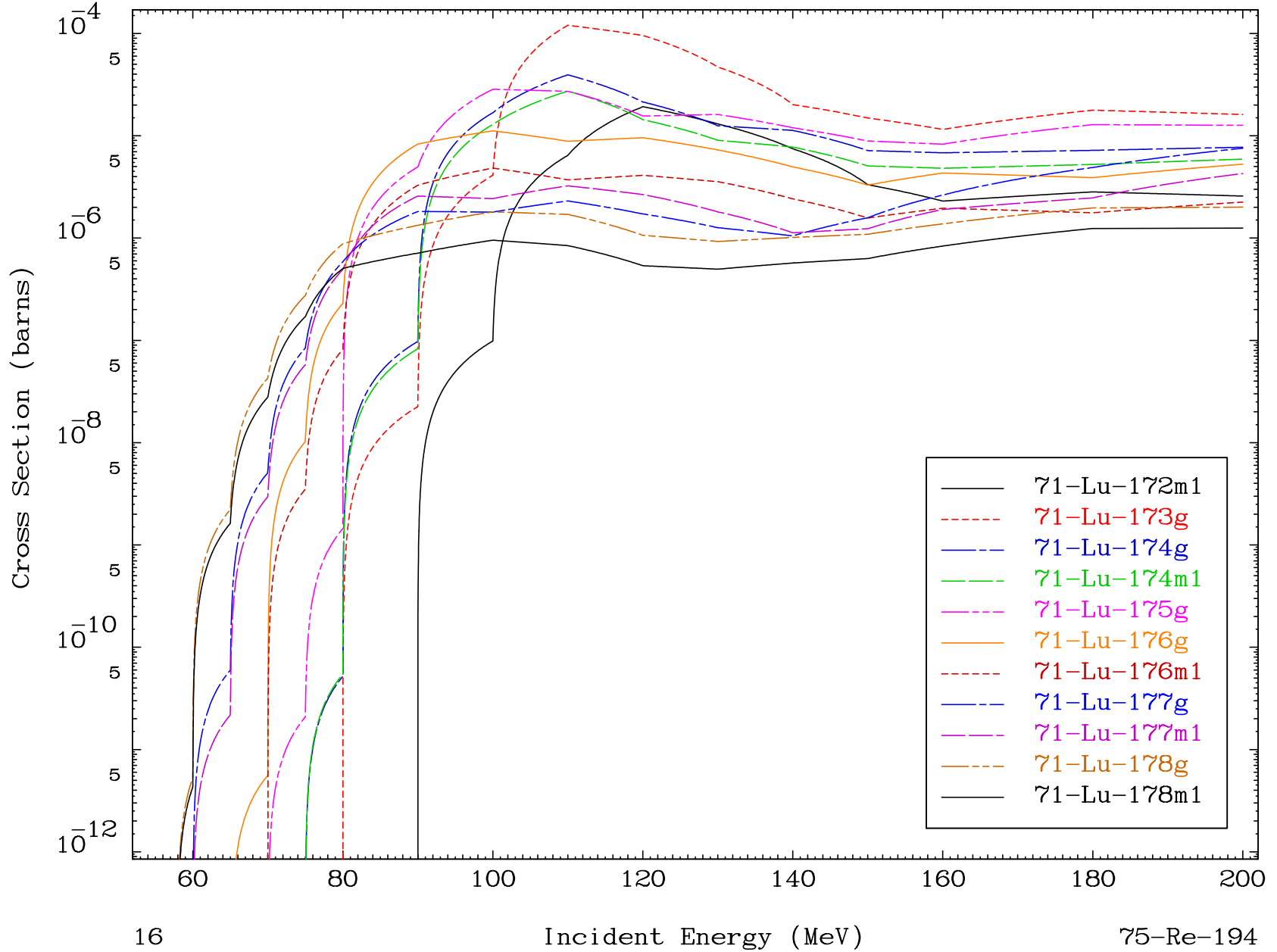




Radionuclide Production Cross Section

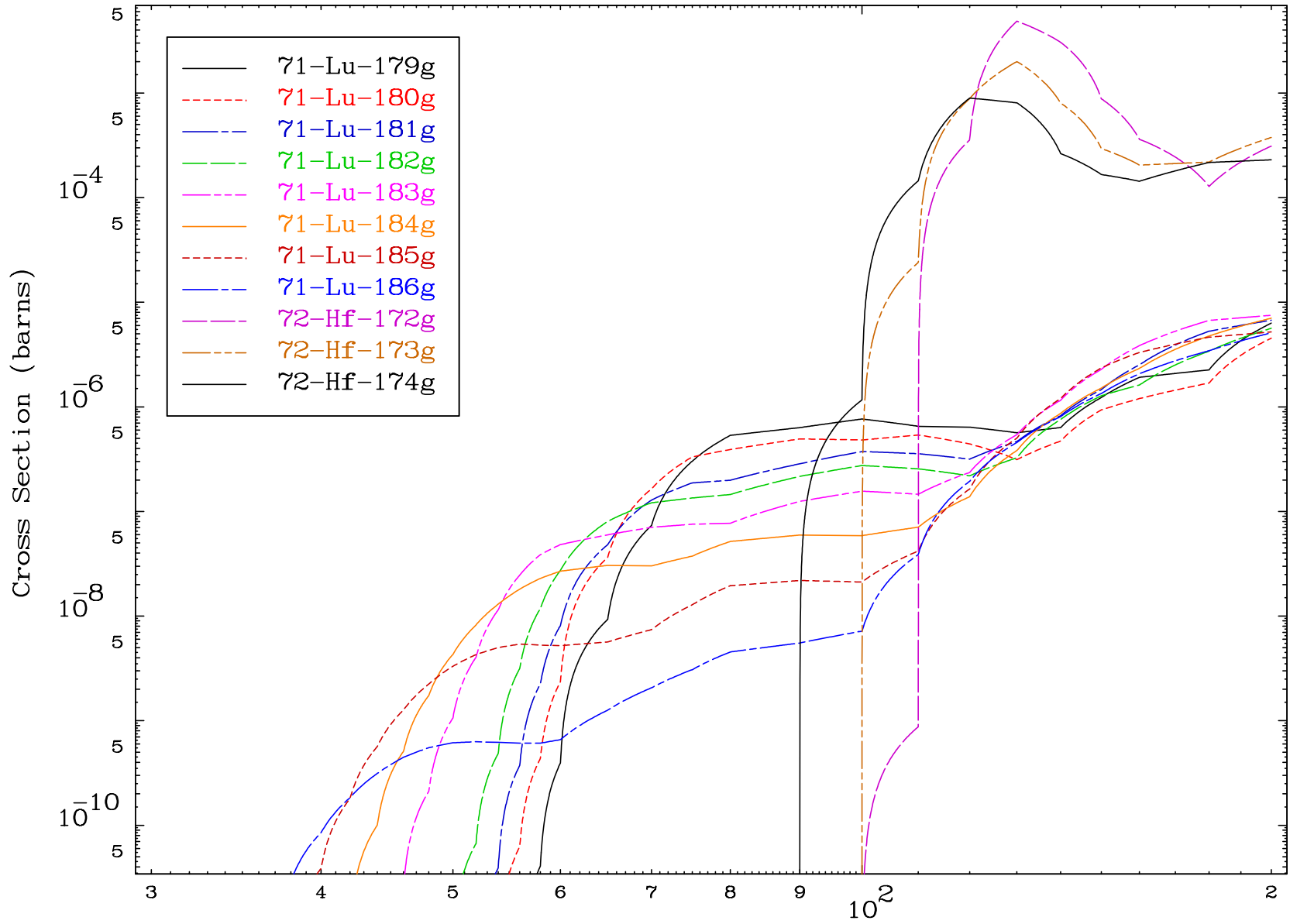


Radionuclide Production Cross Section





Radionuclide Production Cross Section

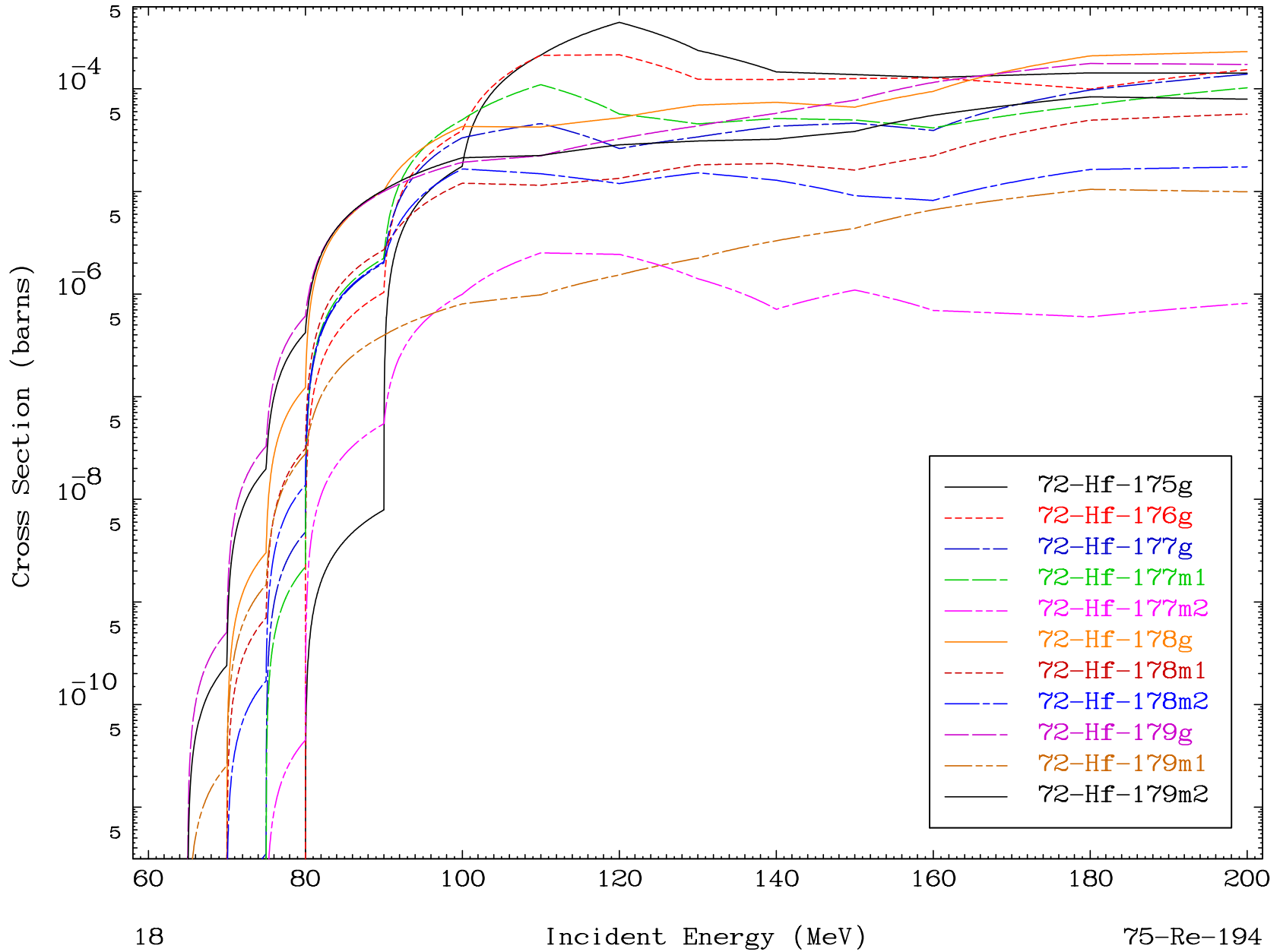


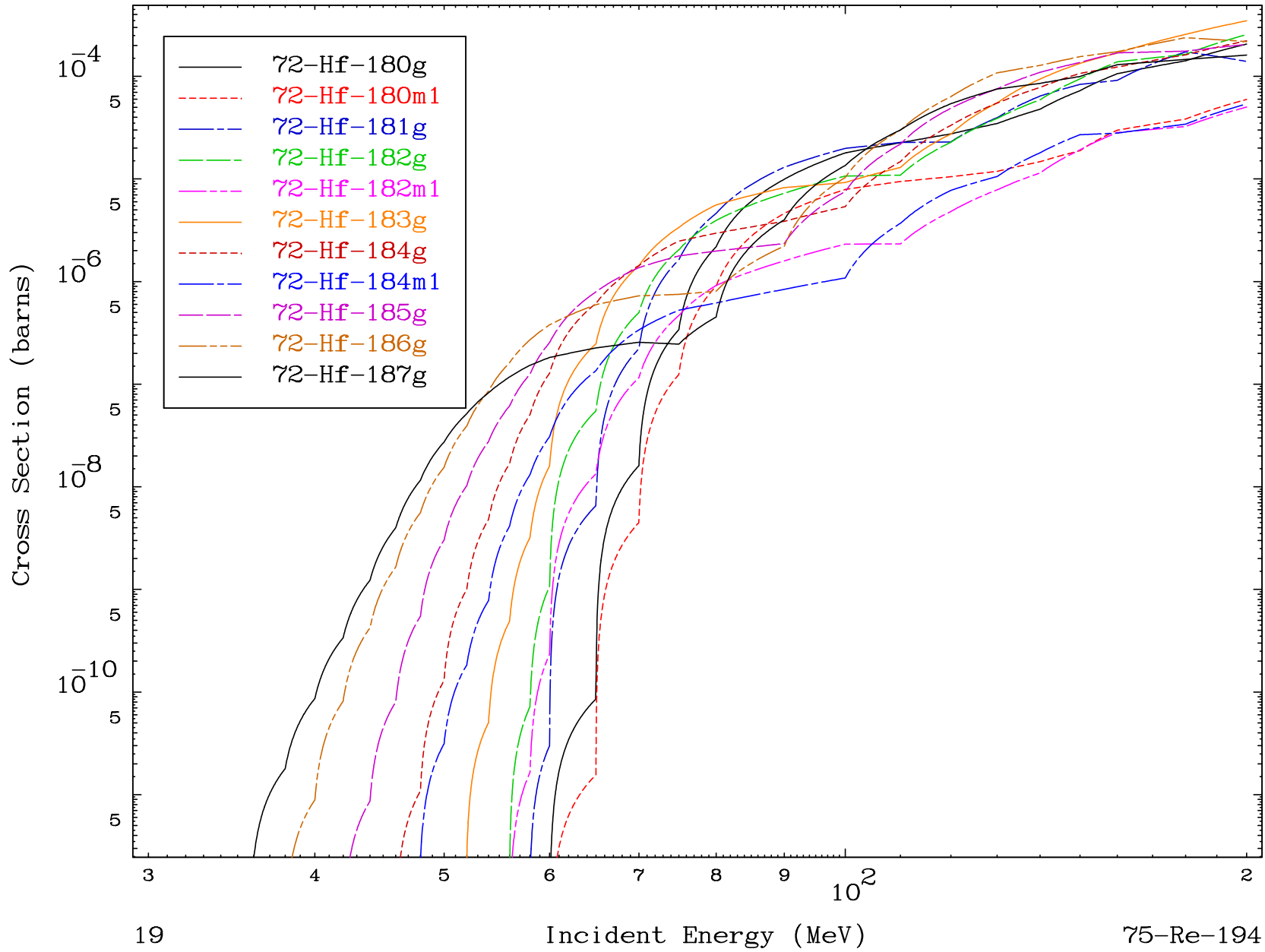
MAT 7552

(n,remainder)

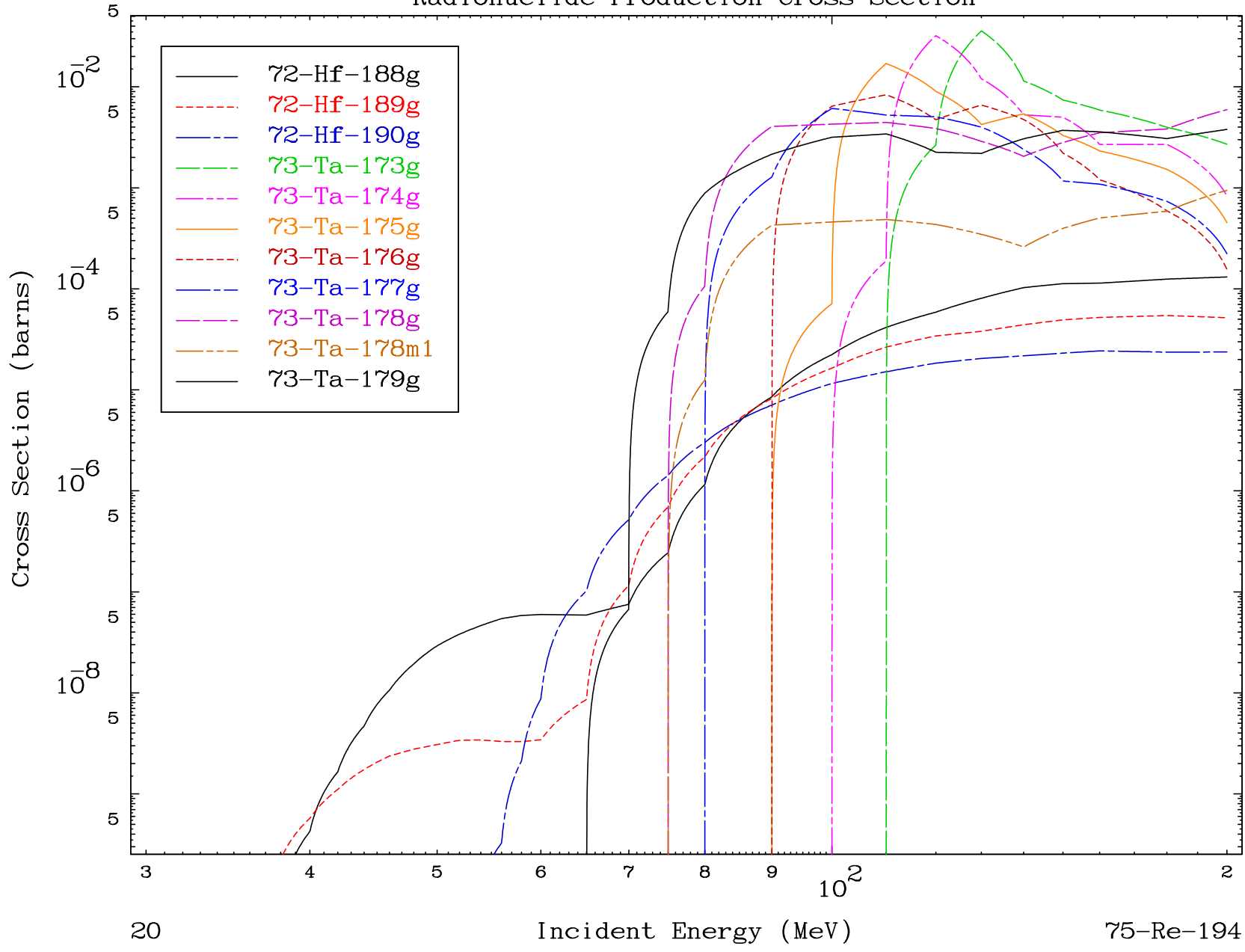
75-Re-194

### Radionuclide Production Cross Section

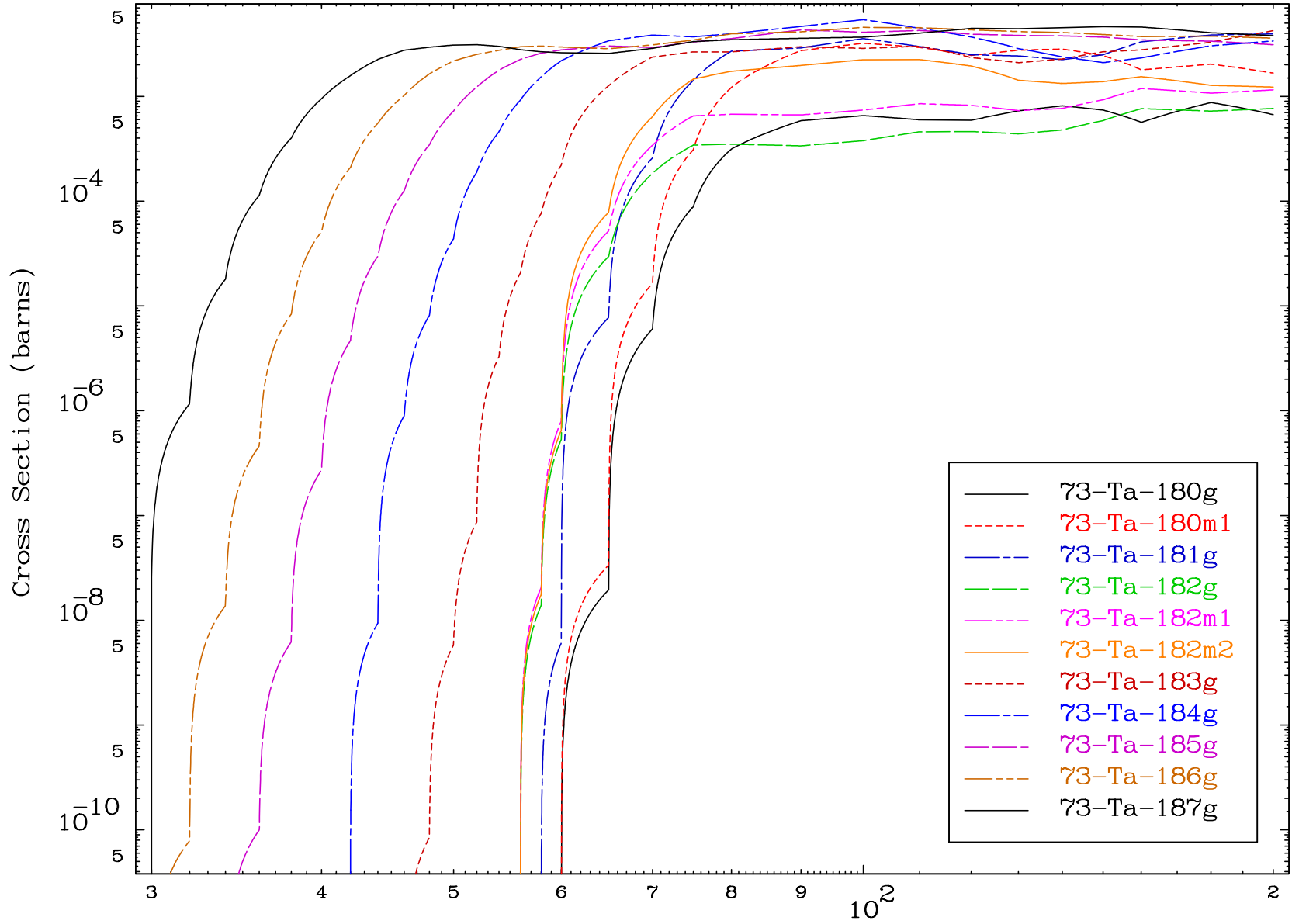




Radionuclide Production Cross Section



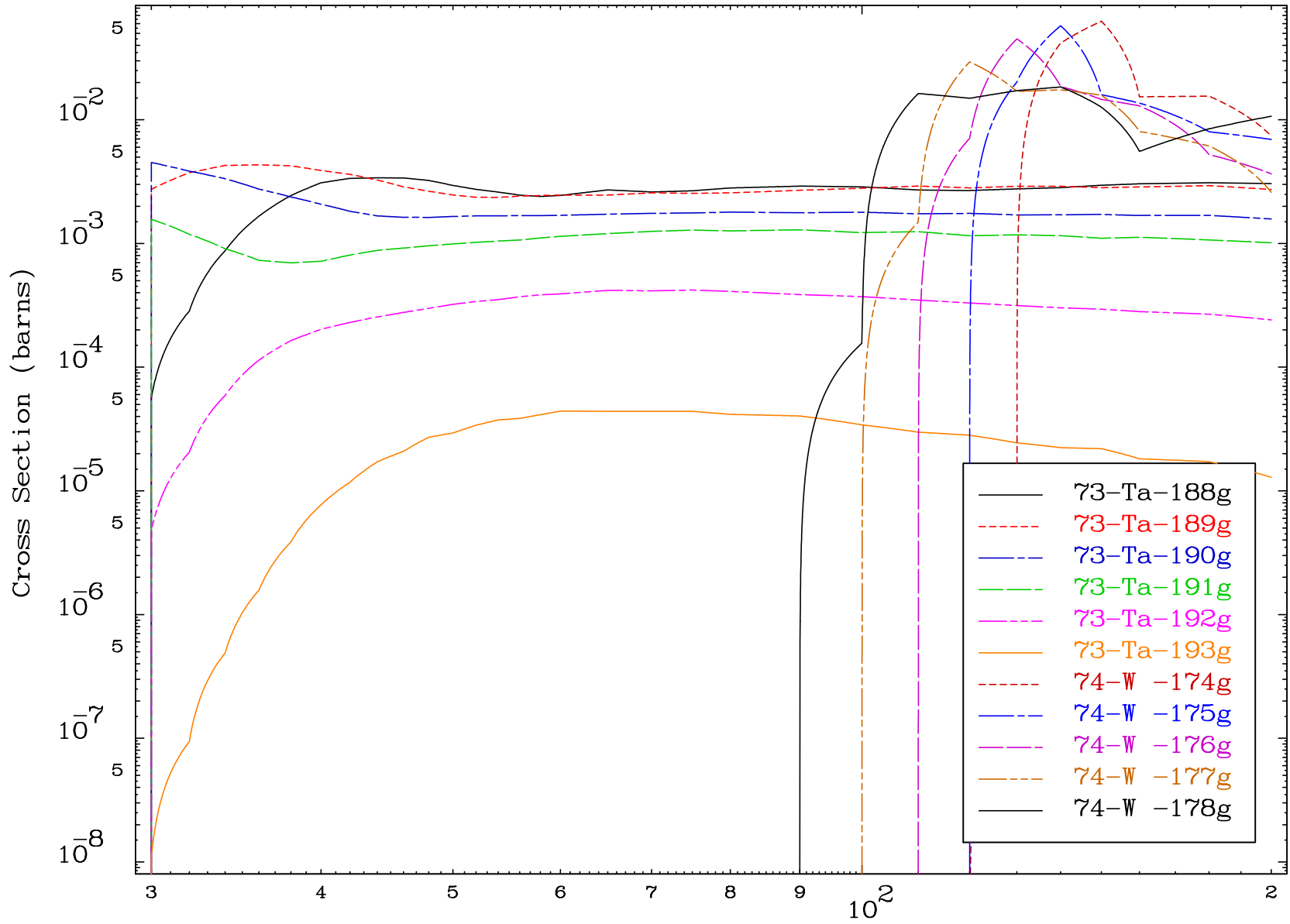
Radionuclide Production Cross Section



MAT 7552

(n,remainder)  
Radionuclide Production Cross Section

75-Re-194



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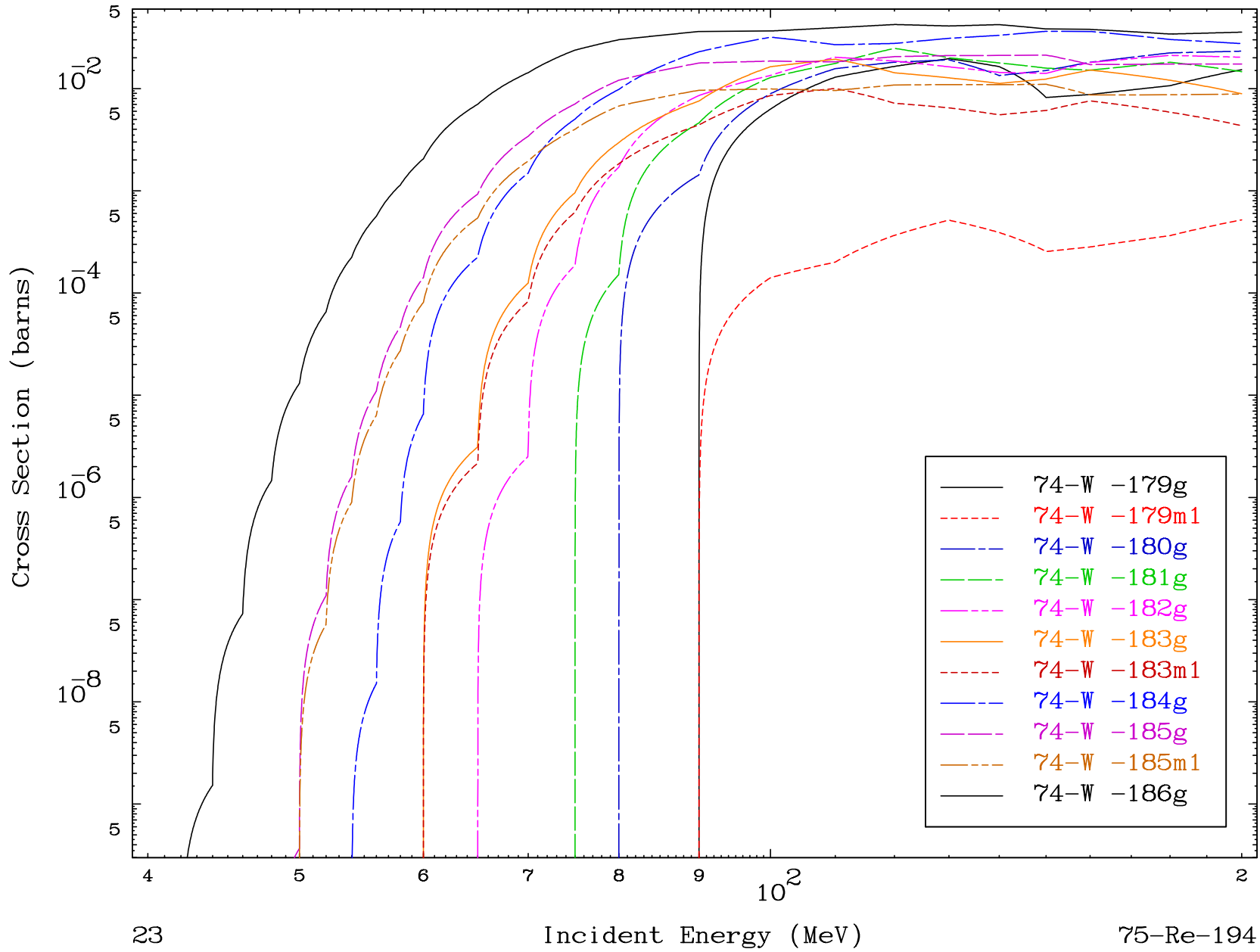
Incident Energy (MeV)

75-Re-194

MAT 7552

(n,remainder)  
Radionuclide Production Cross Section

75-Re-194

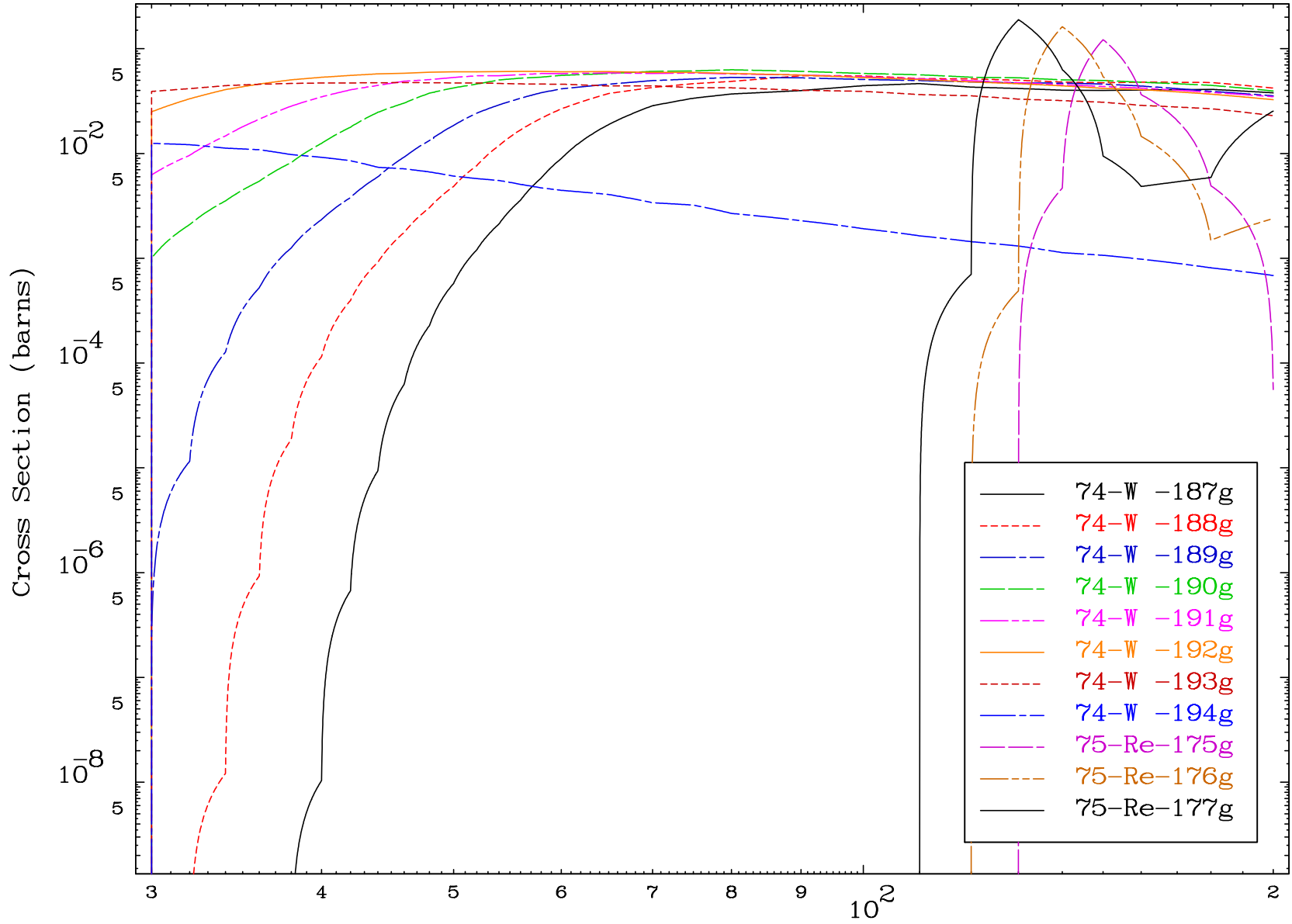


MAT 7552

(n,remainder)

75-Re-194

Radionuclide Production Cross Section



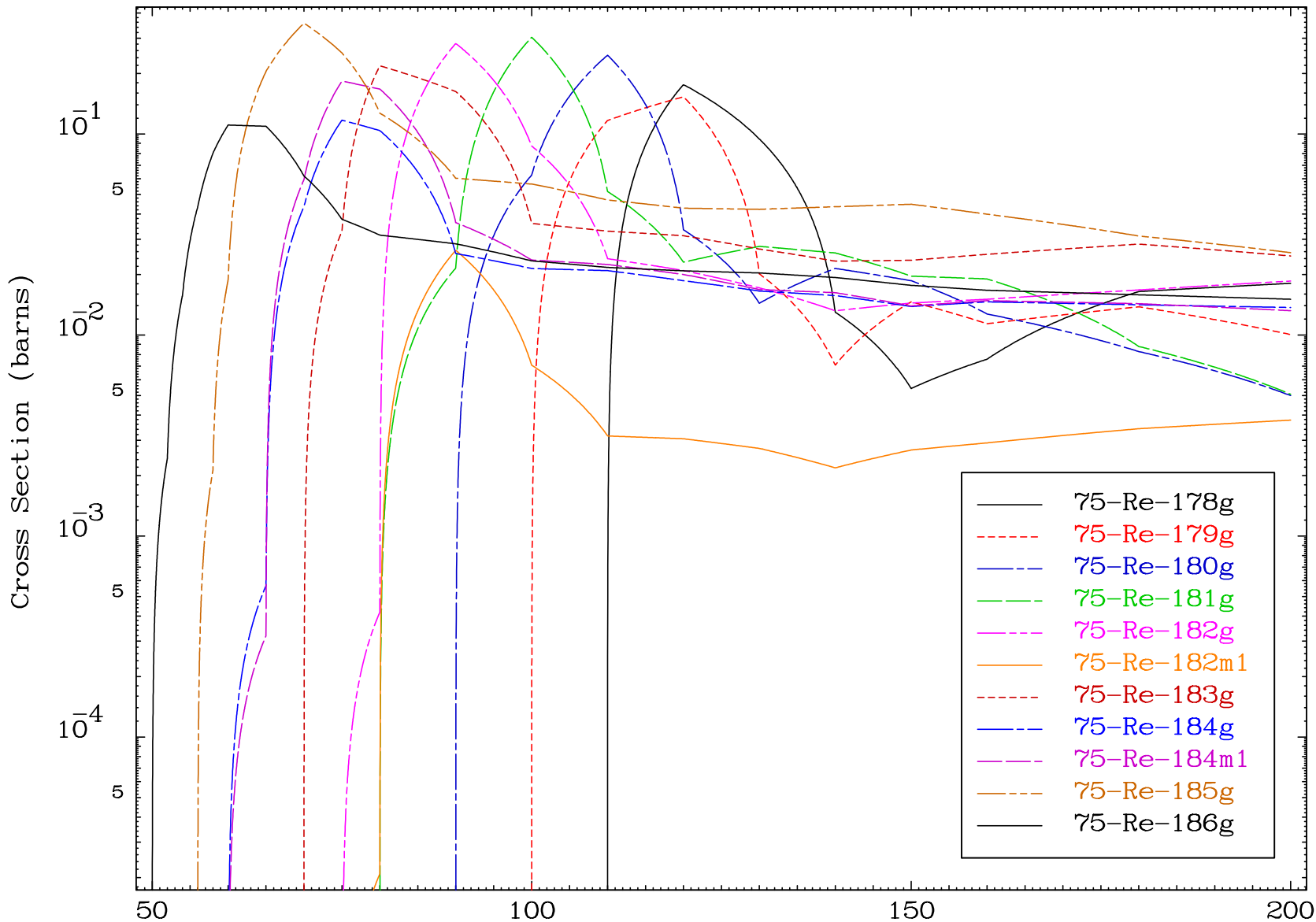
24

Incident Energy (MeV)

75-Re-194



Radionuclide Production Cross Section

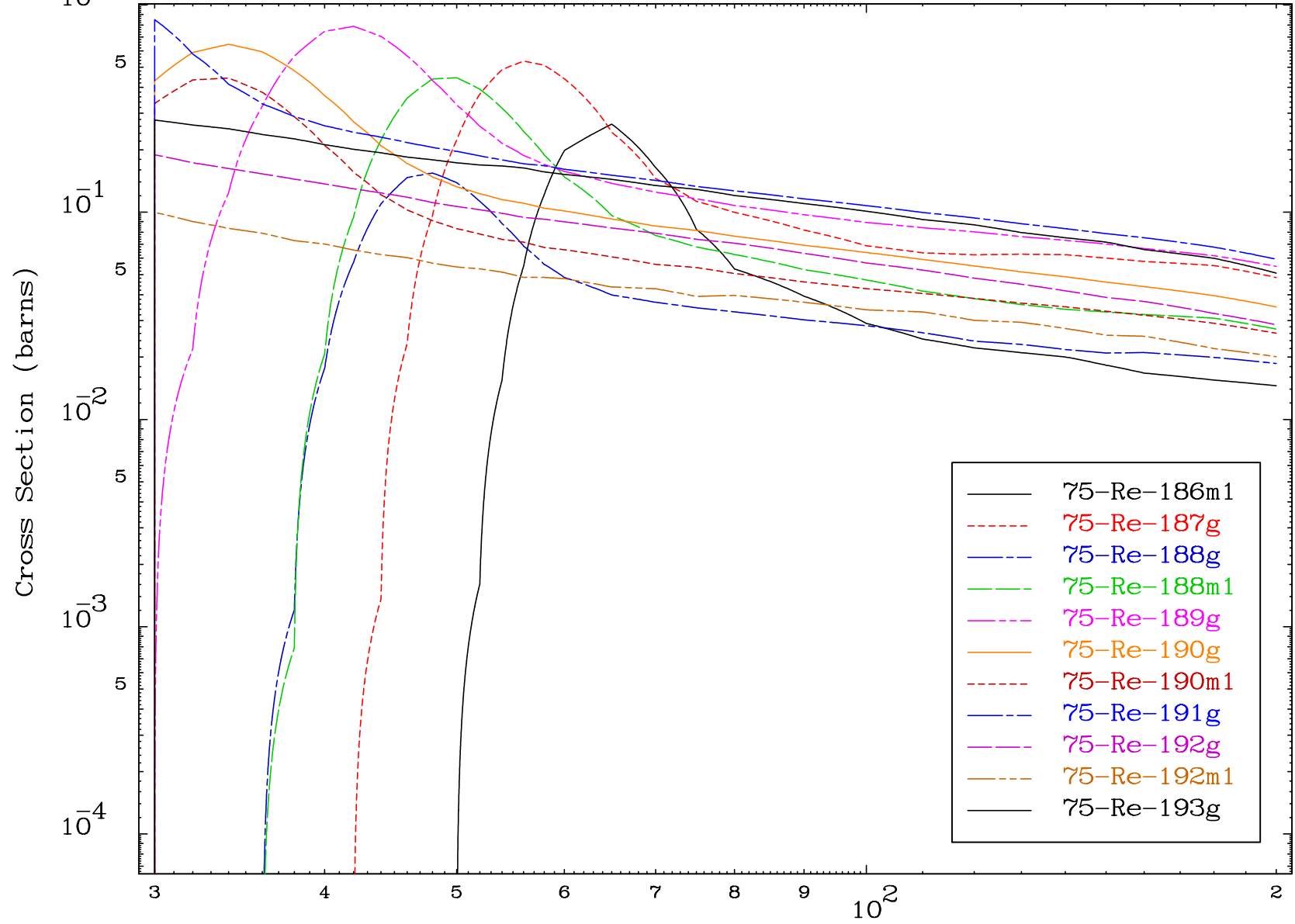


MAT 7552

(n,remainder)

75-Re-194

### Radionuclide Production Cross Section



26

Incident Energy (MeV)

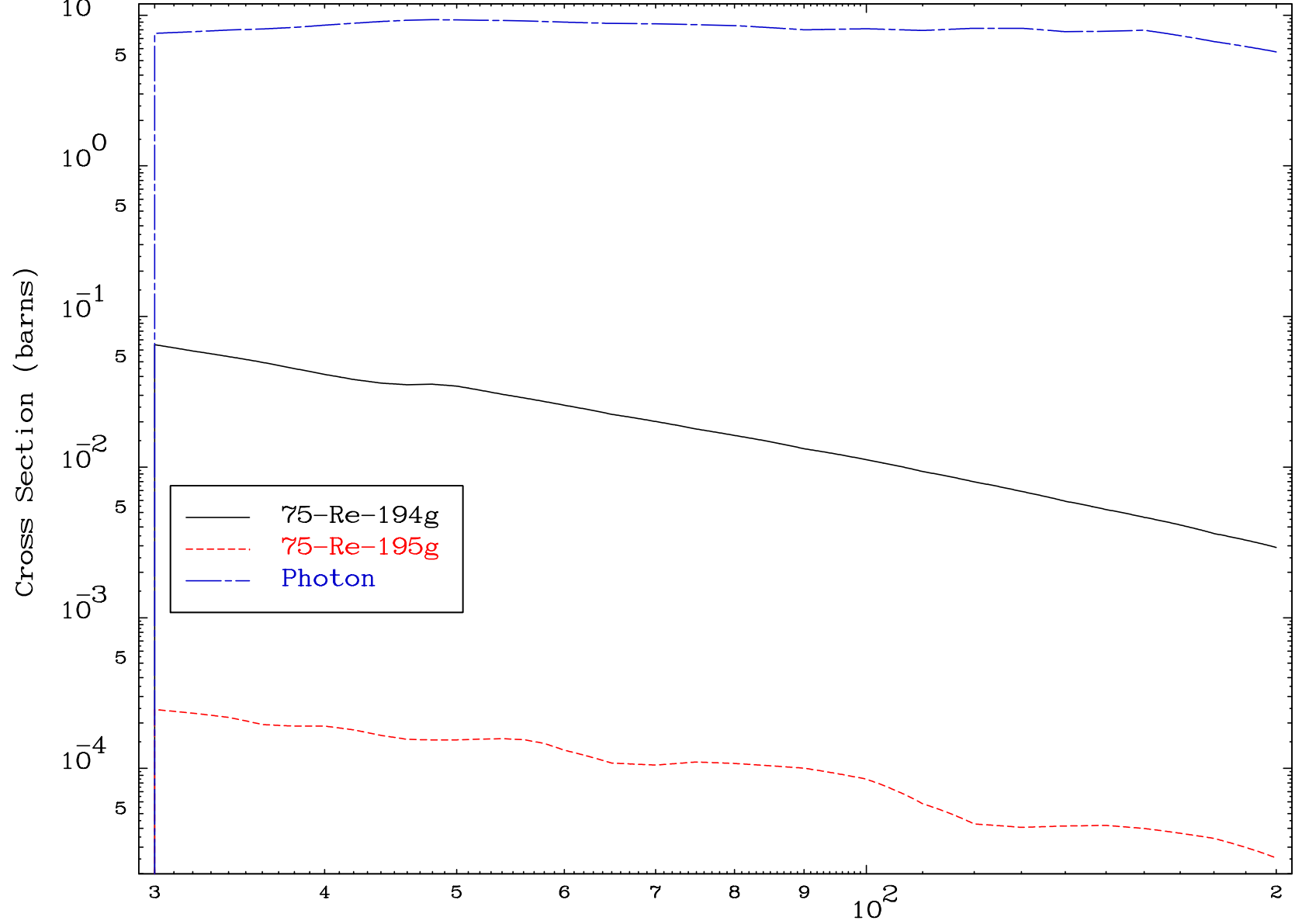
75-Re-194

MAT 7552

(n,remainder)

75-Re-194

Radionuclide Production Cross Section



27

Incident Energy (MeV)

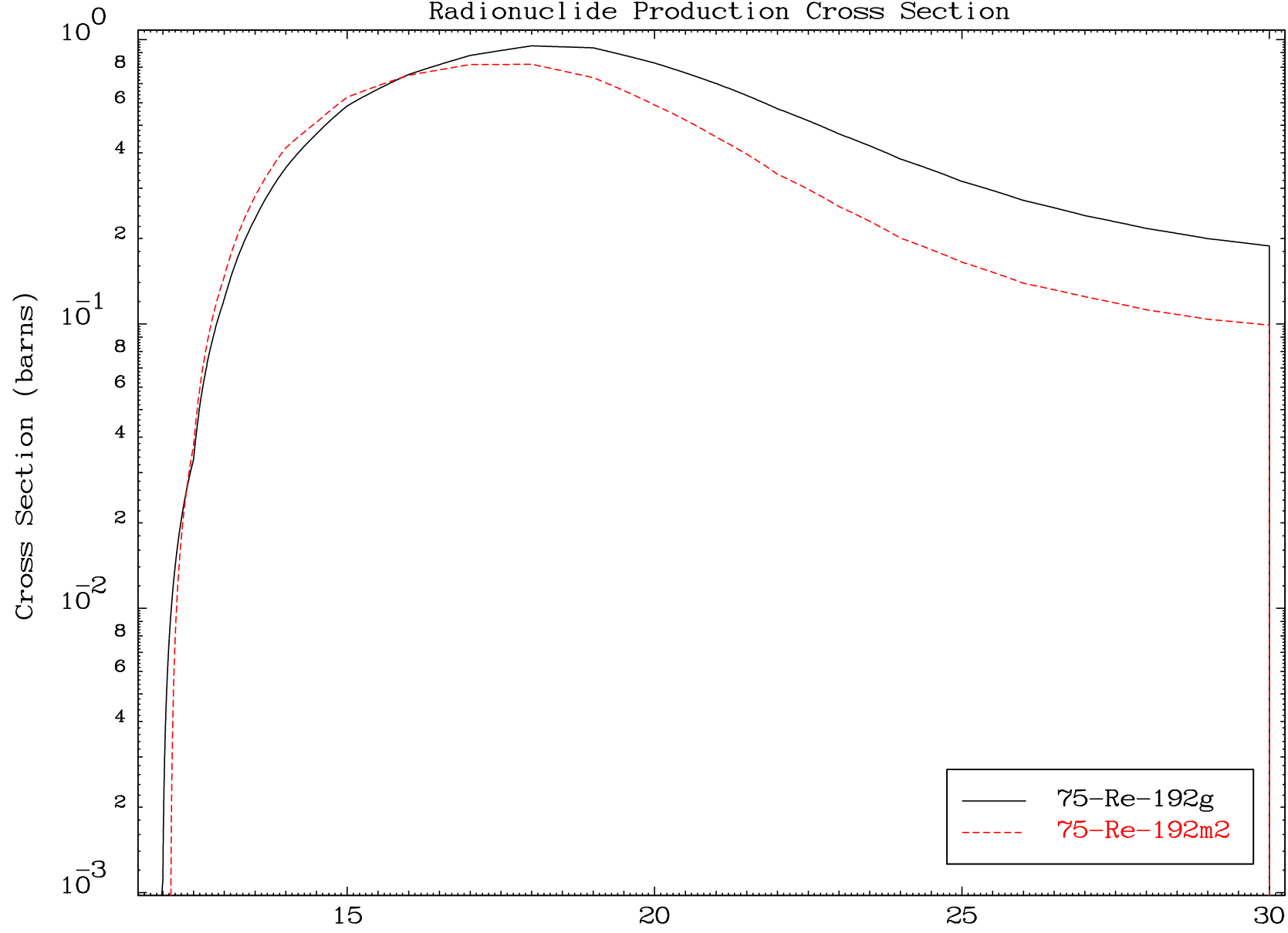
75-Re-194

MAT 7552

(n,3n)

75-Re-194

Radionuclide Production Cross Section



28

Incident Energy (MeV)

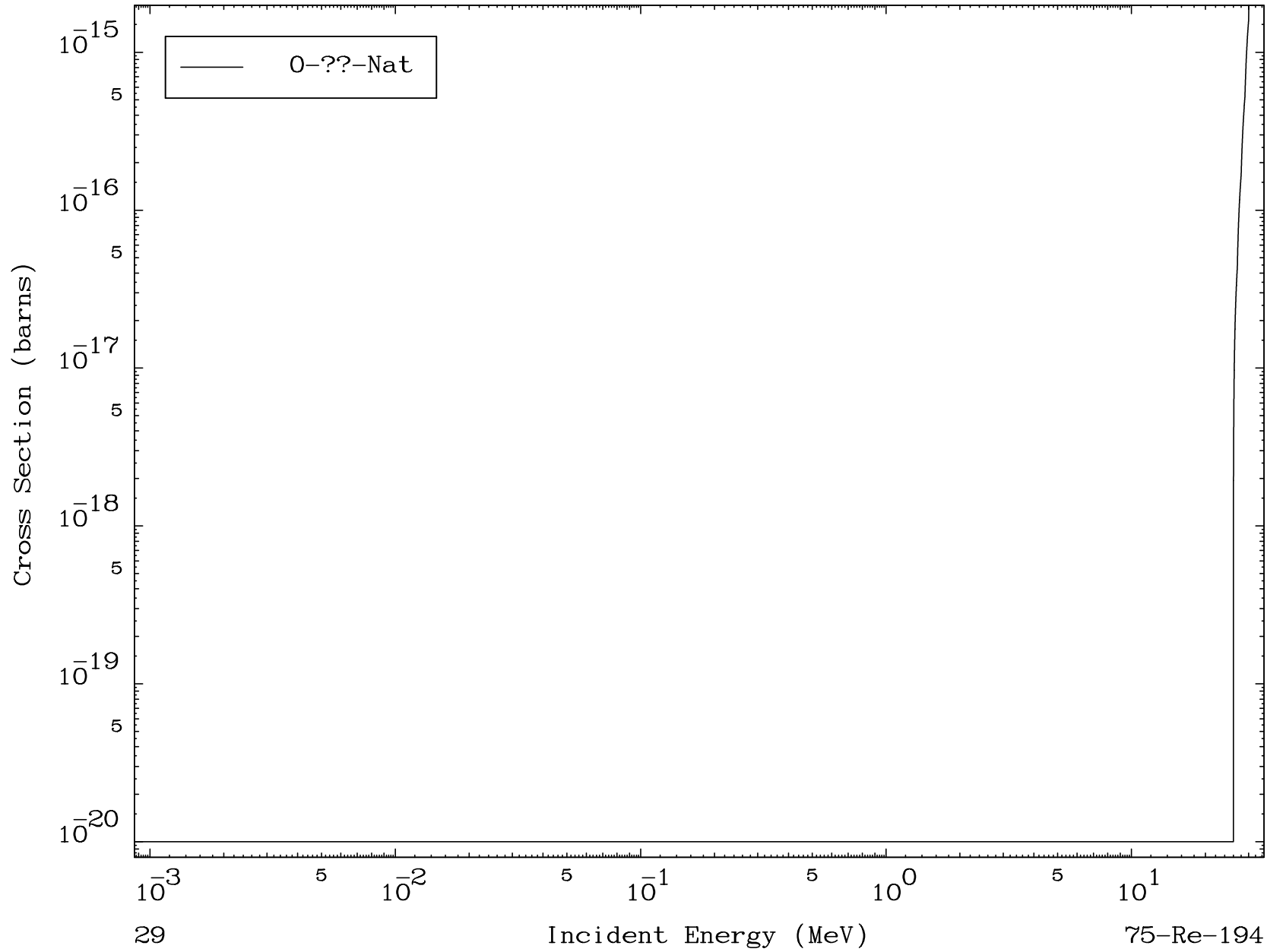
75-Re-194

— 75-Re-192g  
- - - 75-Re-192m2

MAT 7552

Fission  
Radionuclide Production Cross Section

75-Re-194



29

Incident Energy (MeV)

75-Re-194