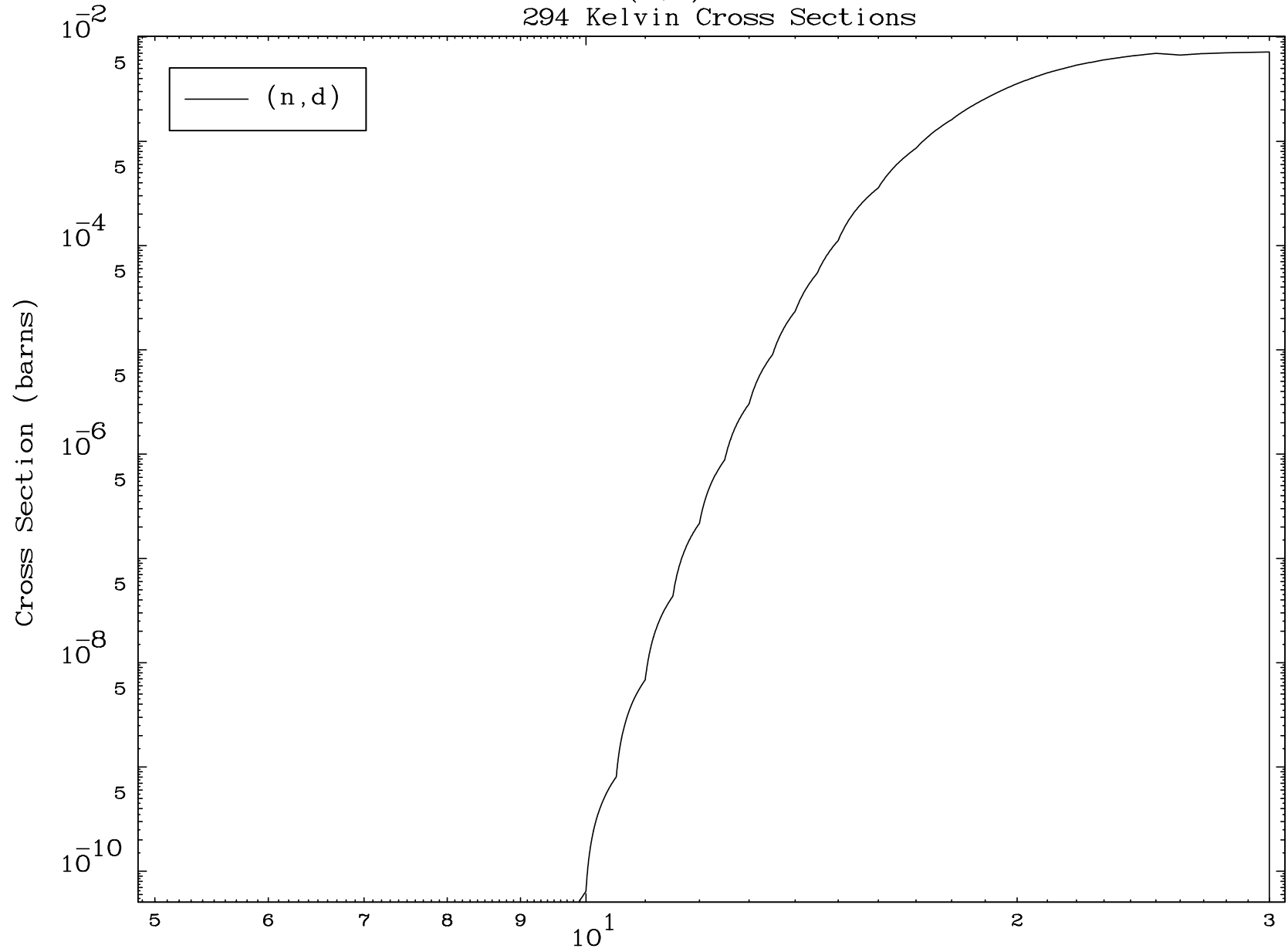


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(n,d) Levels  
294 Kelvin Cross Sections

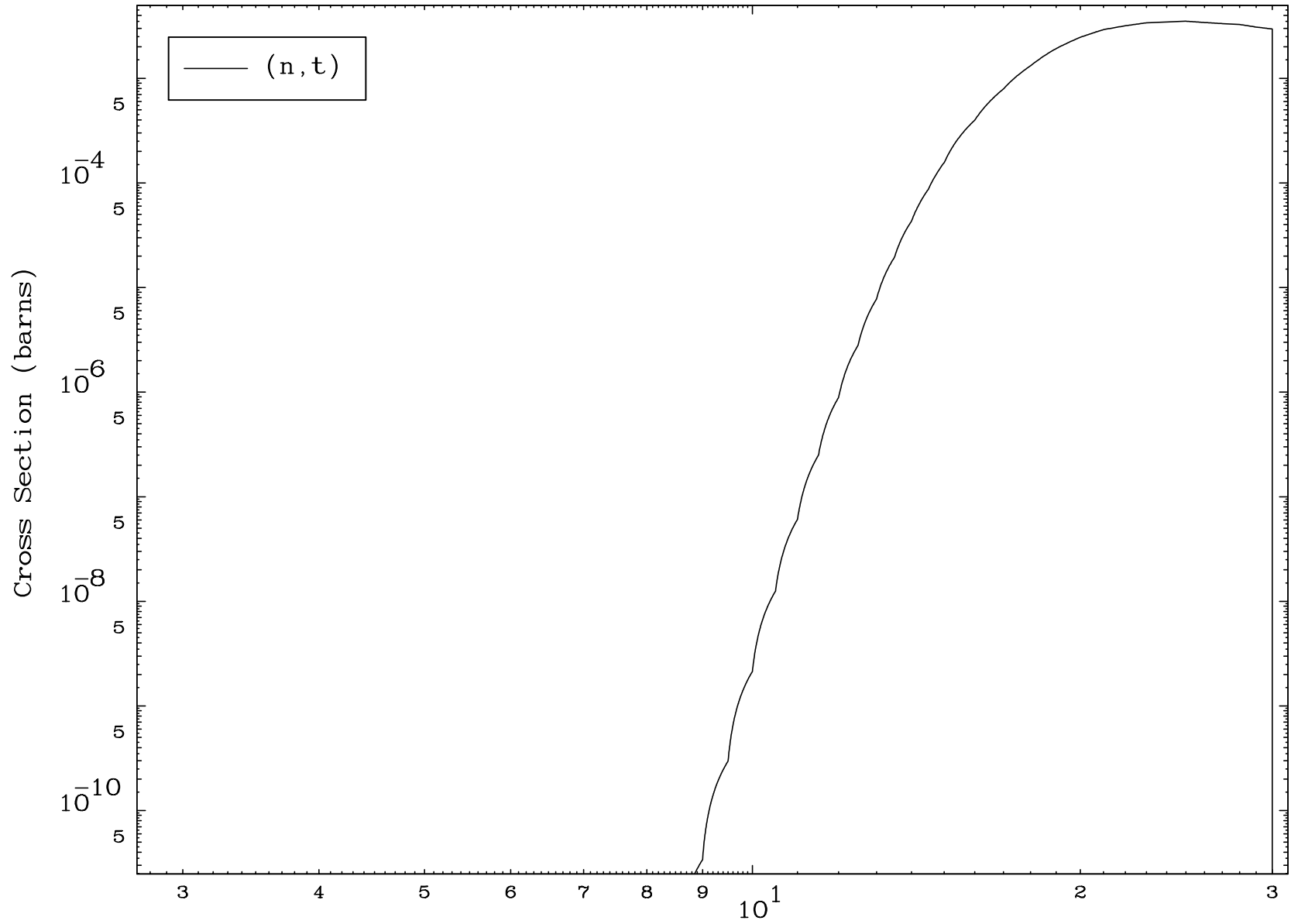
86-Rn-221

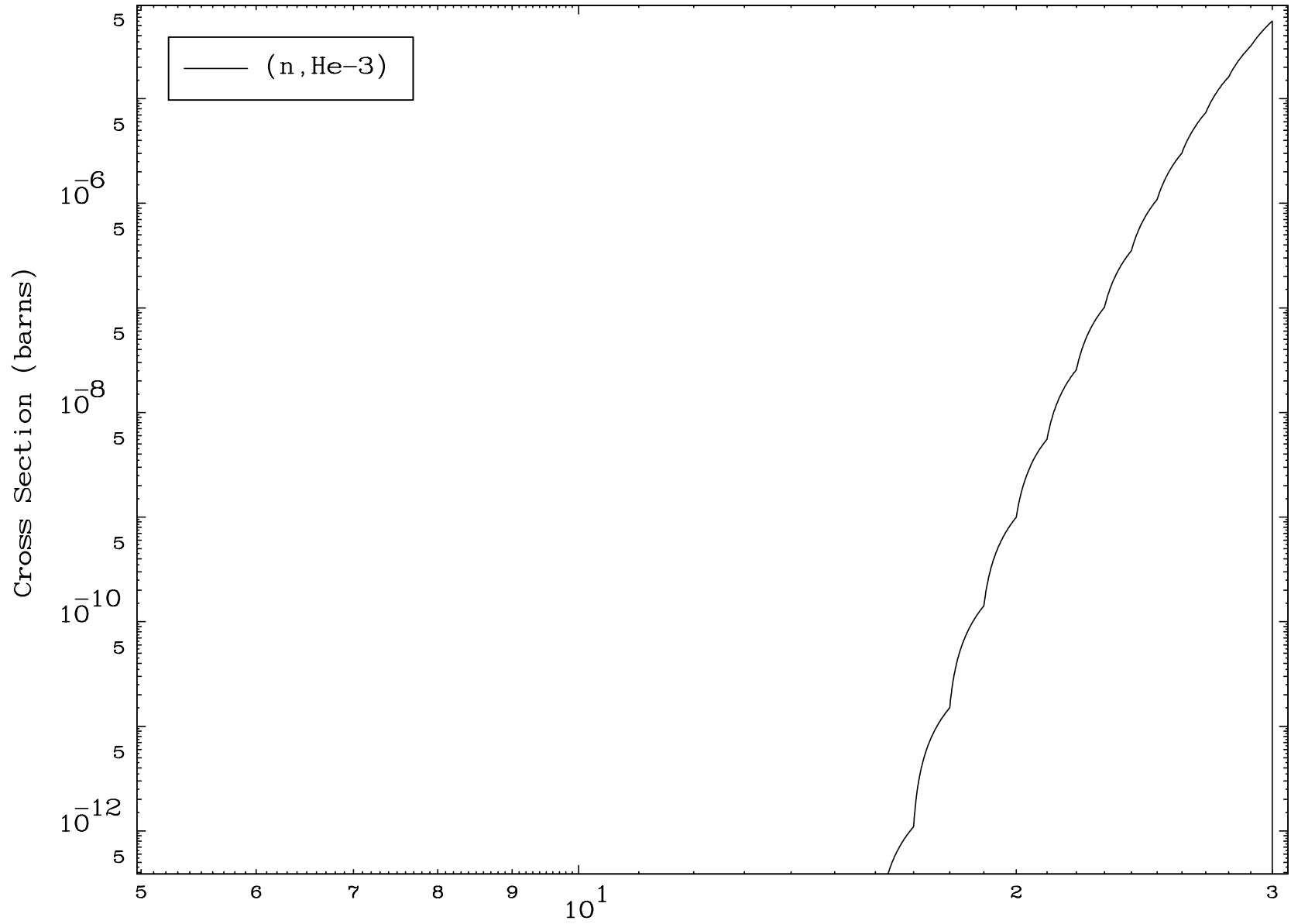


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Incident Energy (MeV)

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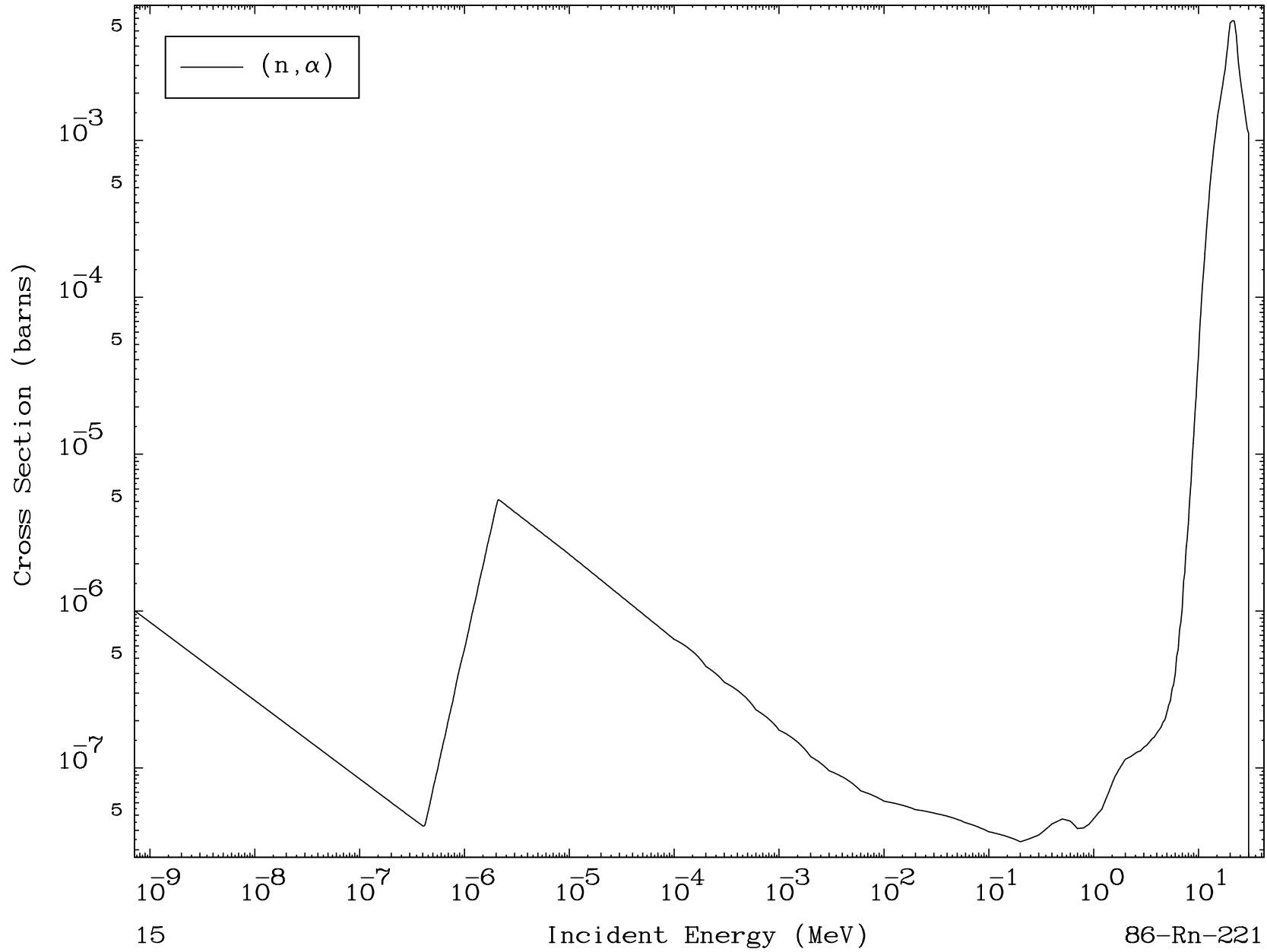




MAT 8655

(n,  $\alpha$ ) Levels  
294 Kelvin Cross Sections

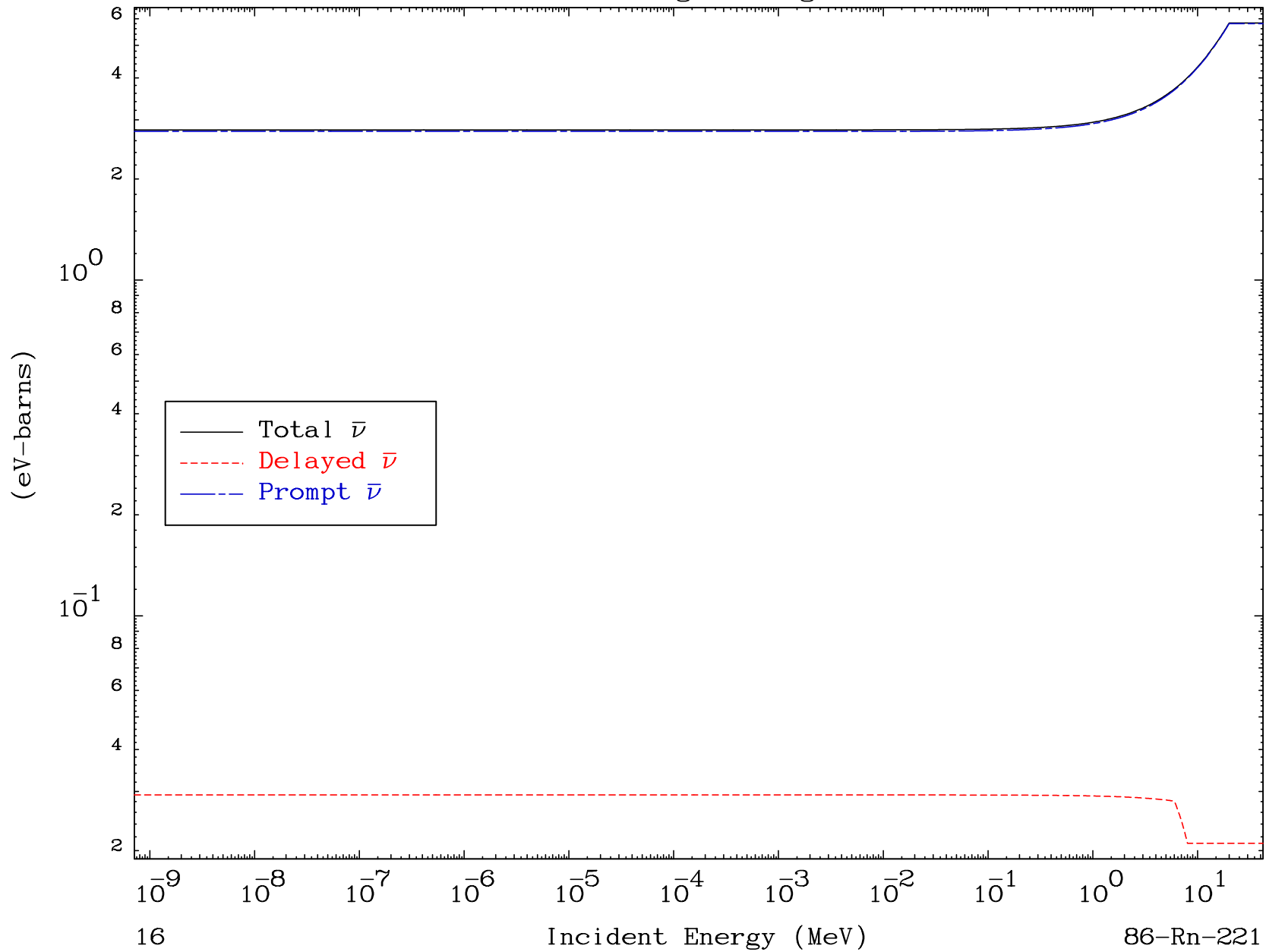
86-Rn-221



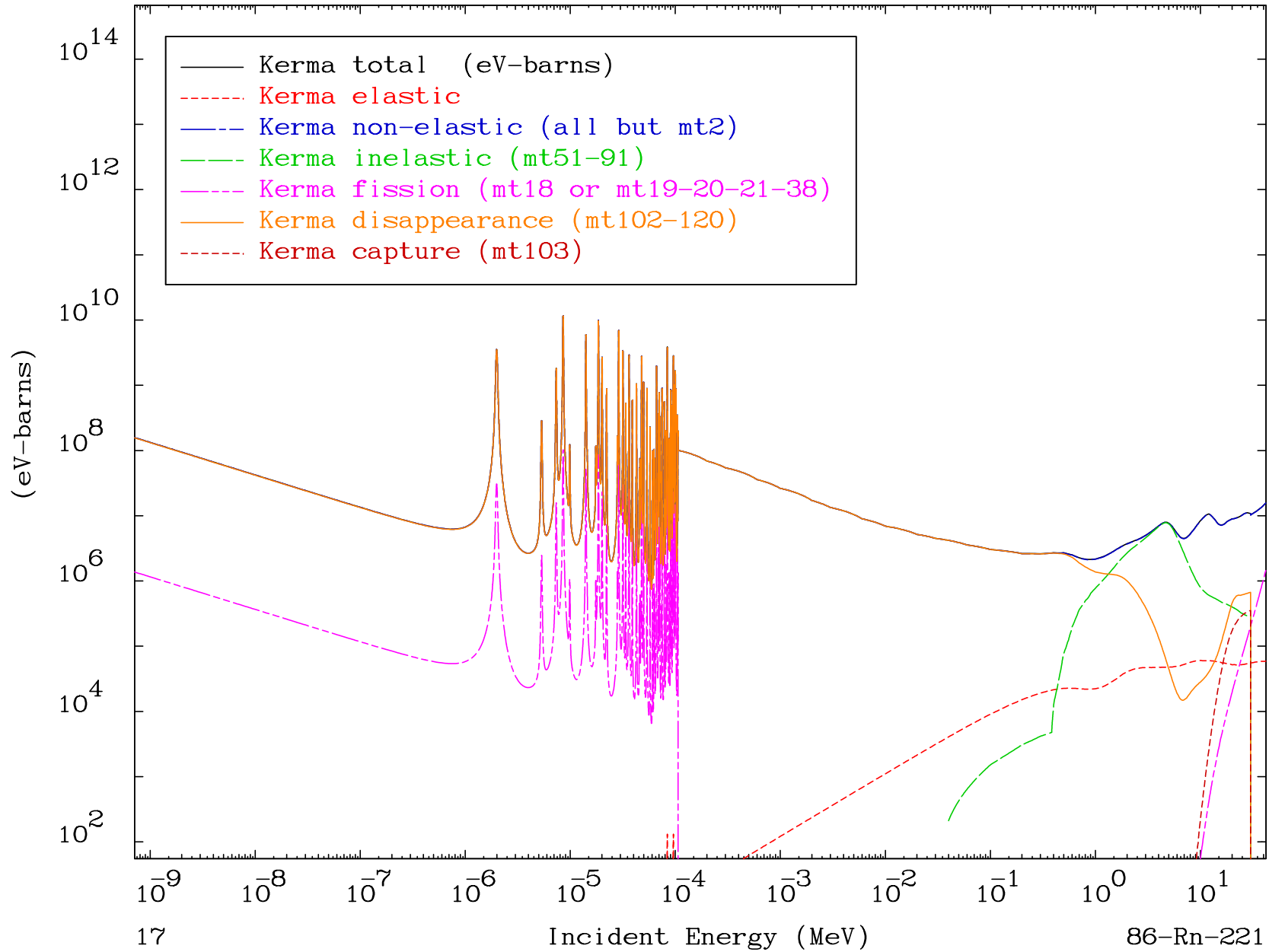
MAT 8655

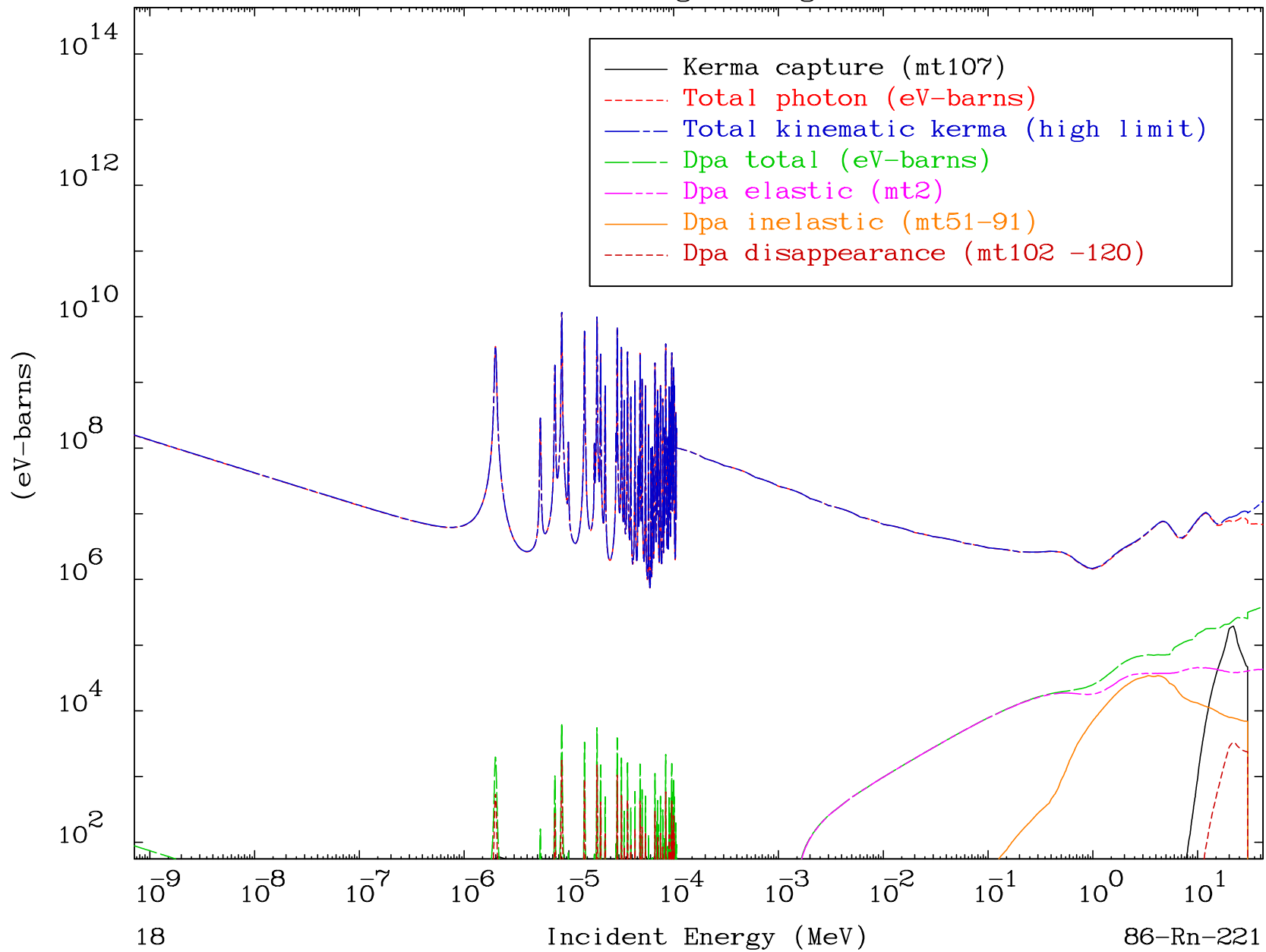
Energy Release  
Heating - Damage

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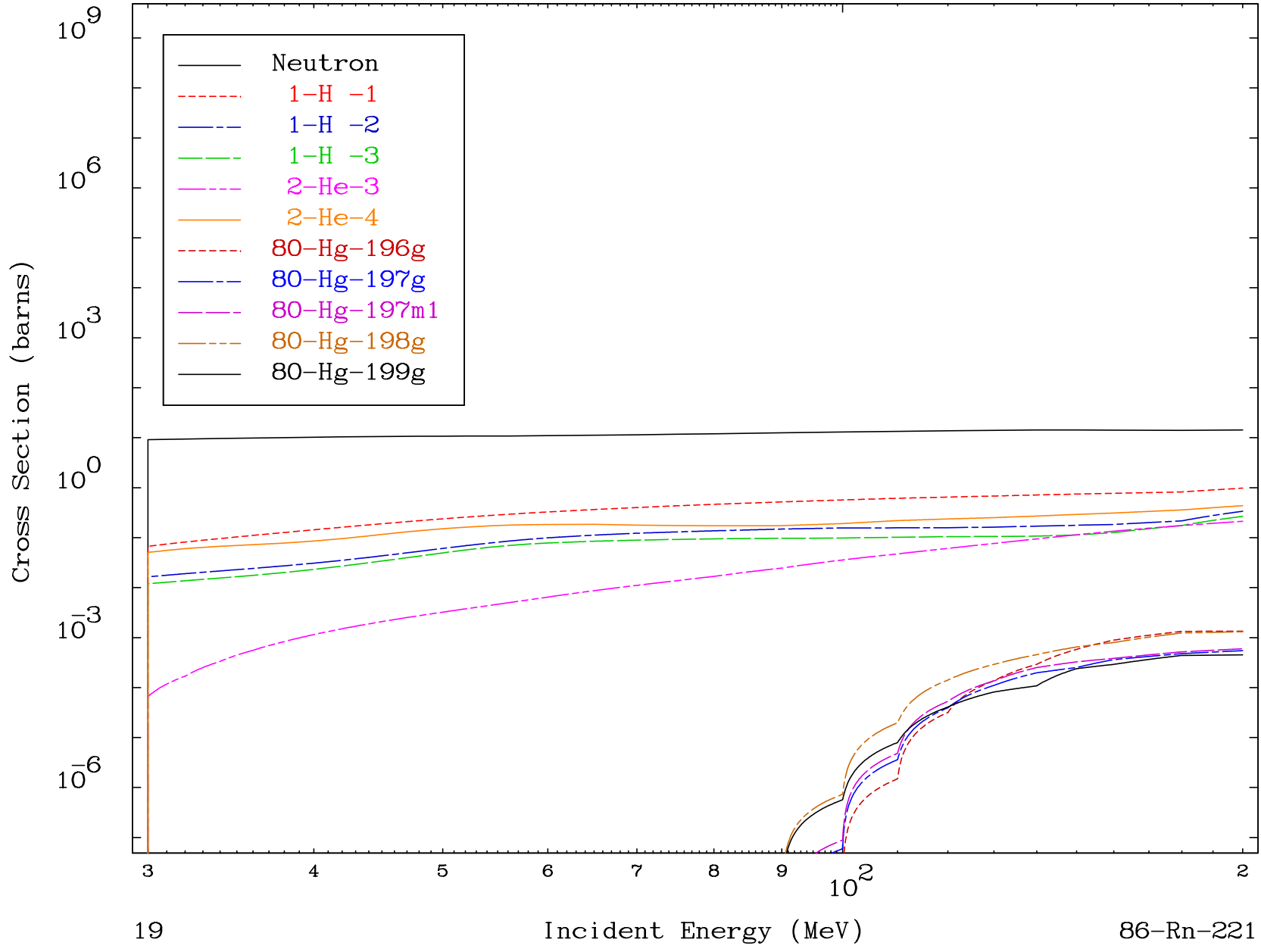




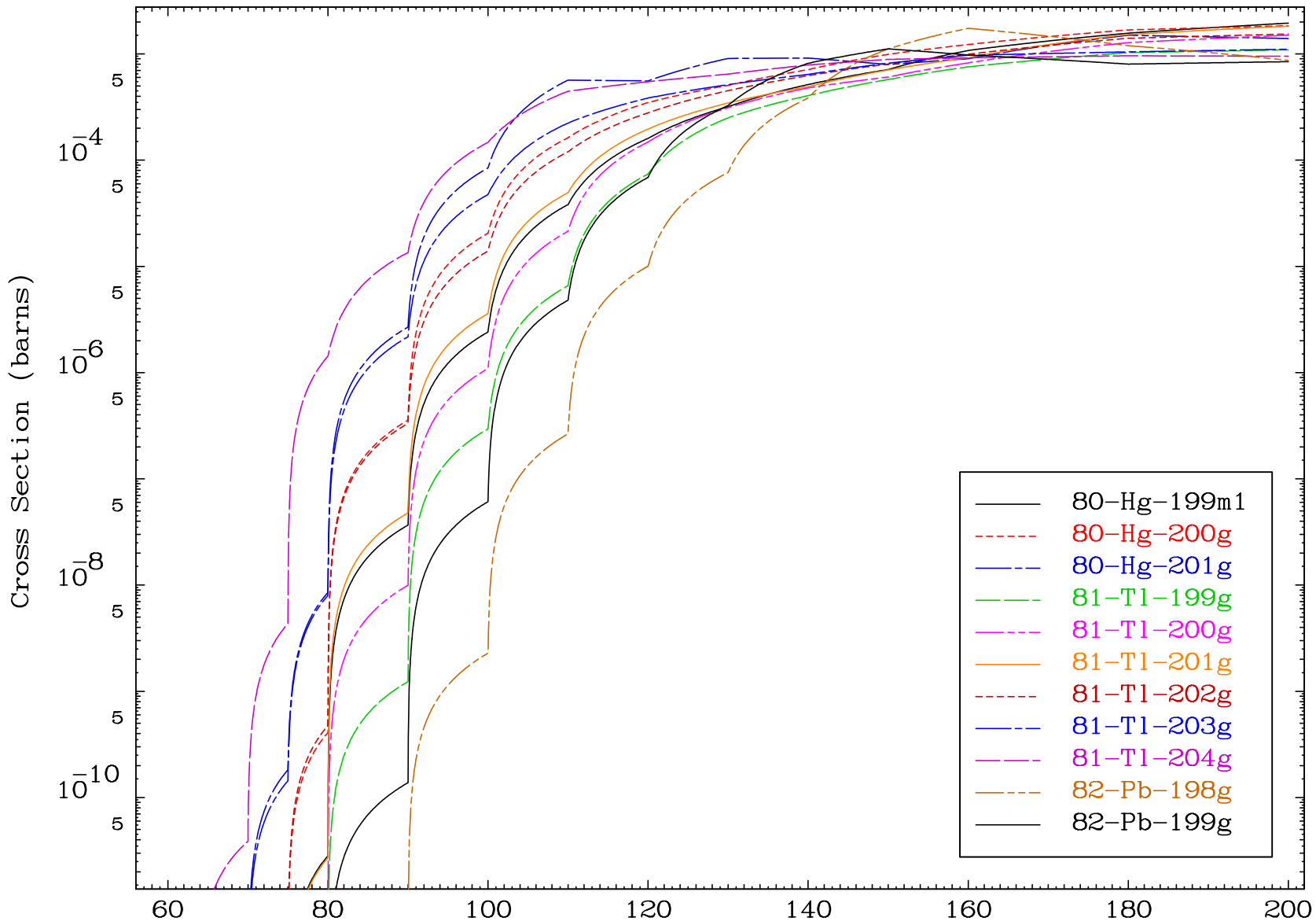




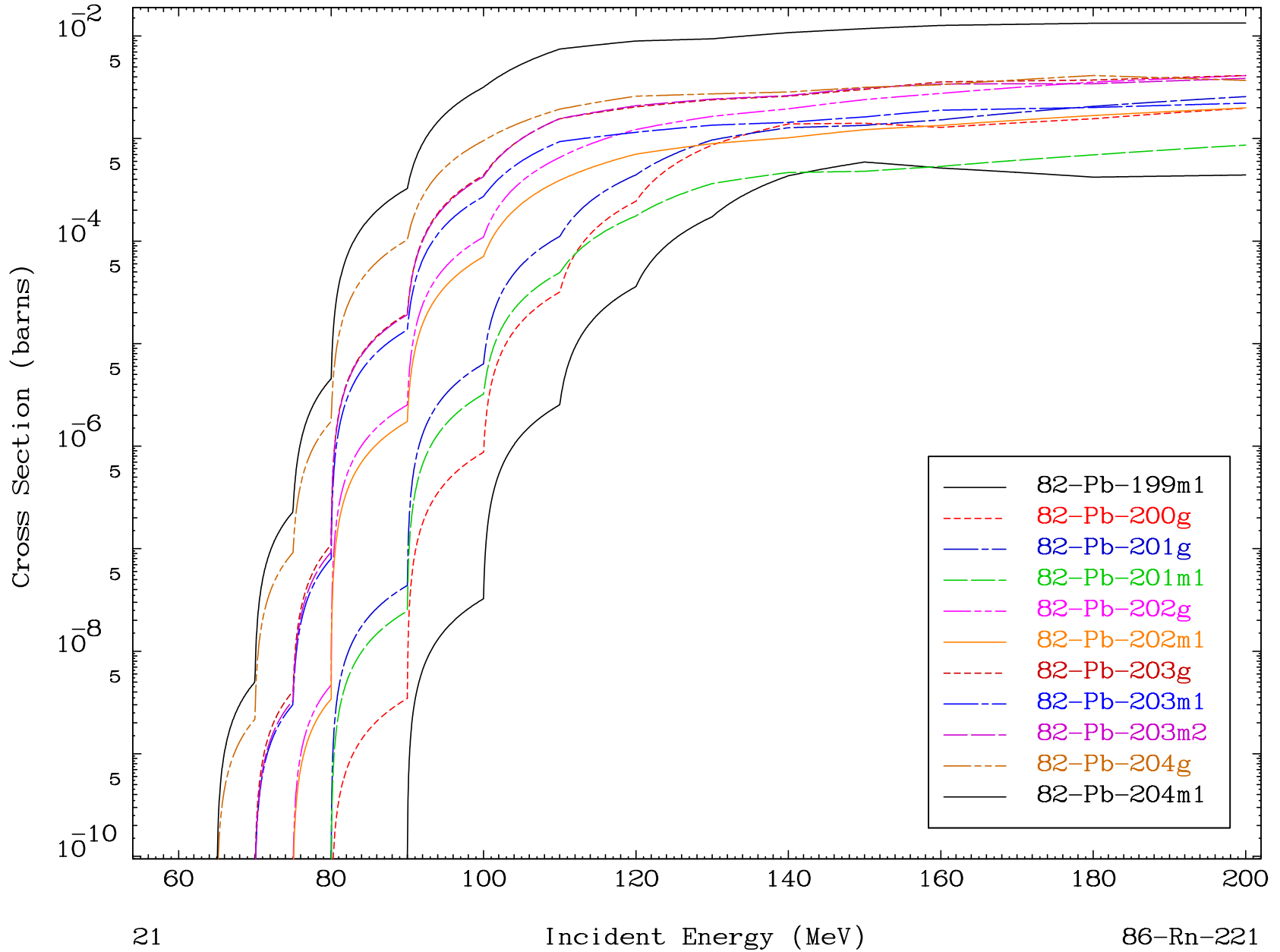
Radionuclide Production Cross Section

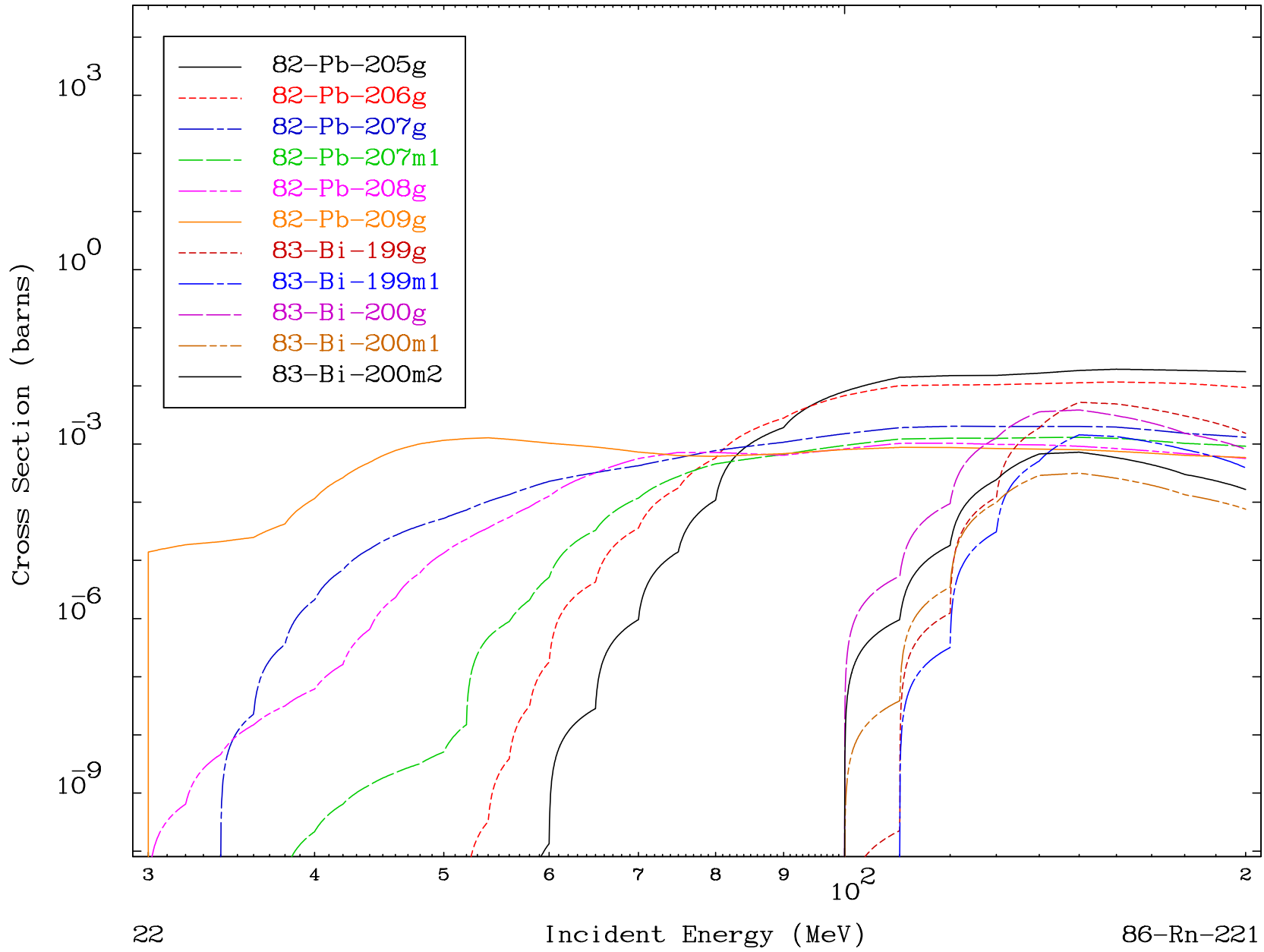


Radionuclide Production Cross Section



Radionuclide Production Cross Section

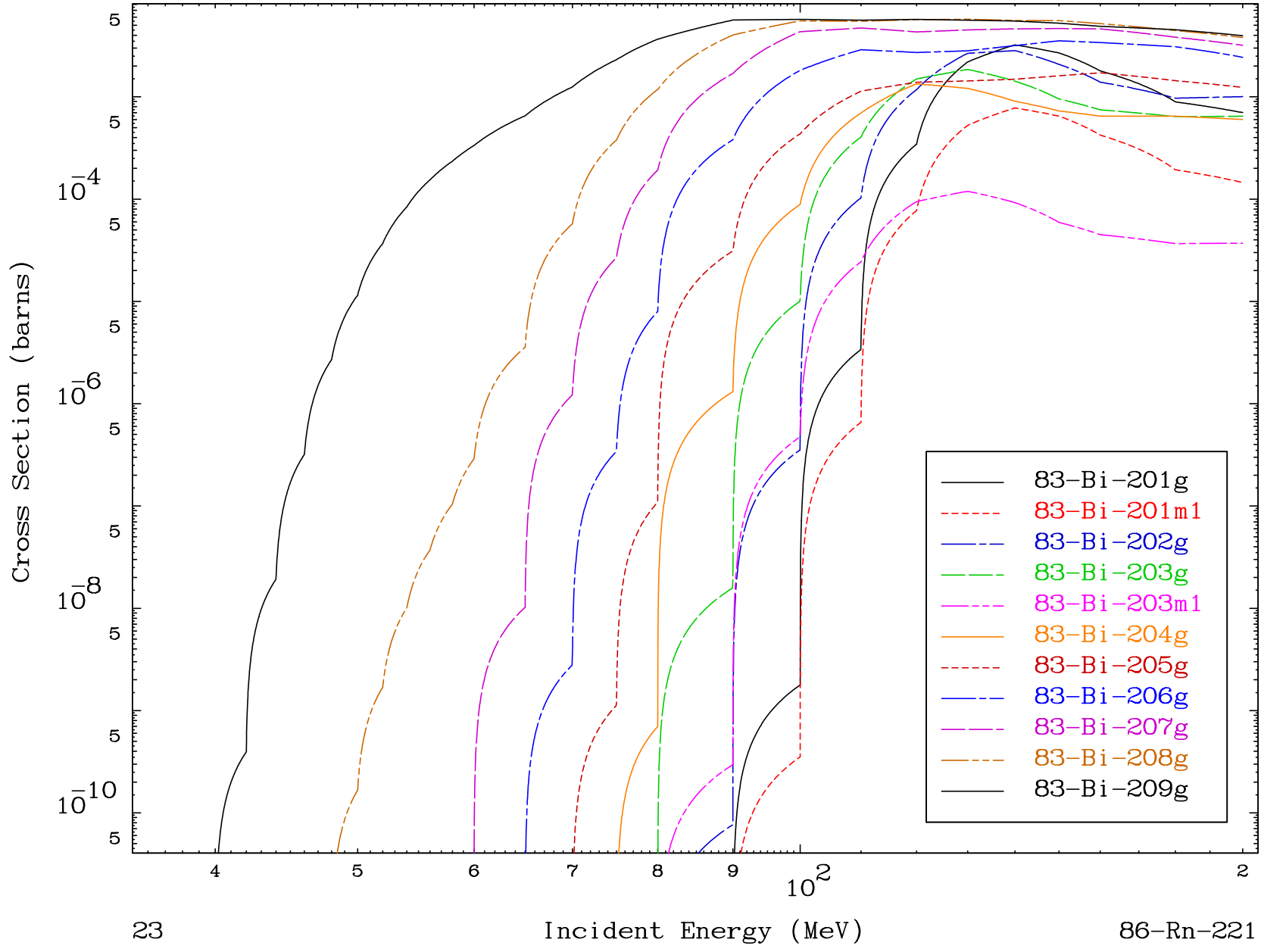




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(n,remainder)  
Radionuclide Production Cross Section

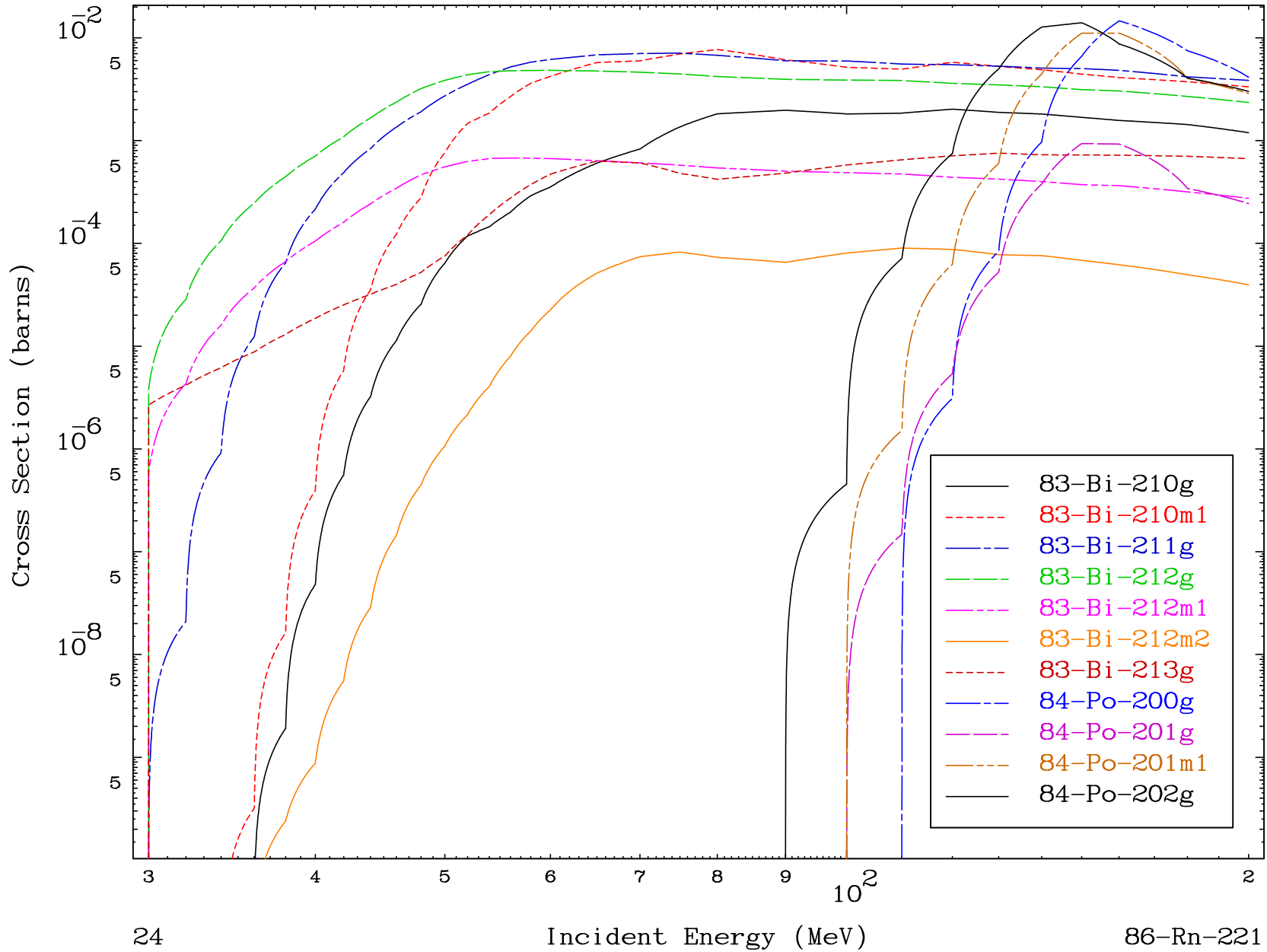
86-Rn-221



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(n,remainder)  
Radionuclide Production Cross Section

86-Rn-221



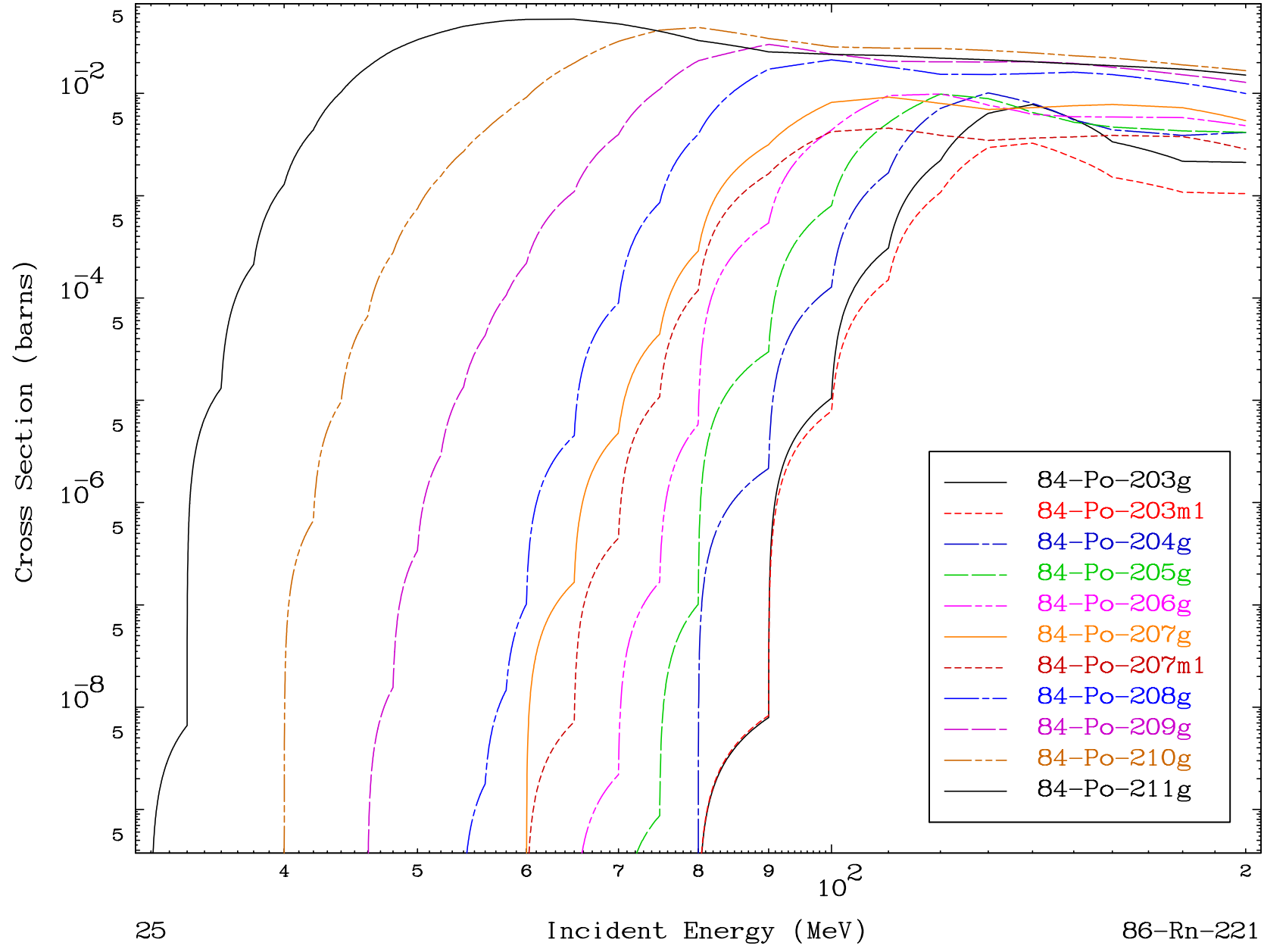


MAT 8655

(n,remainder)

86-Rn-221

### Radionuclide Production Cross Section

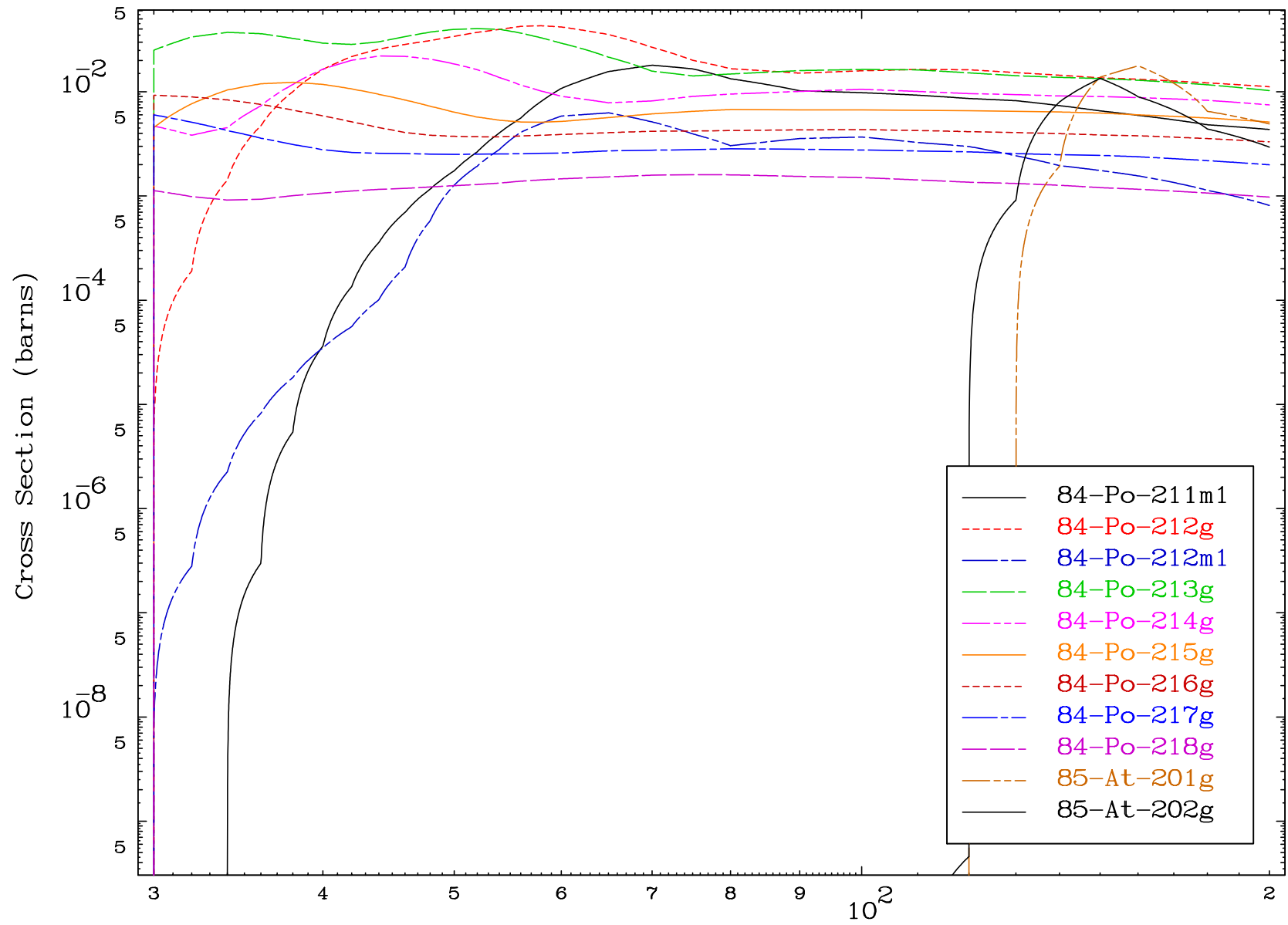


MAT 8655

(n,remainder)

86-Rn-221

### Radionuclide Production Cross Section

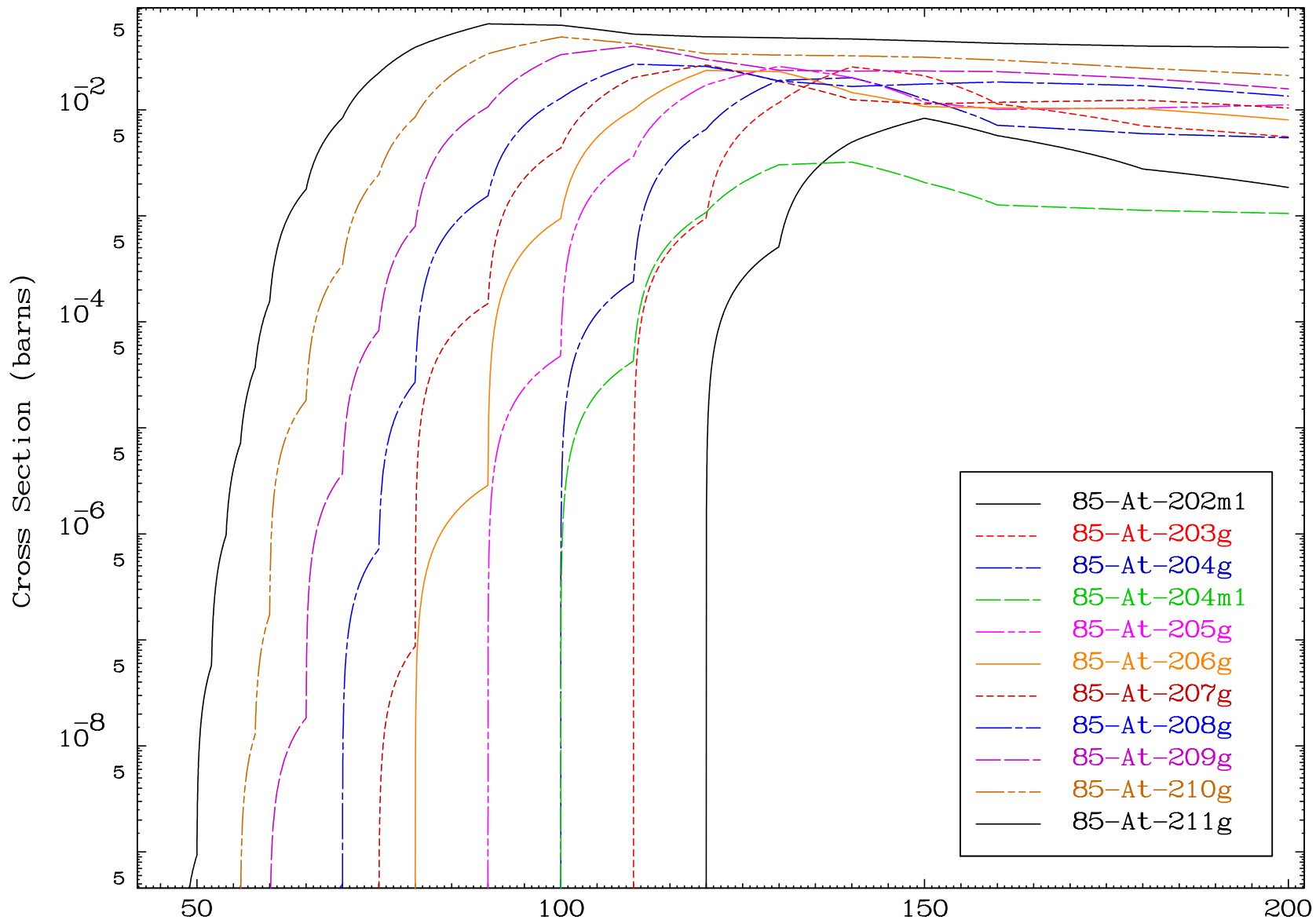


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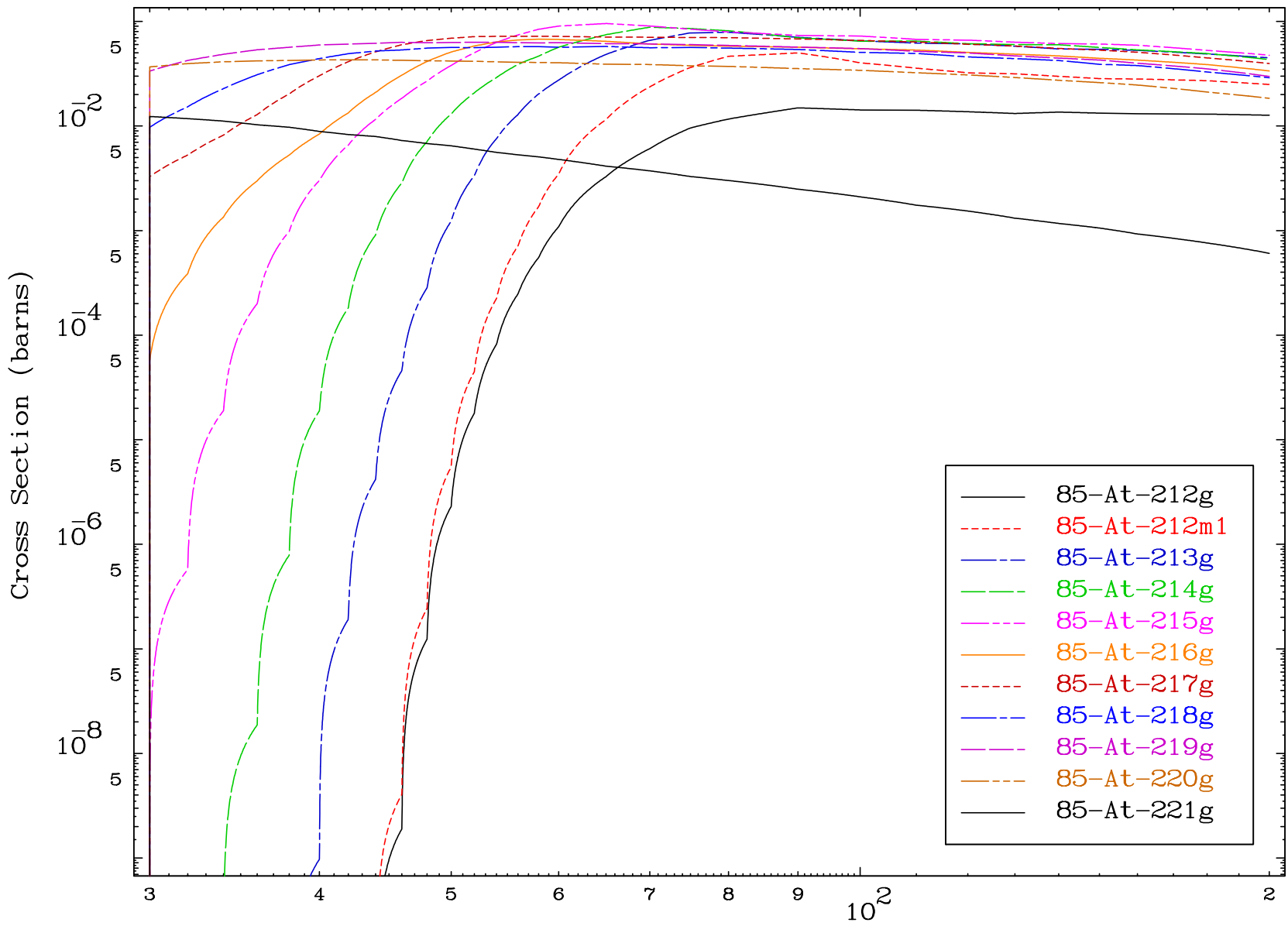
Incident Energy (MeV)

86-Rn-221

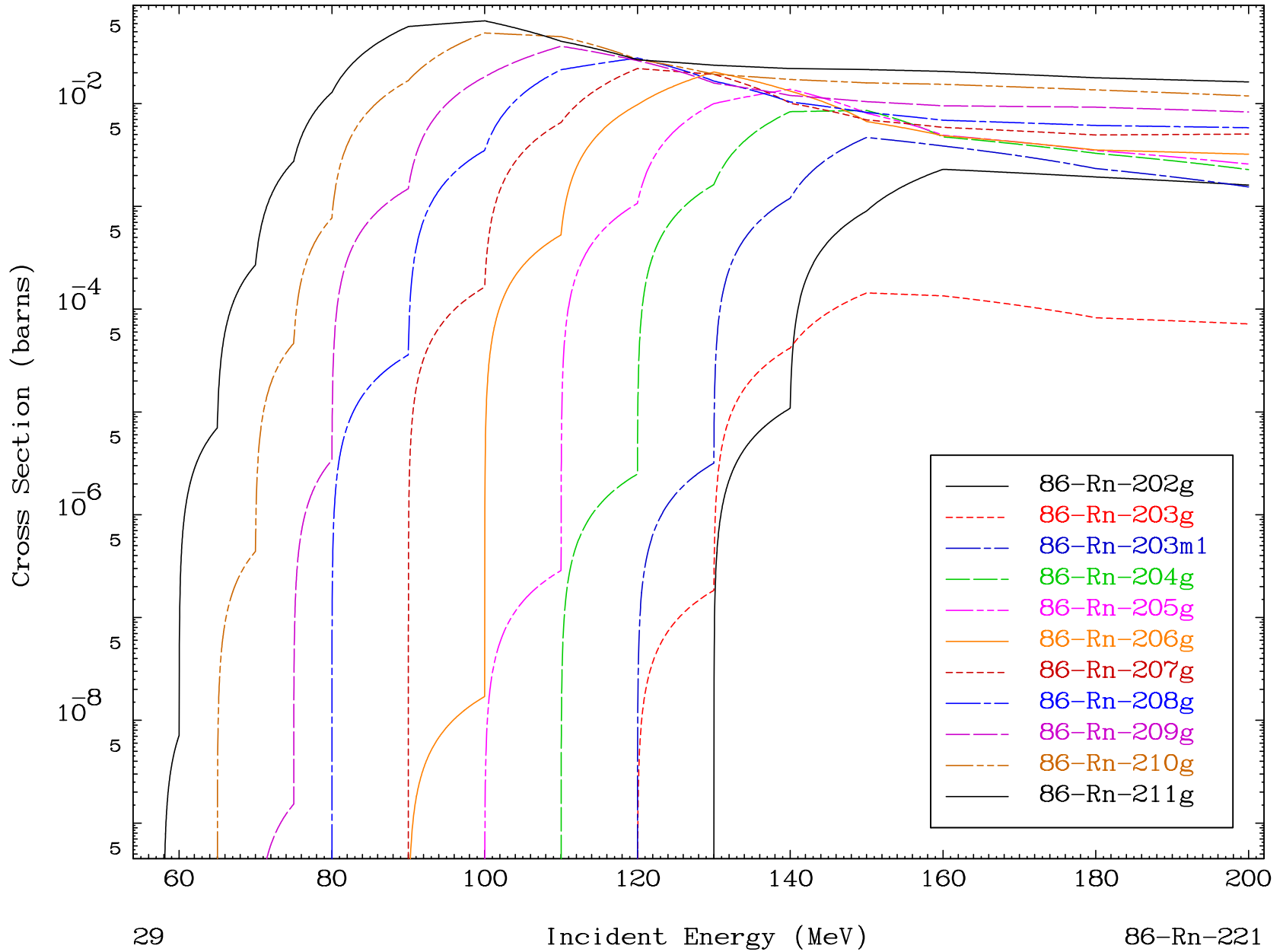
Radionuclide Production Cross Section

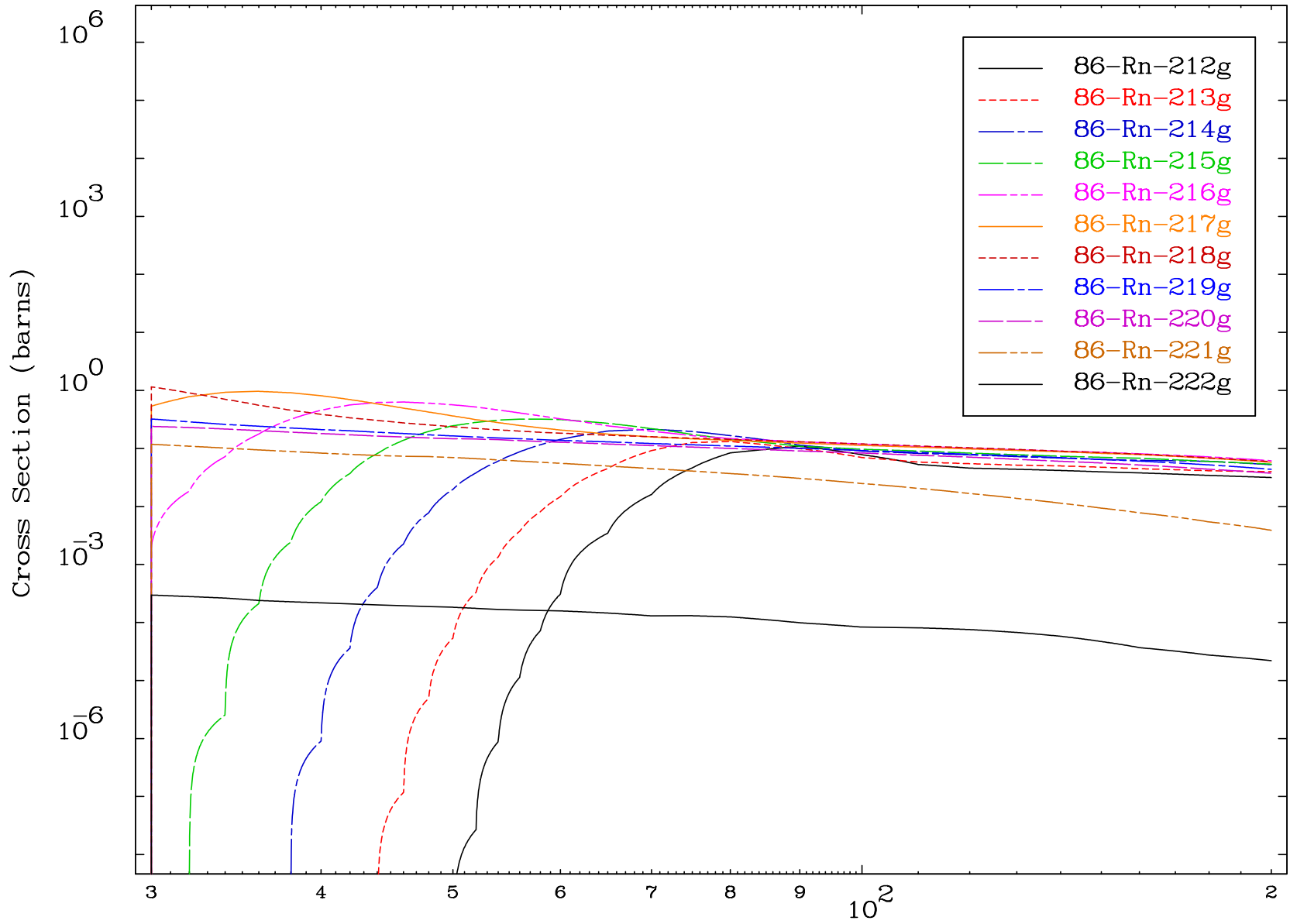


Radionuclide Production Cross Section



Radionuclide Production Cross Section



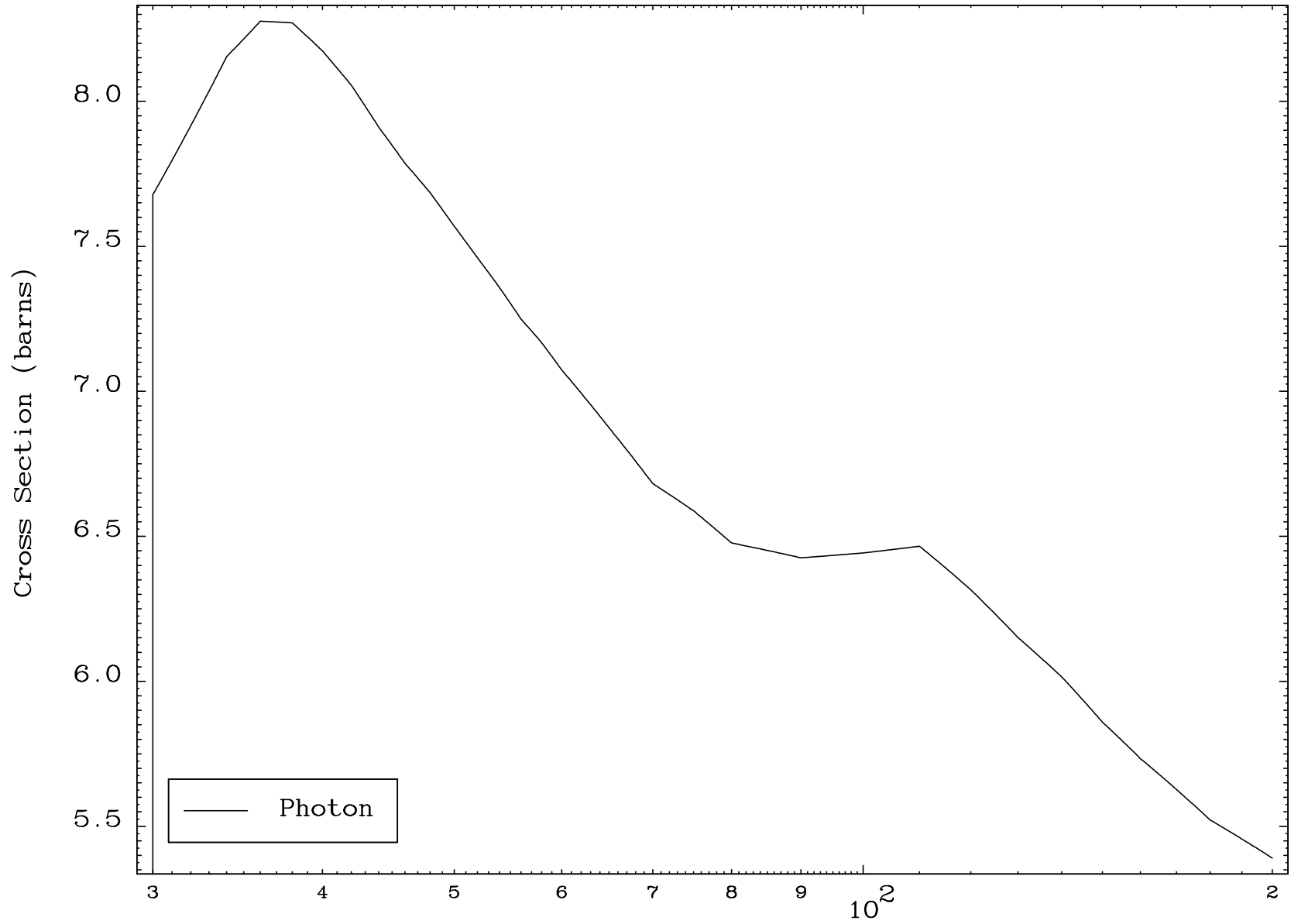


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(n,remainder)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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