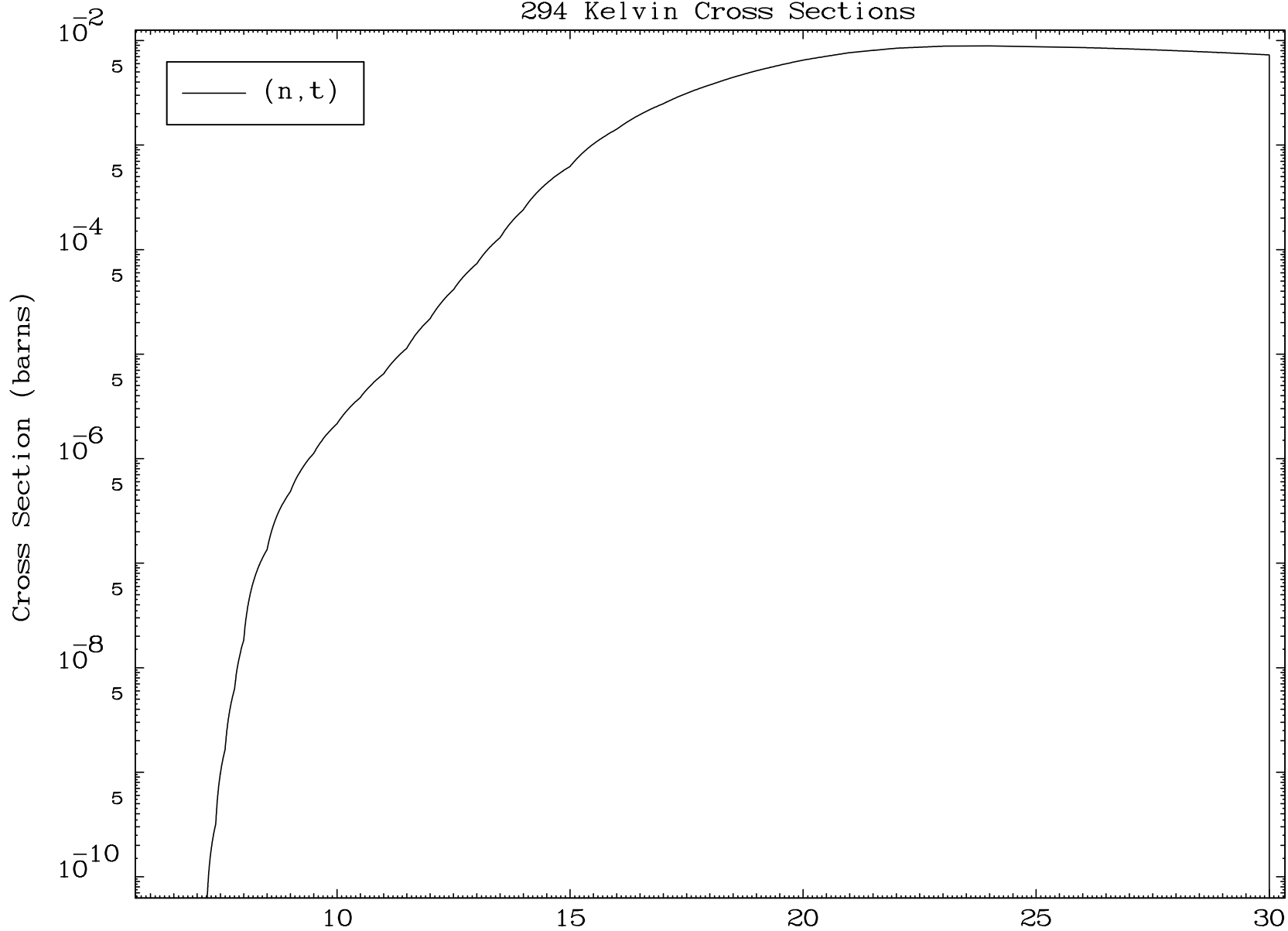


MAT 2123

(n,t) Levels  
294 Kelvin Cross Sections

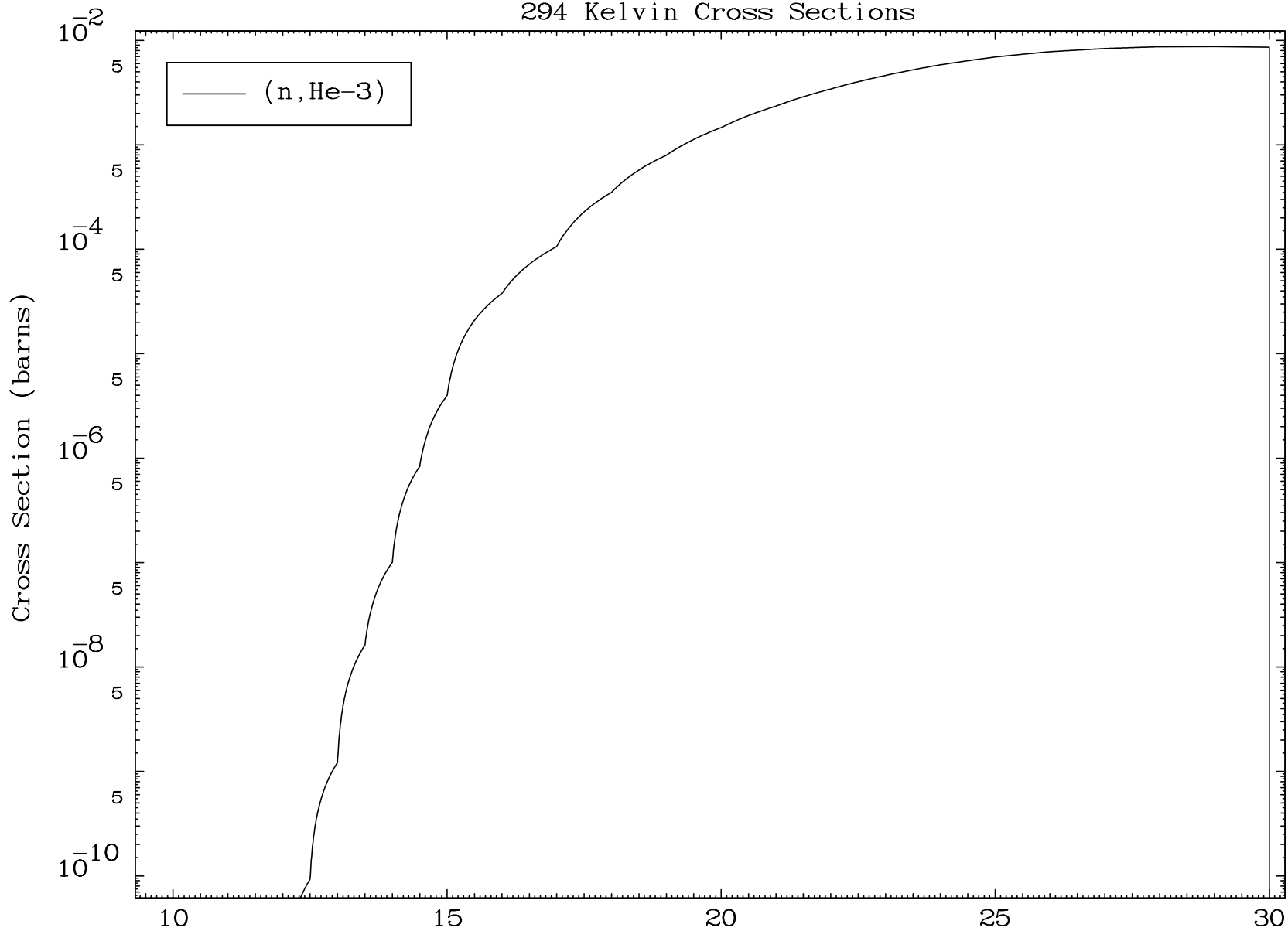
21-Sc-44

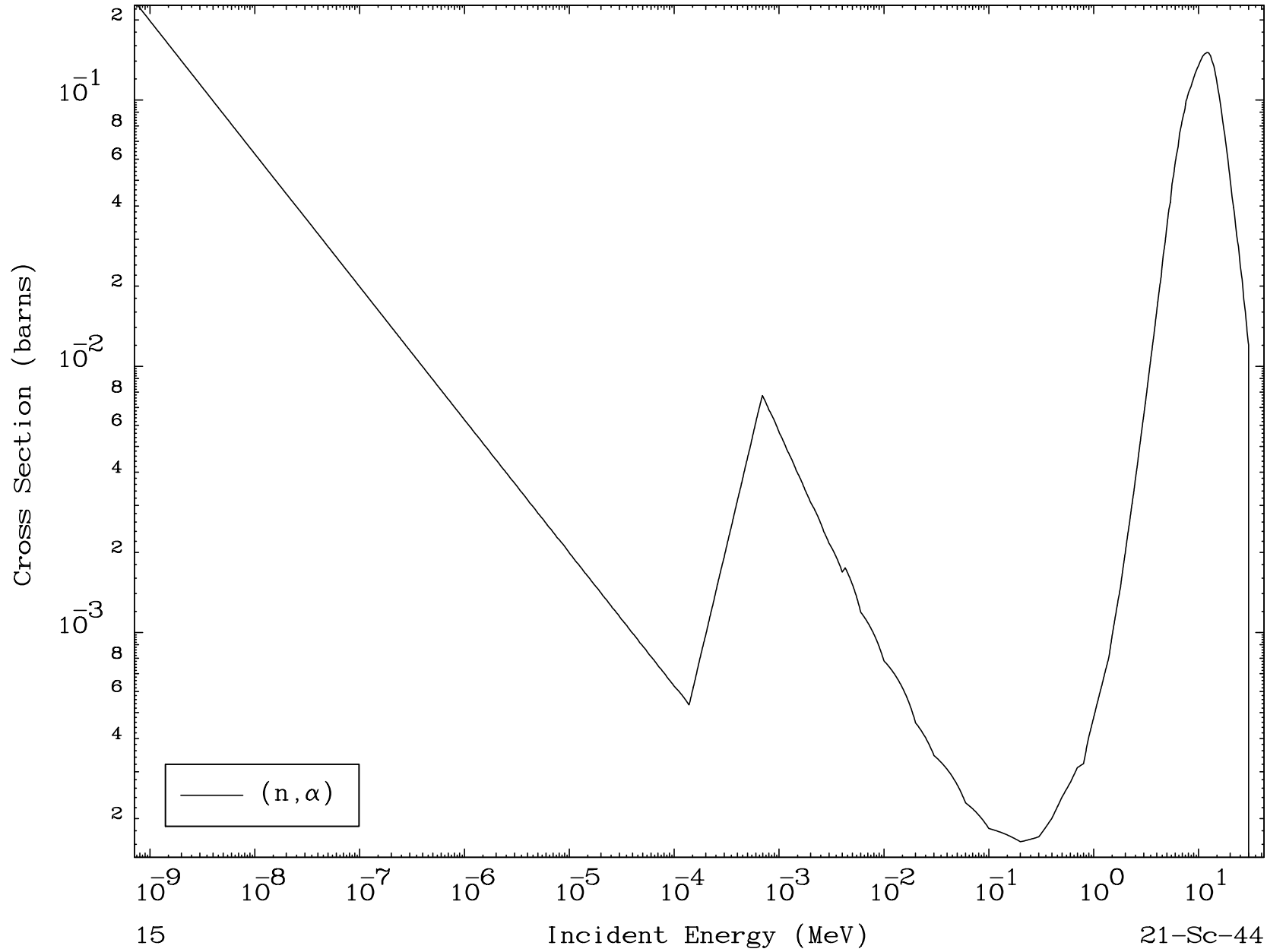


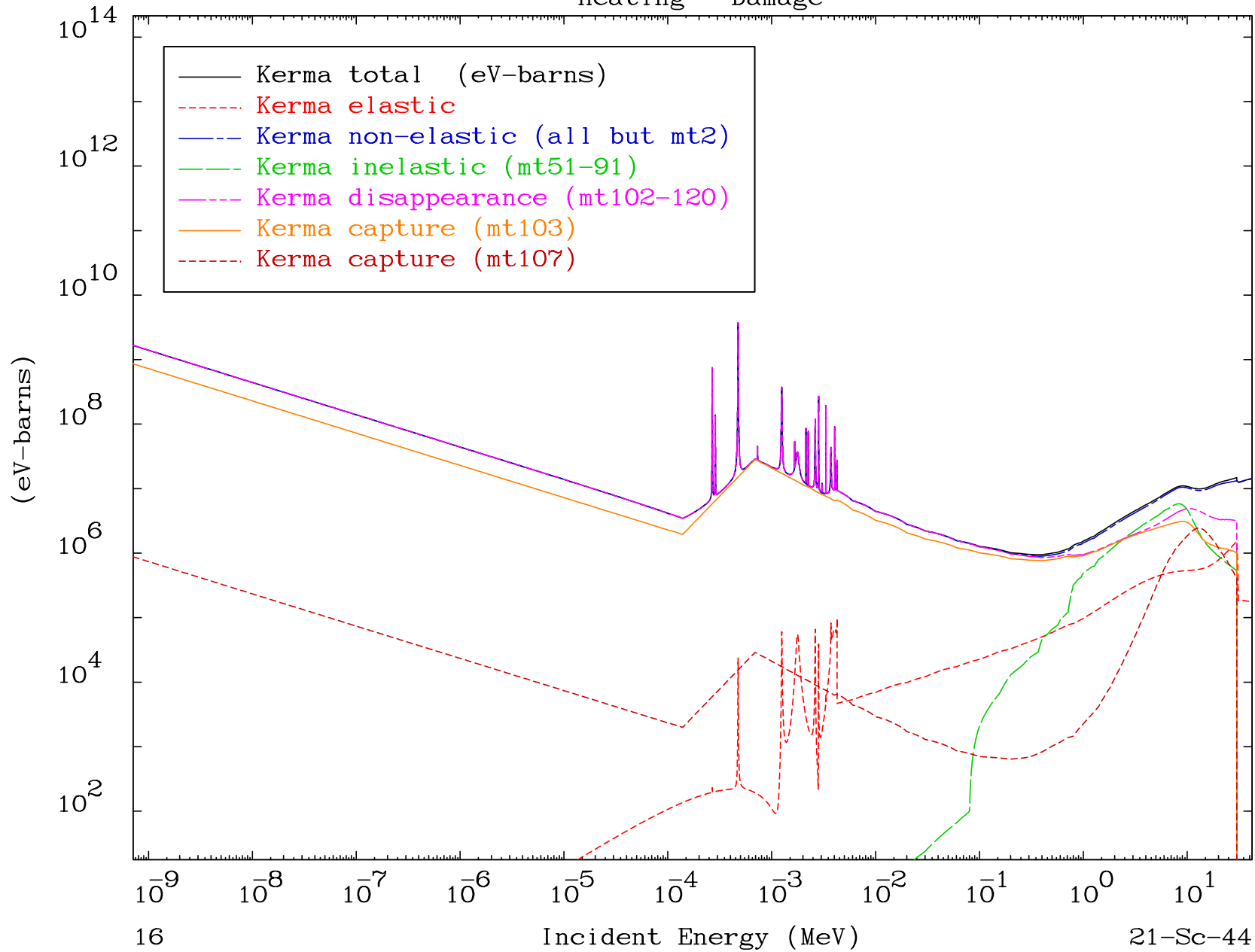
13

Incident Energy (MeV)

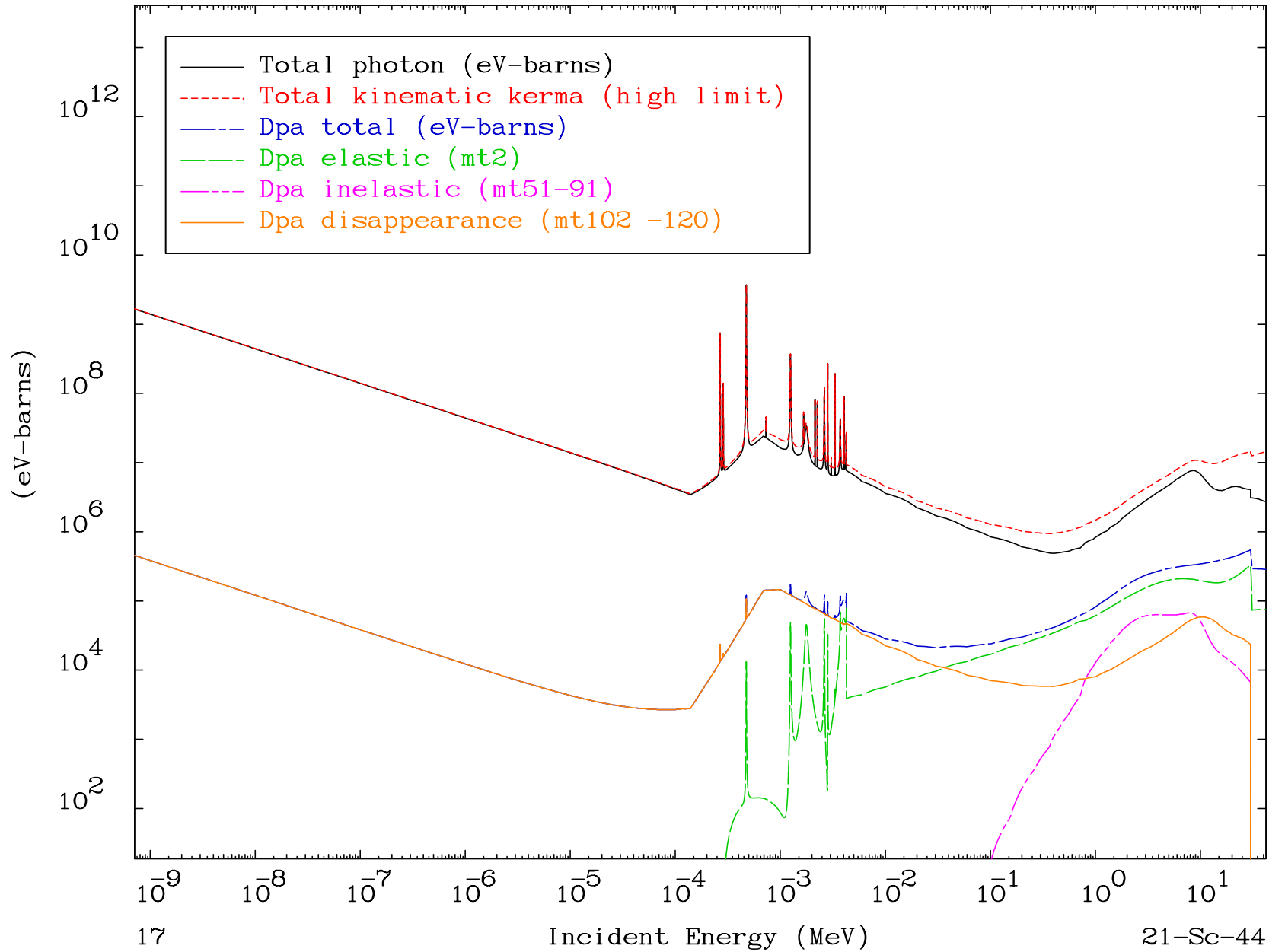
21-Sc-44

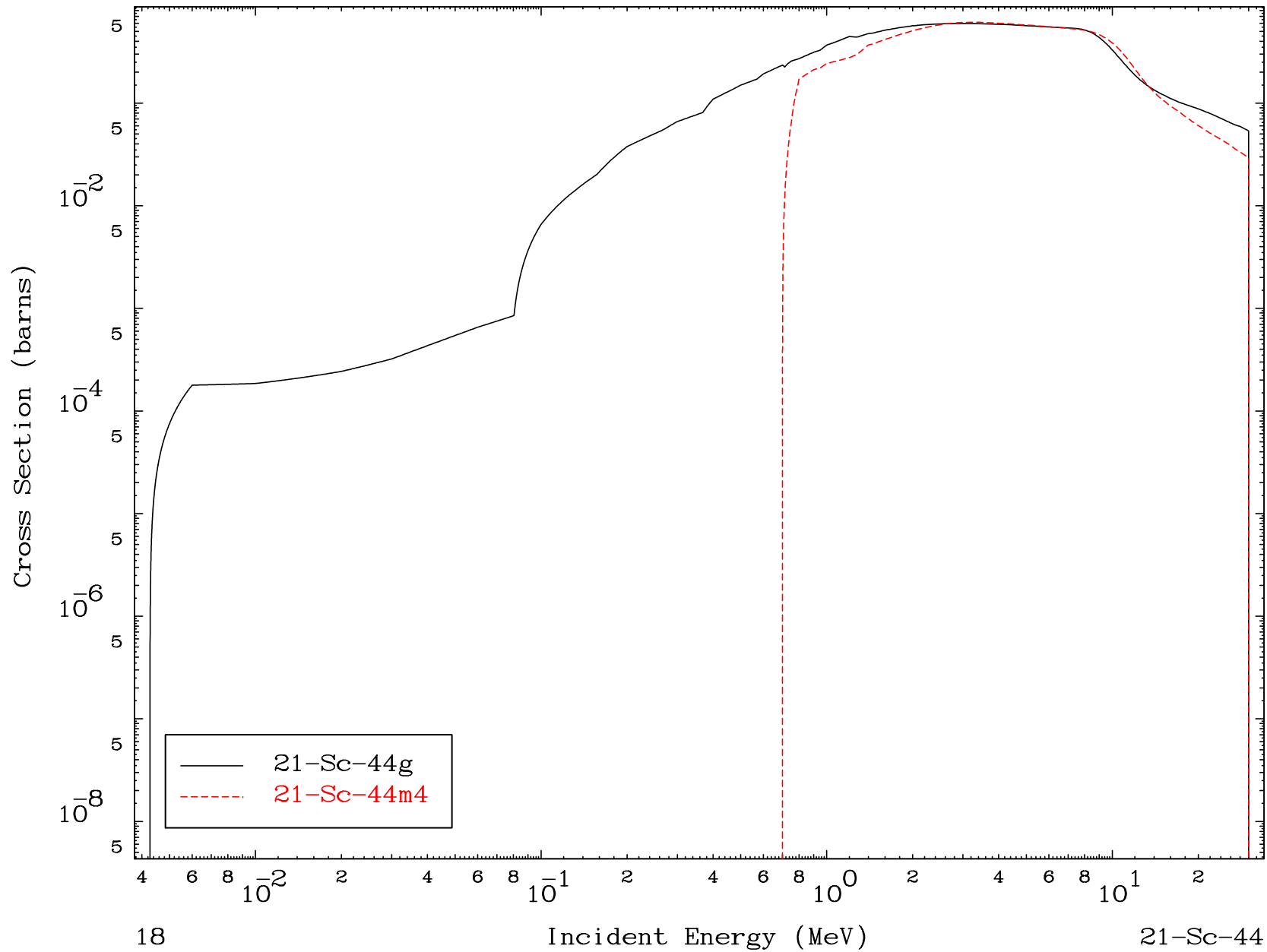




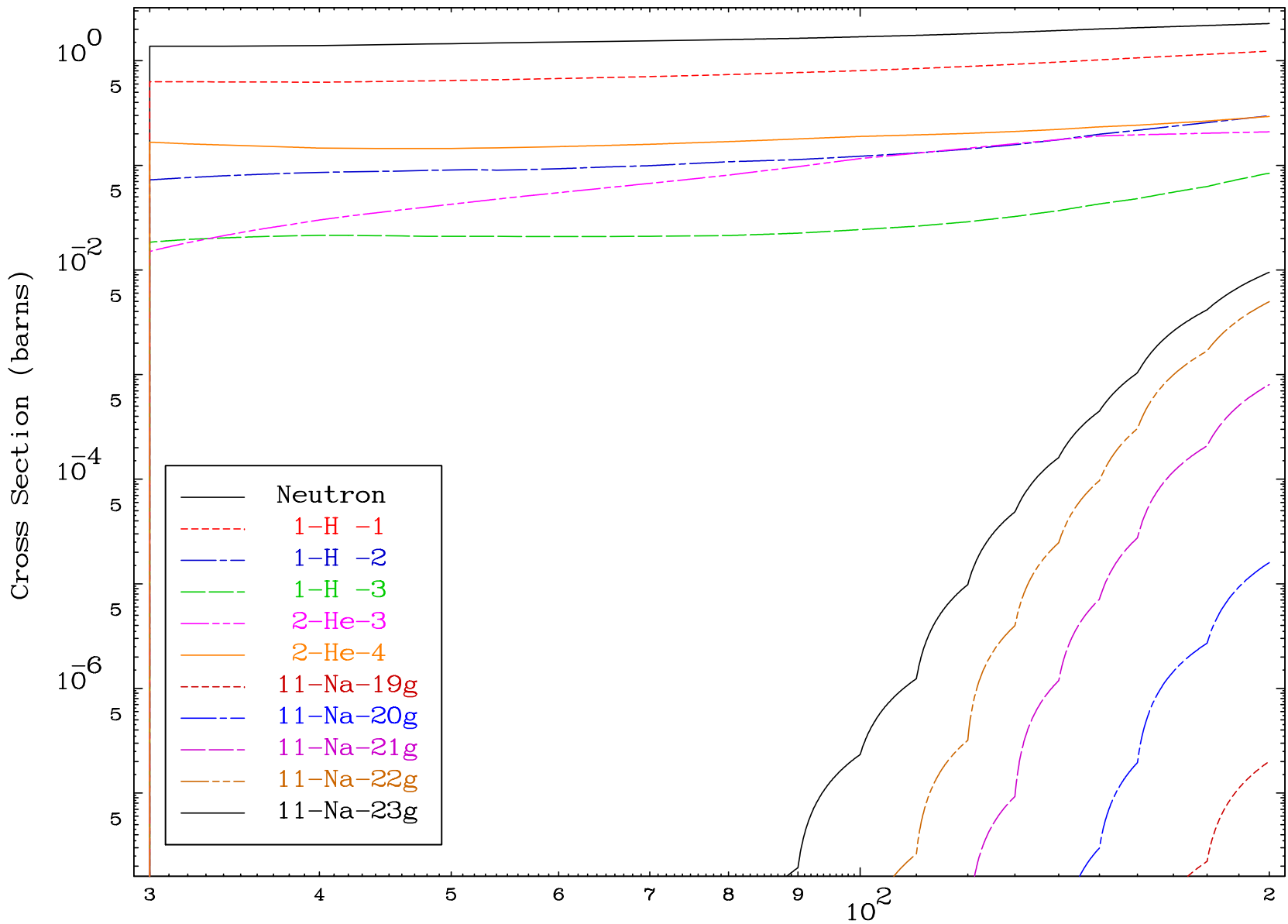




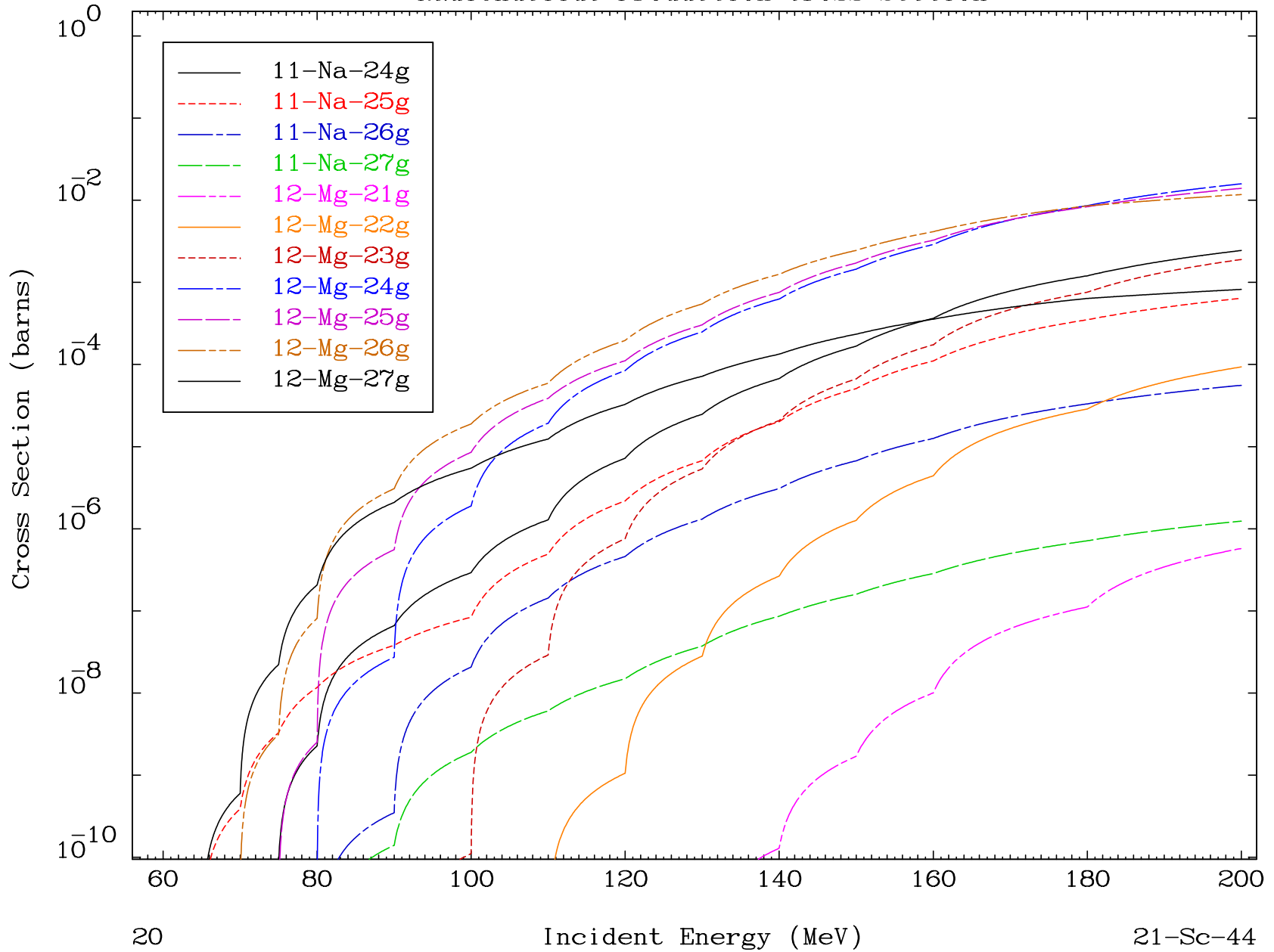




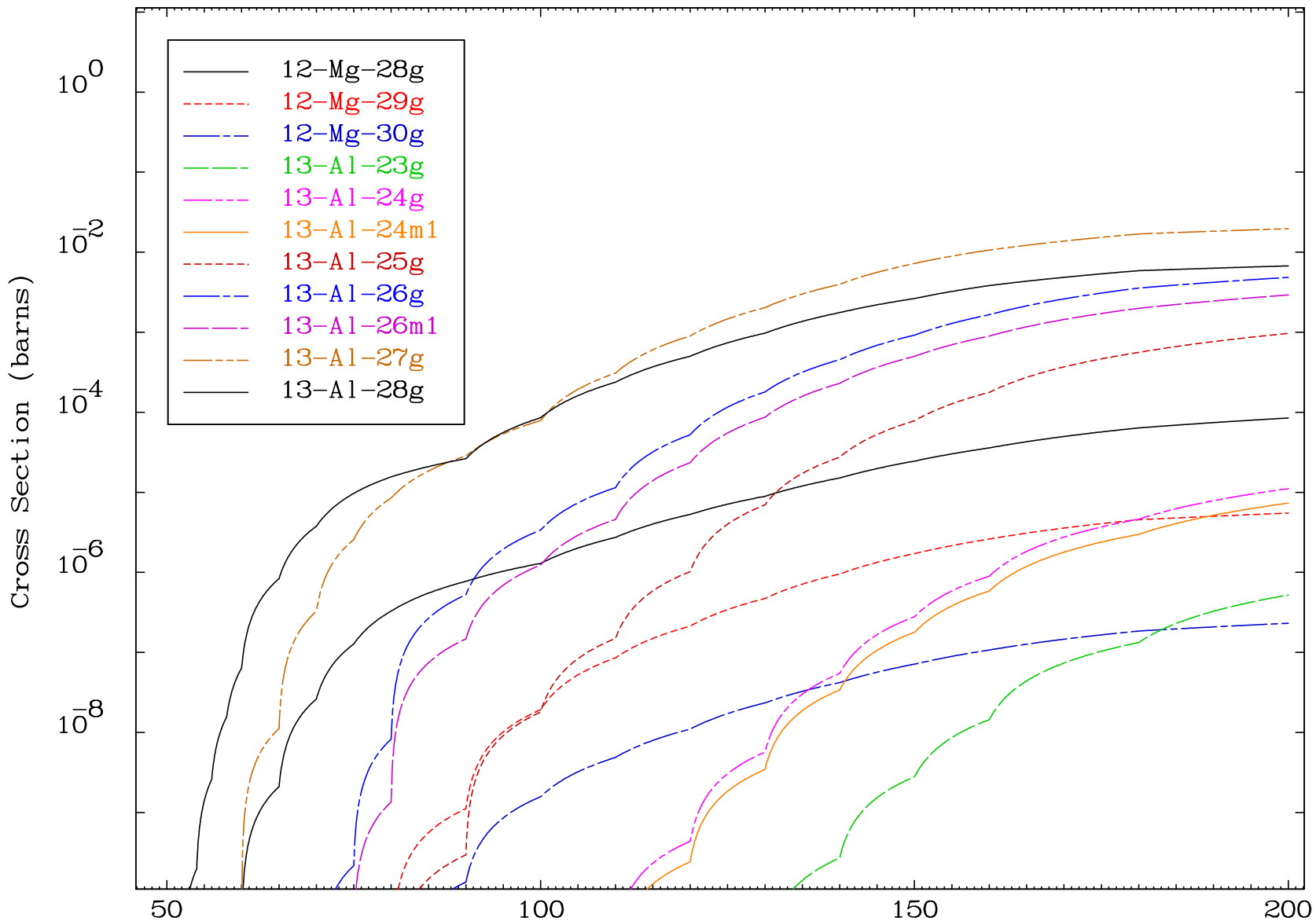
Radionuclide Production Cross Section

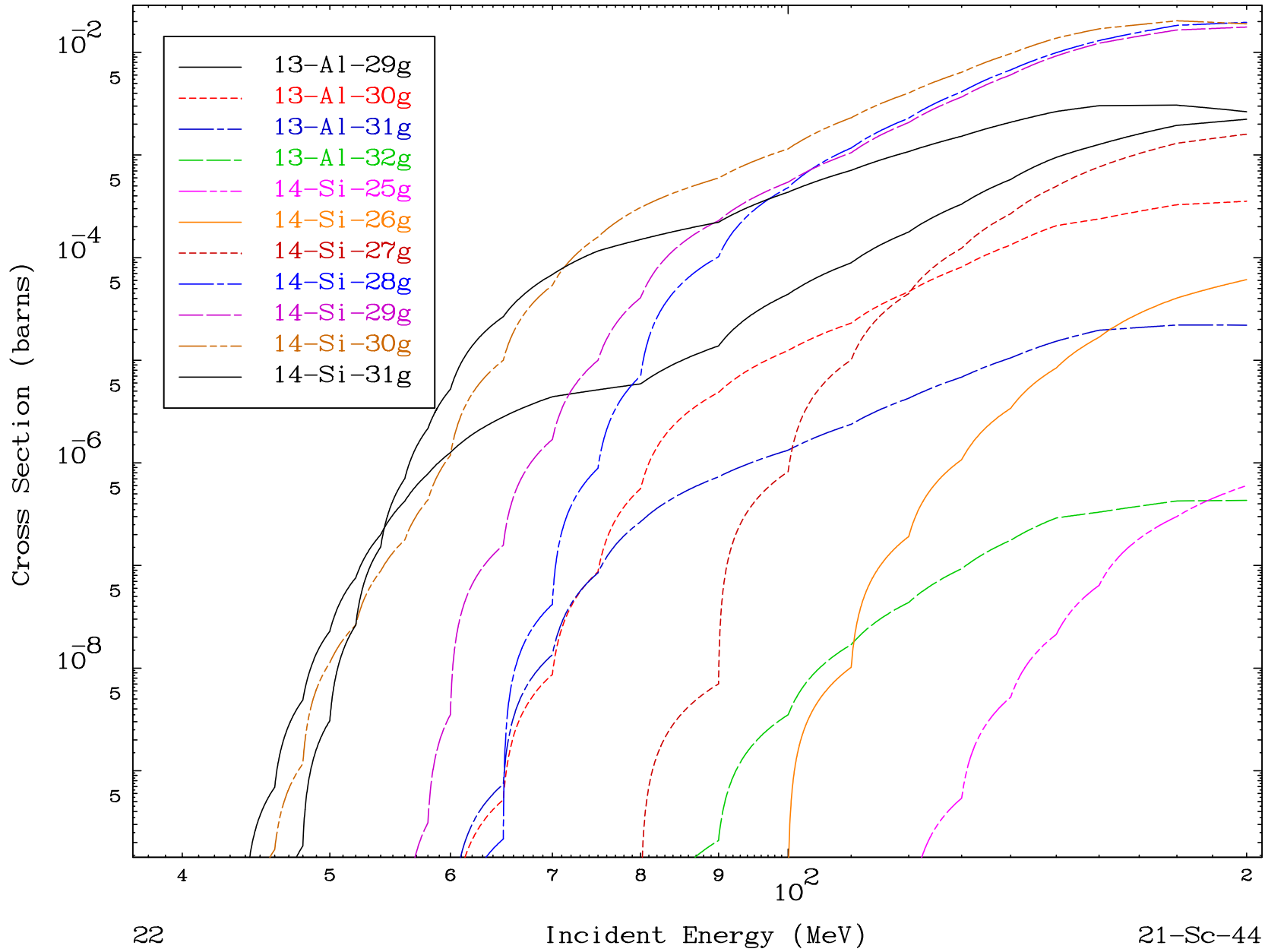


Radionuclide Production Cross Section

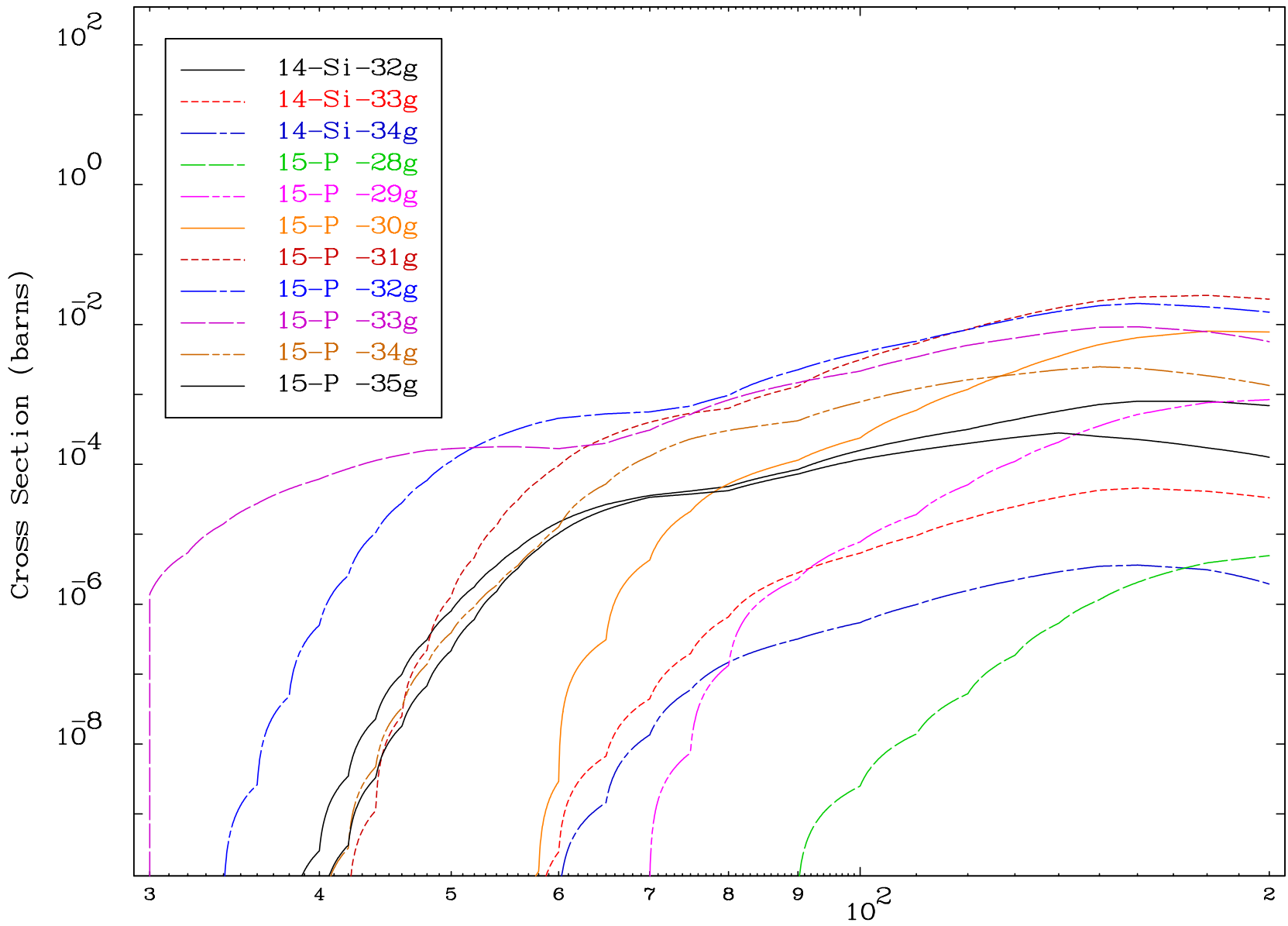


Radionuclide Production Cross Section

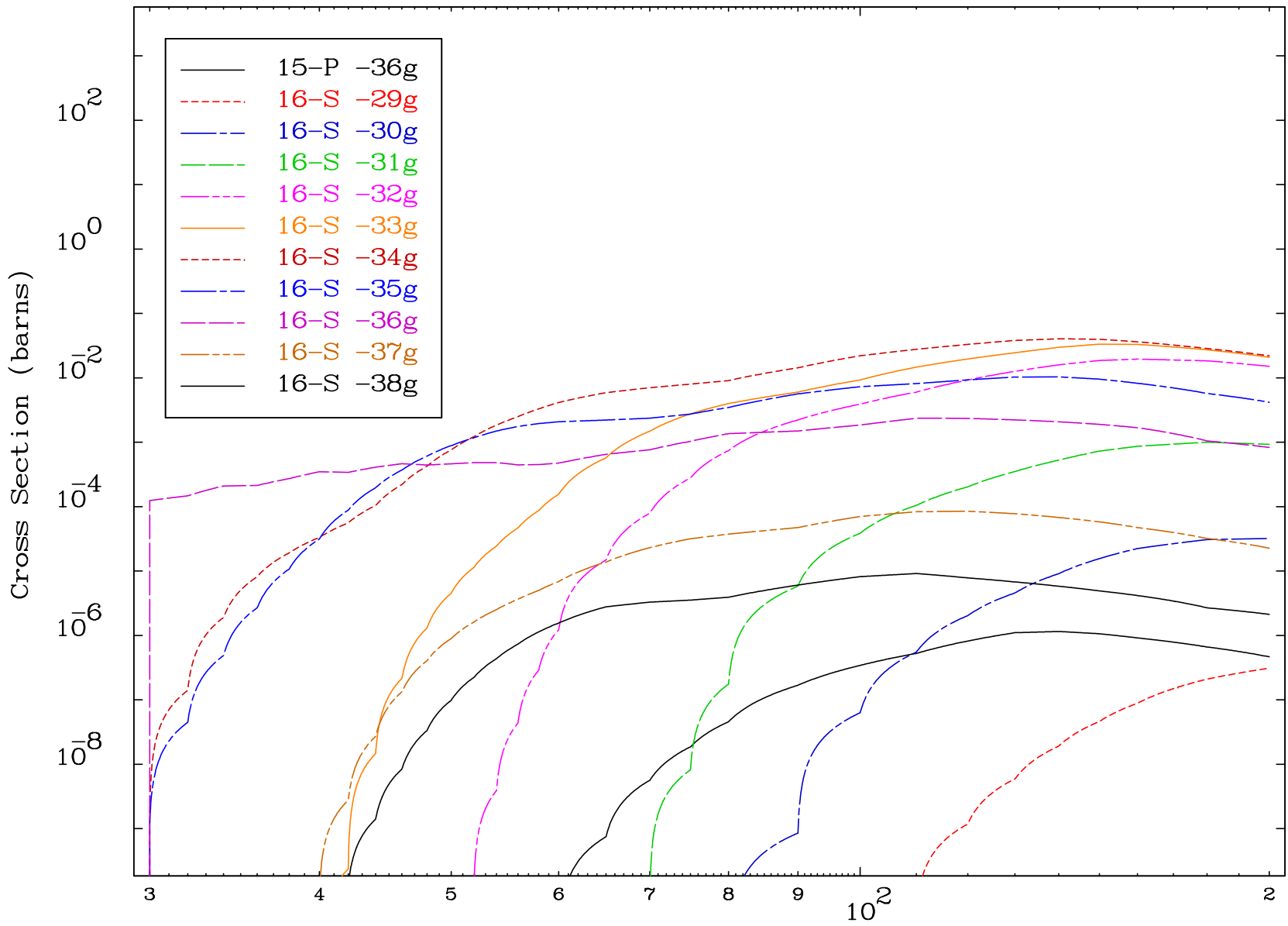




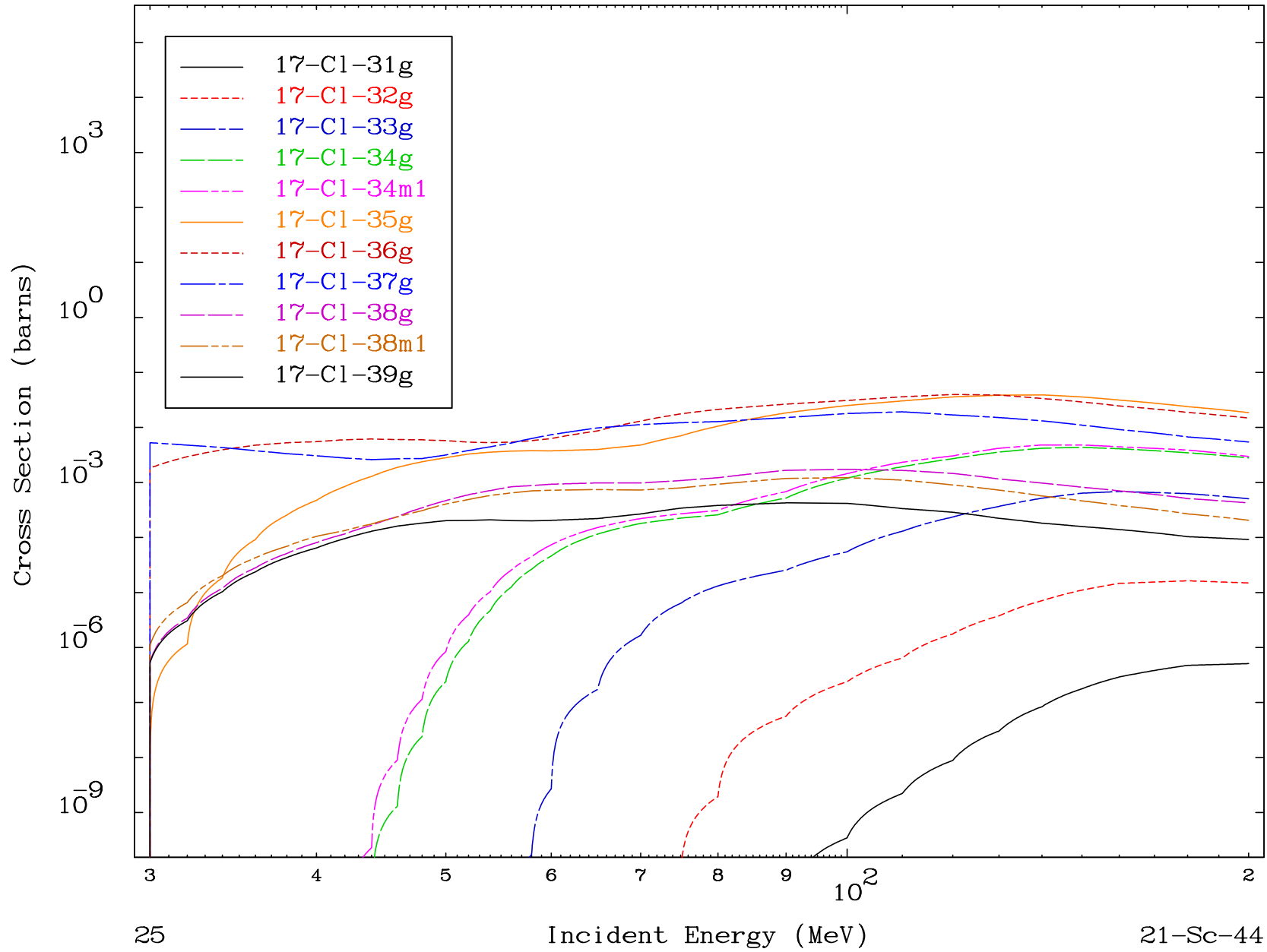
Radionuclide Production Cross Section



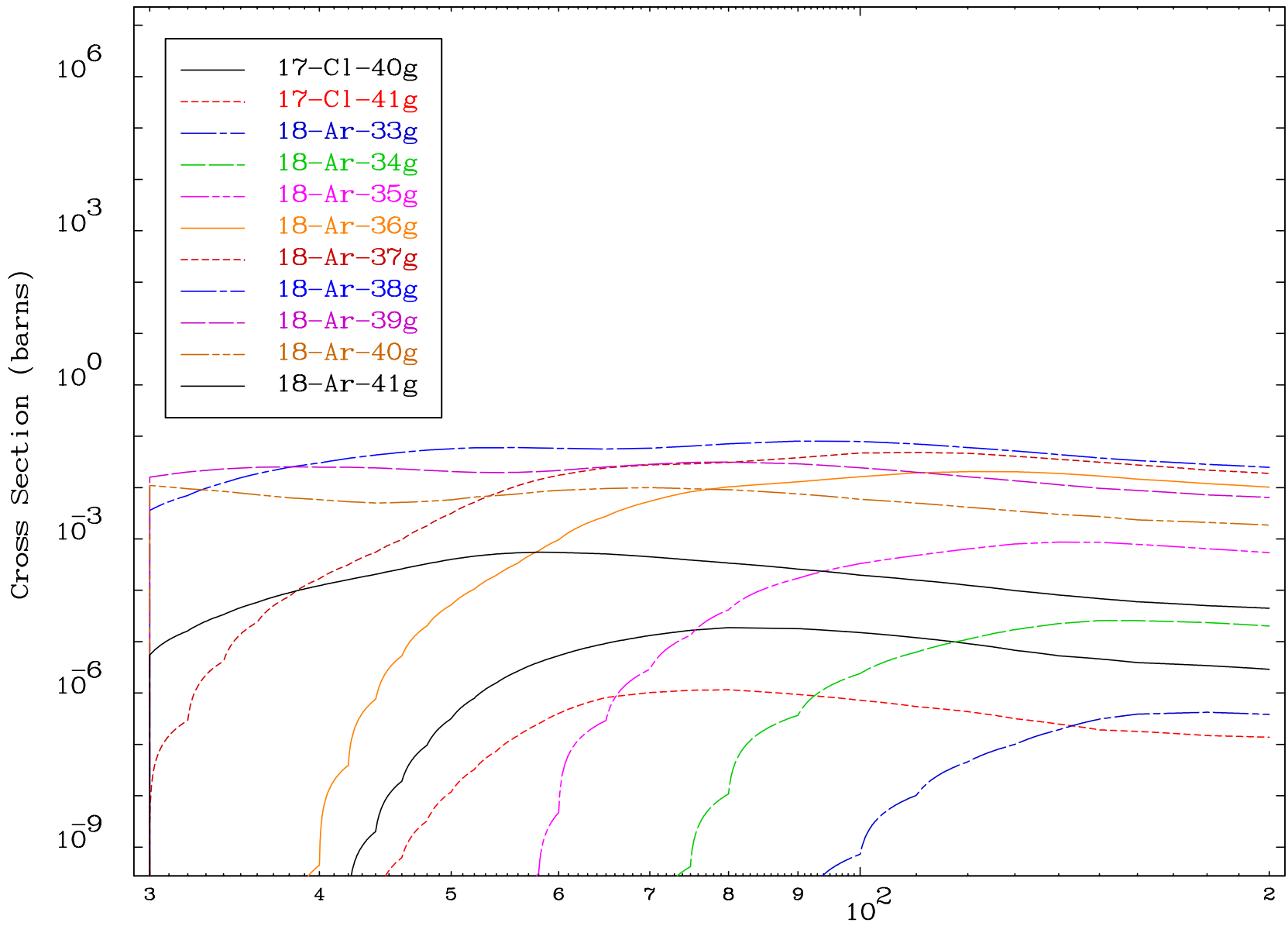
Radionuclide Production Cross Section



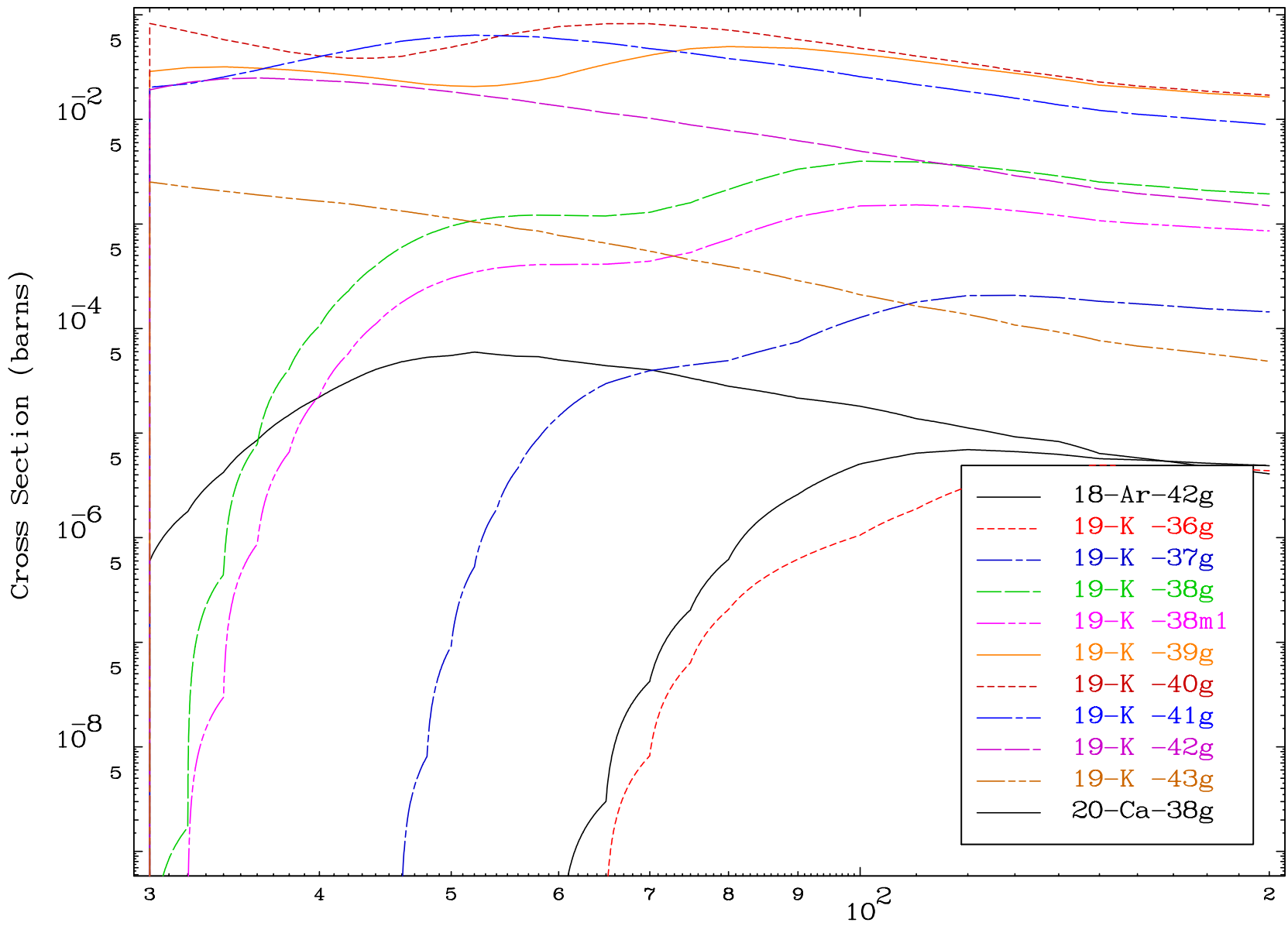


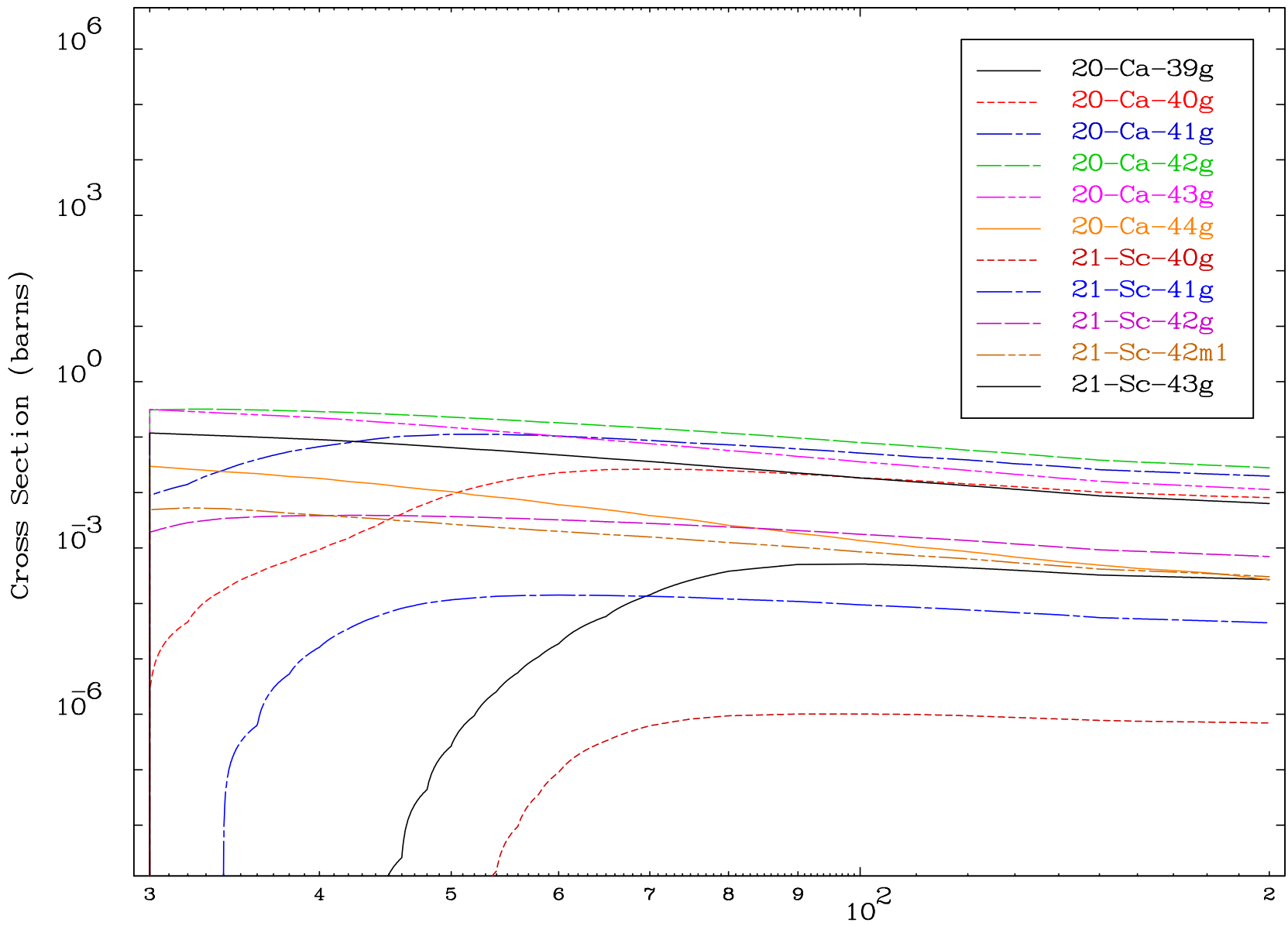


Radionuclide Production Cross Section



Radionuclide Production Cross Section



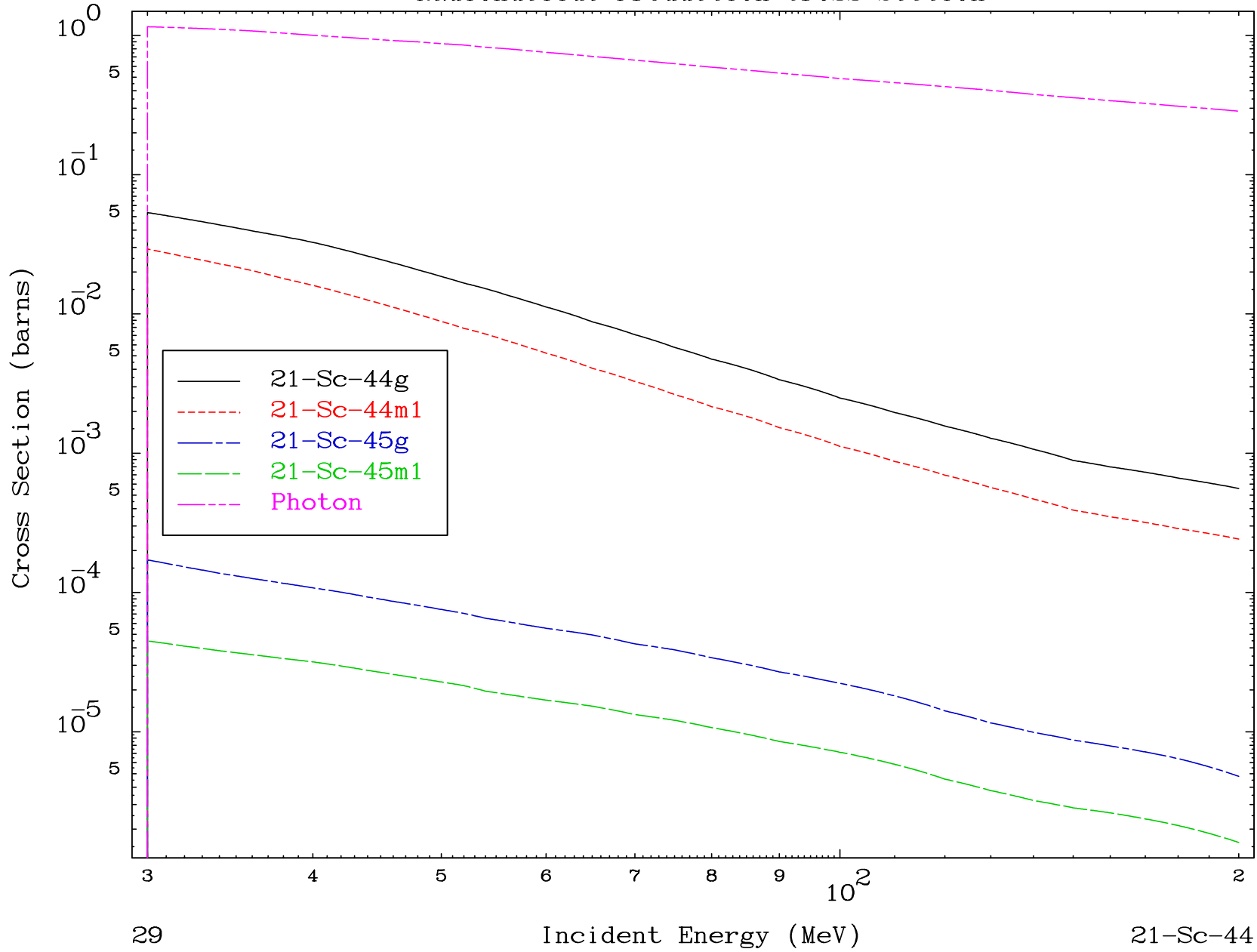


MAT 2123

(n,remainder)

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### Radionuclide Production Cross Section



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Incident Energy (MeV)

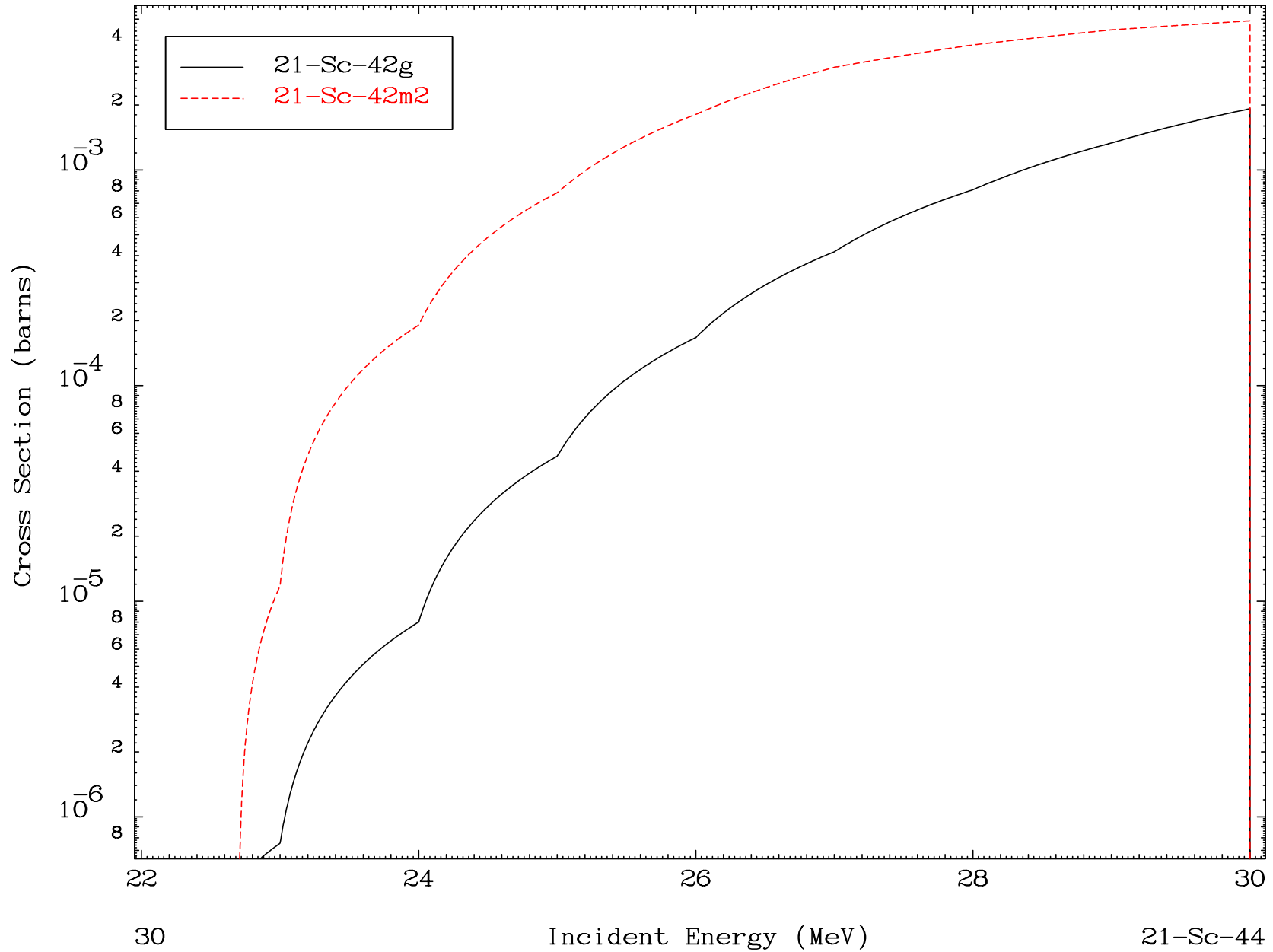
21-Sc-44

MAT 2123

(n,3n)

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Radionuclide Production Cross Section



MAT 2123

(n,  $\gamma$ )  
Radionuclide Production Cross Section

21-Sc-44

