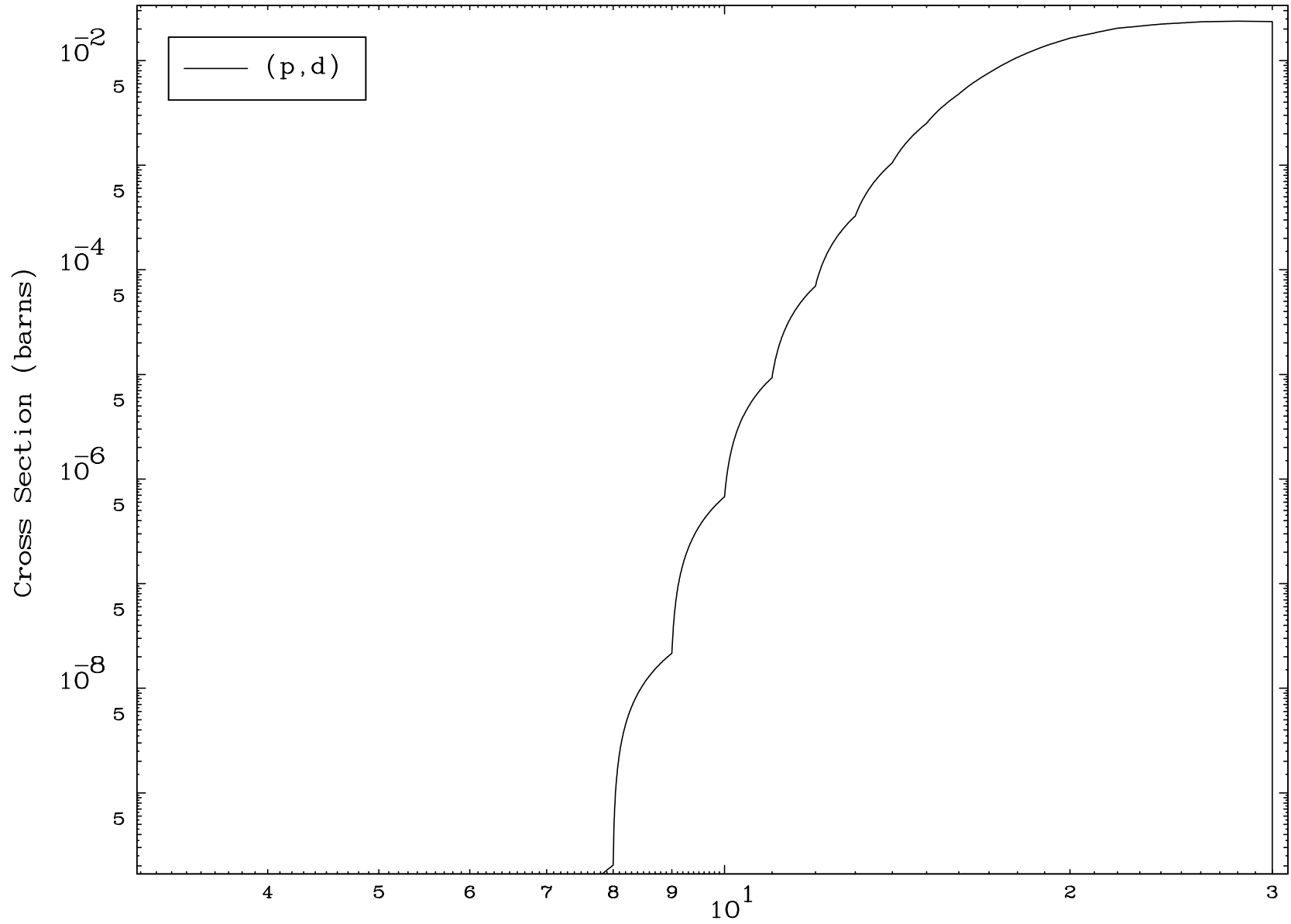


MAT 8591

(p,d) Levels  
0 Kelvin Cross Sections

85-At-225

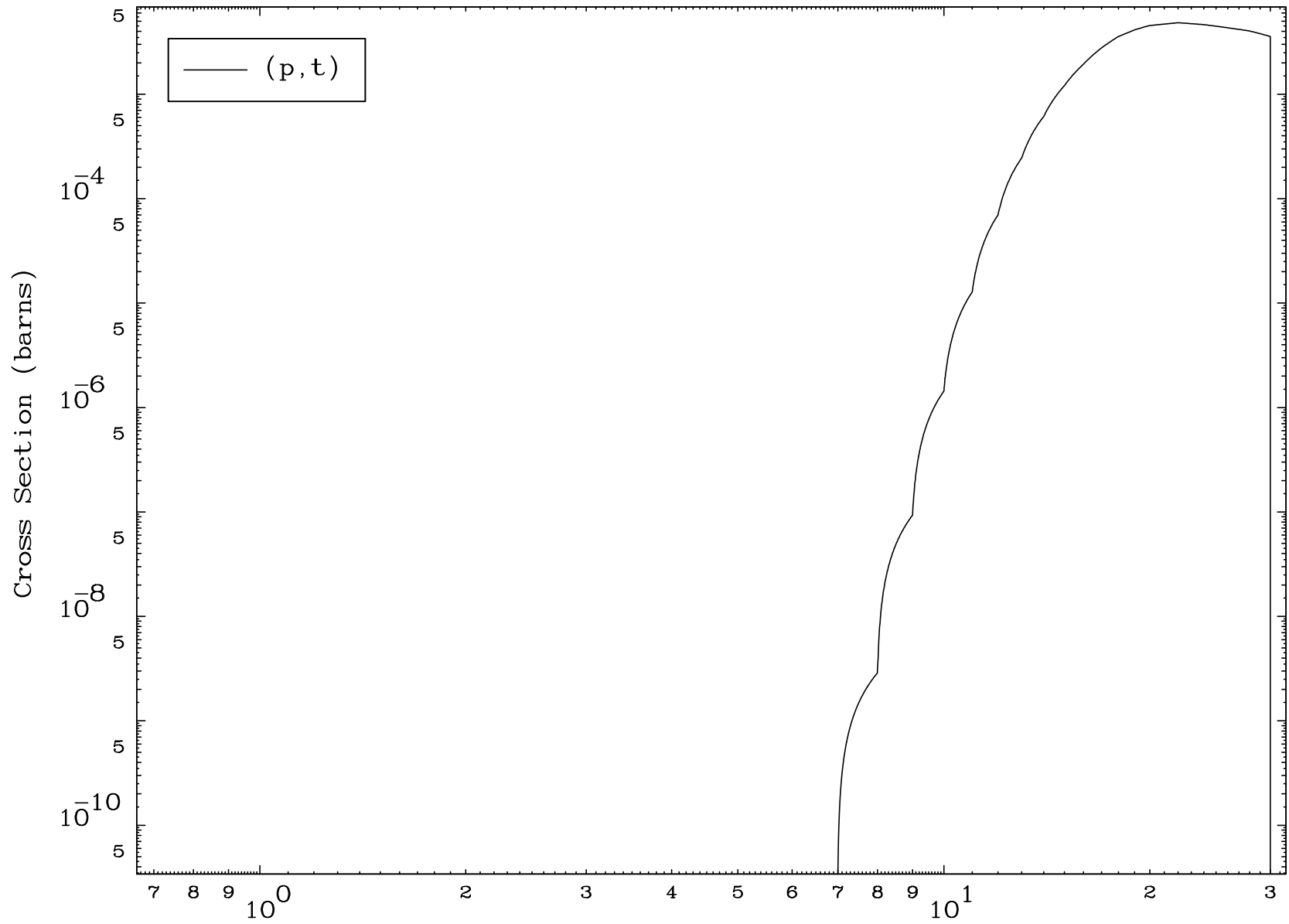


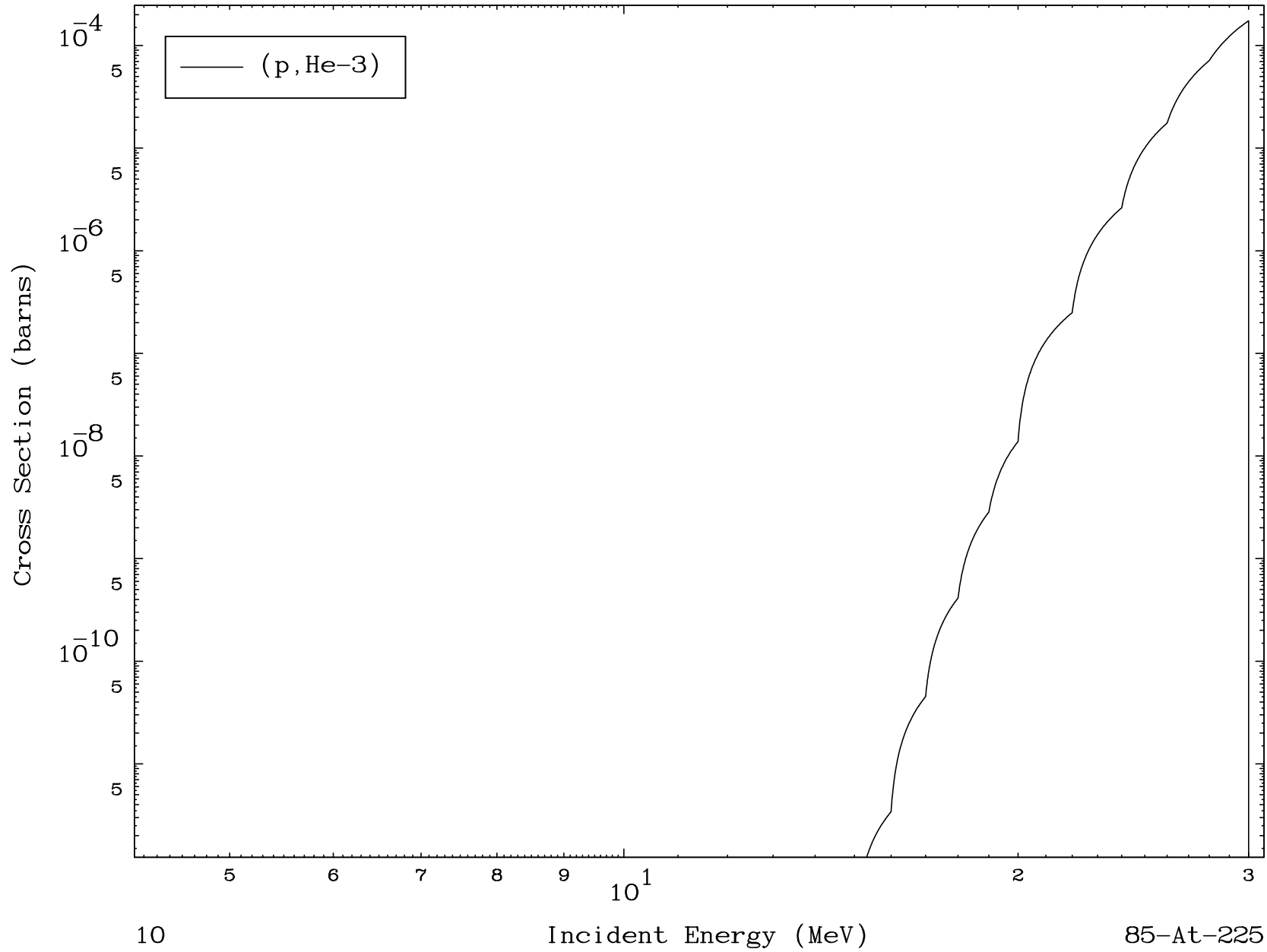
8

Incident Energy (MeV)

85-At-225



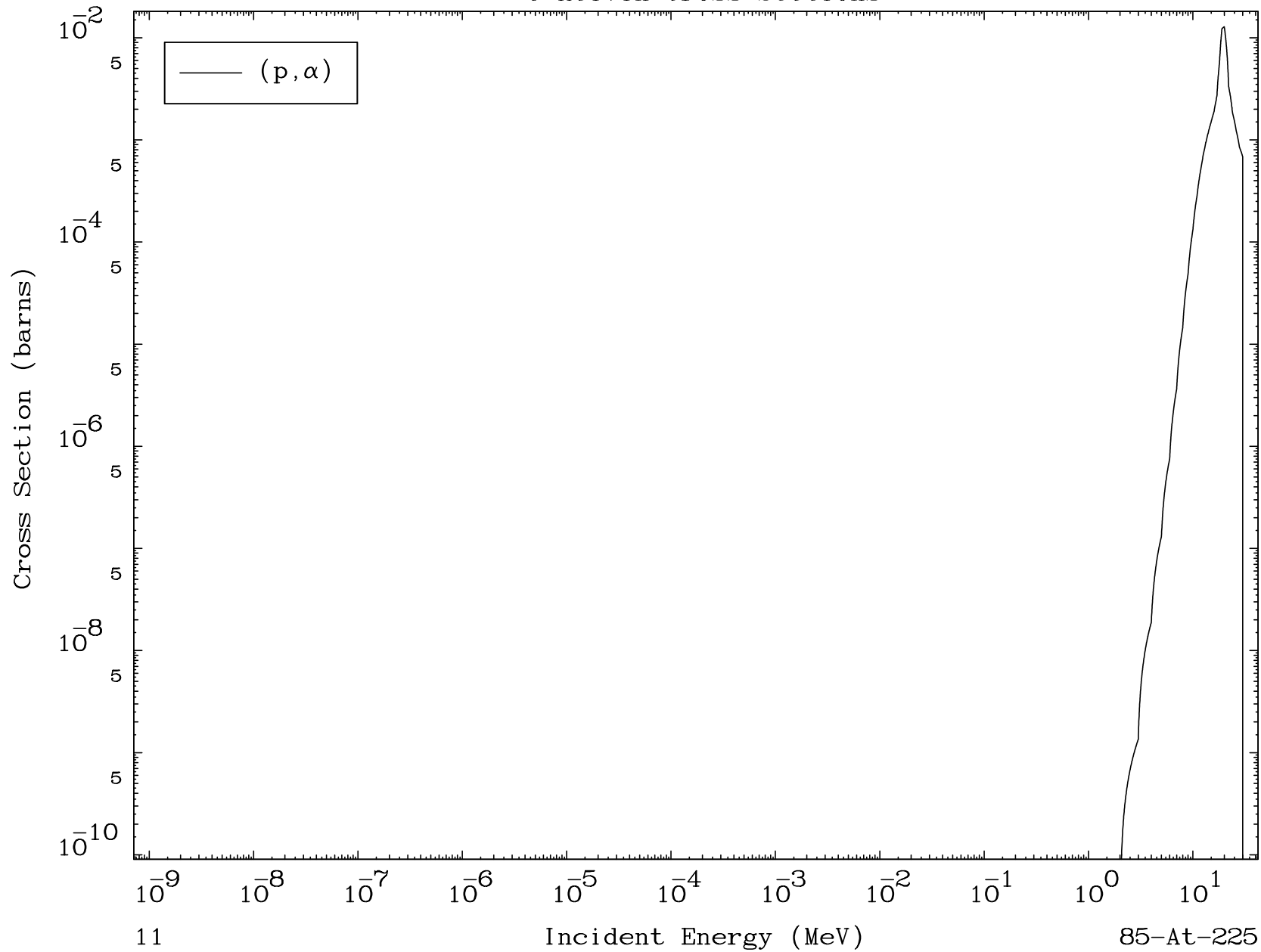




MAT 8591

(p, $\alpha$ ) Levels  
0 Kelvin Cross Sections

85-At-225

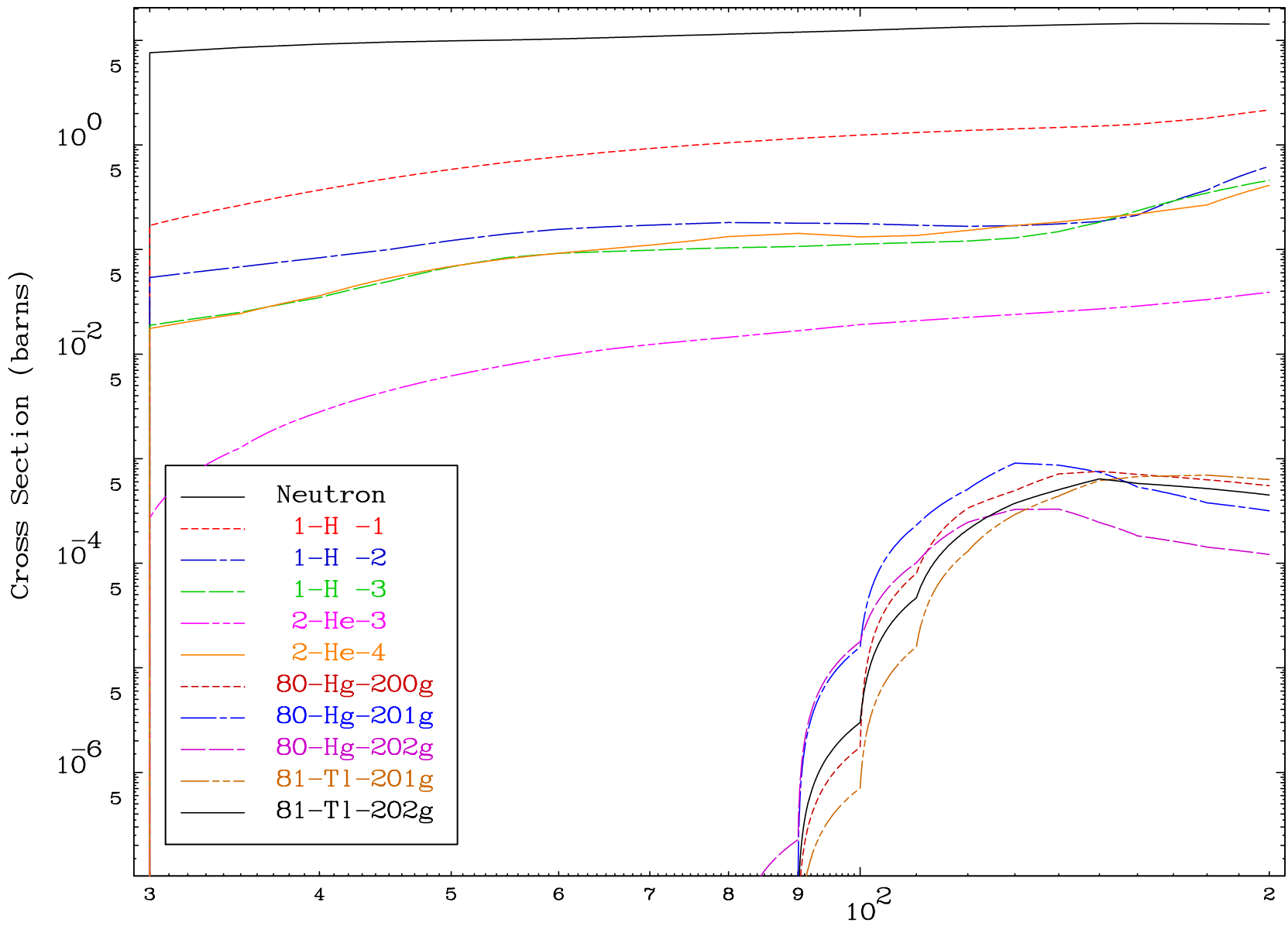


11

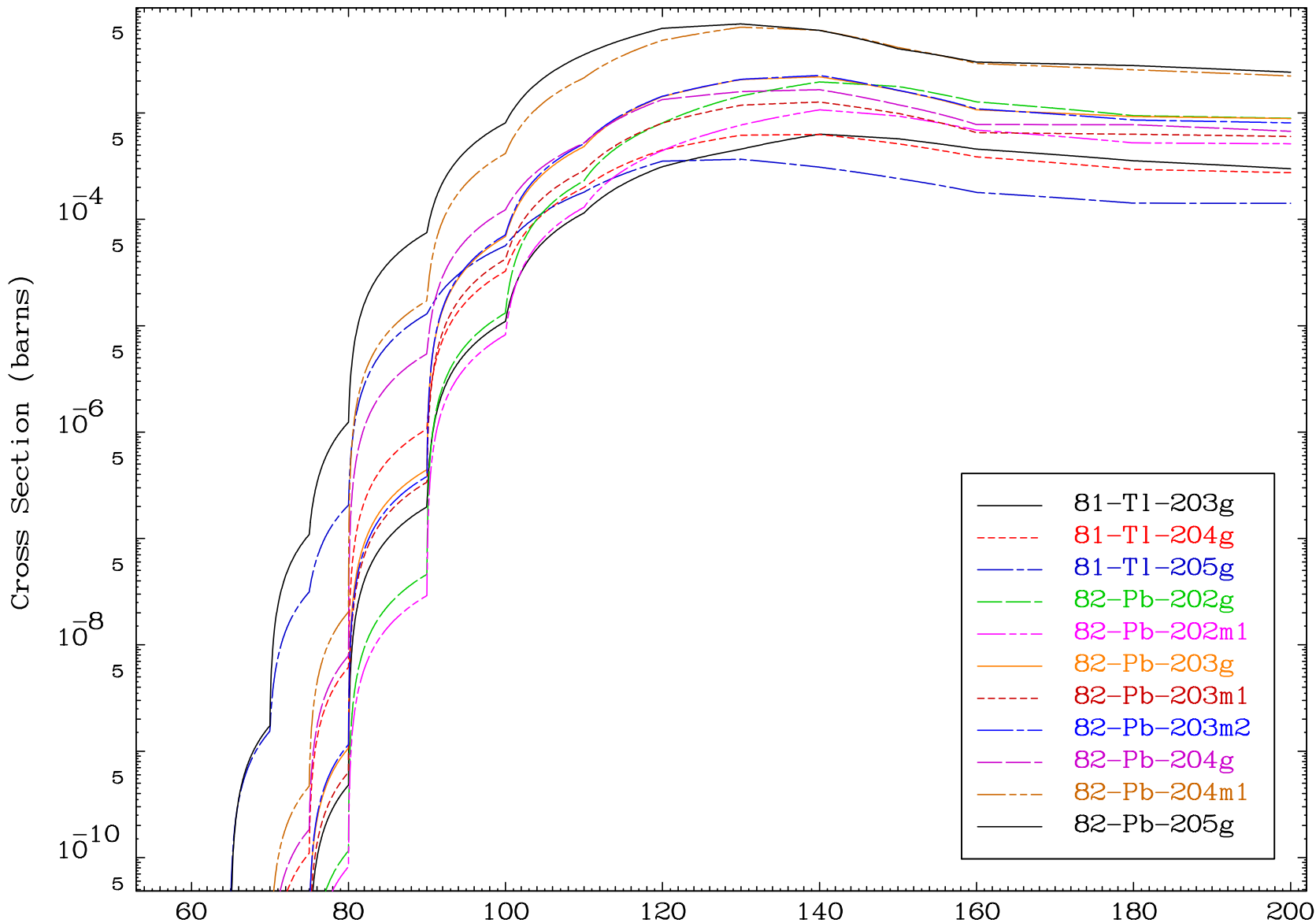
Incident Energy (MeV)

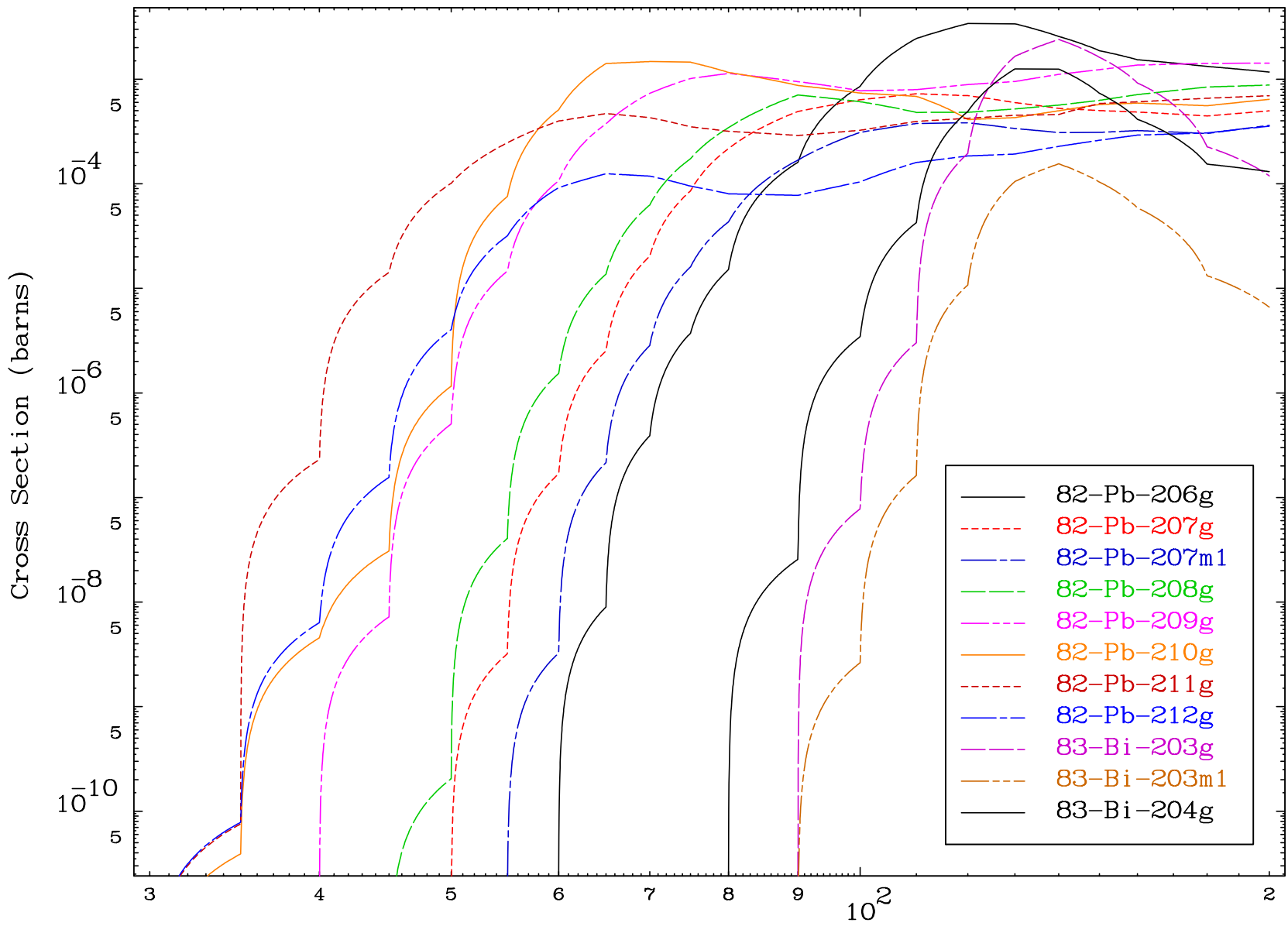
85-At-225

Radionuclide Production Cross Section

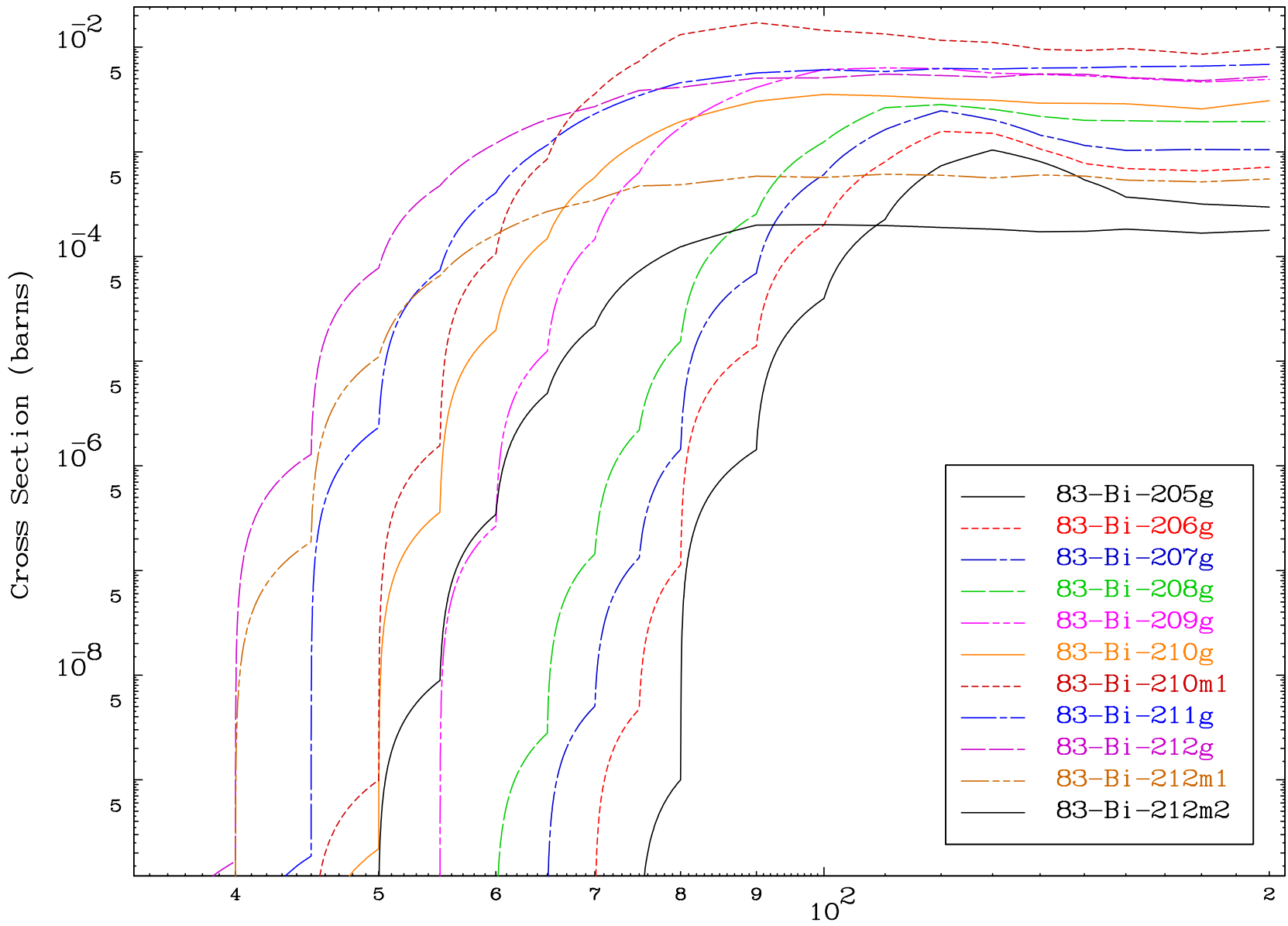


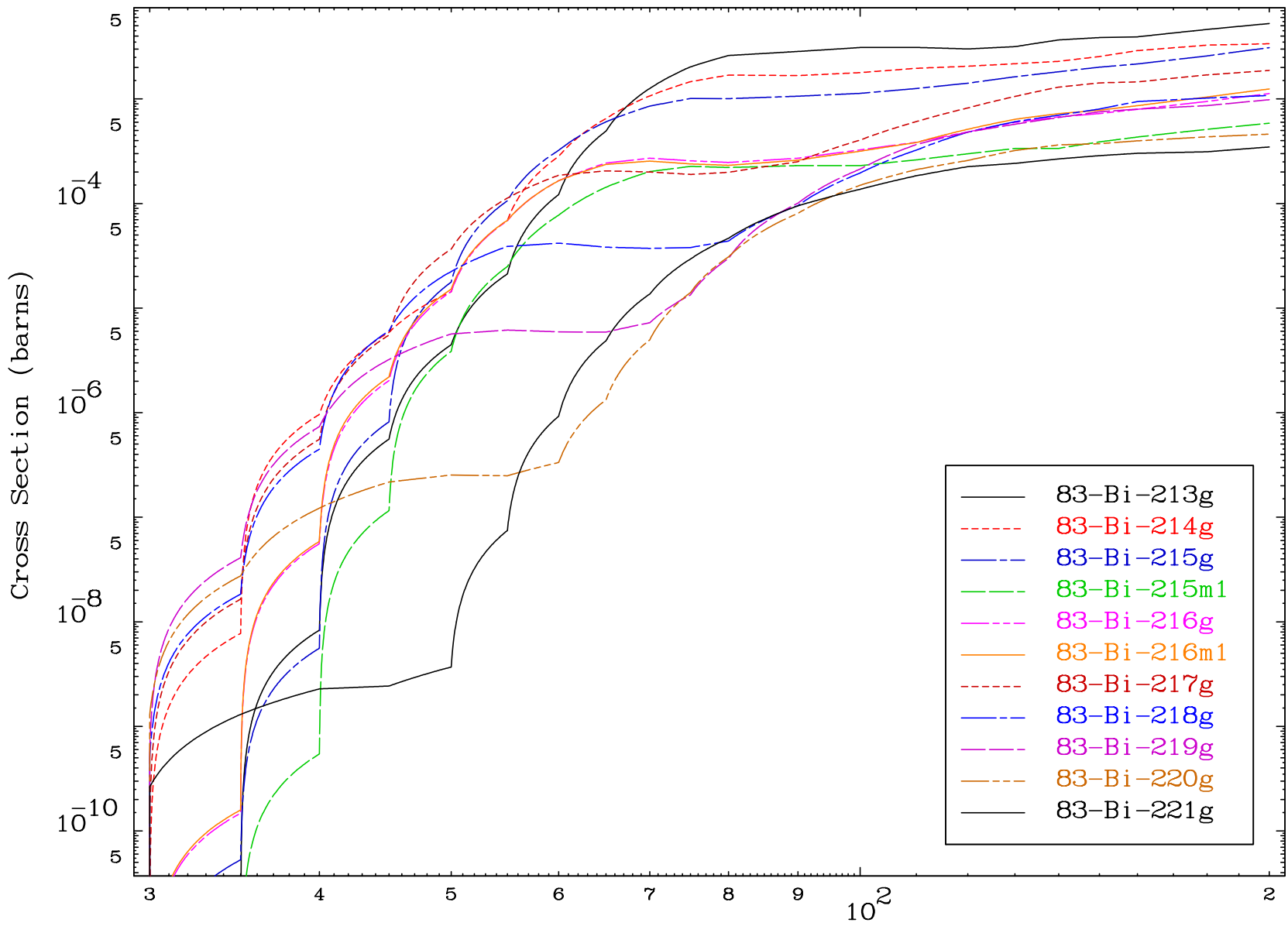
Radionuclide Production Cross Section





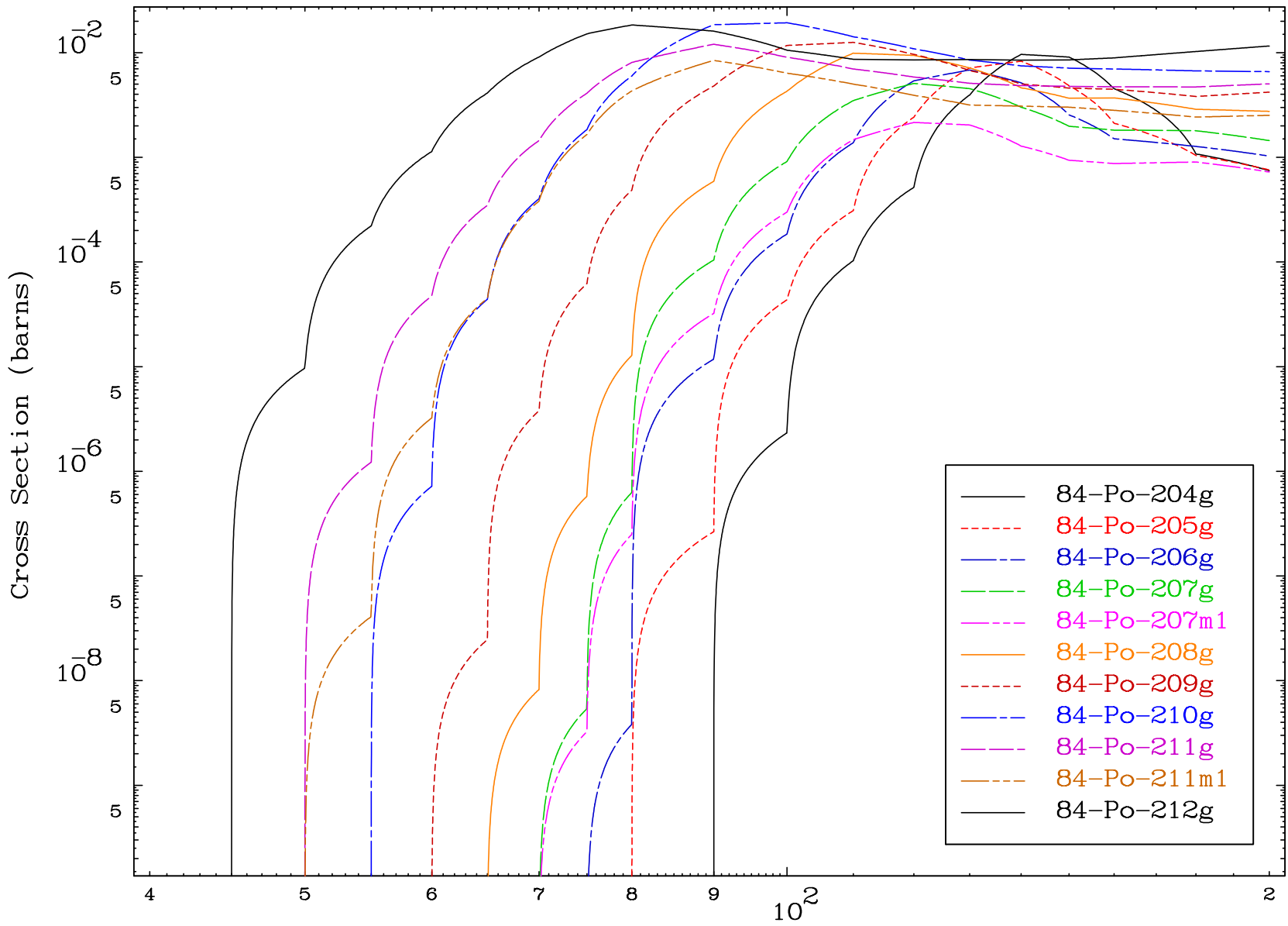
Radionuclide Production Cross Section



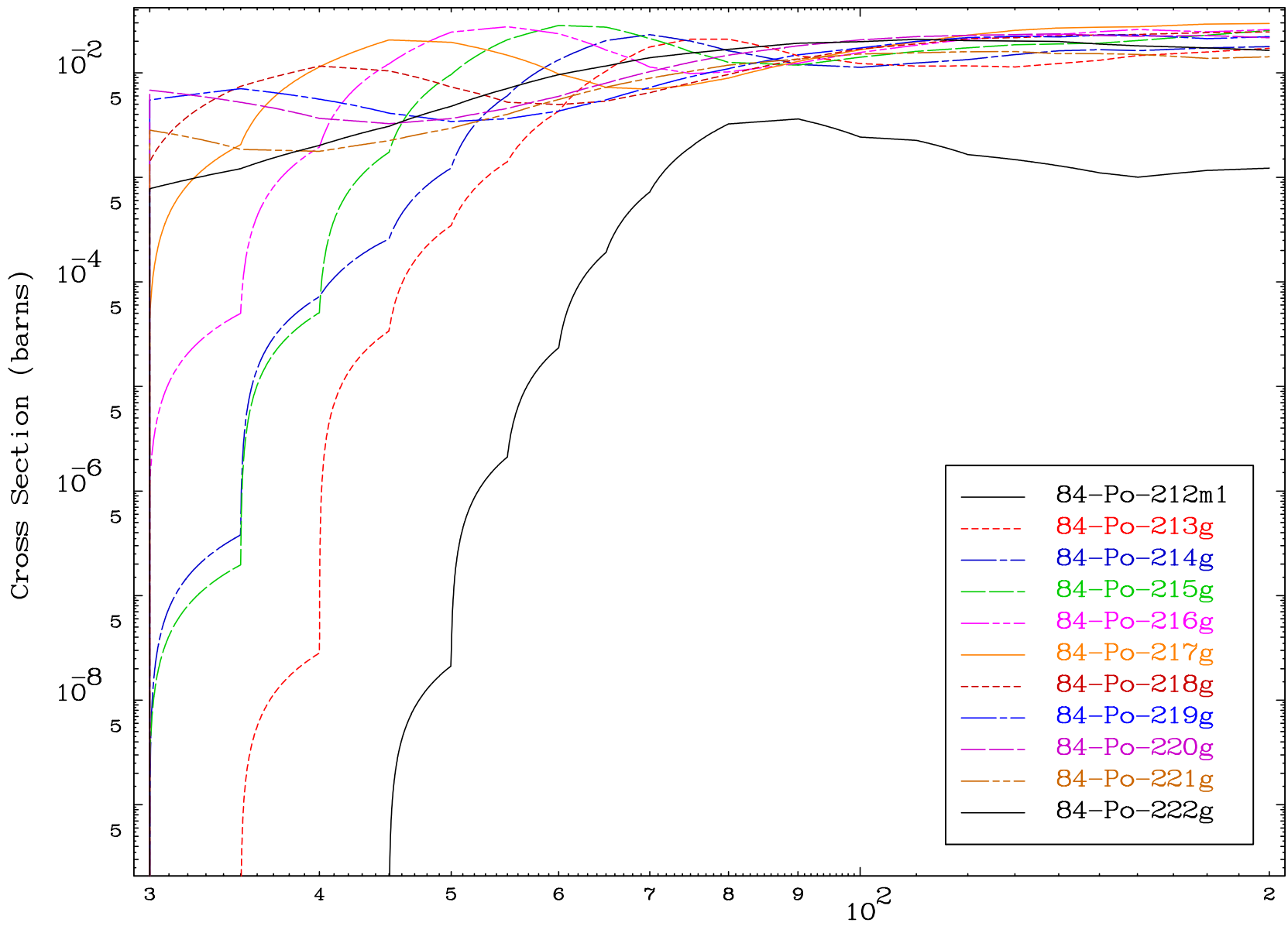




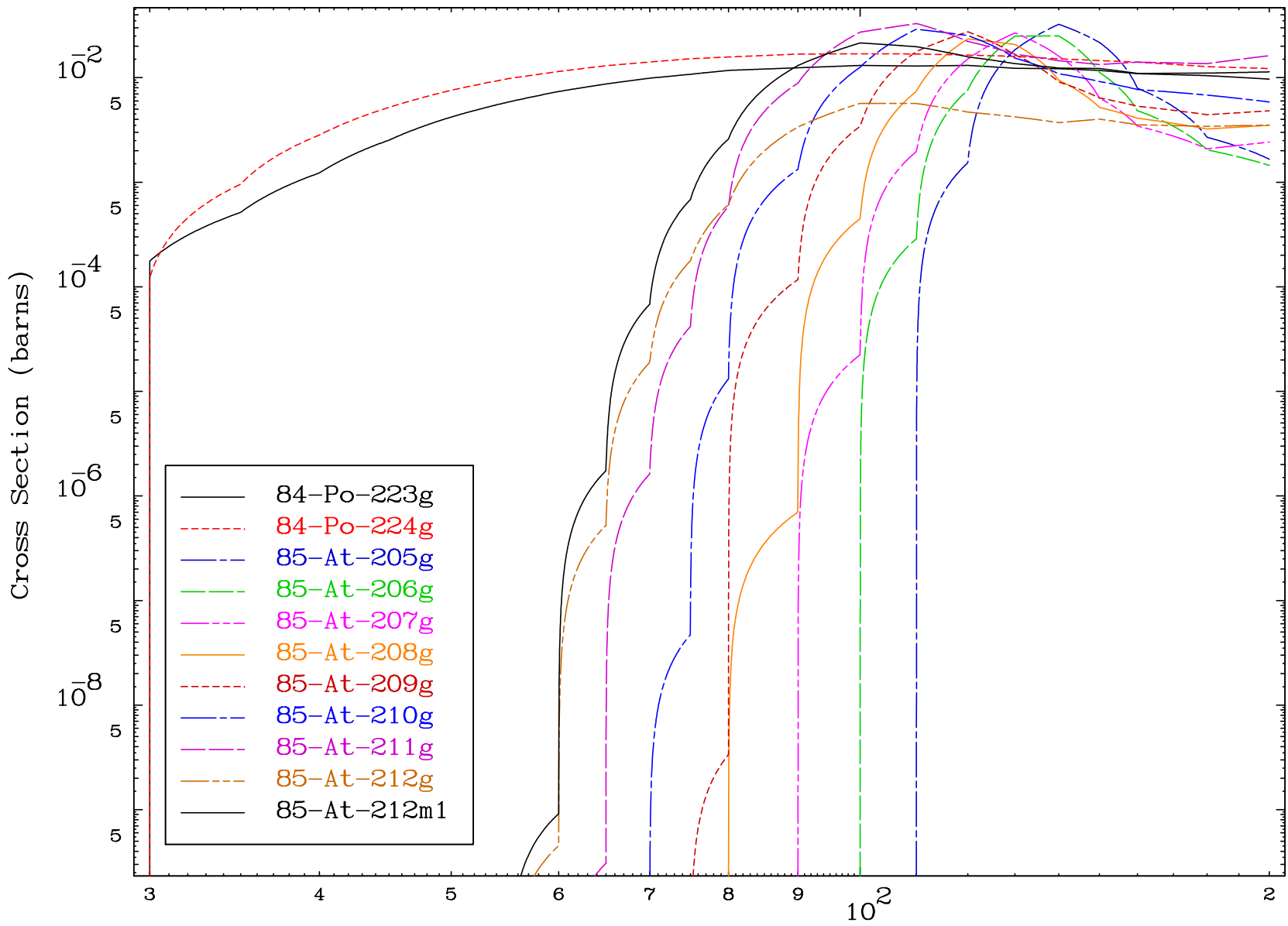
Radionuclide Production Cross Section



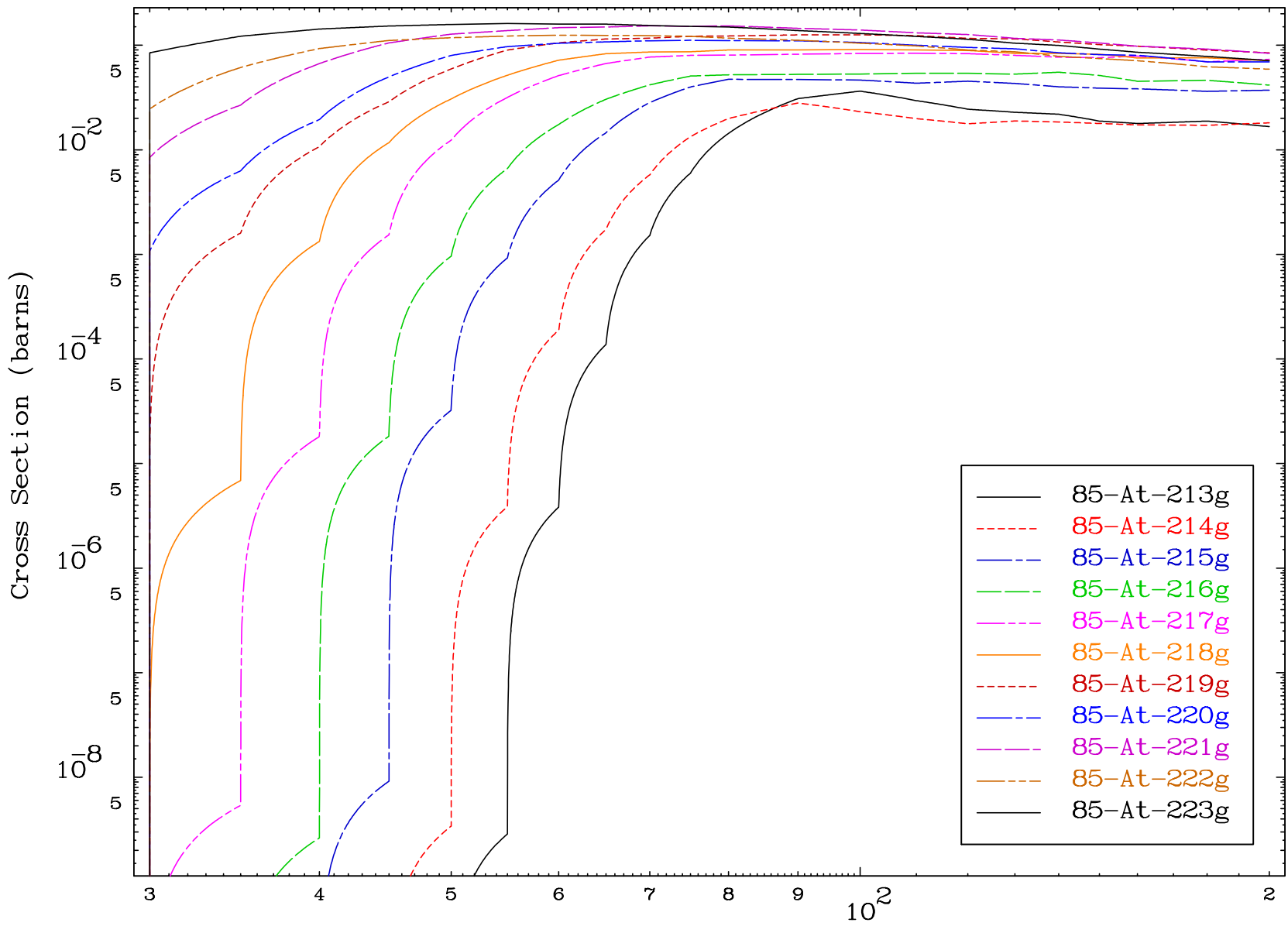
Radionuclide Production Cross Section



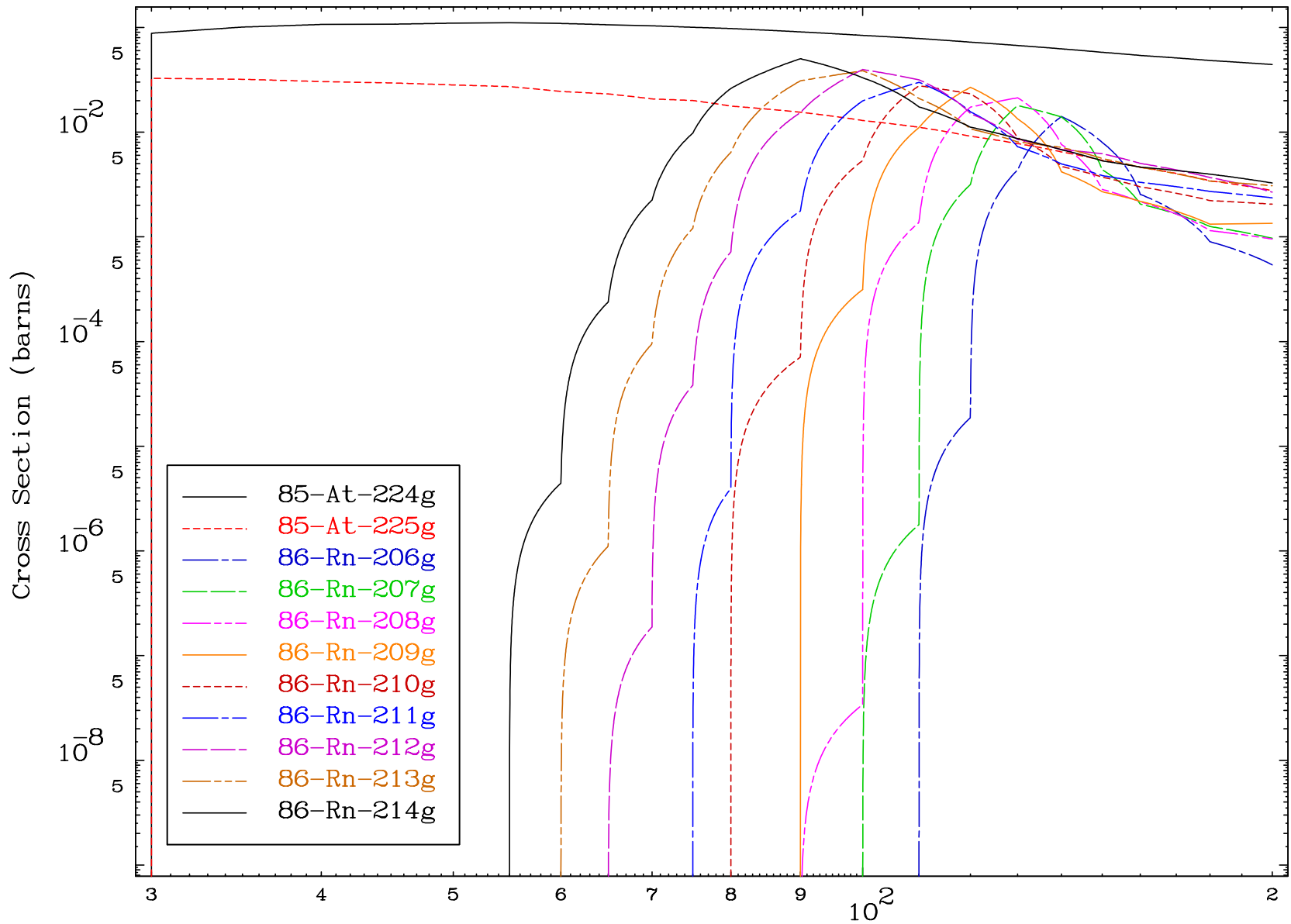
Radionuclide Production Cross Section



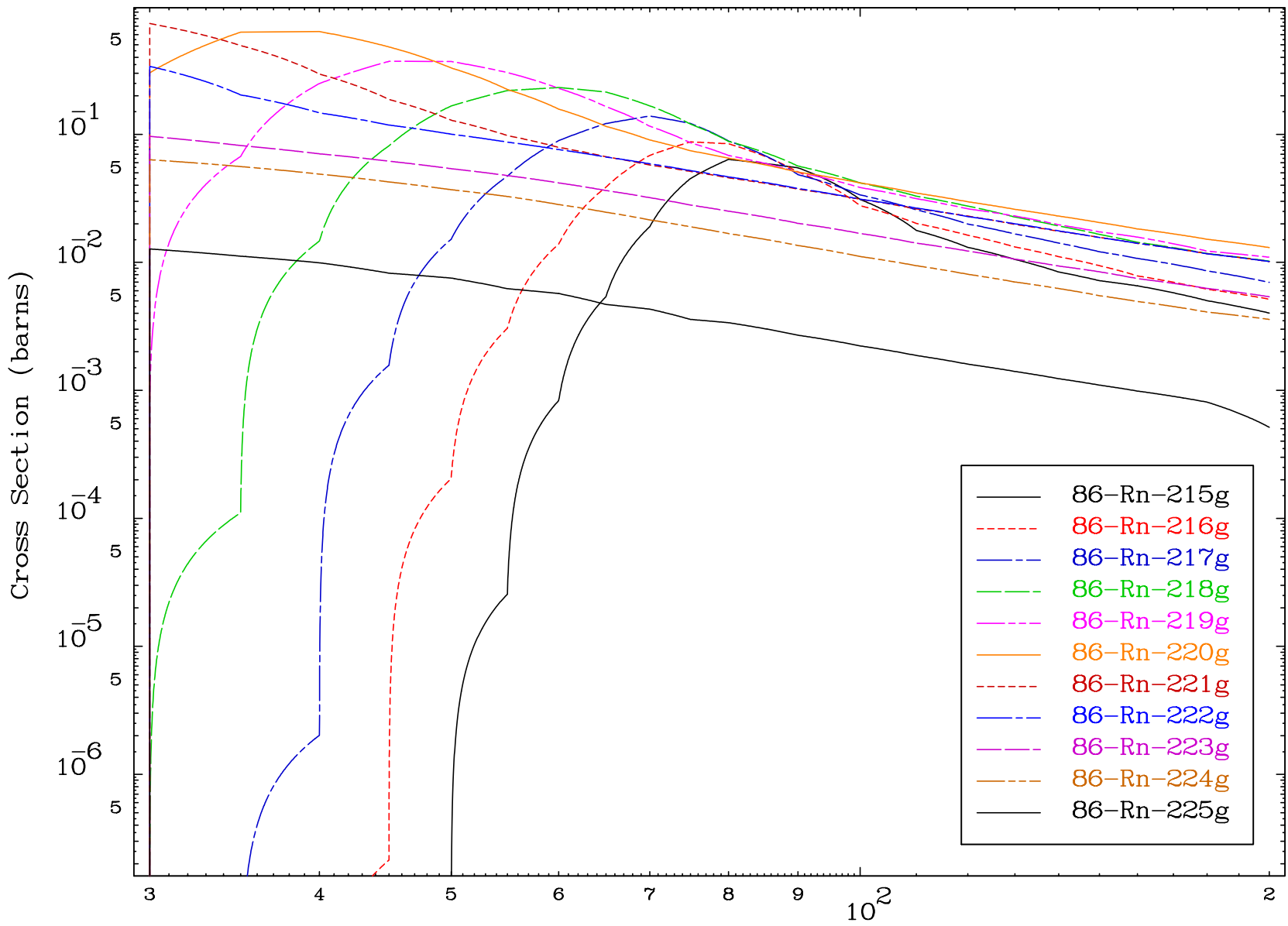
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

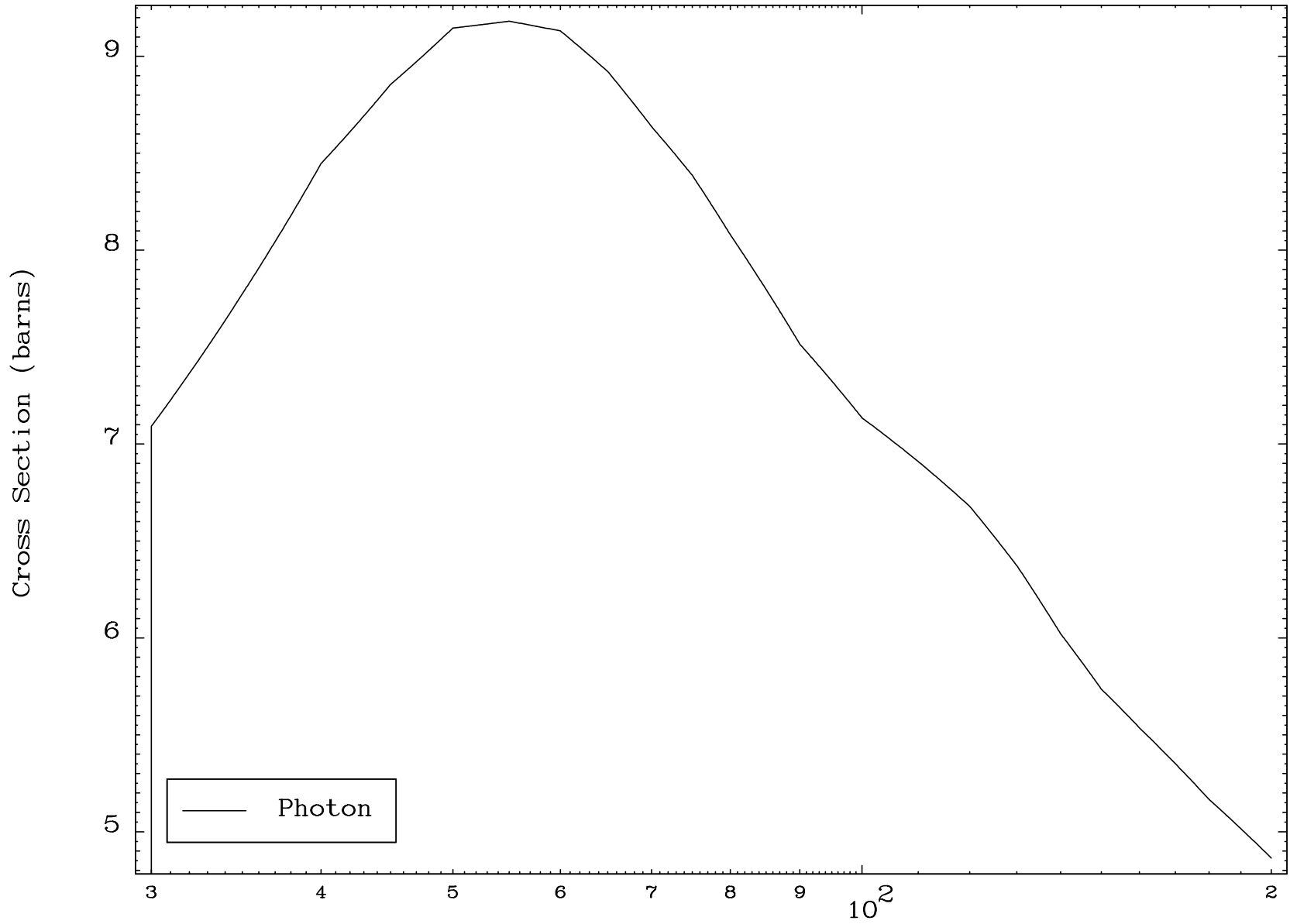


MAT 8591

(p,remainder)

85-At-225

Radionuclide Production Cross Section



23

Incident Energy (MeV)

85-At-225

