

Program EVALPLOT  
(Version 2018-1)

by

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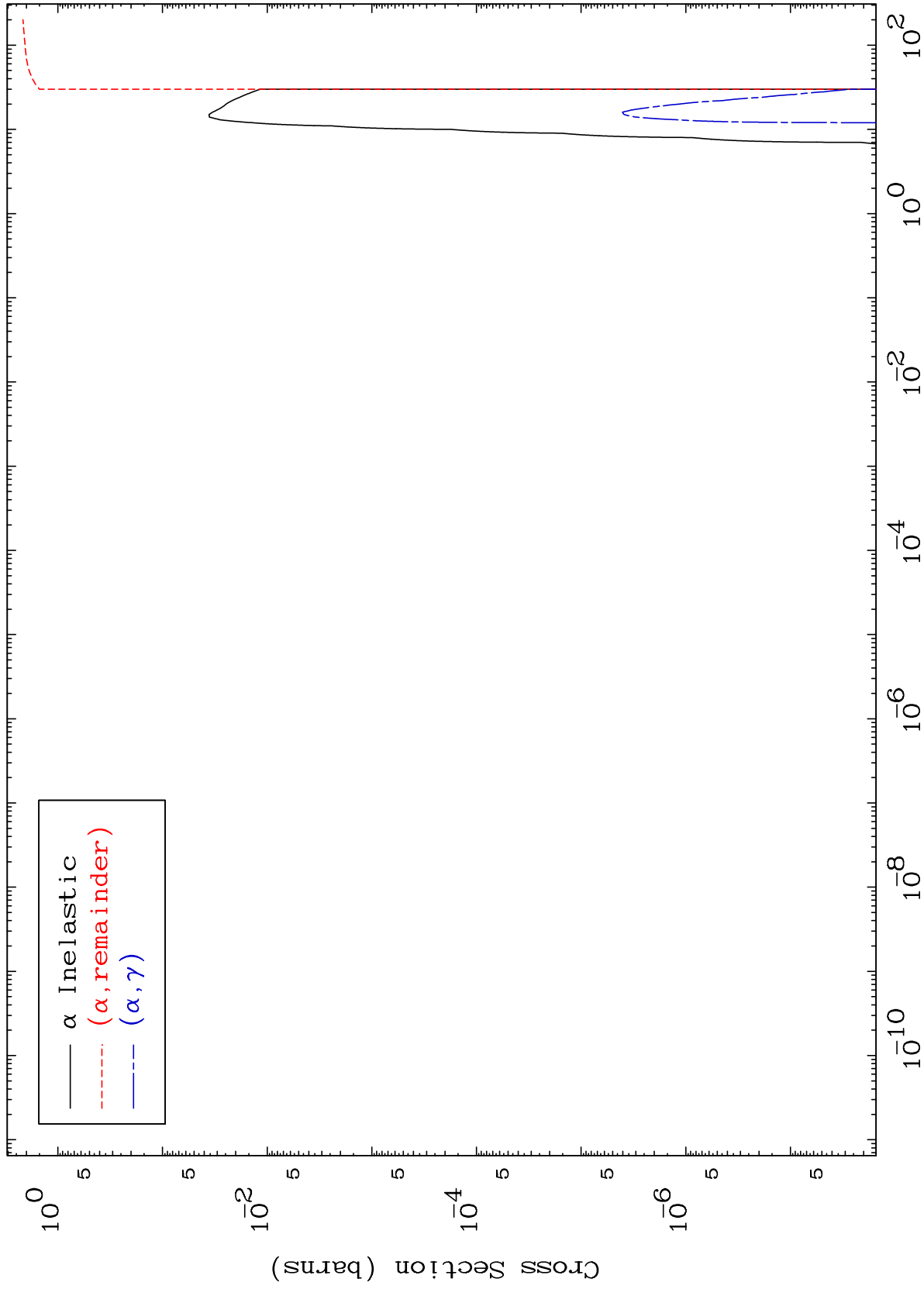
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 4746

0 Kelvin  $\alpha$  Major  
Cross Sections

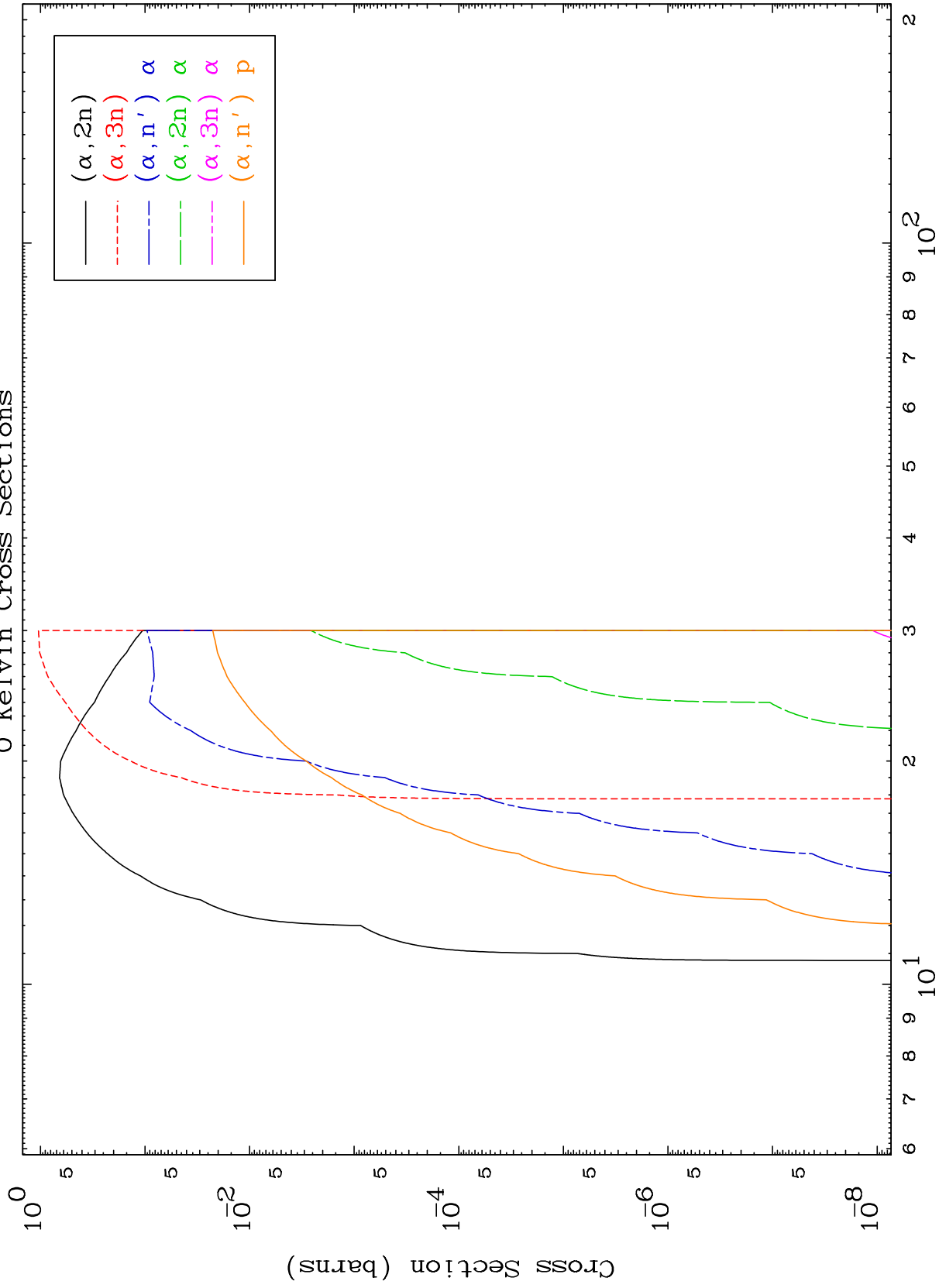
47-Ag-114



MAT 4746

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

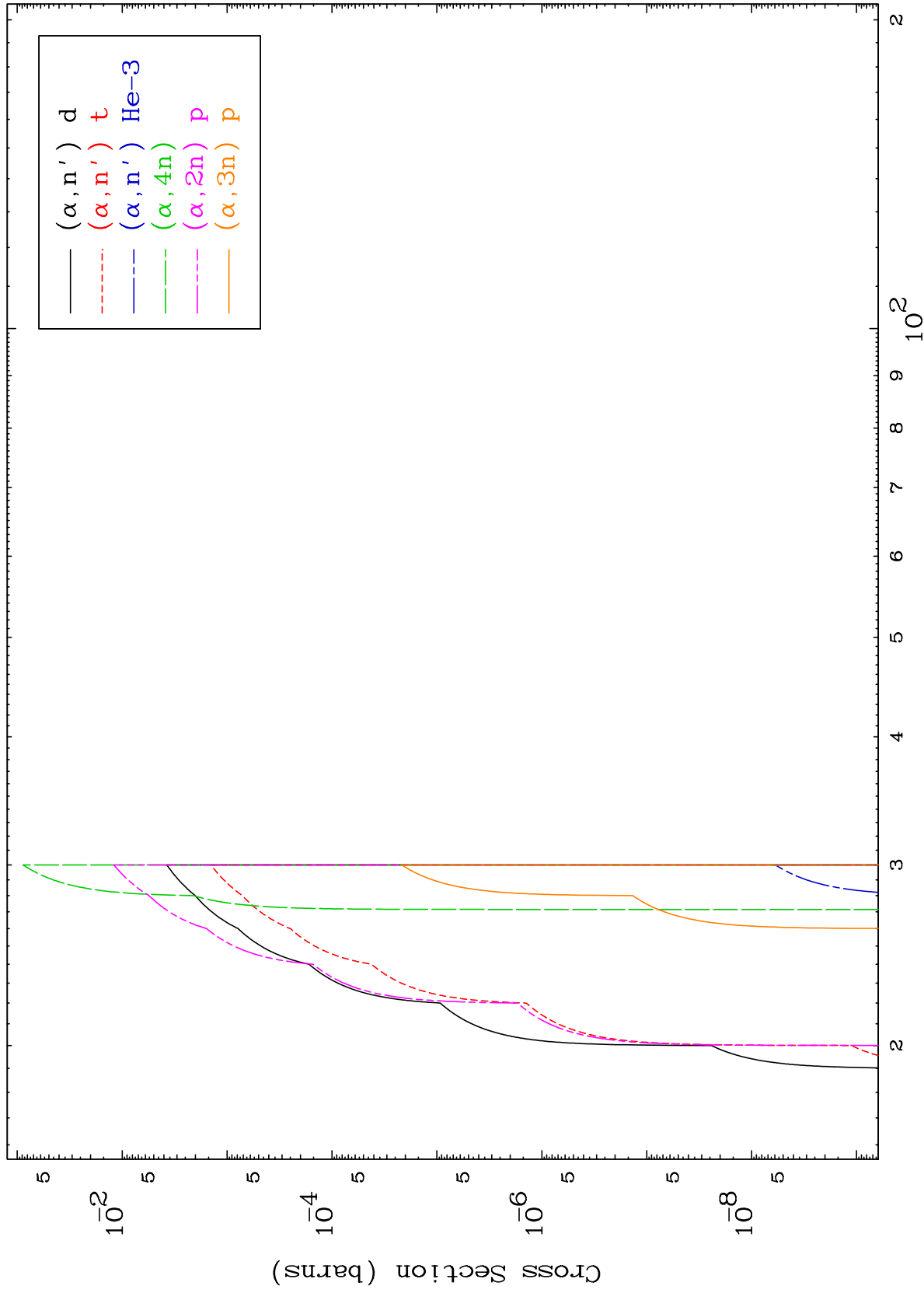
47-Ag-114

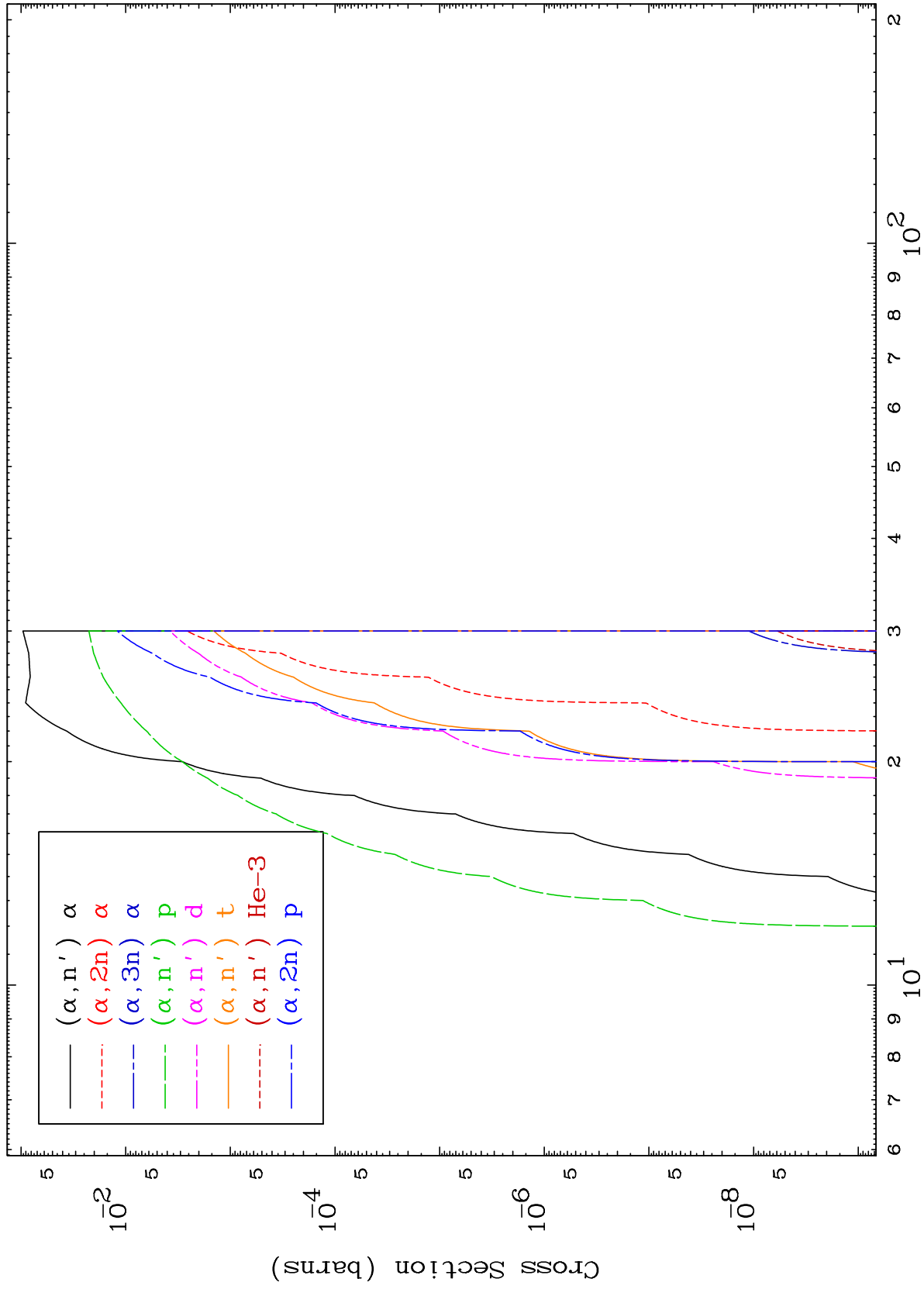


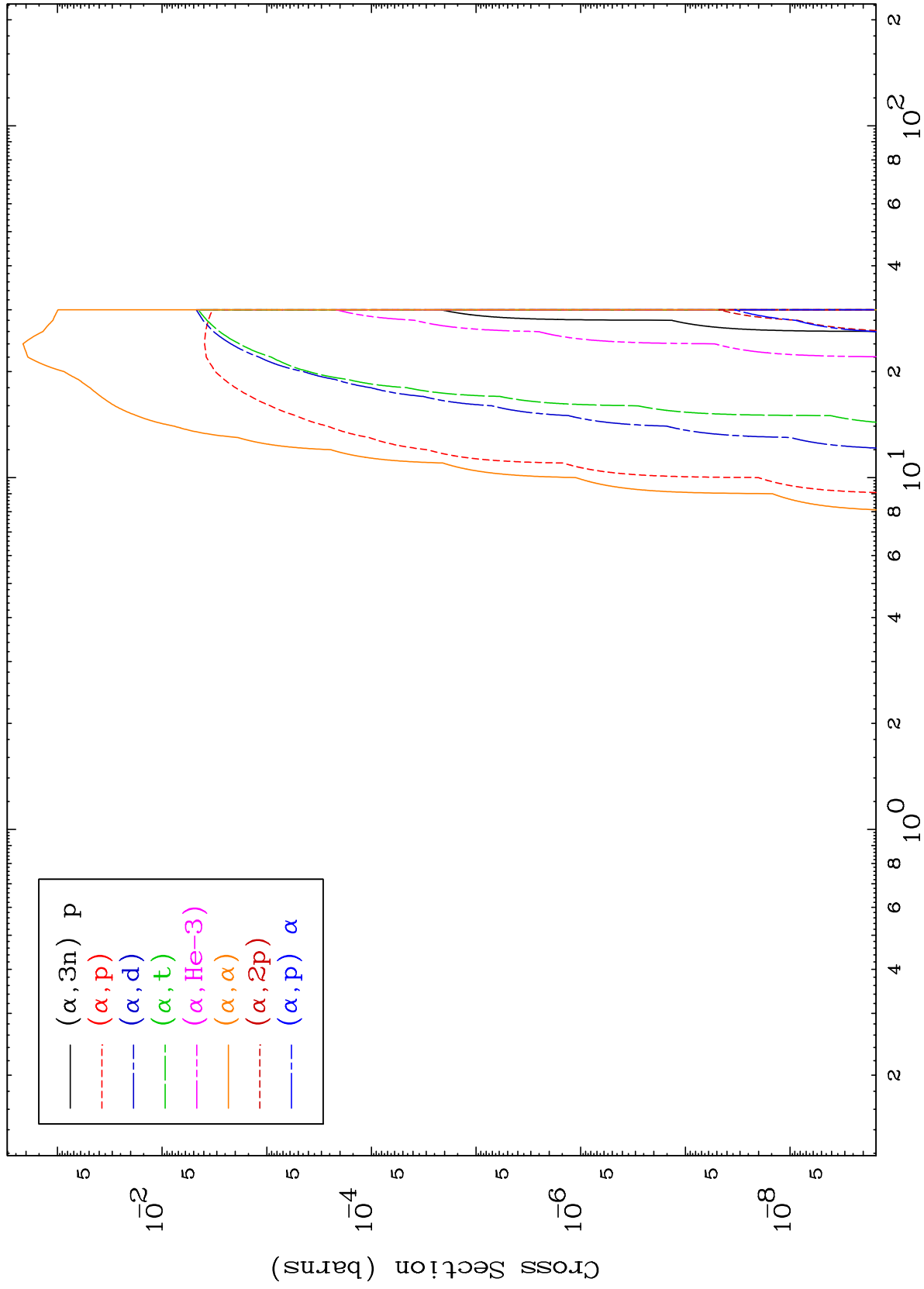
2

Incident Energy (MeV)

47-Ag-114



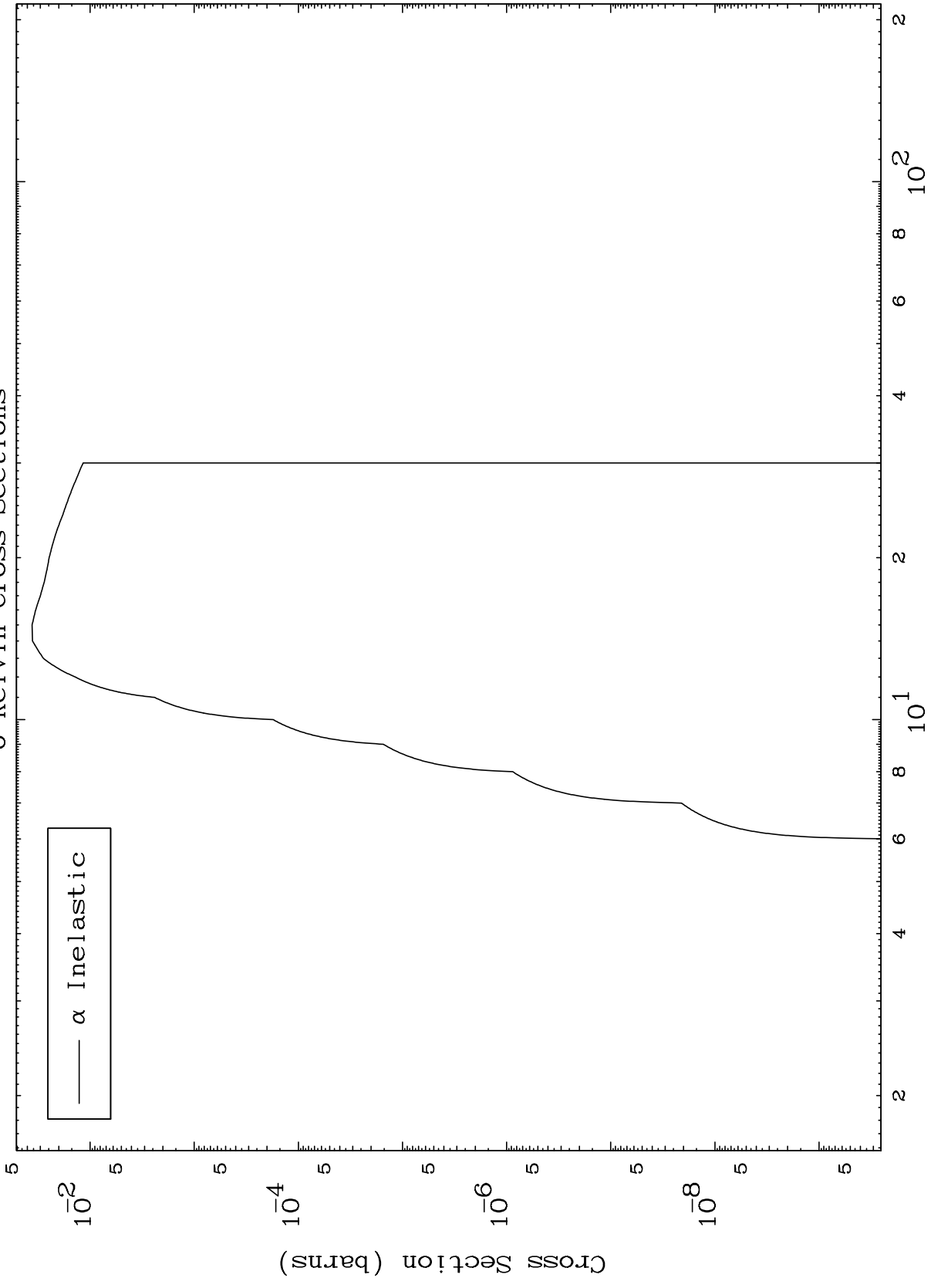




MAT 4746

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

47-Ag-114



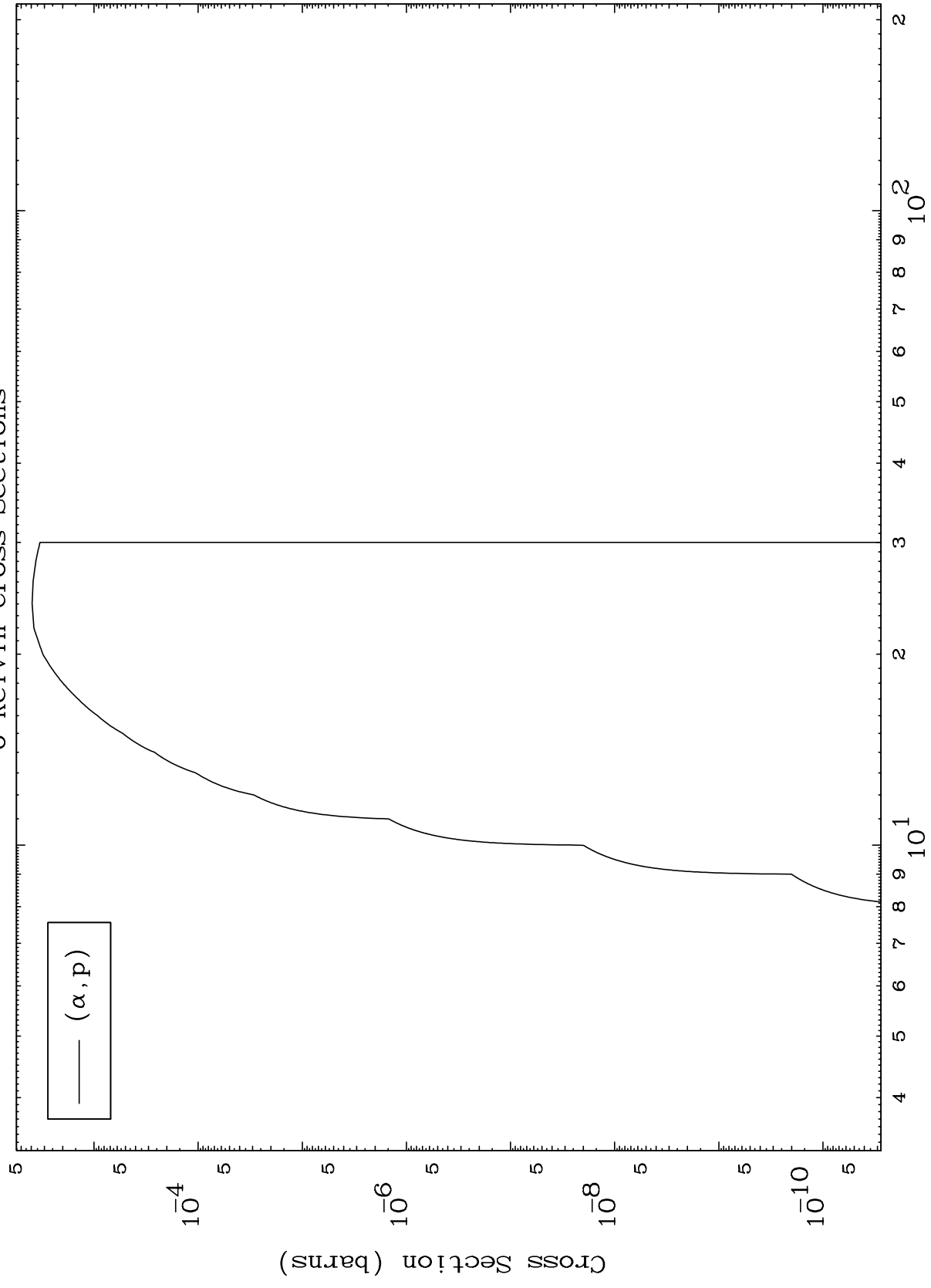
6

47-Ag-114

MAT 4746

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

47-Ag-114



7

Incident Energy (MeV)

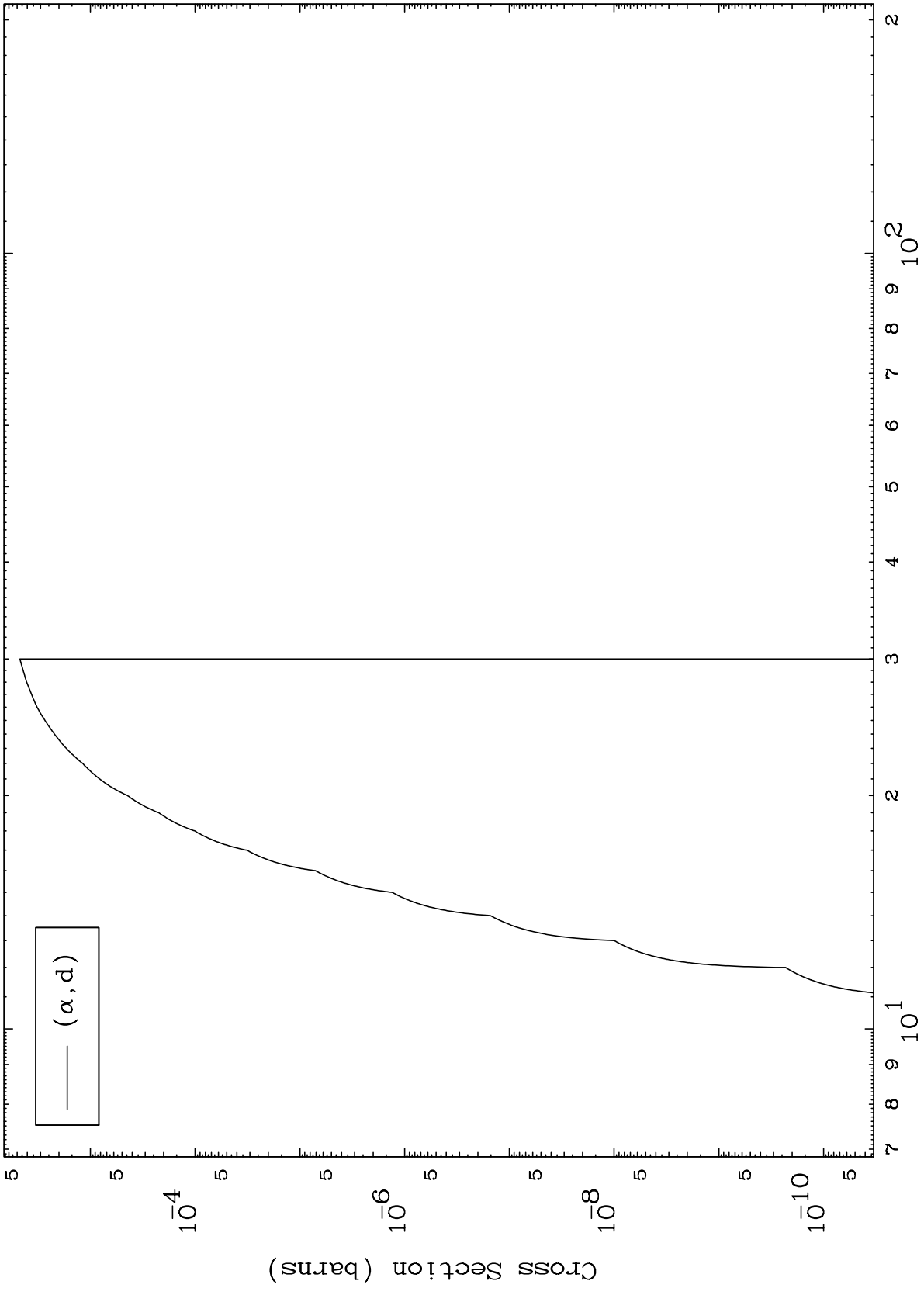
47-Ag-114



MAT 4746

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

47-Ag-114



8

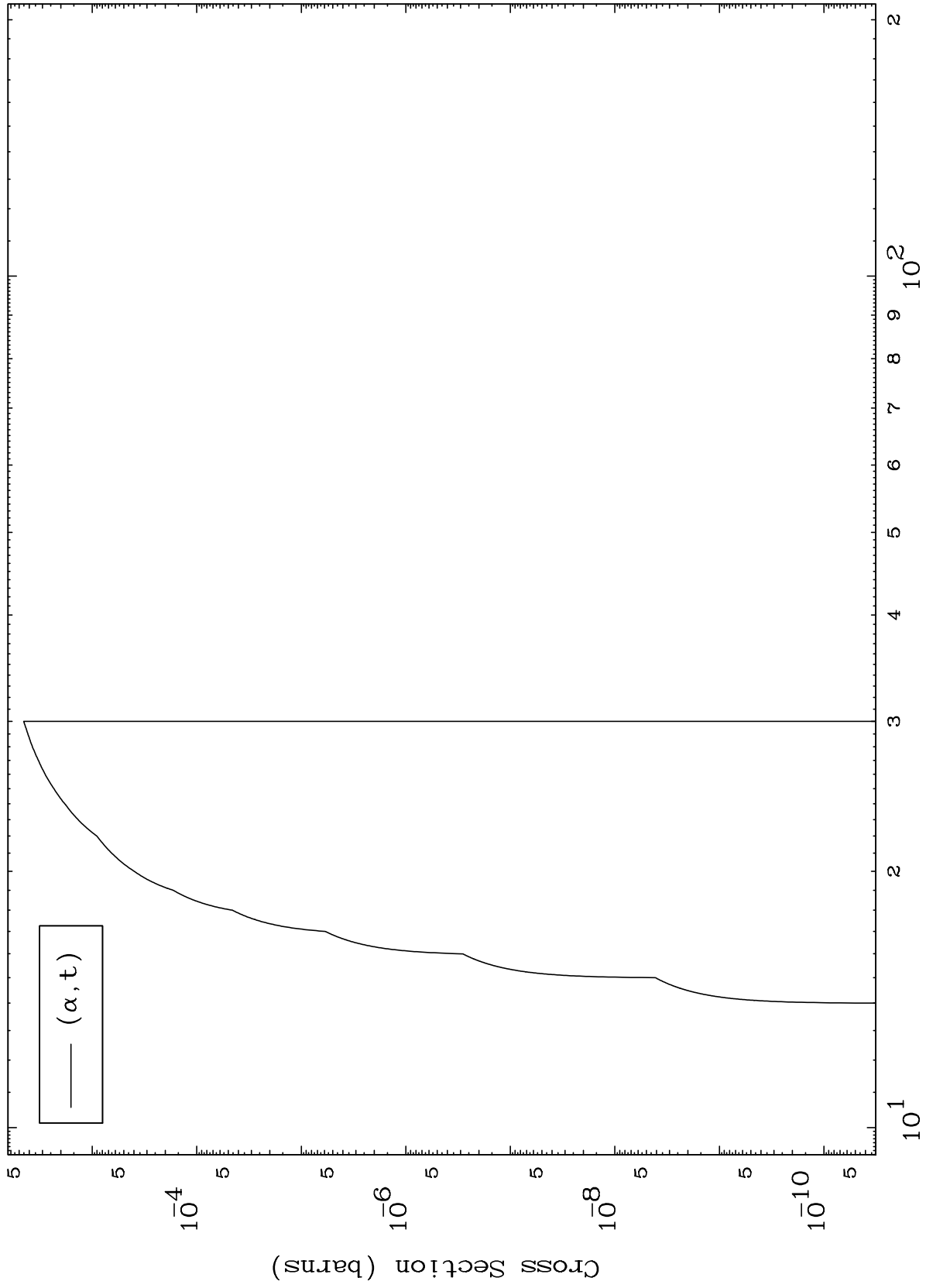
Incident Energy (MeV)

47-Ag-114

MAT 4746

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

47-Ag-114



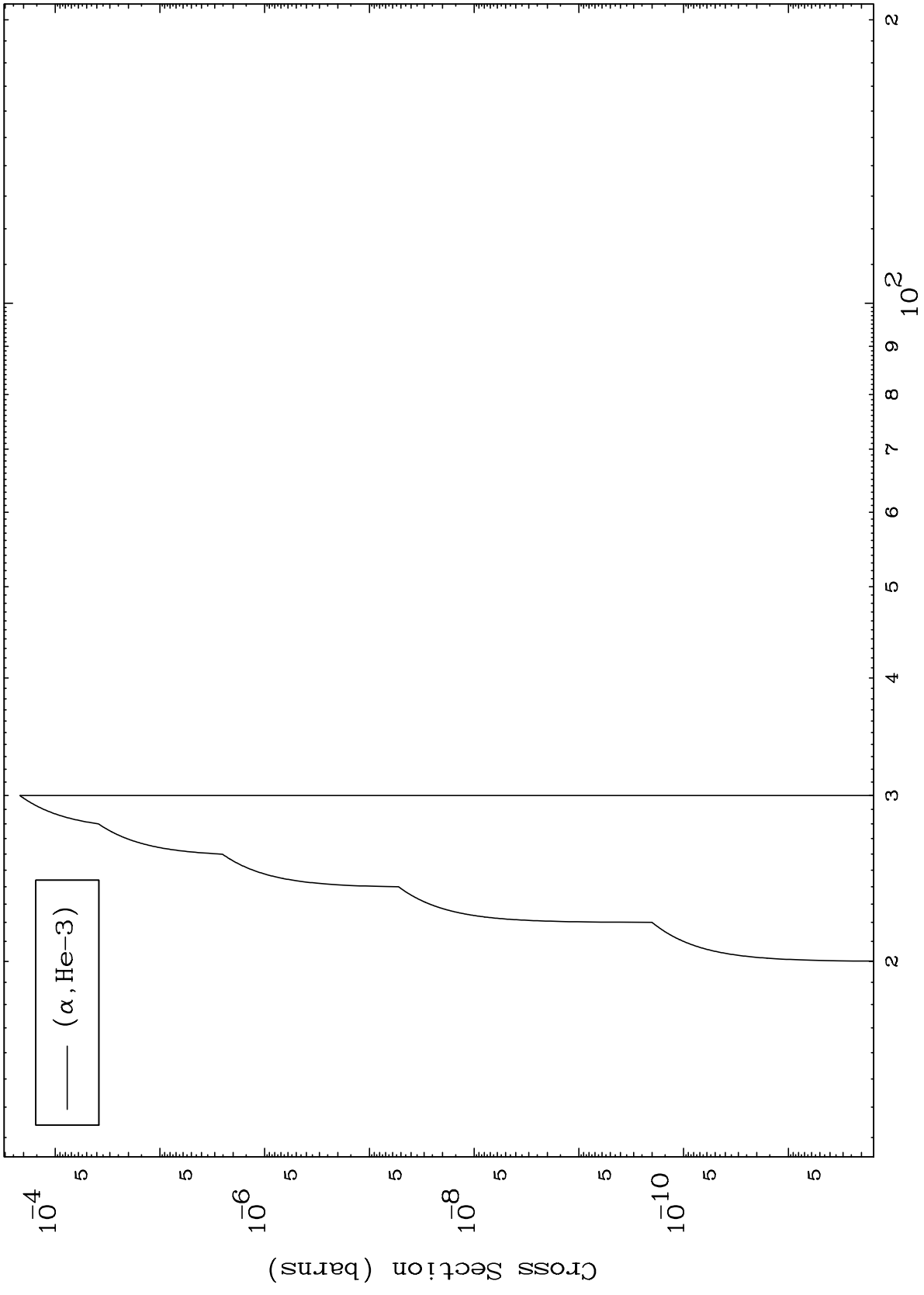
Incident Energy (MeV)

47-Ag-114

MAT 4746

( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

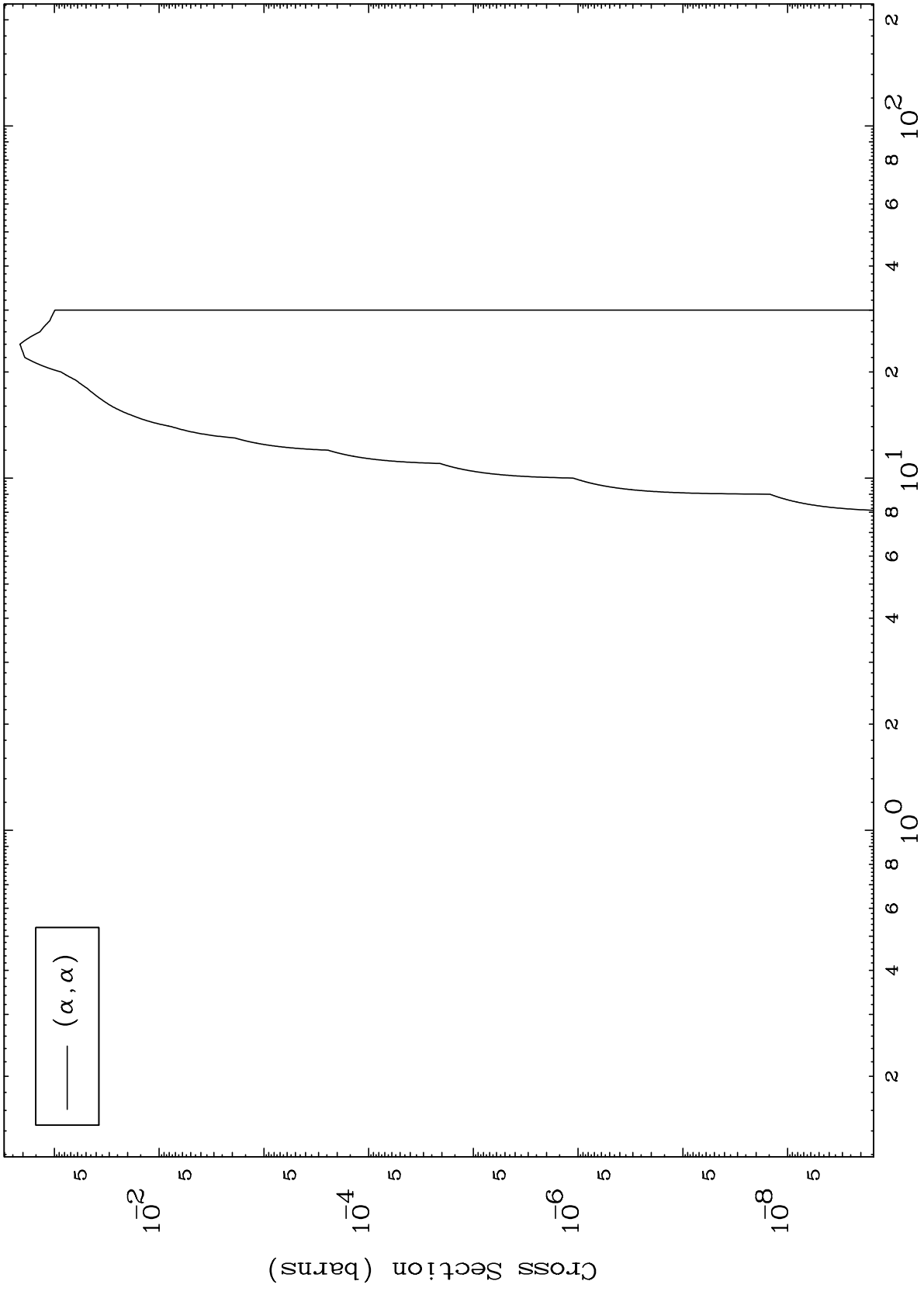
47-Ag-114



10

Incident Energy (MeV)

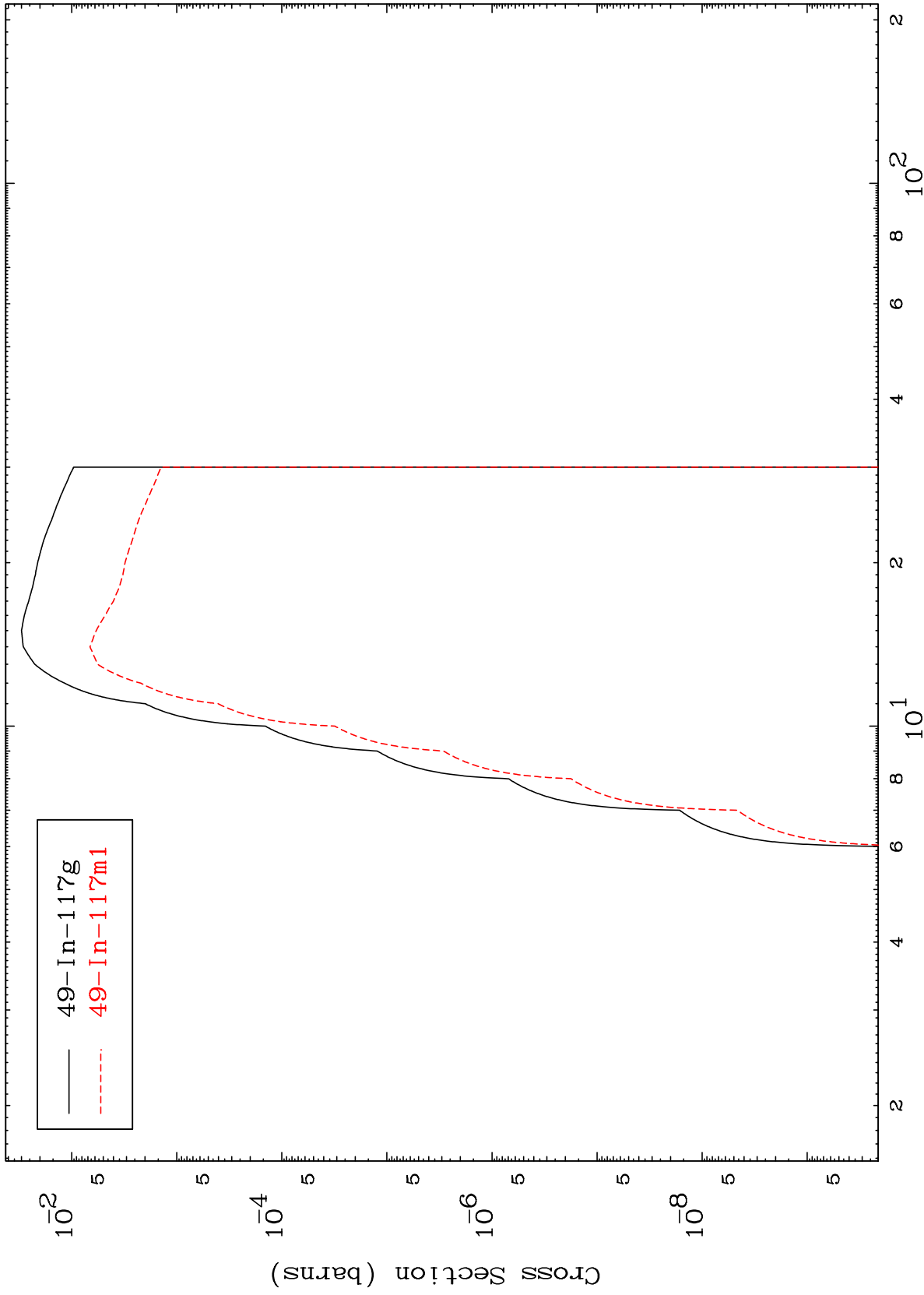
47-Ag-114



MAT 4746

Radionuclide Production Cross Section

47-Ag-114



12

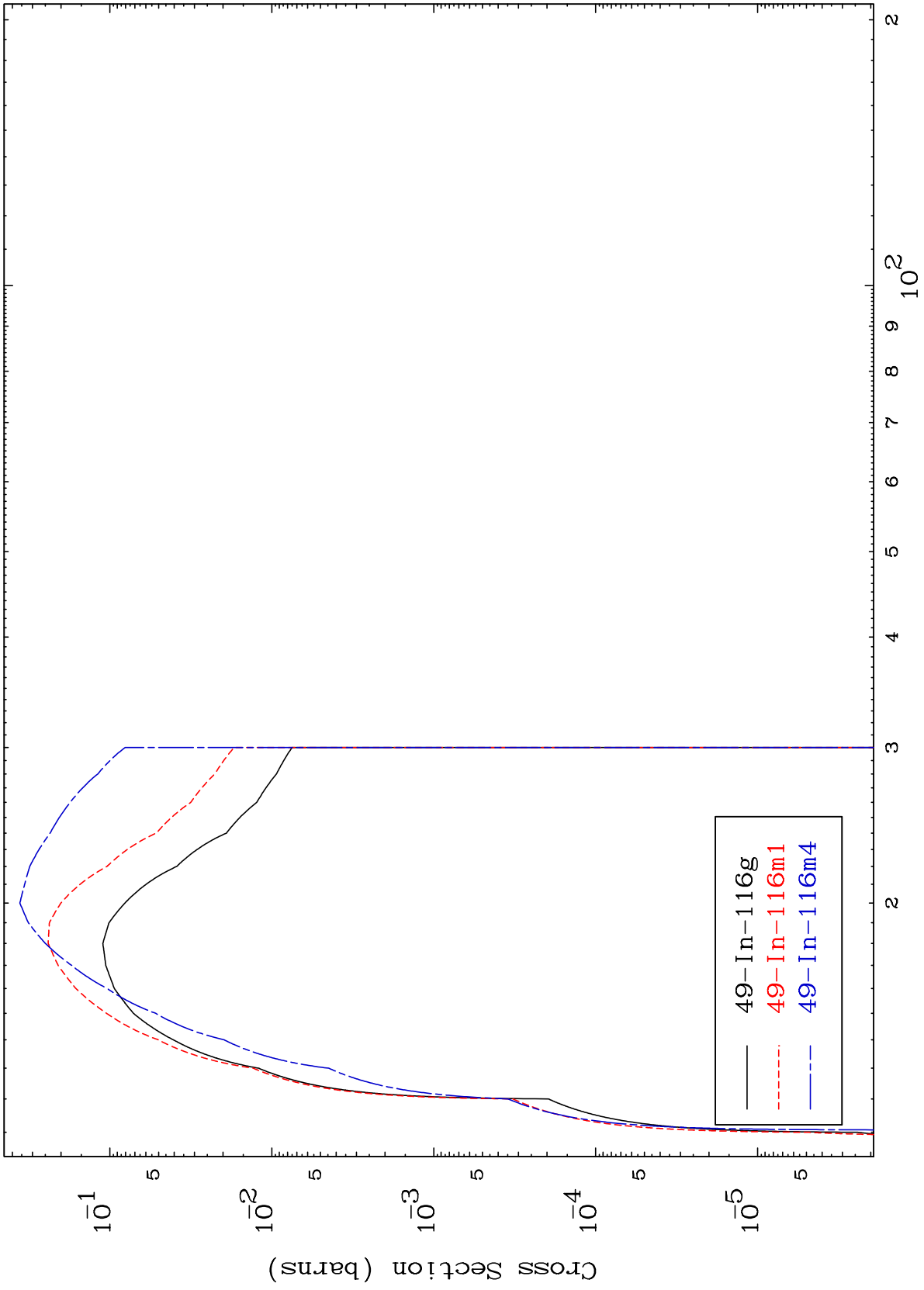
Incident Energy (MeV)

47-Ag-114

MAT 4746

47-Ag-114

Radionuclide Production Cross Section  
( $\alpha, 2n$ )



13

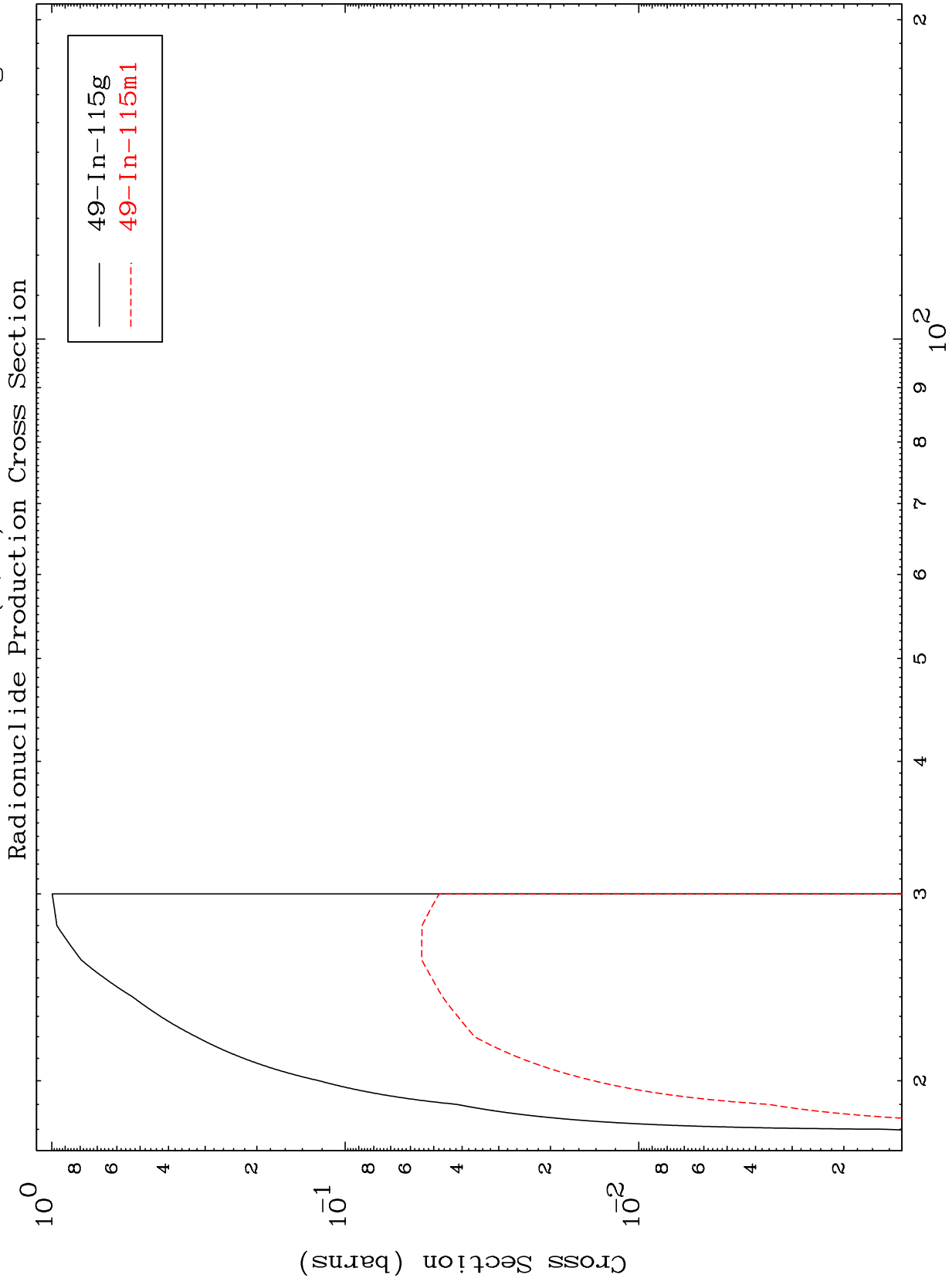
Incident Energy (MeV)

47-Ag-114

MAT 4746

( $\alpha, 3n$ )

47-Ag-114



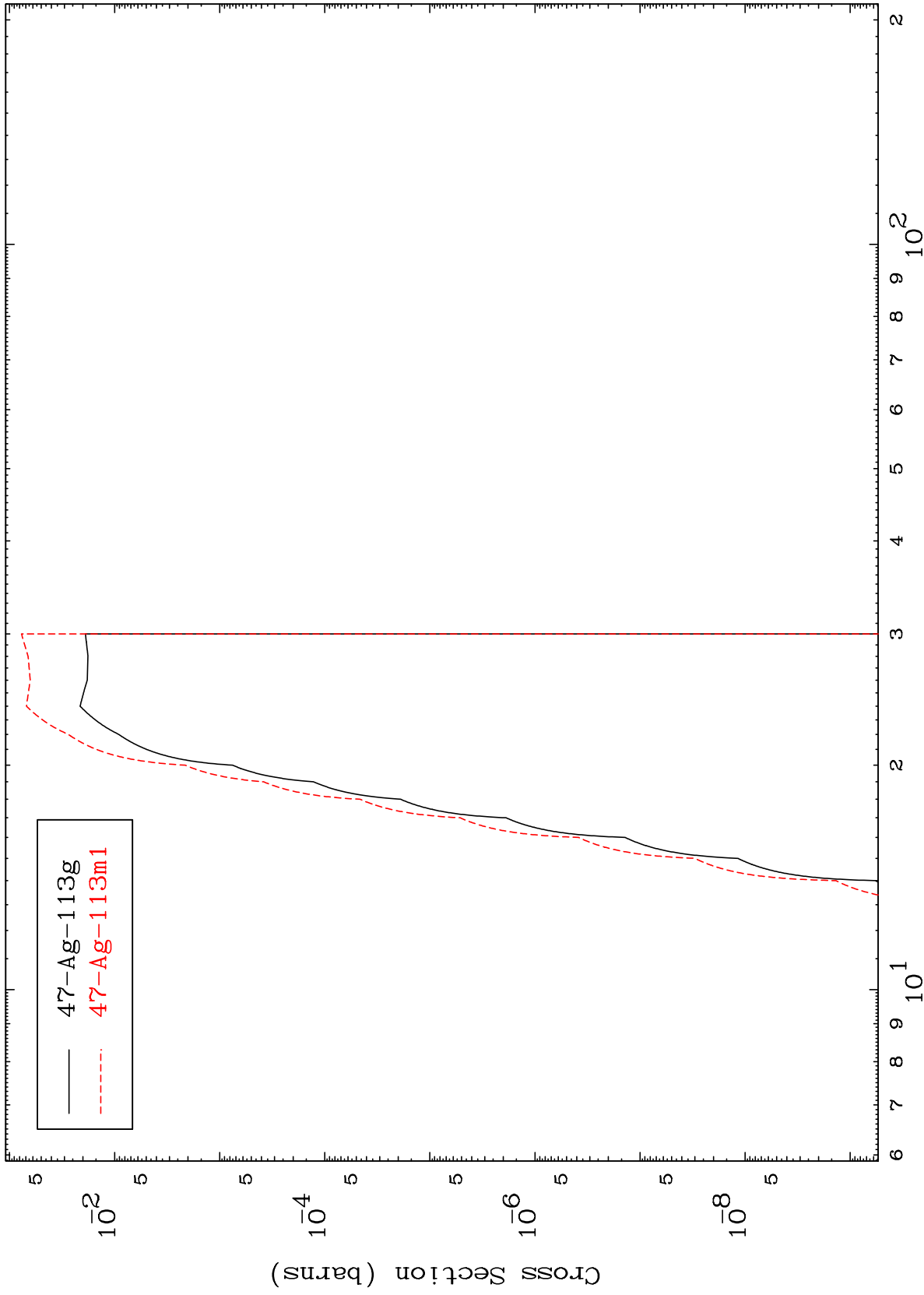
14

47-Ag-114

MAT 4746

47-Ag-114

$(\alpha, n')$   $\alpha$   
Radionuclide Production Cross Section



15

Incident Energy (MeV)

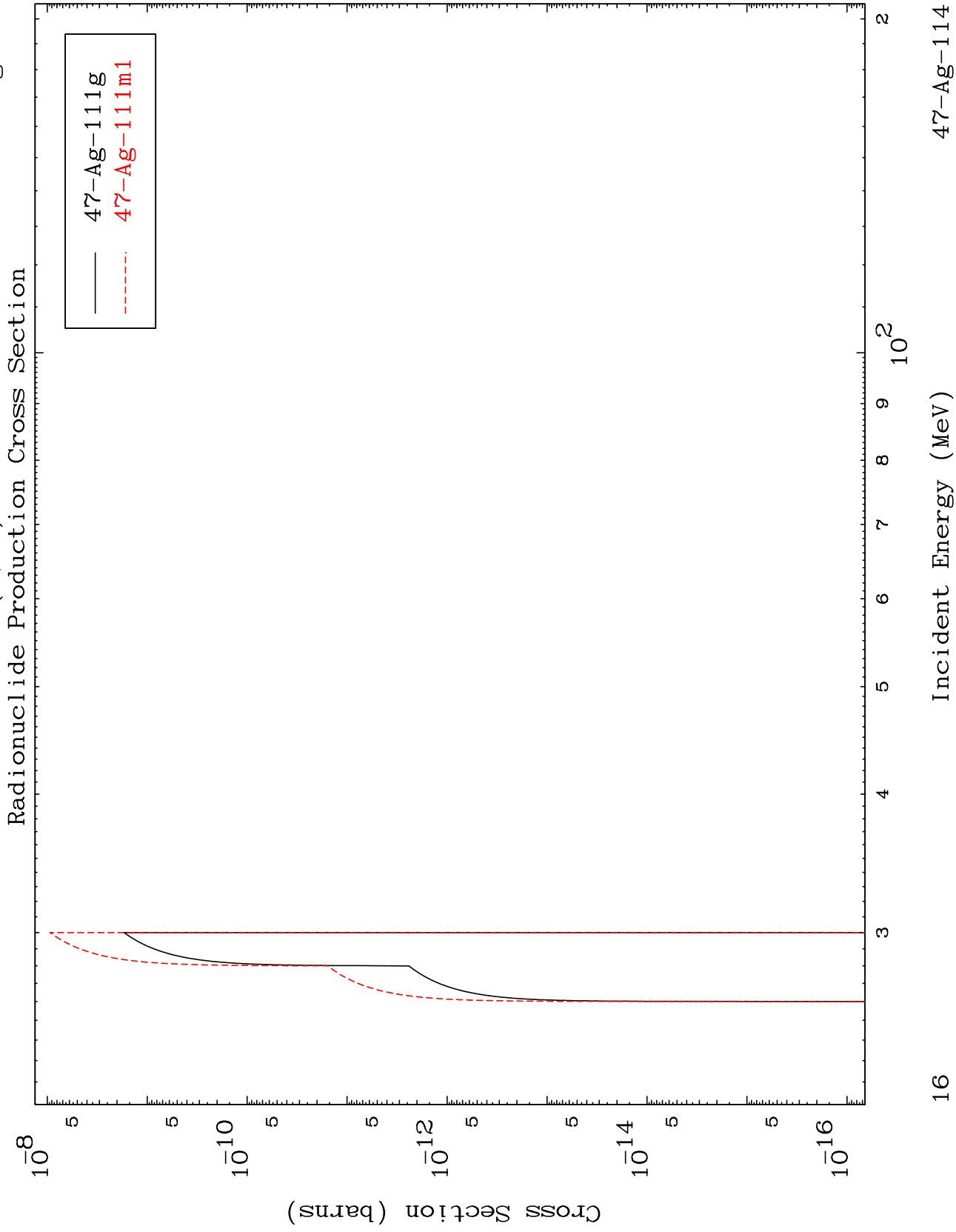
47-Ag-114



MAT 4746

( $\alpha, 3n$ )  $\alpha$

47-Ag-114



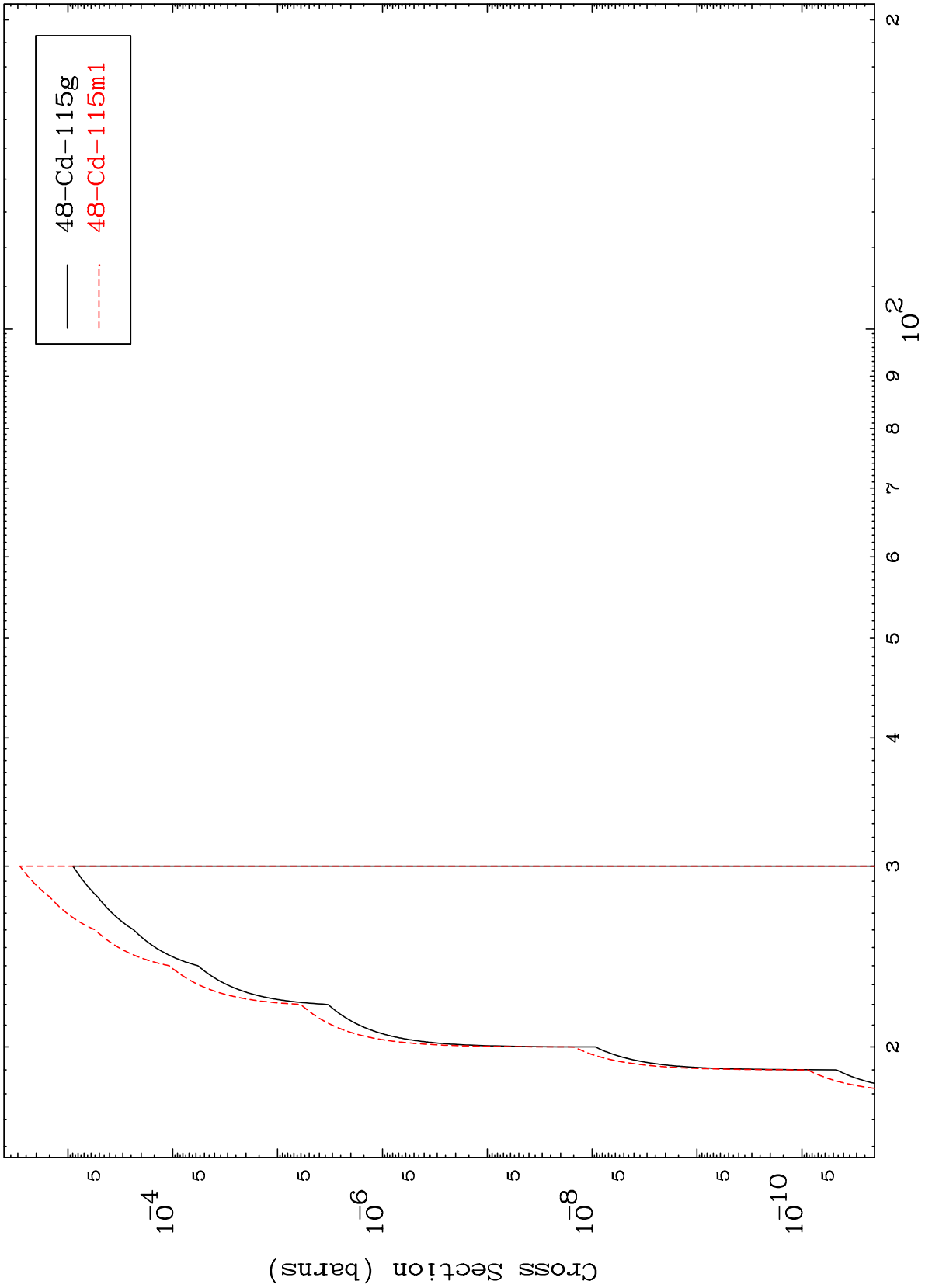
16

MAT 4746

( $\alpha, n'$ ) d

47-Ag-114

Radionuclide Production Cross Section



17

Incident Energy (MeV)

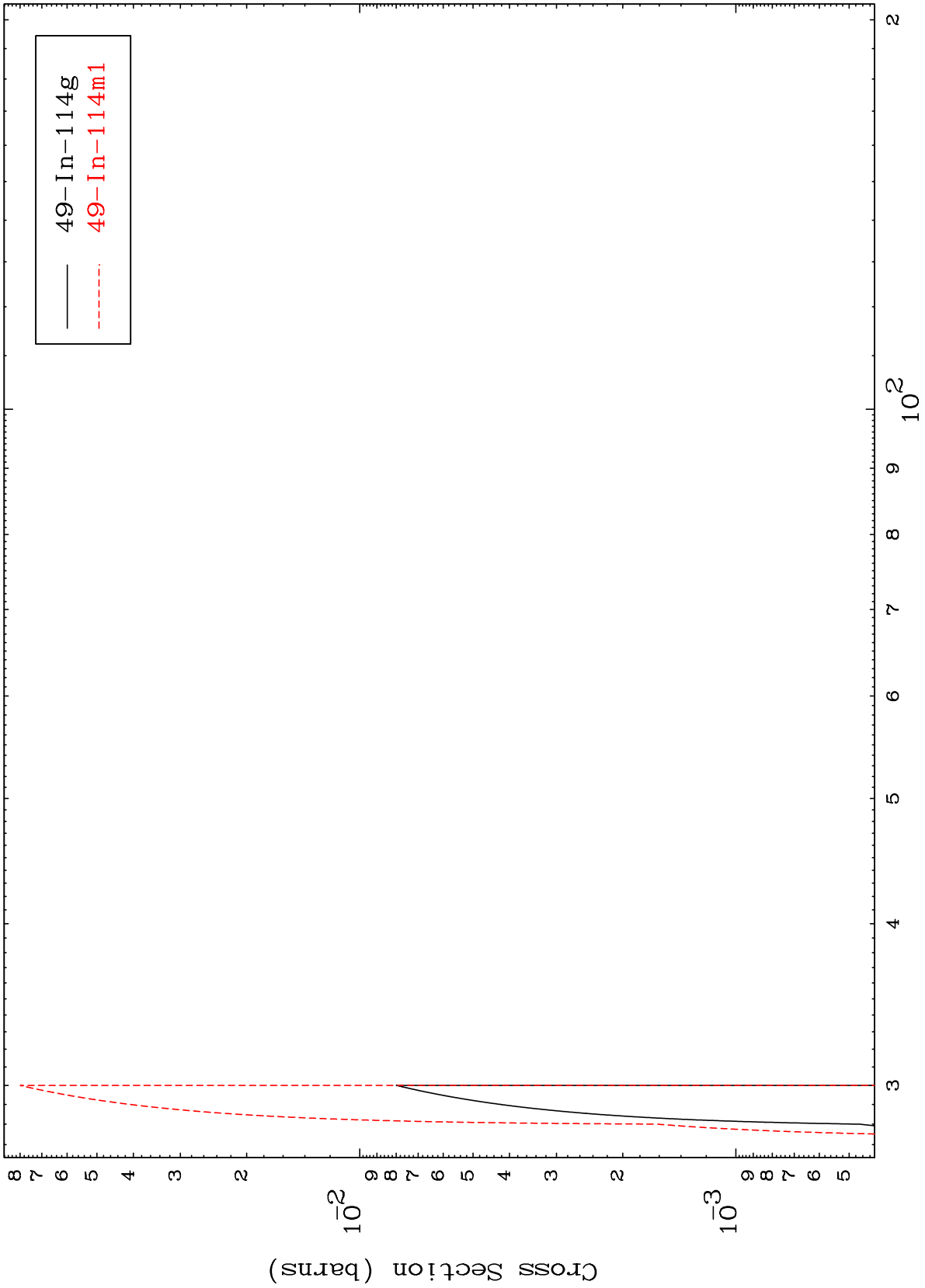
47-Ag-114

MAT 4746

( $\alpha, 4n$ )

47-Ag-114

Radionuclide Production Cross Section



49-In-114g  
49-In-114m1

18

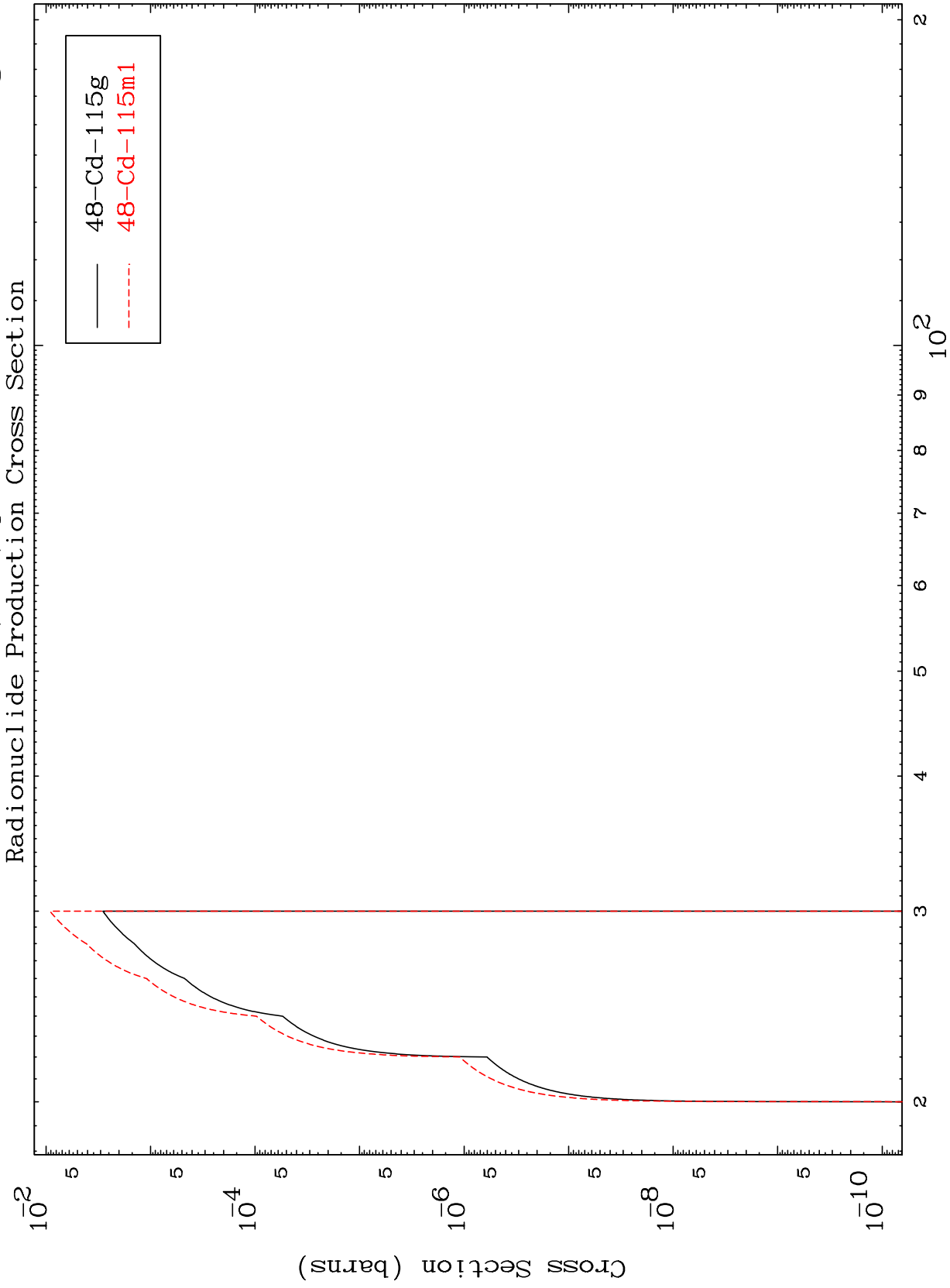
Incident Energy (MeV)

47-Ag-114

MAT 4746

( $\alpha, 2n$ ) p

47-Ag-114



19

Incident Energy (MeV)

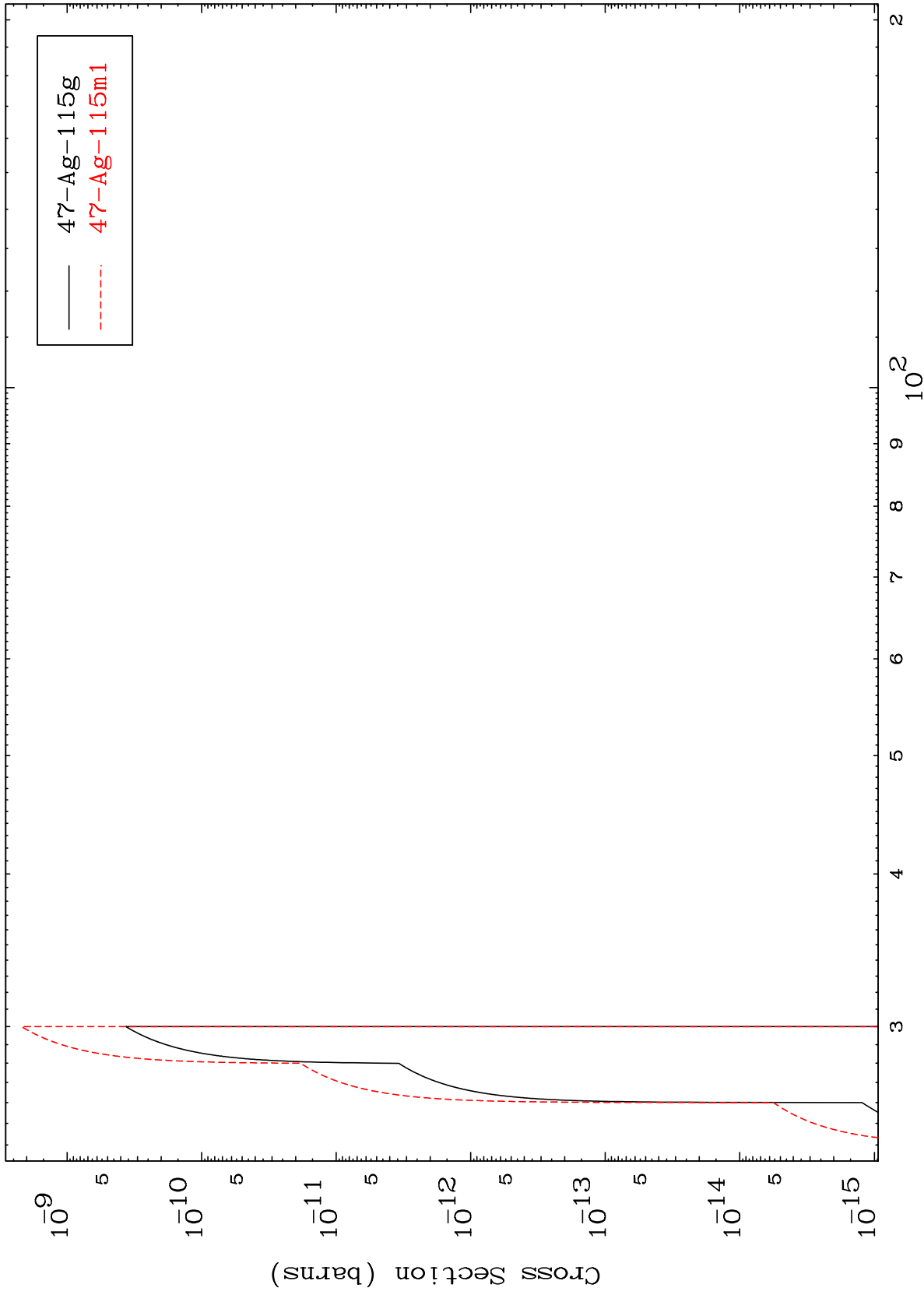
47-Ag-114

MAT 4746

( $\alpha, 2n$ ) p

47-Ag-114

Radionuclide Production Cross Section



20

Incident Energy (MeV)

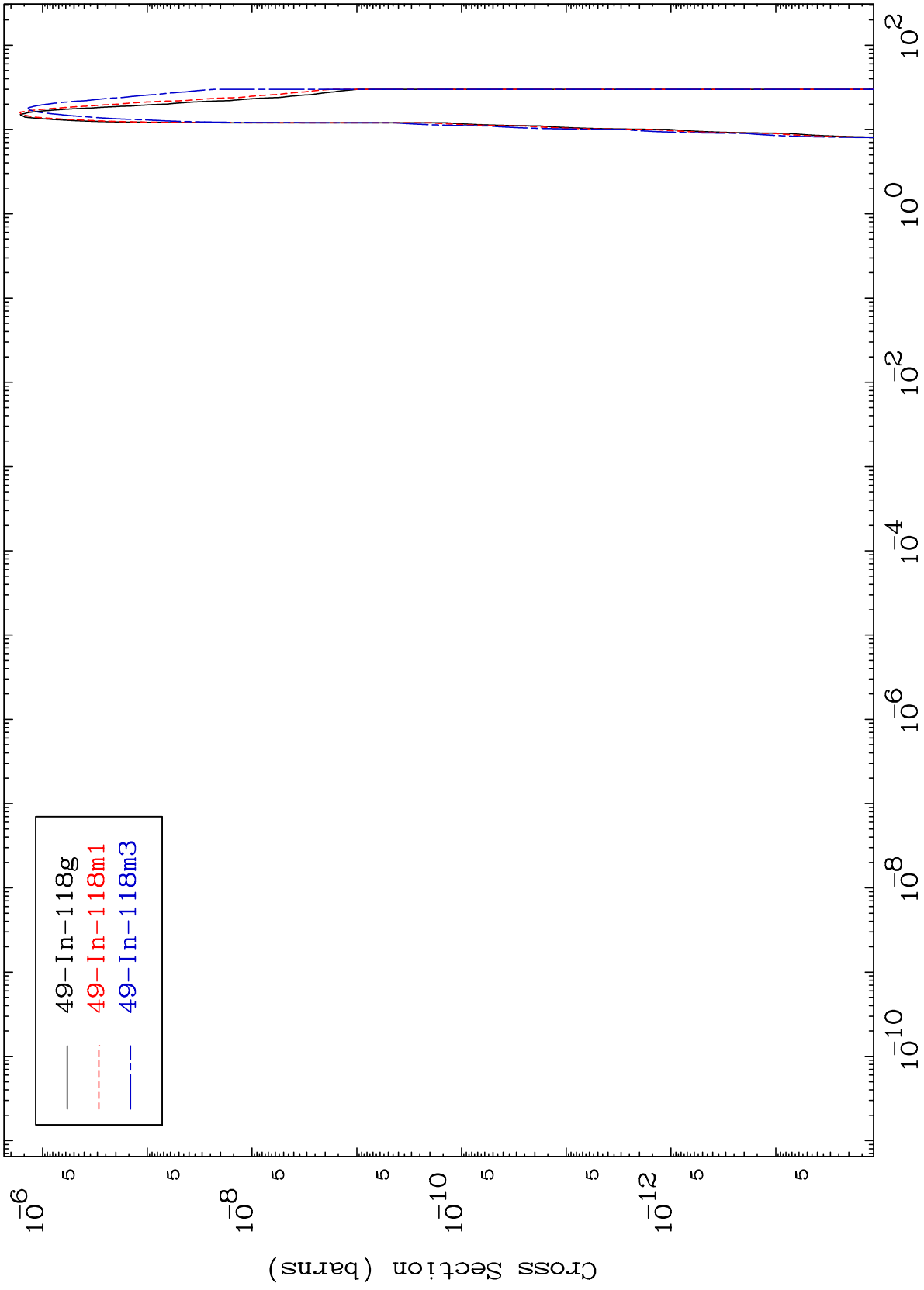
47-Ag-114

MAT 4746

( $\alpha, \gamma$ )

47-Ag-114

Radionuclide Production Cross Section



21

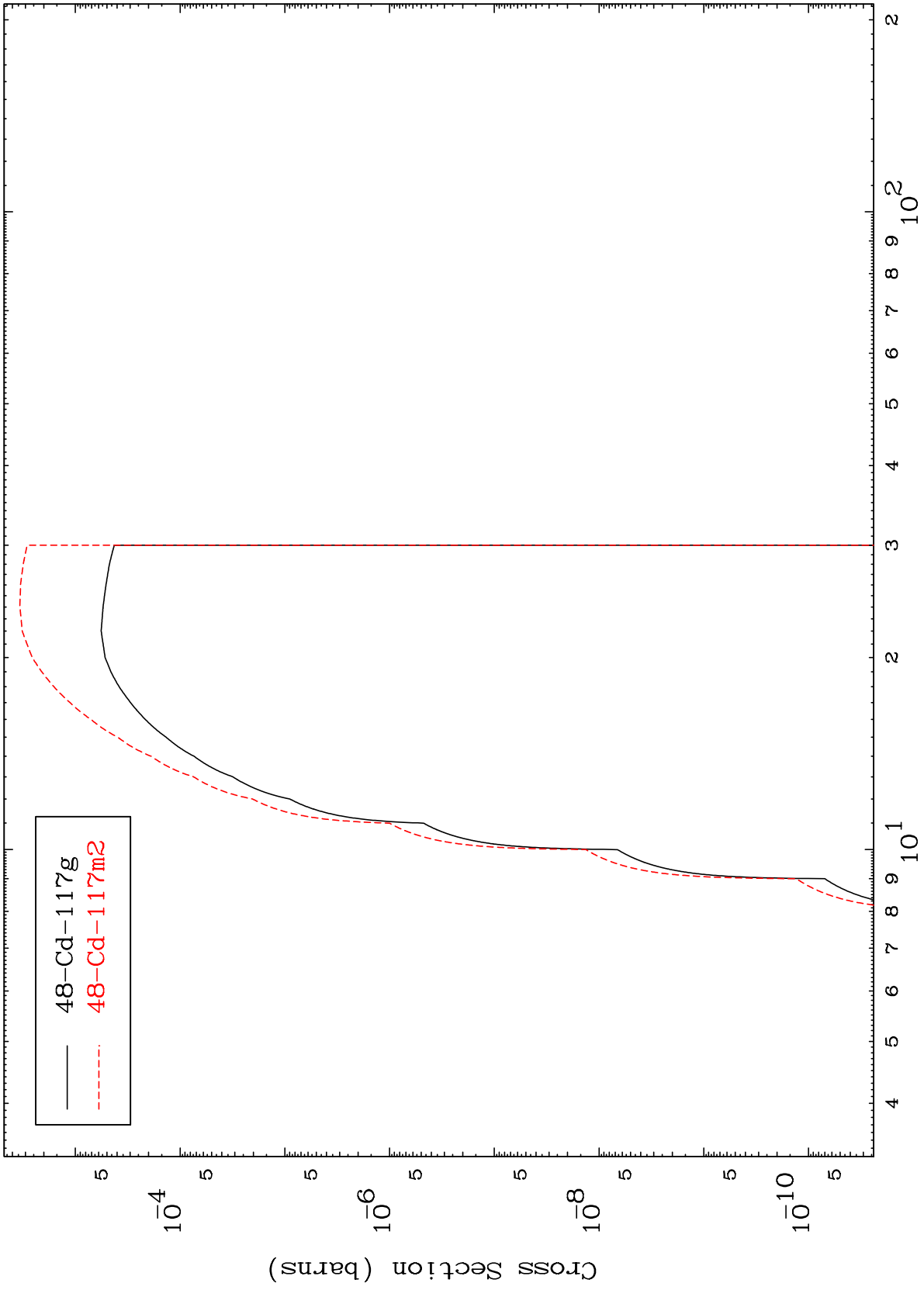
Incident Energy (MeV)

47-Ag-114

MAT 4746

47-Ag-114

( $\alpha, p$ )  
Radionuclide Production Cross Section



22

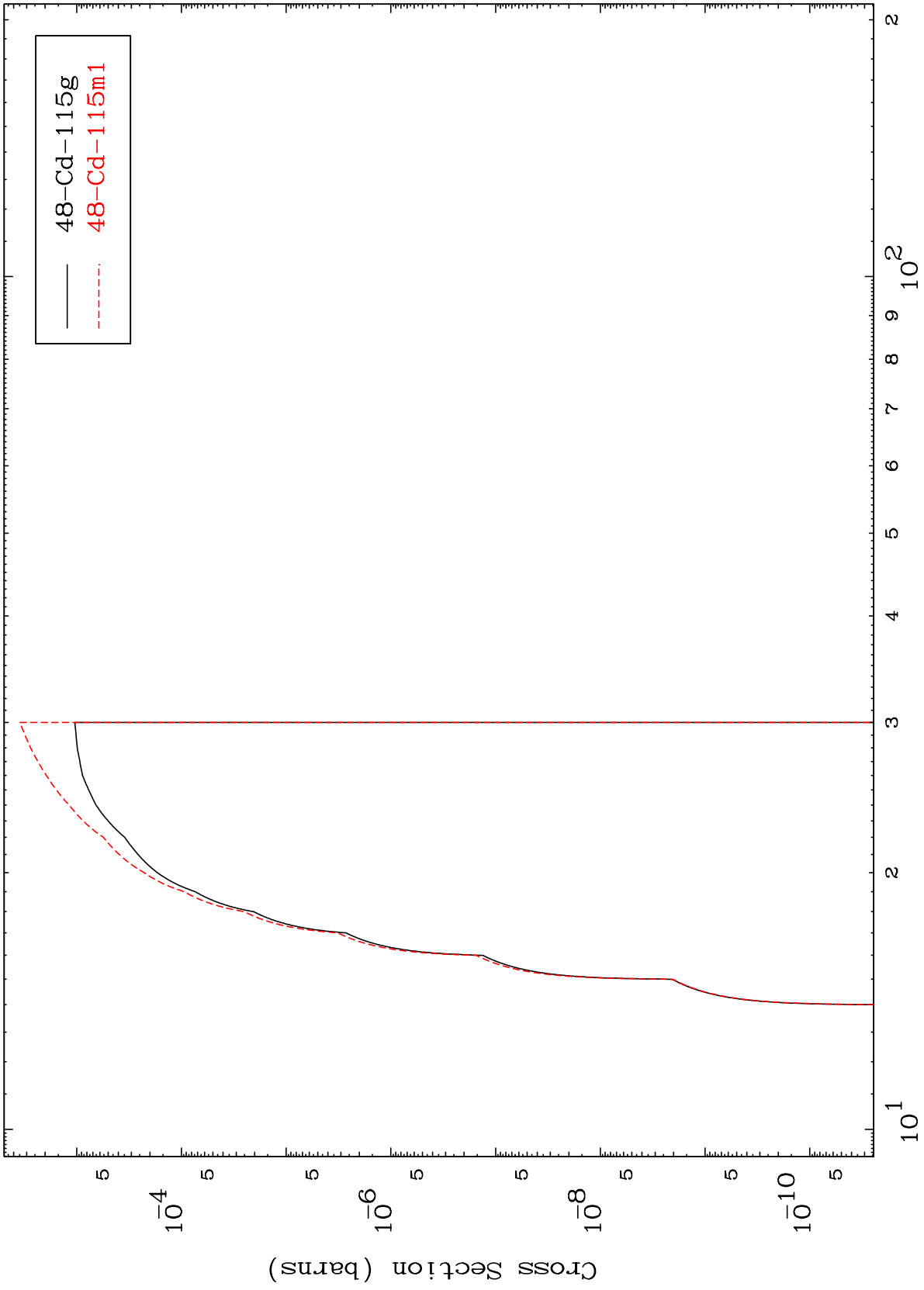
47-Ag-114

MAT 4746

( $\alpha, t$ )

47-Ag-114

Radionuclide Production Cross Section



23

Incident Energy (MeV)

47-Ag-114

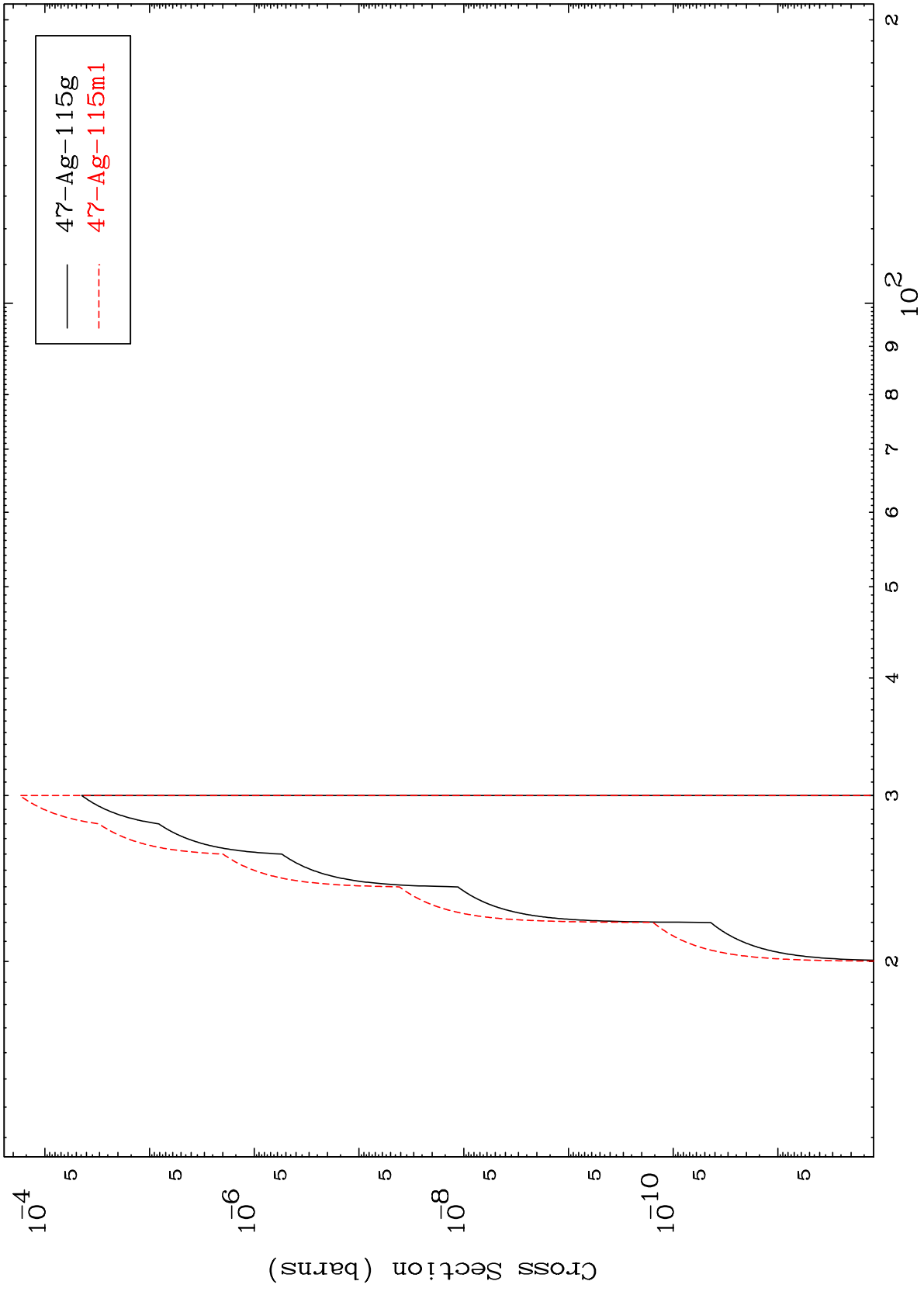


MAT 4746

( $\alpha, \text{He-3}$ )

47-Ag-114

Radionuclide Production Cross Section



24

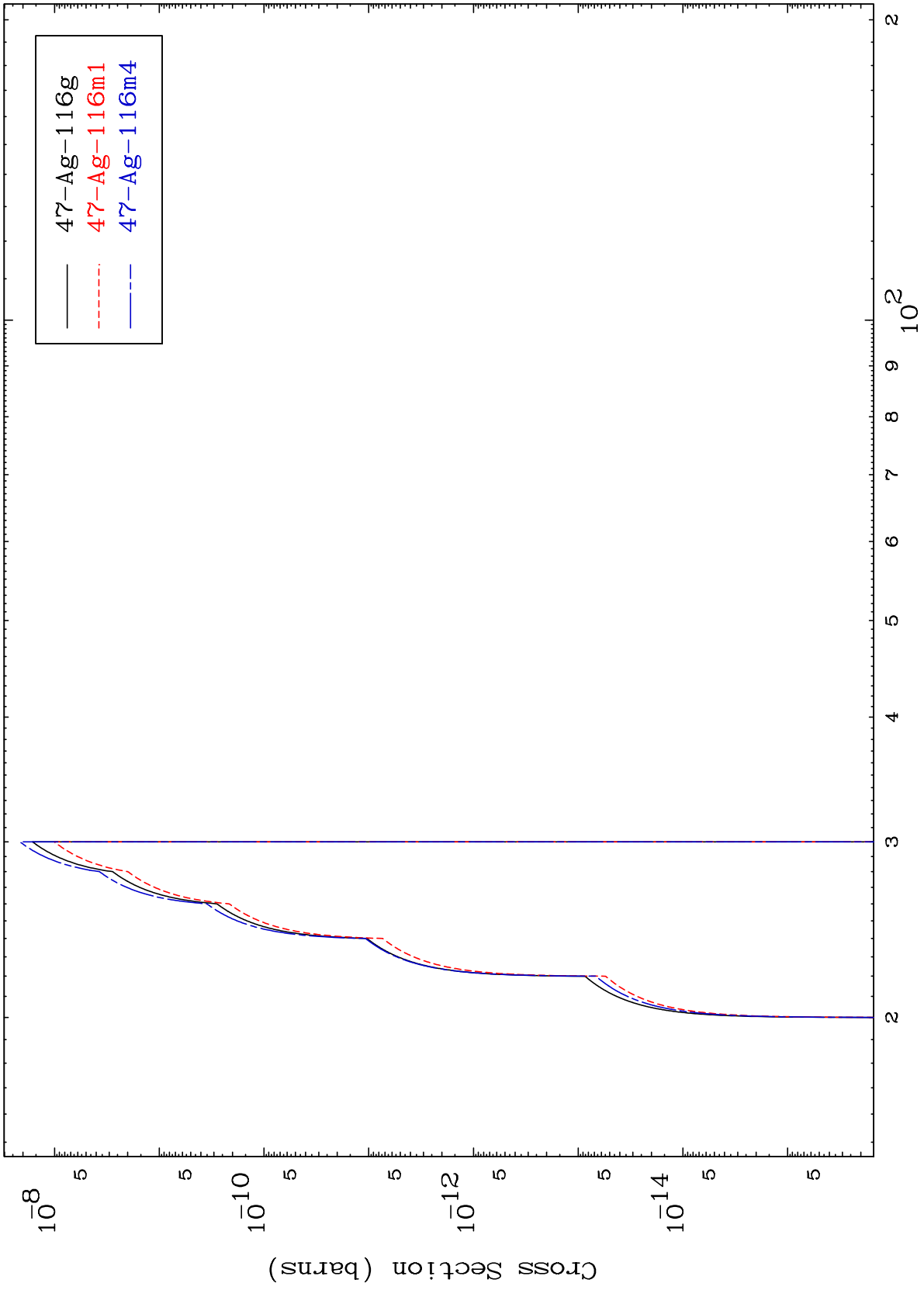
Incident Energy (MeV)

47-Ag-114

MAT 4746

47-Ag-114

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



25

Incident Energy (MeV)

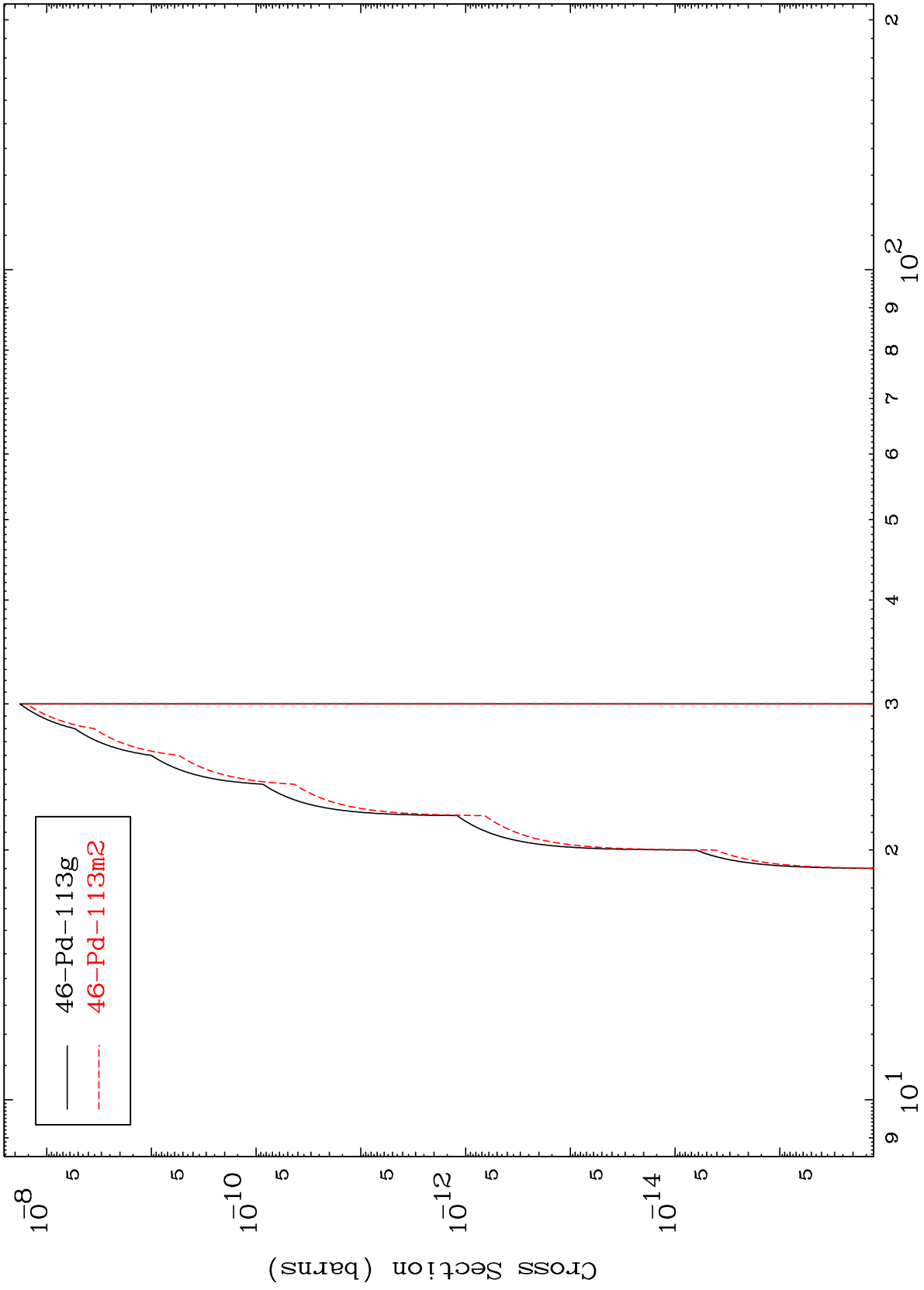
47-Ag-114

MAT 4746

( $\alpha, p$ )  $\alpha$

47-Ag-114

Radionuclide Production Cross Section



26

Incident Energy (MeV)

47-Ag-114