

Program EVALPLOT  
(Version 2018-1)

by

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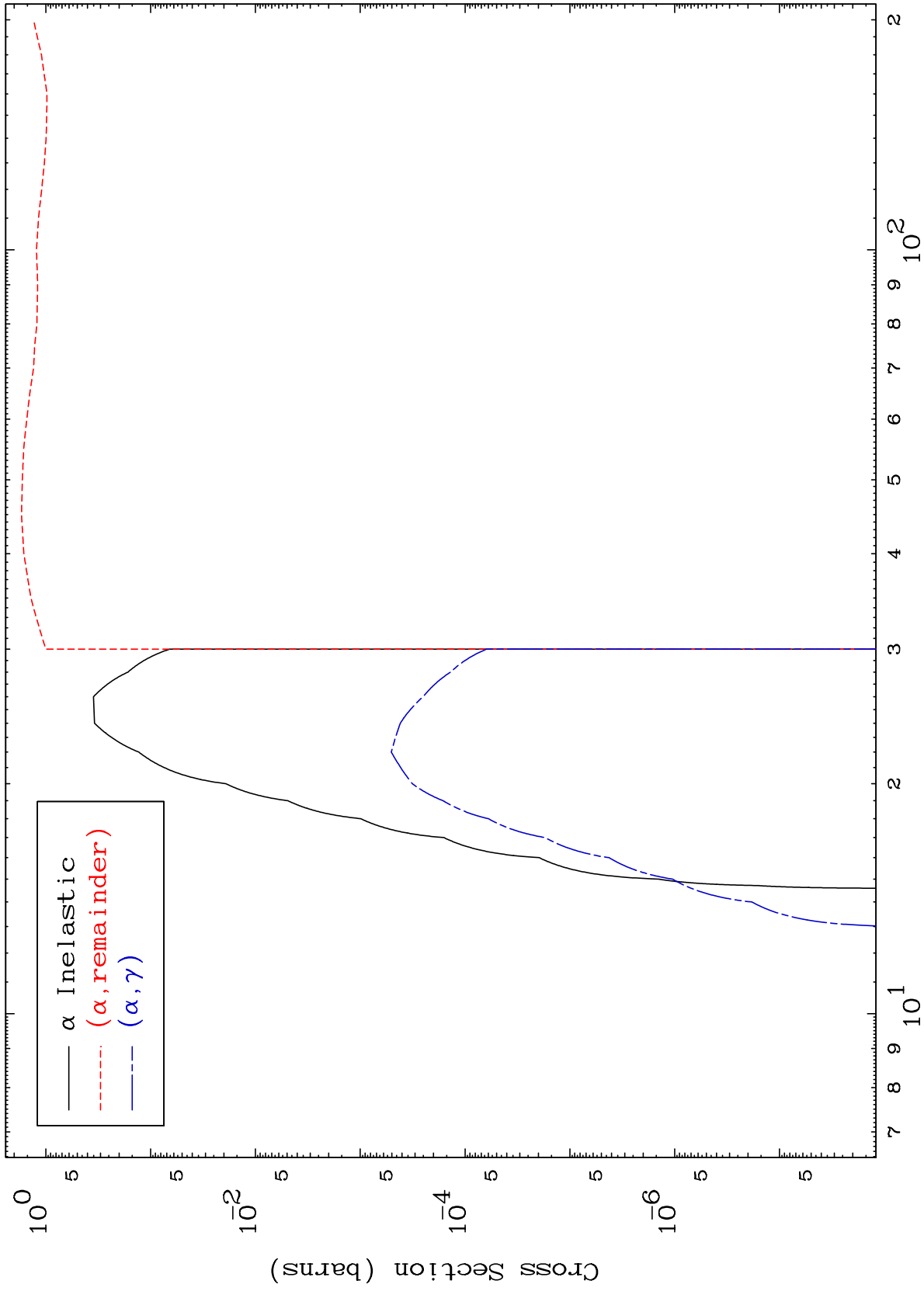
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8534

0 Kelvin  $\alpha$  Major  
Cross Sections

85-At-206



1

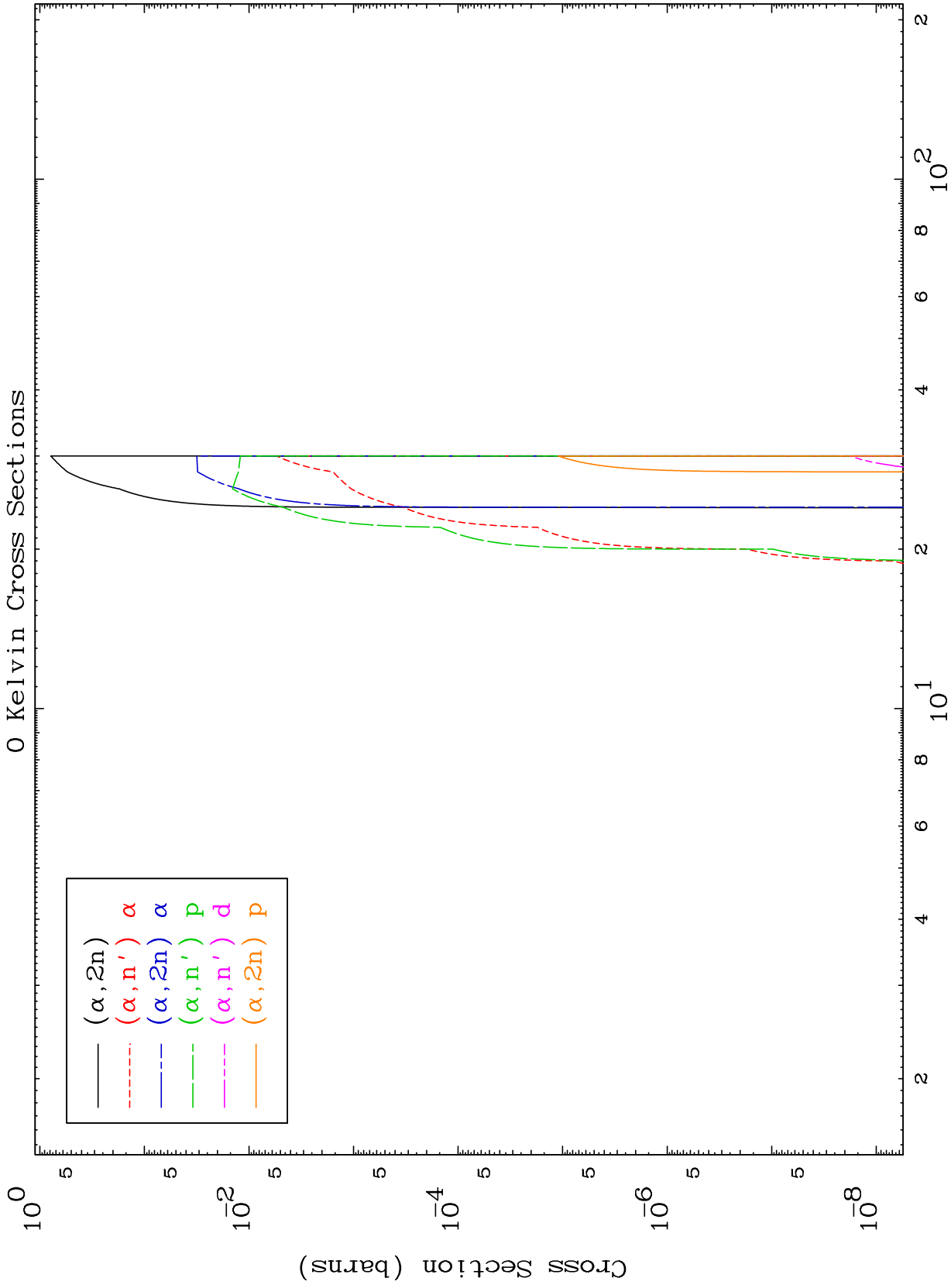
Incident Energy (MeV)

85-At-206

MAT 8534

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

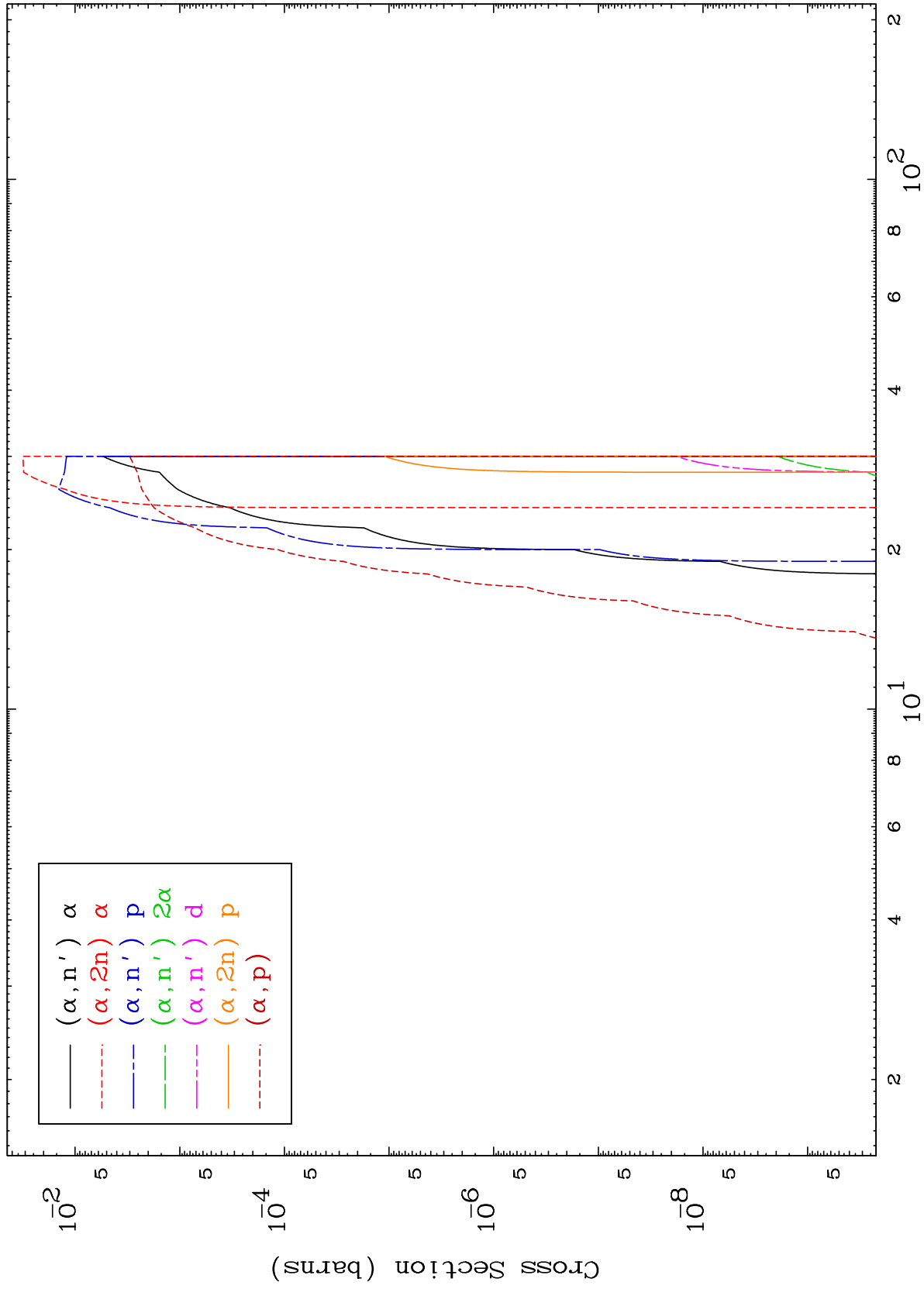
85-At-206



2

Incident Energy (MeV)

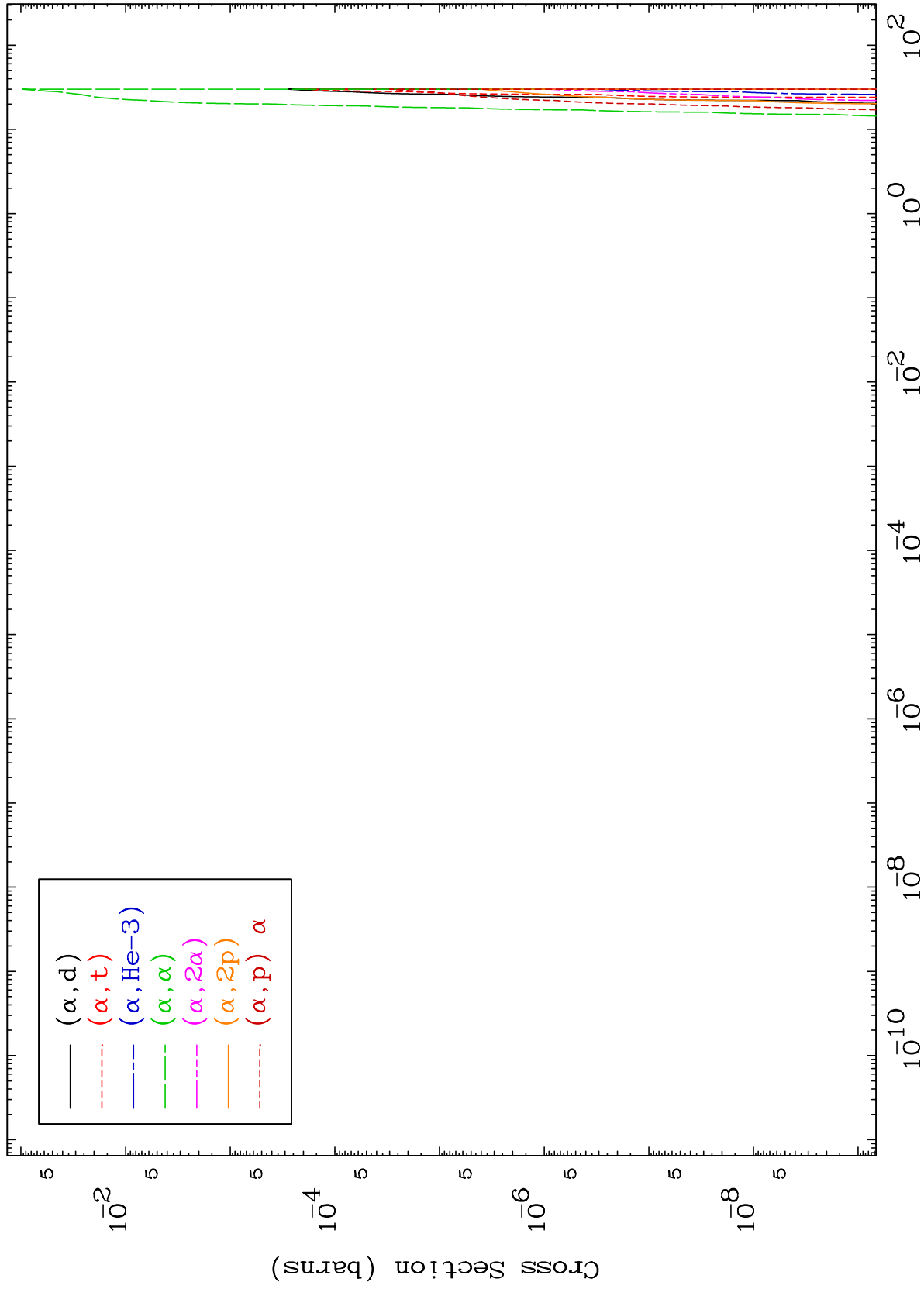
85-At-206



MAT 8534

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

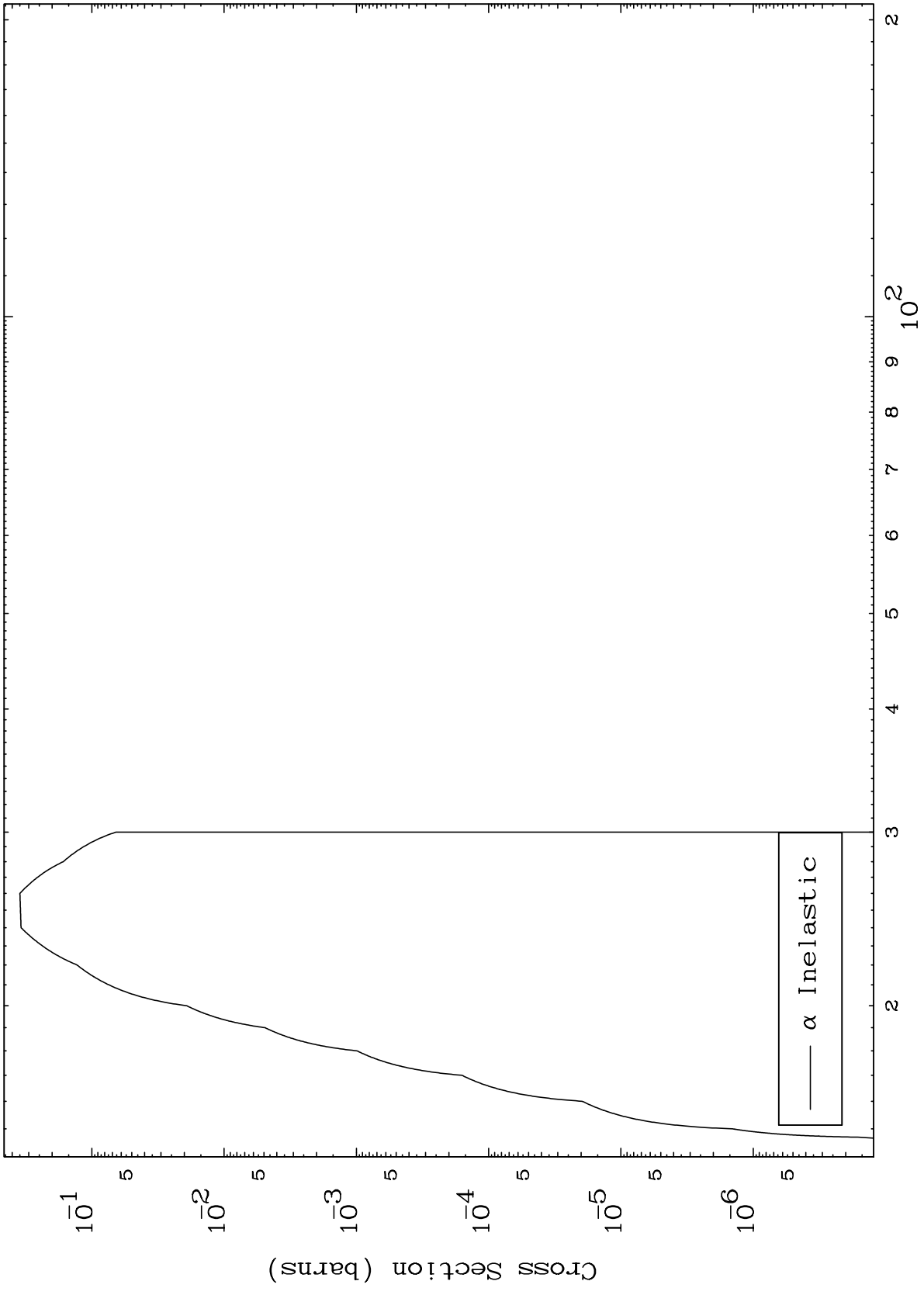
85-At-206



4

Incident Energy (MeV)

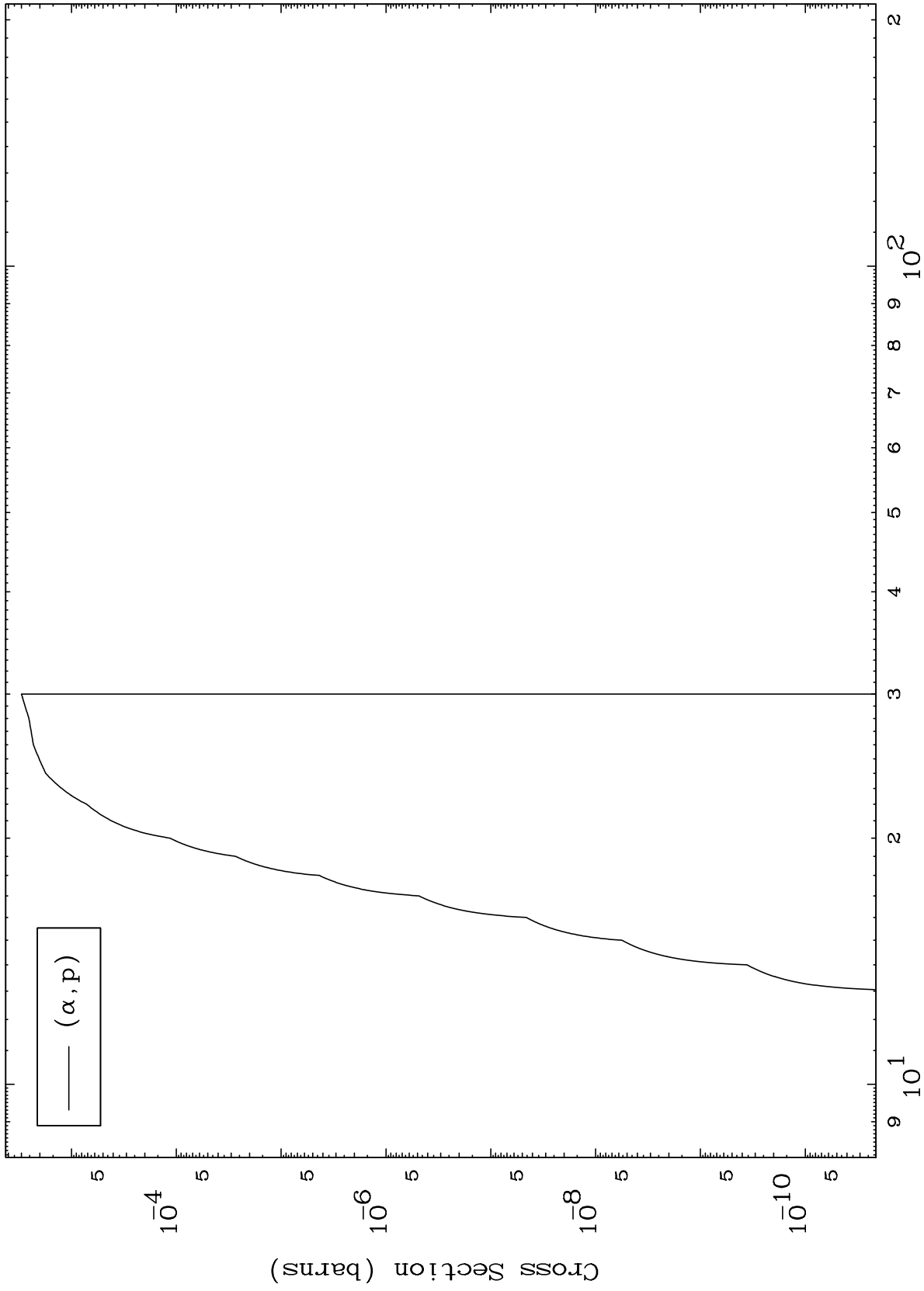
85-At-206



MAT 8534

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

85-At-206



6

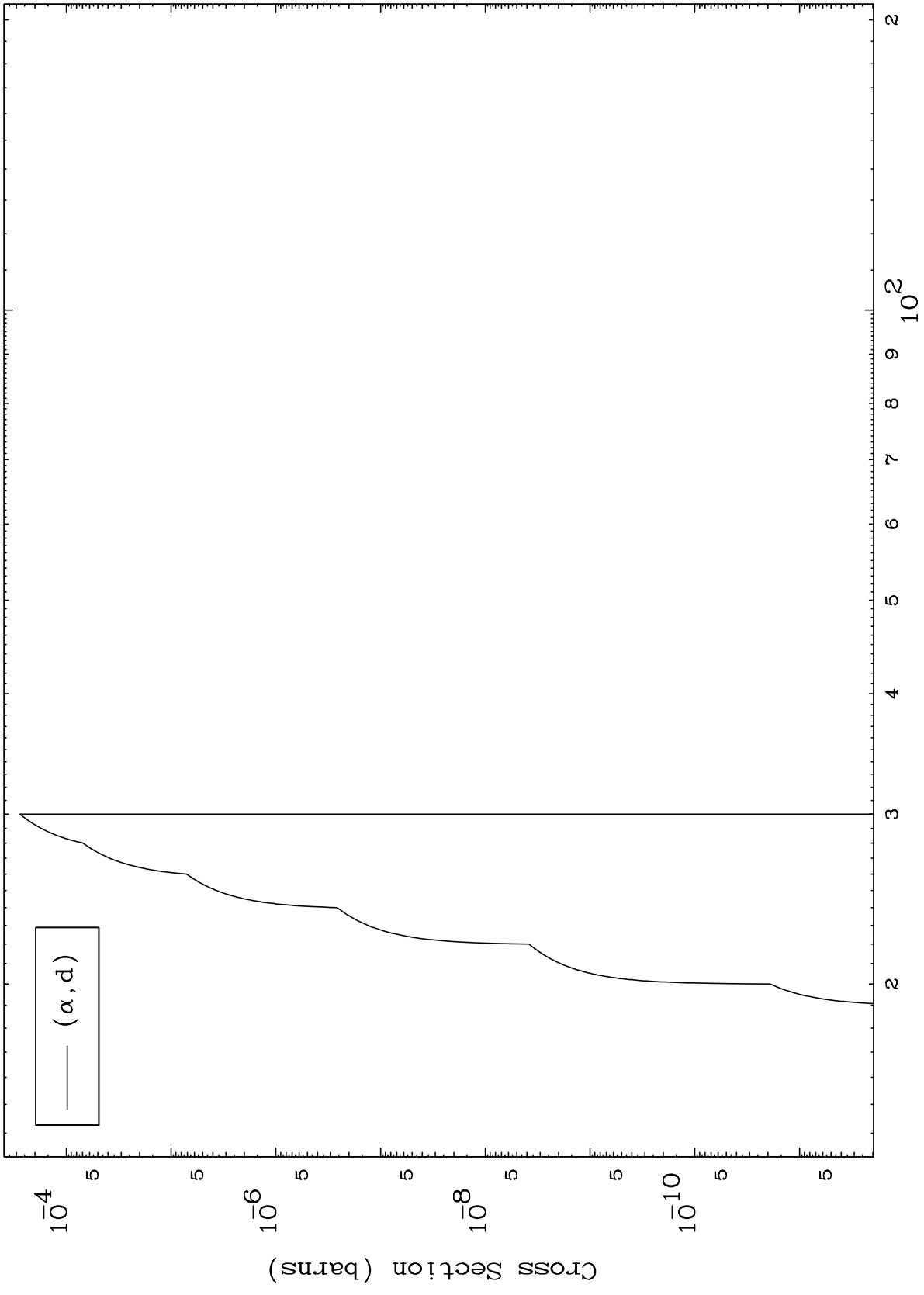
Incident Energy (MeV)

85-At-206

MAT 8534

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

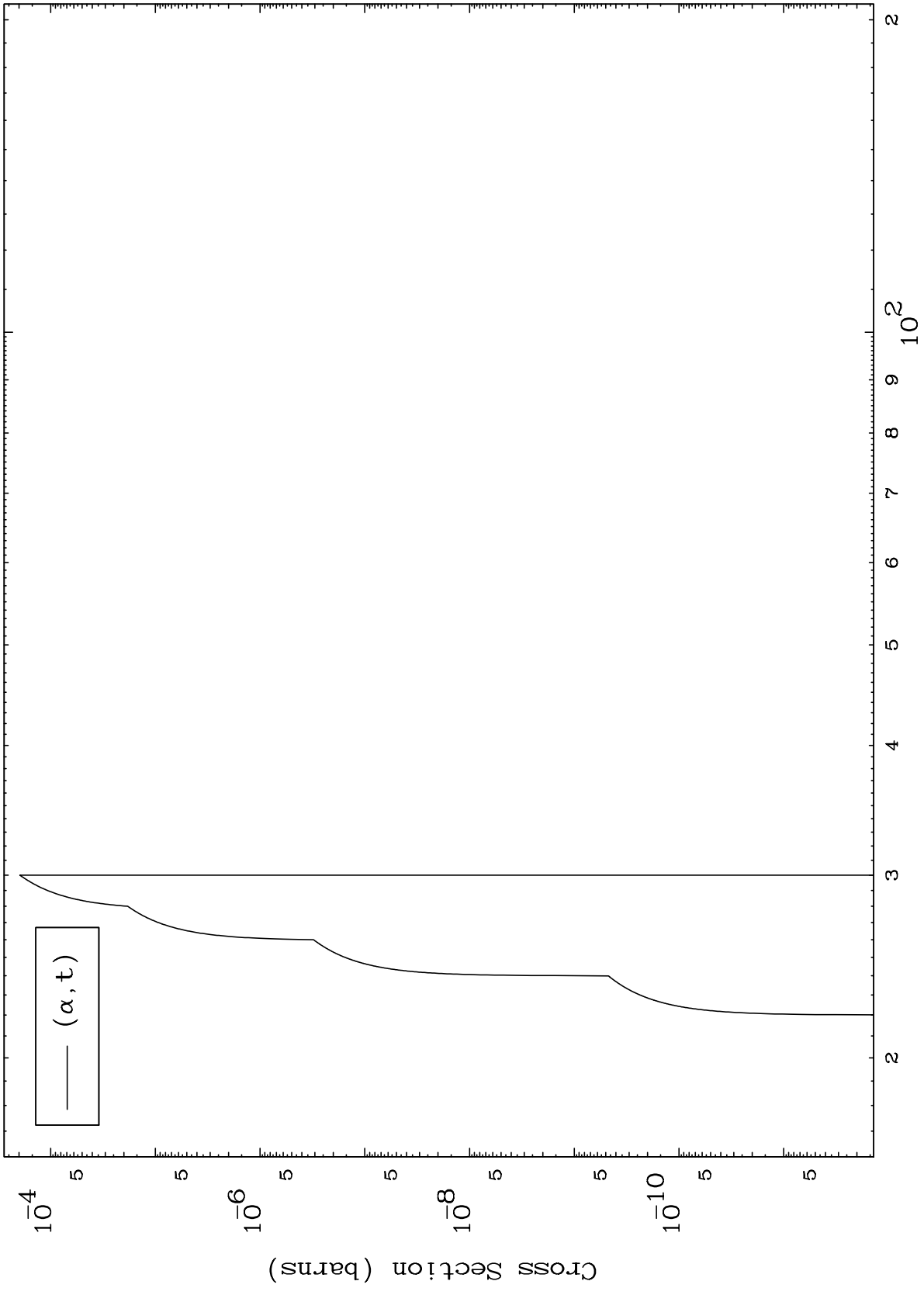
85-At-206

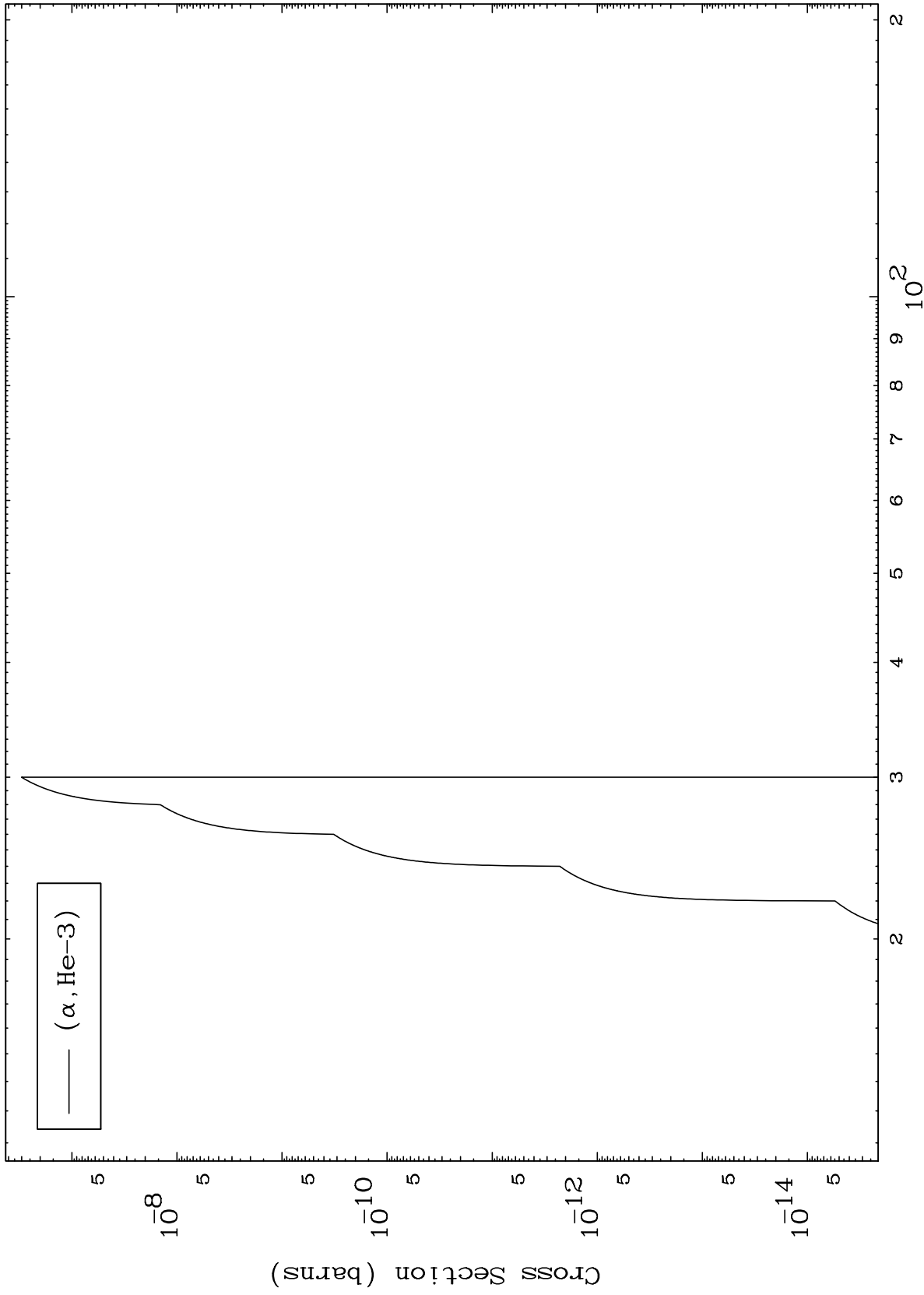


7

85-At-206



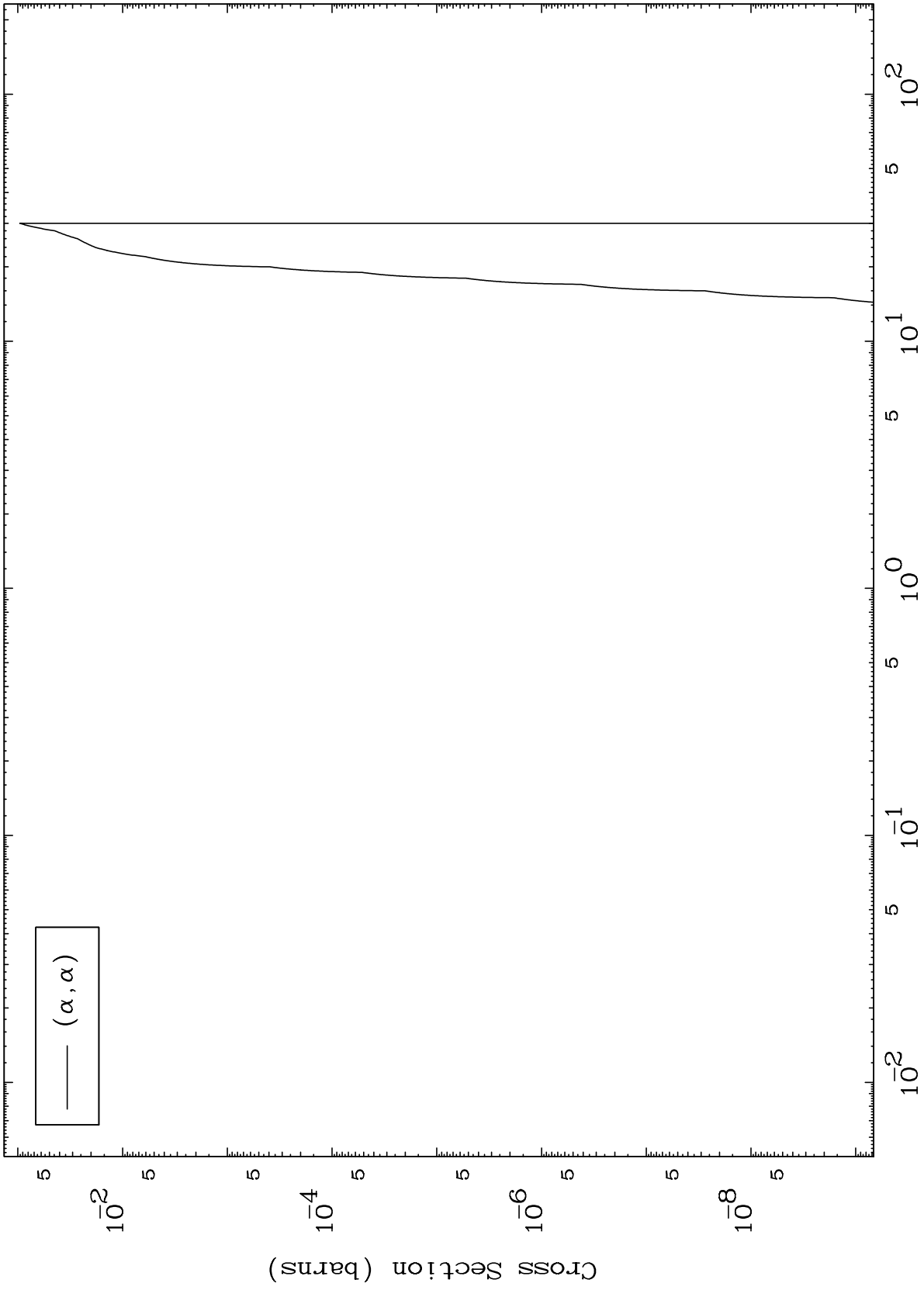




MAT 8534

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

85-At-206



10

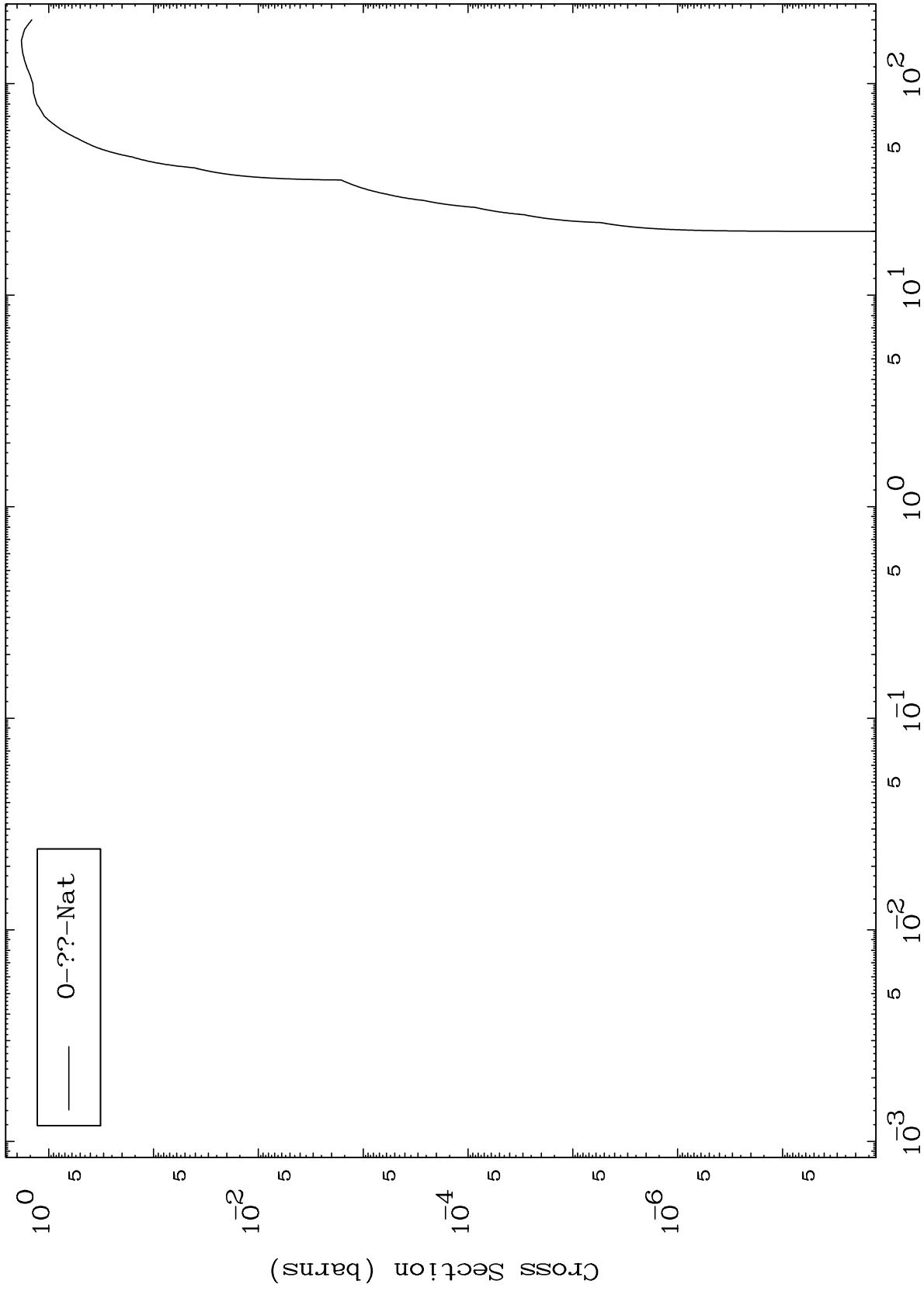
Incident Energy (MeV)

85-At-206

MAT 8534

85-At-206

$\alpha$  Fission  
Radionuclide Production Cross Section



11

Incident Energy (MeV)

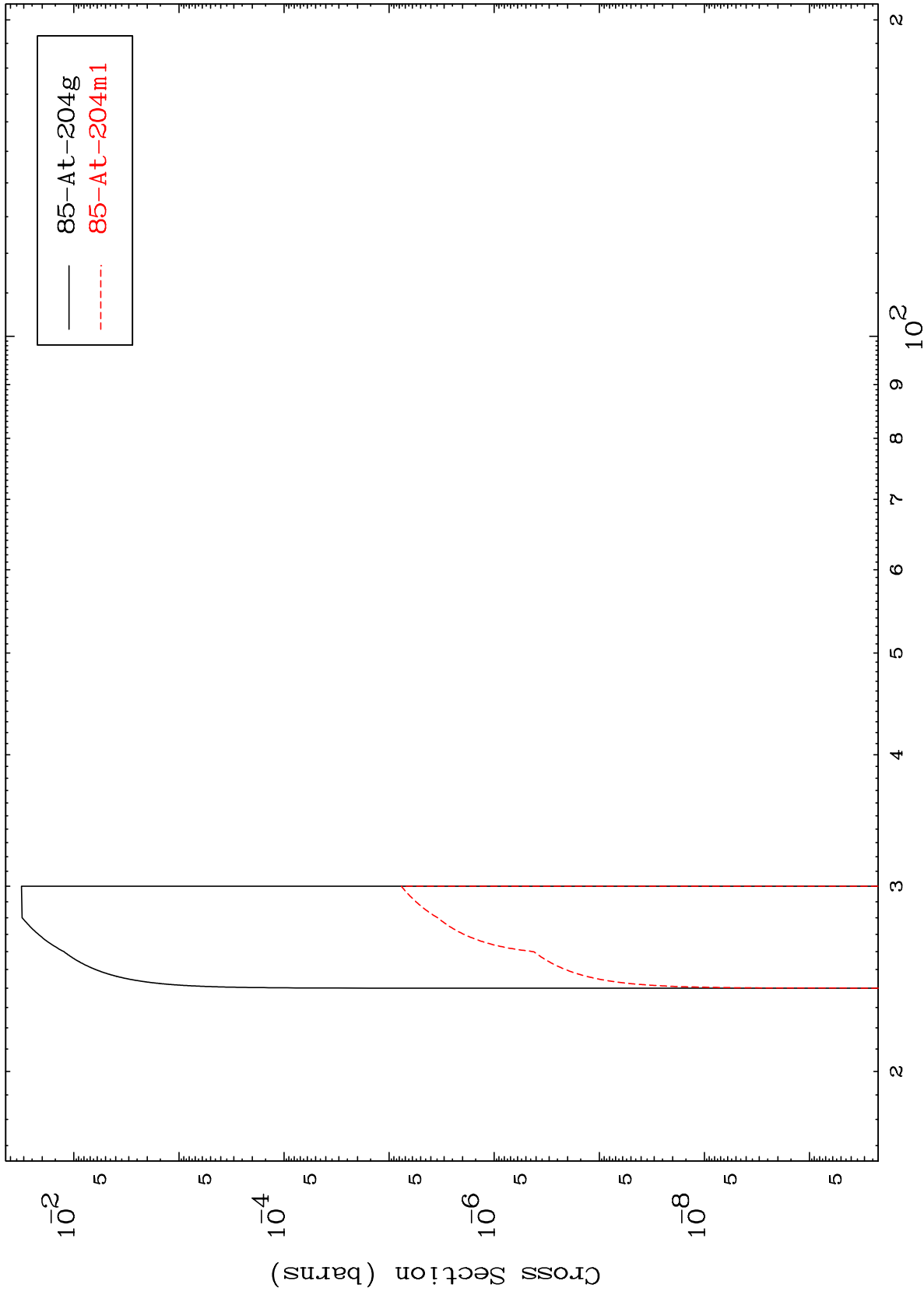
85-At-206

MAT 8534

( $\alpha, 2n$ )  $\alpha$

85-At-206

Radionuclide Production Cross Section



12

Incident Energy (MeV)

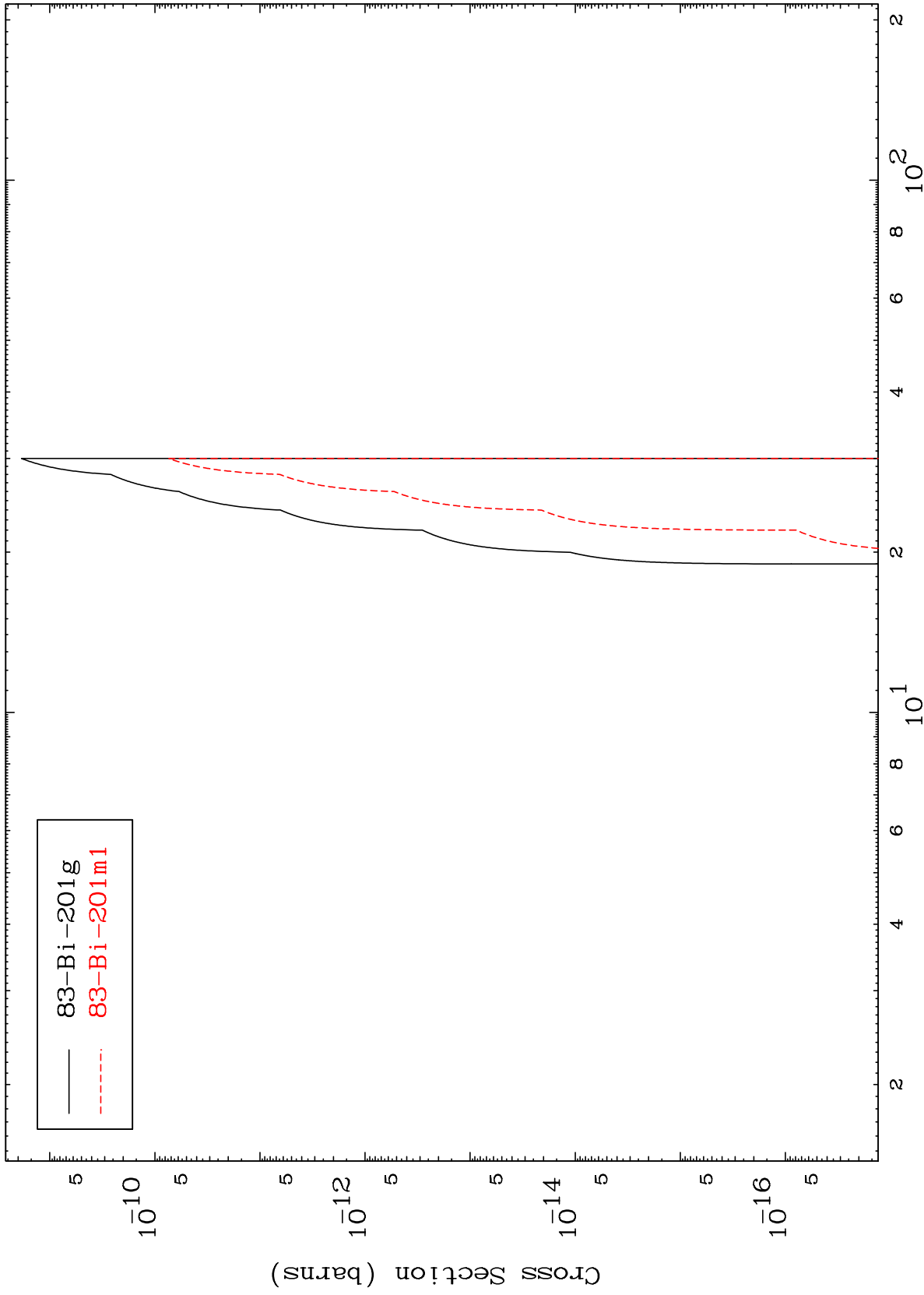
85-At-206

MAT 8534

( $\alpha, n'$ )  $2\alpha$

85-At-206

Radionuclide Production Cross Section



13

Incident Energy (MeV)

85-At-206