

Program EVALPLOT  
(Version 2018-1)

by

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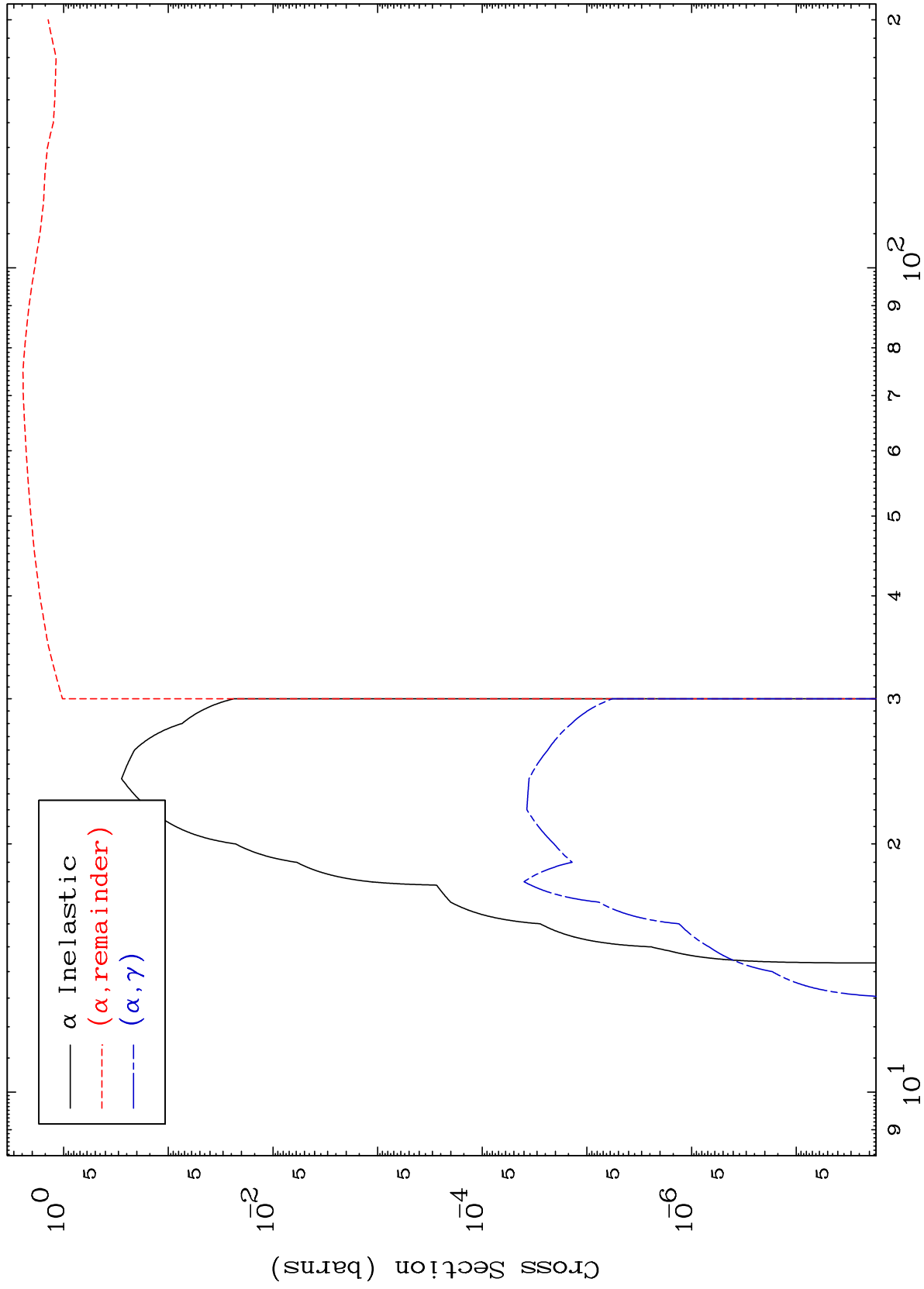
Press Mouse Button to Start

MAT 8546

$\alpha$  Major

85-At-210

0 Kelvin Cross Sections



1

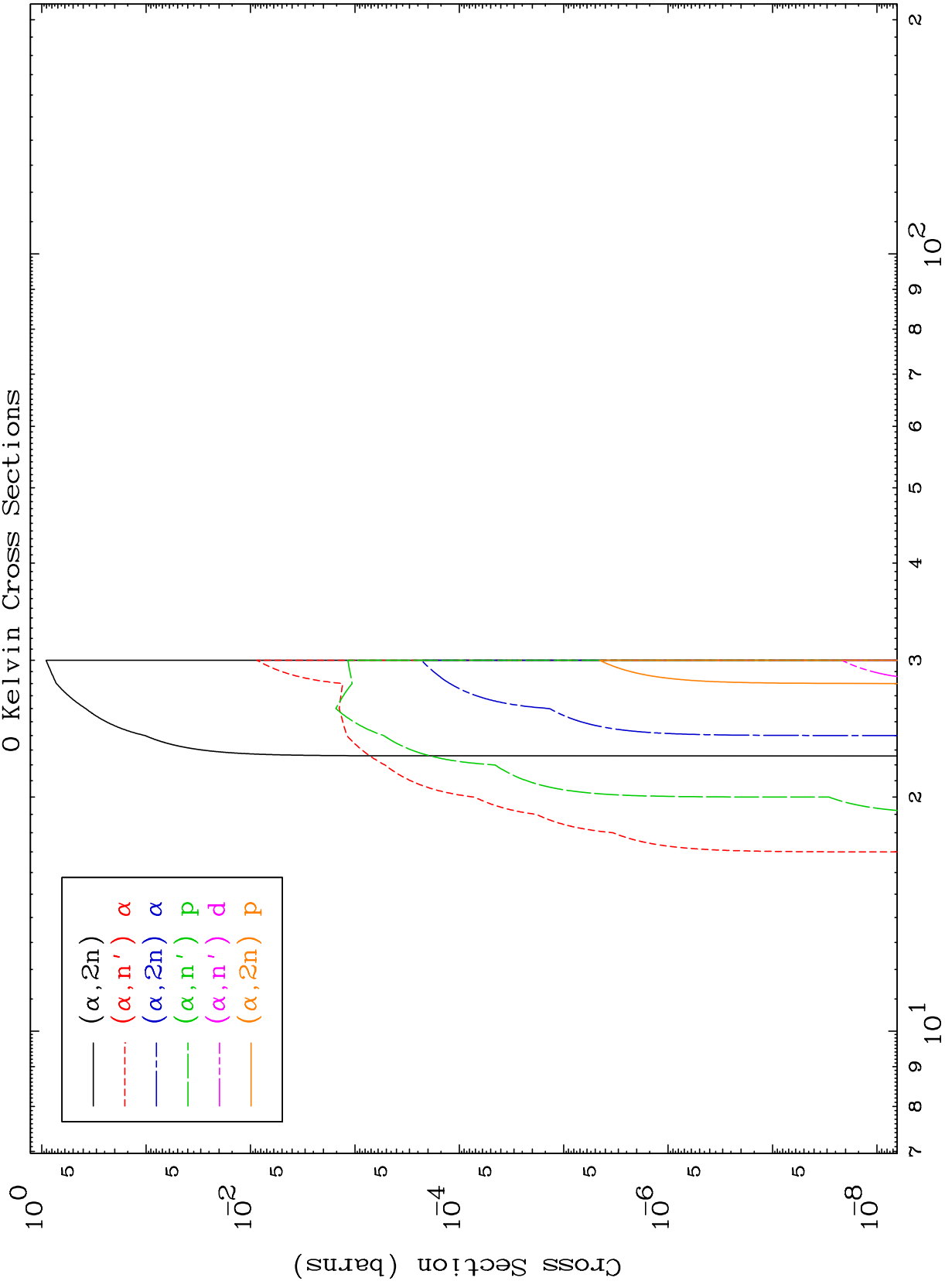
Incident Energy (MeV)

85-At-210

MAT 8546

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

85-At-210



2

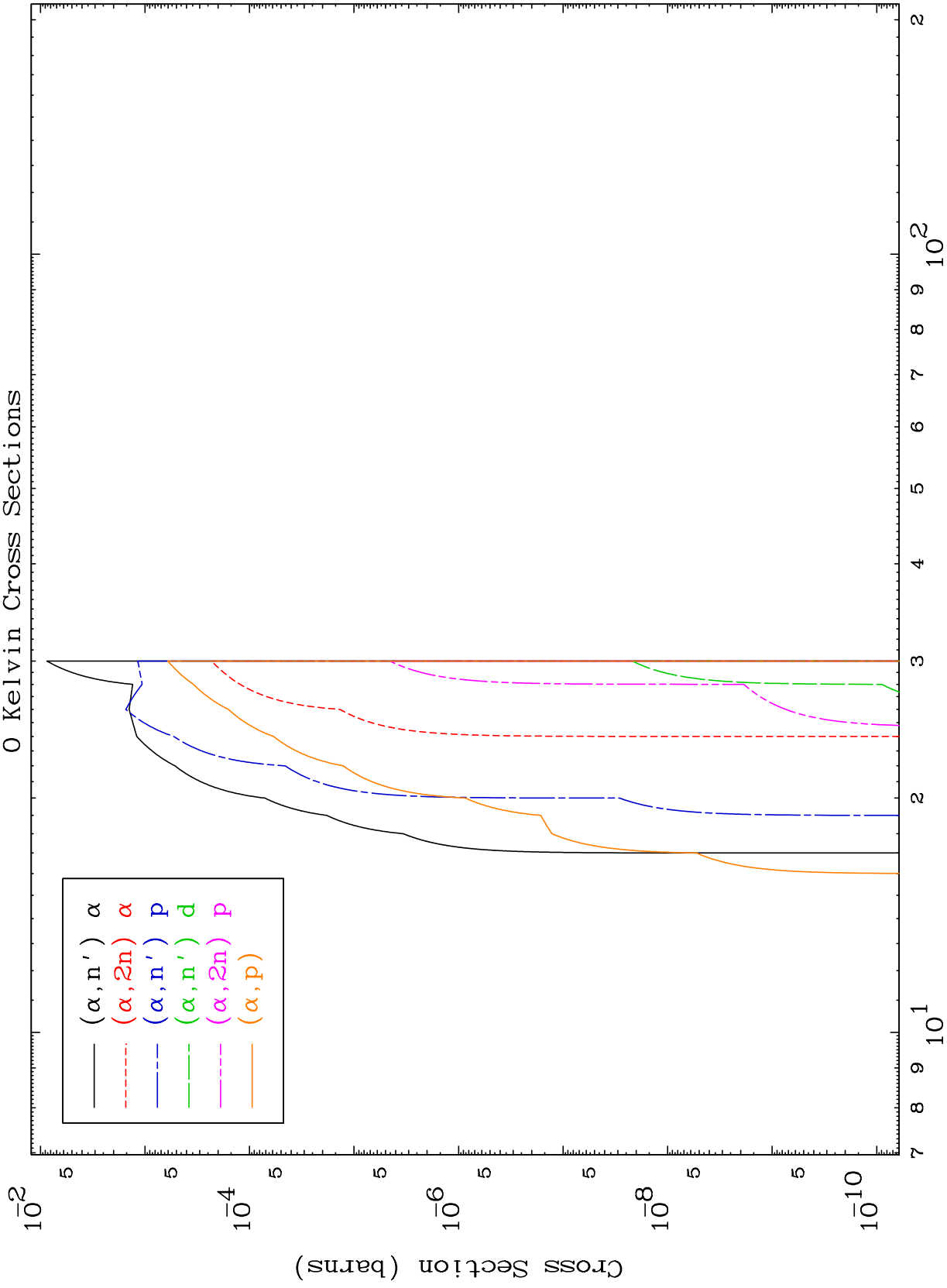
Incident Energy (MeV)

85-At-210

MAT 8546

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

85-At-210



3

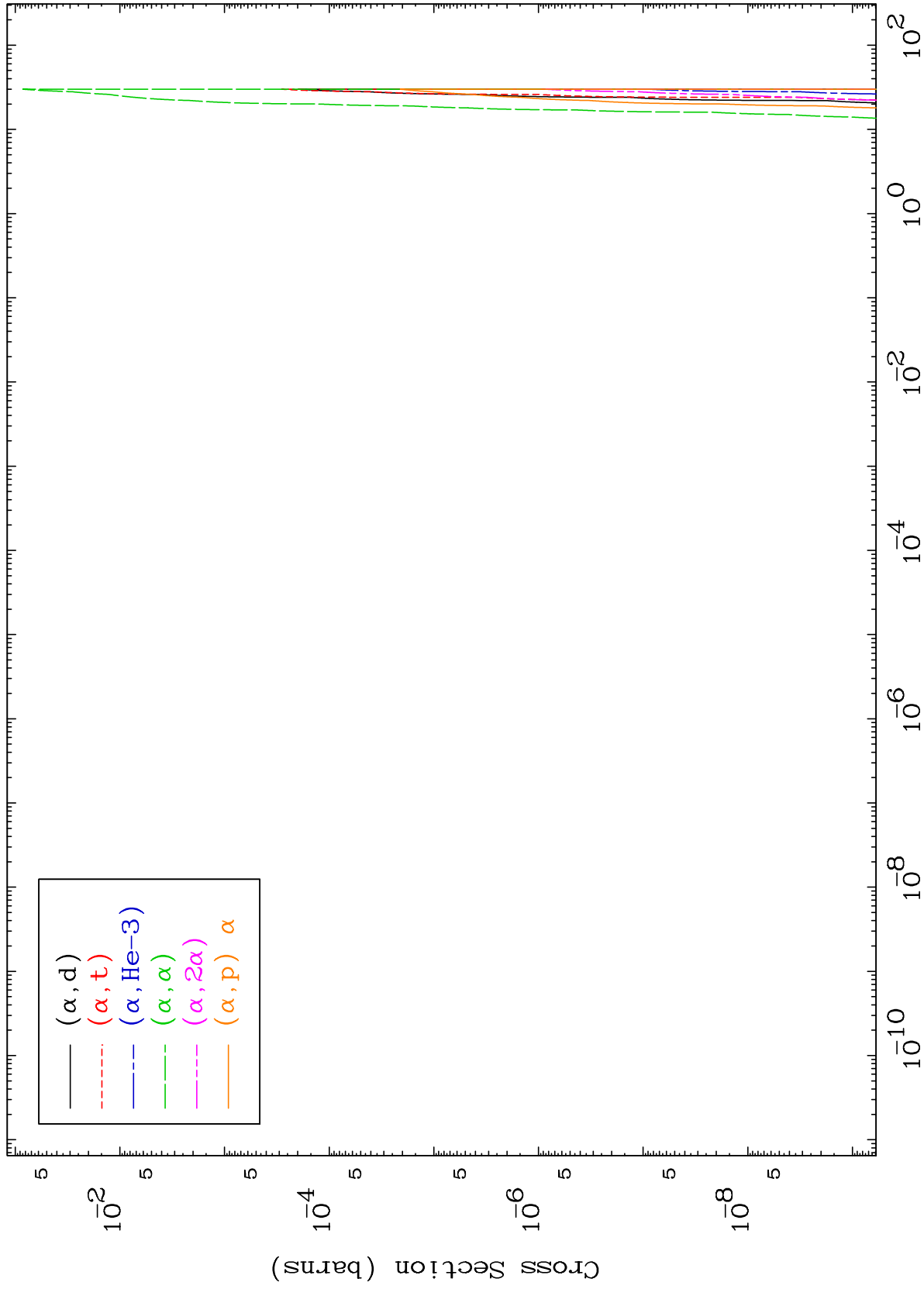
Incident Energy (MeV)

85-At-210

MAT 8546

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

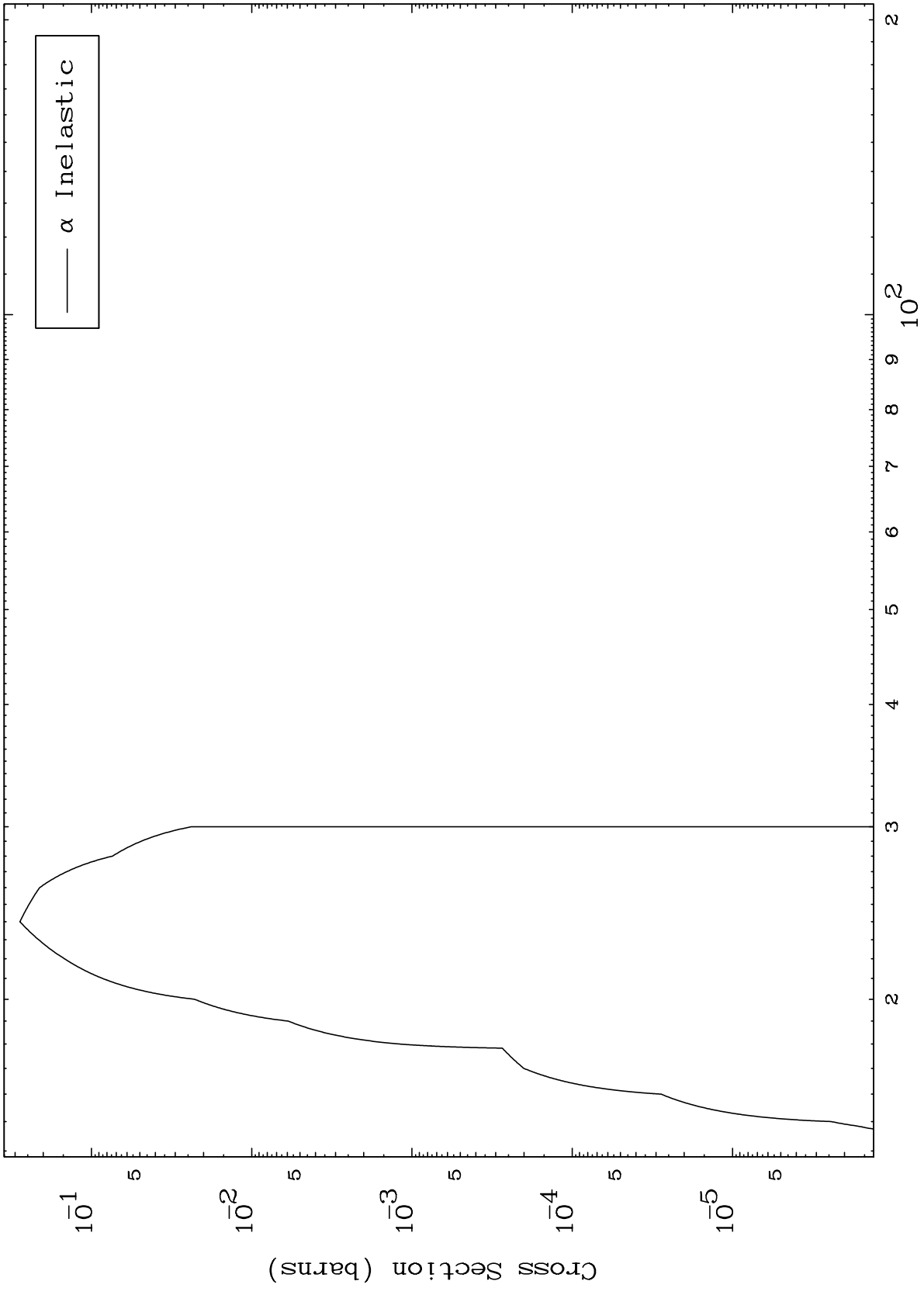
85-At-210



MAT 8546

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

85-At-210



5

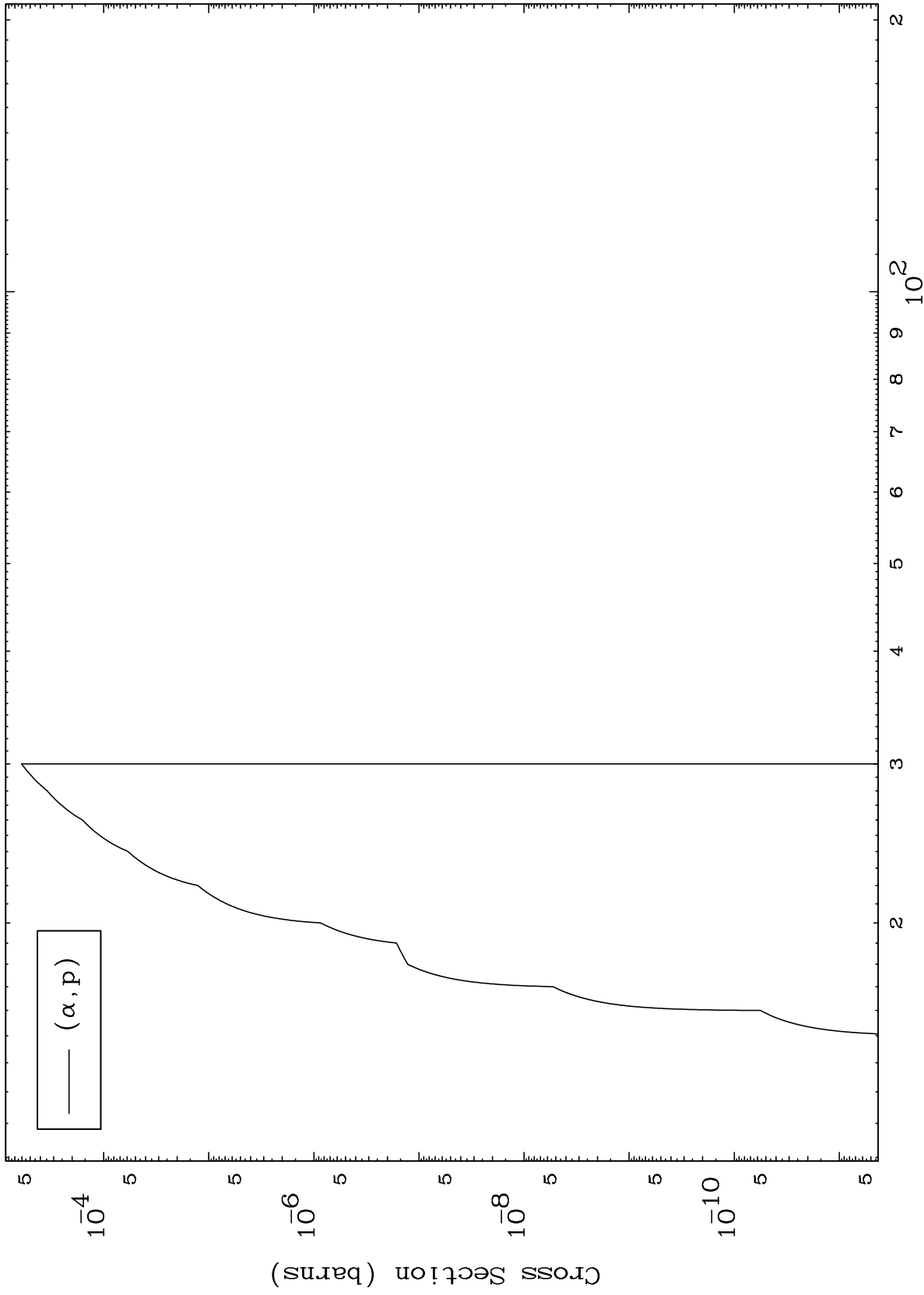
Incident Energy (MeV)

85-At-210

MAT 8546

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

85-At-210



6

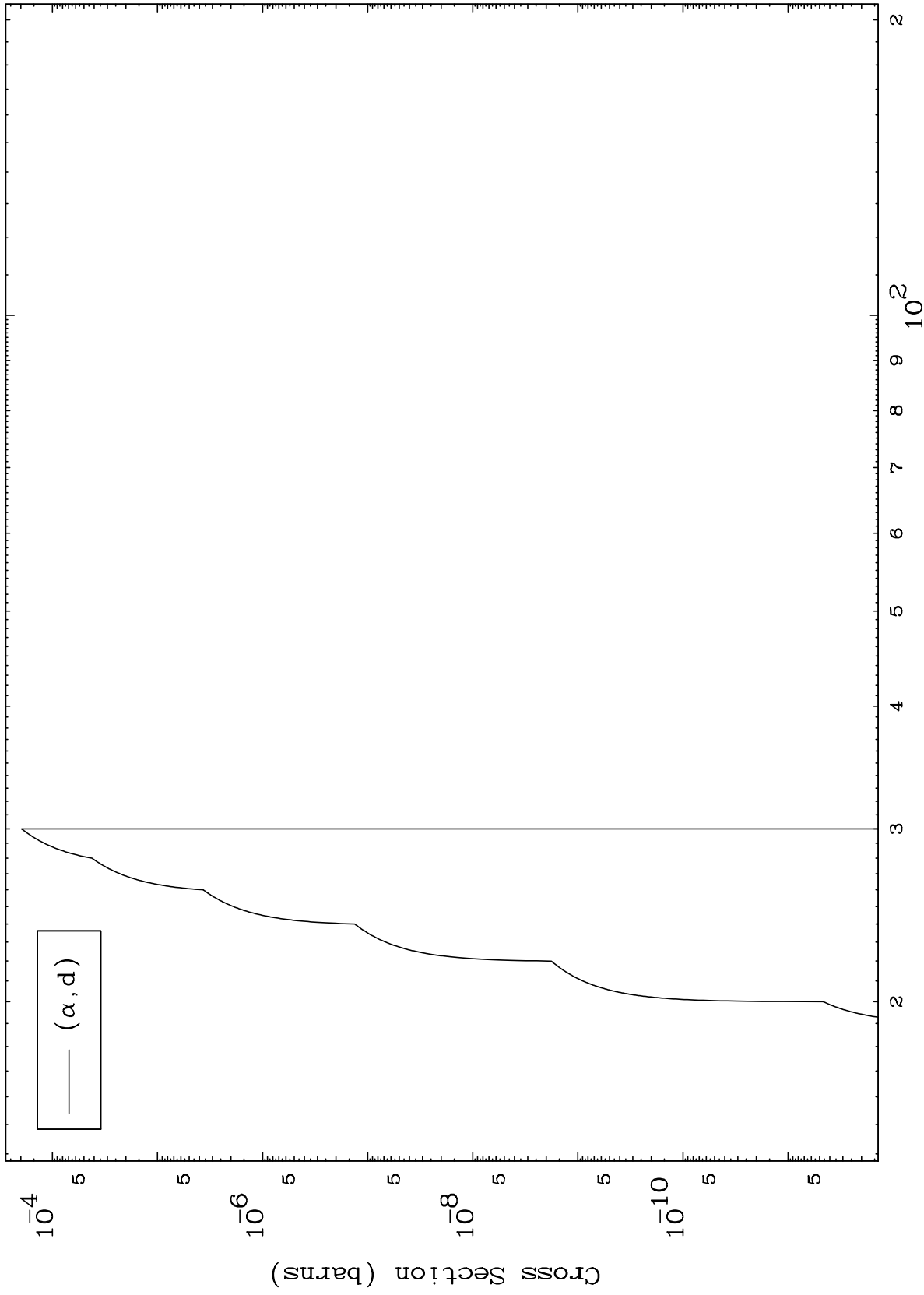
Incident Energy (MeV)

85-At-210

MAT 8546

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

85-At-210



7

Incident Energy (MeV)

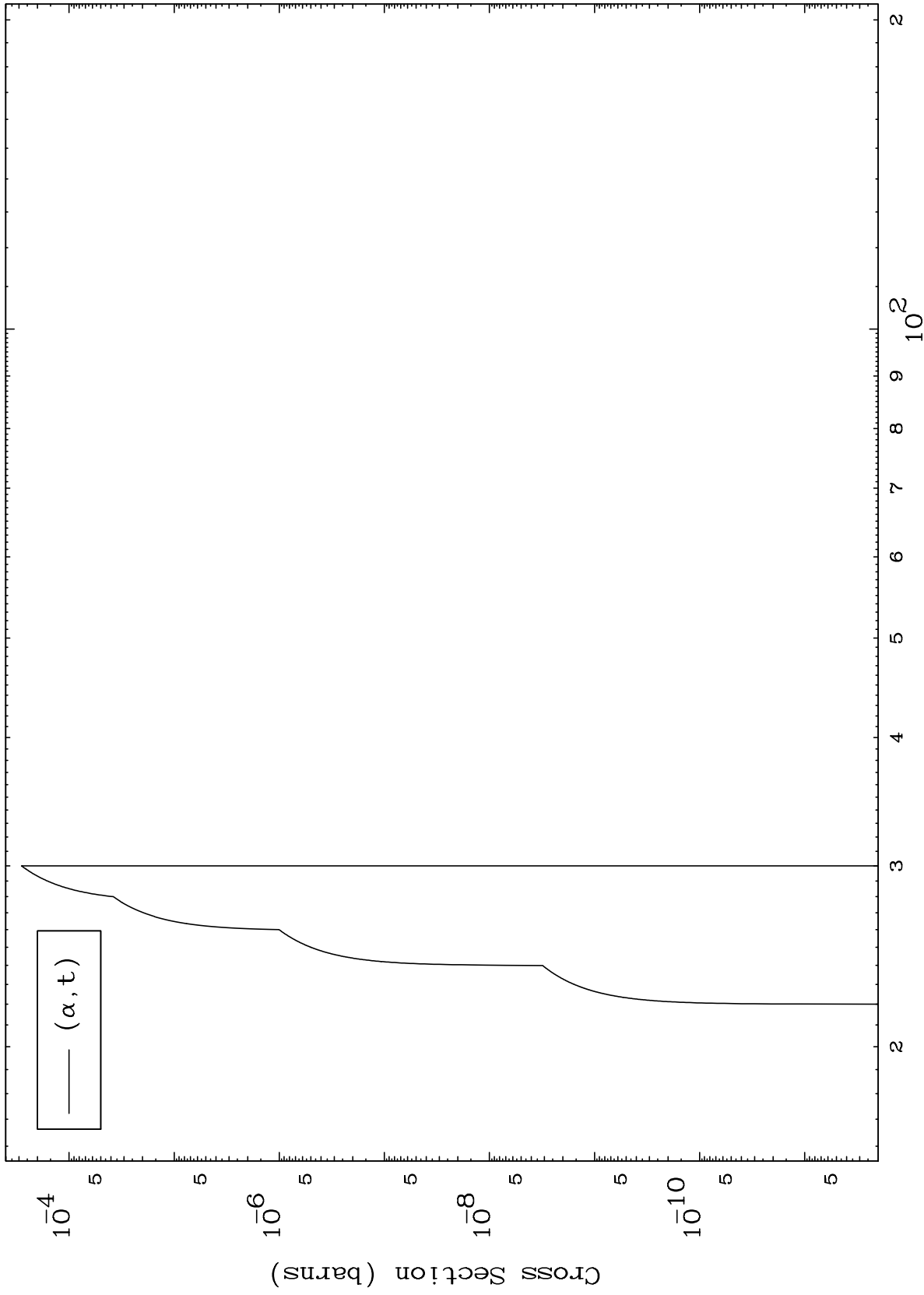
85-At-210



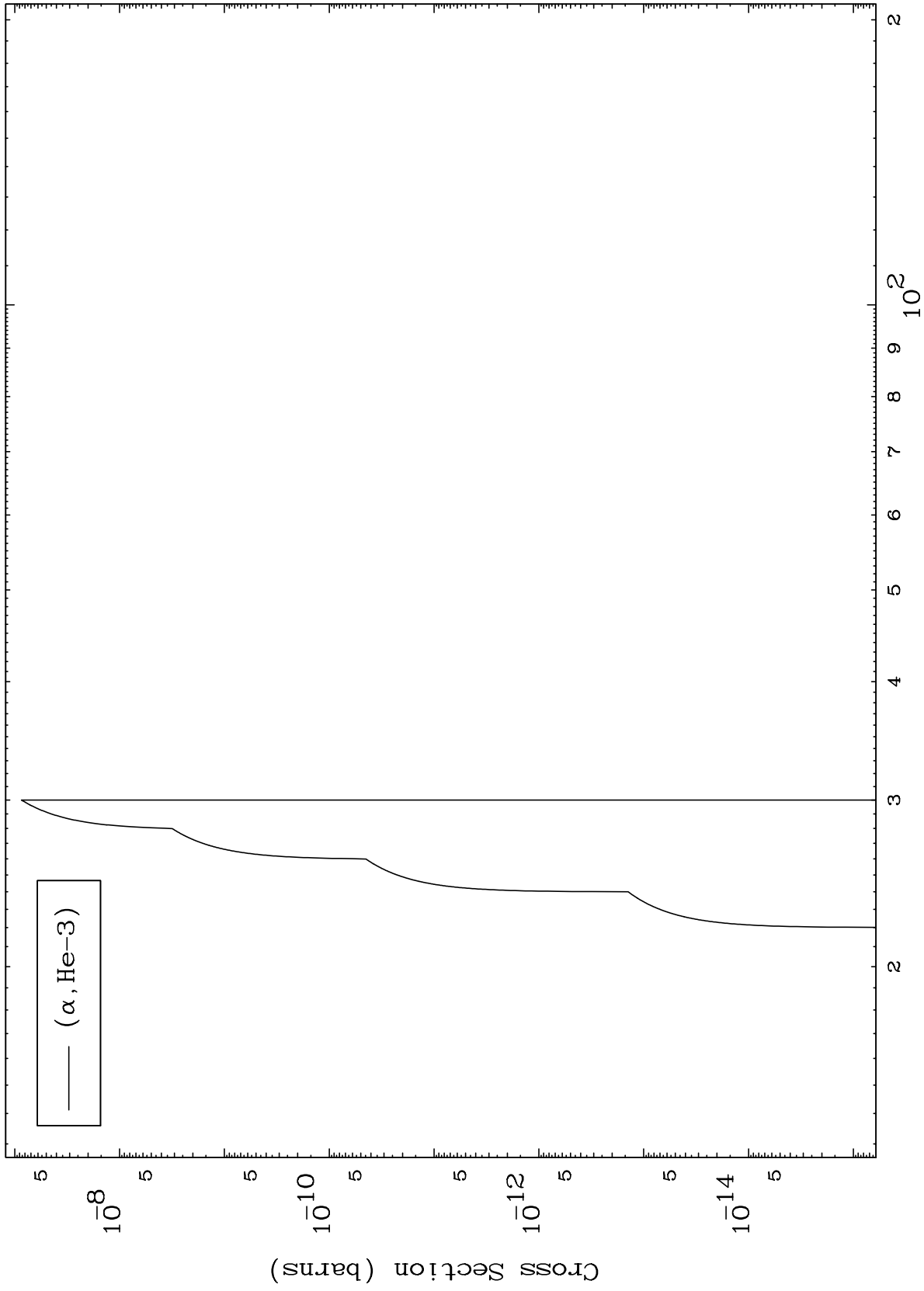
MAT 8546

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

85-At-210



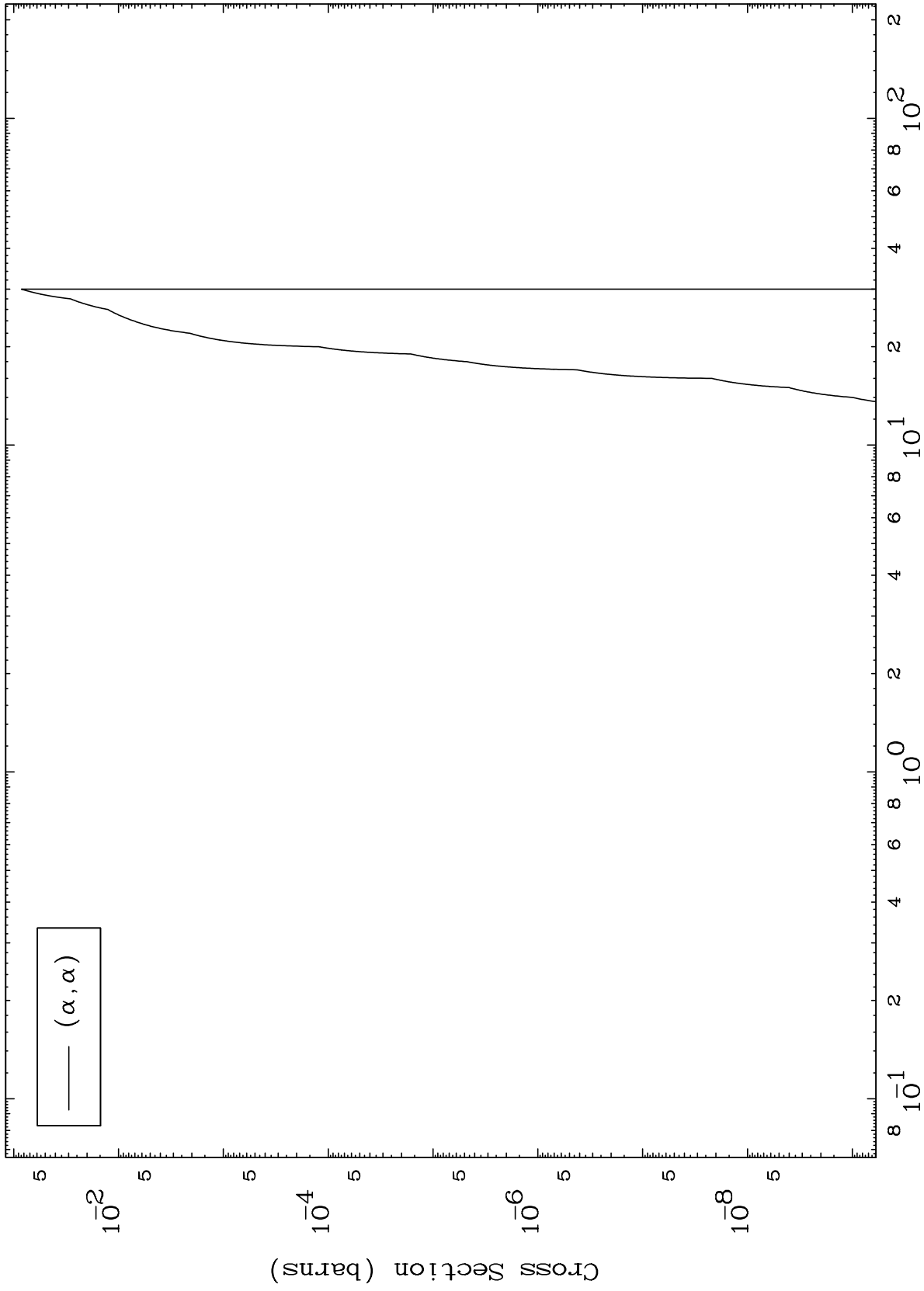
8



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( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

85-At-210



10

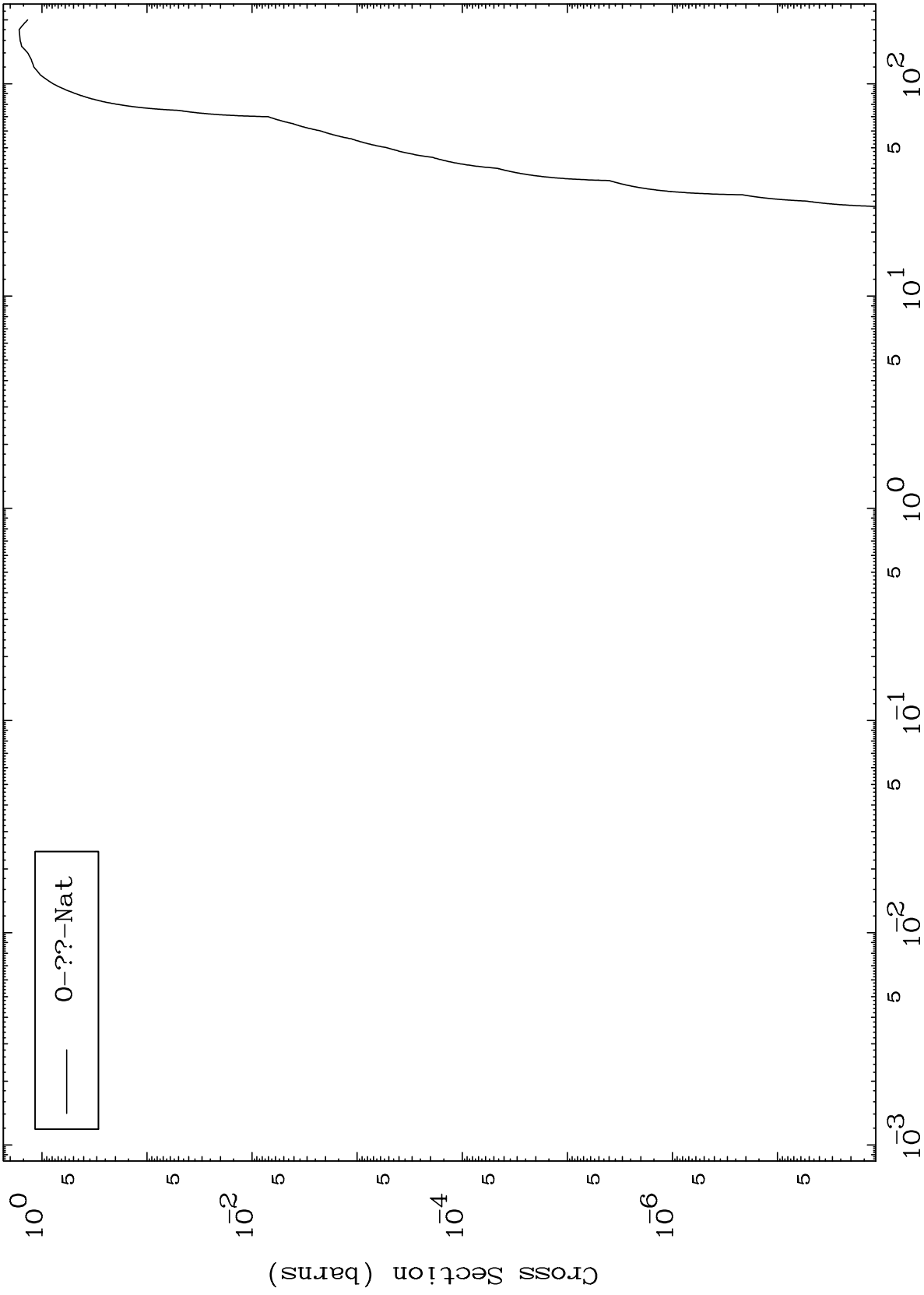
Incident Energy (MeV)

85-At-210

MAT 8546

$\alpha$  Fission  
Radionuclide Production Cross Section

85-At-210



11

Incident Energy (MeV)

85-At-210