

Program EVALPLOT  
(Version 2018-1)

by

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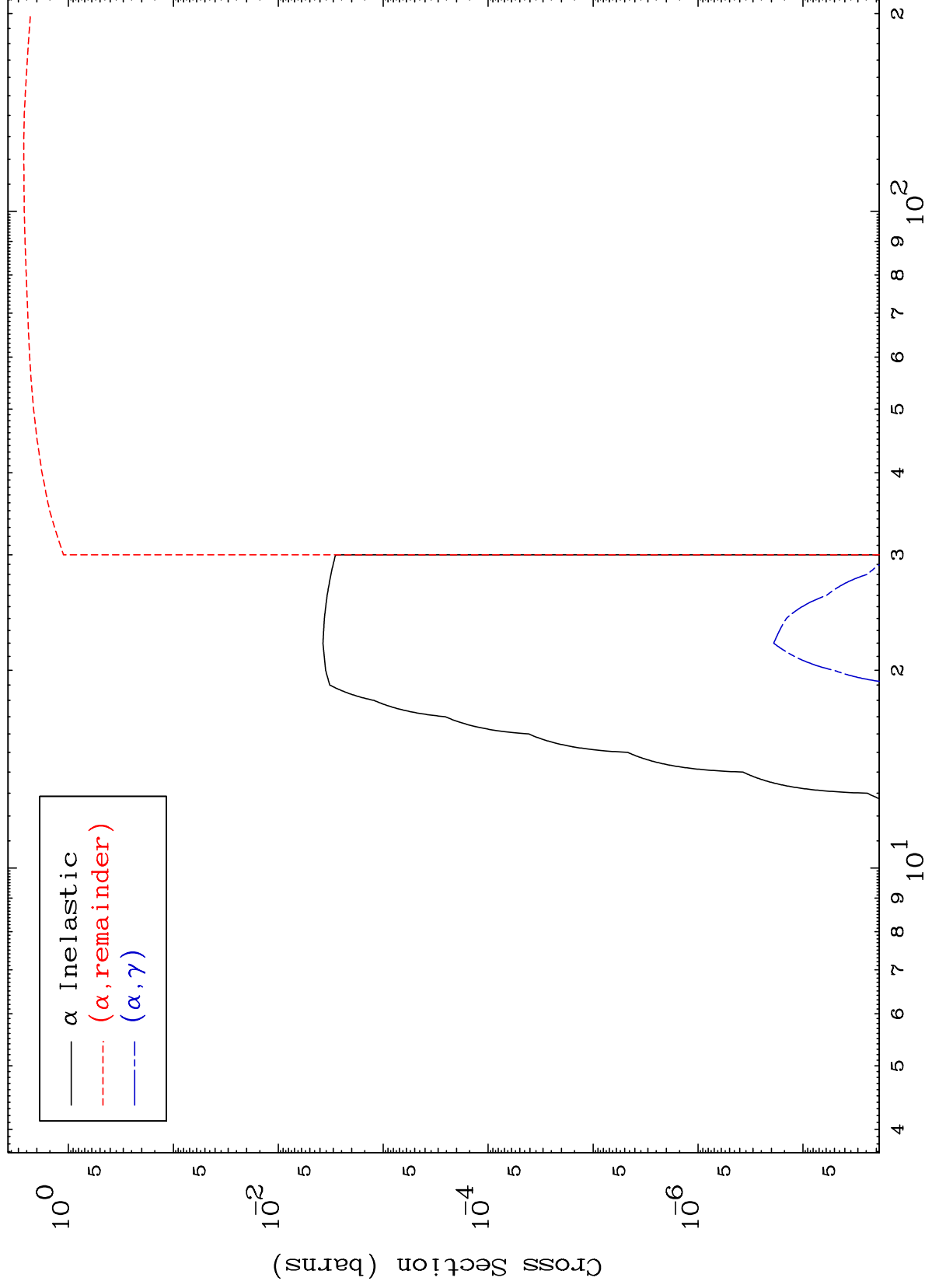
Press Mouse Button to Start

MAT 8585

$\alpha$  Major

0 Kelvin Cross Sections

85-At-223



1

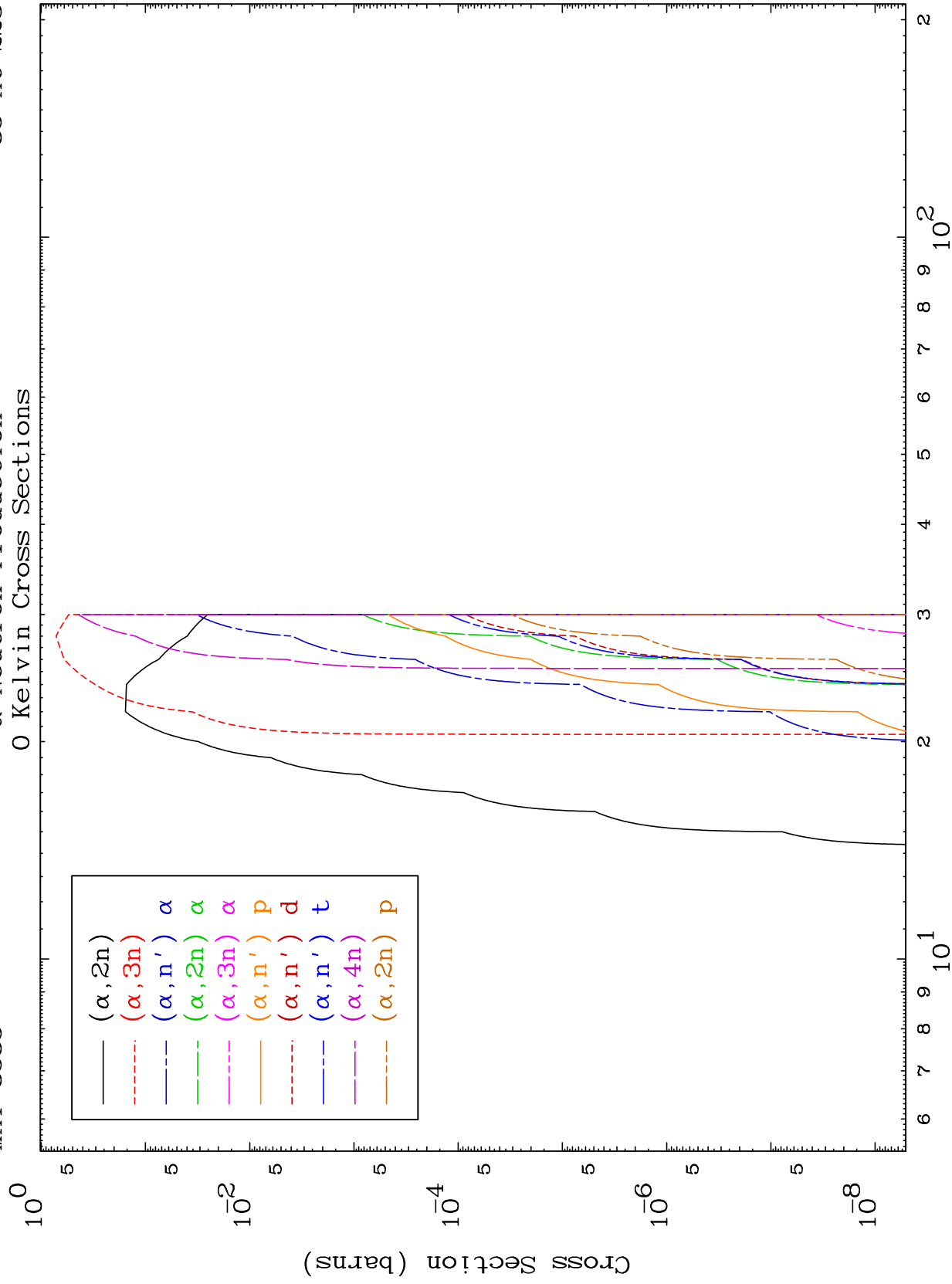
Incident Energy (MeV)

85-At-223

MAT 8585

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

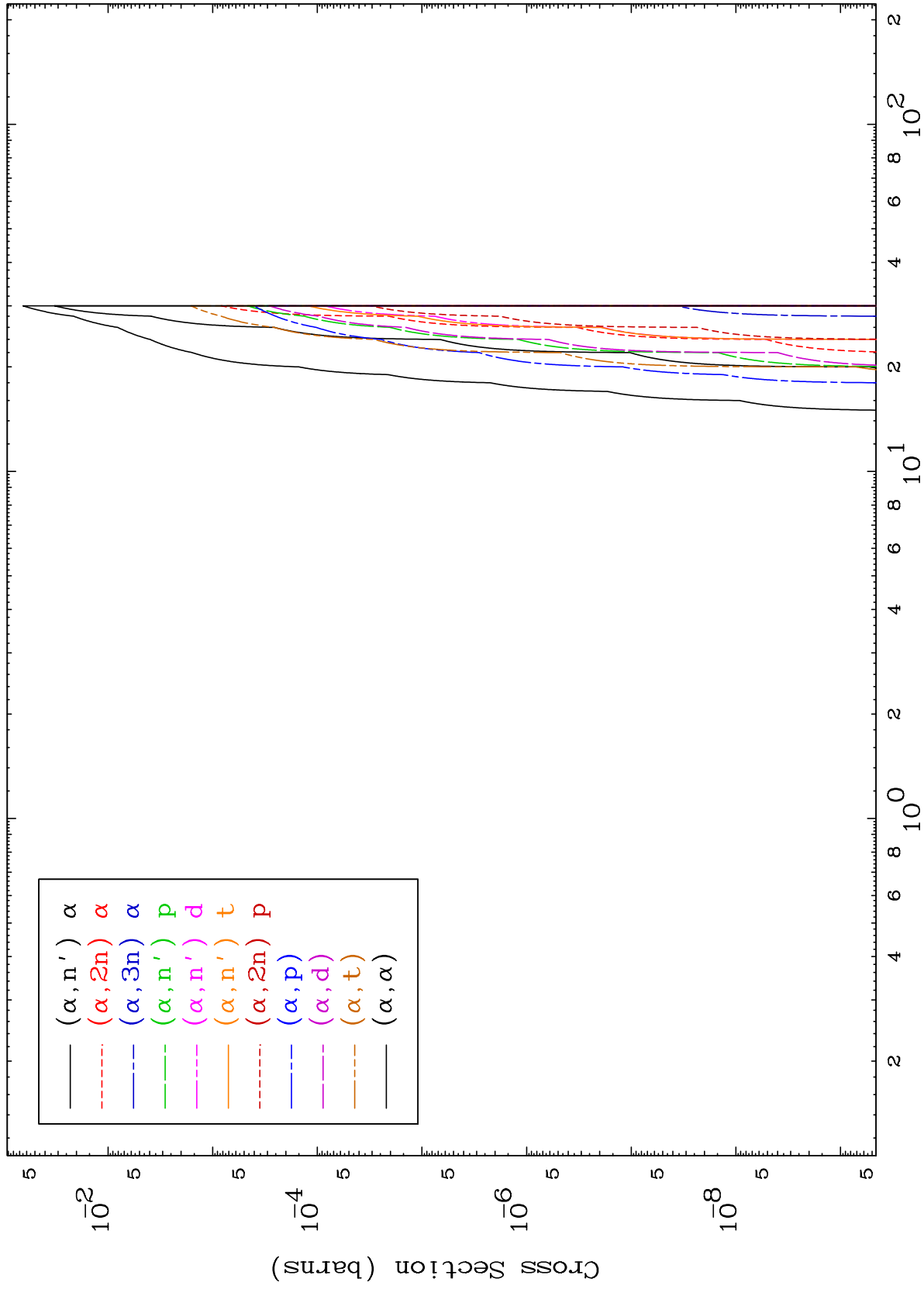
85-At-223



2

Incident Energy (MeV)

85-At-223

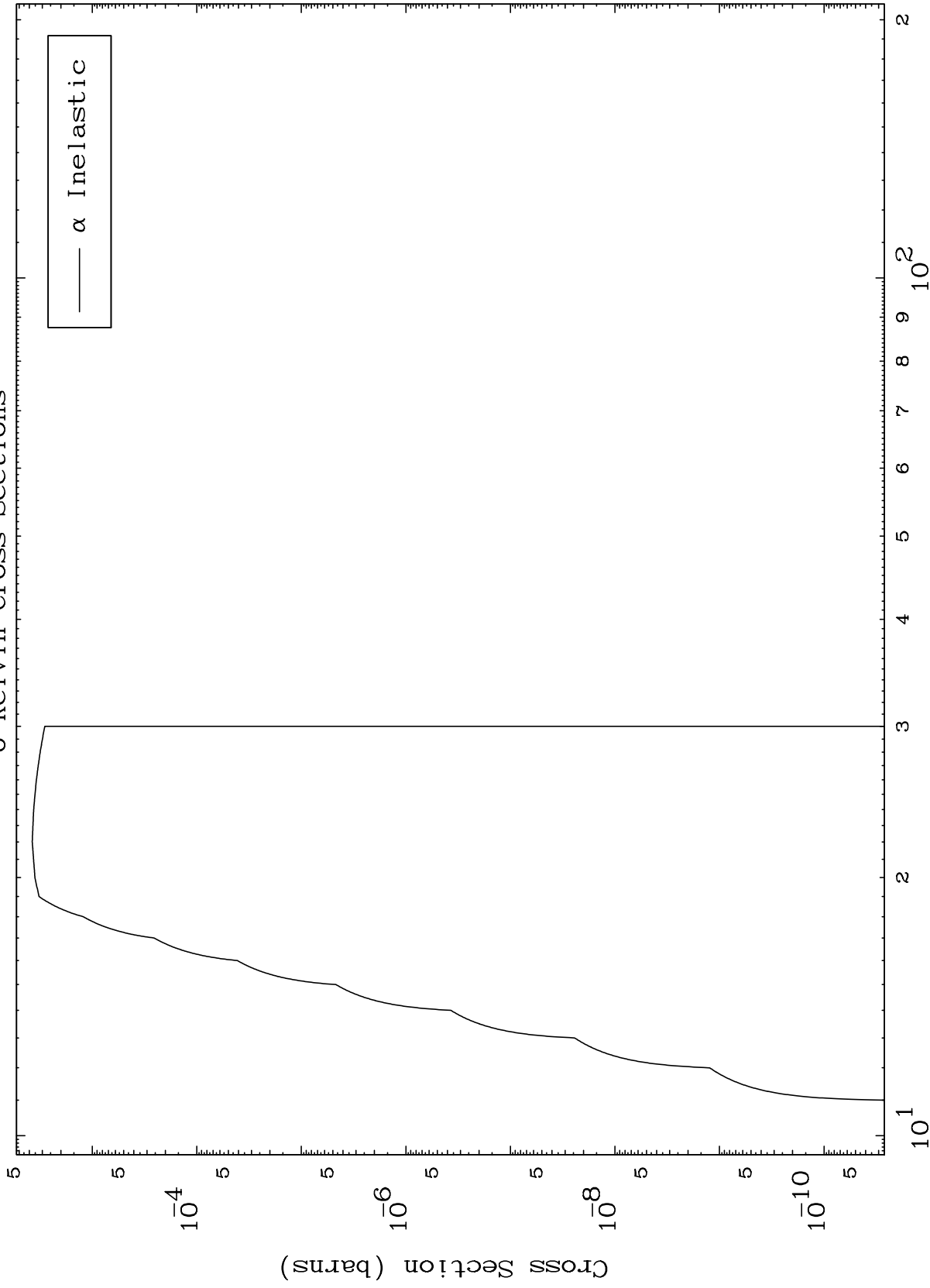


MAT 8585

( $\alpha, n'$ ) Level

0 Kelvin Cross Sections

85-At-223



Incident Energy (MeV)

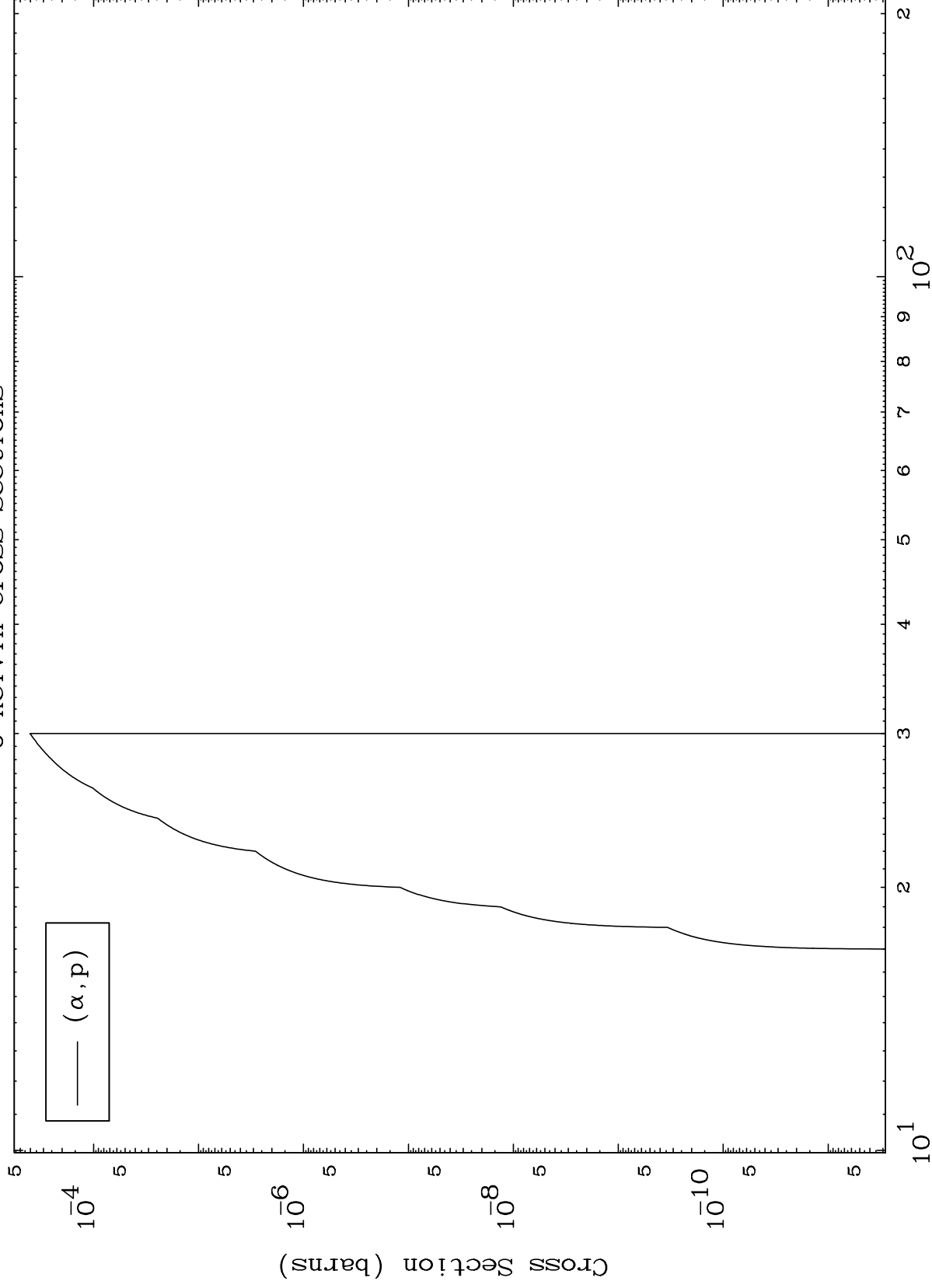
85-At-223

MAT 8585

( $\alpha$ ,p) Levels

85-At-223

0 Kelvin Cross Sections



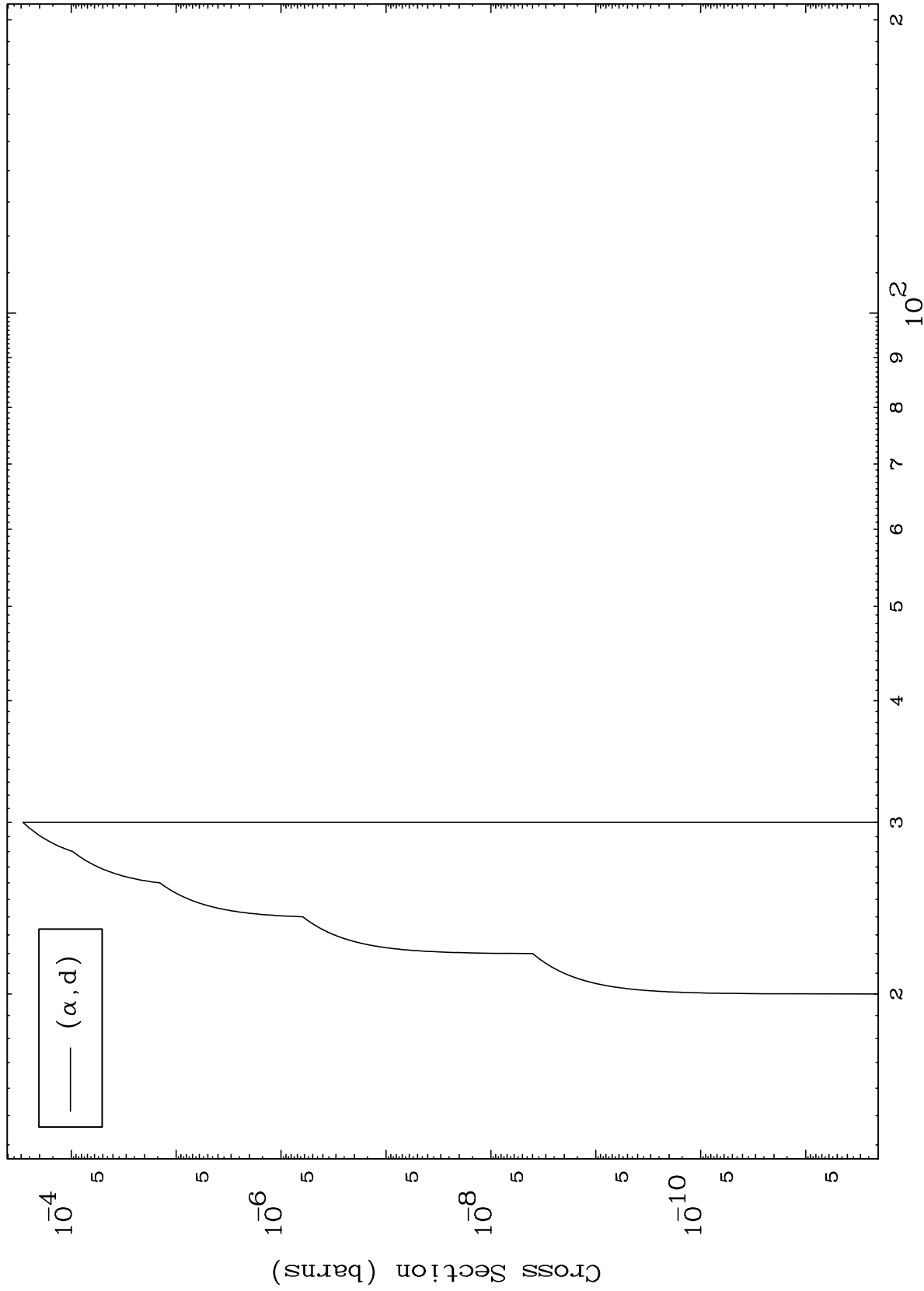
Incident Energy (MeV)

85-At-223

MAT 8585

85-At-223

( $\alpha$ ,d) Levels  
0 Kelvin Cross Sections



6

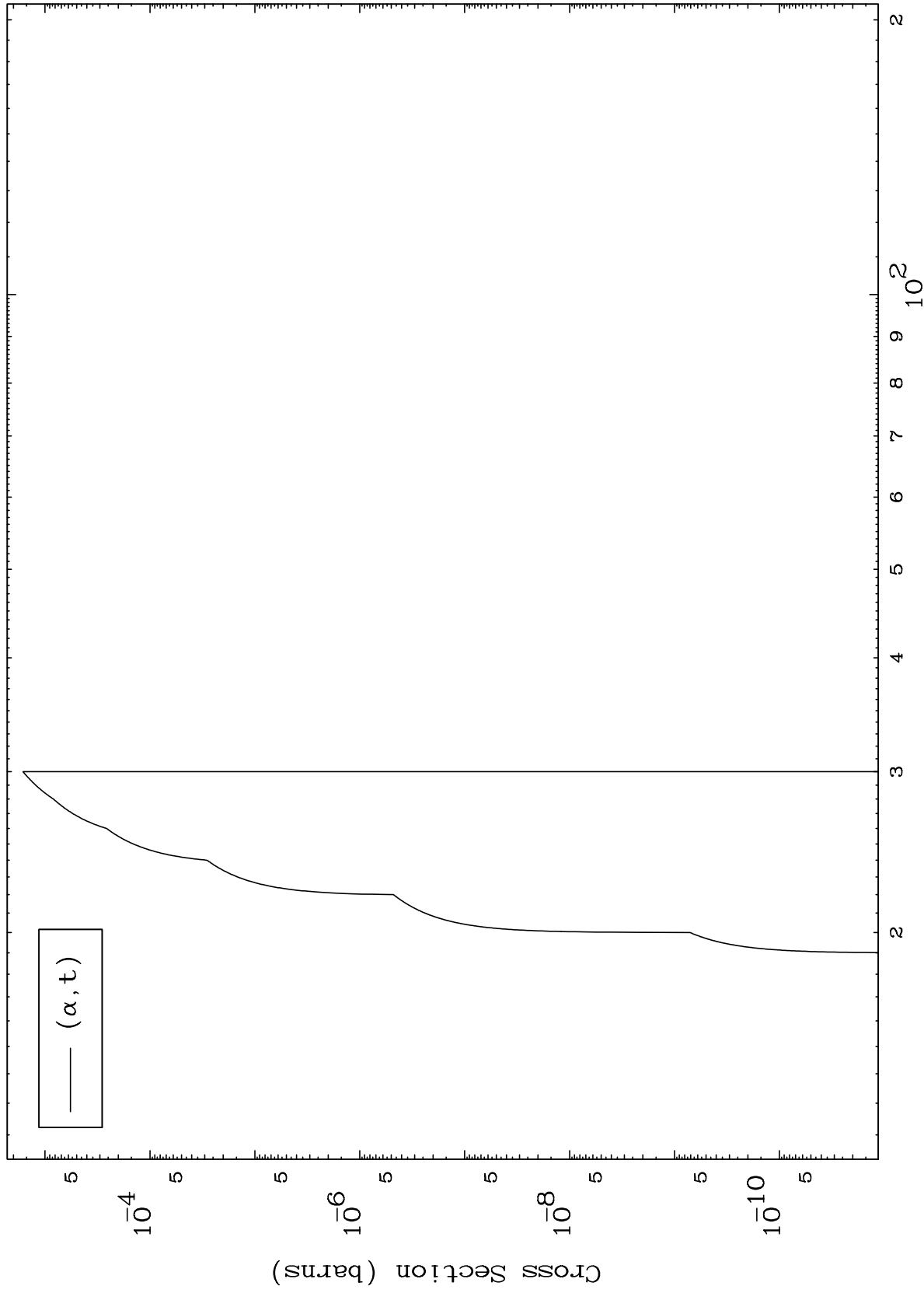
Incident Energy (MeV)

85-At-223

MAT 8585

85-At-223

( $\alpha$ ,t) Levels  
0 Kelvin Cross Sections



7

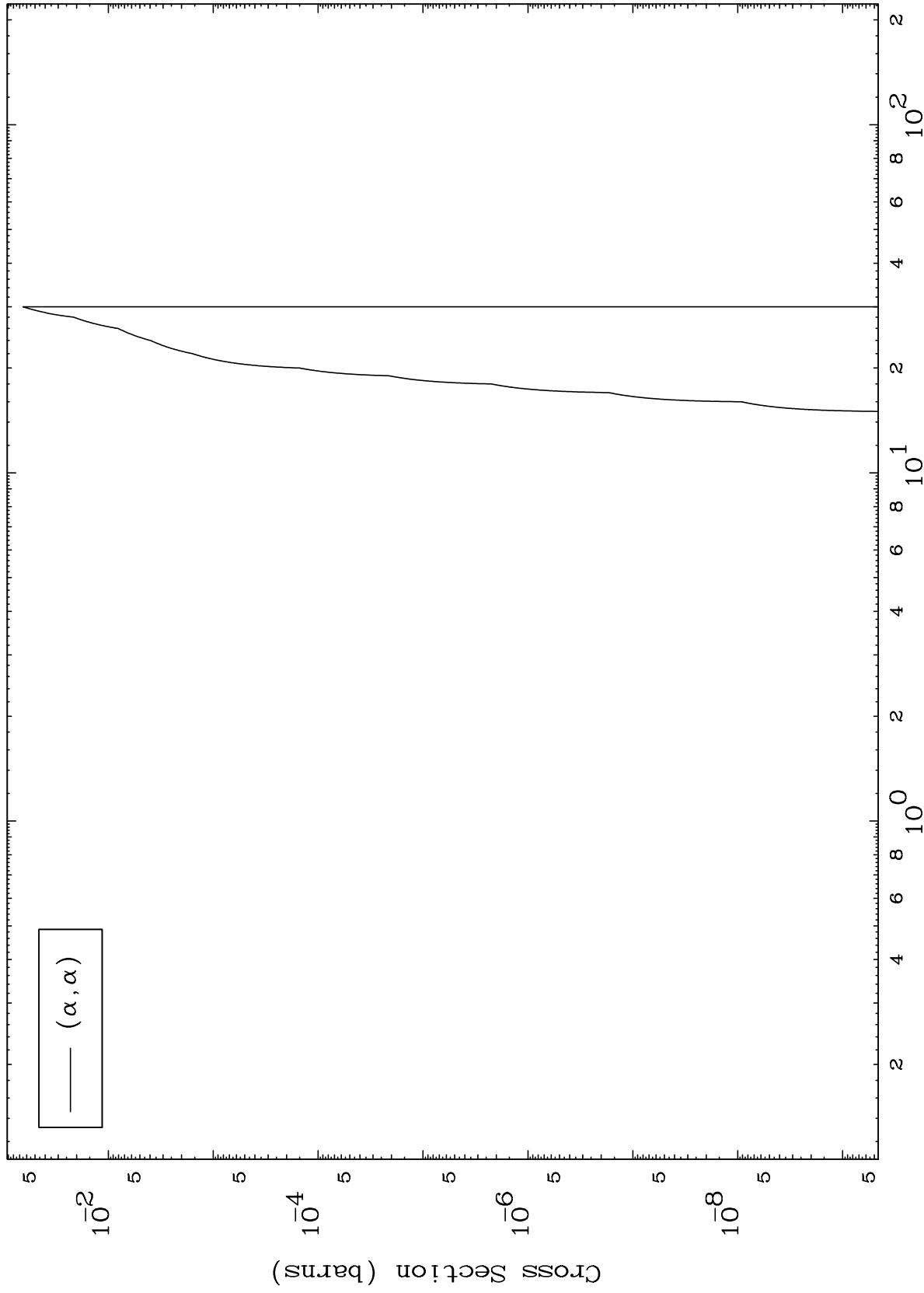
85-At-223

Incident Energy (MeV)

MAT 8585

85-At-223

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections



8

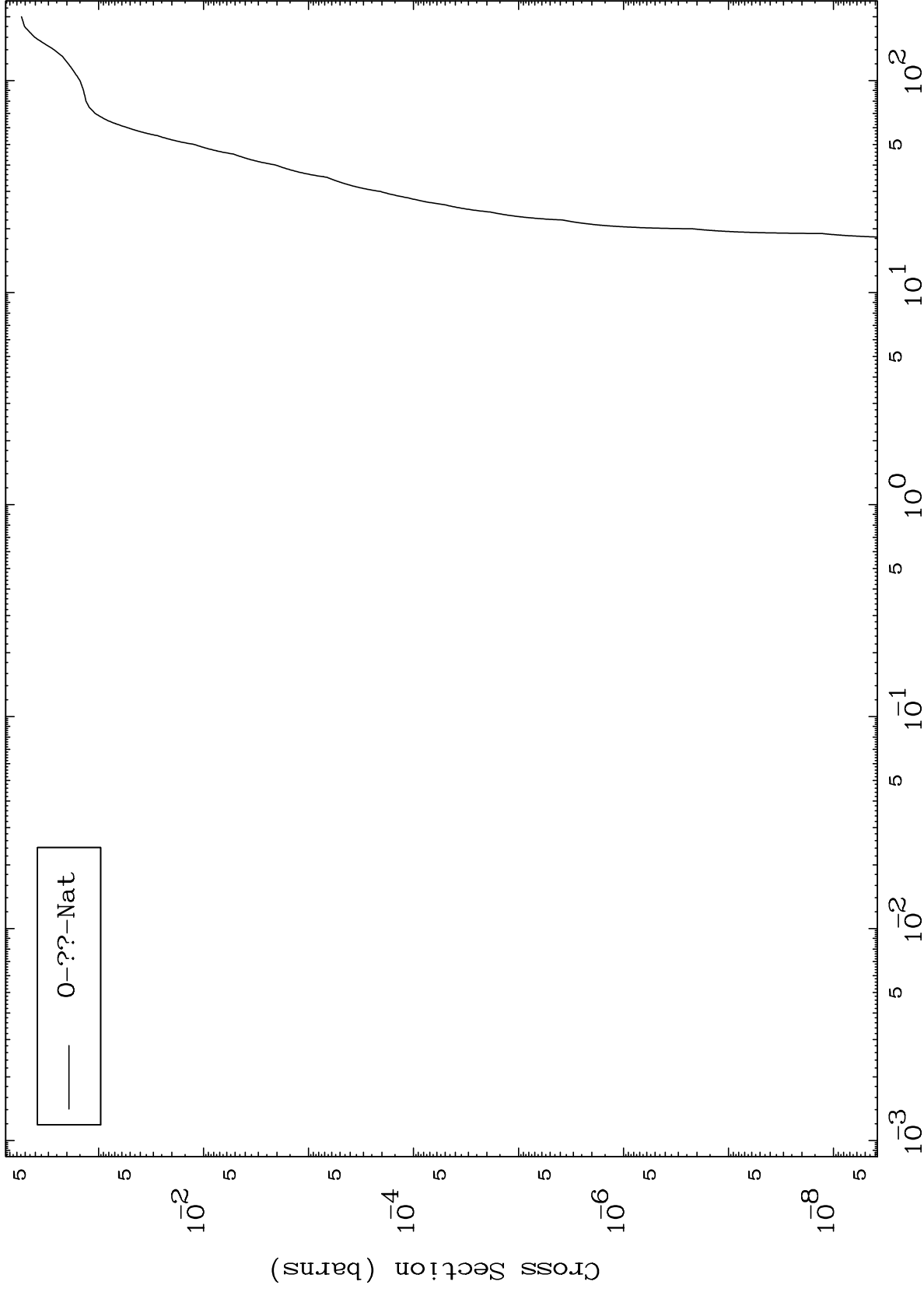
Incident Energy (MeV)

85-At-223

MAT 8585

$\alpha$  Fission  
Radionuclide Production Cross Section

85-At-223



Incident Energy (MeV)

85-At-223

9